

Program EVALPLOT
(Version 2018-1)

by

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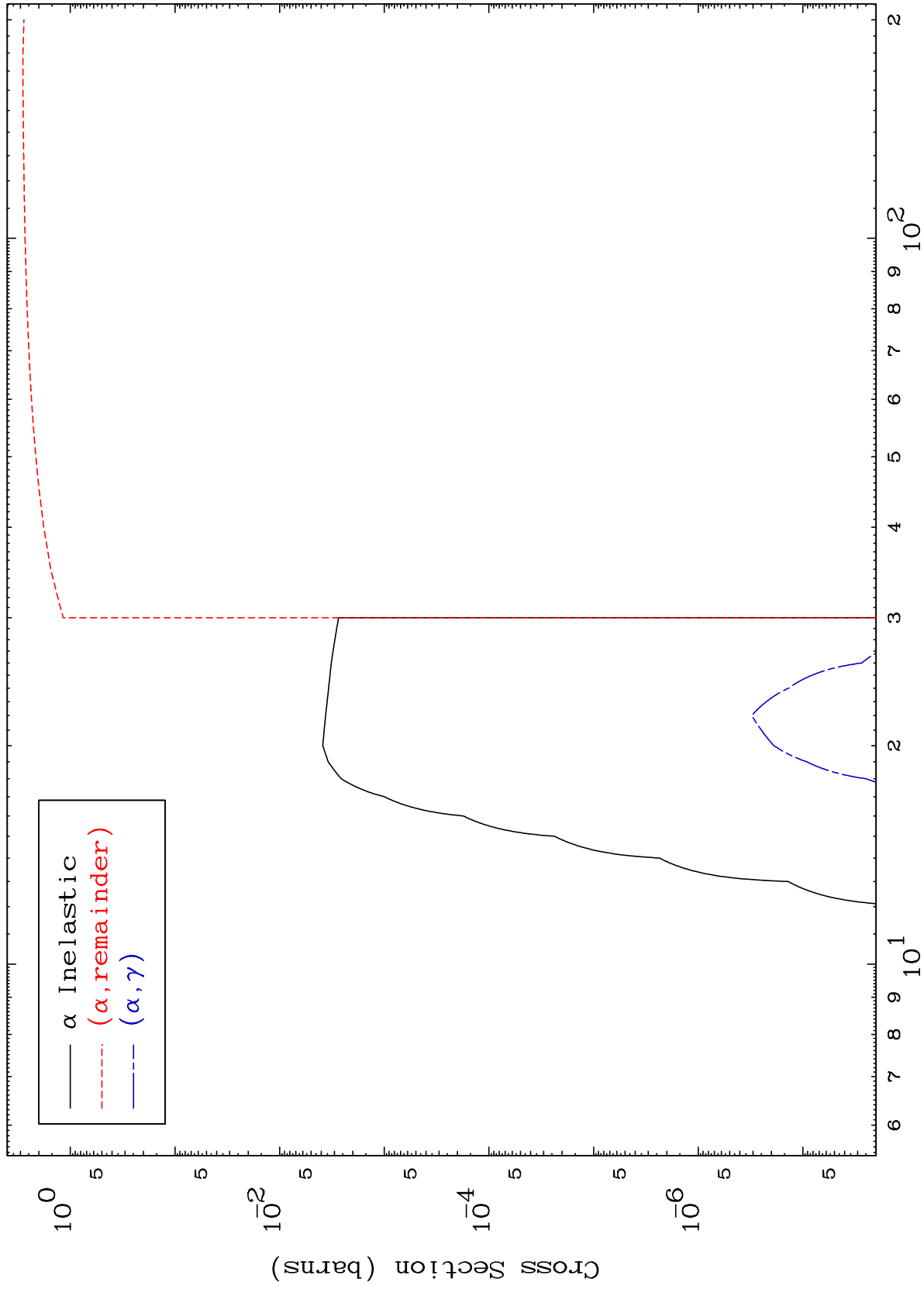
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8146

0 Kelvin α Major
Cross Sections

81-Tl-210



1

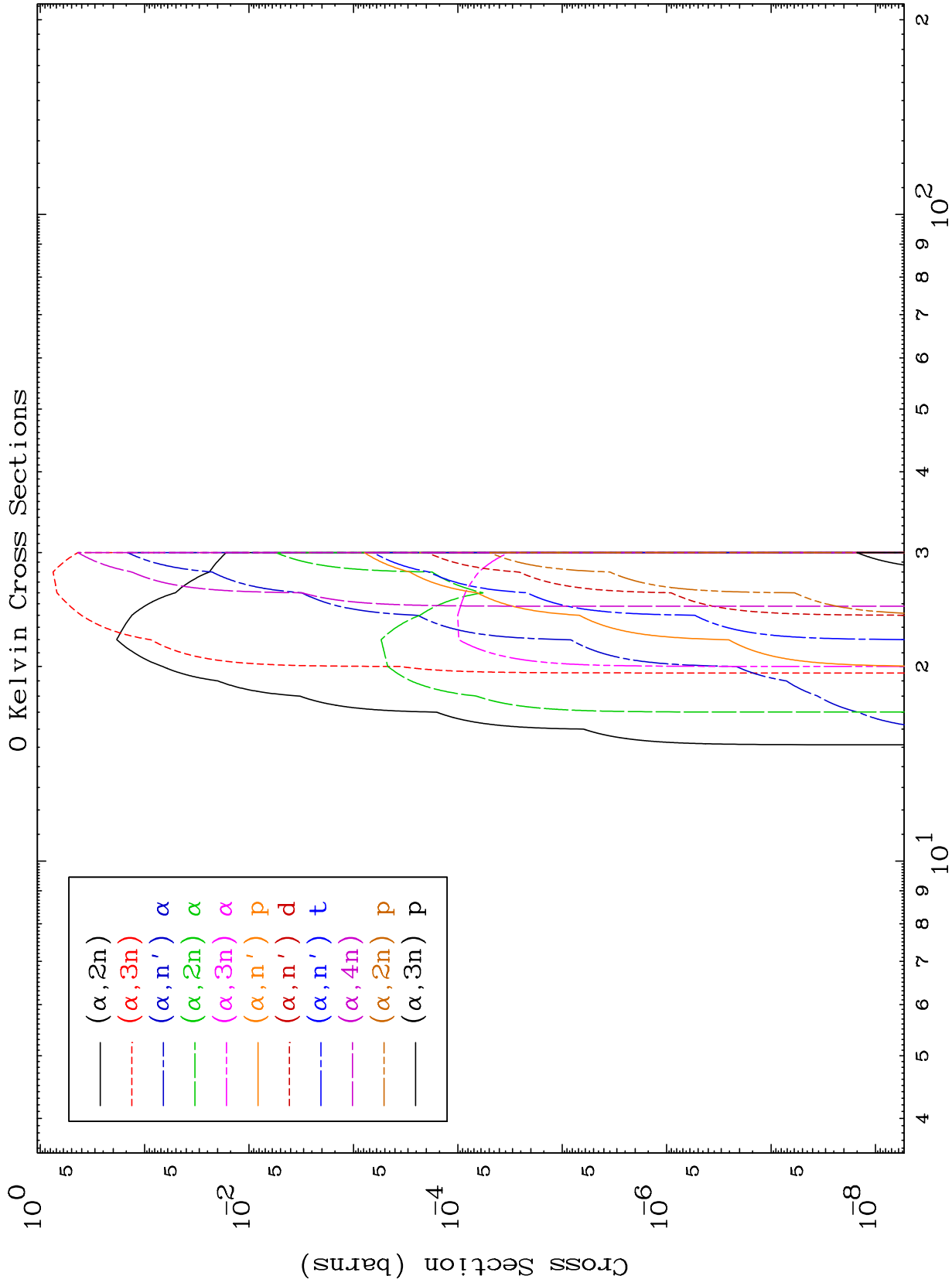
Incident Energy (MeV)

81-Tl-210

MAT 8146

α Neutron Production
0 Kelvin Cross Sections

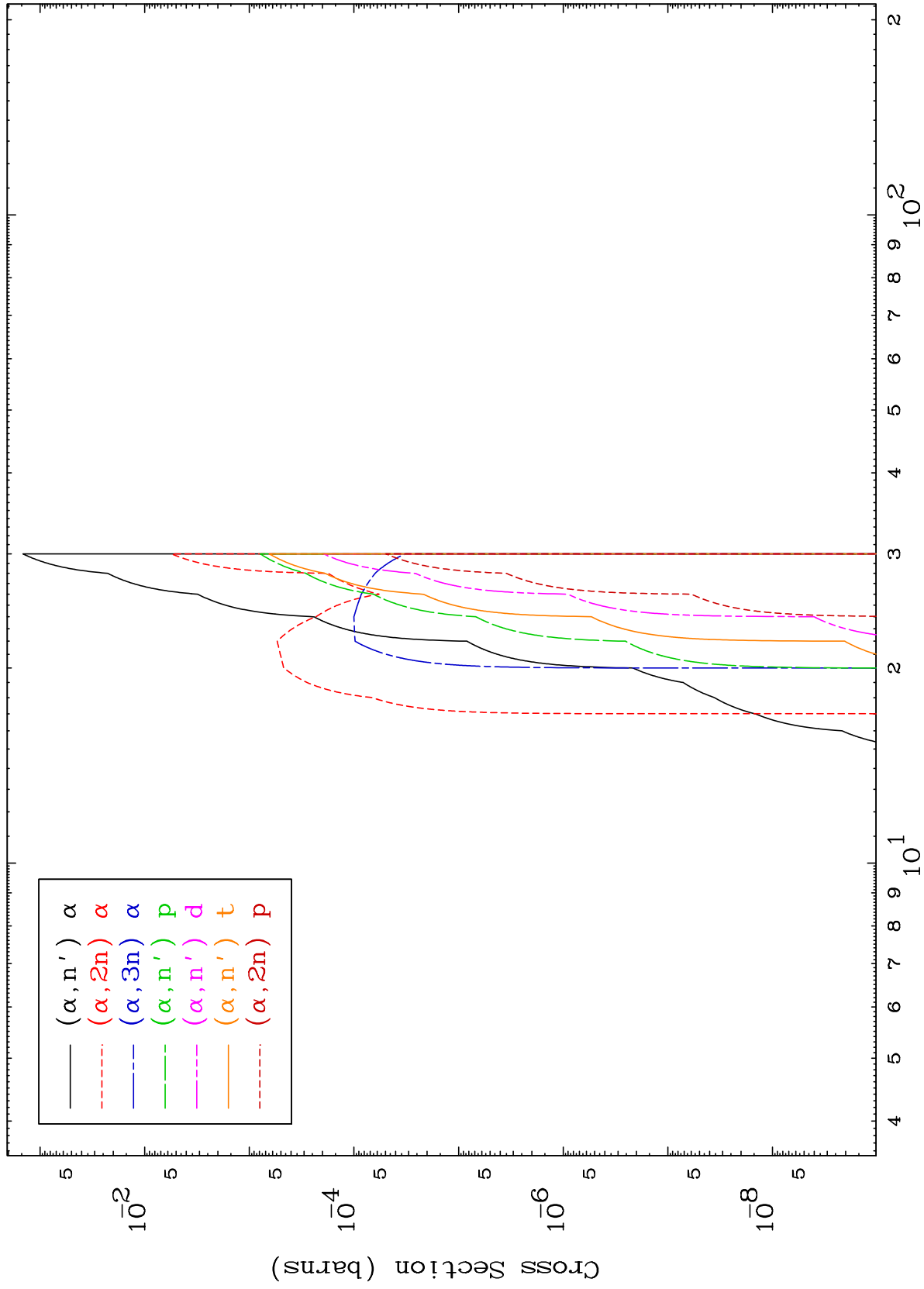
81-T1-210



2

Incident Energy (MeV)

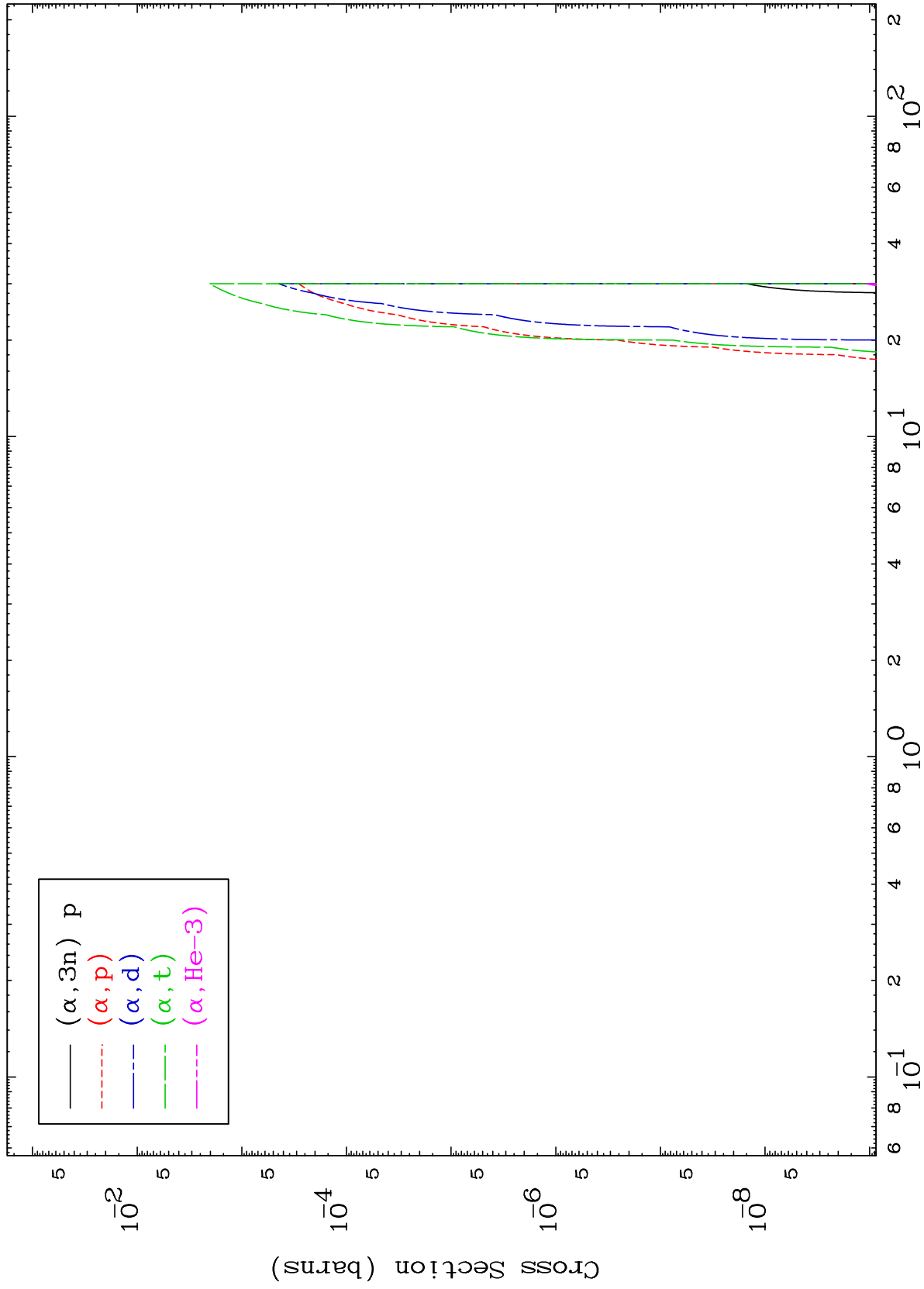
81-T1-210



MAT 8146

α Charged Particle
0 Kelvin Cross Sections

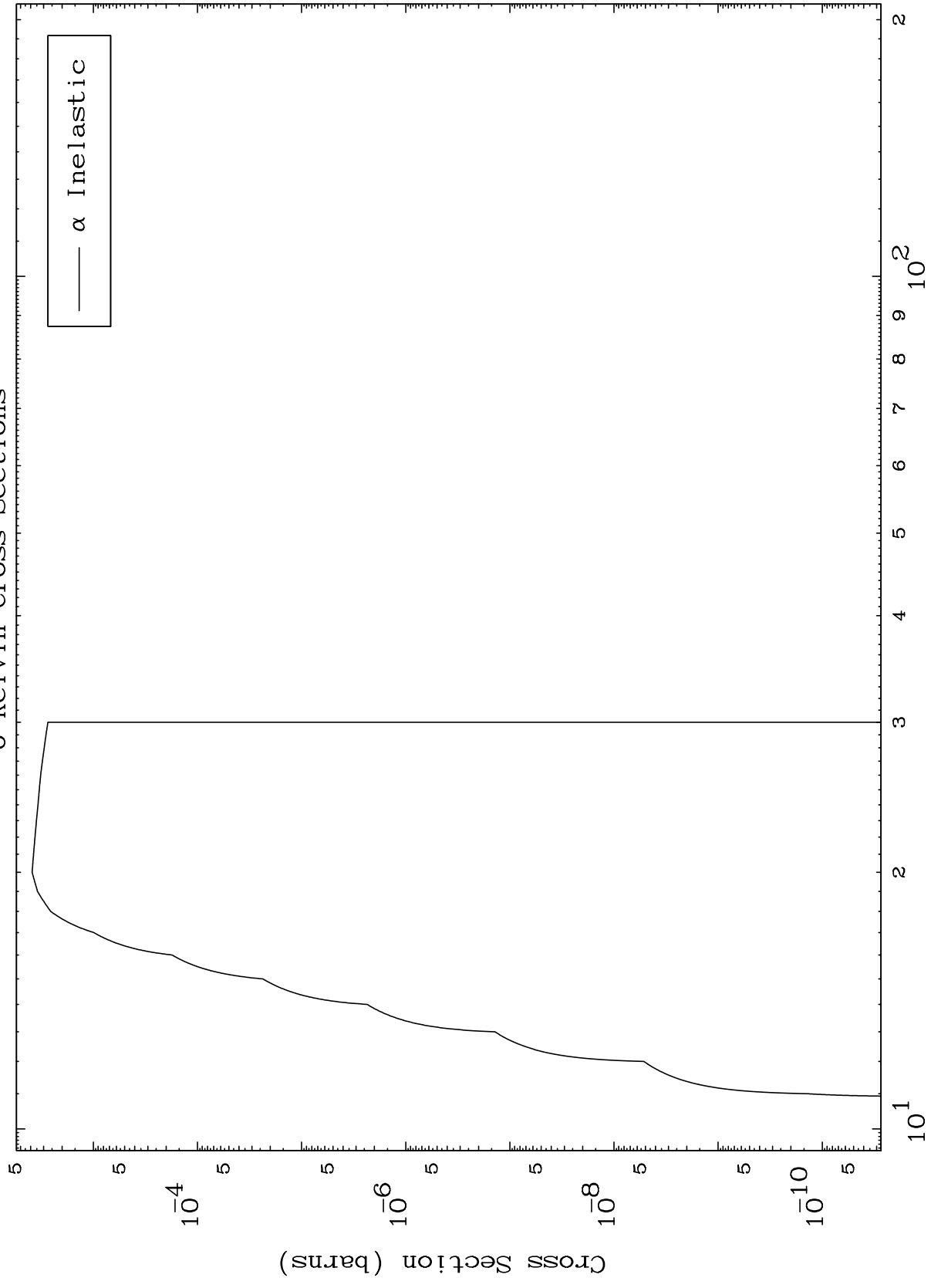
81-Tl-210



MAT 8146

(α, n') Level
0 Kelvin Cross Sections

81-T1-210



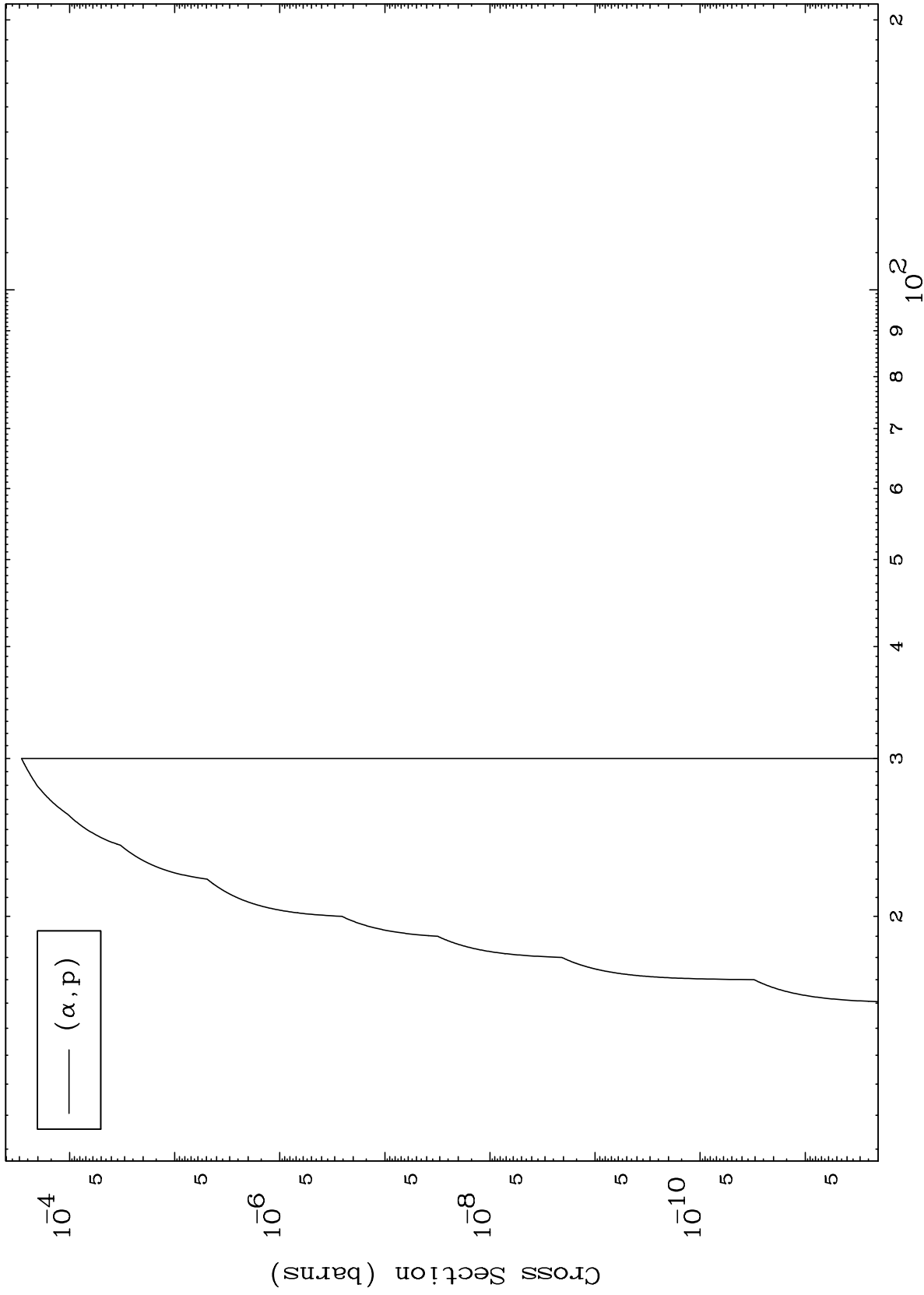
Incident Energy (MeV)

81-T1-210

MAT 8146

(α, p) Levels
0 Kelvin Cross Sections

81-T1-210



6

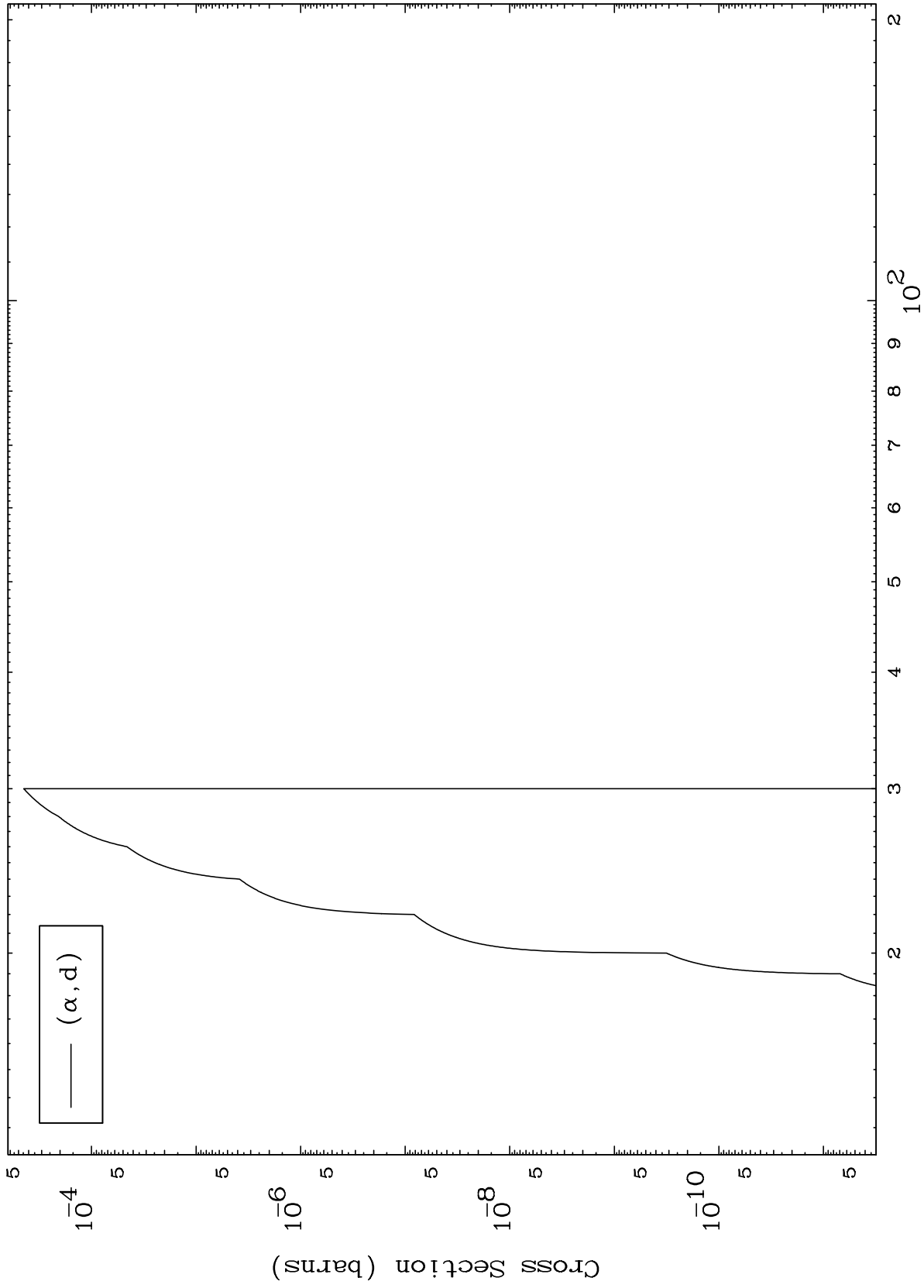
Incident Energy (MeV)

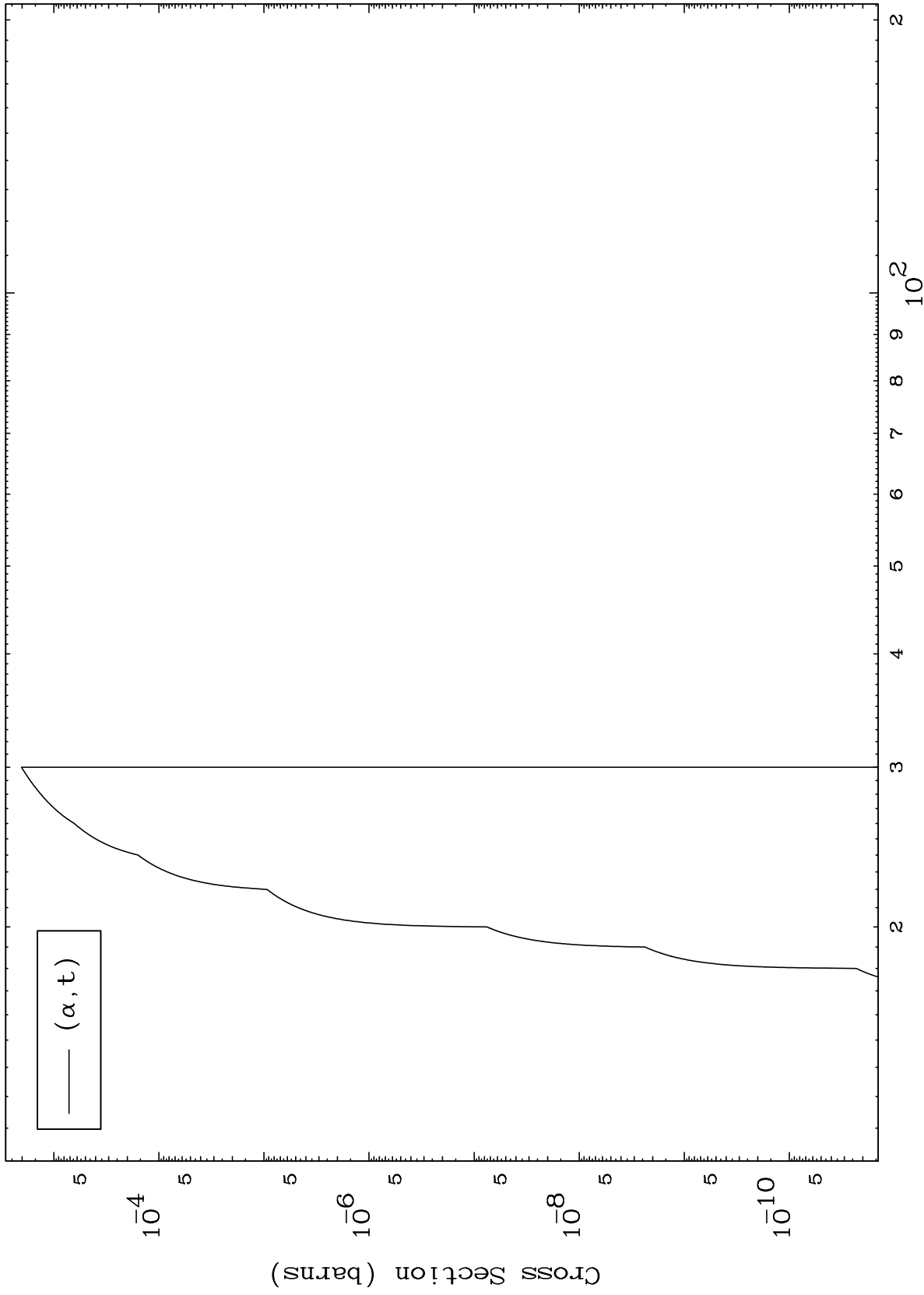
81-T1-210

MAT 8146

(α, d) Levels
0 Kelvin Cross Sections

81-T1-210

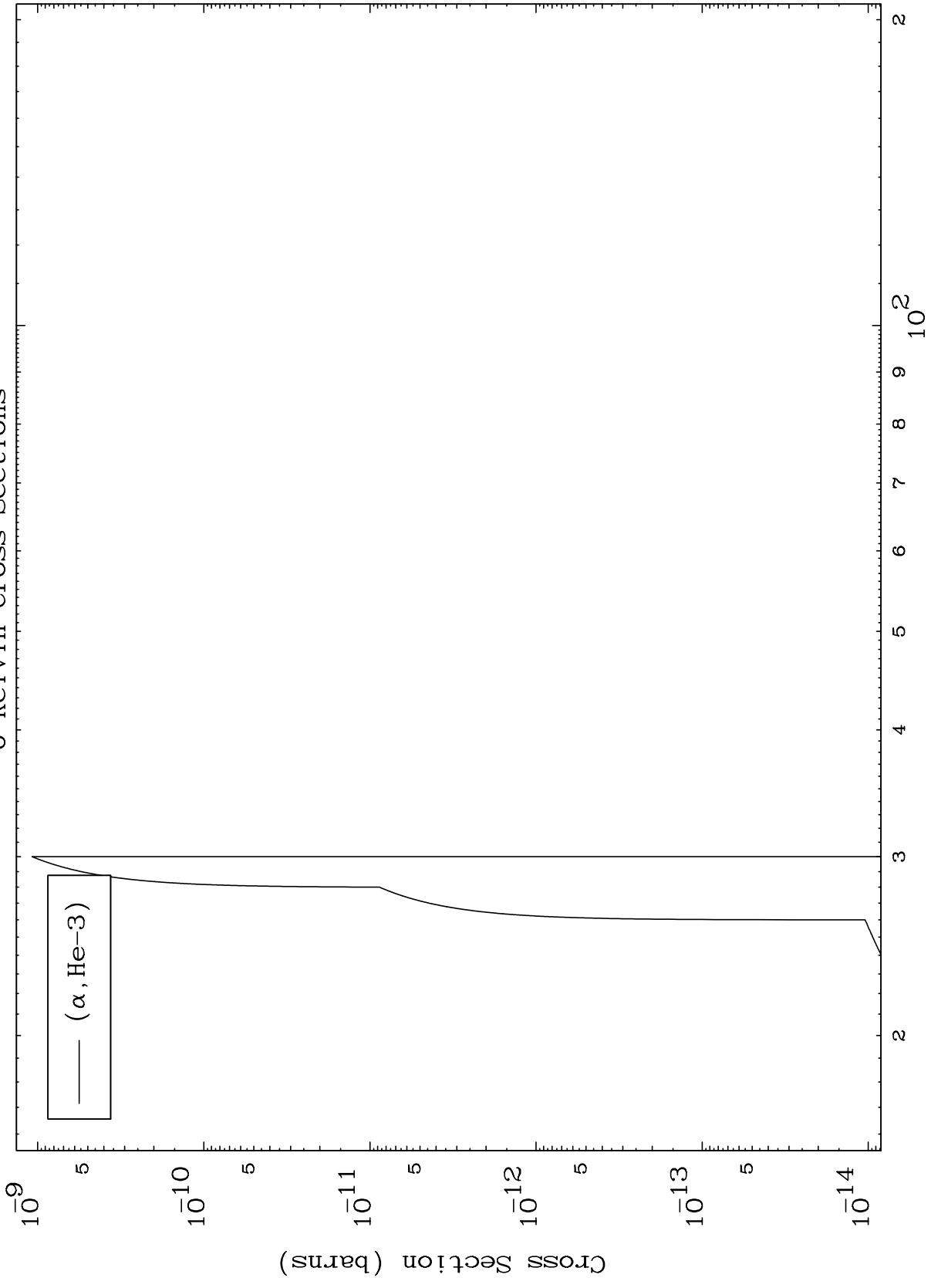




MAT 8146

(α ,He3) Levels
0 Kelvin Cross Sections

81-T1-210



9

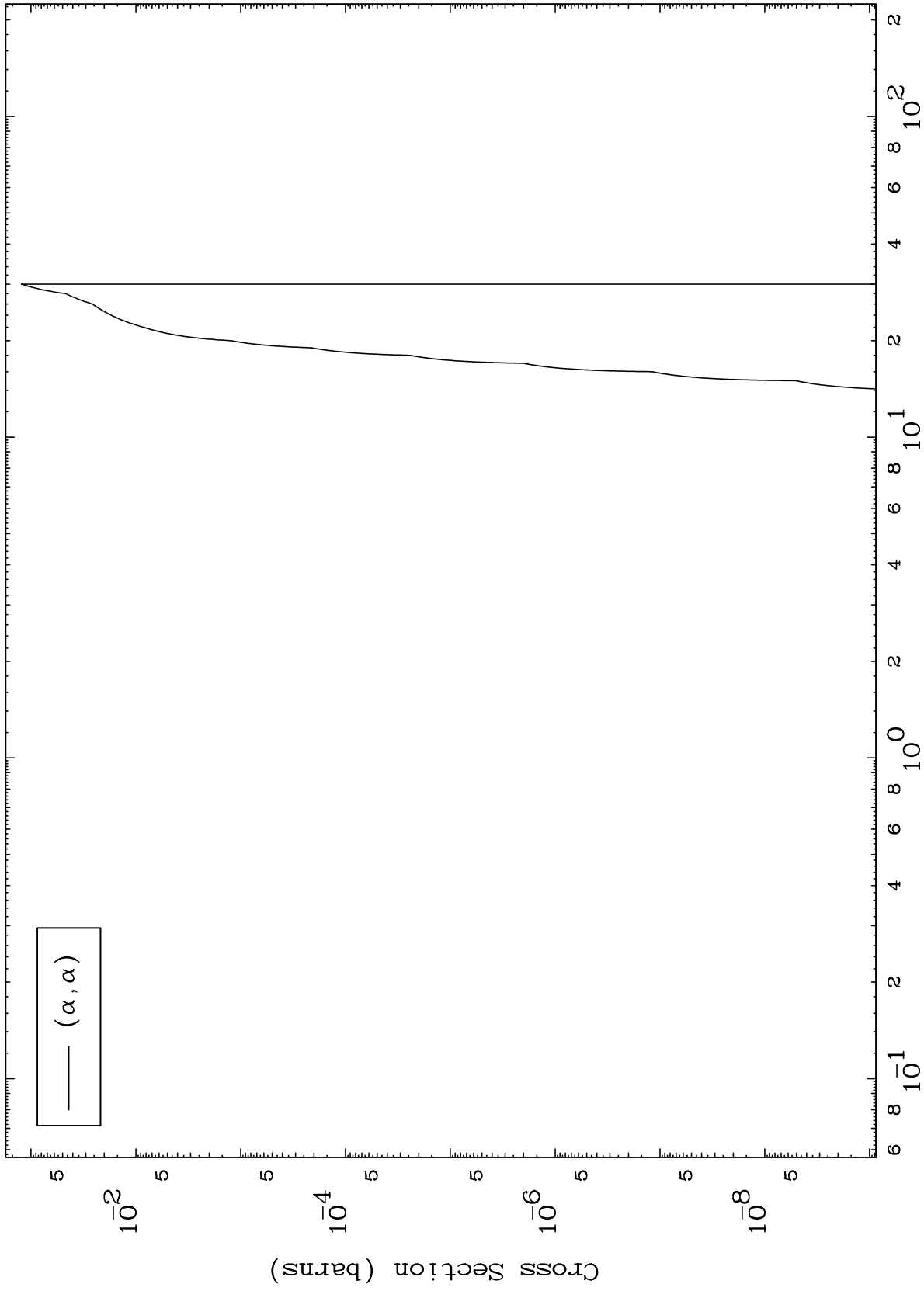
Incident Energy (MeV)

81-T1-210

MAT 8146

(α, α) Levels
0 Kelvin Cross Sections

81-Tl-210



10

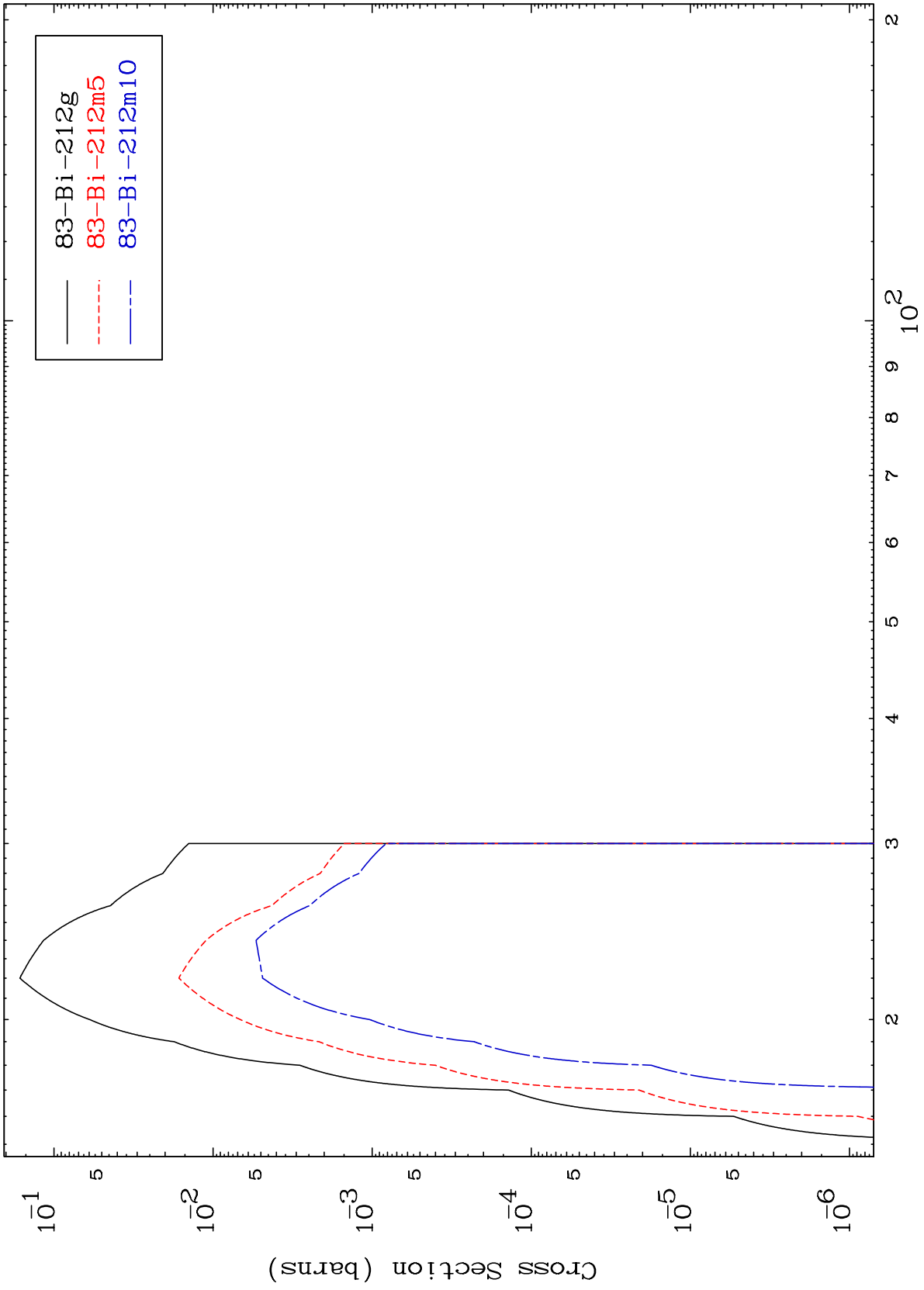
Incident Energy (MeV)

81-Tl-210

MAT 8146

81-Tl-210

Radionuclide Production Cross Section
($\alpha, 2n$)



11

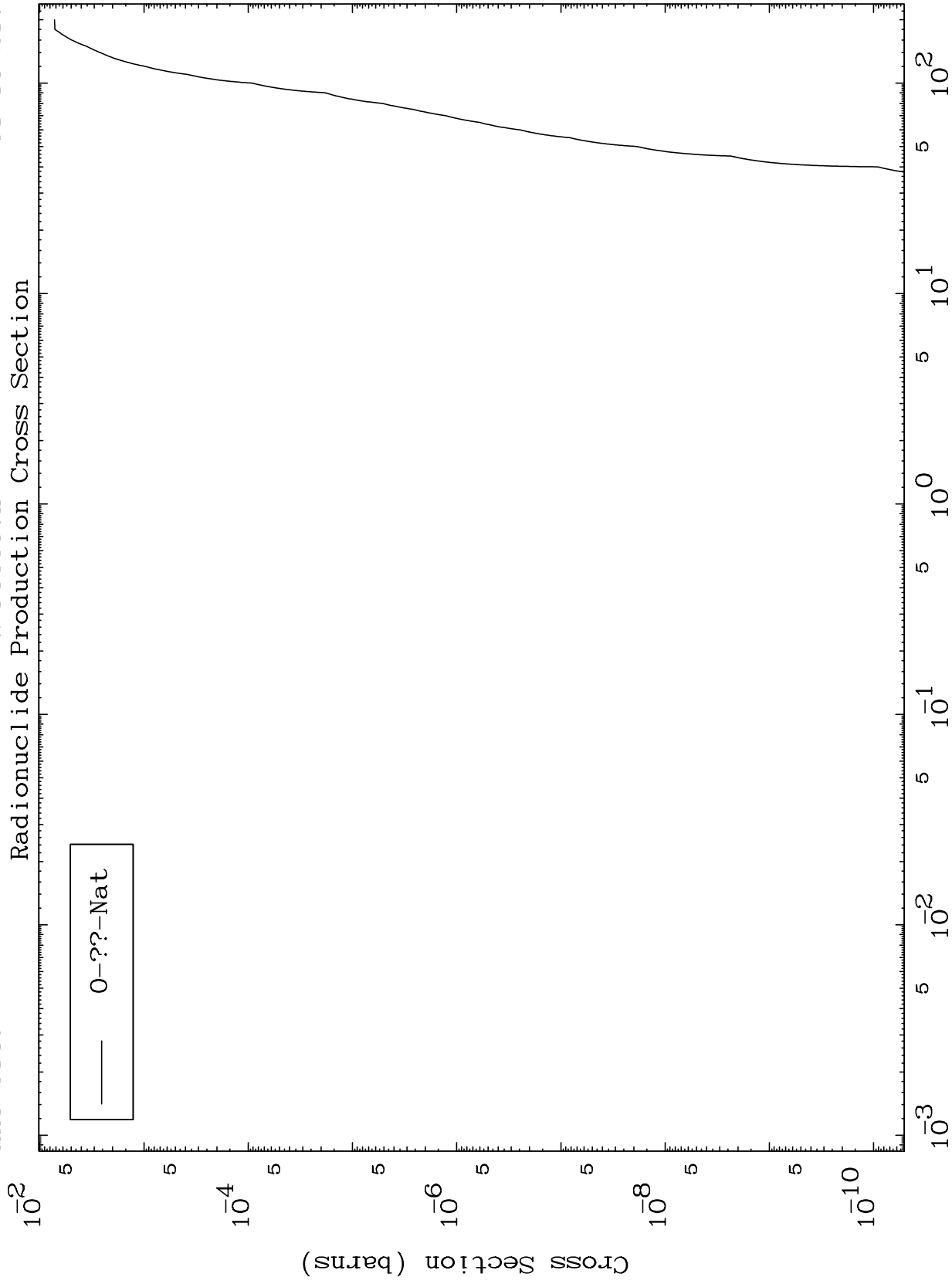
Incident Energy (MeV)

81-Tl-210

MAT 8146

α Fission
Radionuclide Production Cross Section

81-Tl-210



12

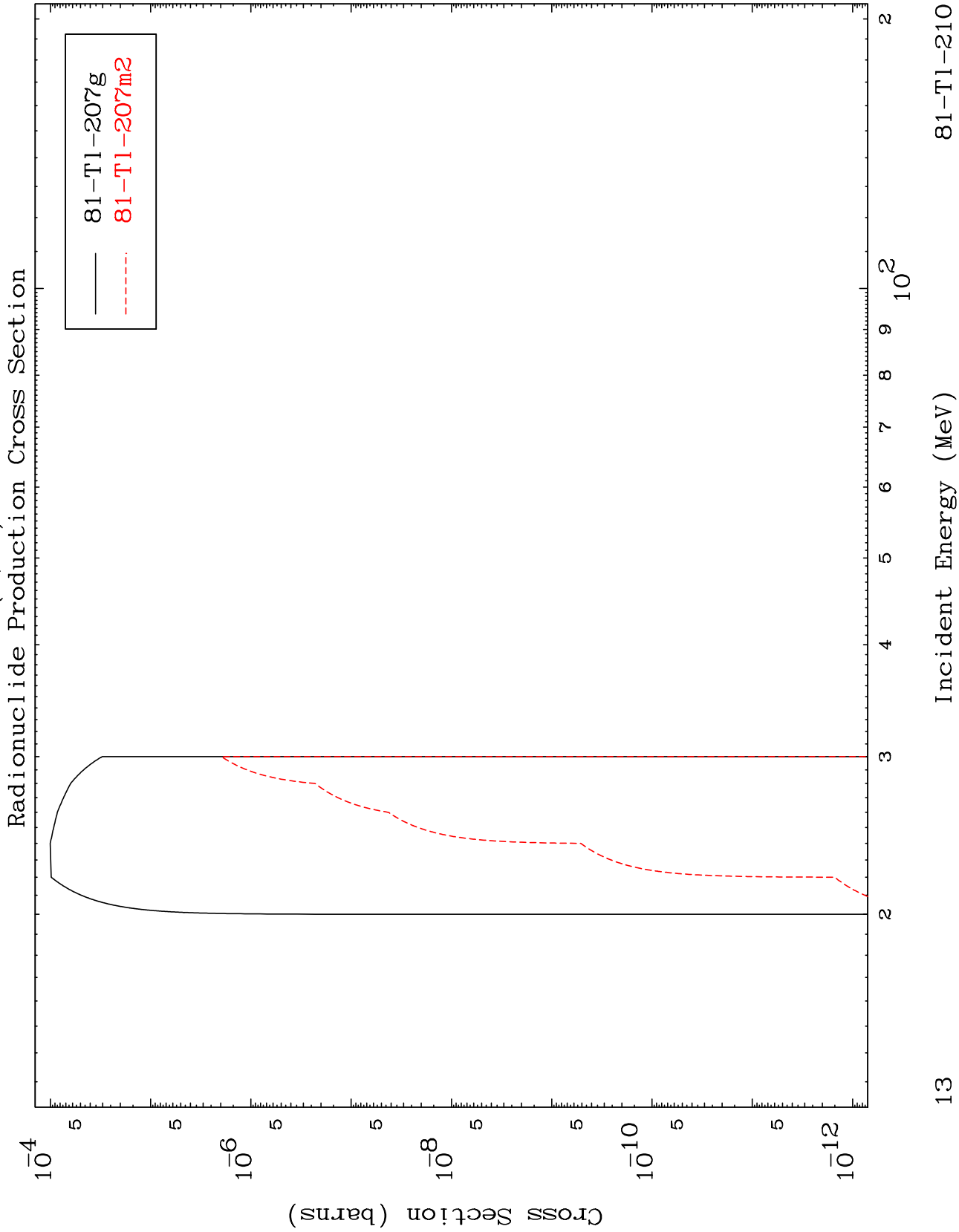
Incident Energy (MeV)

81-Tl-210

MAT 8146

($\alpha, 3n$) α

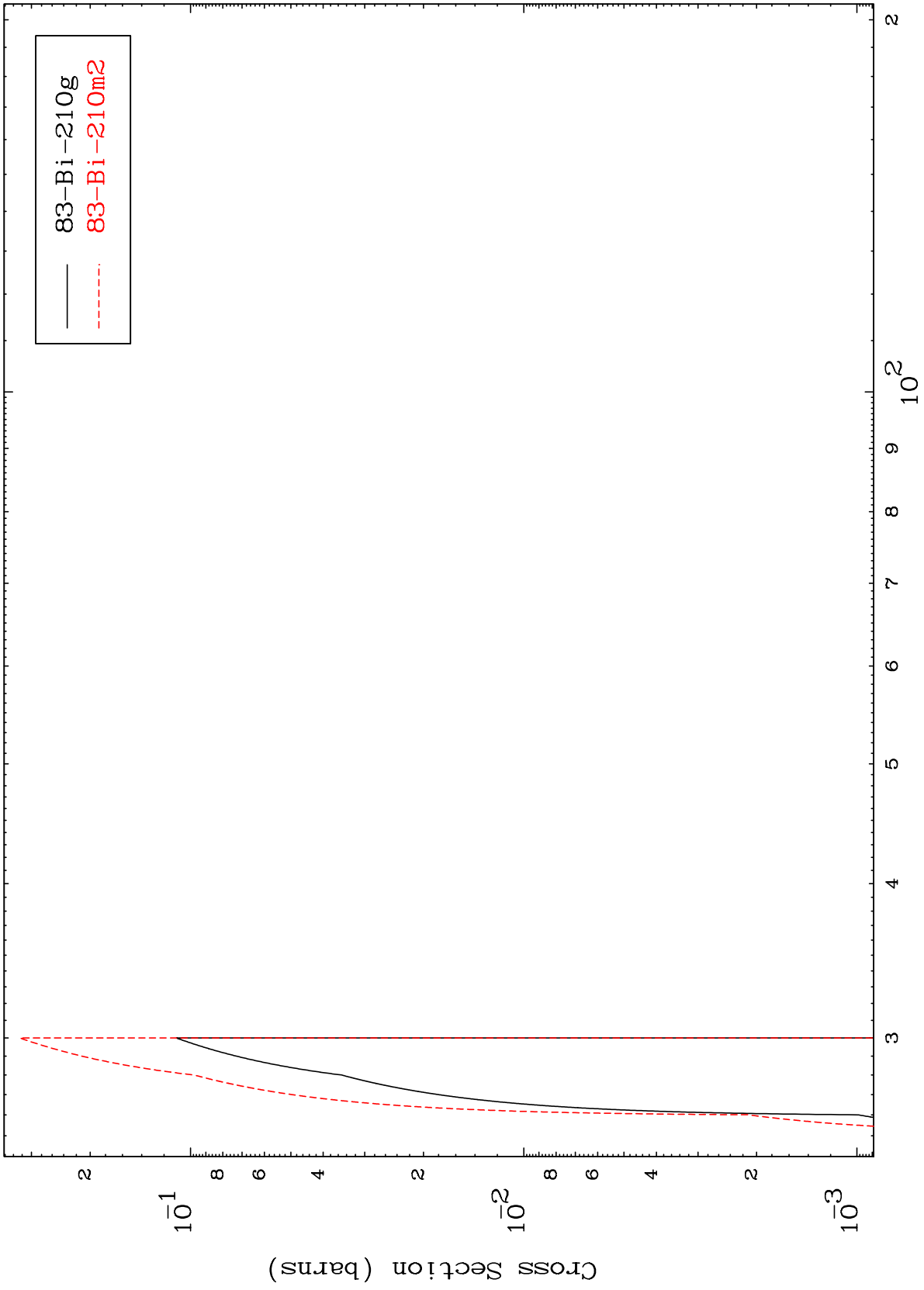
81-Tl-210



MAT 8146

81-Tl-210

($\alpha, 4n$)
Radionuclide Production Cross Section



14

Incident Energy (MeV)

81-Tl-210