

Program EVALPLOT  
(Version 2018-1)

by

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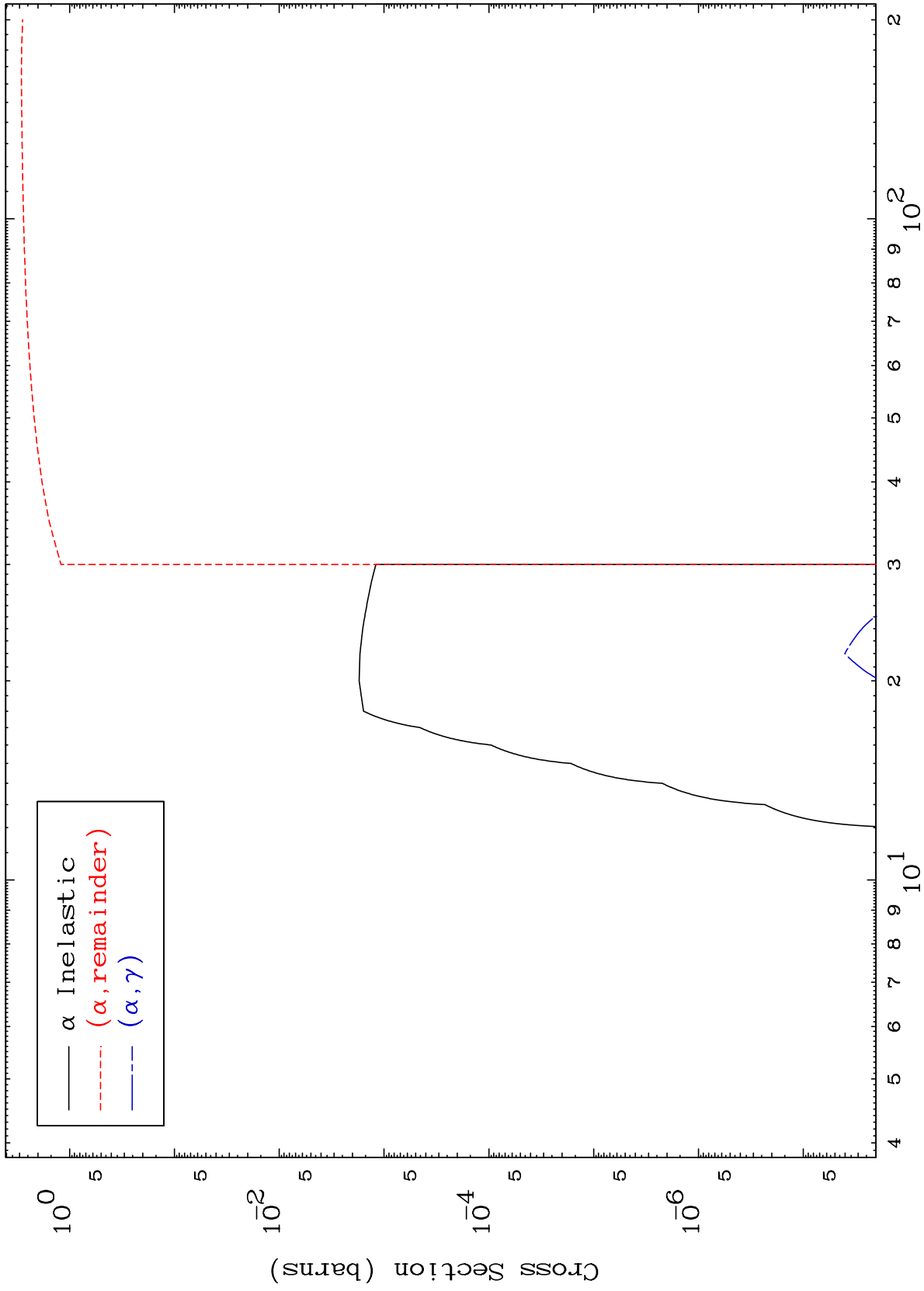
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8161

0 Kelvin  $\alpha$  Major  
Cross Sections

81-Tl-215



1

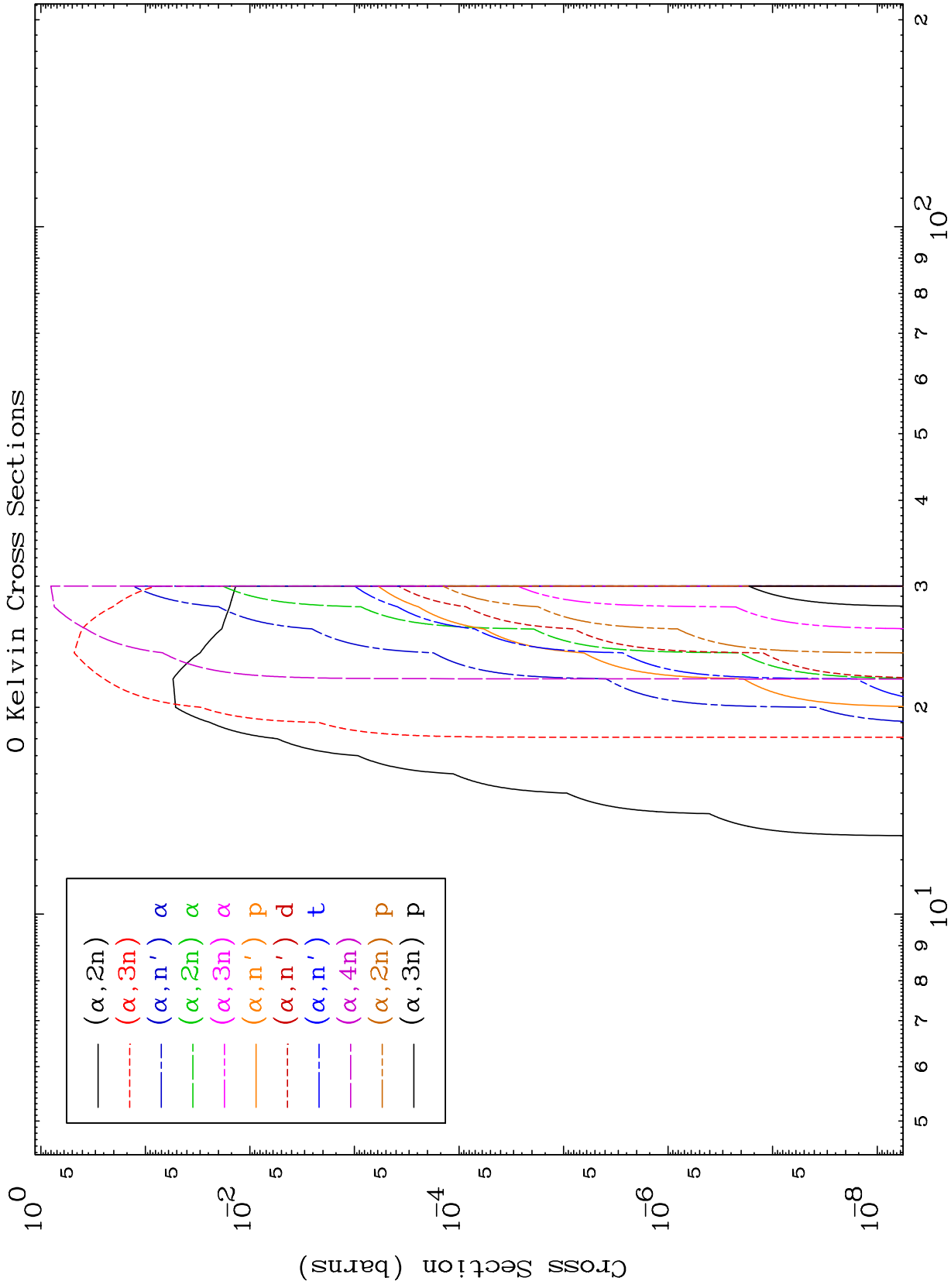
Incident Energy (MeV)

81-Tl-215

MAT 8161

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

81-T1-215

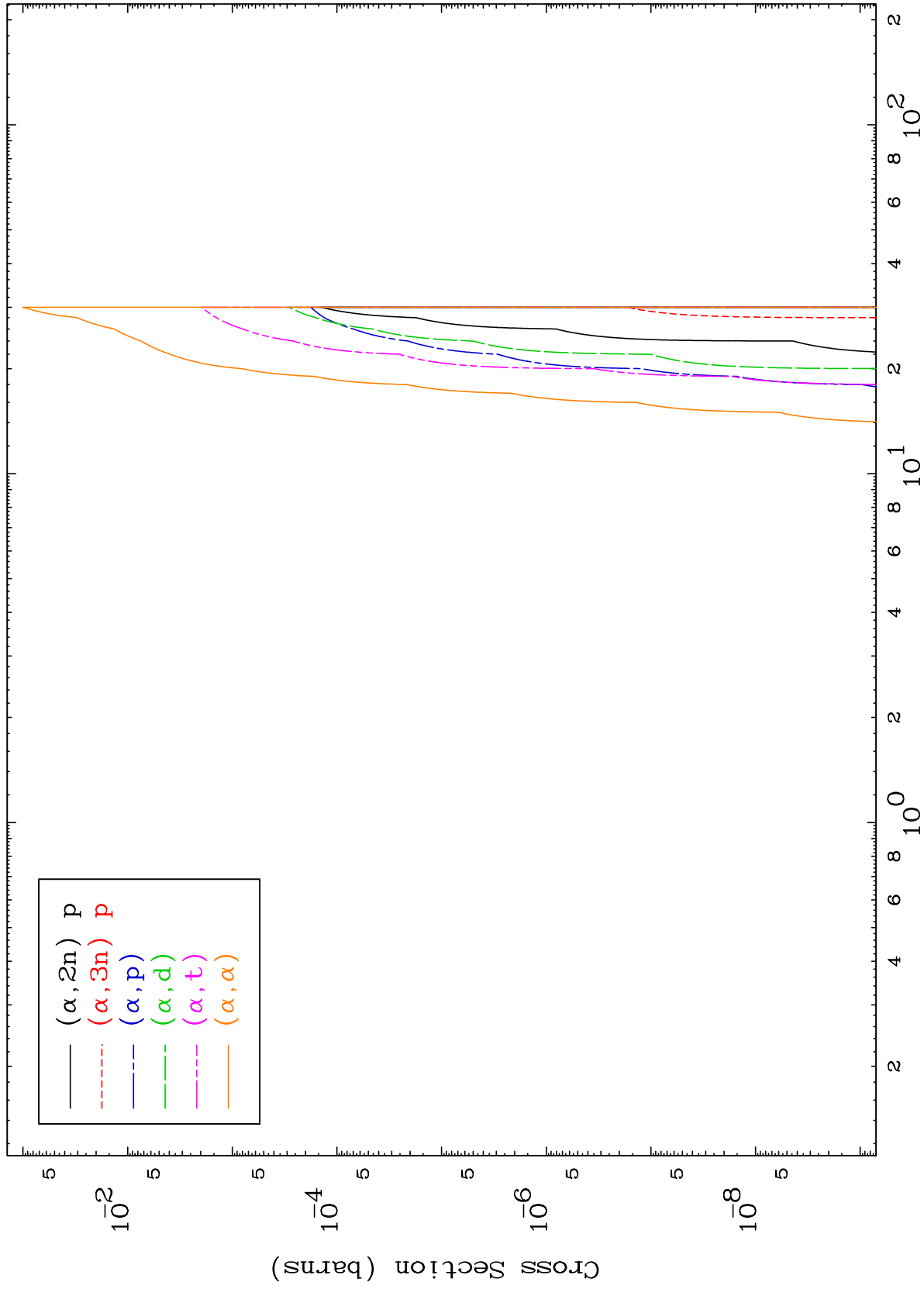


2

Incident Energy (MeV)

81-T1-215

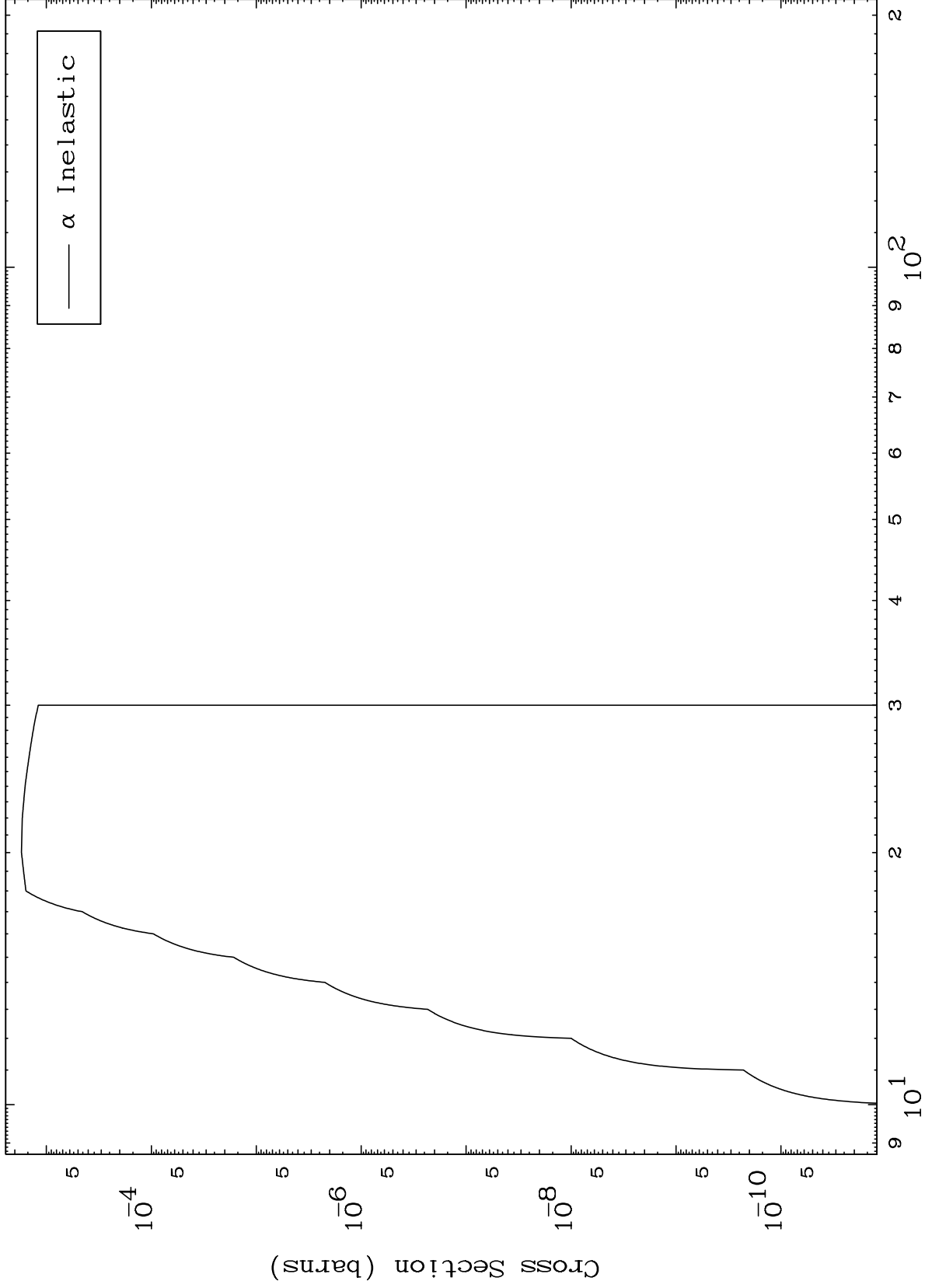




MAT 8161

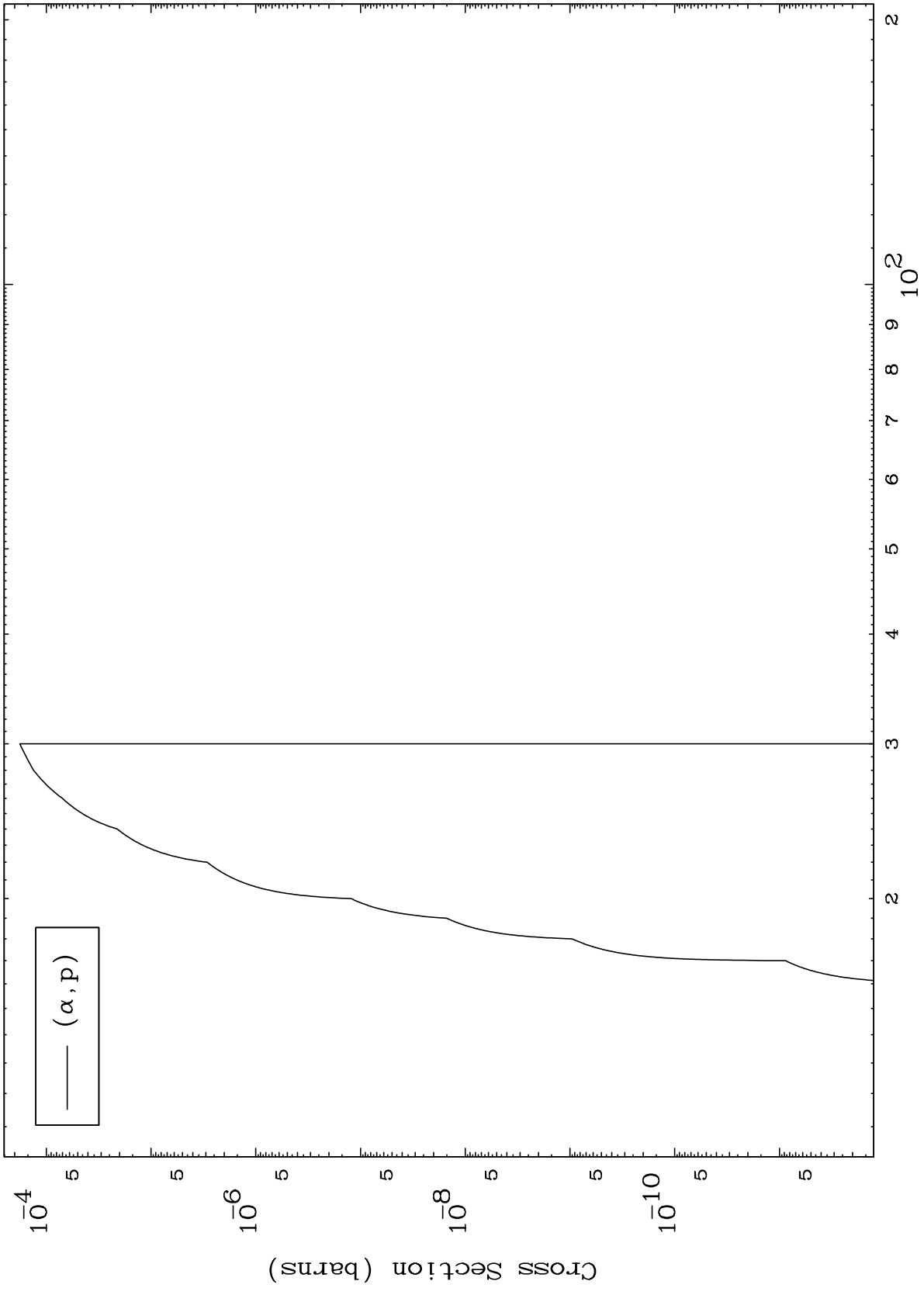
( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

81-T1-215



Incident Energy (MeV)

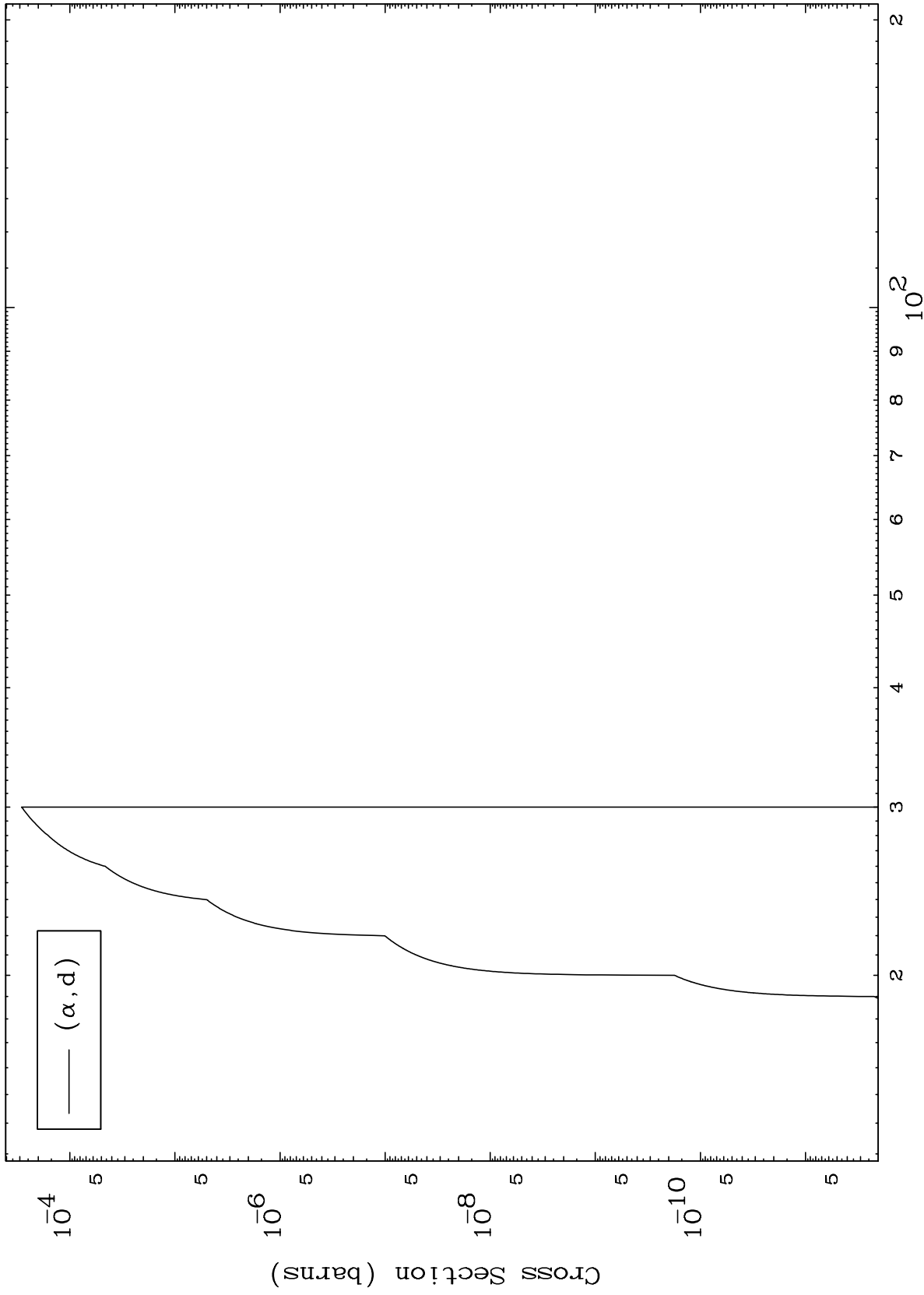
81-T1-215



MAT 8161

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

81-T1-215



7

Incident Energy (MeV)

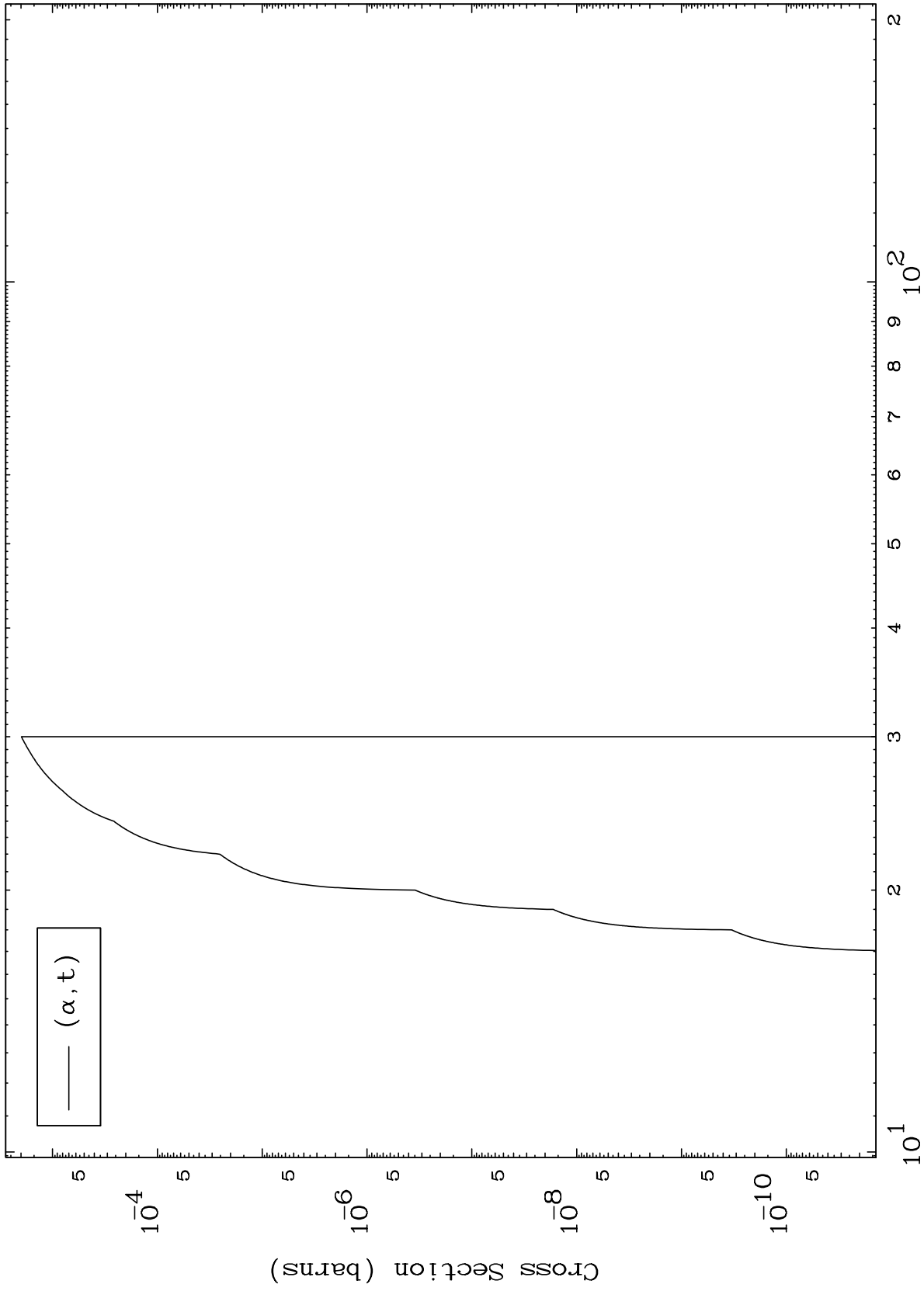
81-T1-215



MAT 8161

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

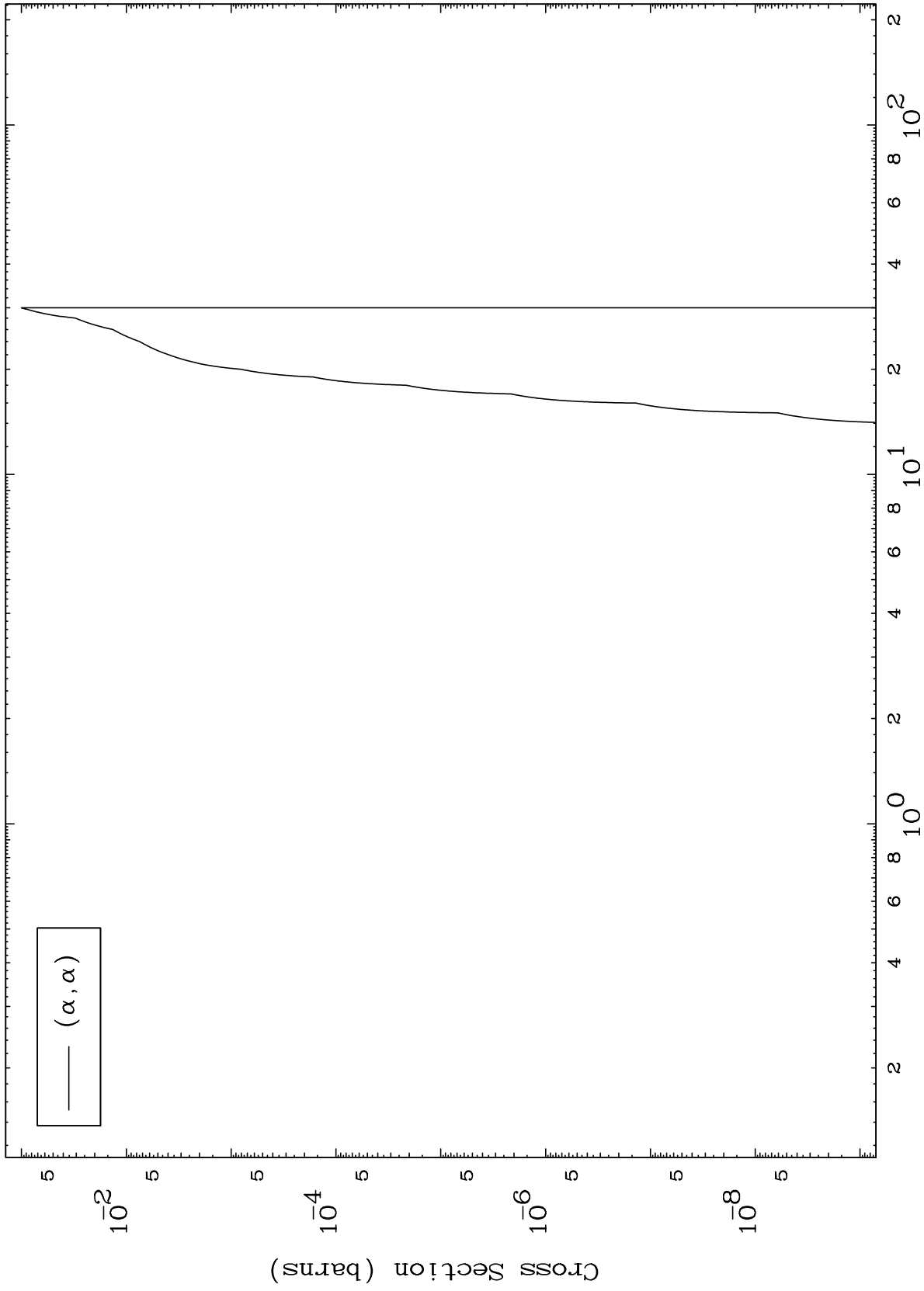
81-T1-215



MAT 8161

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

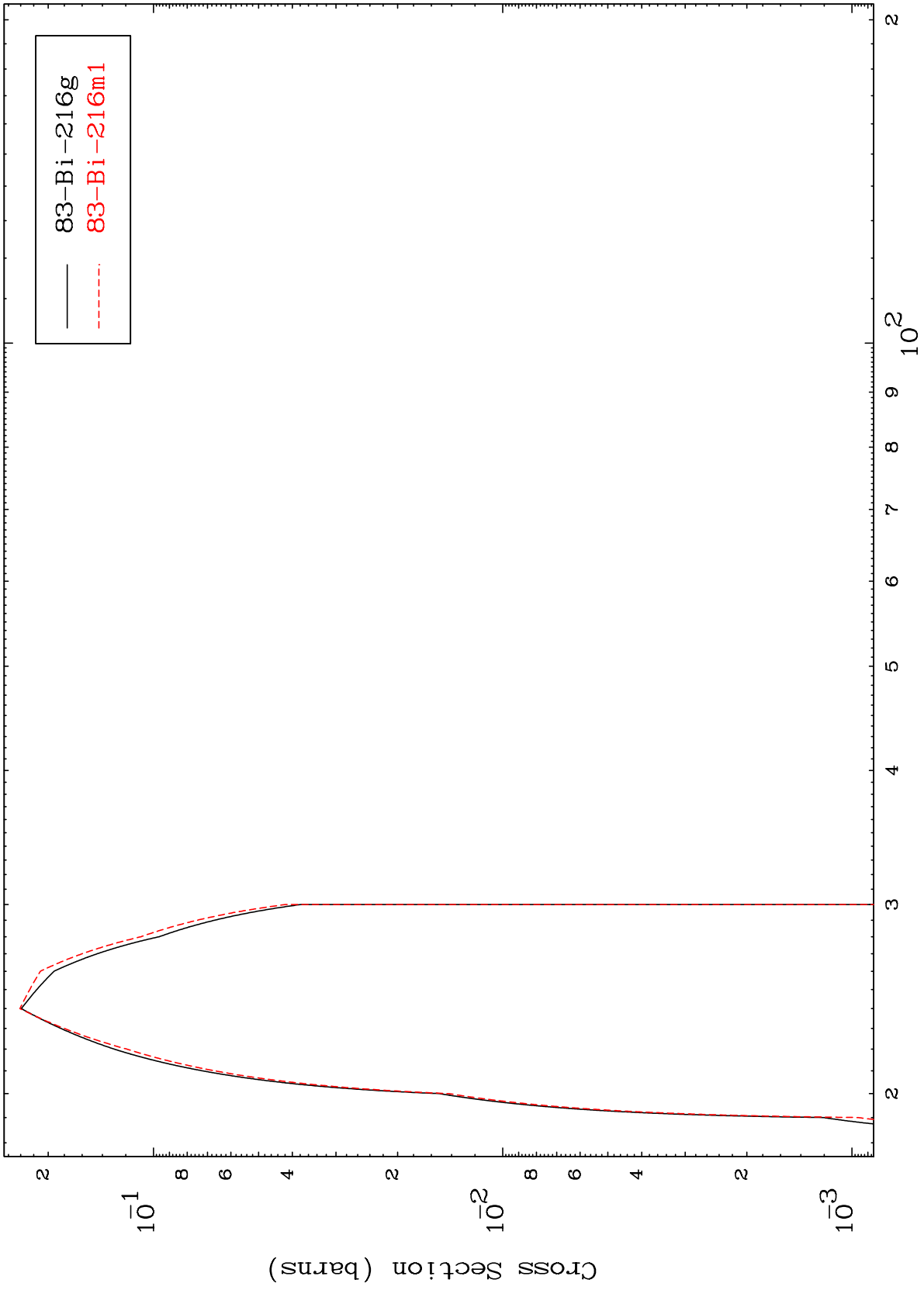
81-Tl-215



MAT 8161

81-T1-215

( $\alpha, 3n$ )  
Radionuclide Production Cross Section



10

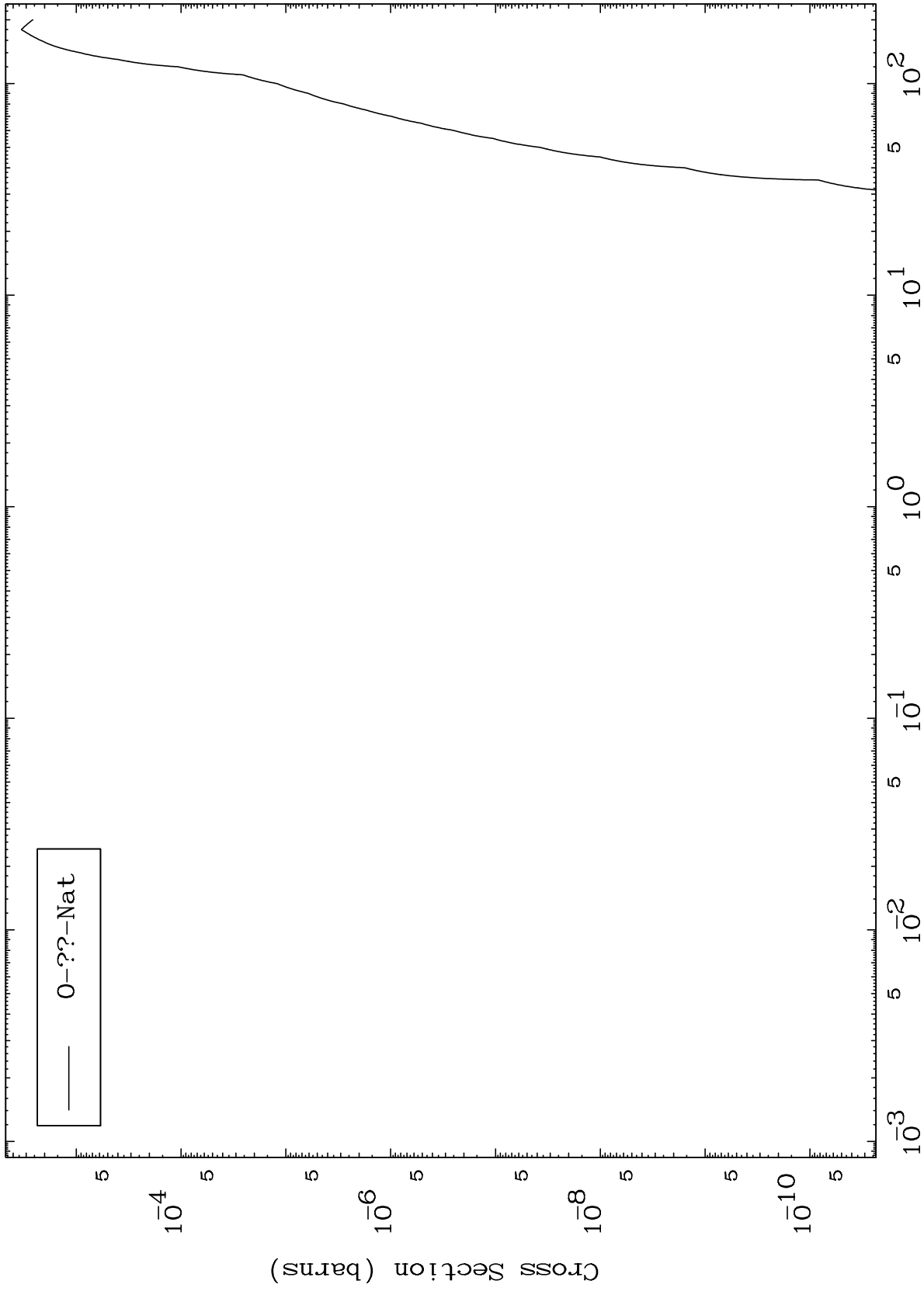
Incident Energy (MeV)

81-T1-215

MAT 8161

$\alpha$  Fission  
Radionuclide Production Cross Section

81-Tl-215



11

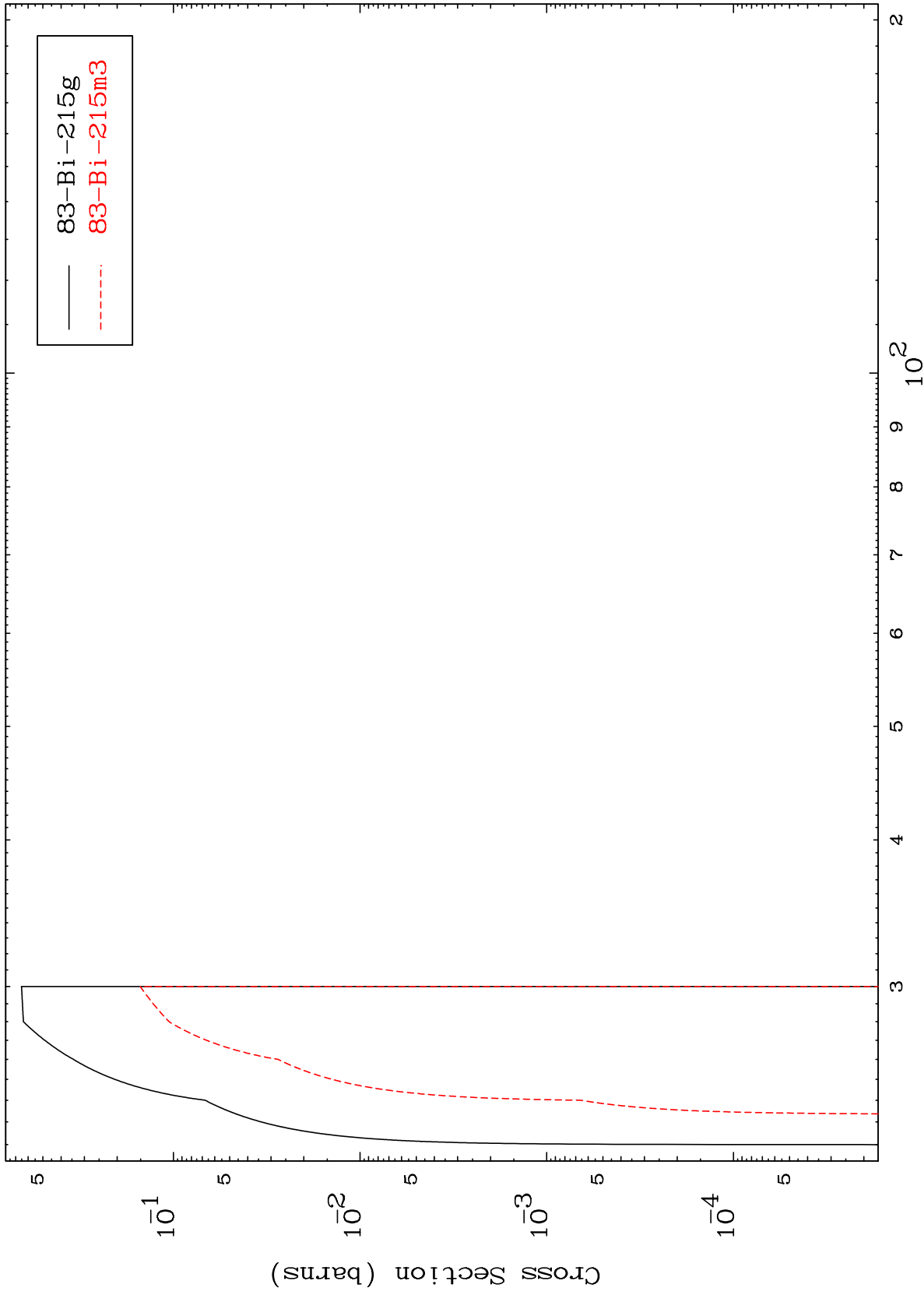
Incident Energy (MeV)

81-Tl-215

MAT 8161

81-T1-215

$(\alpha, 4n)$   
Radionuclide Production Cross Section



12

Incident Energy (MeV)

81-T1-215