

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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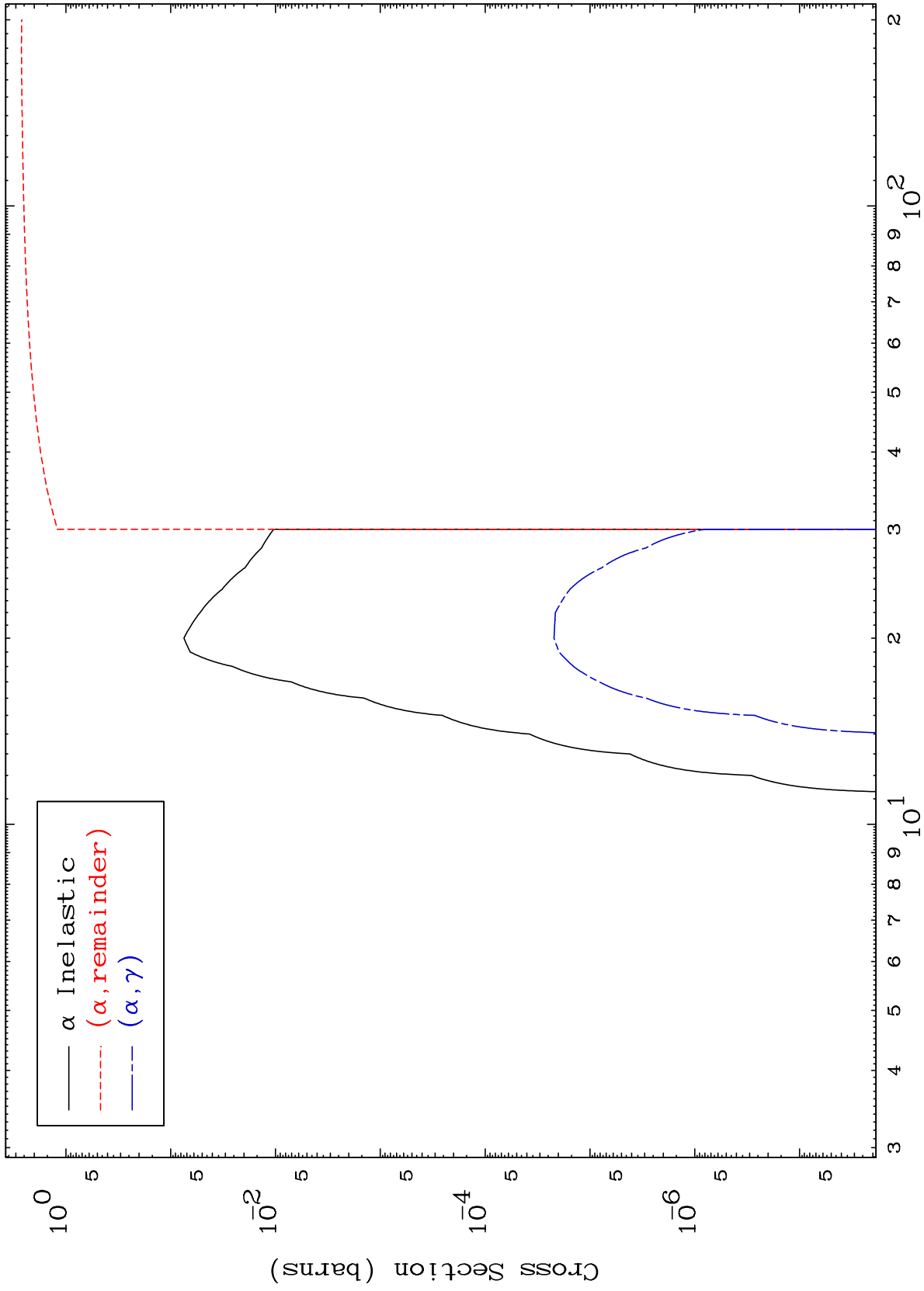
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7428

0 Kelvin  $\alpha$  Major  
Cross Sections

74-W -181



1

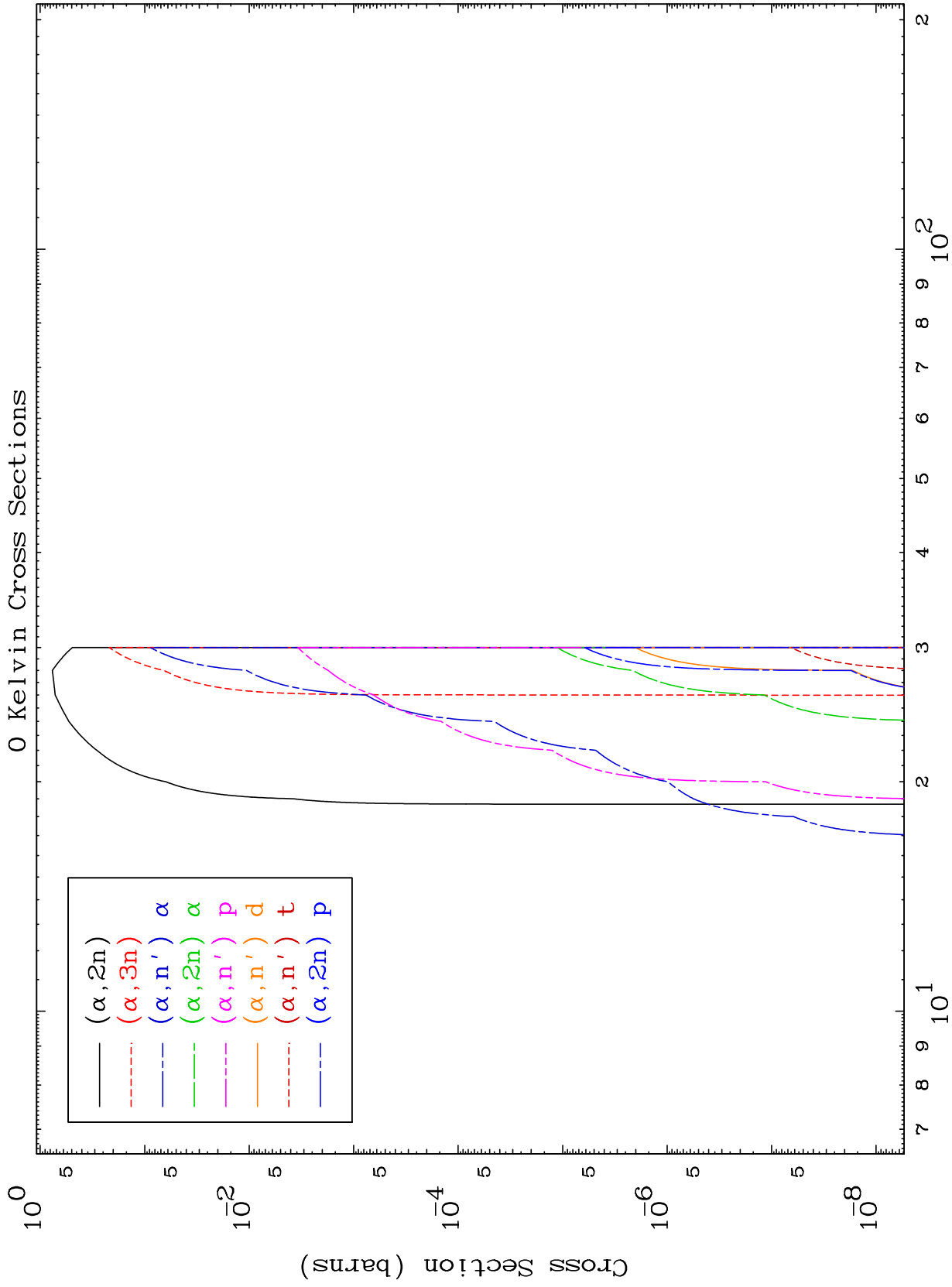
Incident Energy (MeV)

74-W -181

MAT 7428

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

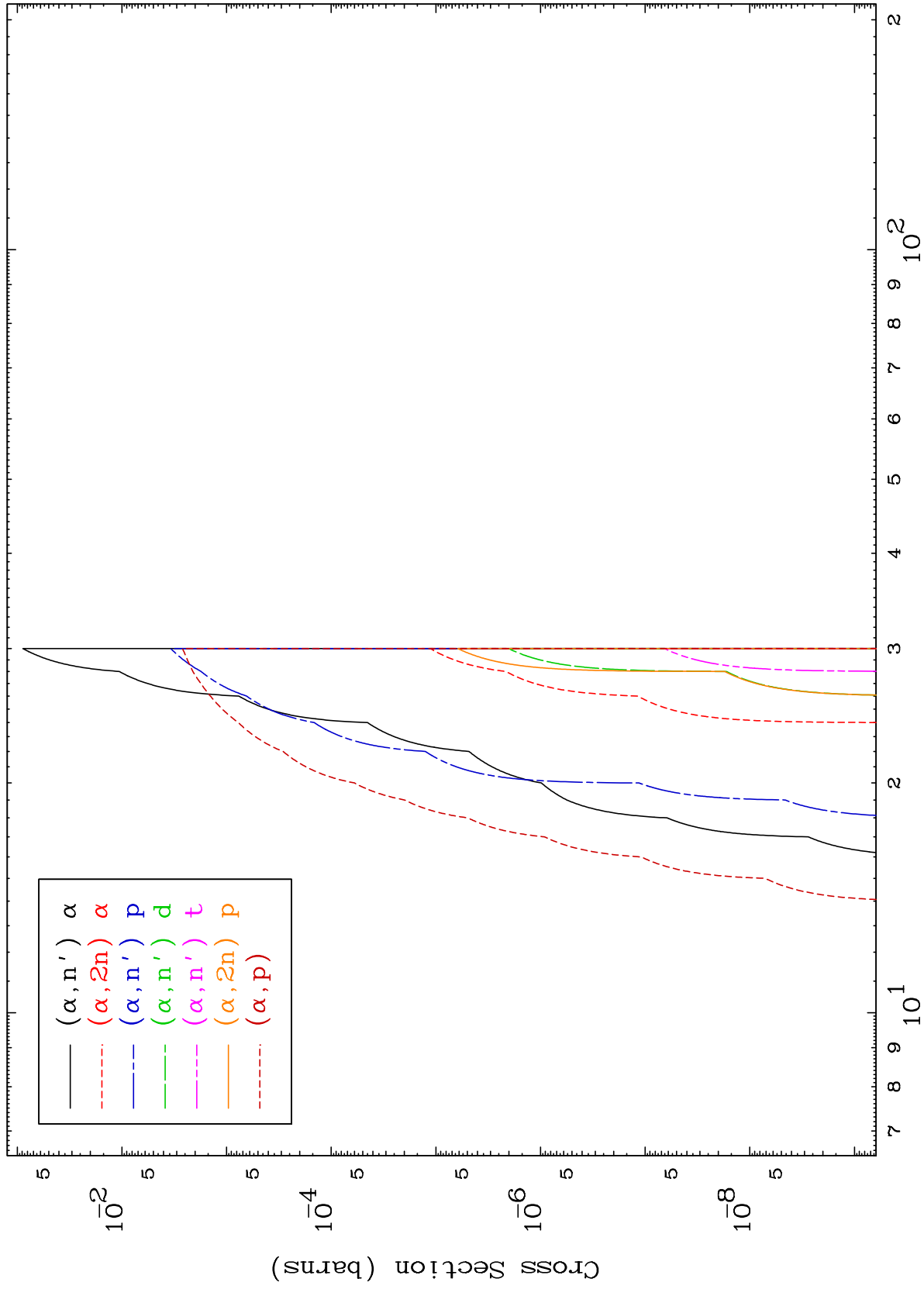
74-W -181

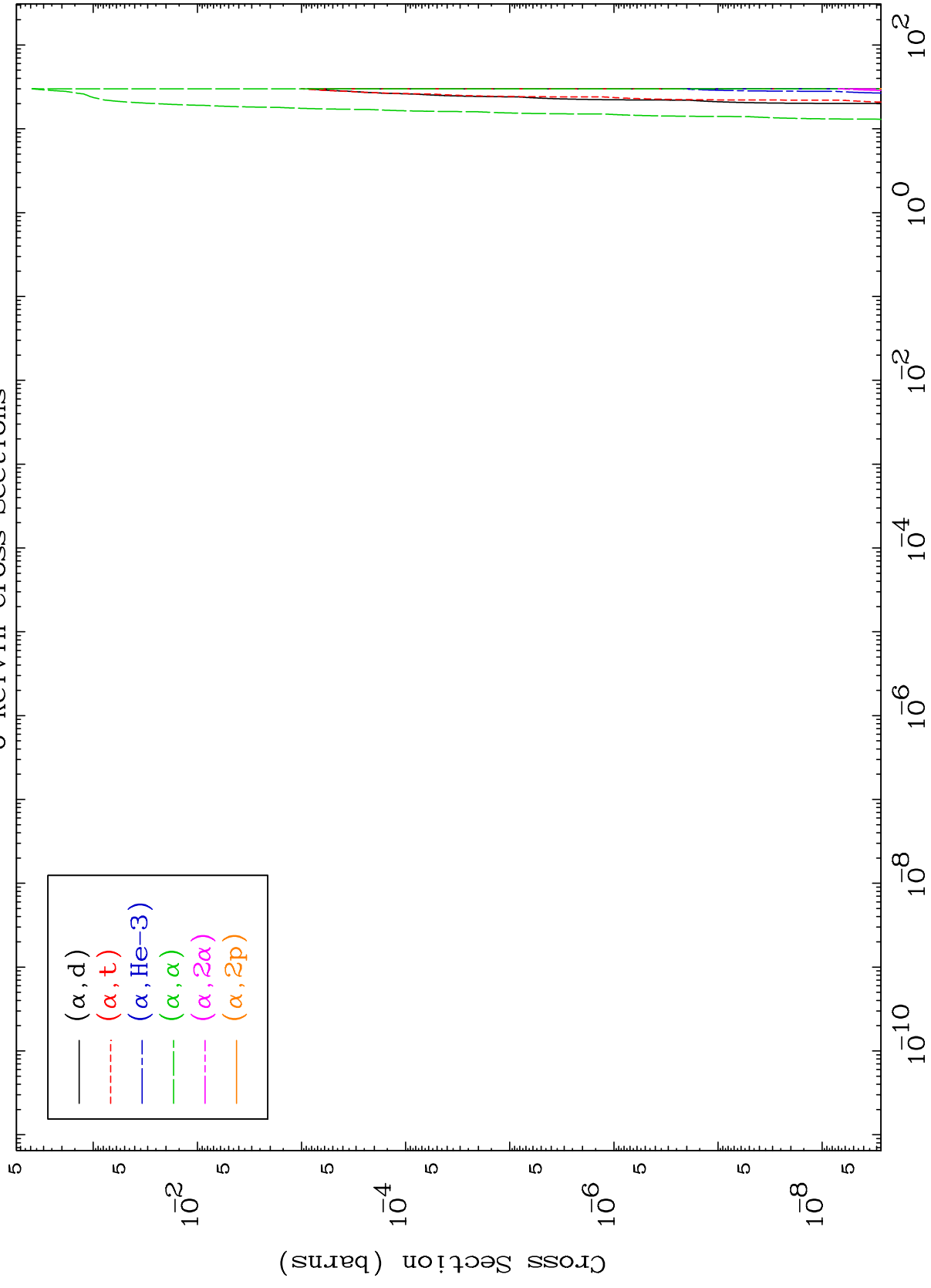


2

Incident Energy (MeV)

74-W -181

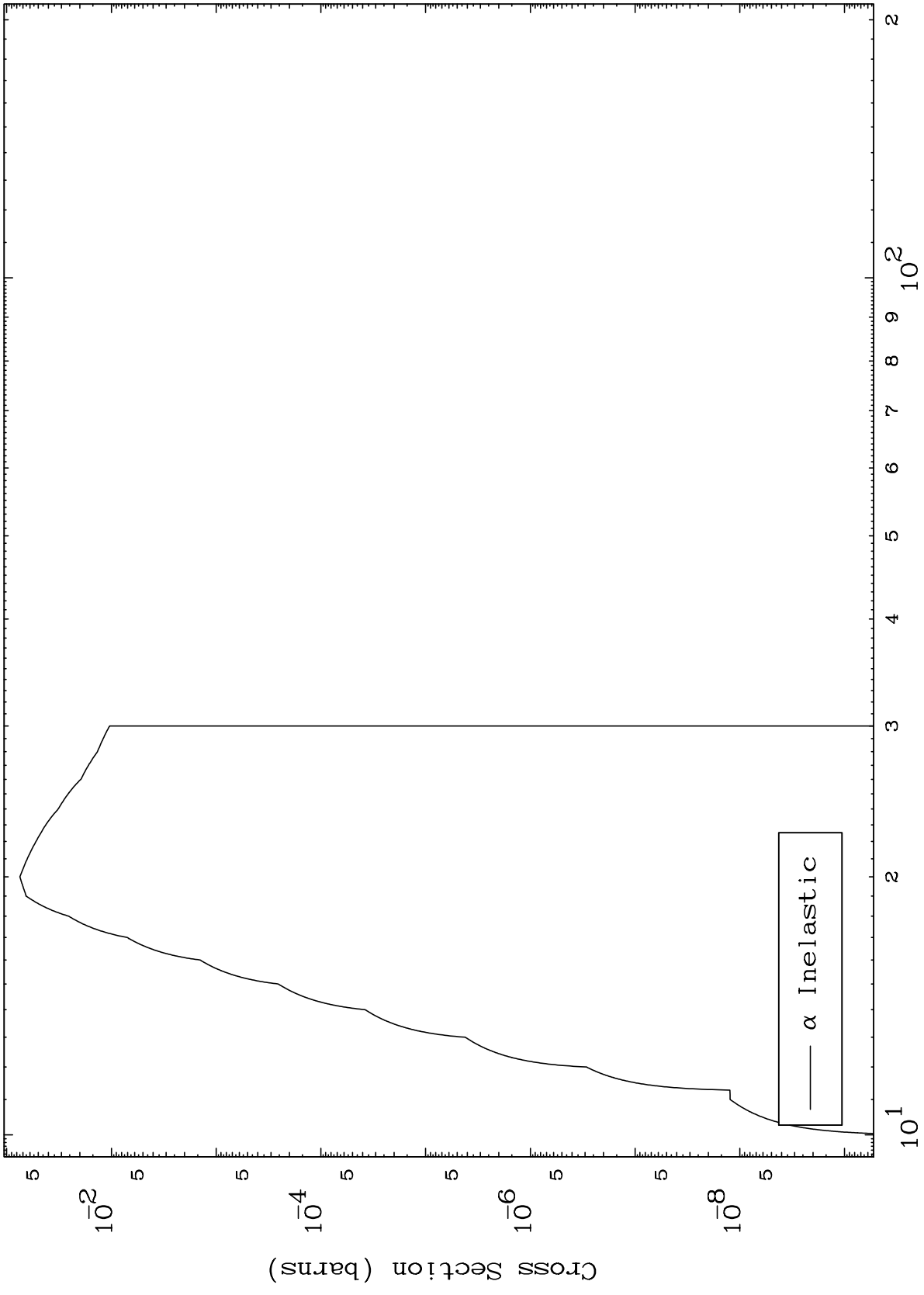




MAT 7428

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

74-W -181



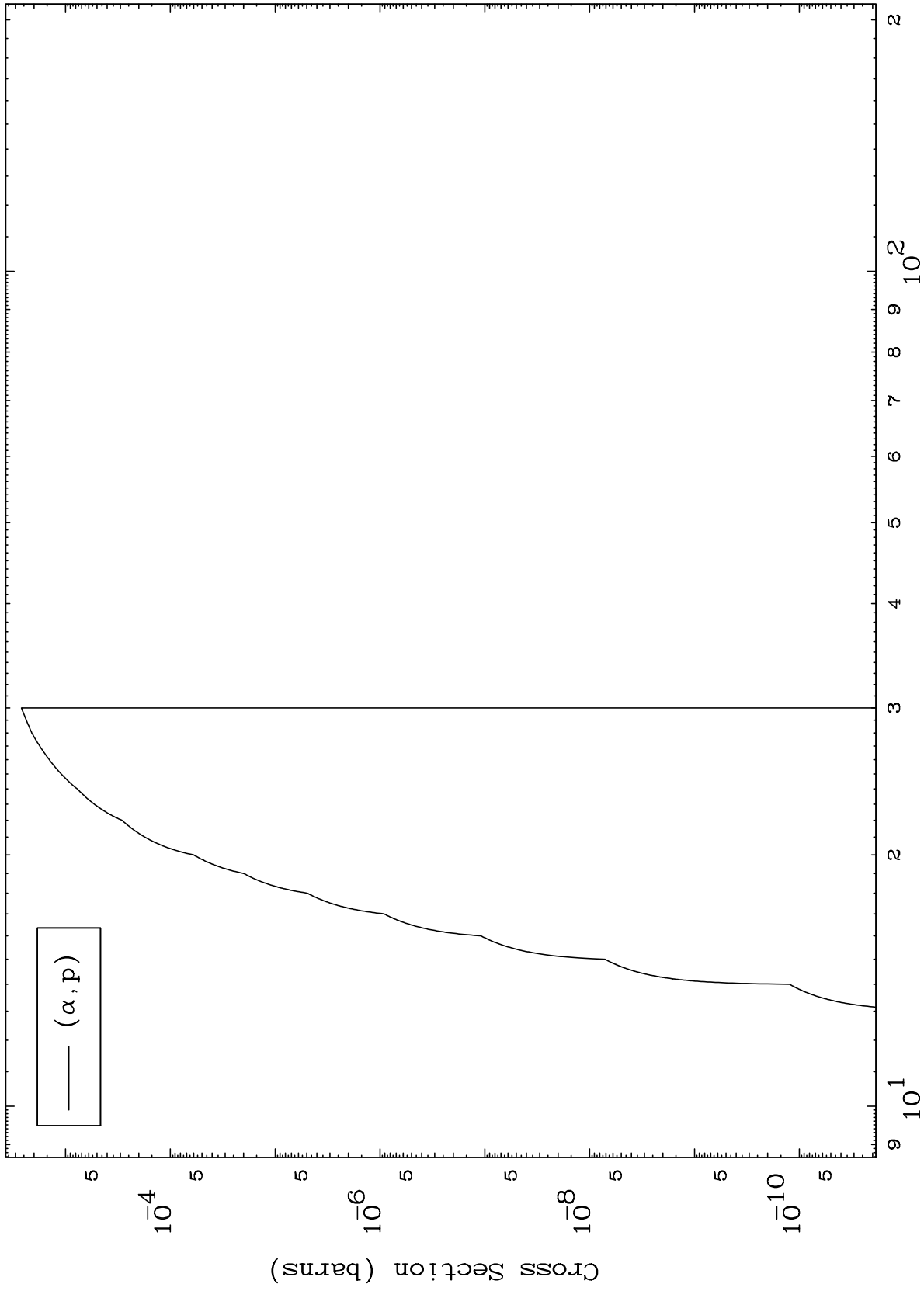
Incident Energy (MeV)

74-W -181

MAT 7428

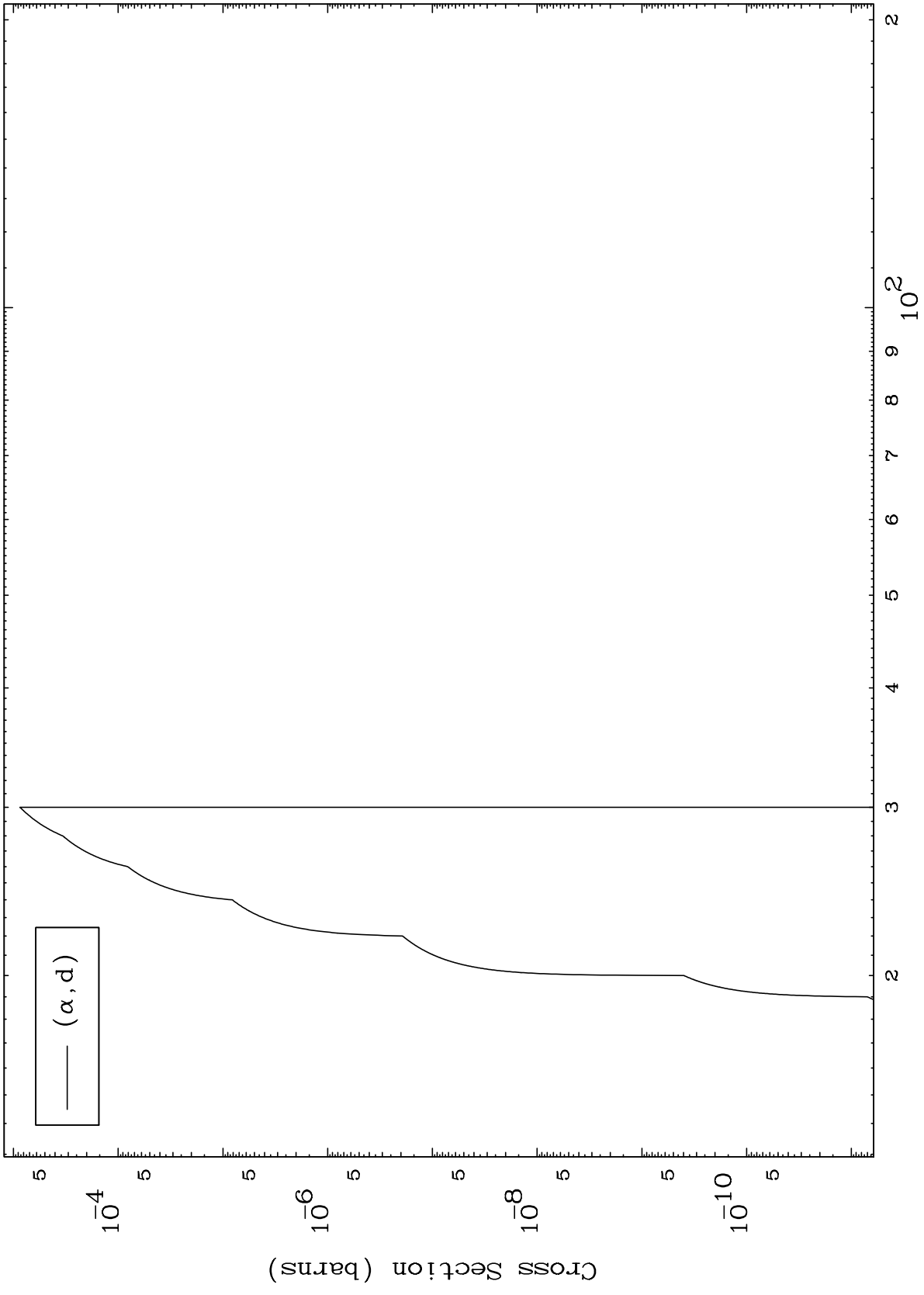
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

74-W -181

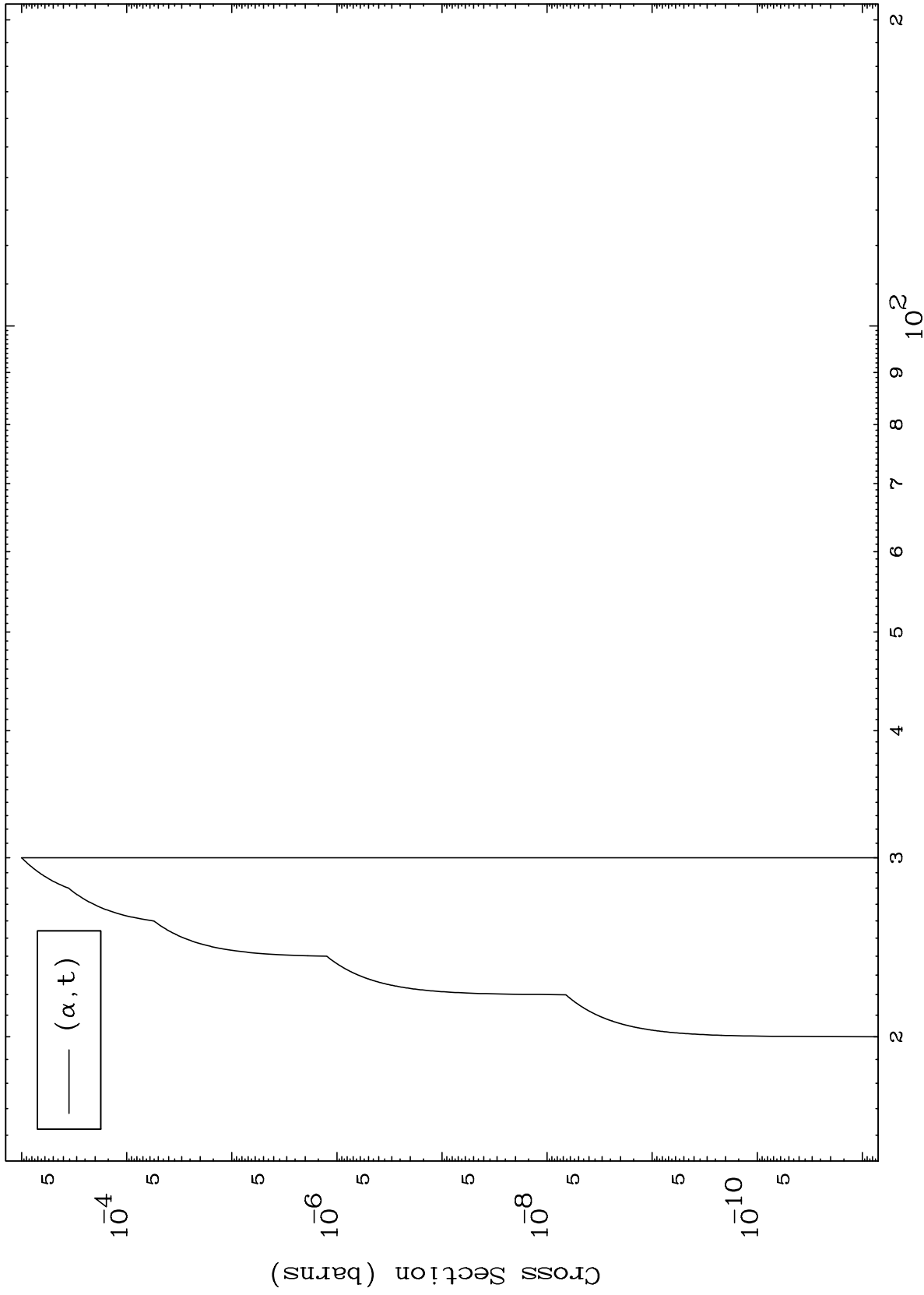


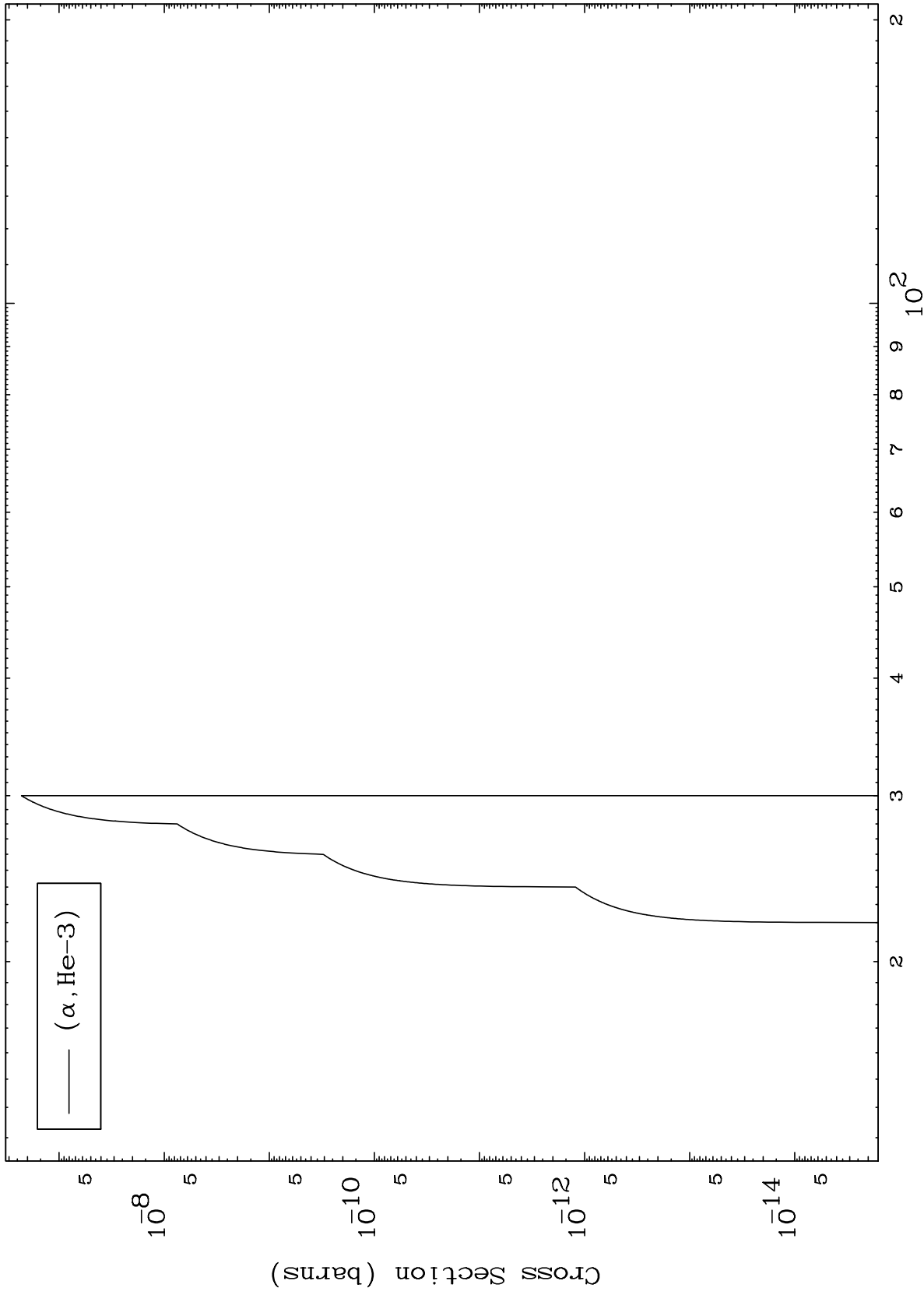
Incident Energy (MeV)

74-W -181





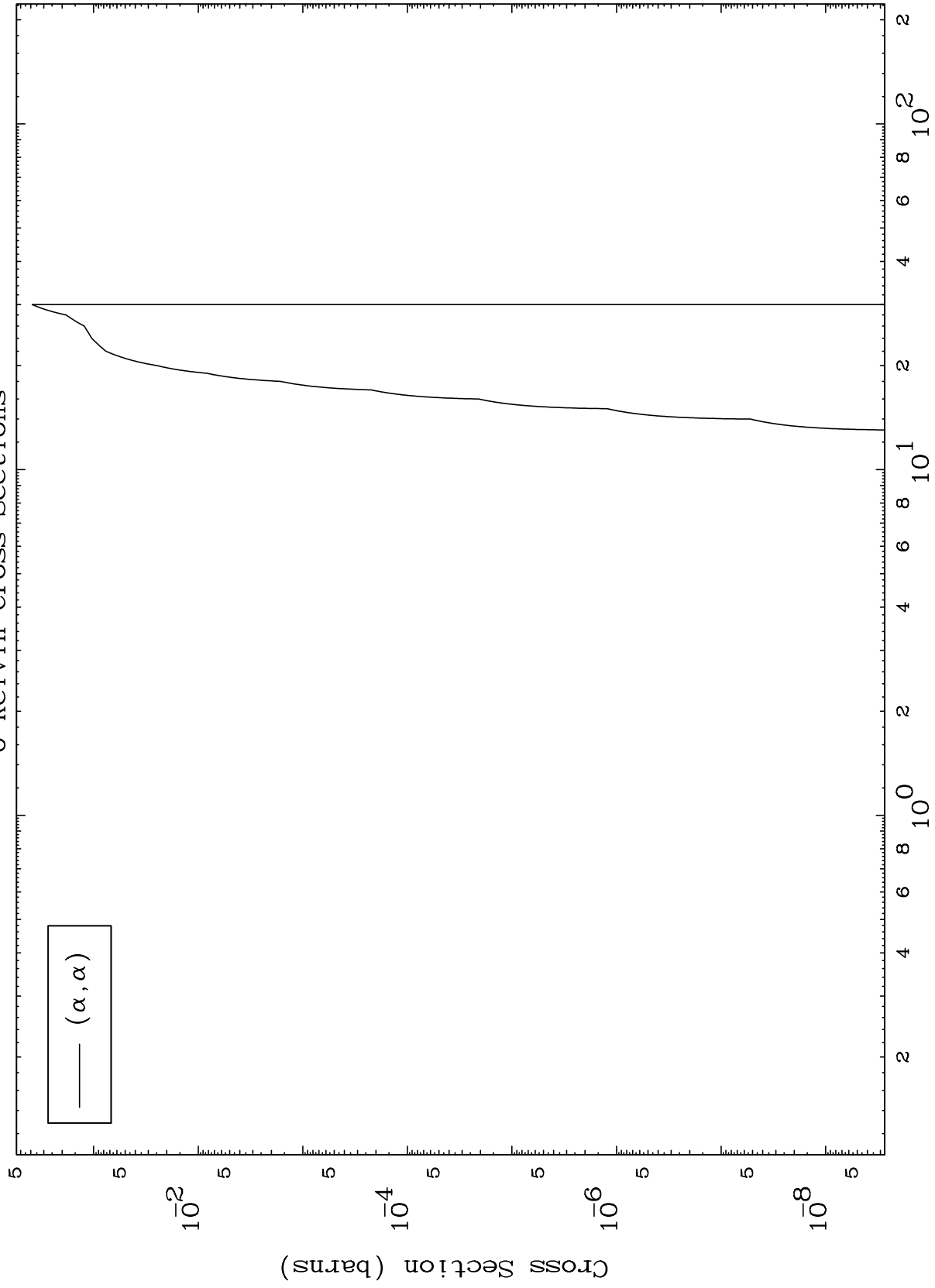




MAT 7428

$(\alpha, \alpha)$  Levels  
0 Kelvin Cross Sections

74-W -181



10

Incident Energy (MeV)

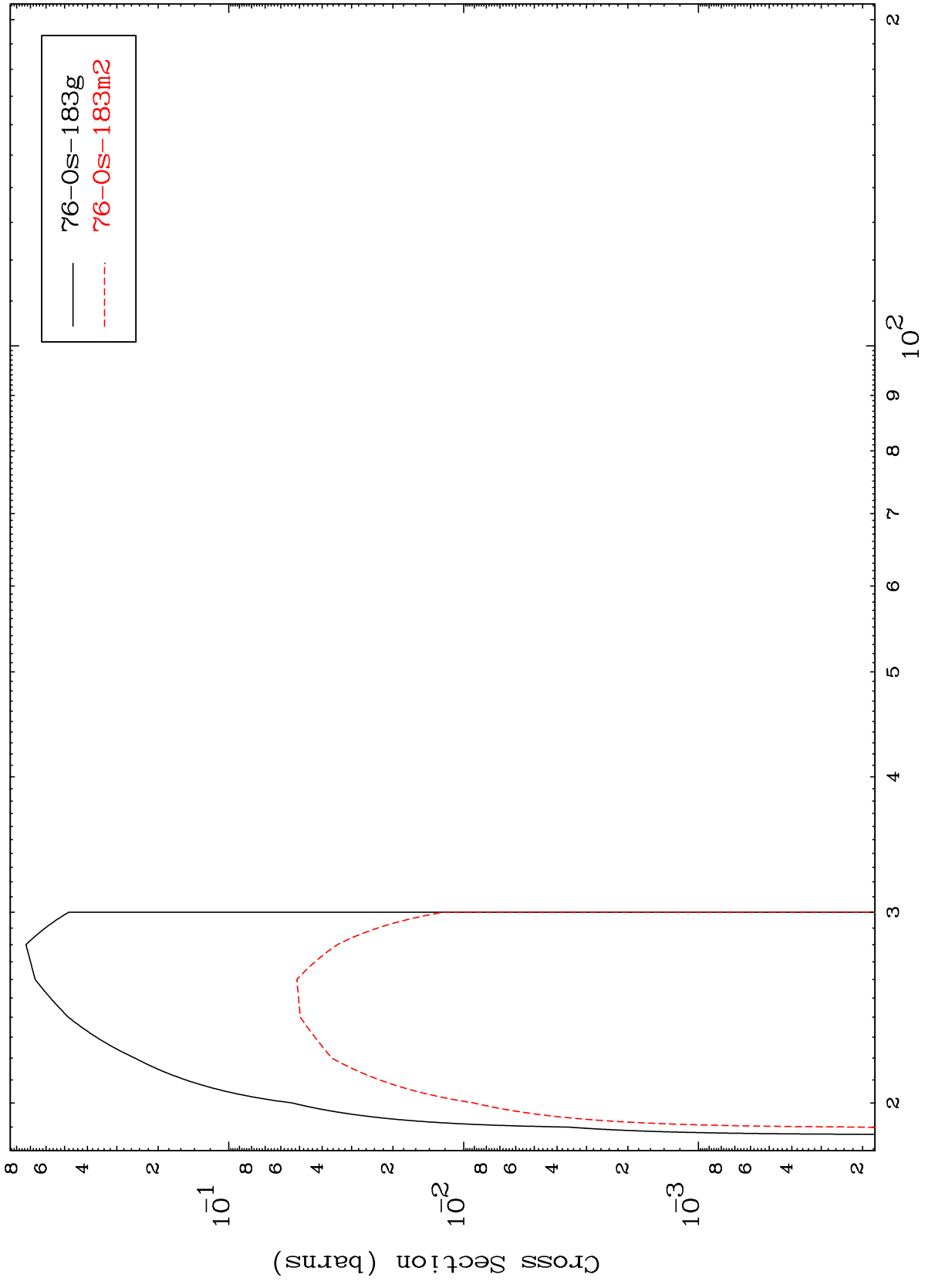
74-W -181

MAT 7428

( $\alpha, 2n$ )

74-W -181

Radionuclide Production Cross Section



11

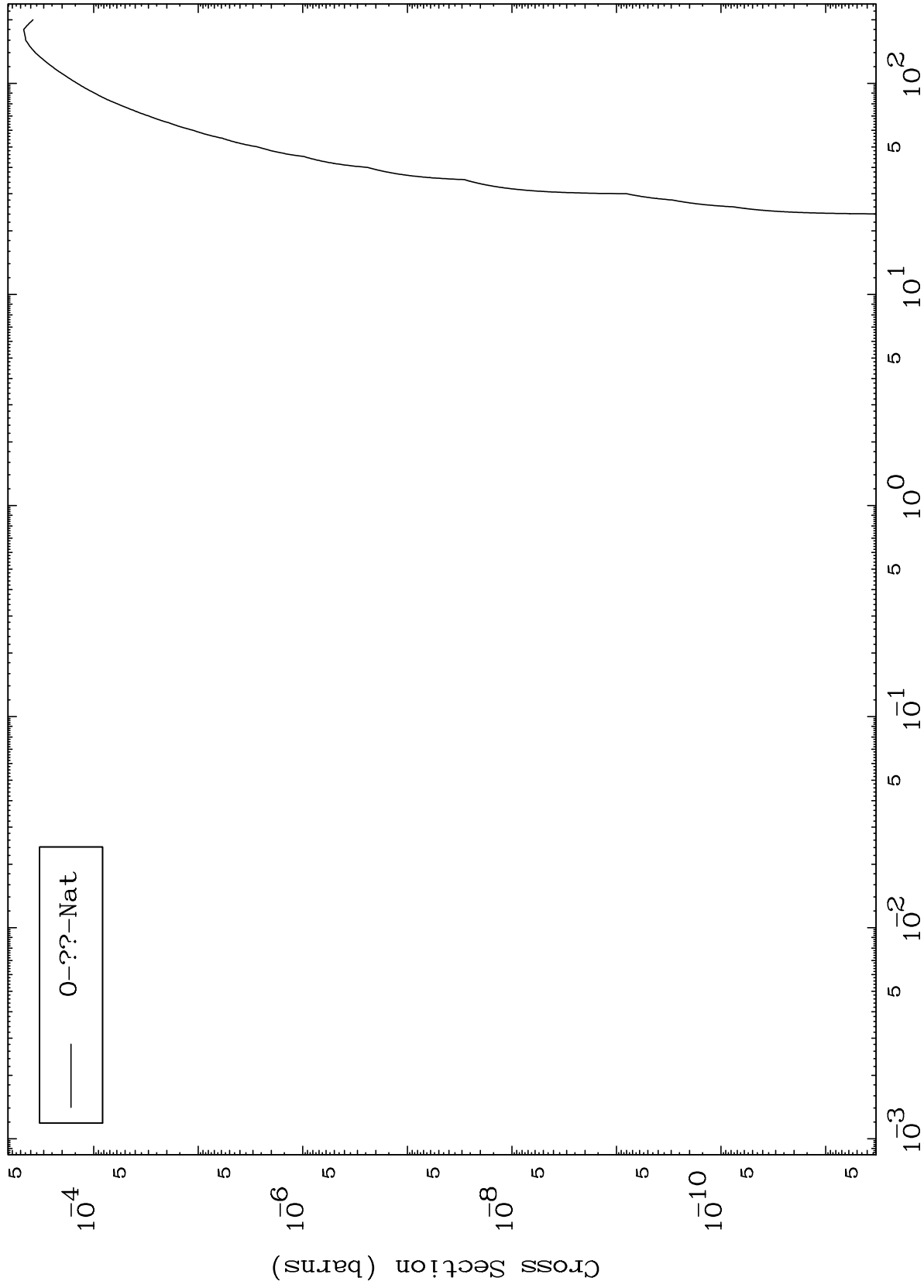
Incident Energy (MeV)

74-W -181

MAT 7428

$\alpha$  Fission  
Radionuclide Production Cross Section

74-W -181



12

Incident Energy (MeV)

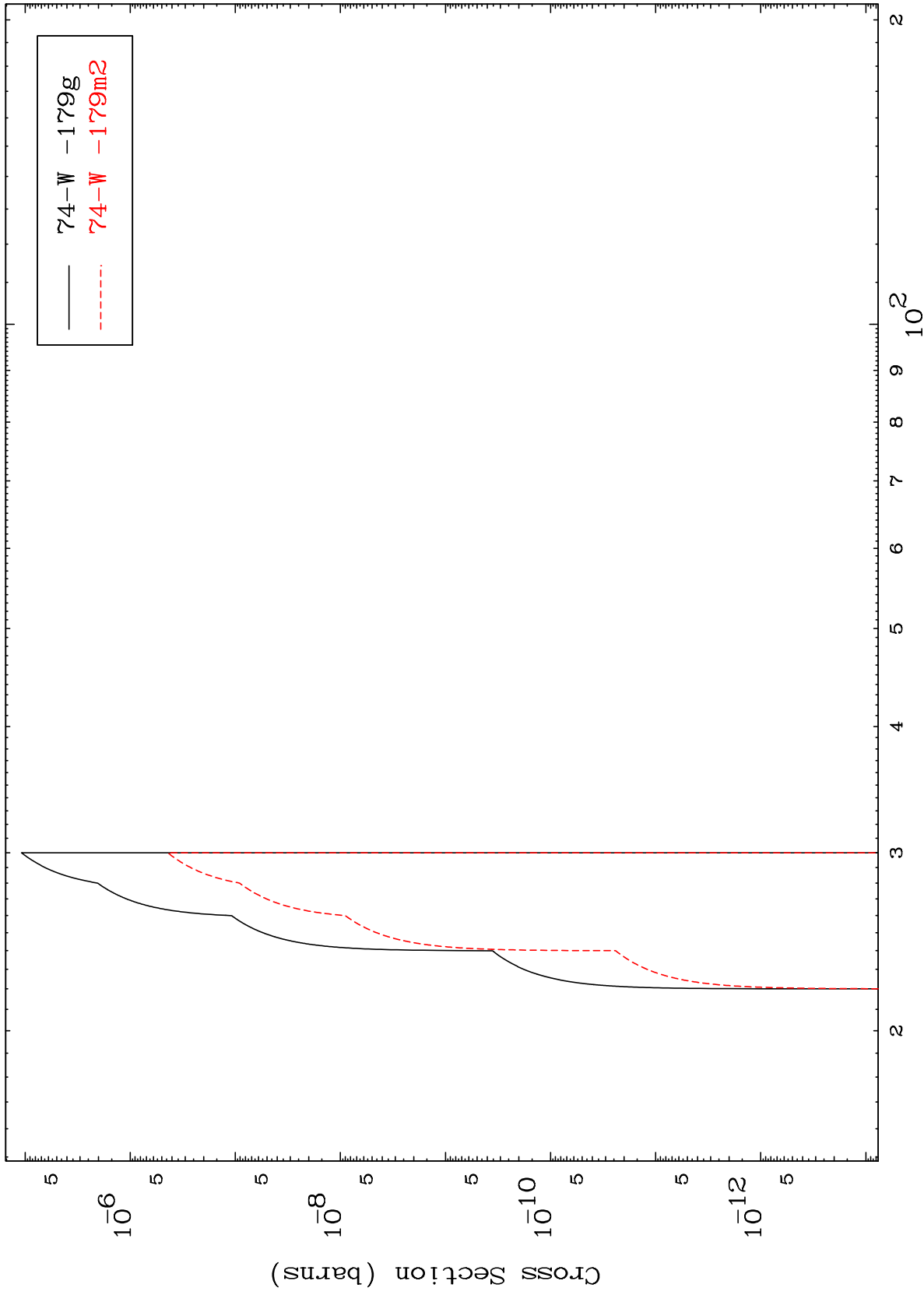
74-W -181

MAT 7428

( $\alpha, 2n$ )  $\alpha$

74-W -181

Radionuclide Production Cross Section



13

Incident Energy (MeV)

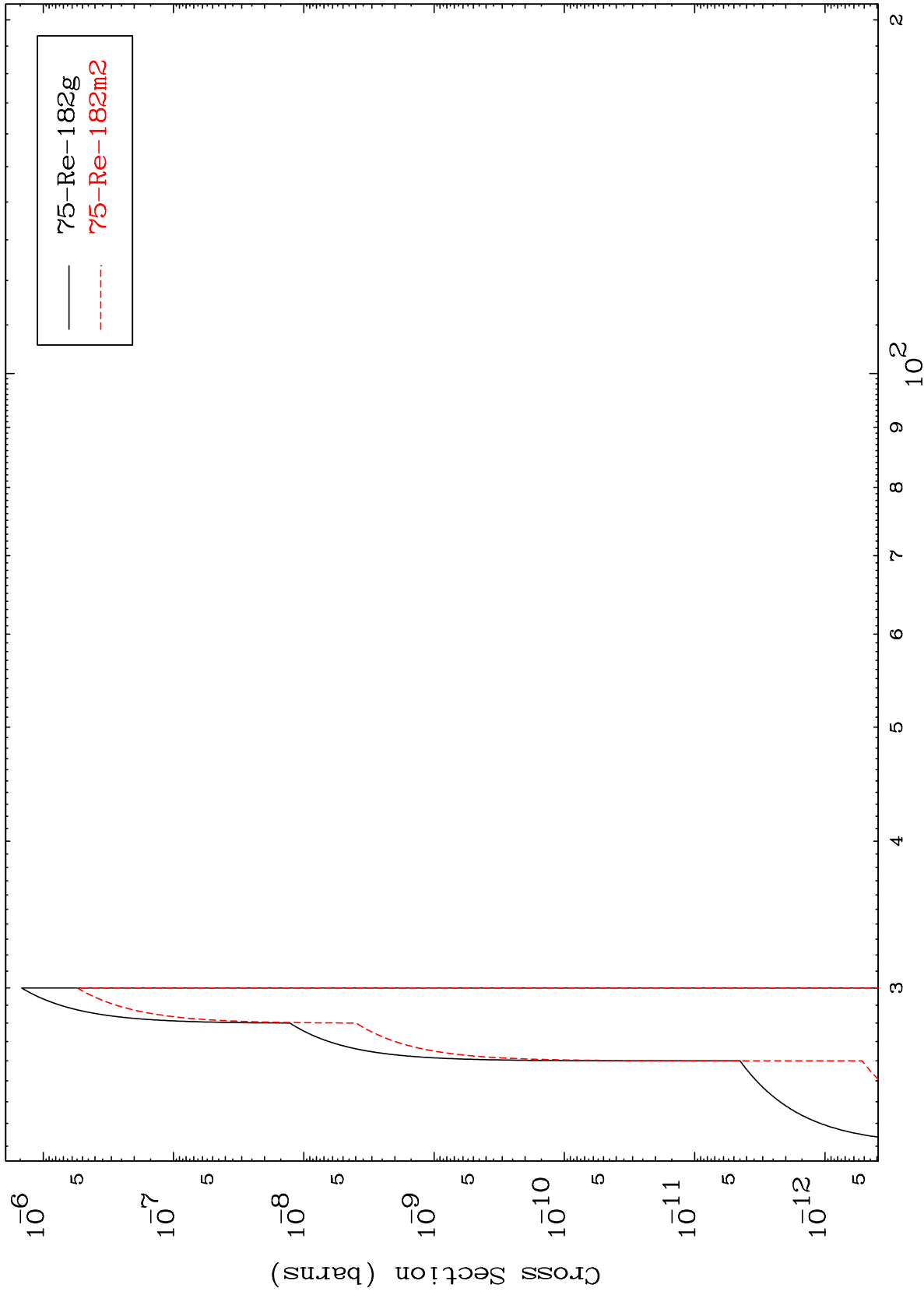
74-W -181

MAT 7428

( $\alpha, n'$ ) d

74-W -181

Radionuclide Production Cross Section



14

Incident Energy (MeV)

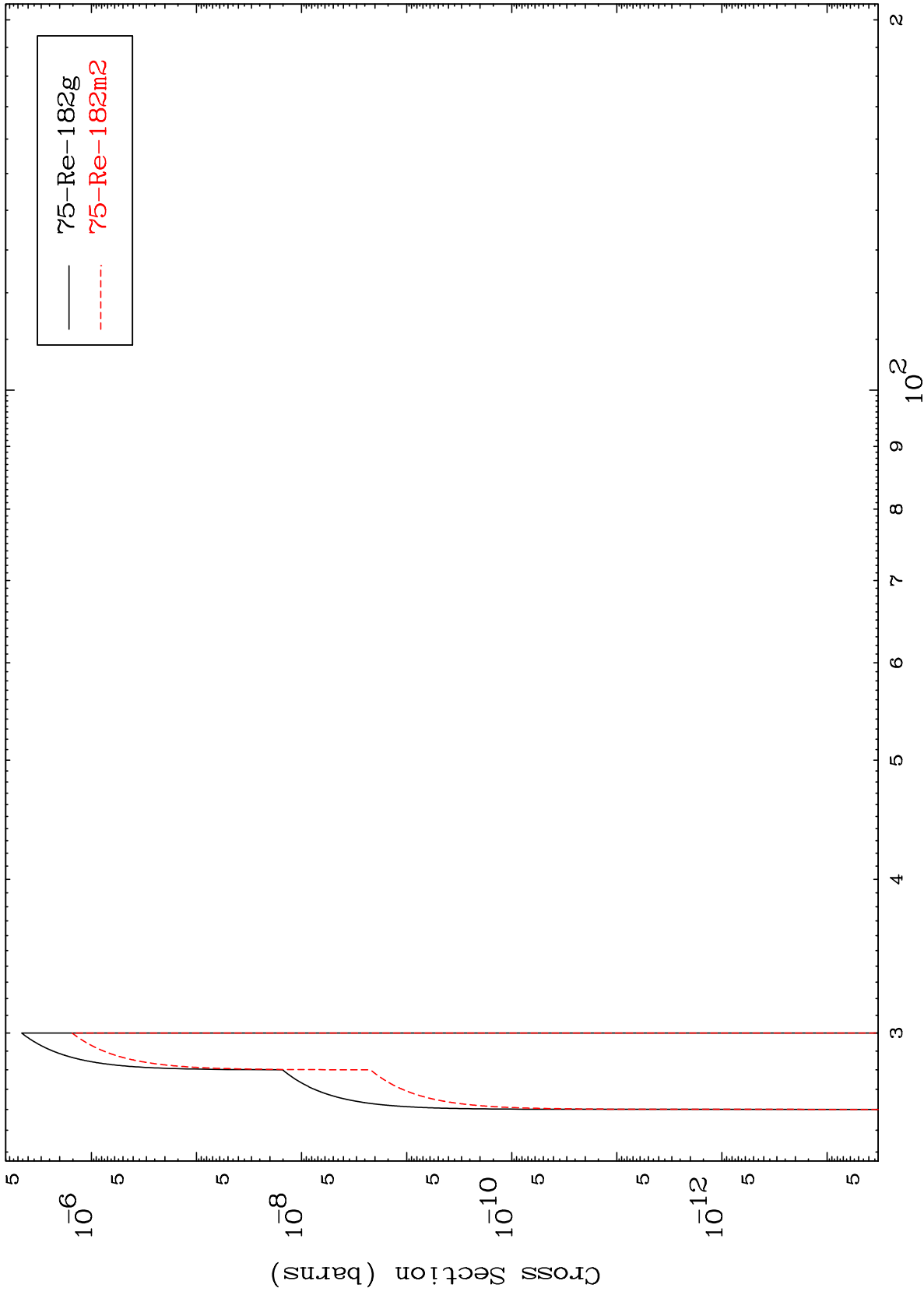
74-W -181

MAT 7428

( $\alpha, 2n$ ) p

74-W -181

Radionuclide Production Cross Section



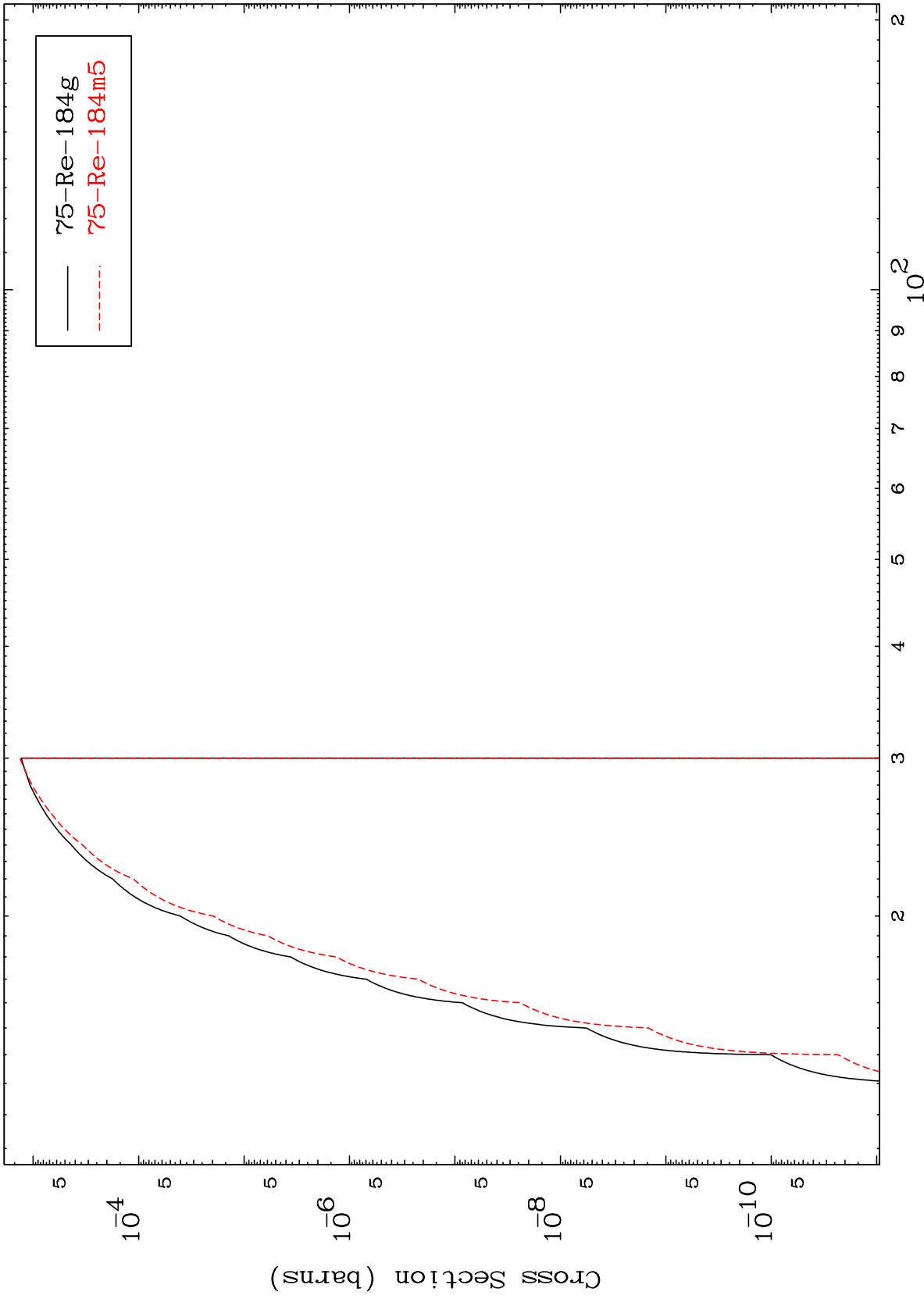
15

Incident Energy (MeV)

74-W -181



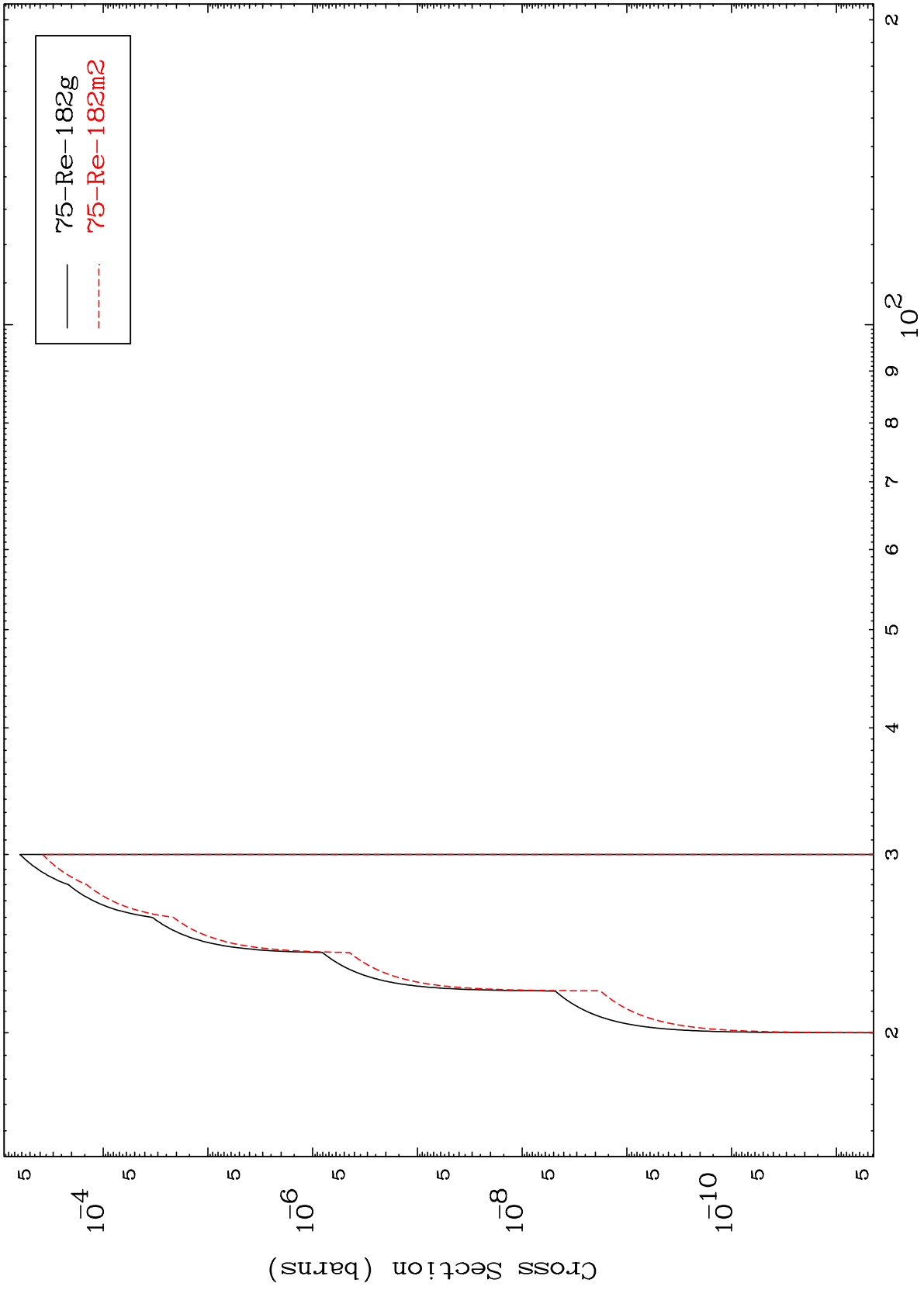
( $\alpha, p$ )  
Radionuclide Production Cross Section



MAT 7428

74-W -181

$(\alpha, t)$   
Radionuclide Production Cross Section



17

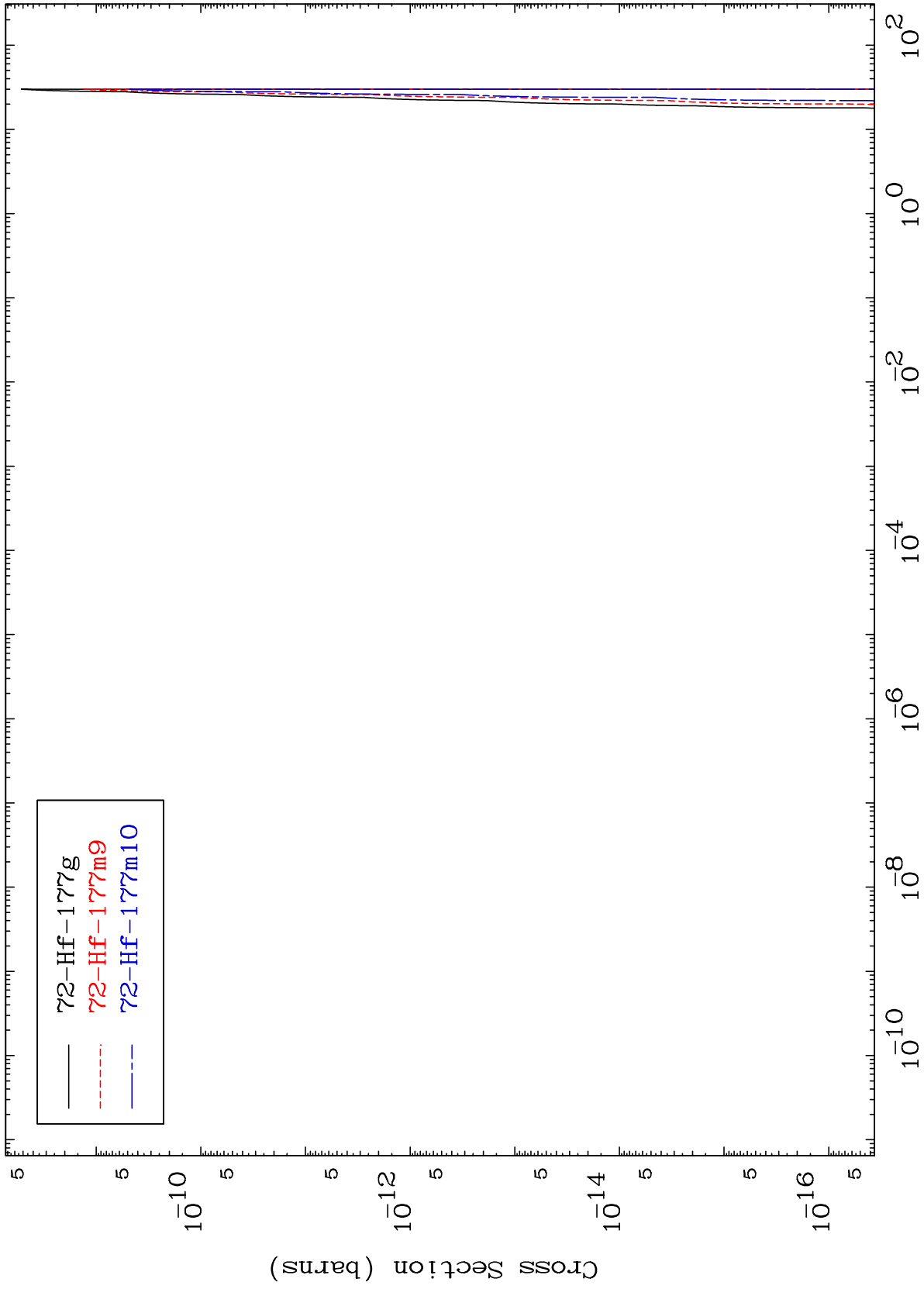
74-W -181

MAT 7428

( $\alpha, 2\alpha$ )

74-W -181

### Radionuclide Production Cross Section



18

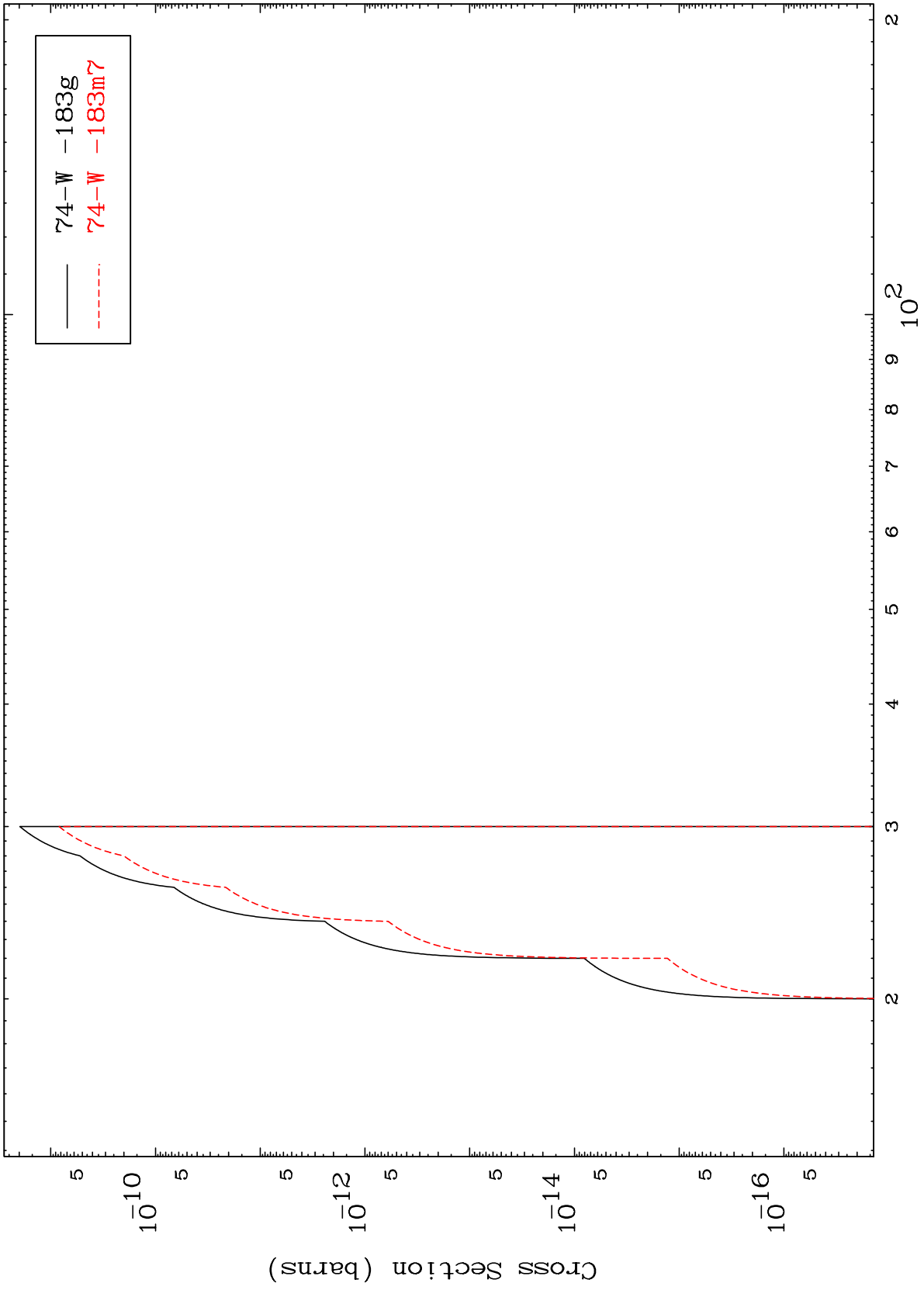
Incident Energy (MeV)

74-W -181

MAT 7428

74-W -181

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



19

74-W -181

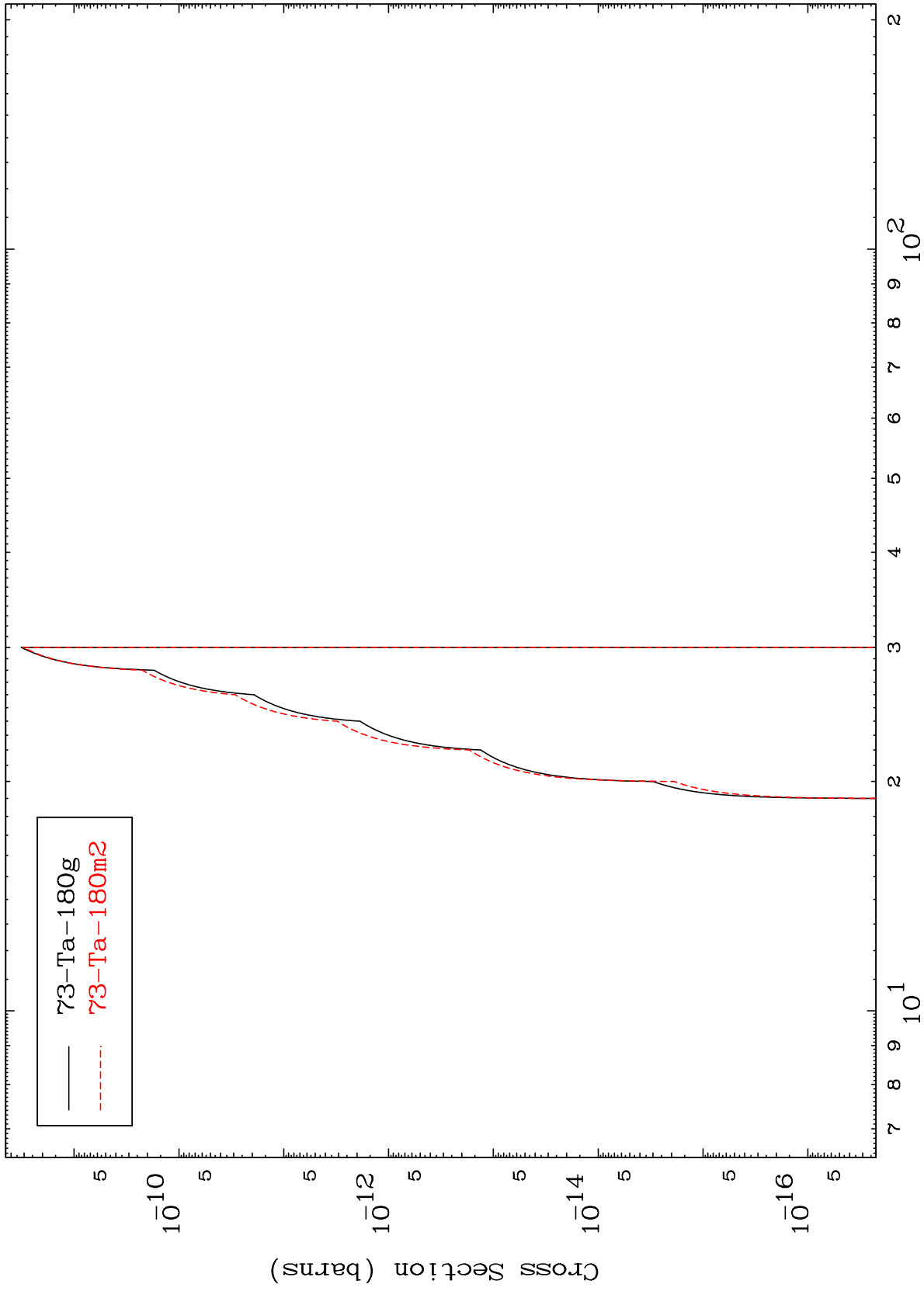
Incident Energy (MeV)

MAT 7428

( $\alpha, p$ )  $\alpha$

74-W -181

Radionuclide Production Cross Section



20

Incident Energy (MeV)

74-W -181