

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

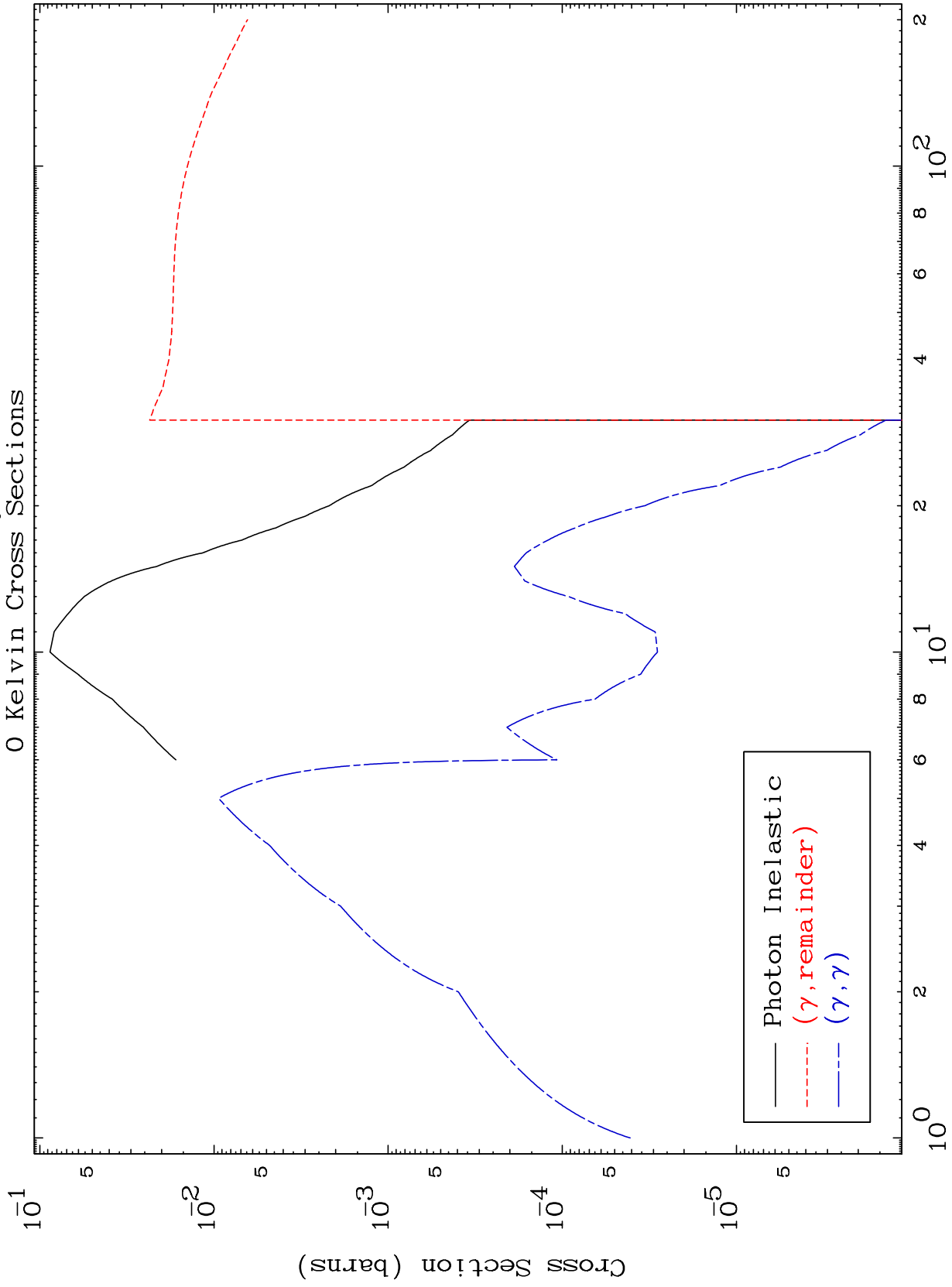
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8337

Photon Major  
0 Kelvin Cross Sections

83-Bi-213



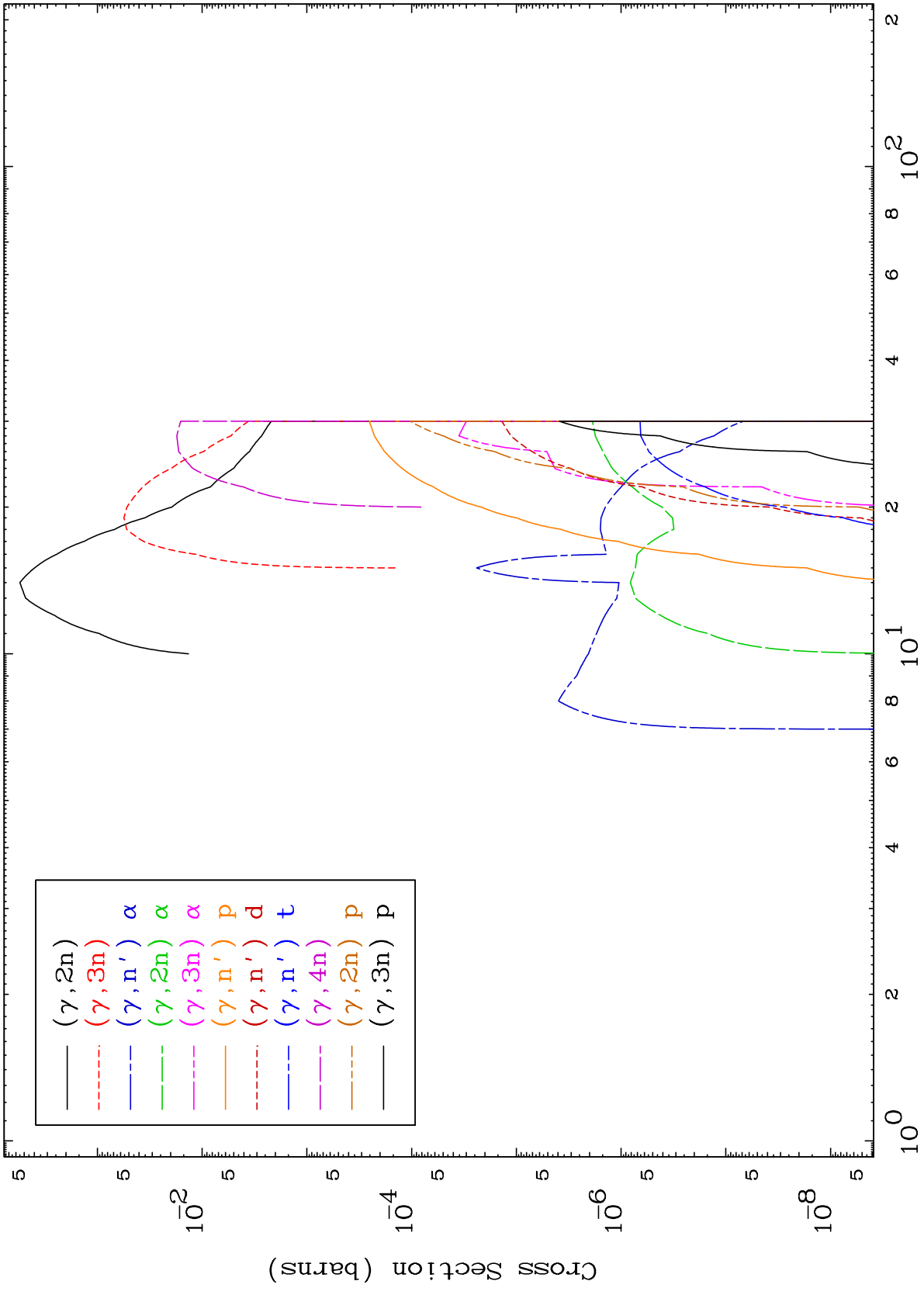
Incident Energy (MeV)

83-Bi-213

MAT 8337

Photon Neutron Production  
0 Kelvin Cross Sections

83-Bi-213



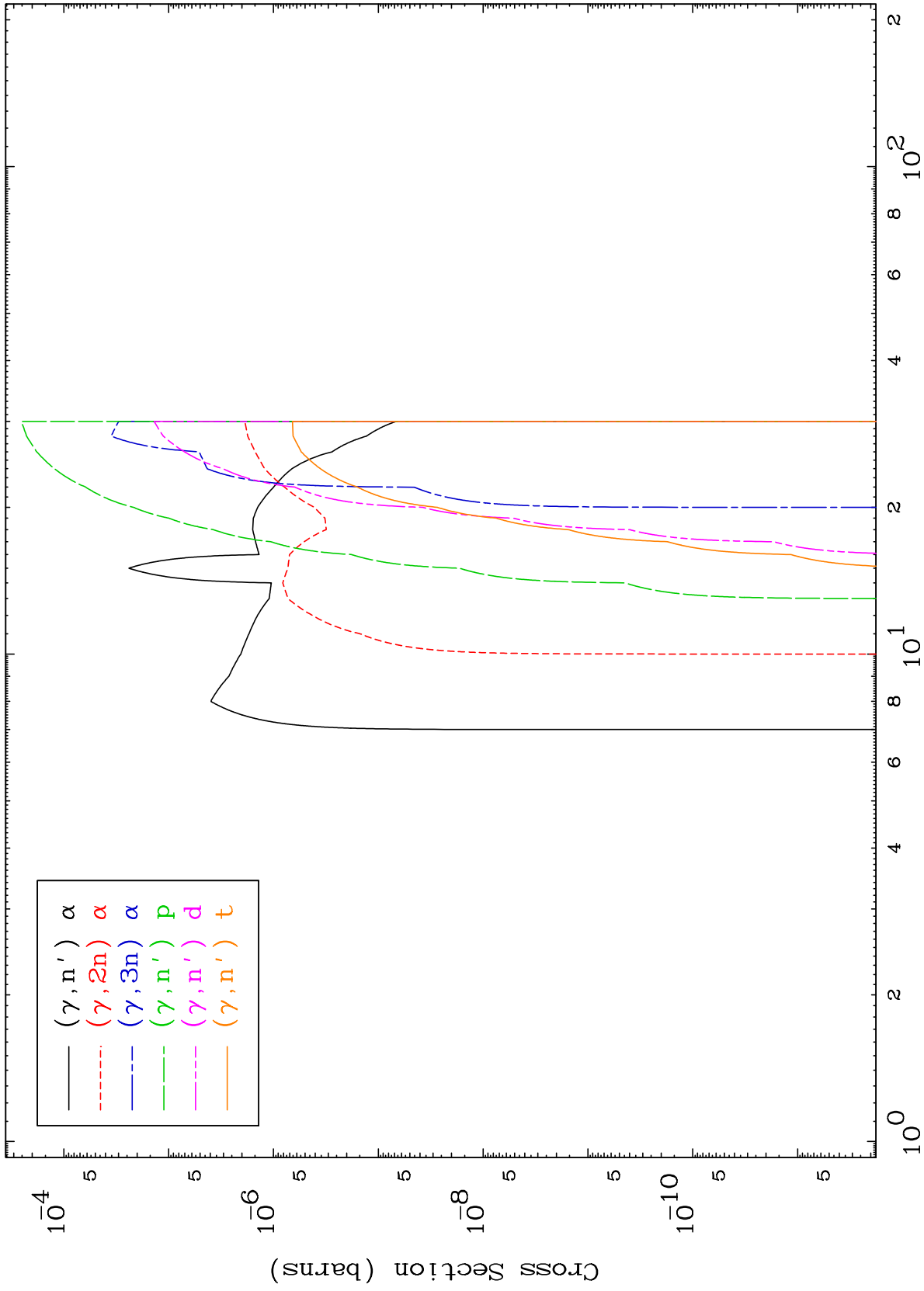
83-Bi-213

Incident Energy (MeV)

MAT 8337

Photon Charged Particle  
0 Kelvin Cross Sections

83-Bi-213



83-Bi-213

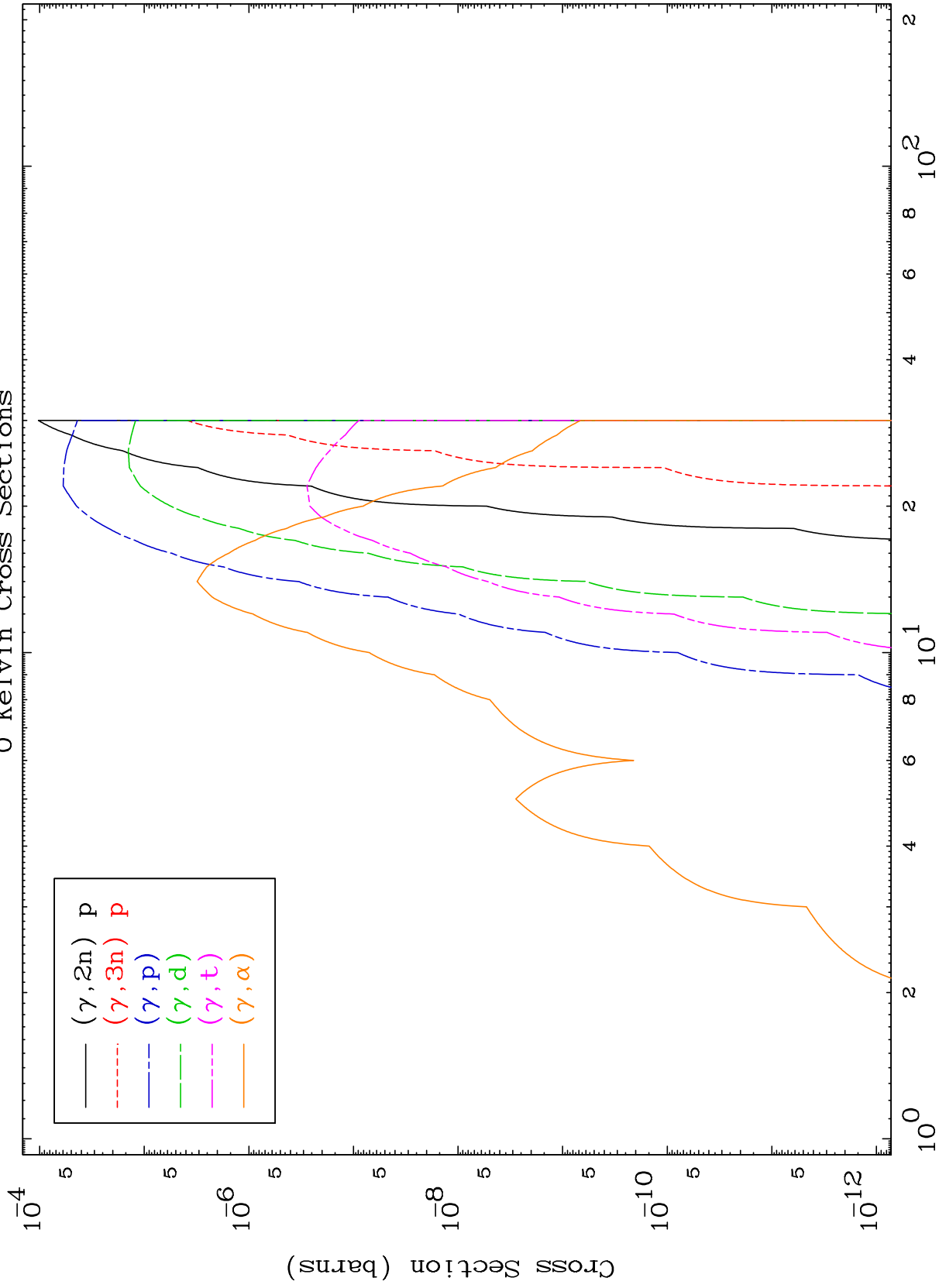
Incident Energy (MeV)

3

MAT 8337

Photon Charged Particle  
0 Kelvin Cross Sections

83-Bi-213



Incident Energy (MeV)

83-Bi-213

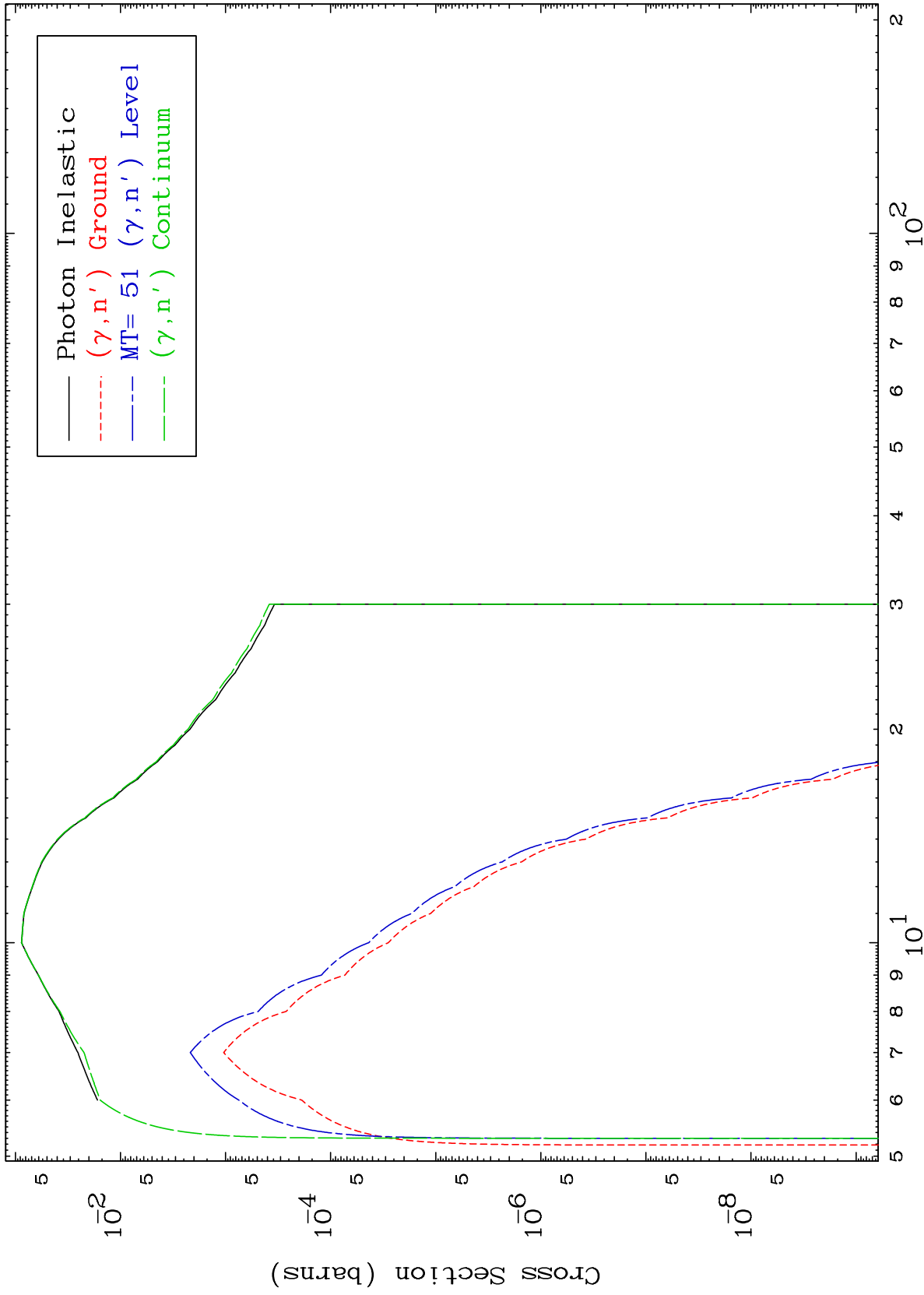
4

MAT 8337

$(\gamma, n')$  Level

83-Bi-213

0 Kelvin Cross Sections



5

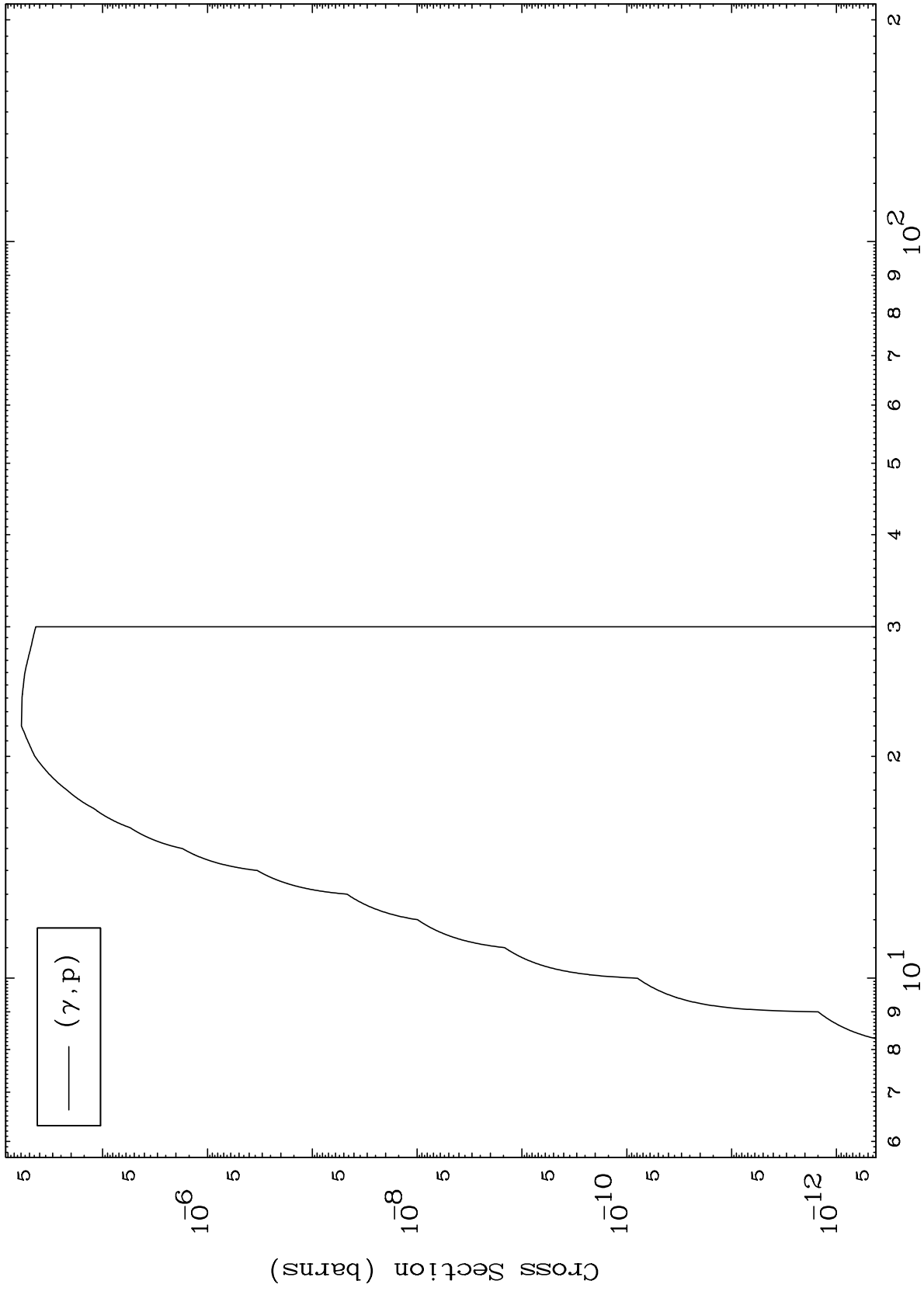
Incident Energy (MeV)

83-Bi-213

MAT 8337

( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections

83-Bi-213



6

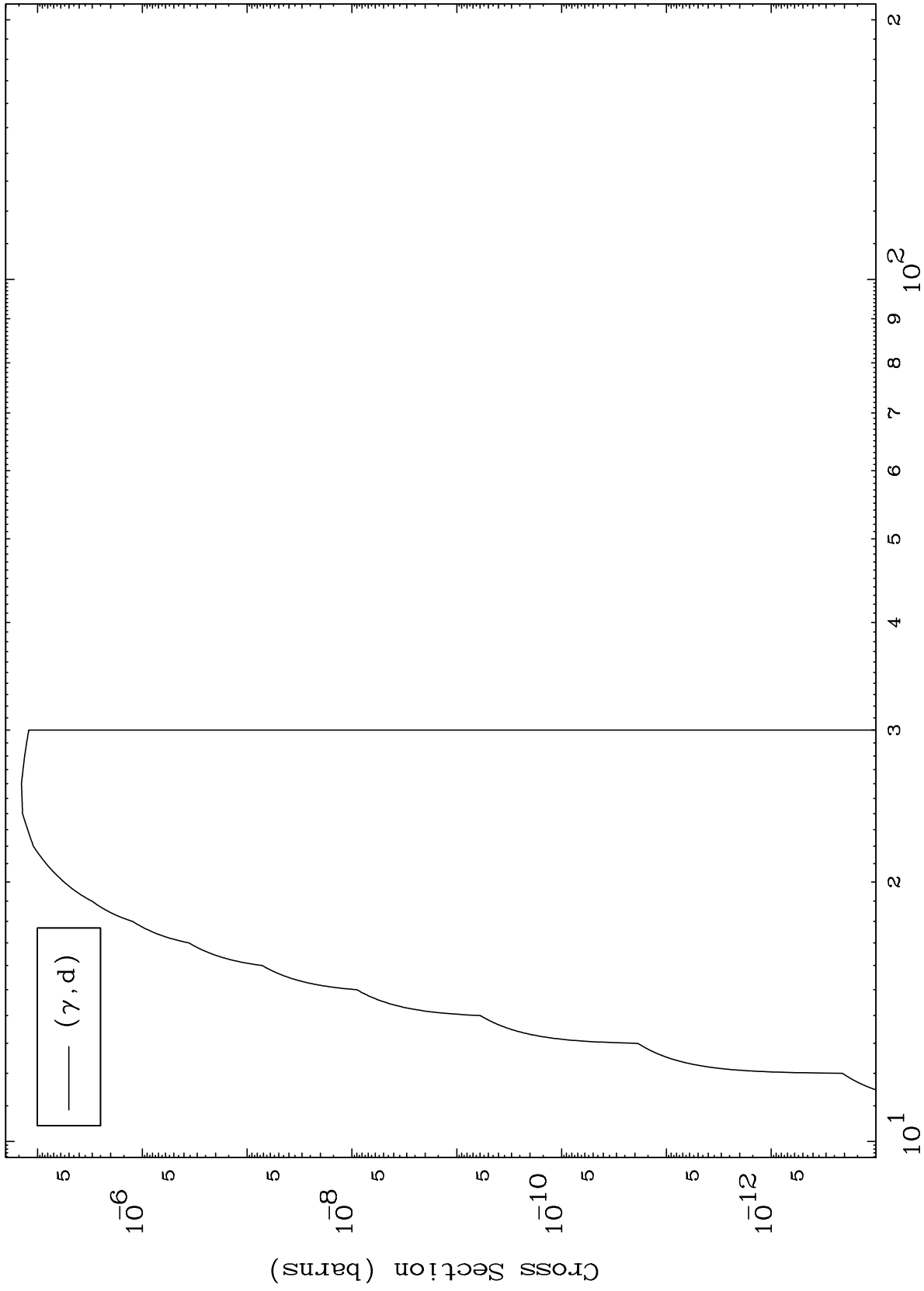
Incident Energy (MeV)

83-Bi-213

MAT 8337

( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

83-Bi-213



Incident Energy (MeV)

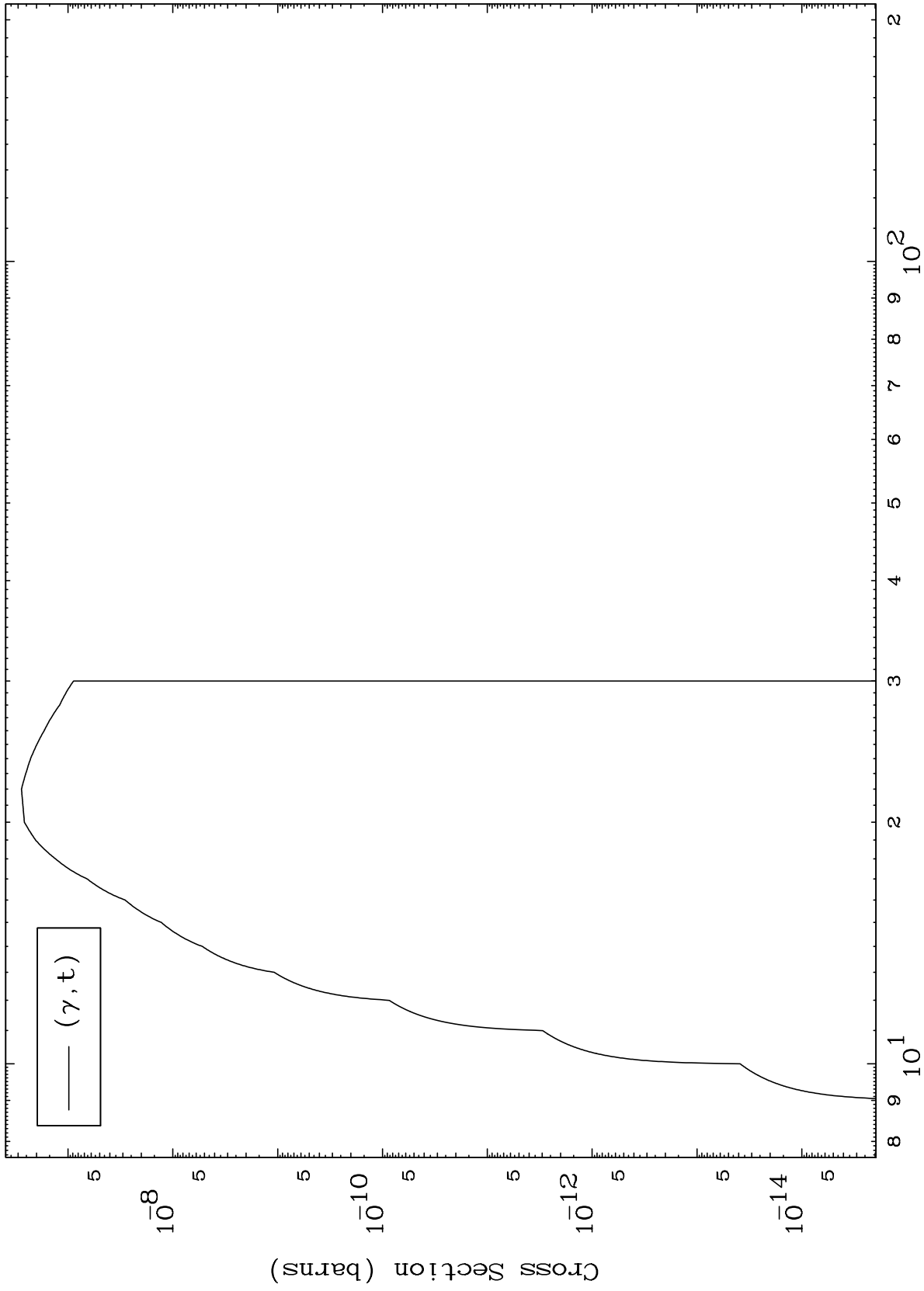
83-Bi-213



MAT 8337

( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

83-Bi-213



8

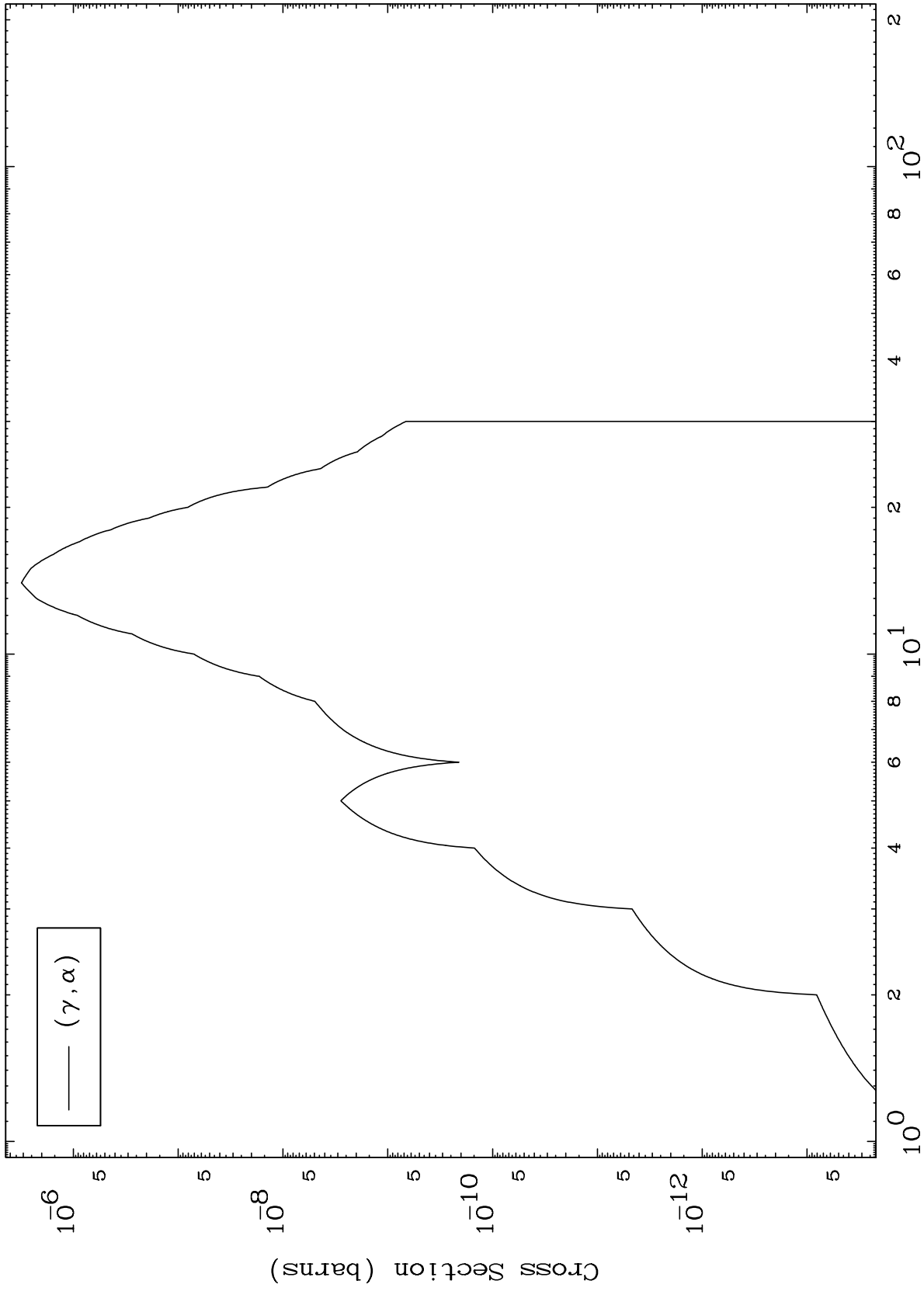
Incident Energy (MeV)

83-Bi-213

MAT 8337

( $\gamma, \alpha$ ) Levels  
0 Kelvin Cross Sections

83-Bi-213



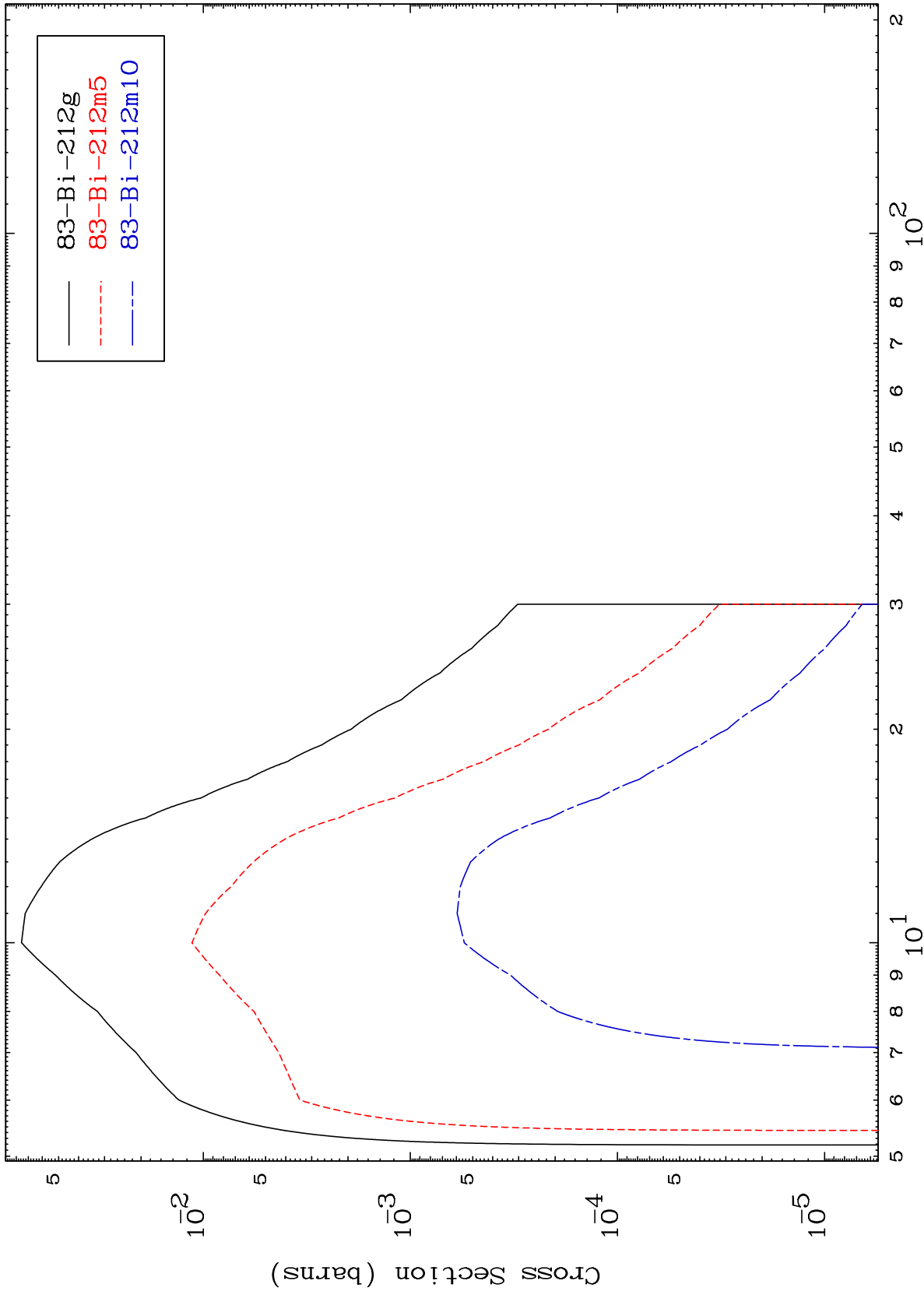
Incident Energy (MeV)

83-Bi-213

MAT 8337

Photon Inelastic  
Radionuclide Production Cross Section

83-Bi-213



10

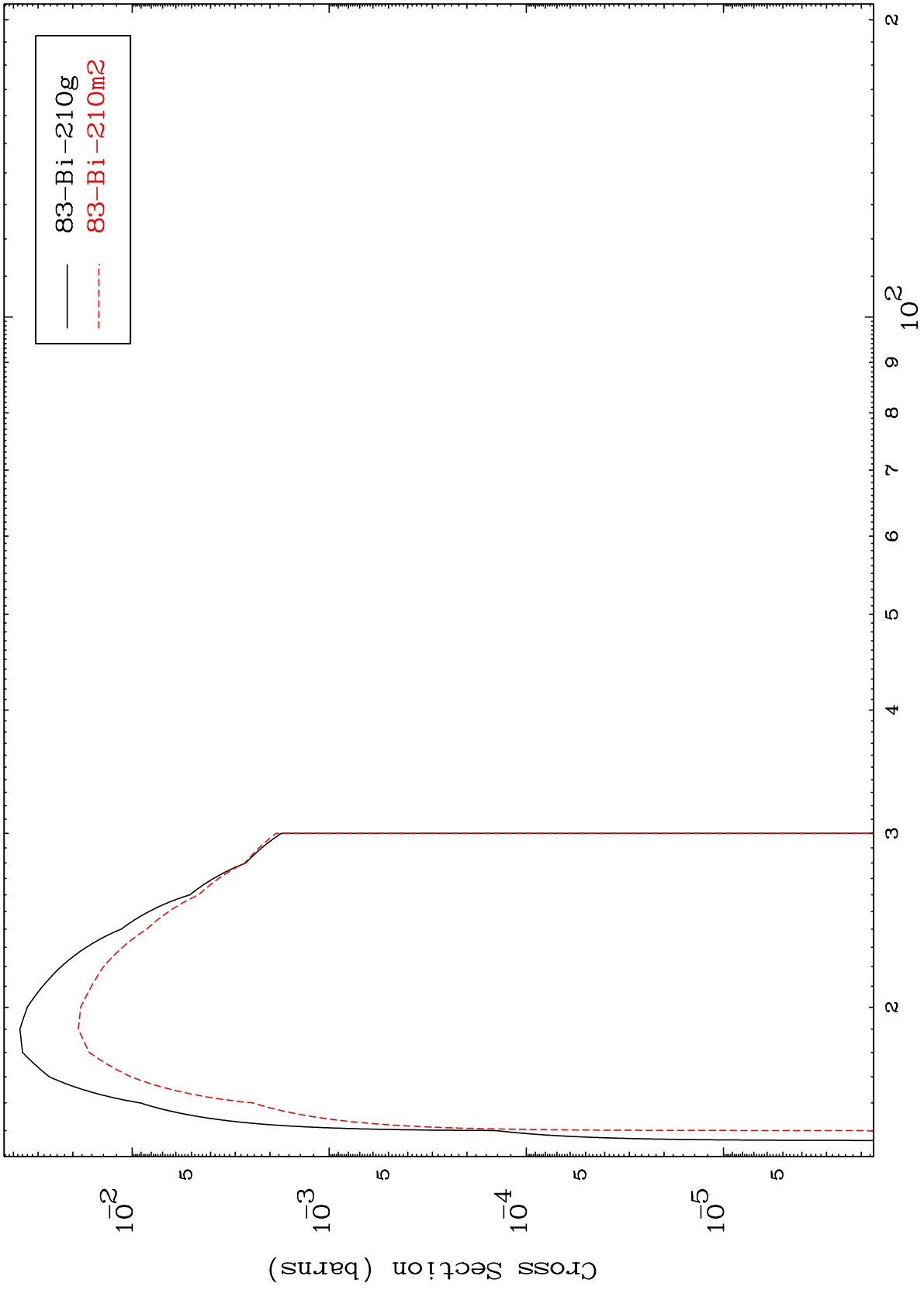
Incident Energy (MeV)

83-Bi-213

MAT 8337

83-Bi-213

( $\gamma, 3n$ )  
Radionuclide Production Cross Section



11

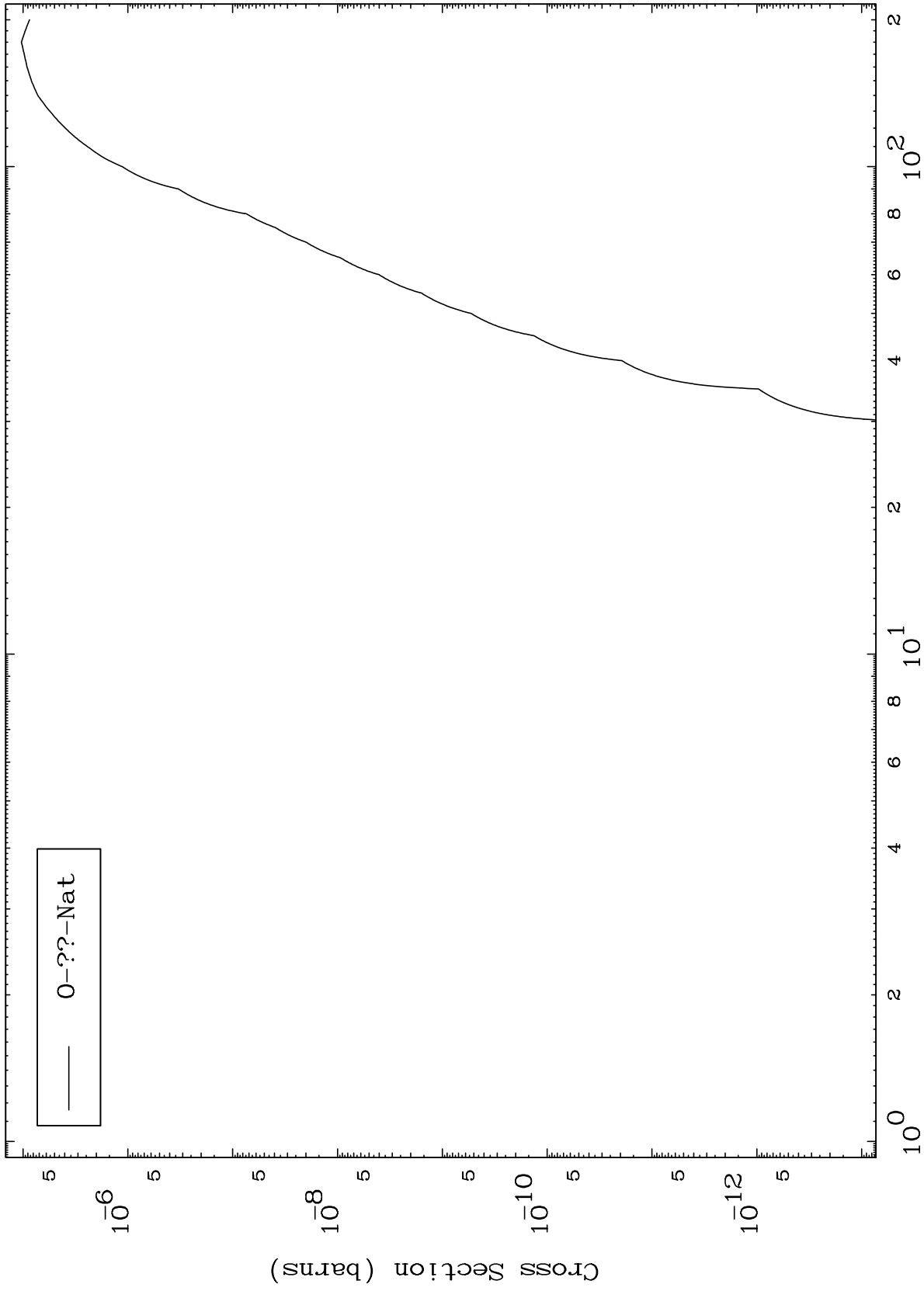
Incident Energy (MeV)

83-Bi-213

MAT 8337

Photon Fission  
Radionuclide Production Cross Section

83-Bi-213



Incident Energy (MeV)

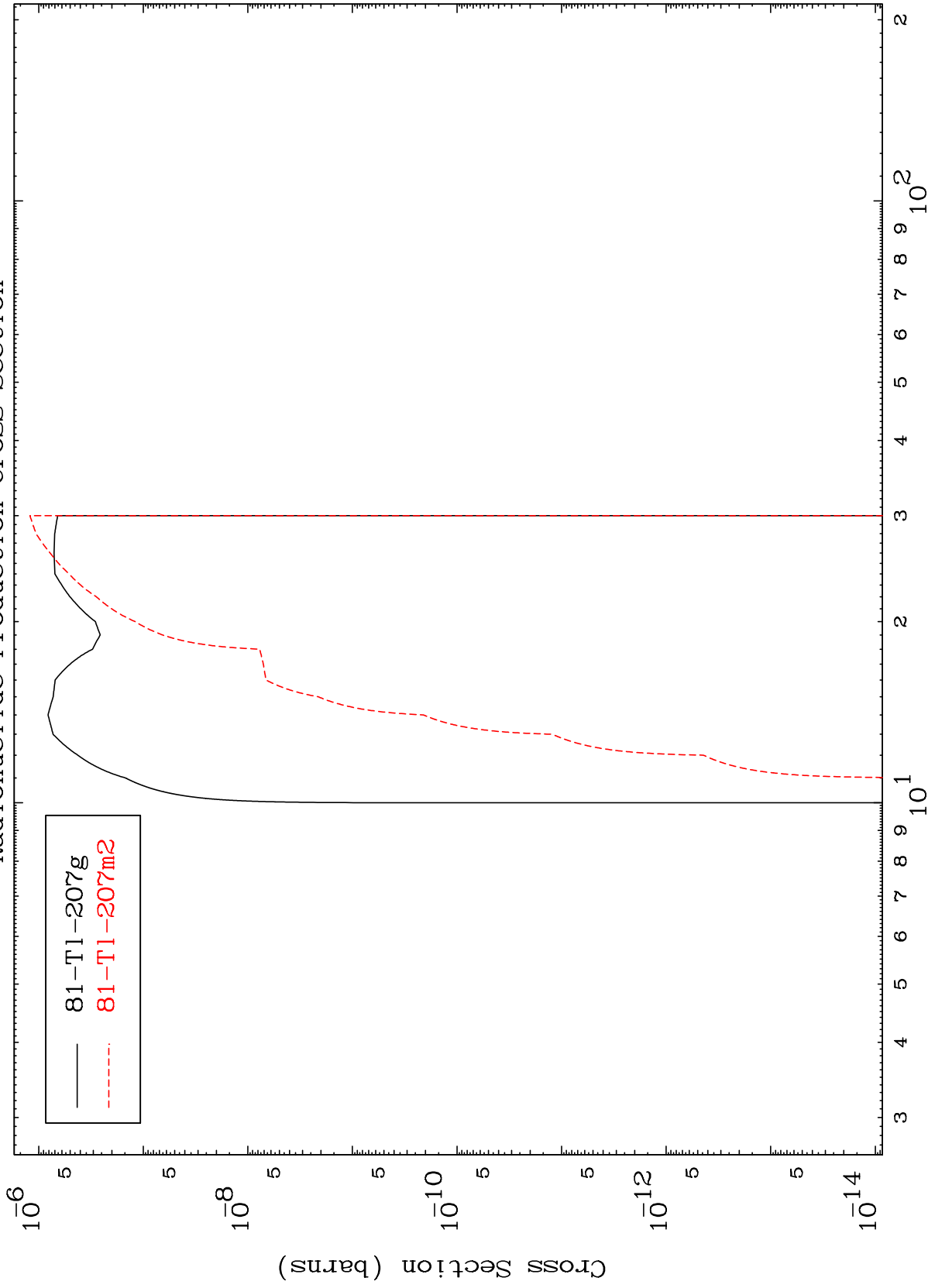
83-Bi-213

MAT 8337

83-Bi-213

( $\gamma, 2n$ )  $\alpha$

Radionuclide Production Cross Section



13

Incident Energy (MeV)

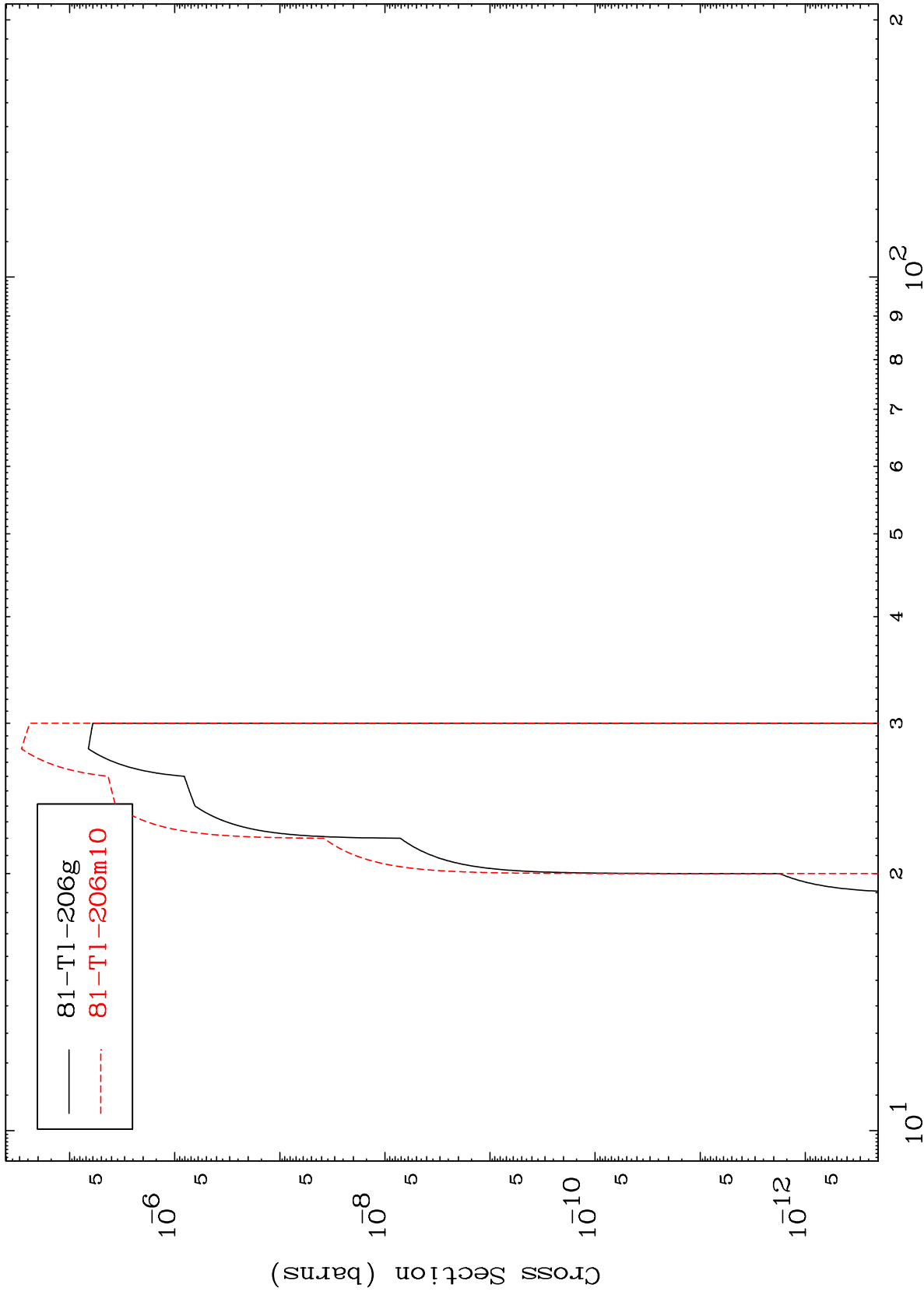
83-Bi-213

MAT 8337

$(\gamma, 3n) \alpha$

$^{83}\text{Bi}-213$

Radionuclide Production Cross Section



14

Incident Energy (MeV)

$^{83}\text{Bi}-213$