

Program EVALPLOT  
(Version 2018-1)

by

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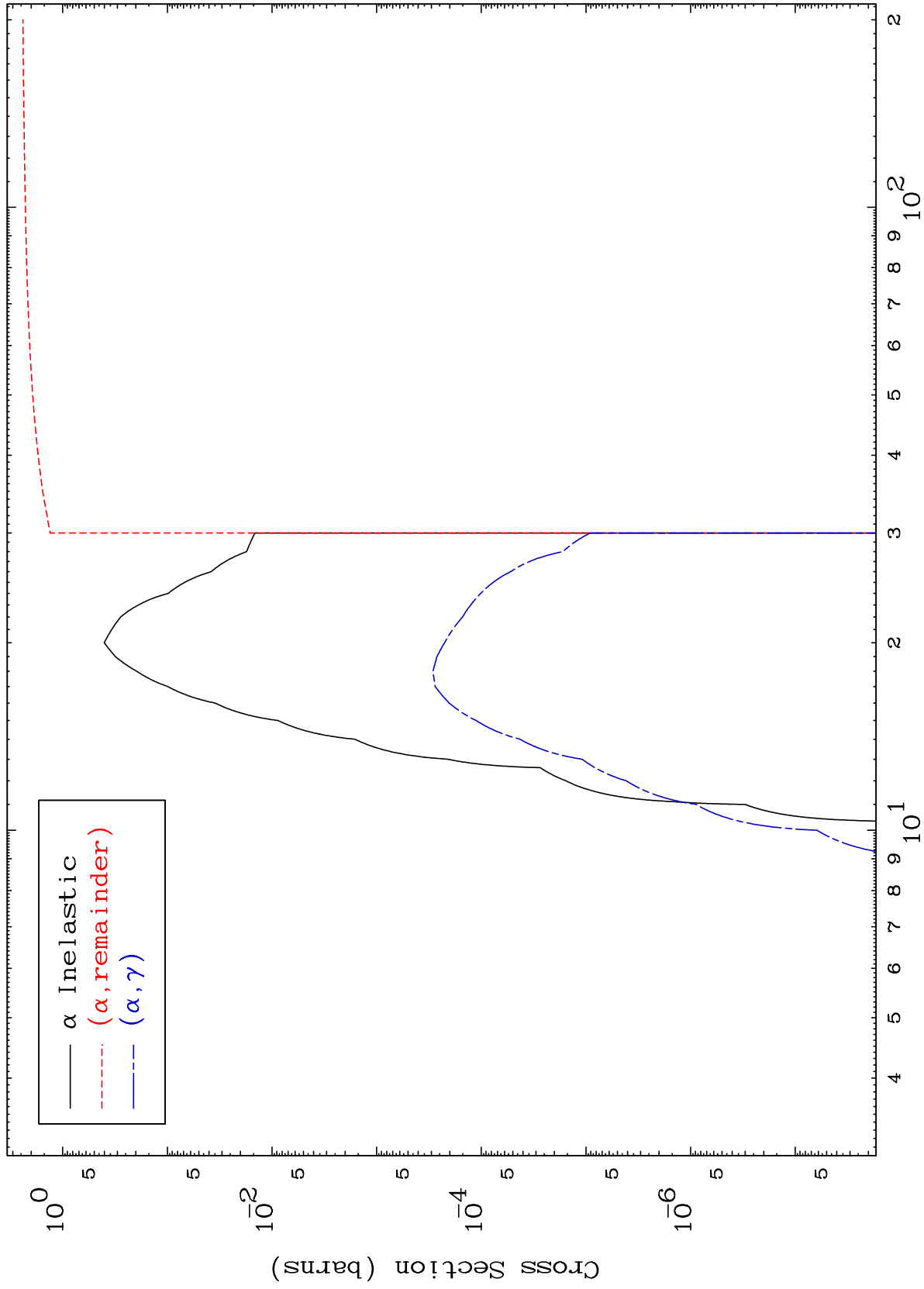
Press Mouse Button to Start

MAT 6228

$\alpha$  Major

62-Sm-145

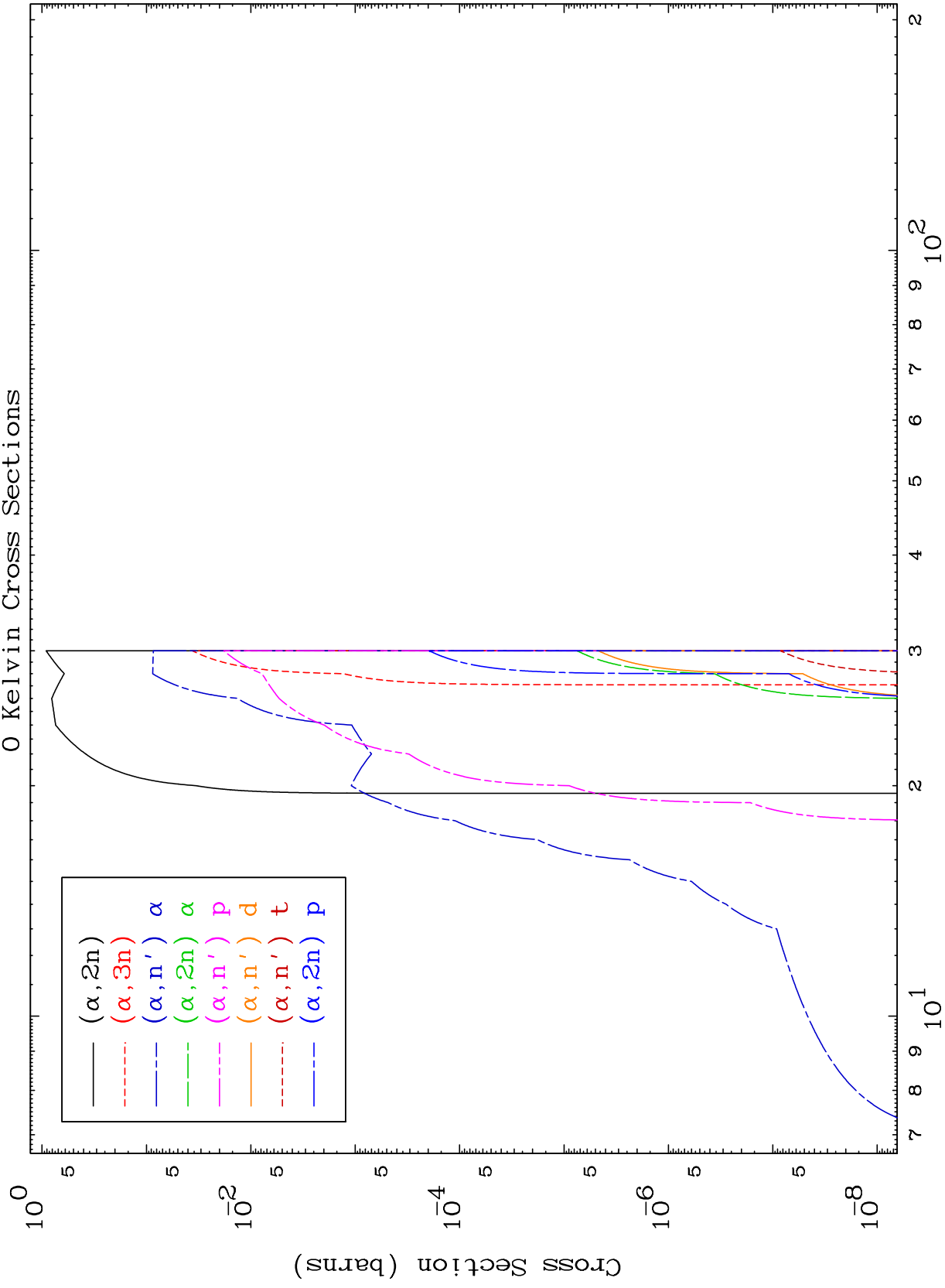
0 Kelvin Cross Sections



MAT 6228

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

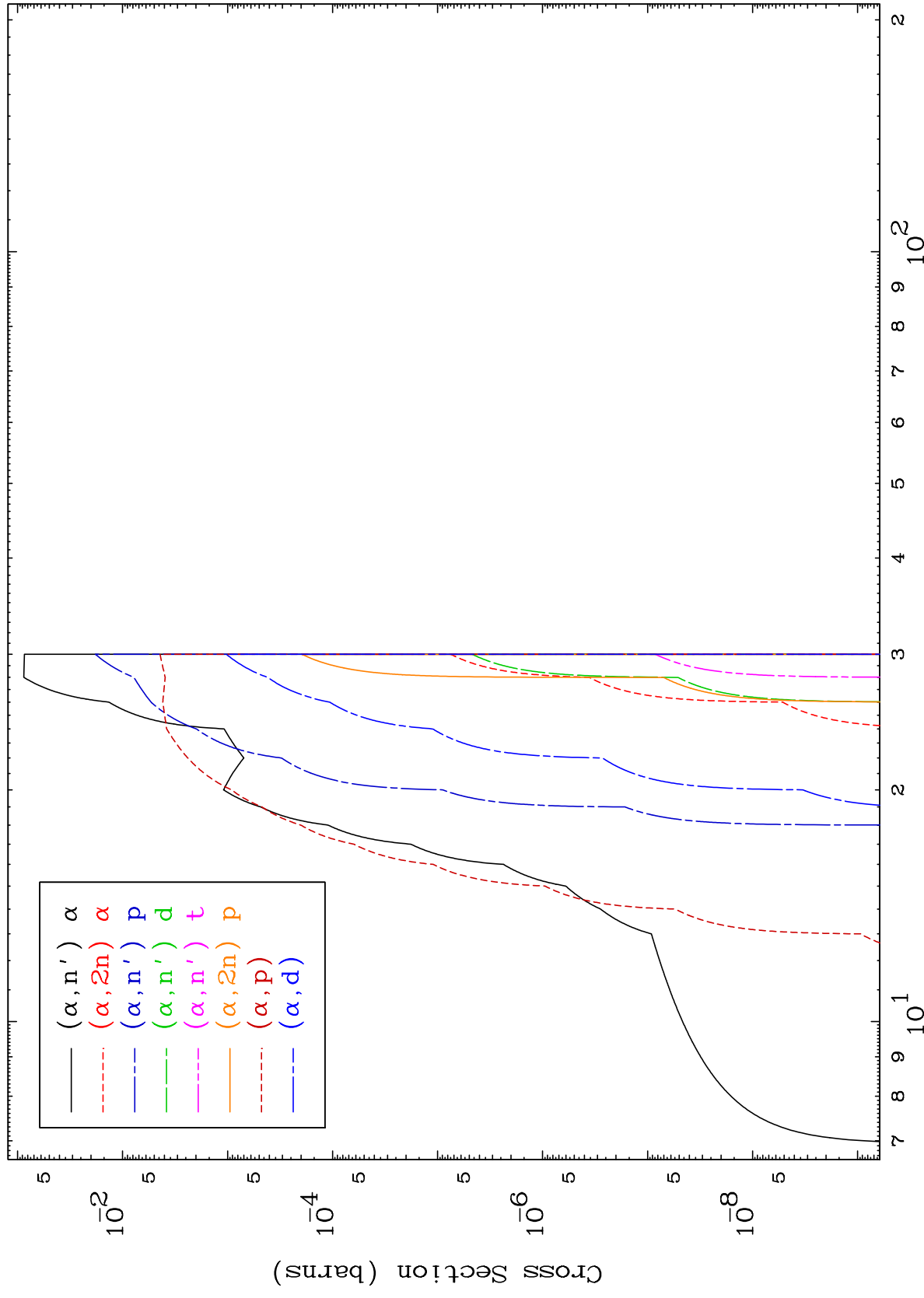
62-Sm-145



2

Incident Energy (MeV)

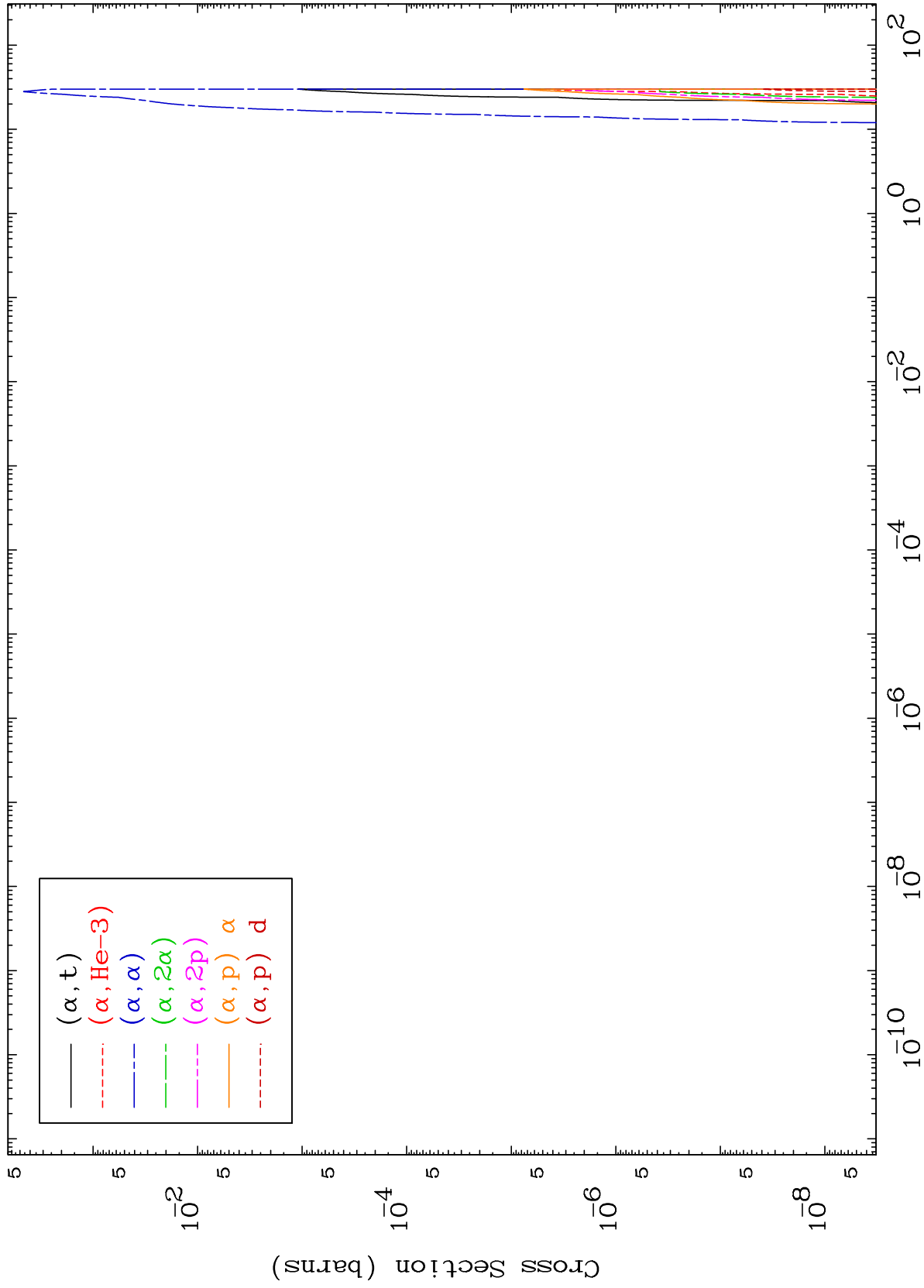
62-Sm-145



MAT 6228

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

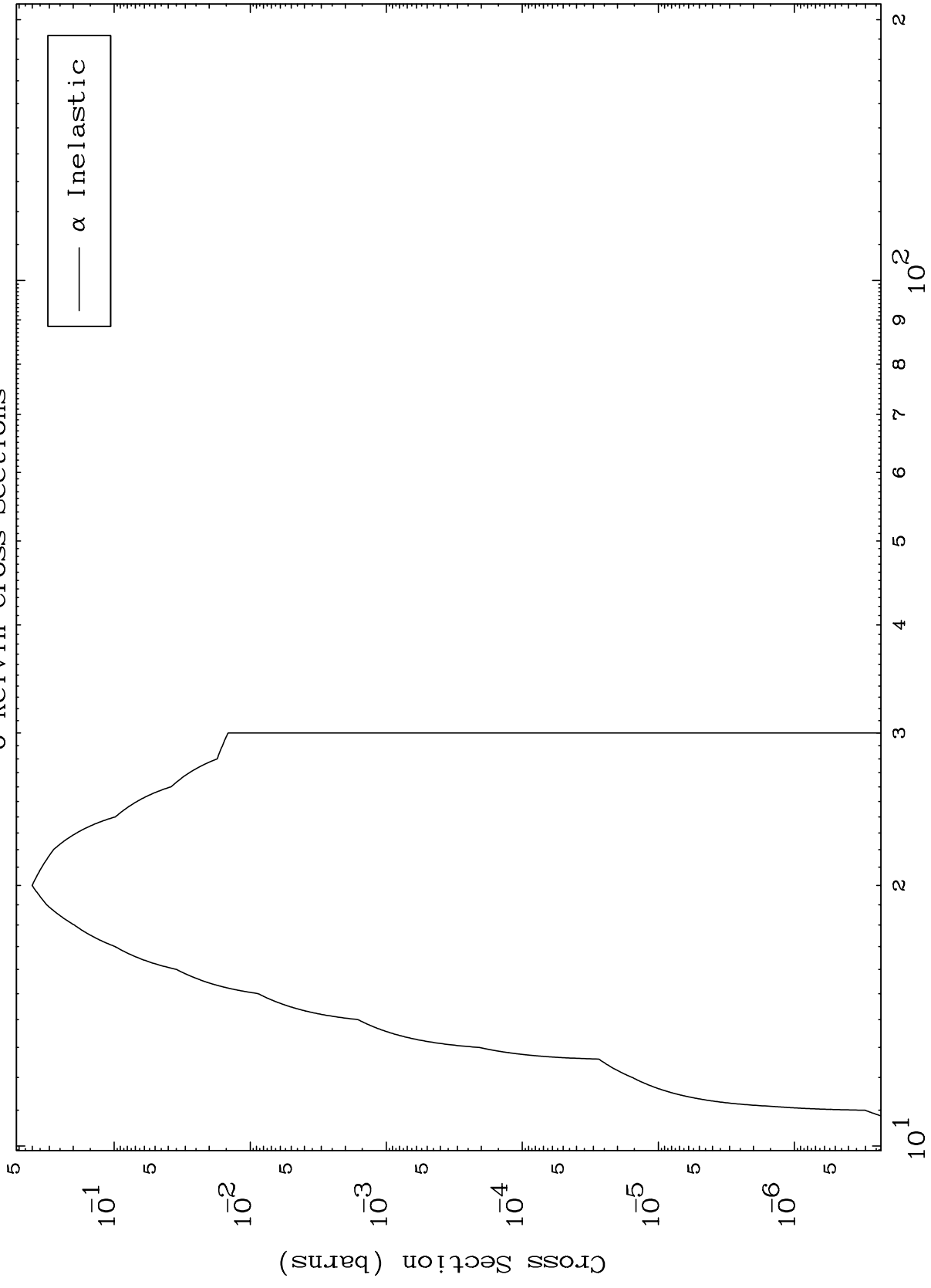
62-Sm-145



MAT 6228

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

62-Sm-145



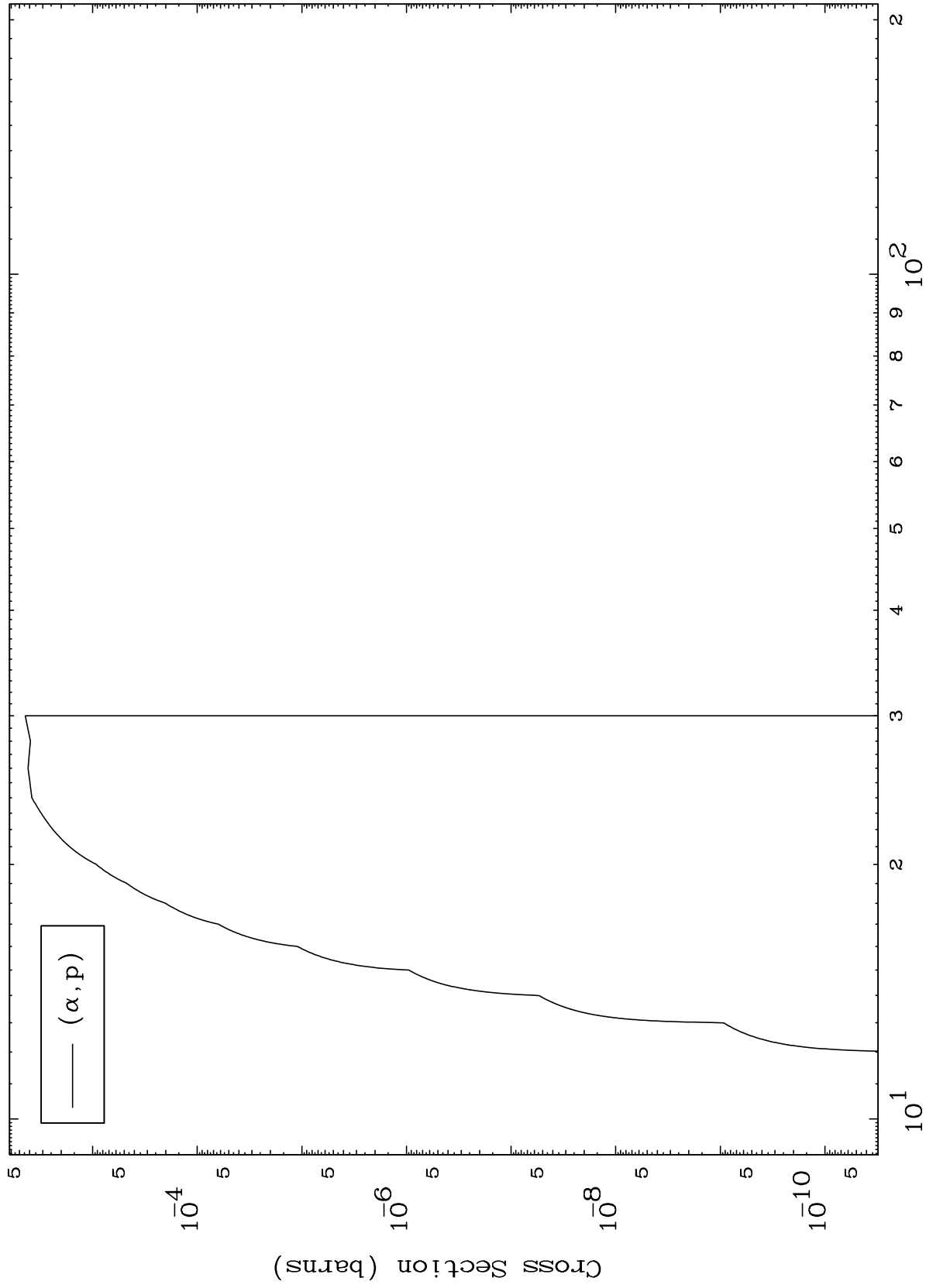
Incident Energy (MeV)

62-Sm-145

MAT 6228

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

62-Sm-145



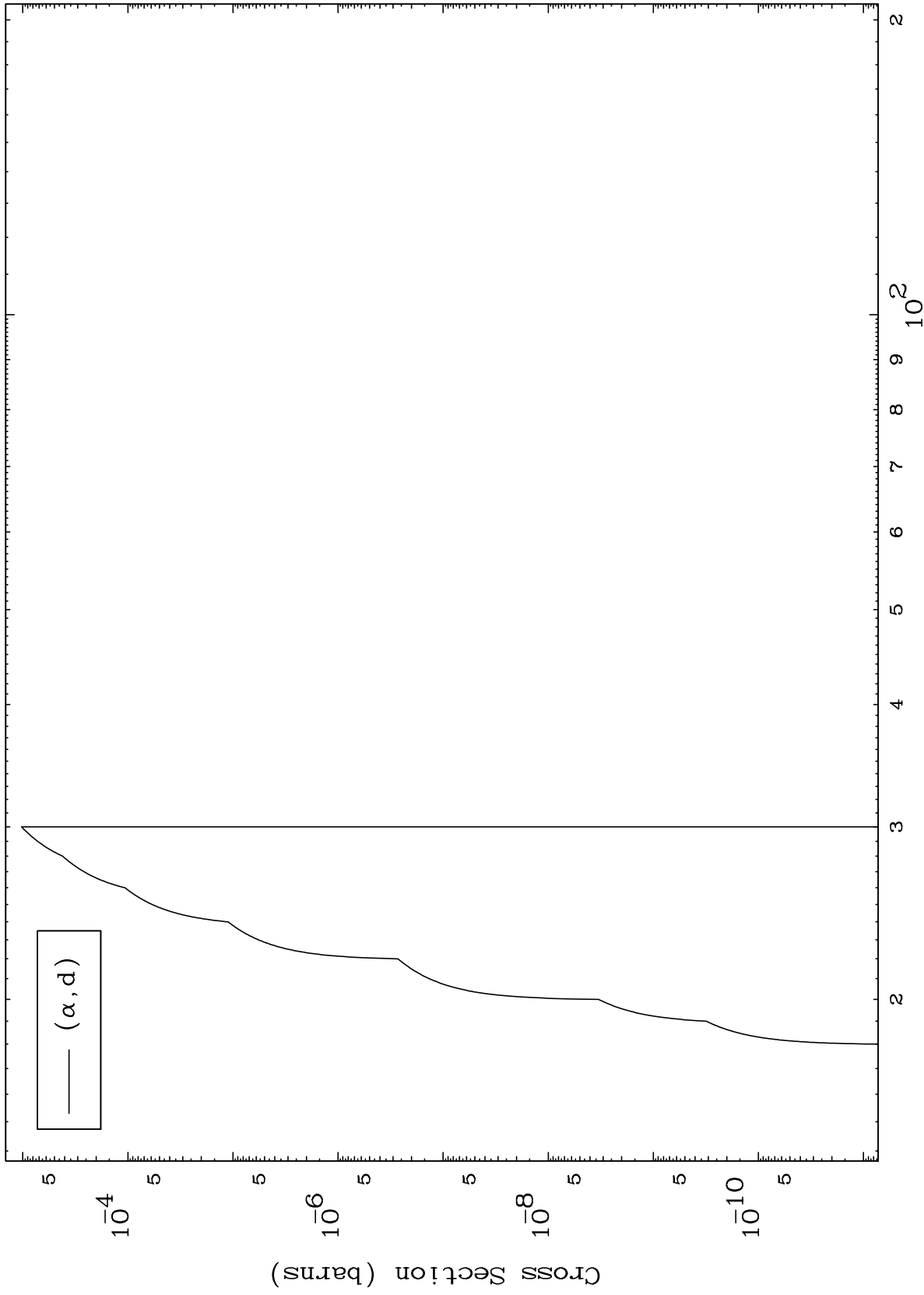
Incident Energy (MeV)

62-Sm-145

MAT 6228

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

62-Sm-145



7

Incident Energy (MeV)

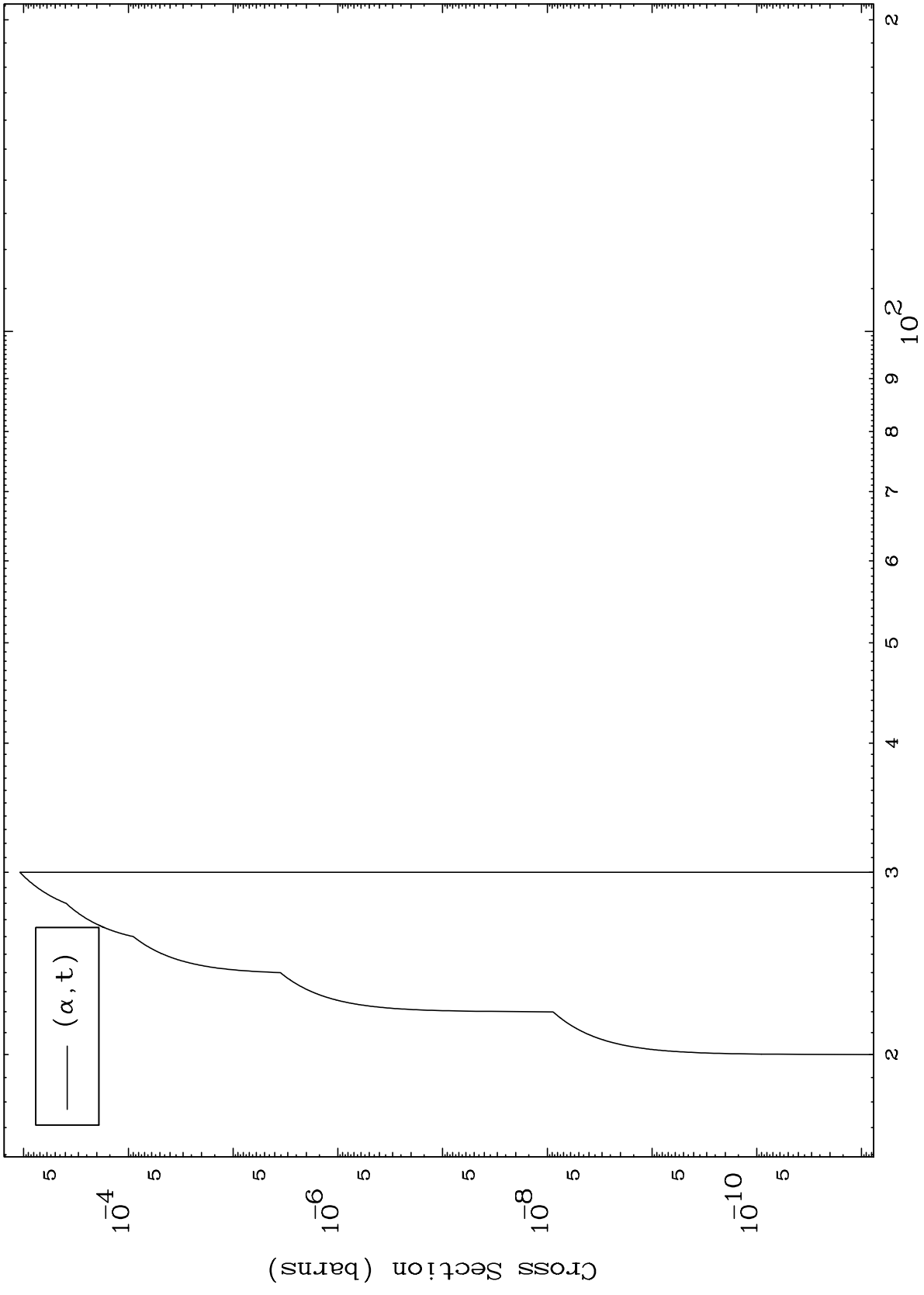
62-Sm-145



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( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

62-Sm-145



8

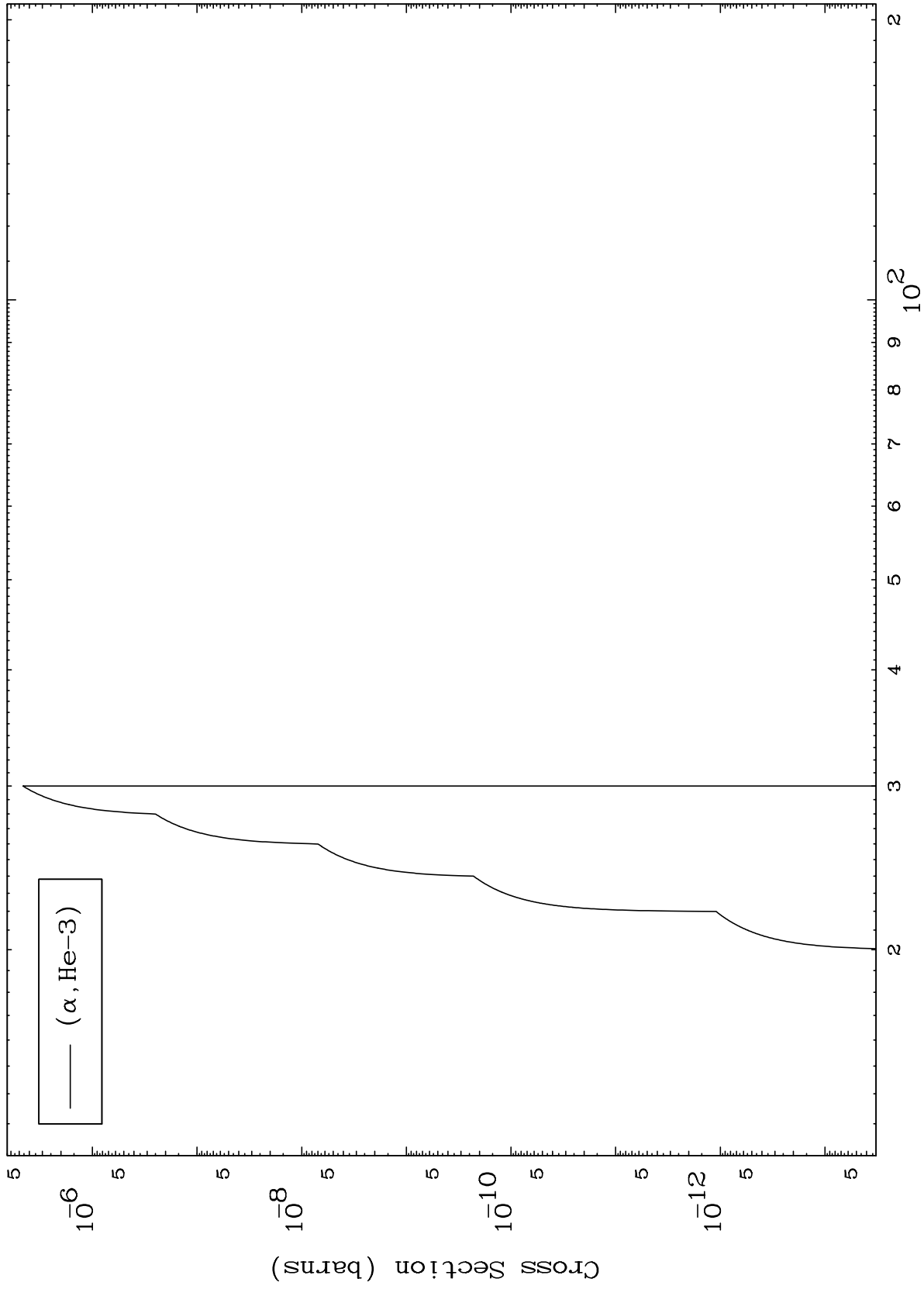
Incident Energy (MeV)

62-Sm-145

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( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

62-Sm-145



9

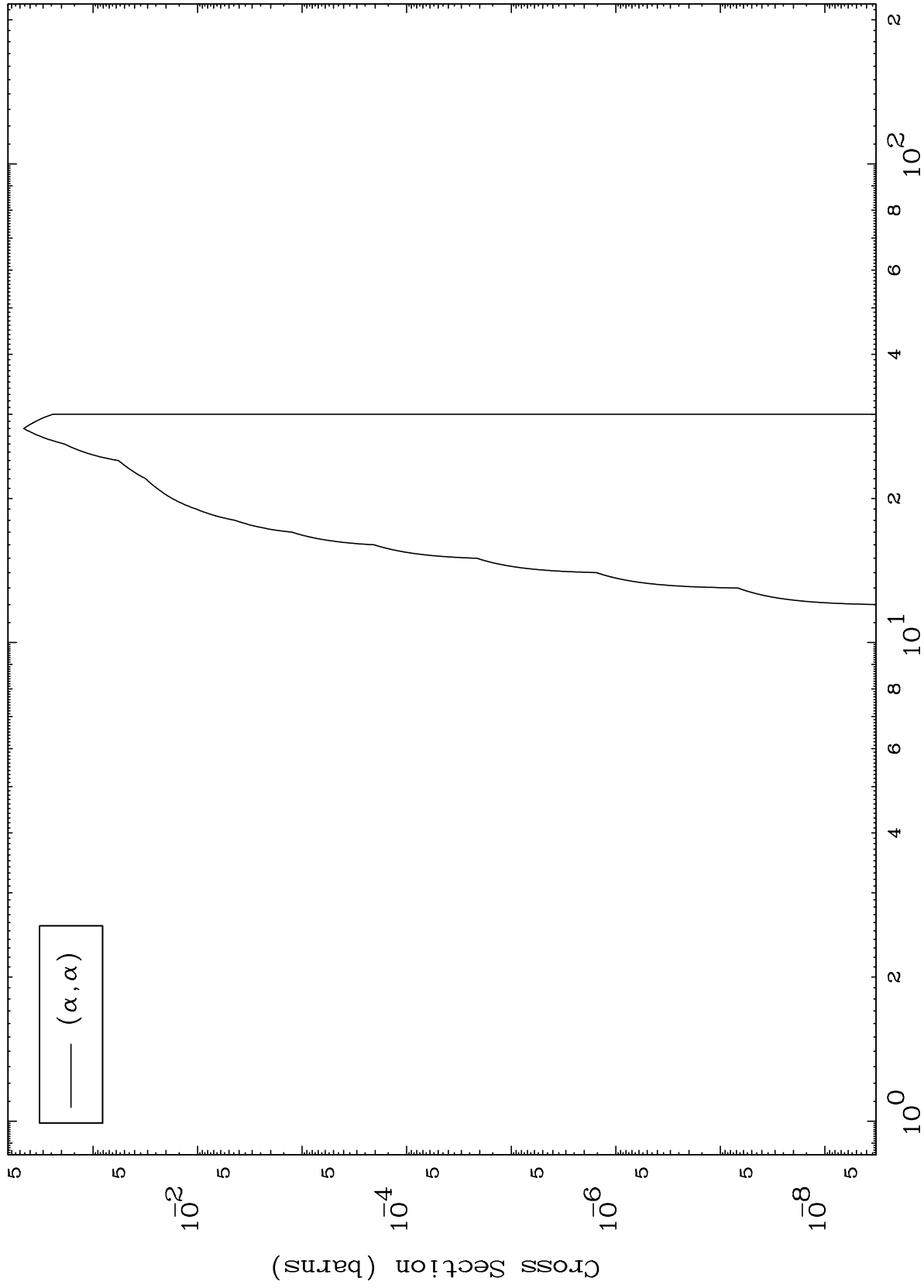
Incident Energy (MeV)

62-Sm-145

MAT 6228

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

62-Sm-145



Incident Energy (MeV)

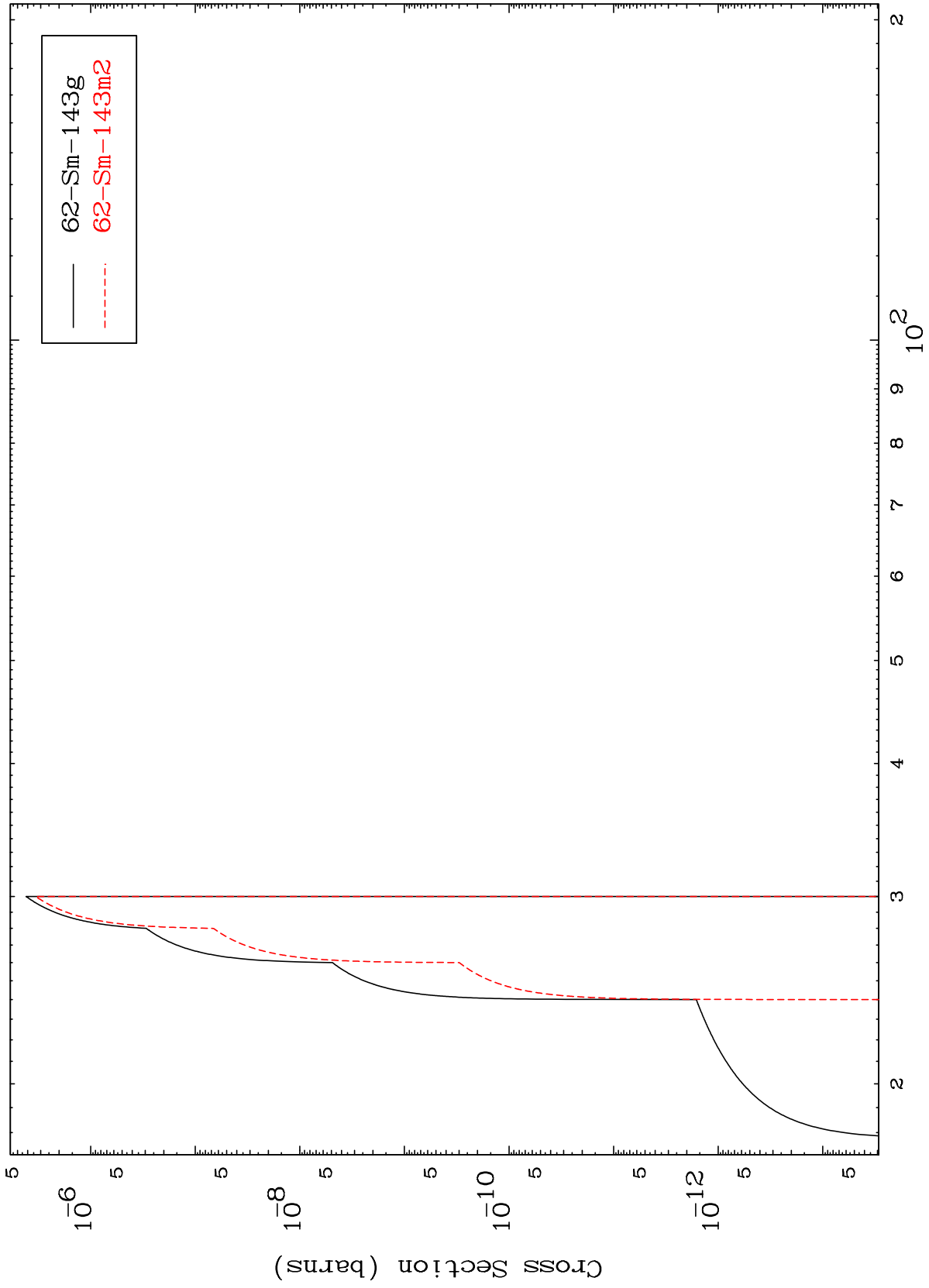
62-Sm-145

MAT 6228

$(\alpha, 2n) \alpha$

62-Sm-145

Radionuclide Production Cross Section



11

Incident Energy (MeV)

62-Sm-145

MAT 6228

( $\alpha, 2\alpha$ )

62-Sm-145

Radionuclide Production Cross Section

