

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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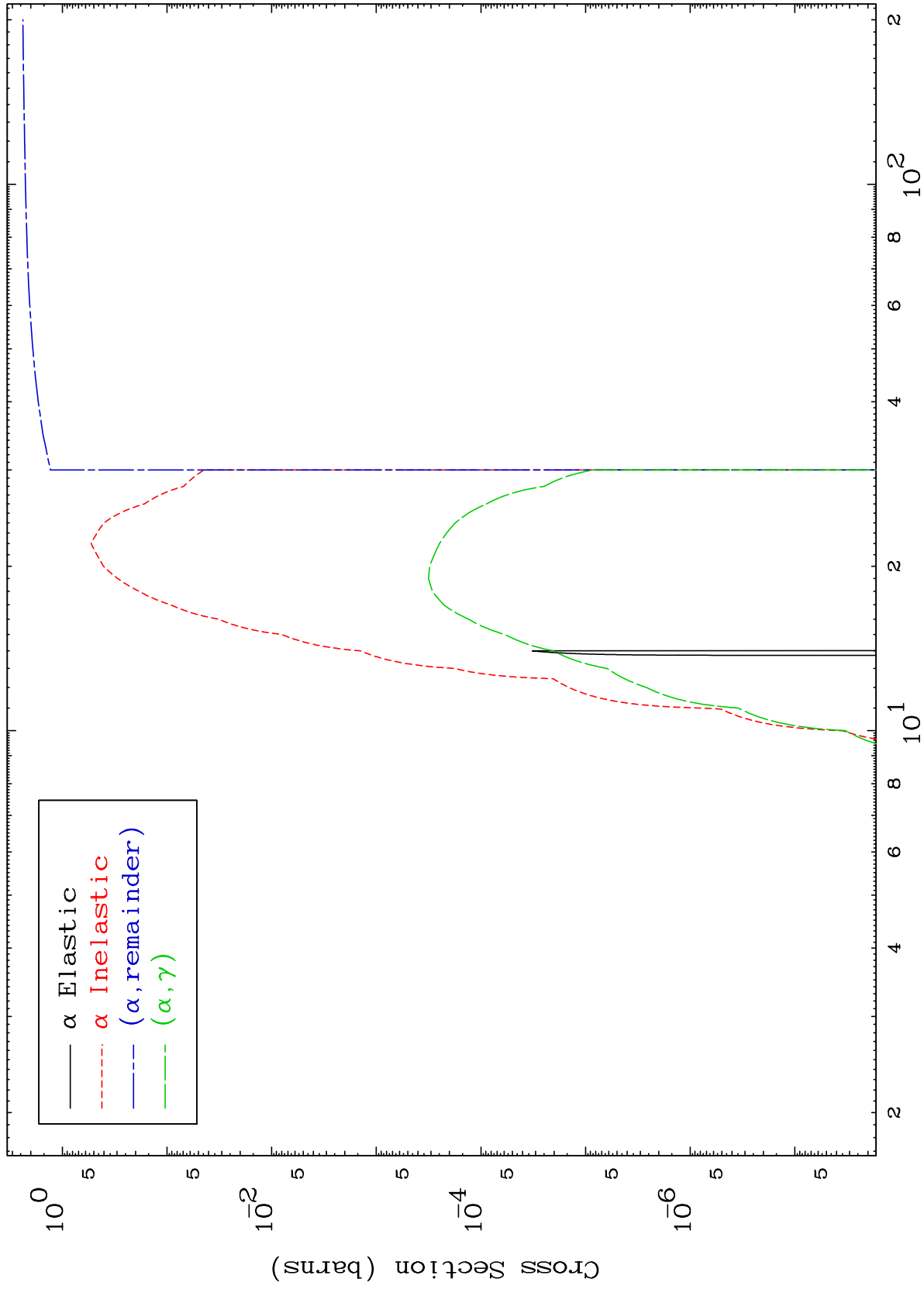
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 6222

0 Kelvin  $\alpha$  Major

62-Sm-143



1

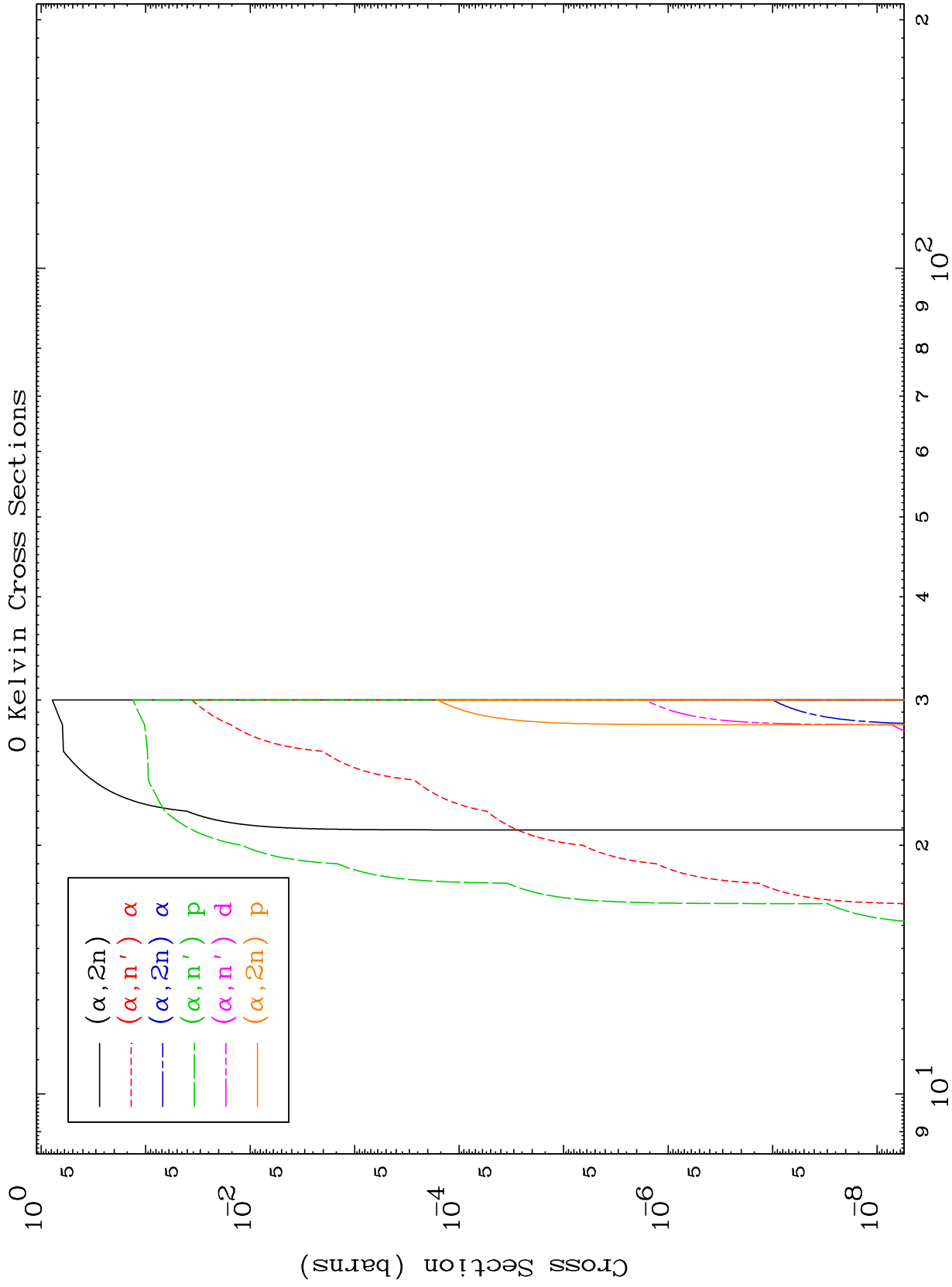
Incident Energy (MeV)

62-Sm-143

MAT 6222

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

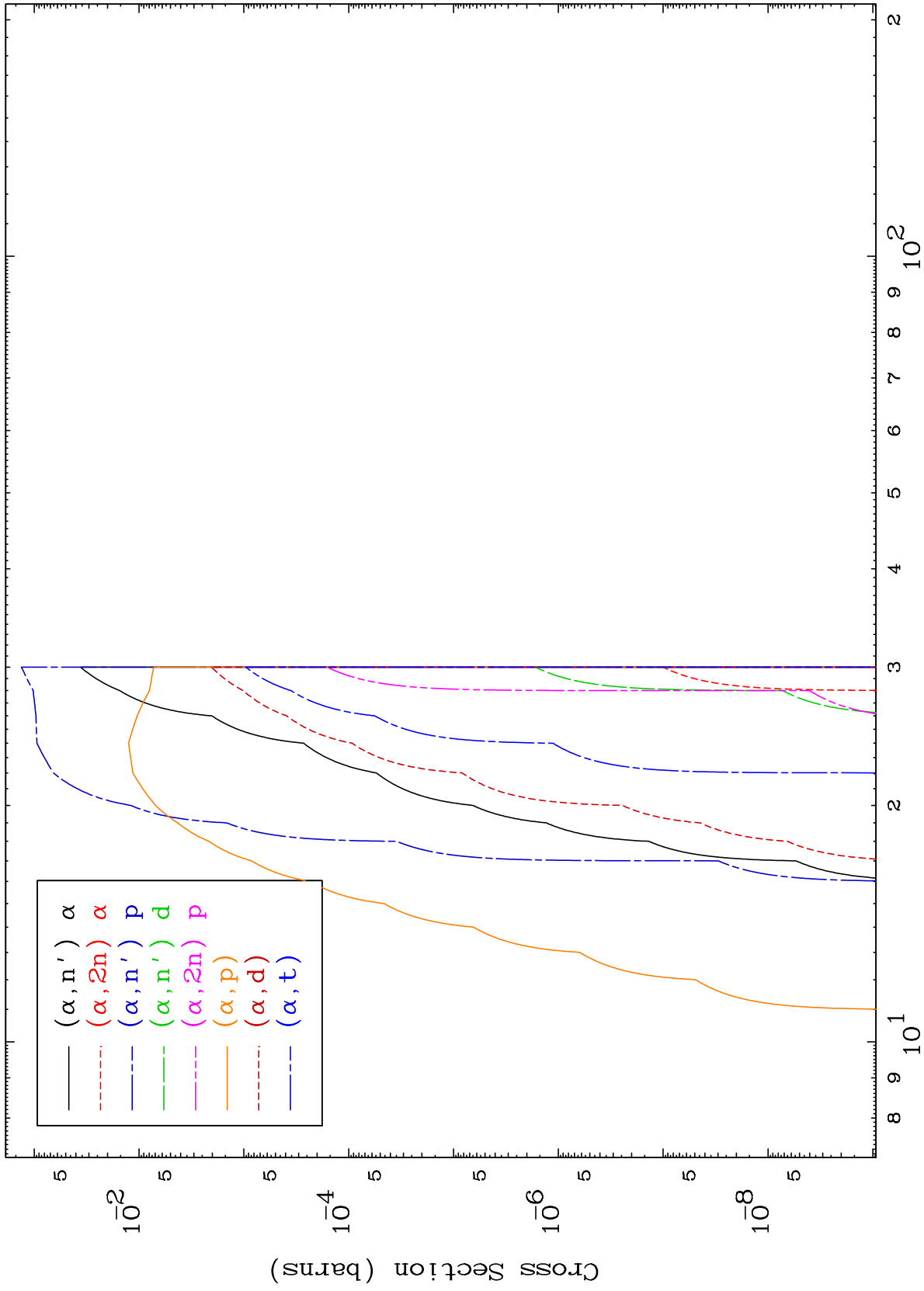
62-Sm-143



2

Incident Energy (MeV)

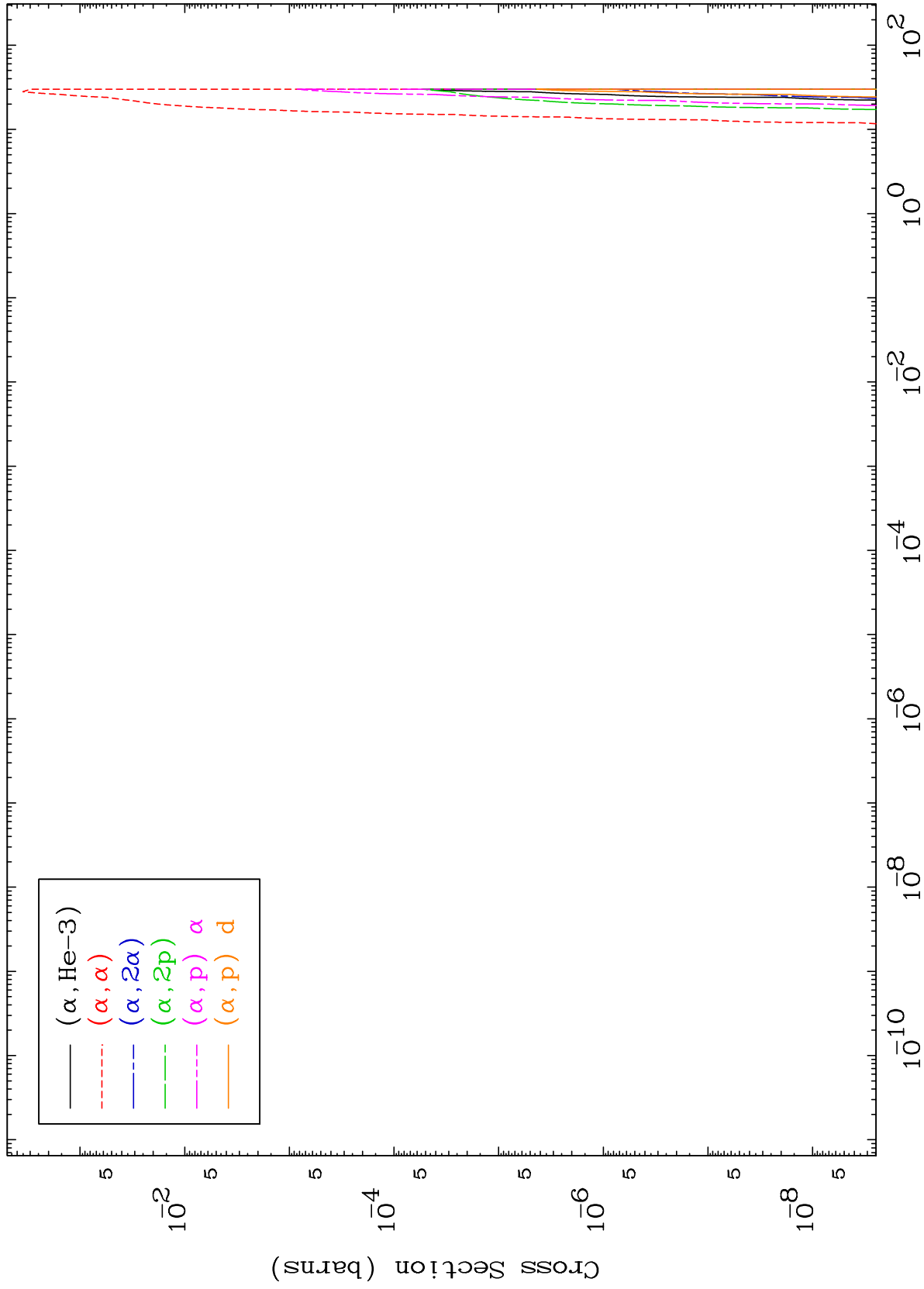
62-Sm-143



MAT 6222

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

62-Sm-143

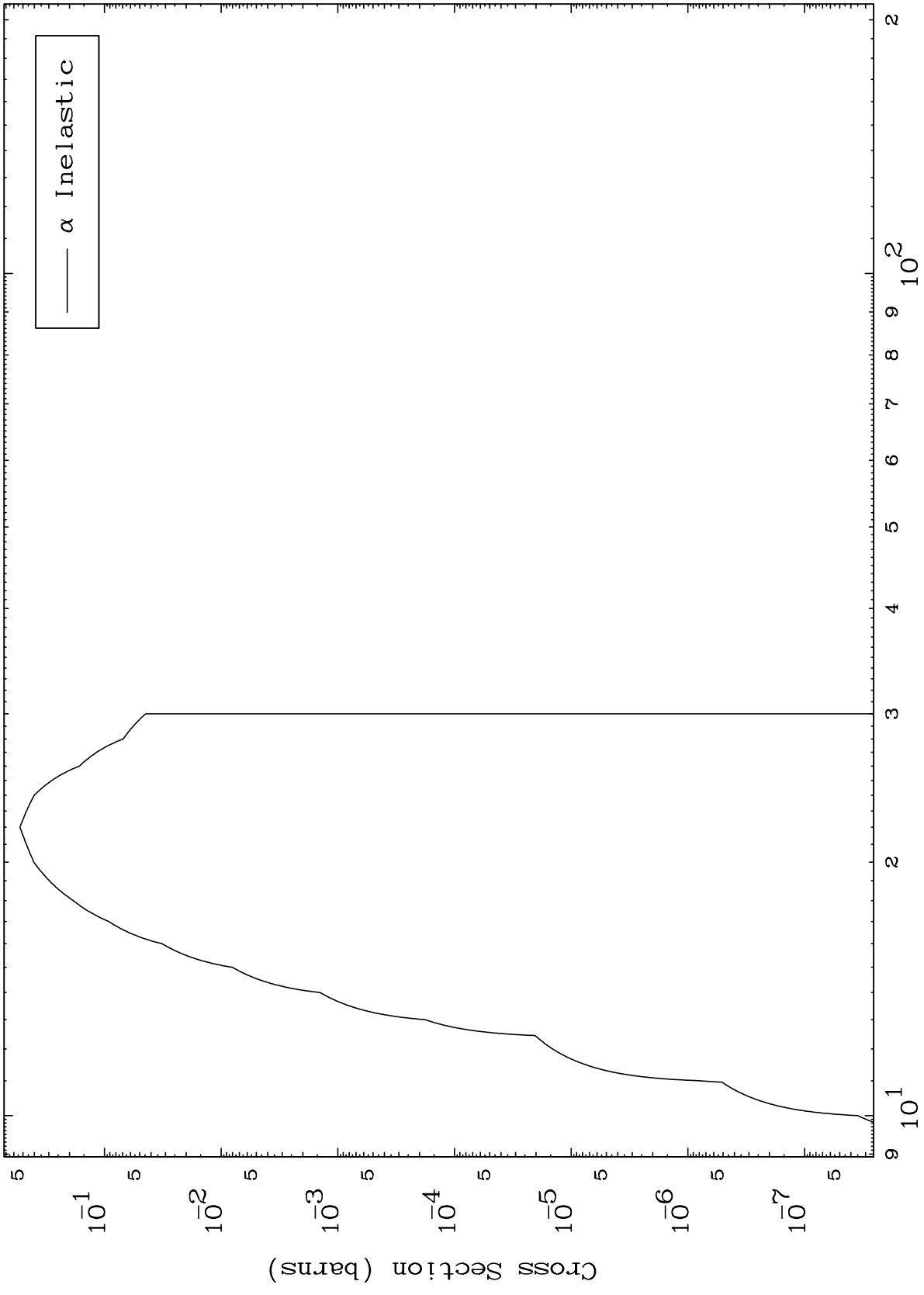


62-Sm-143

MAT 6222

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

62-Sm-143



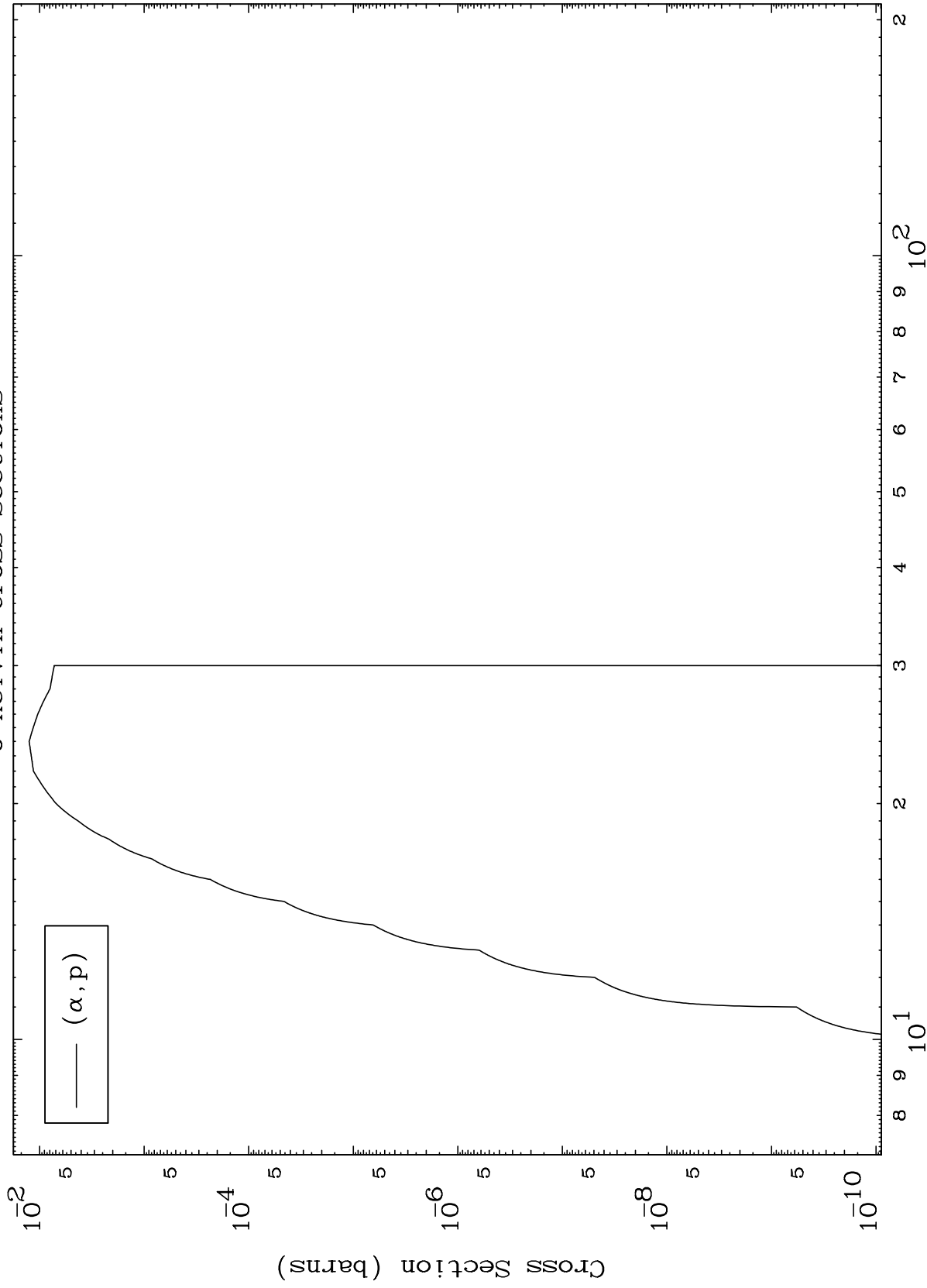
62-Sm-143

Incident Energy (MeV)

MAT 6222

62-Sm-143

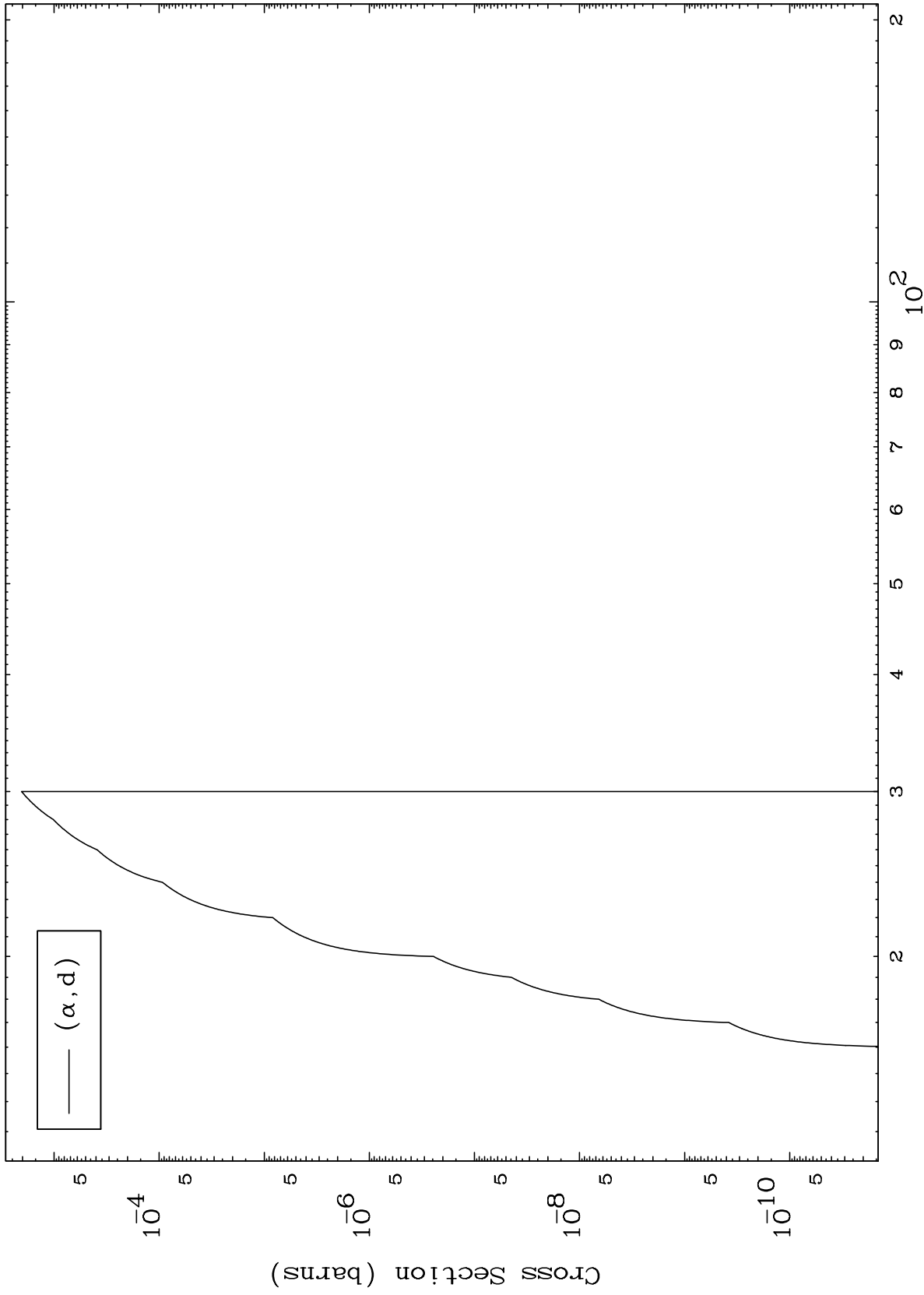
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections



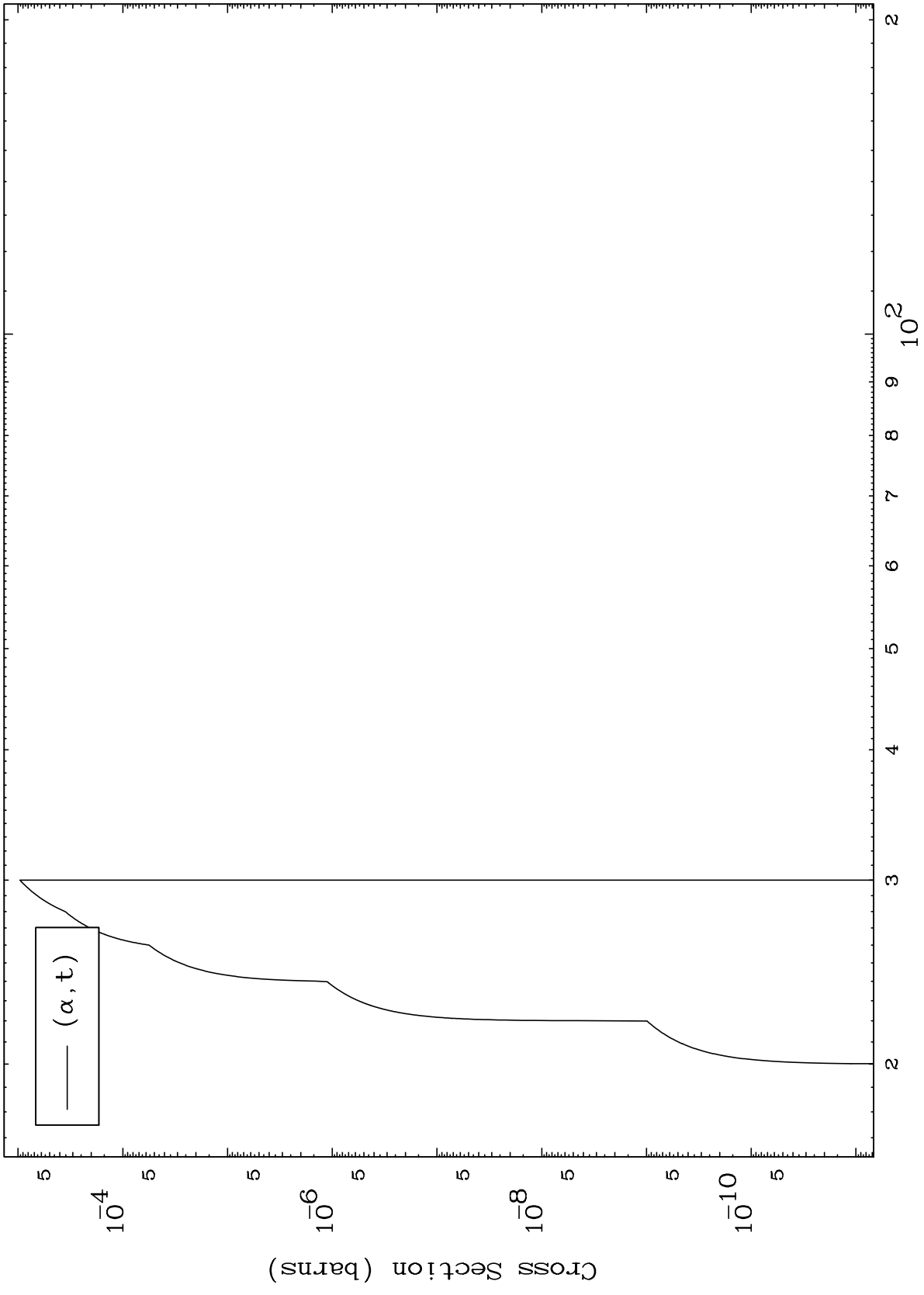
6

Incident Energy (MeV)

62-Sm-143



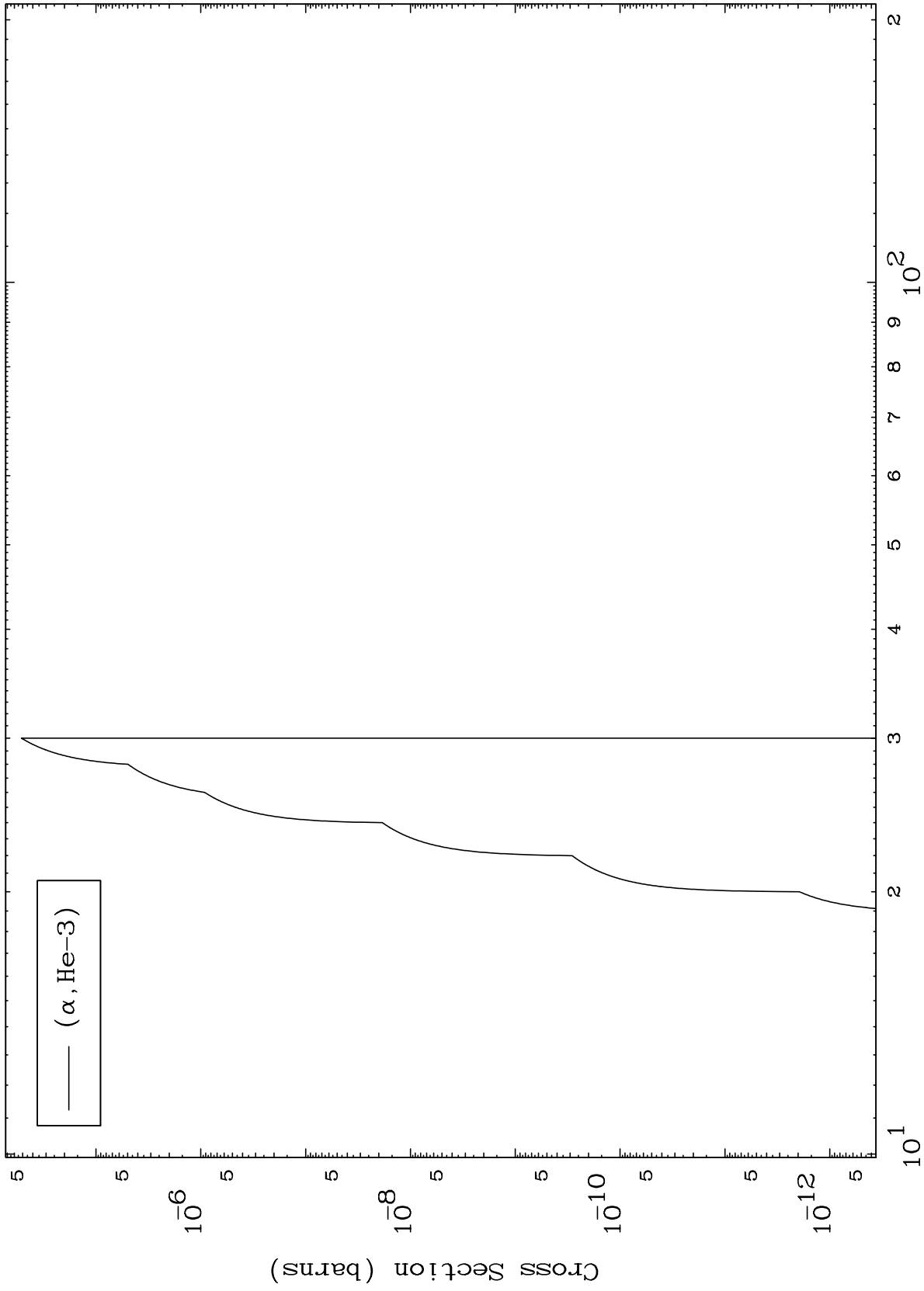




MAT 6222

( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

62-Sm-143



9

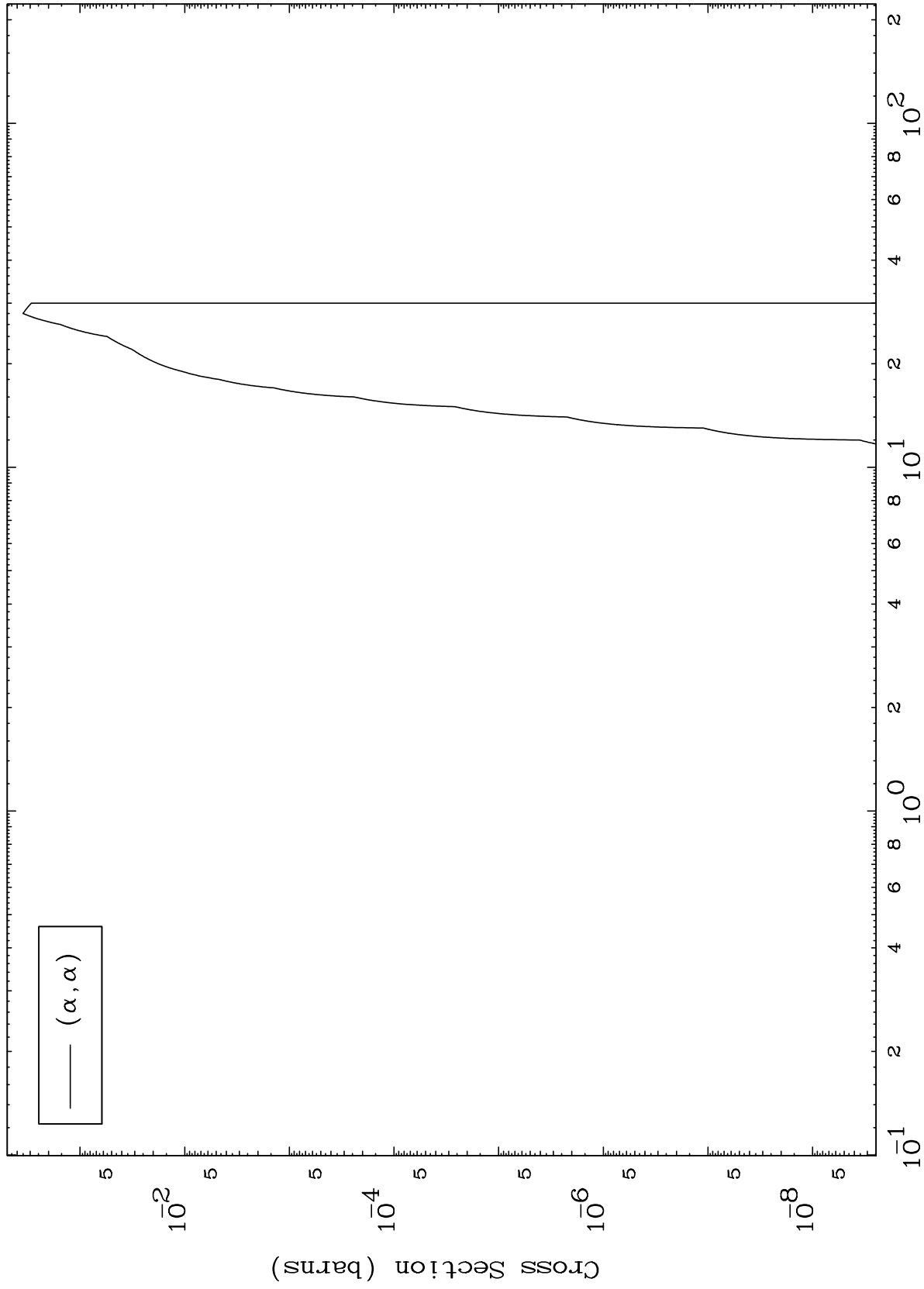
Incident Energy (MeV)

62-Sm-143

MAT 6222

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

62-Sm-143



10

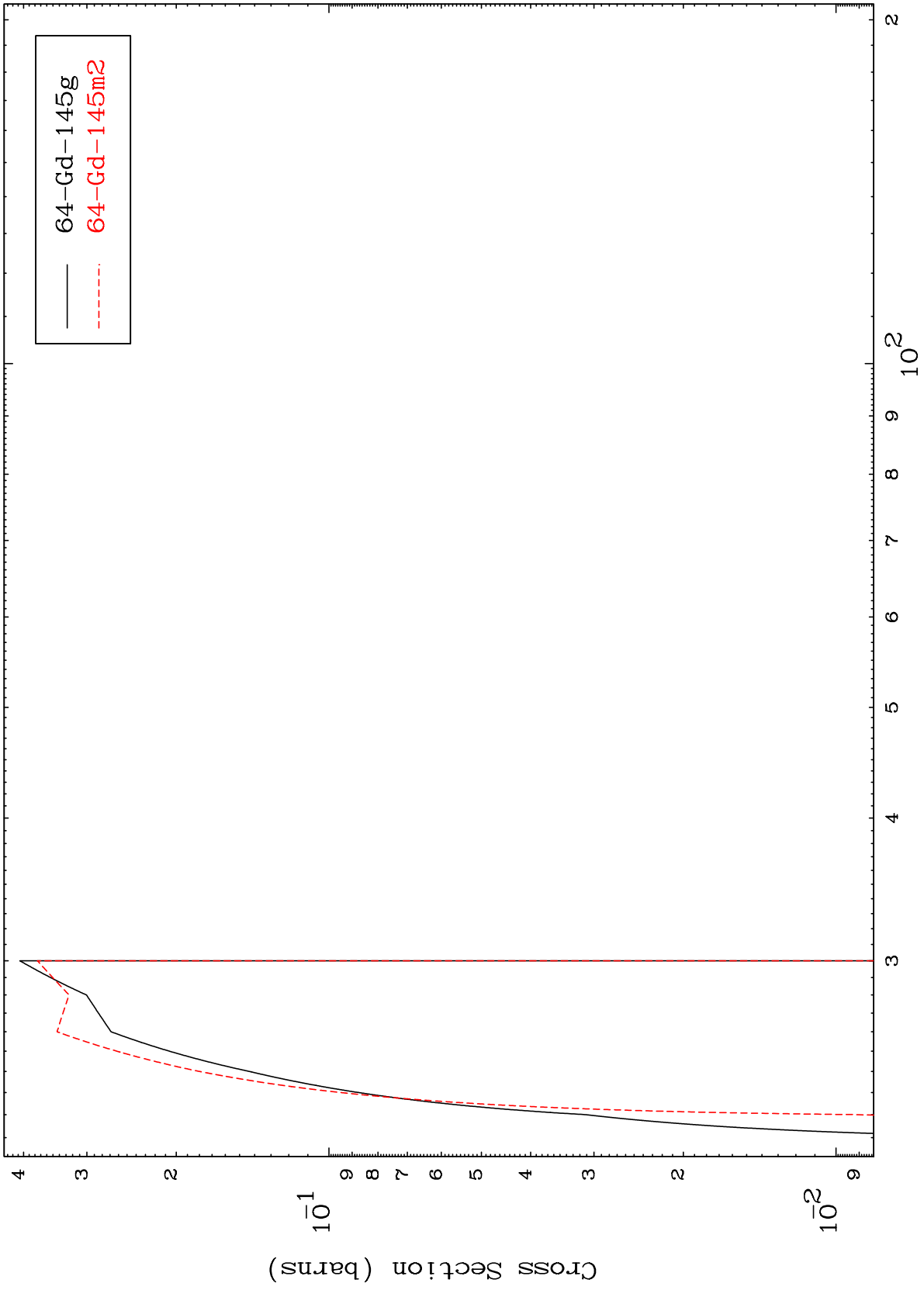
Incident Energy (MeV)

62-Sm-143

MAT 6222

62-Sm-143

$(\alpha, 2n)$   
Radionuclide Production Cross Section



11

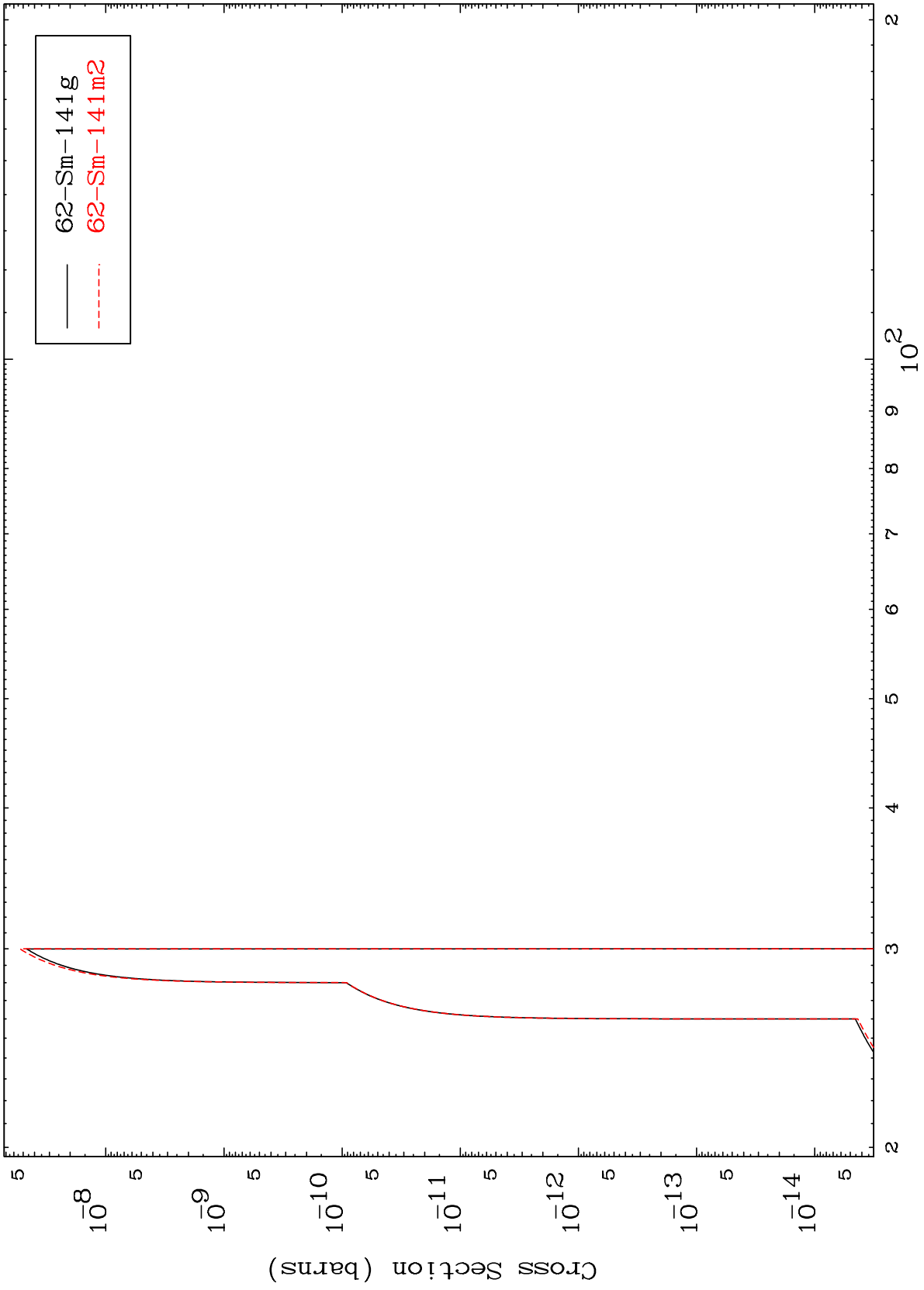
62-Sm-143

Incident Energy (MeV)

MAT 6222

62-Sm-143

$(\alpha, 2n) \alpha$   
Radionuclide Production Cross Section



12

Incident Energy (MeV)

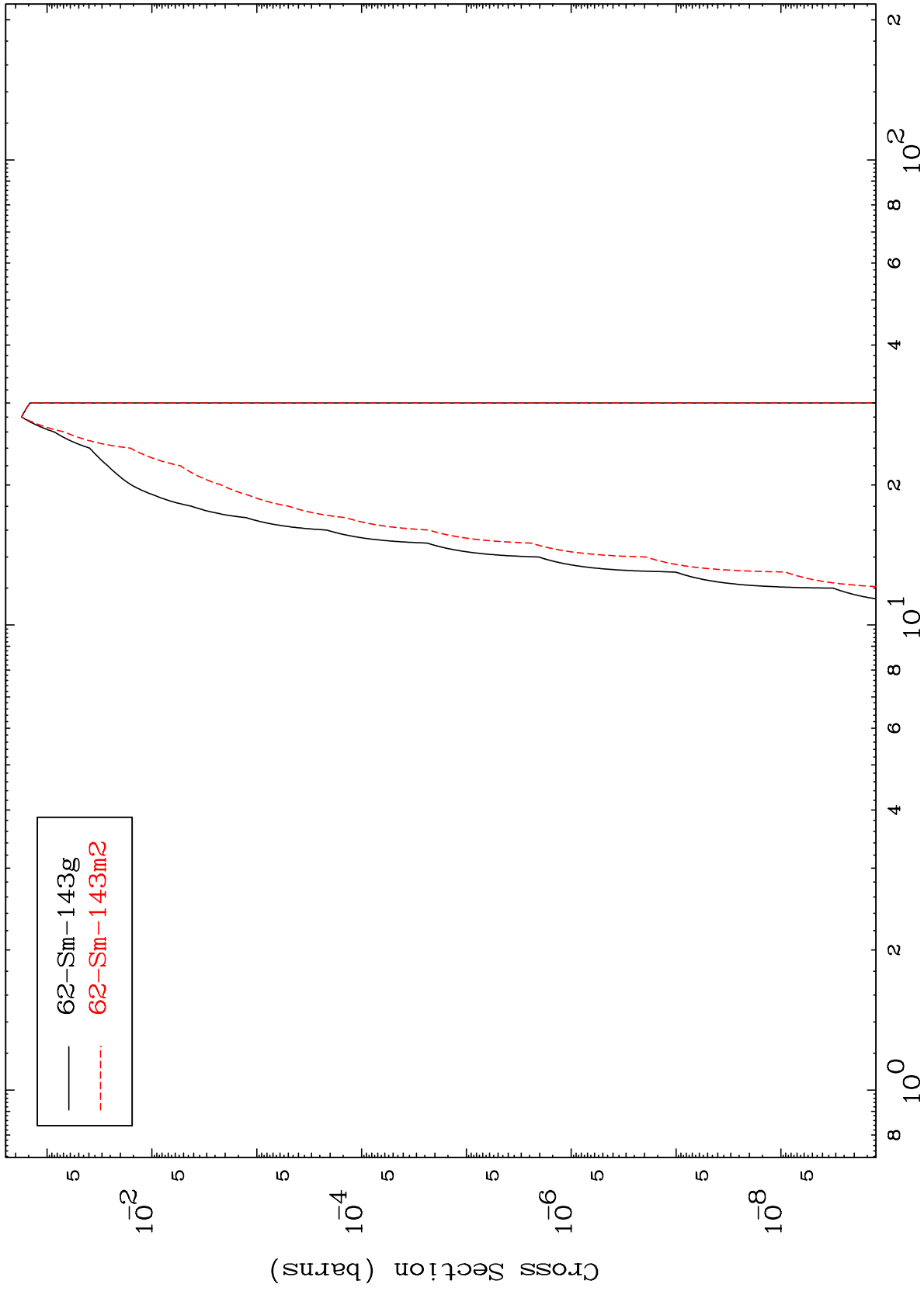
62-Sm-143

MAT 6222

( $\alpha, \alpha$ )

62-Sm-143

Radionuclide Production Cross Section



13

Incident Energy (MeV)

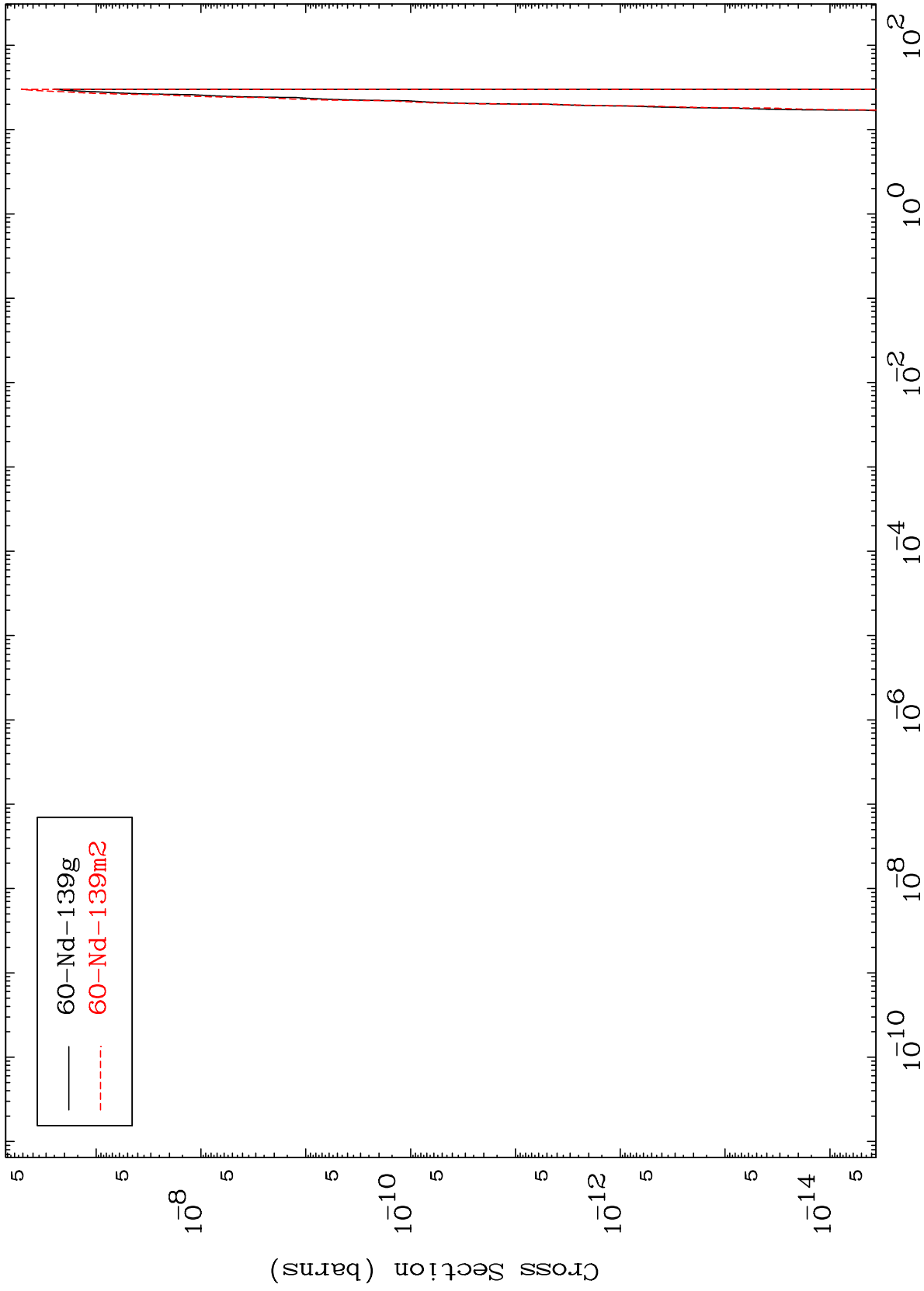
62-Sm-143

MAT 6222

( $\alpha, 2\alpha$ )

62-Sm-143

Radionuclide Production Cross Section



14

Incident Energy (MeV)

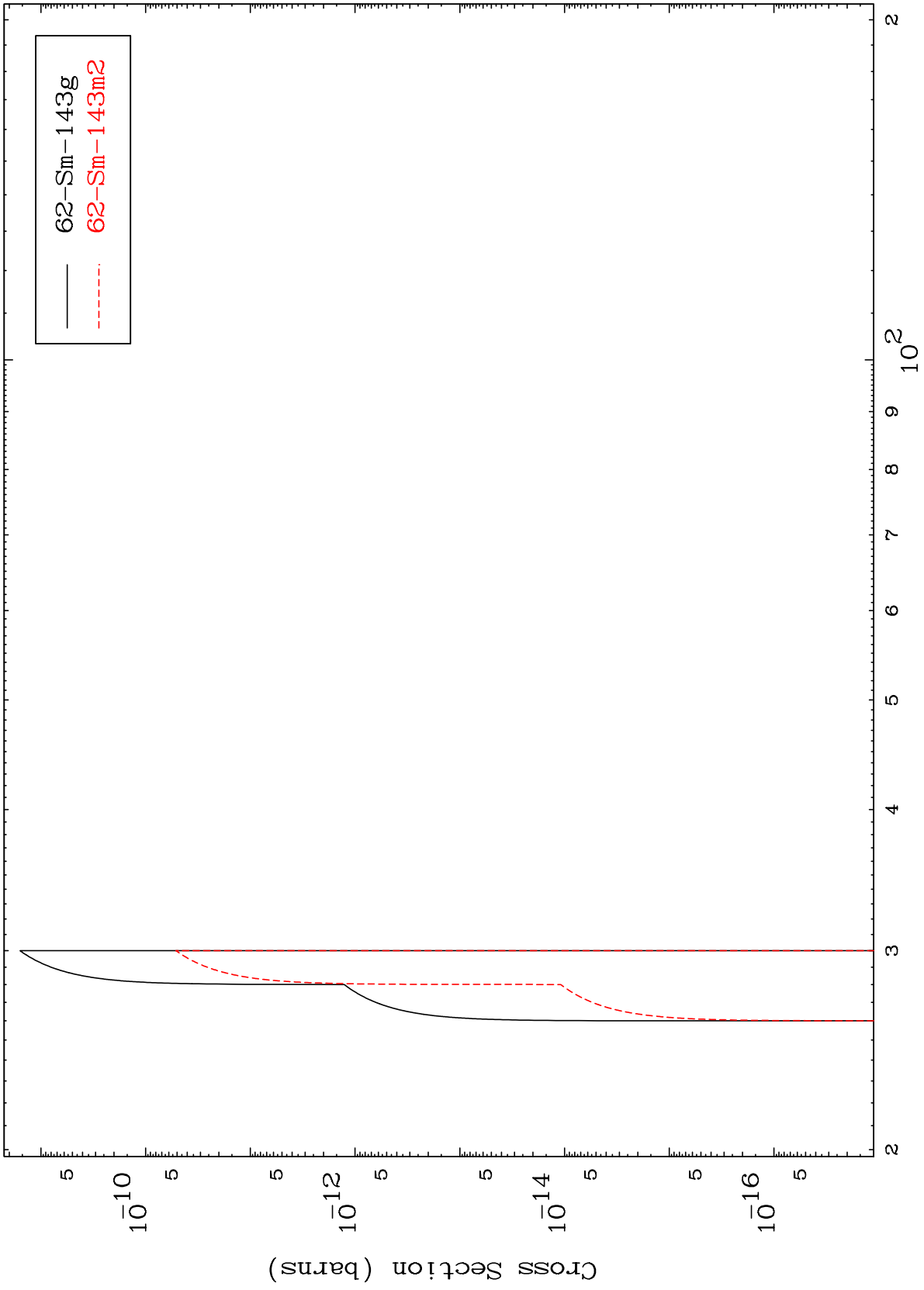
62-Sm-143

MAT 6222

( $\alpha, p$ ) t

62-Sm-143

Radionuclide Production Cross Section



15

Incident Energy (MeV)

62-Sm-143