

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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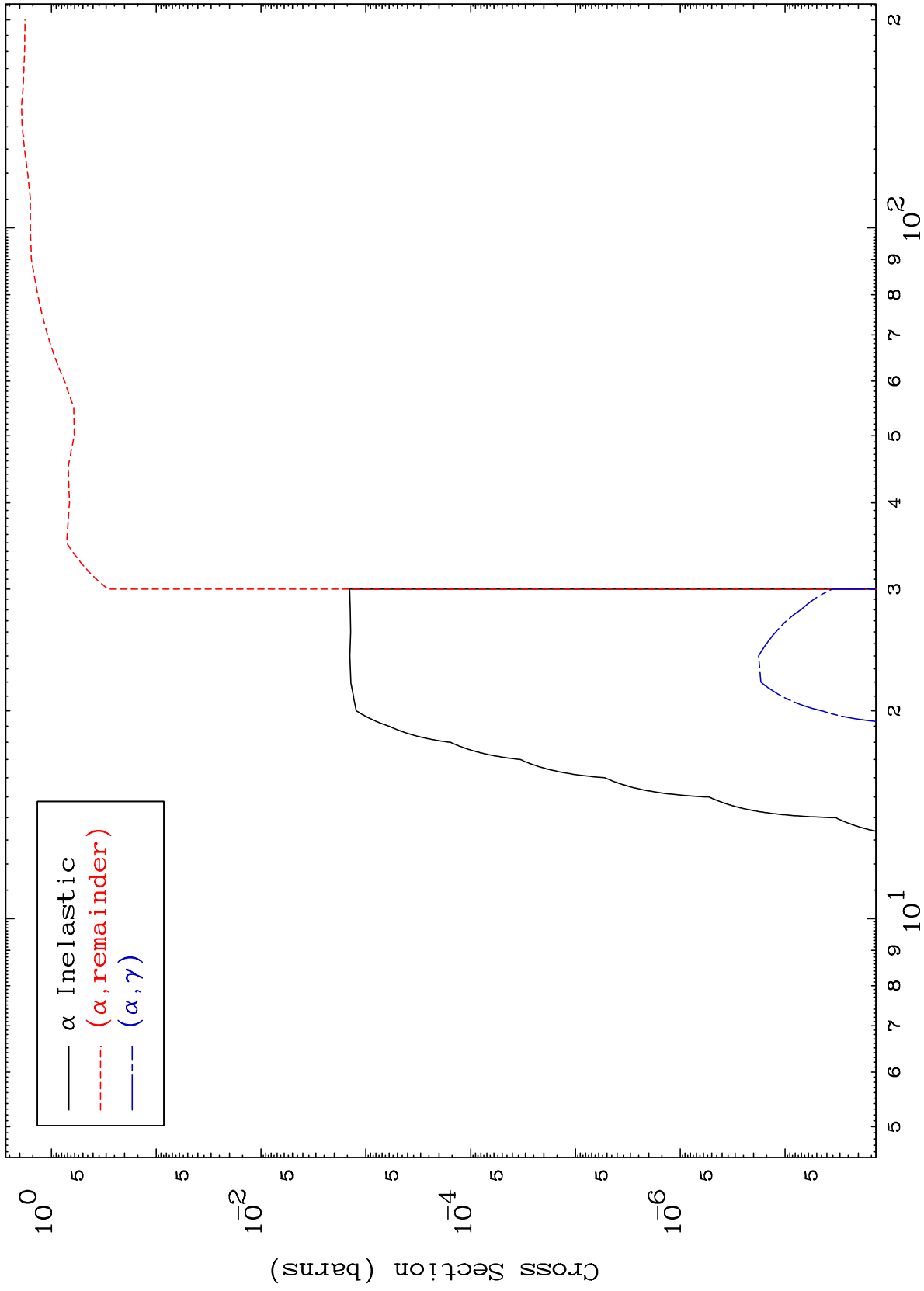
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9037

0 Kelvin  $\alpha$  Major Cross Sections

90-Th-231



1

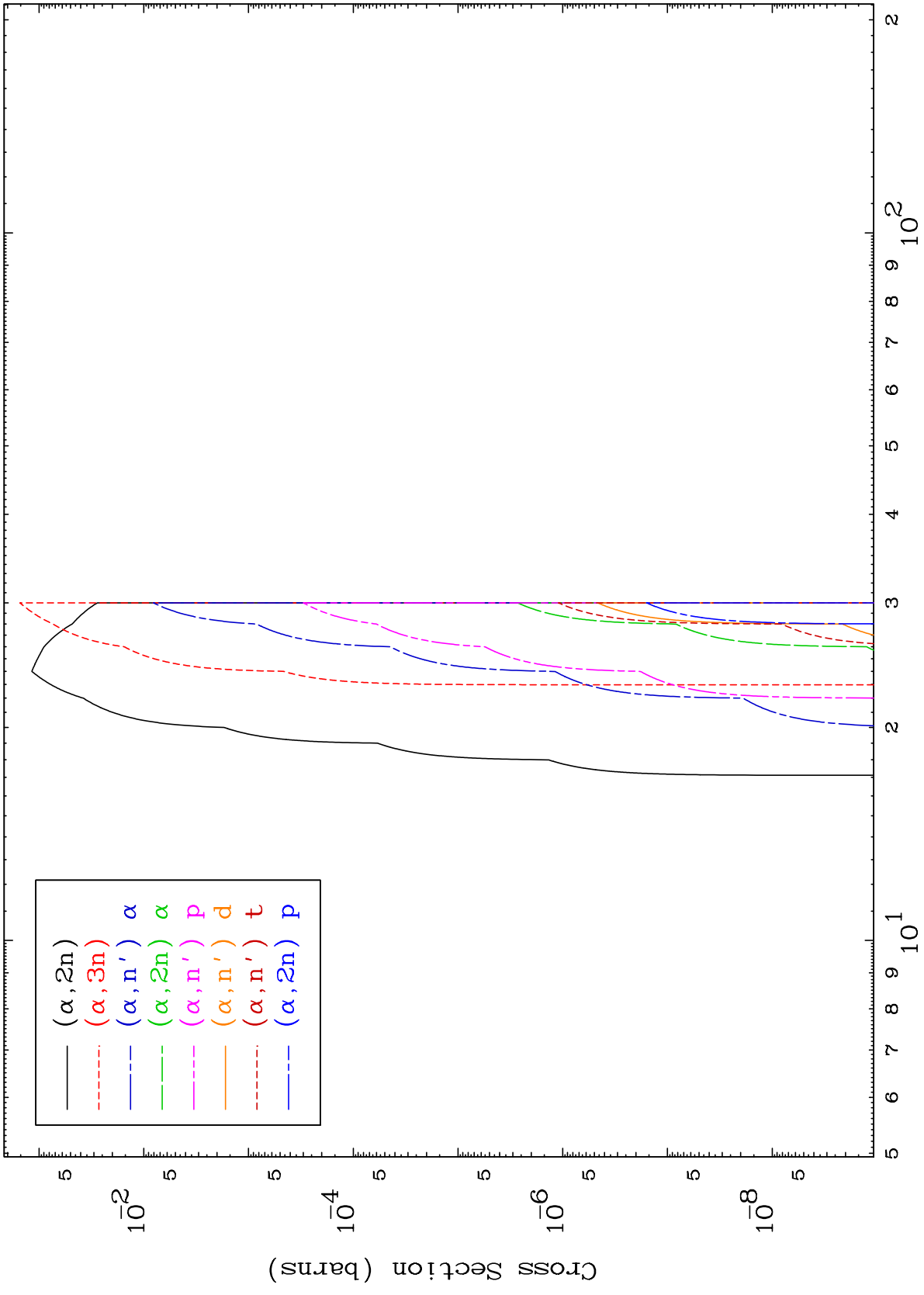
Incident Energy (MeV)

90-Th-231

MAT 9037

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

90-Th-231



2

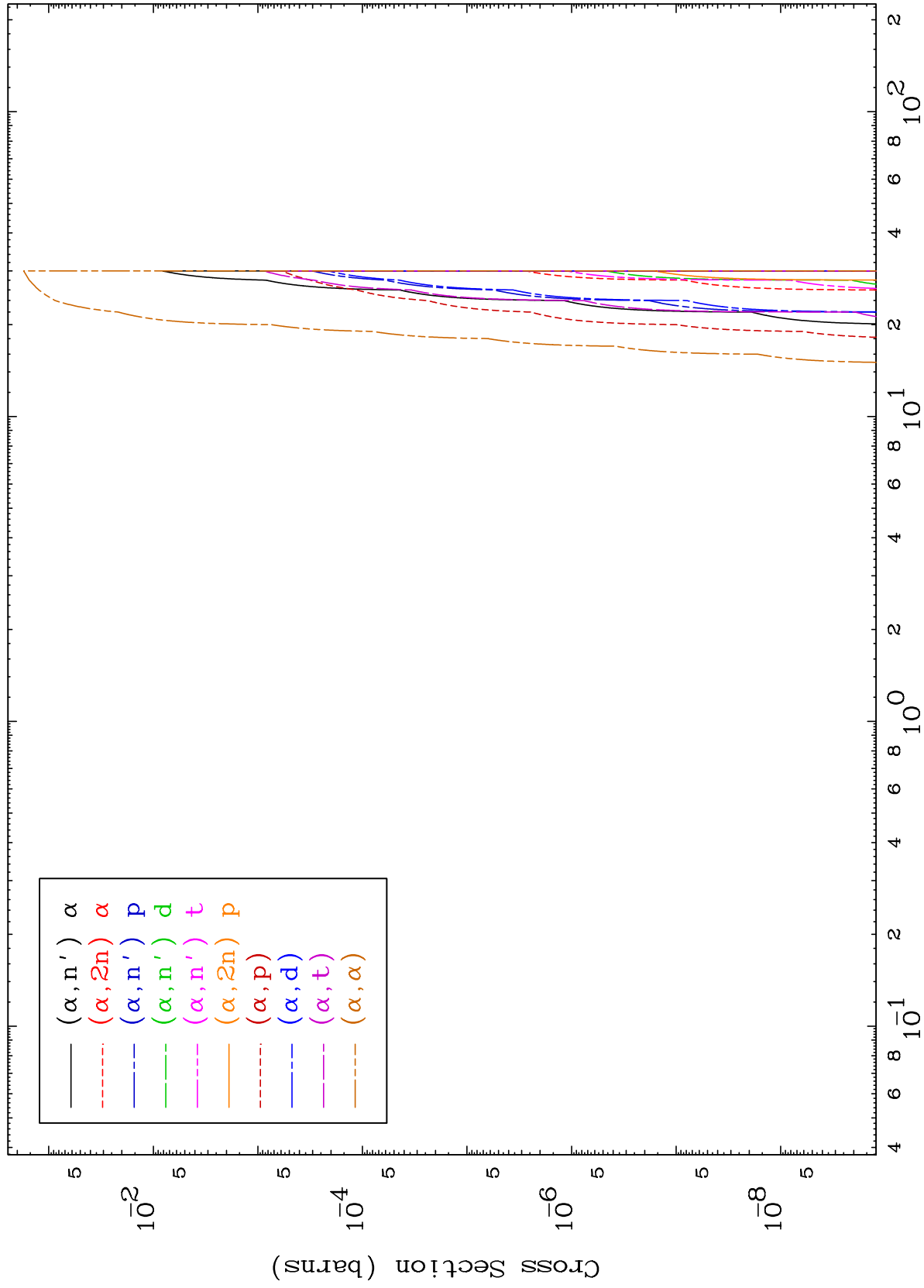
Incident Energy (MeV)

90-Th-231

MAT 9037

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

90-Th-231



3

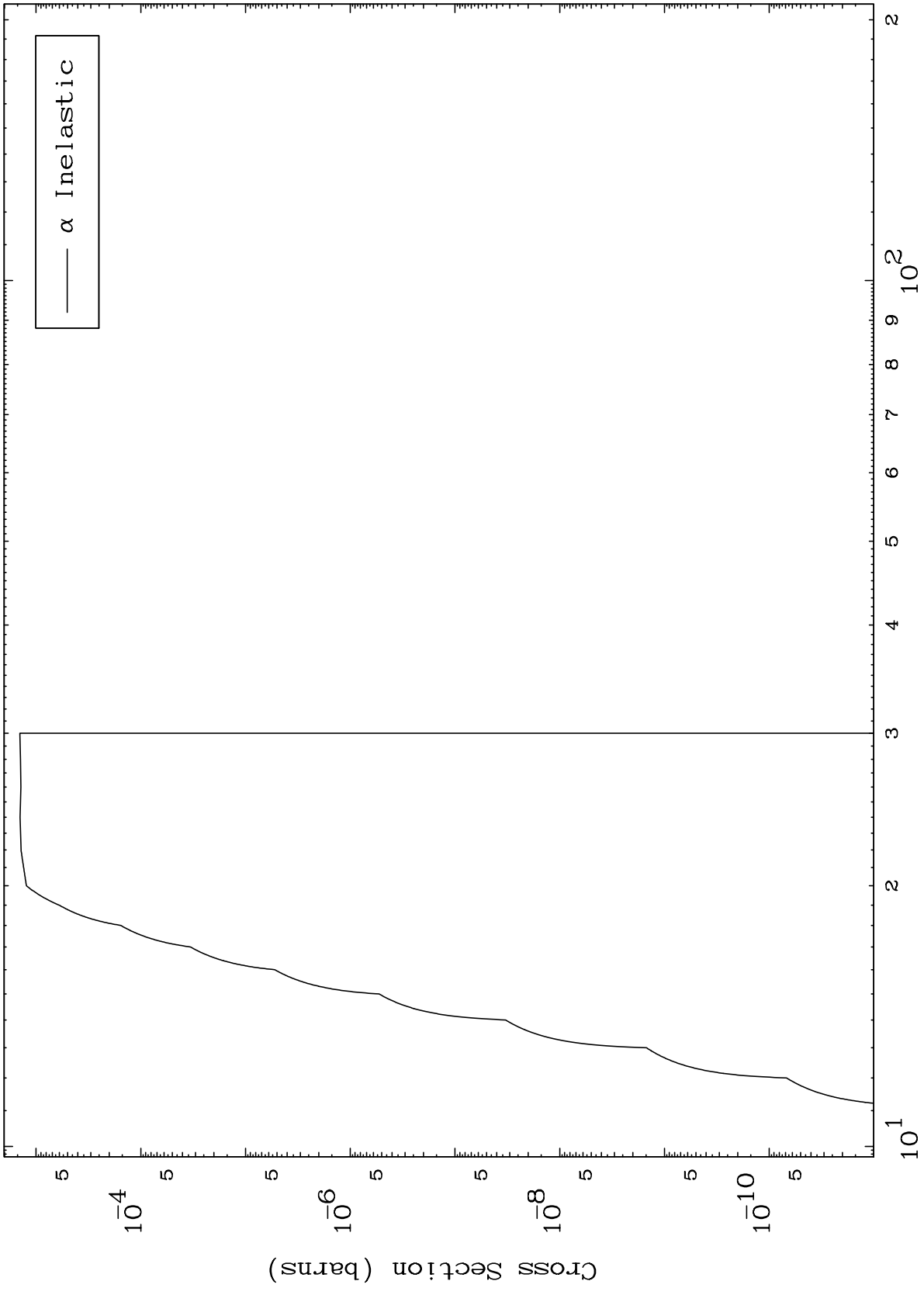
Incident Energy (MeV)

90-Th-231

MAT 9037

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

90-Th-231



90-Th-231

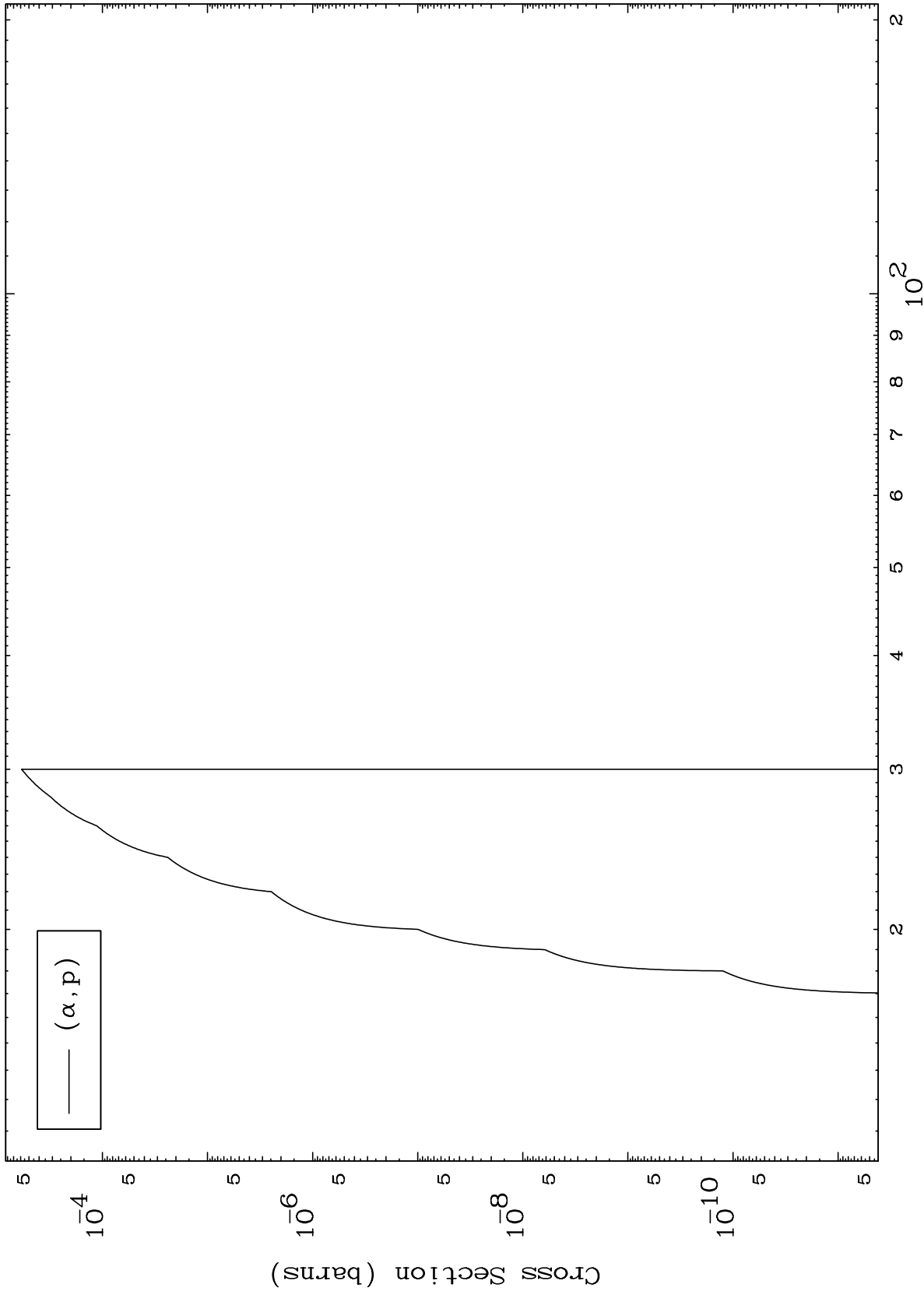
Incident Energy (MeV)

4

MAT 9037

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

90-Th-231



5

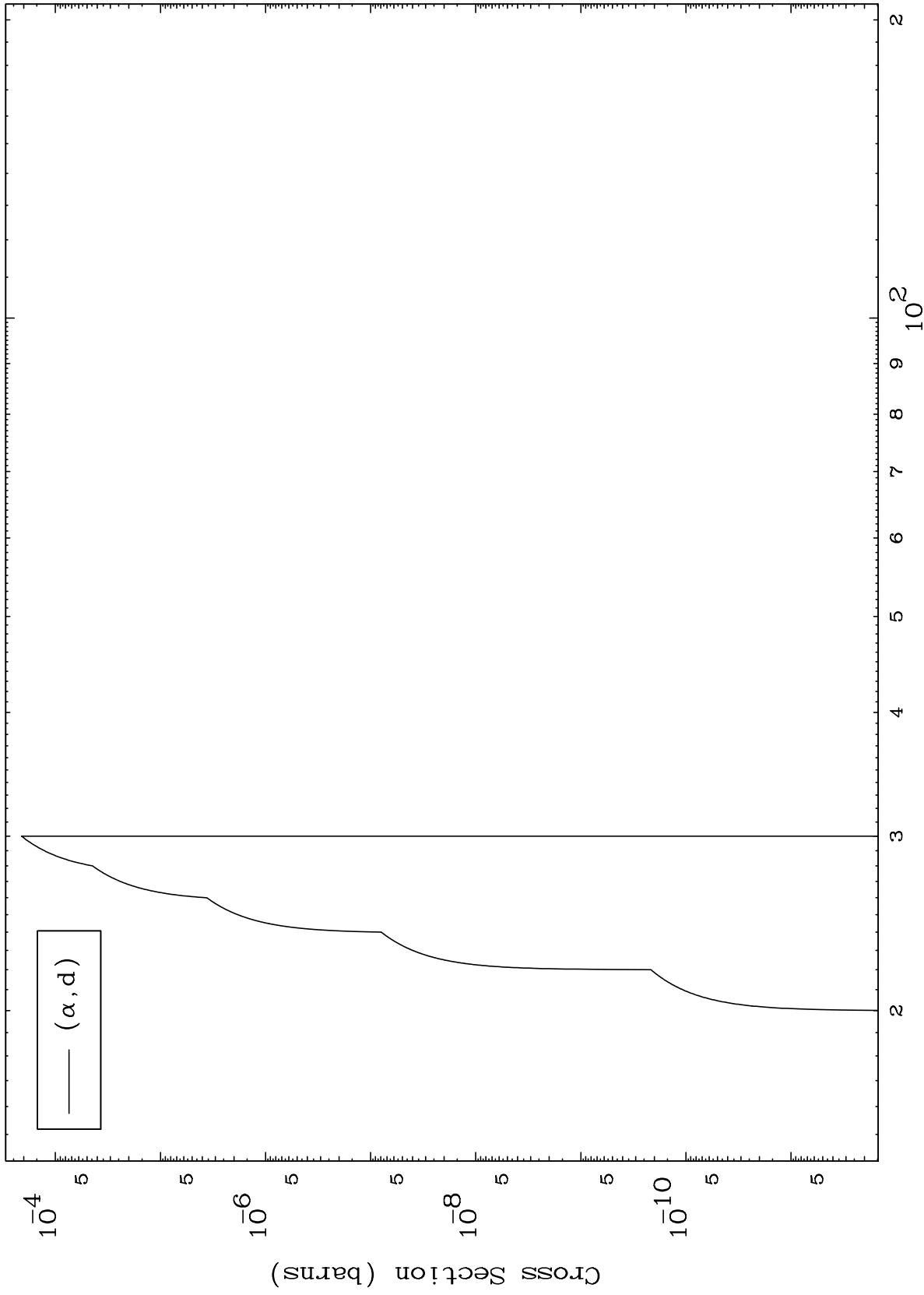
Incident Energy (MeV)

90-Th-231

MAT 9037

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

90-Th-231



6

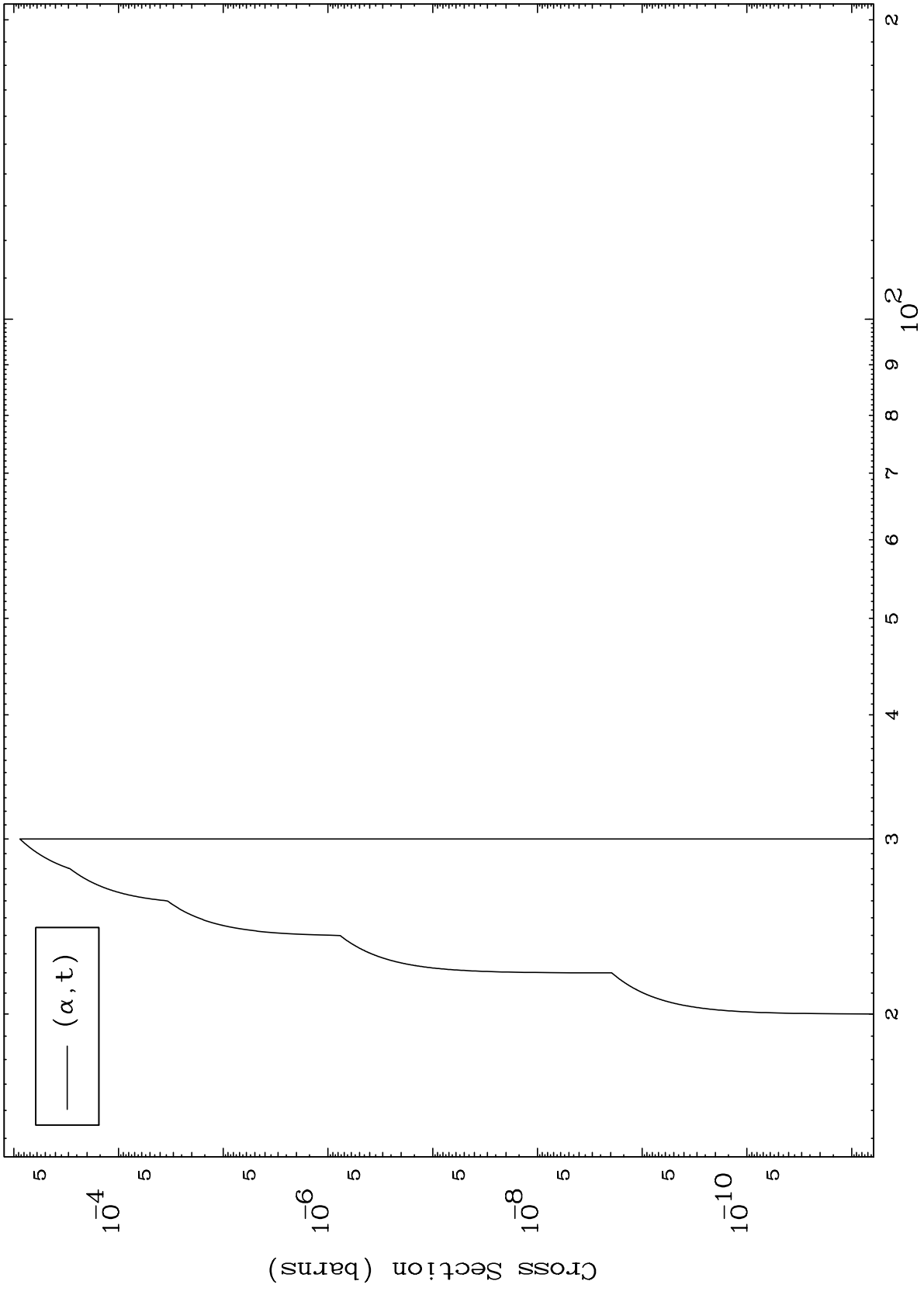
Incident Energy (MeV)

90-Th-231

MAT 9037

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

90-Th-231



7

Incident Energy (MeV)

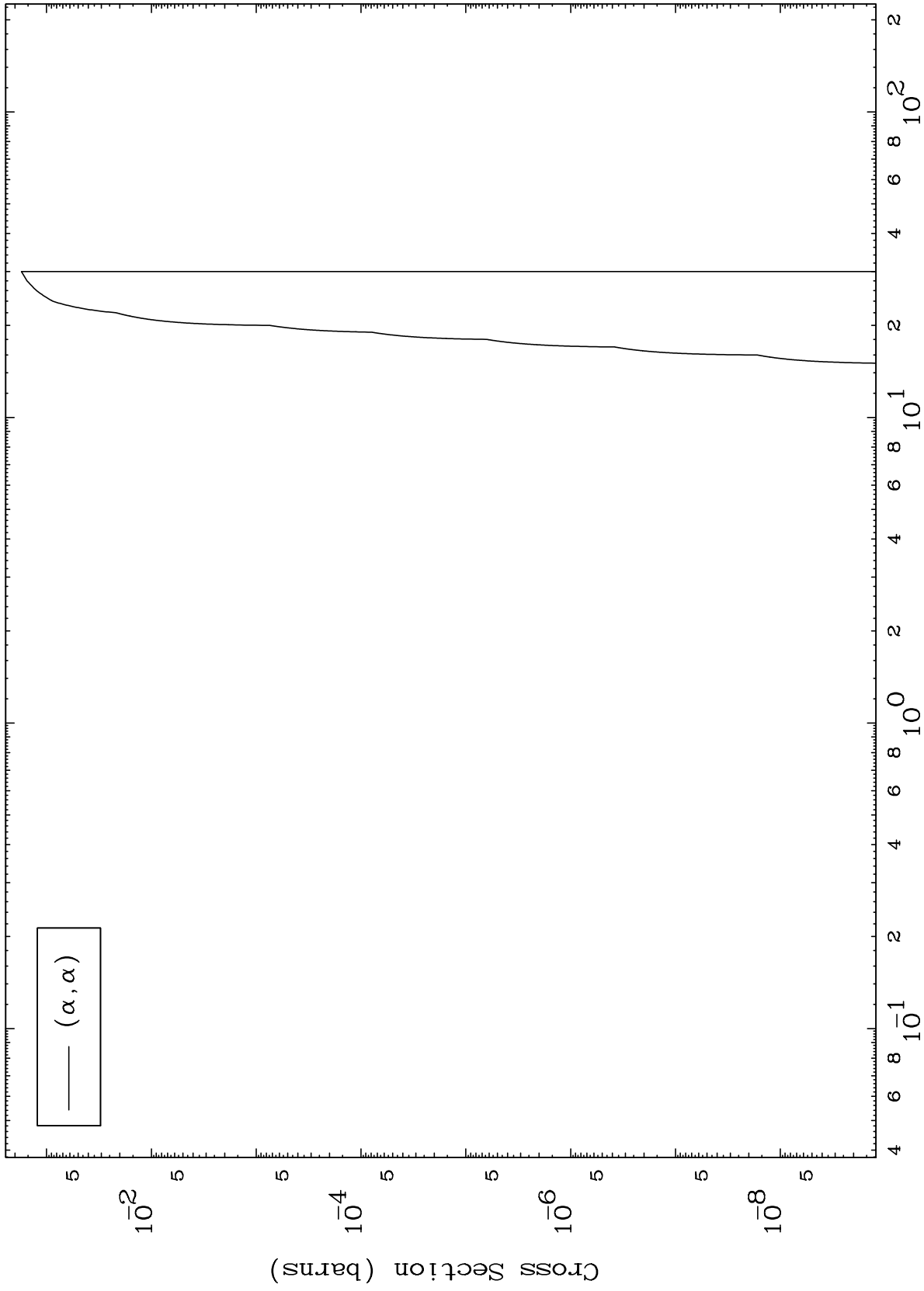
90-Th-231



MAT 9037

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

90-Th-231



8

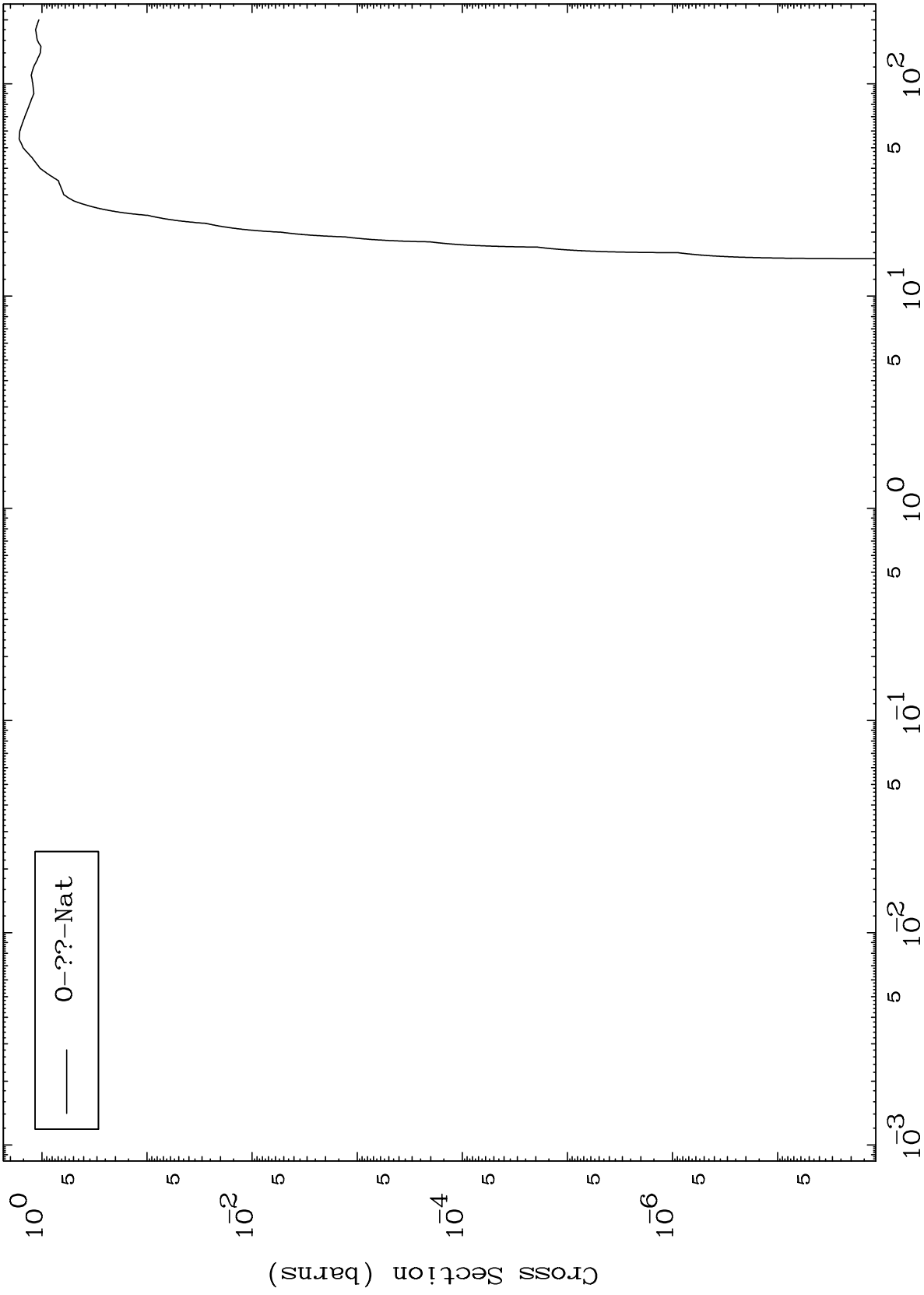
Incident Energy (MeV)

90-Th-231

MAT 9037

$\alpha$  Fission  
Radionuclide Production Cross Section

90-Th-231



0-?-?-Nat

Incident Energy (MeV)

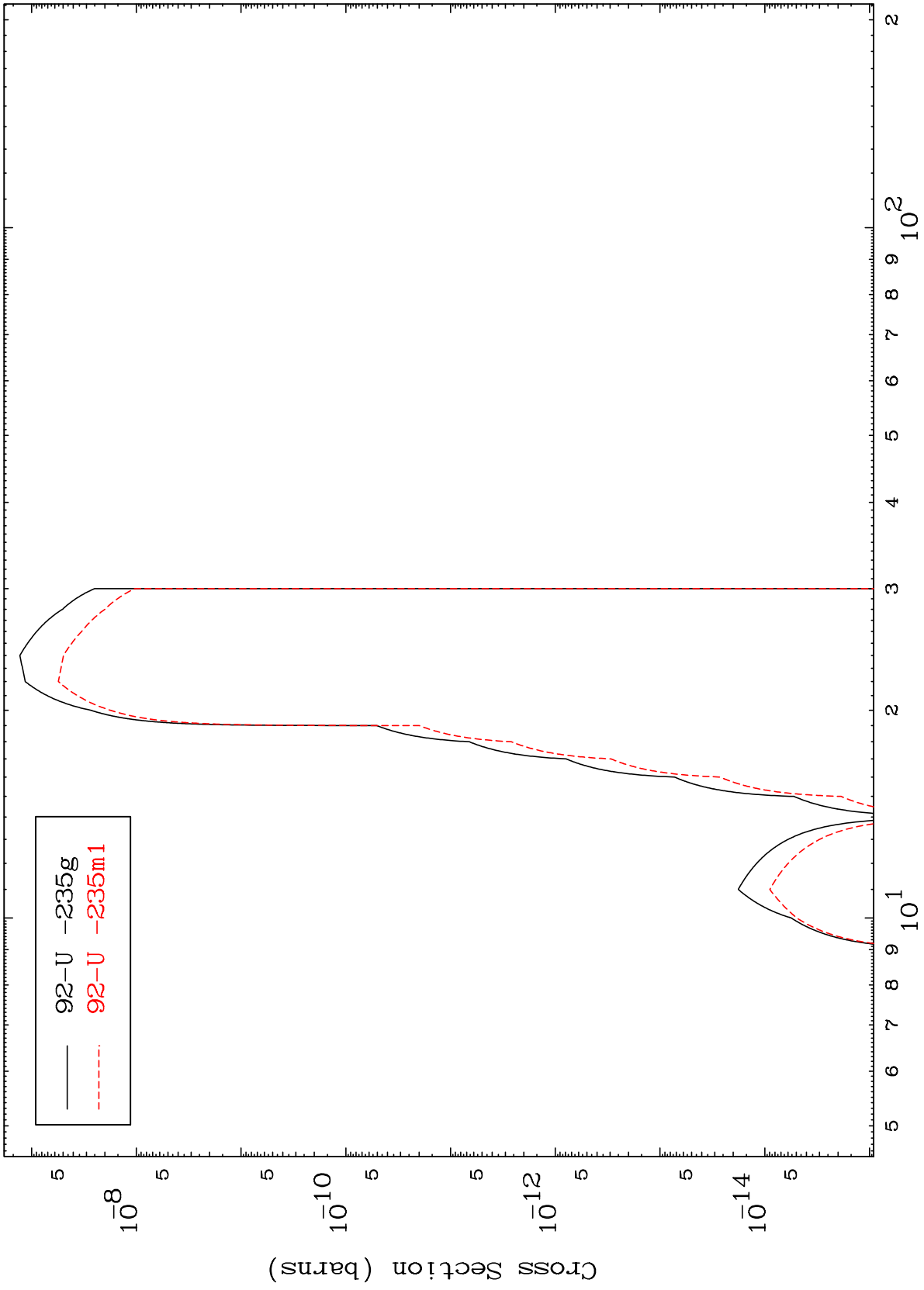
90-Th-231

9

MAT 9037

90-Th-231

Radionuclide Production Cross Section



10

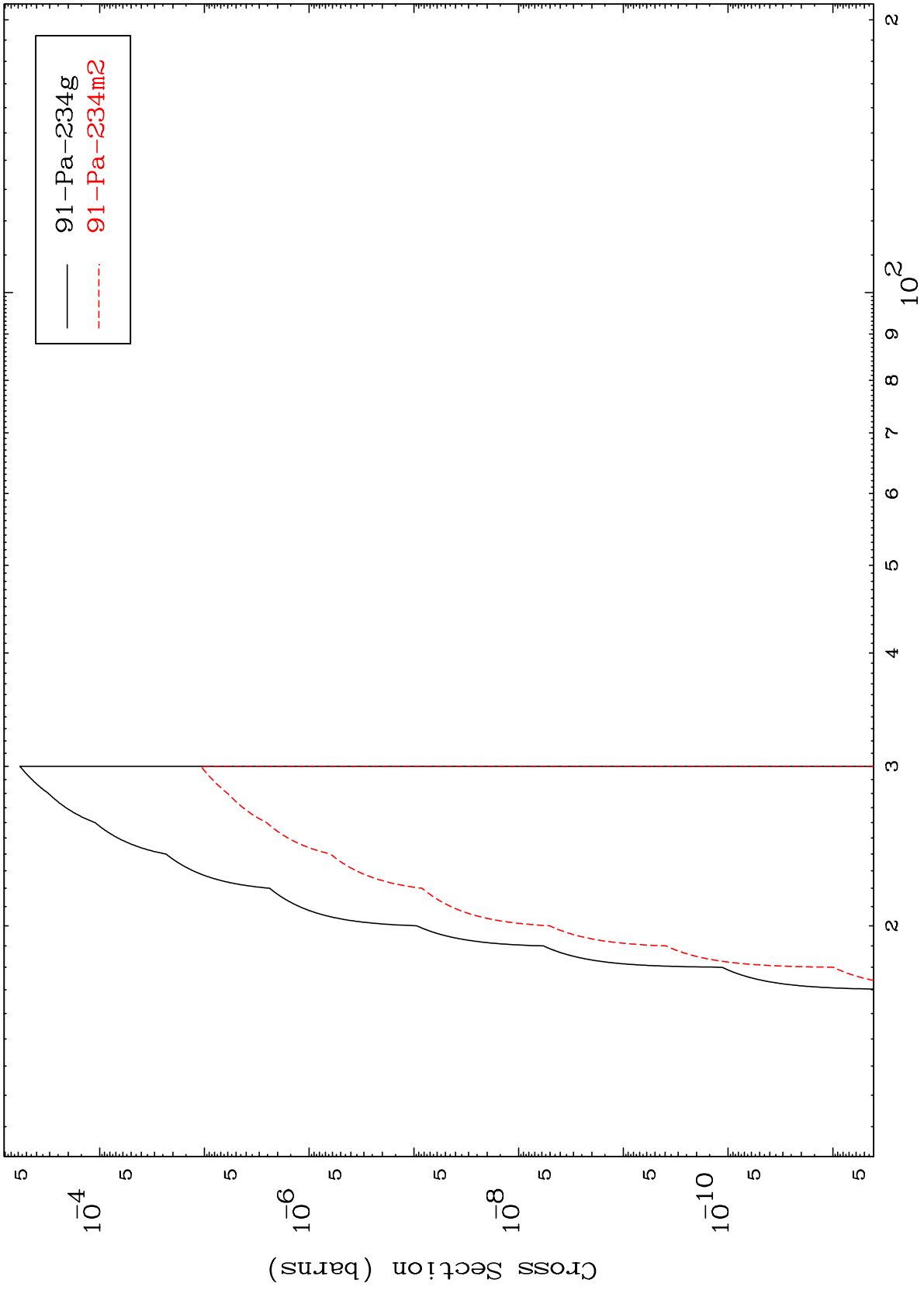
Incident Energy (MeV)

90-Th-231

MAT 9037

90-Th-231

( $\alpha, p$ )  
Radionuclide Production Cross Section



11

Incident Energy (MeV)

90-Th-231