

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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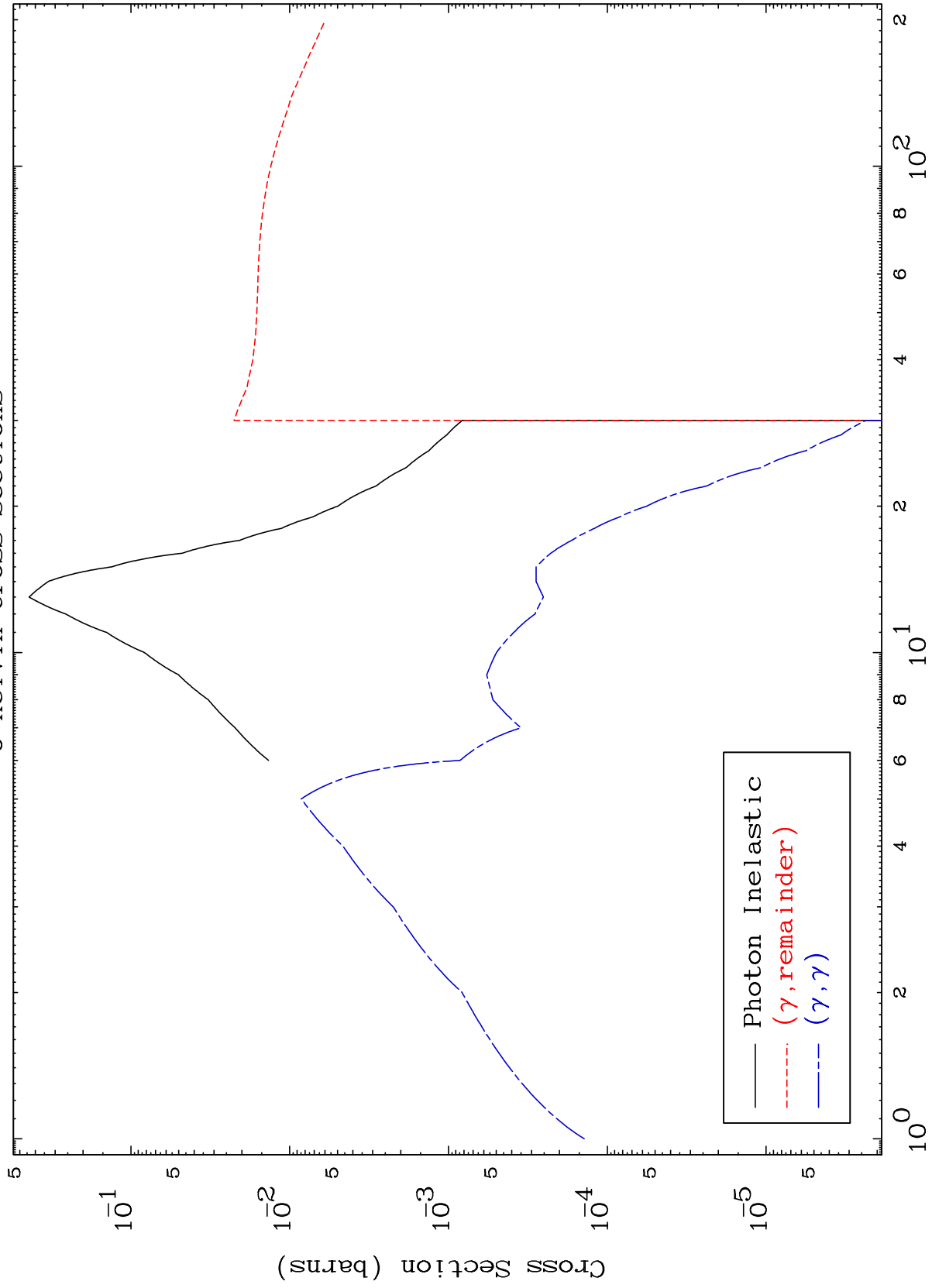
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7740

Photon Major  
0 Kelvin Cross Sections

77-Ir-196

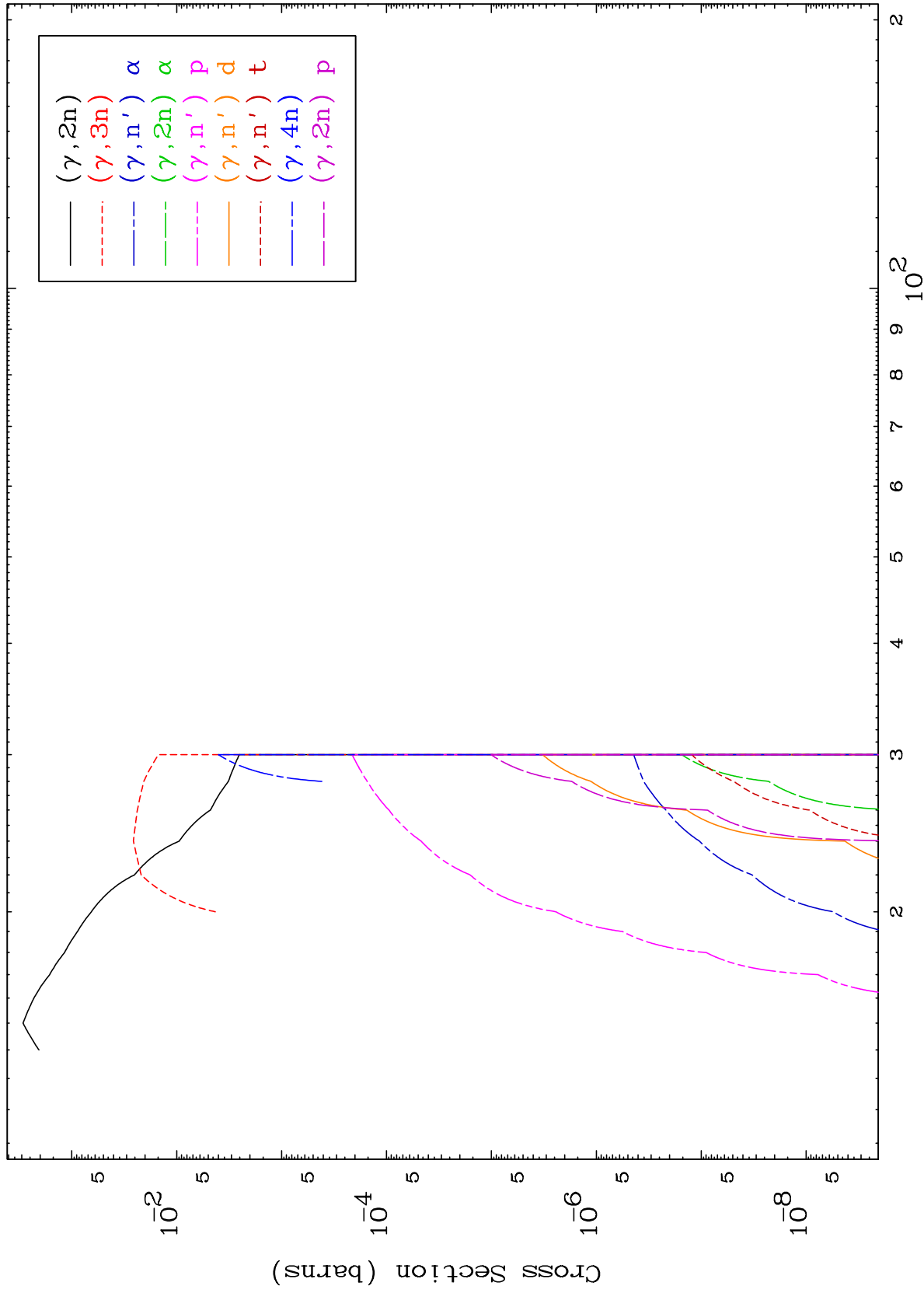


77-Ir-196

MAT 7740

Photon Neutron Production  
0 Kelvin Cross Sections

77-Ir-196



2

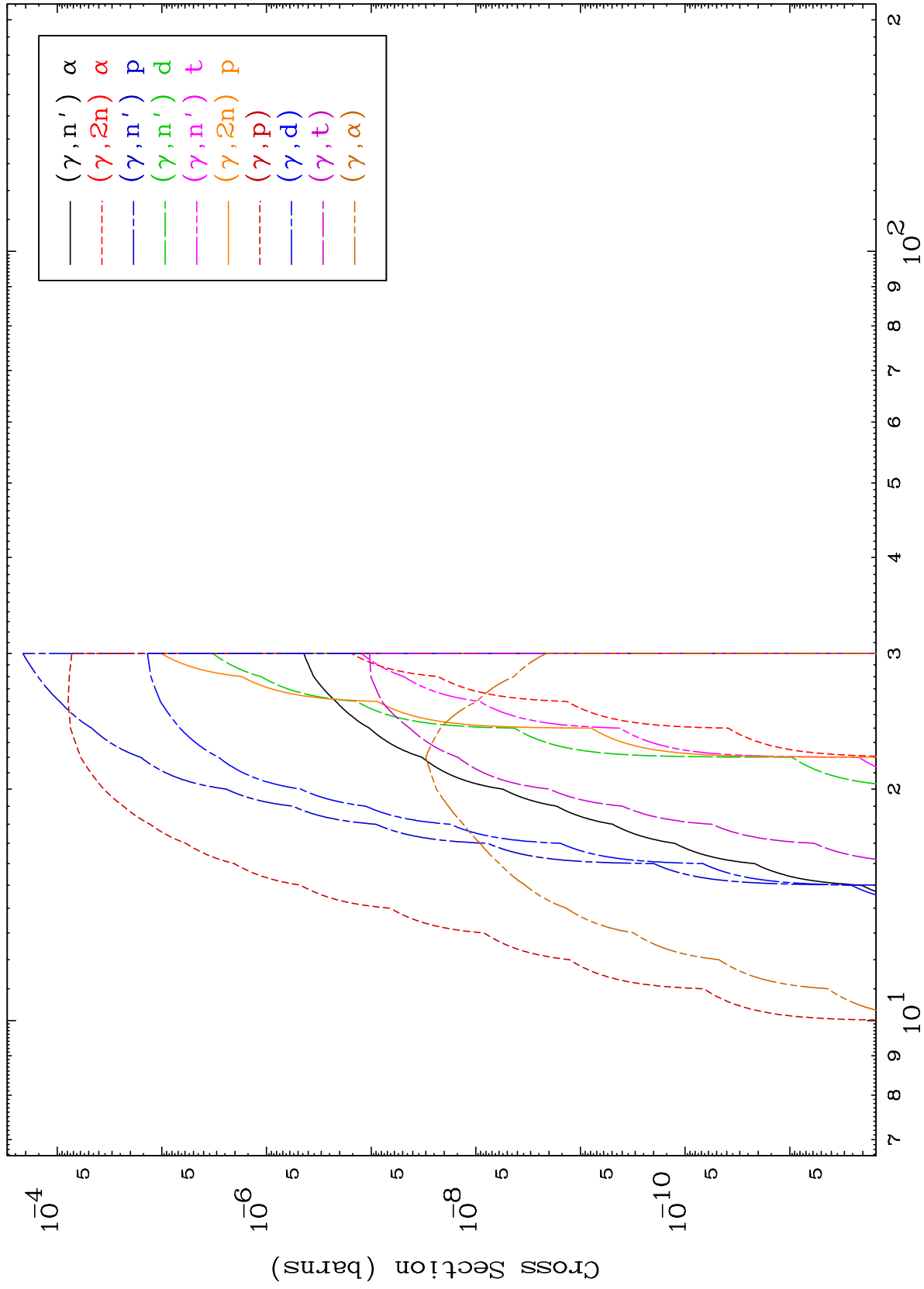
Incident Energy (MeV)

77-Ir-196

MAT 7740

Photon Charged Particle  
0 Kelvin Cross Sections

77-Ir-196



3

Incident Energy (MeV)

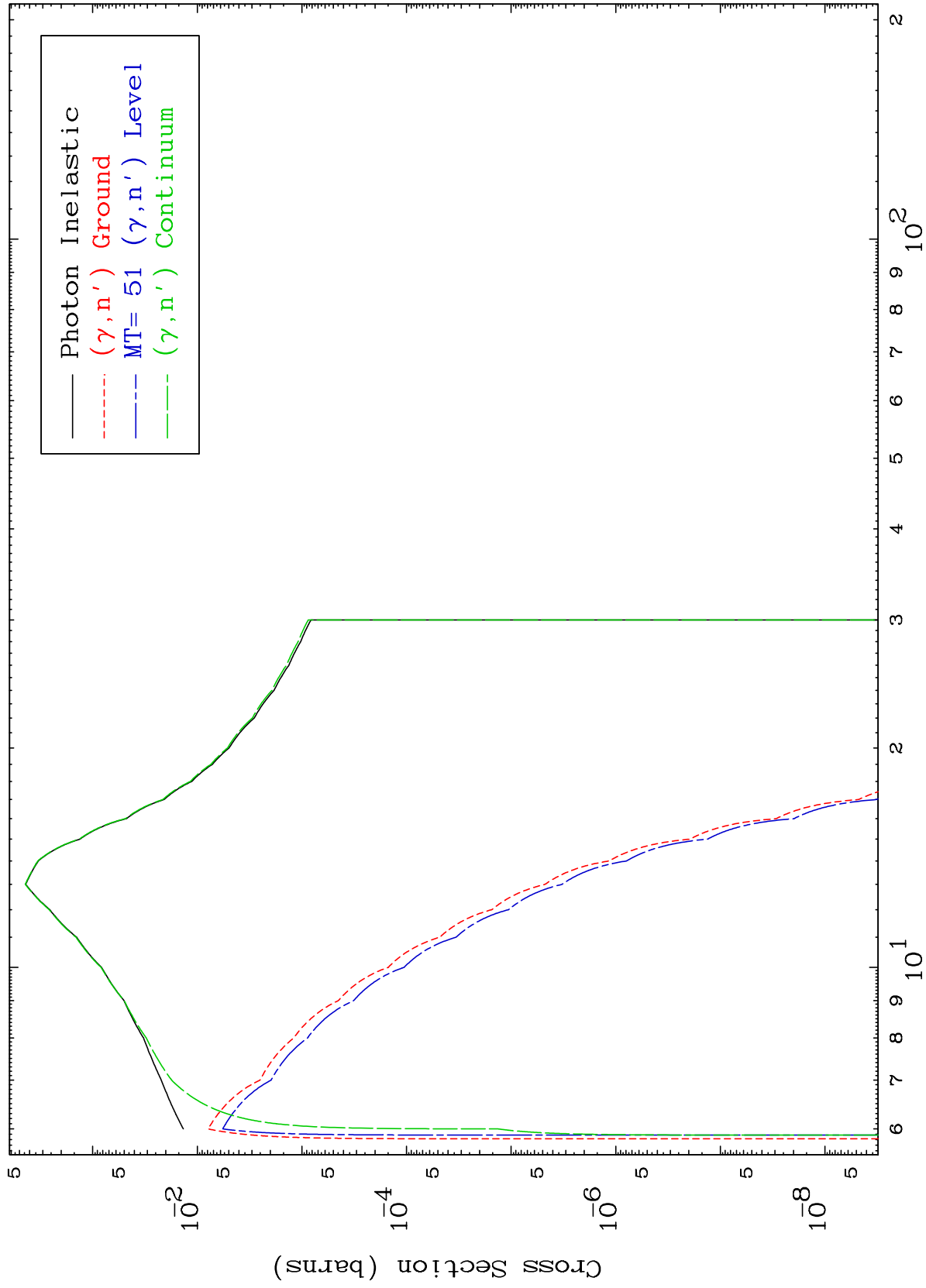
77-Ir-196

MAT 7740

$(\gamma, n')$  Level

0 Kelvin Cross Sections

77-Ir-196



4

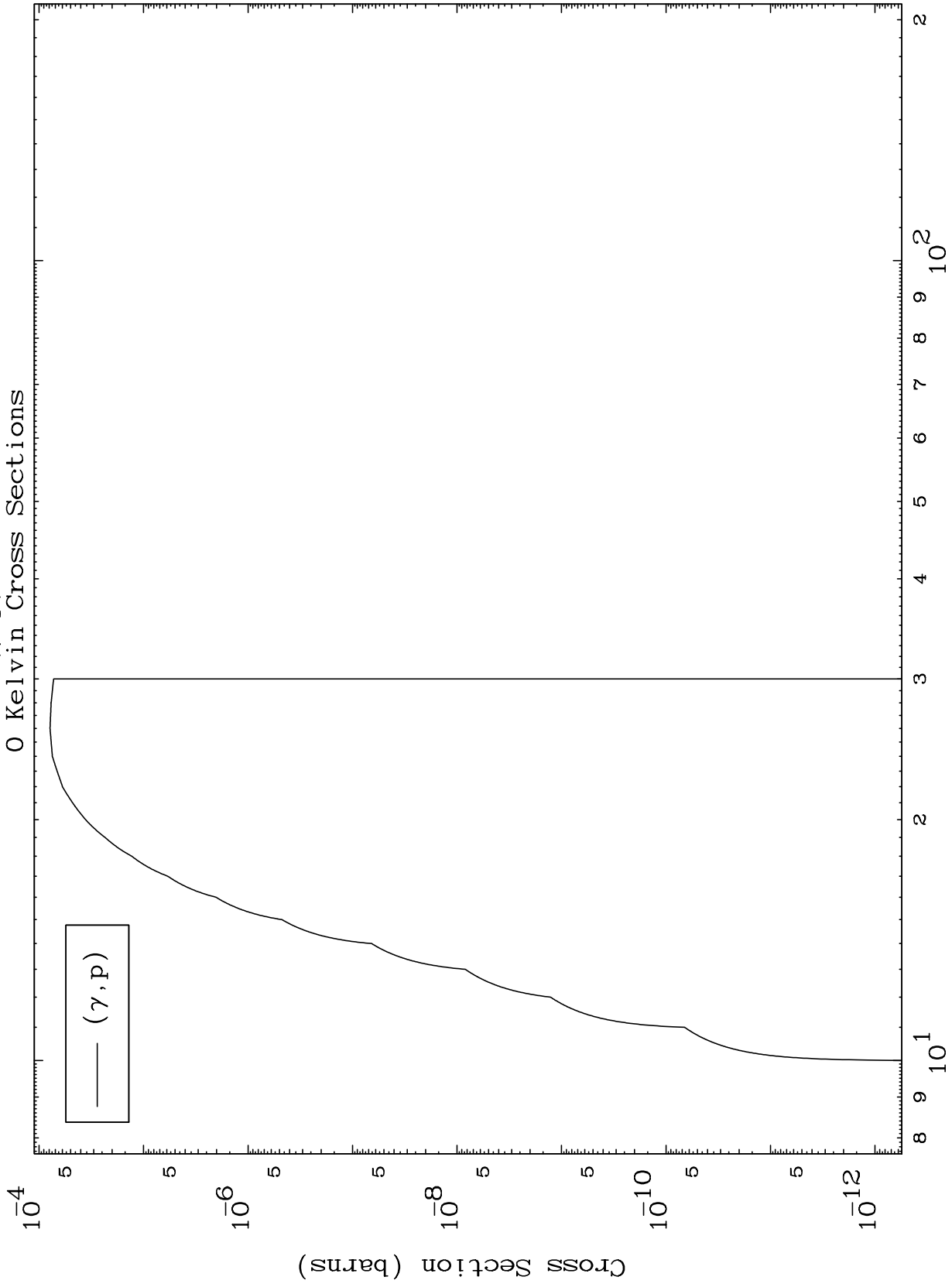
Incident Energy (MeV)

77-Ir-196

MAT 7740

( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections

77-Ir-196



5

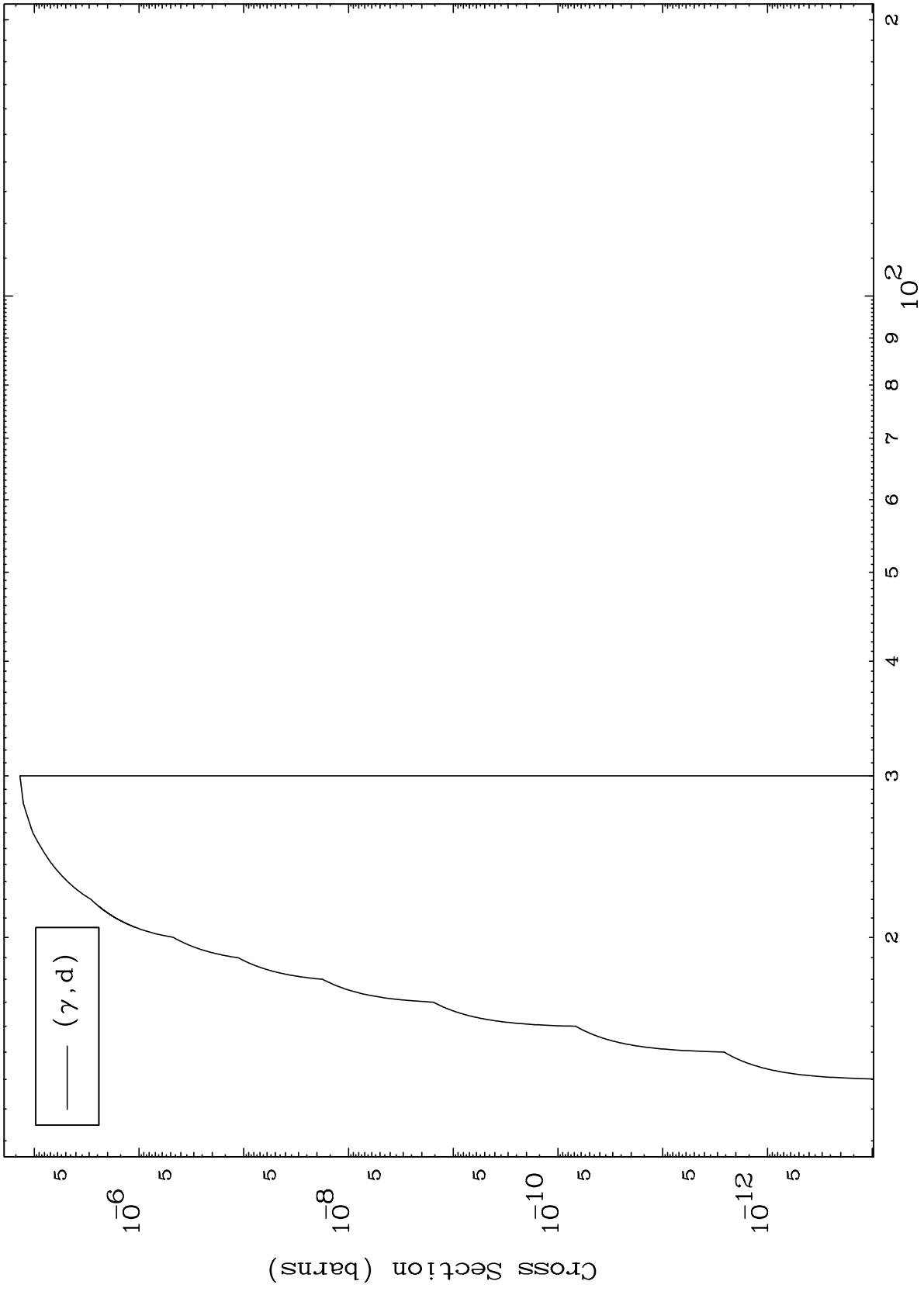
Incident Energy (MeV)

77-Ir-196

MAT 7740

( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

77-Ir-196



6

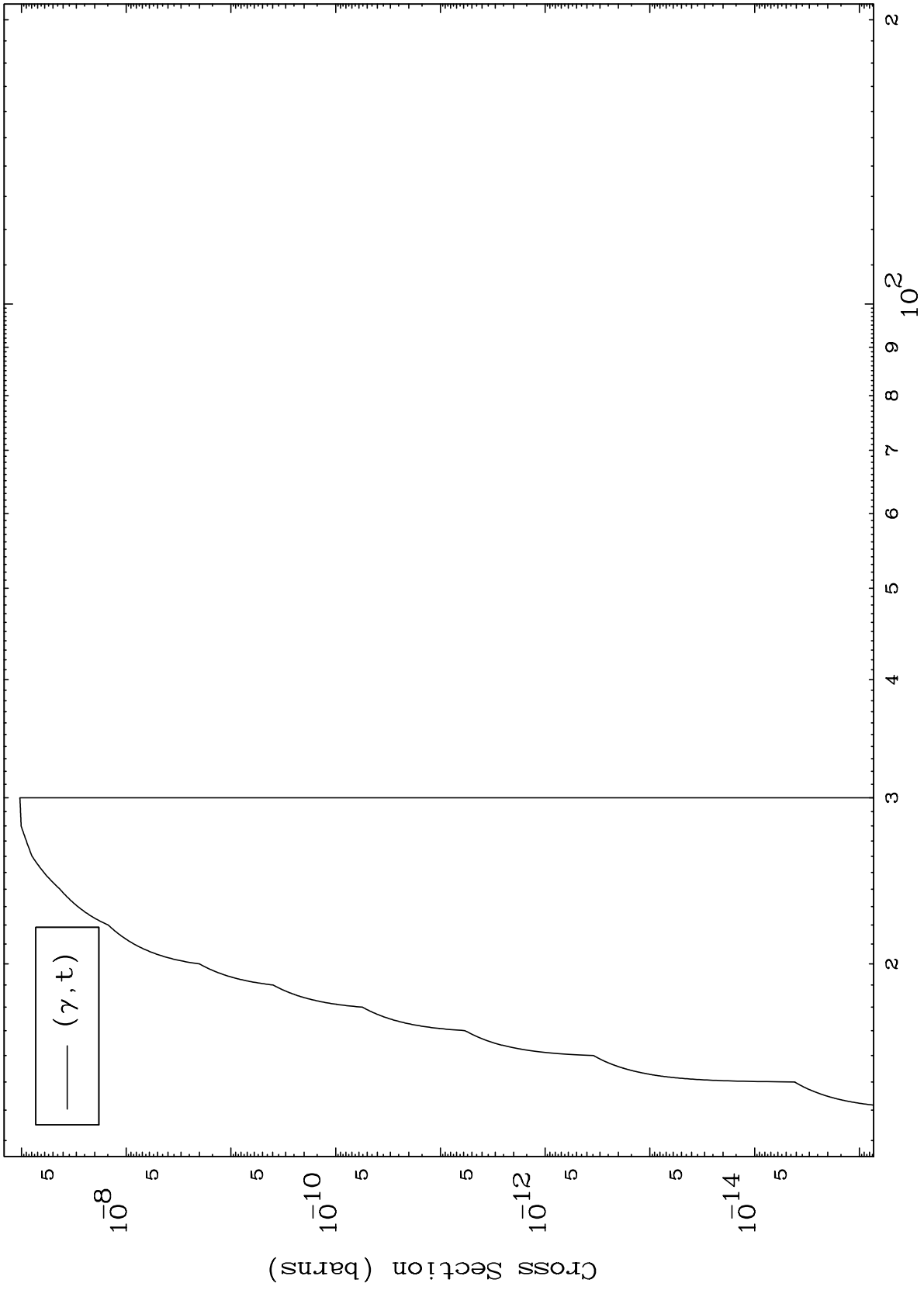
Incident Energy (MeV)

77-Ir-196

MAT 7740

( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

77-Ir-196



7

Incident Energy (MeV)

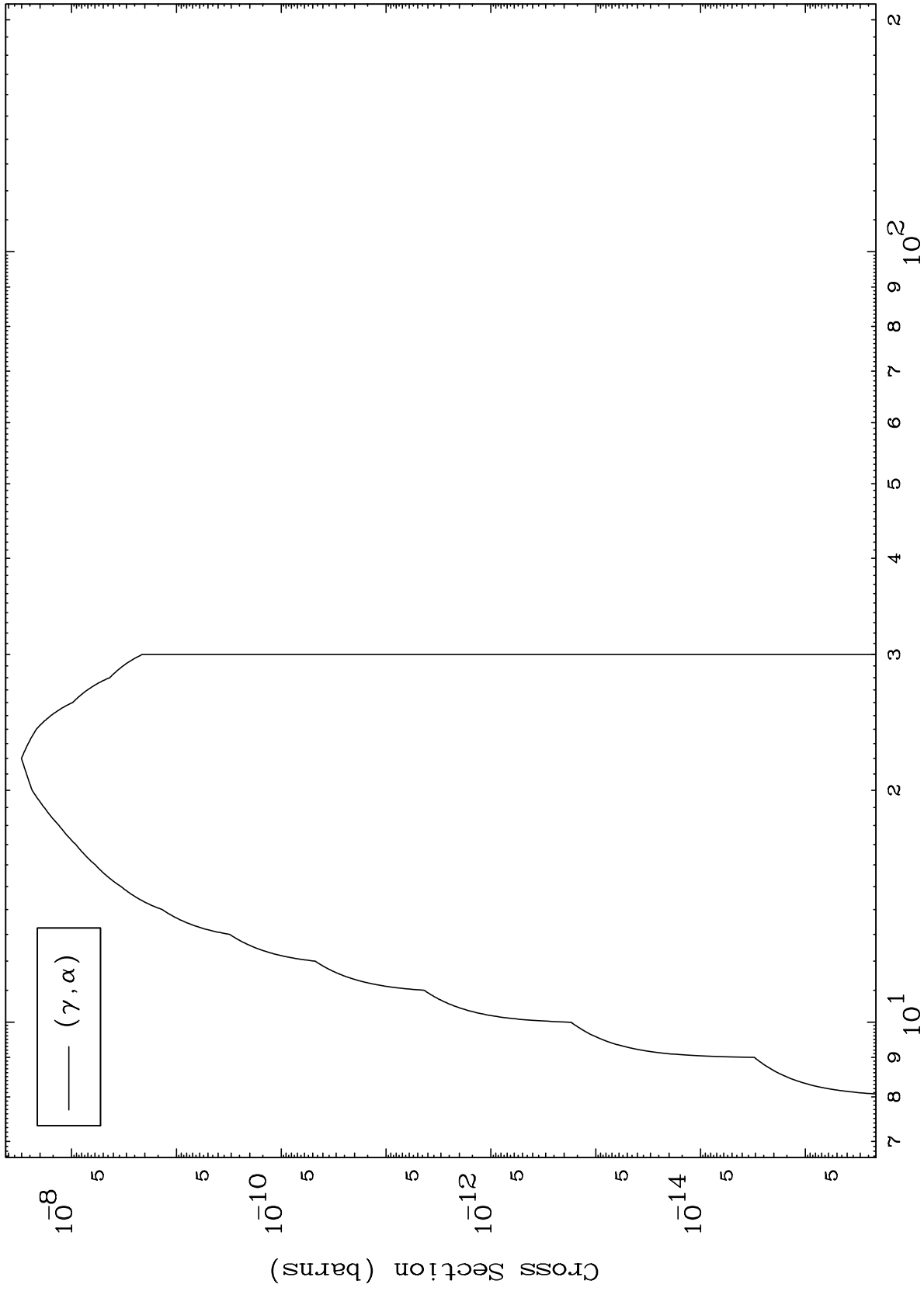
77-Ir-196



MAT 7740

( $\gamma, \alpha$ ) Levels  
0 Kelvin Cross Sections

77-Ir-196



8

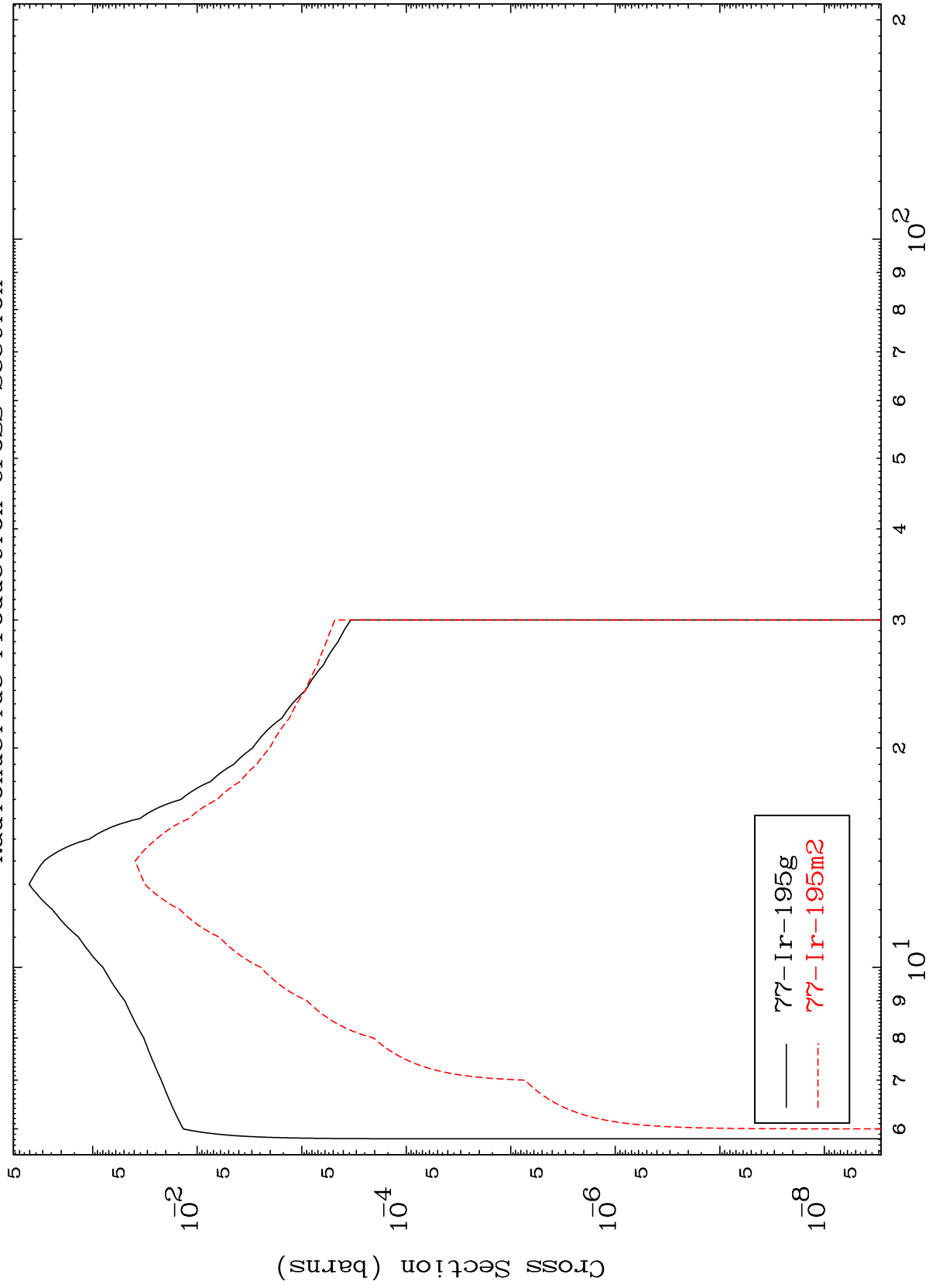
Incident Energy (MeV)

77-Ir-196

MAT 7740

Photon Inelastic  
Radionuclide Production Cross Section

77-Ir-196



9

Incident Energy (MeV)

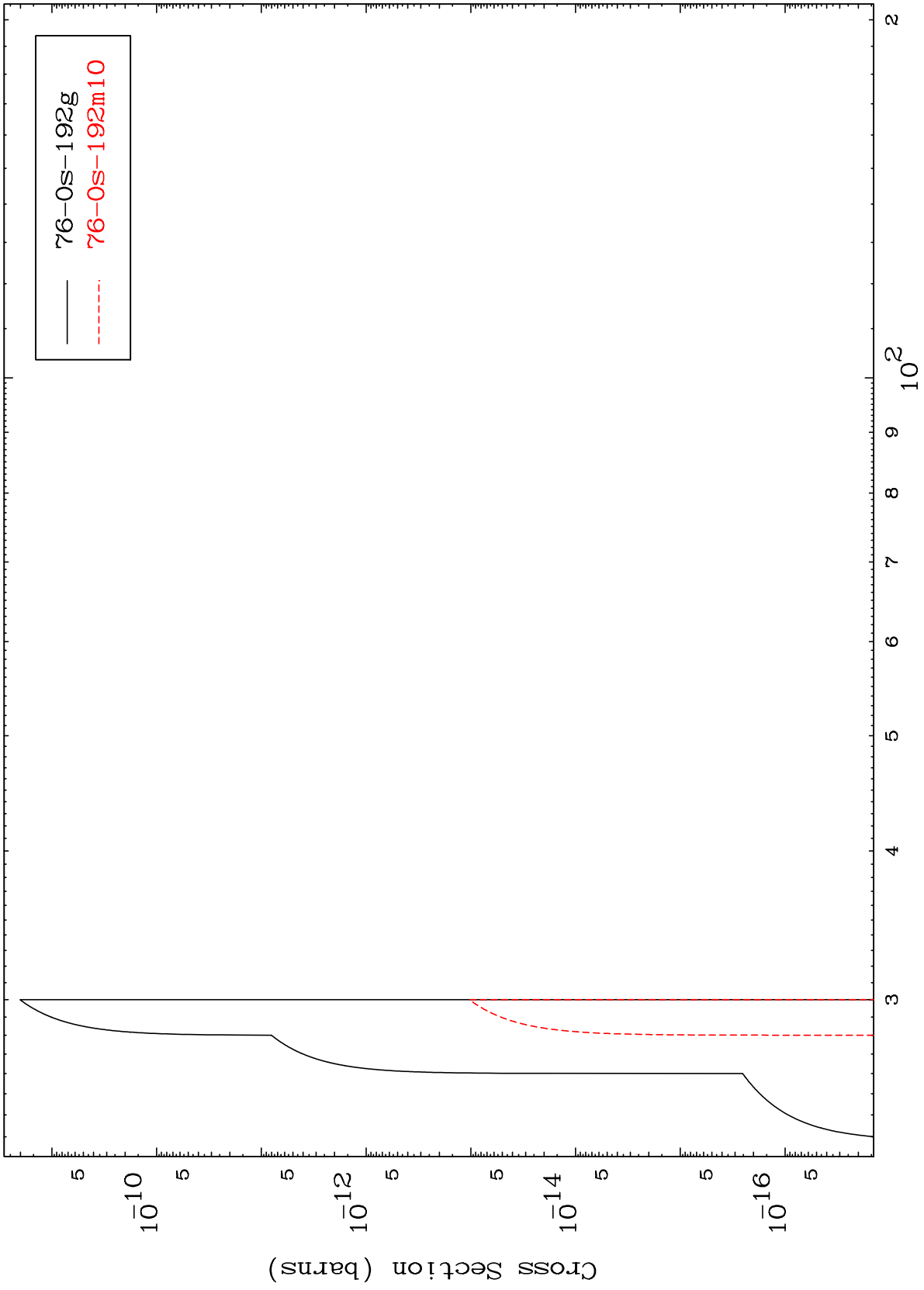
77-Ir-196

MAT 7740

( $\gamma, 2n$ ) d

77-Ir-196

Radionuclide Production Cross Section



10

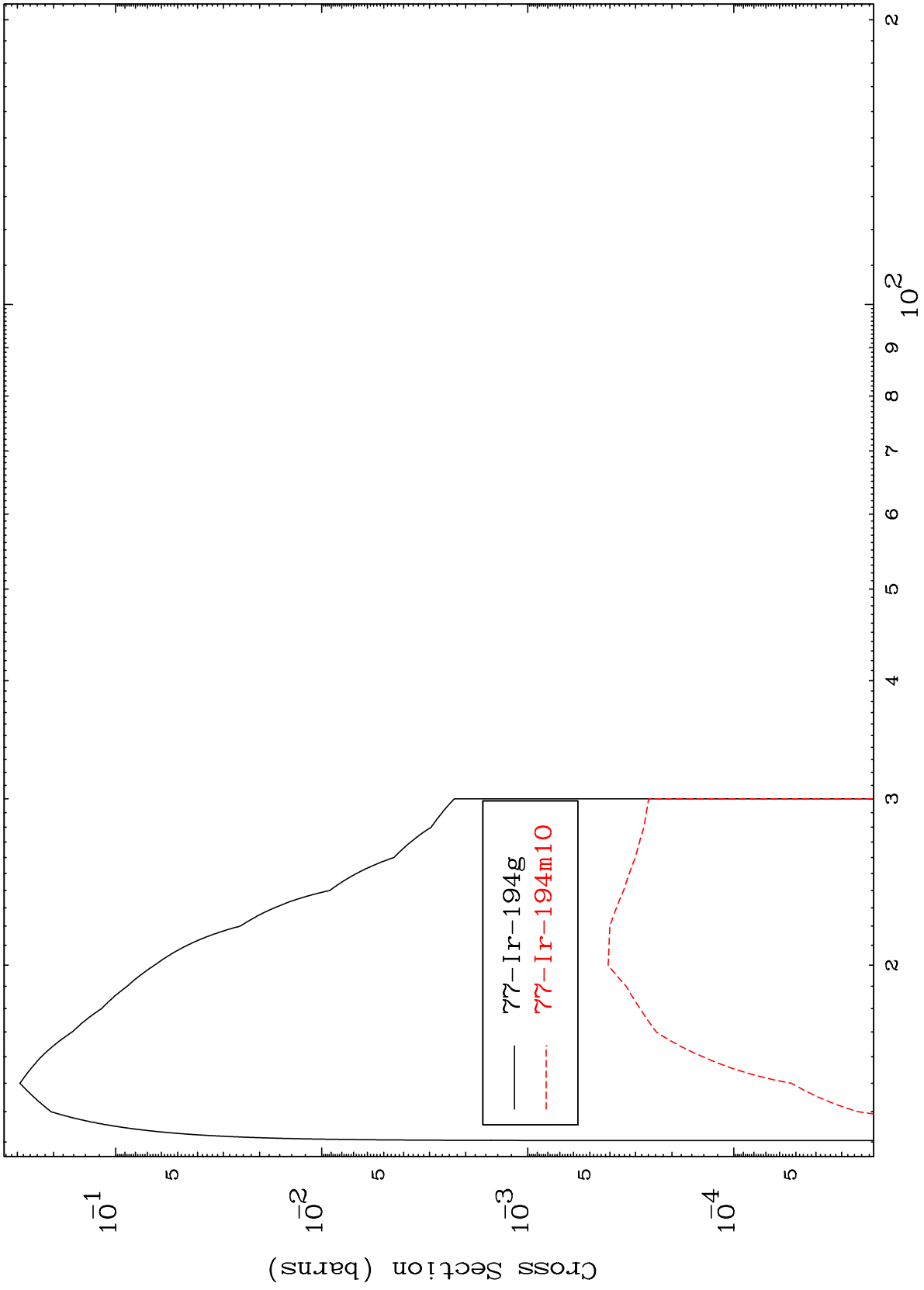
Incident Energy (MeV)

77-Ir-196

MAT 7740

Radionuclide Production Cross Section  
( $\gamma, 2n$ )

<sup>77</sup>Ir-196



11

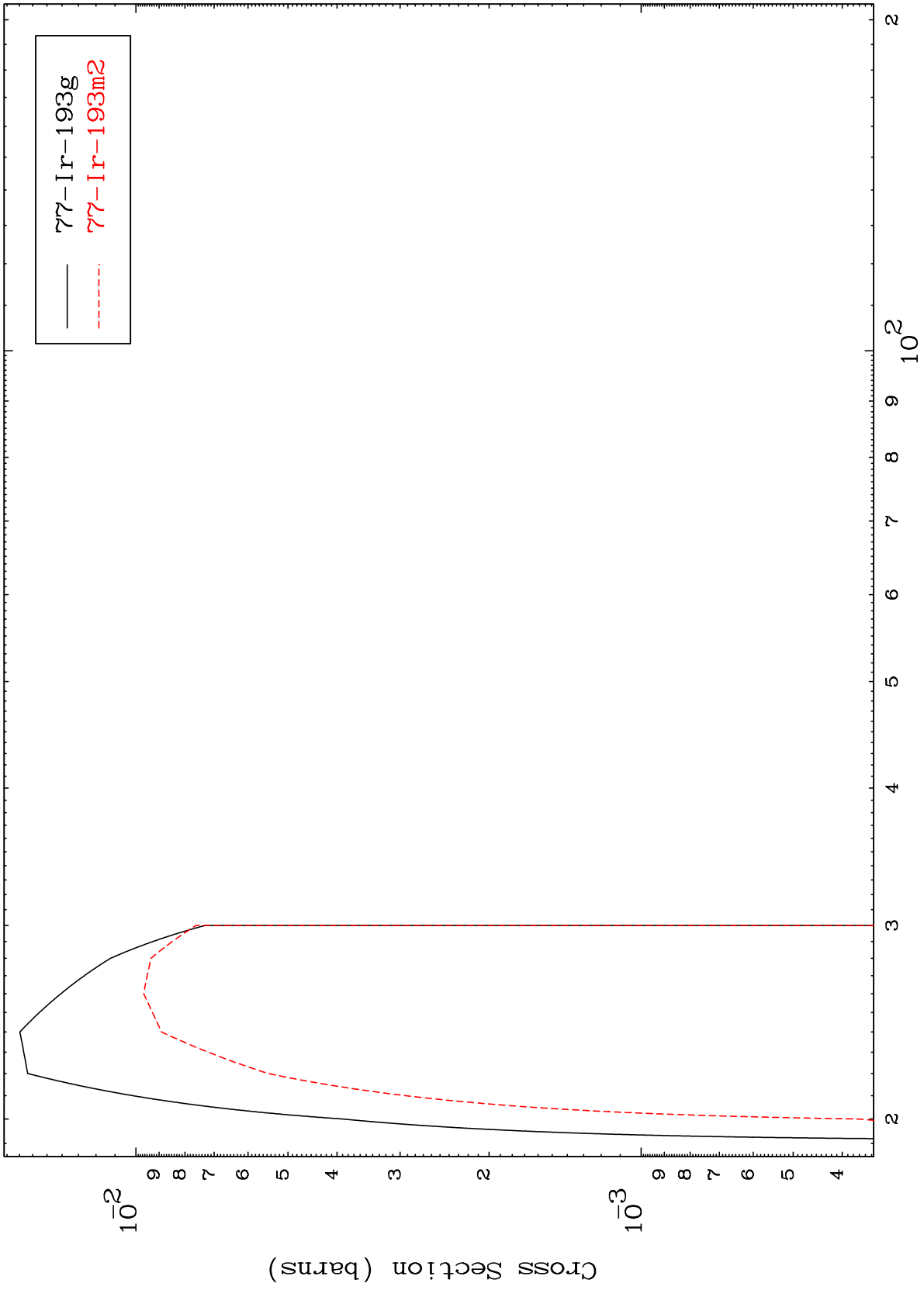
Incident Energy (MeV)

<sup>77</sup>Ir-196

MAT 7740

77-Ir-196

( $\gamma, 3n$ )  
Radionuclide Production Cross Section



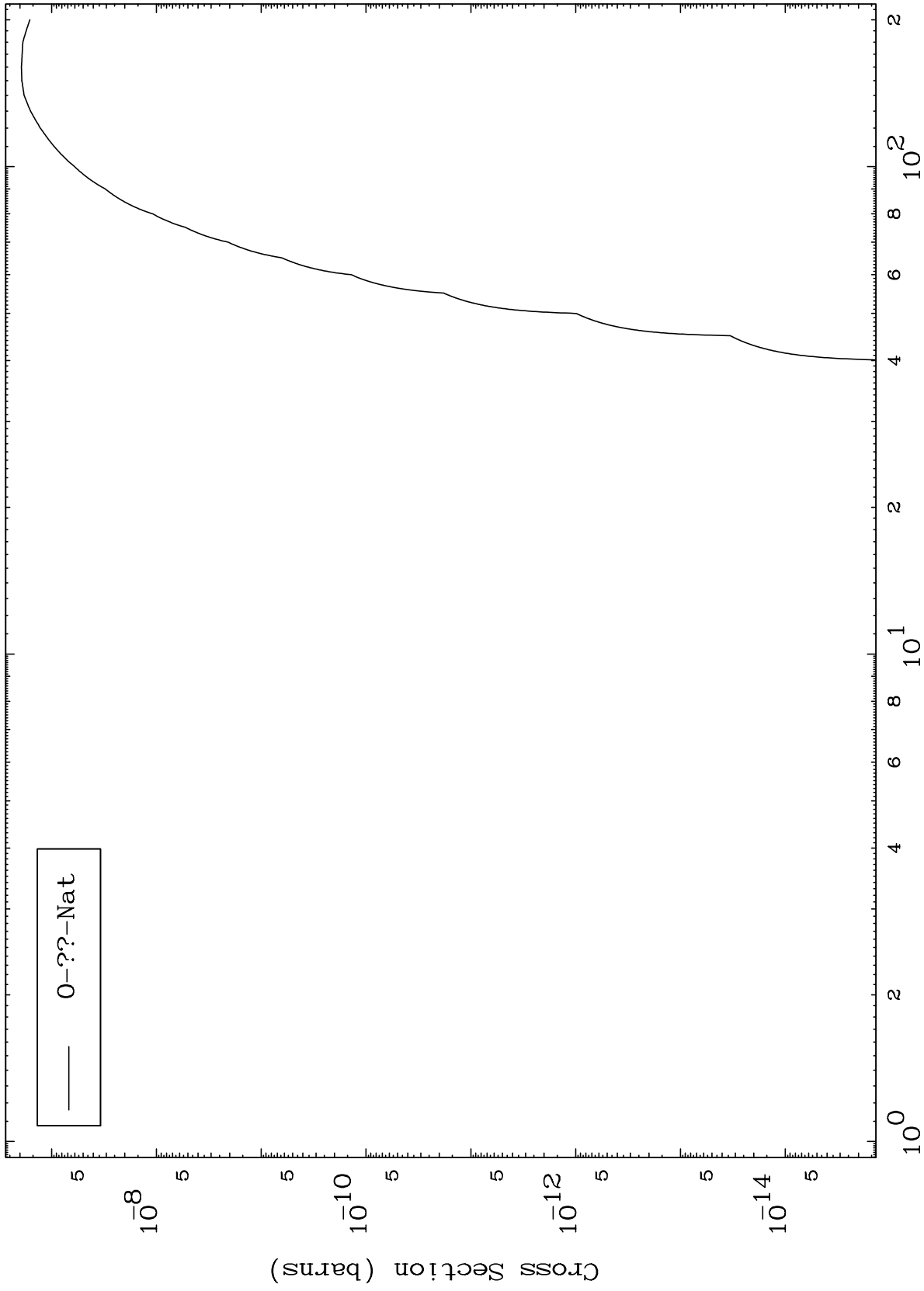
12

77-Ir-196

MAT 7740

Photon Fission  
Radionuclide Production Cross Section

77-Ir-196



13

Incident Energy (MeV)

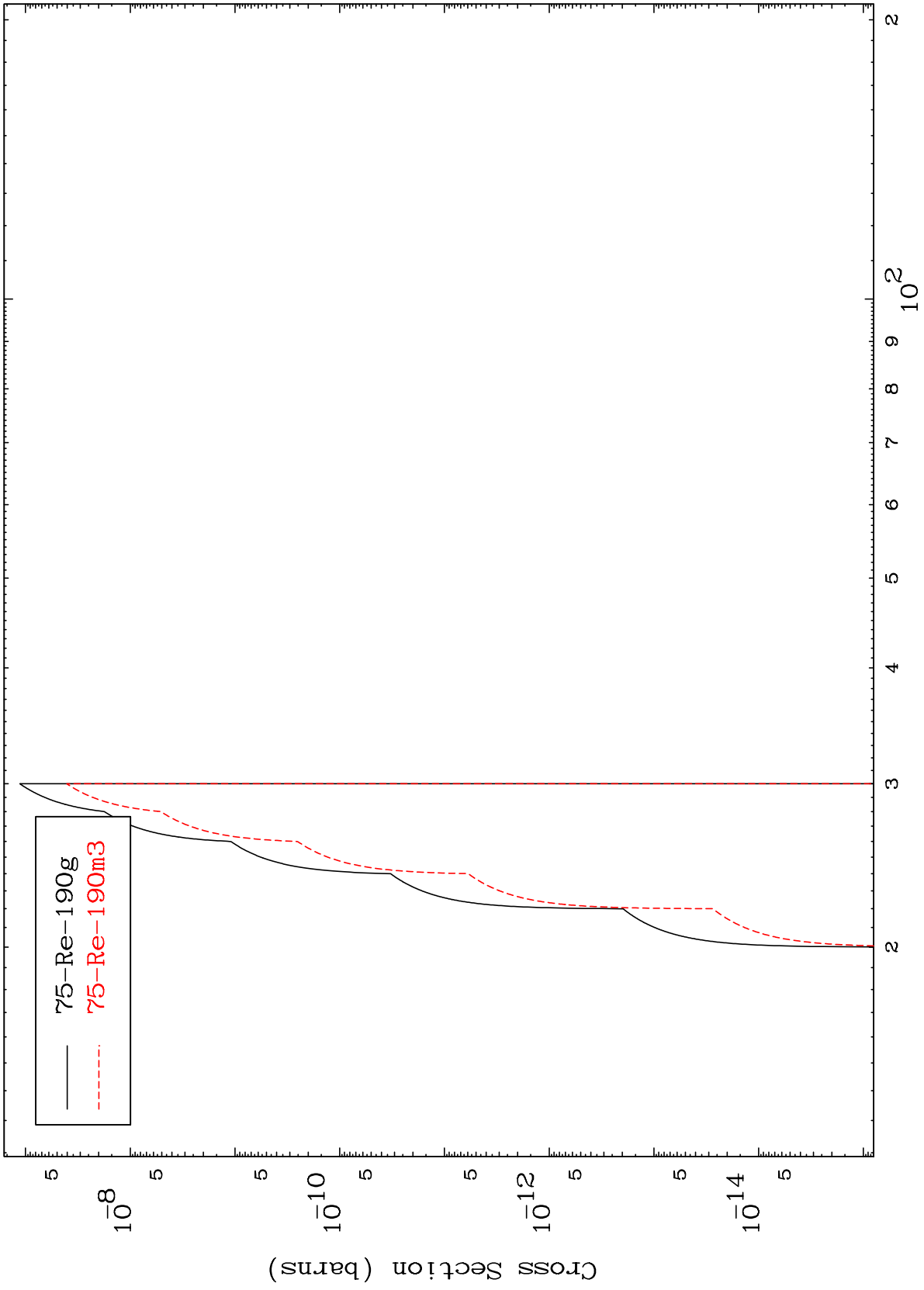
77-Ir-196

MAT 7740

$(\gamma, 2n) \alpha$

77-Ir-196

Radionuclide Production Cross Section



14

Incident Energy (MeV)

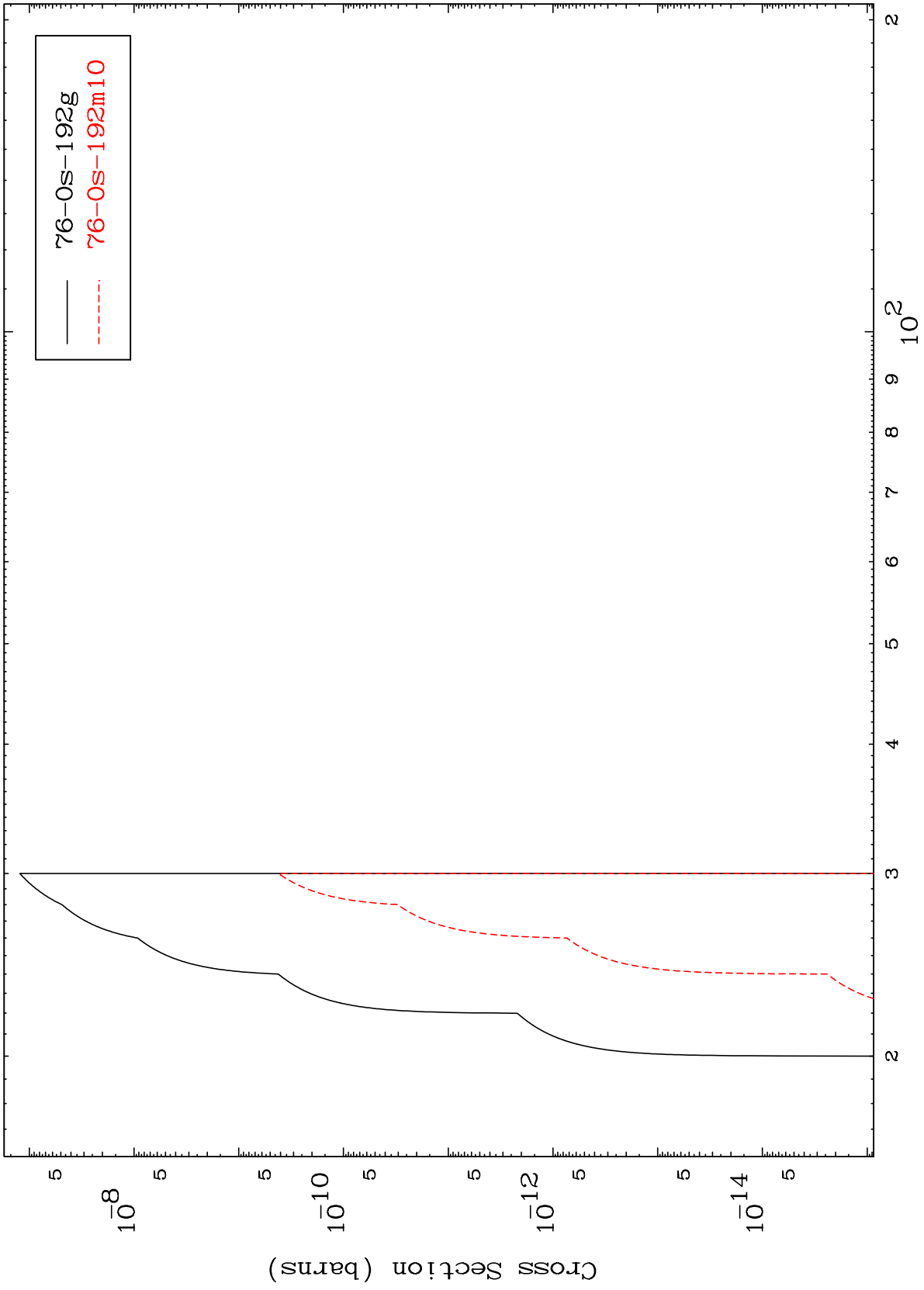
77-Ir-196

MAT 7740

( $\gamma, n'$ ) t

77-Ir-196

Radionuclide Production Cross Section



15

Incident Energy (MeV)

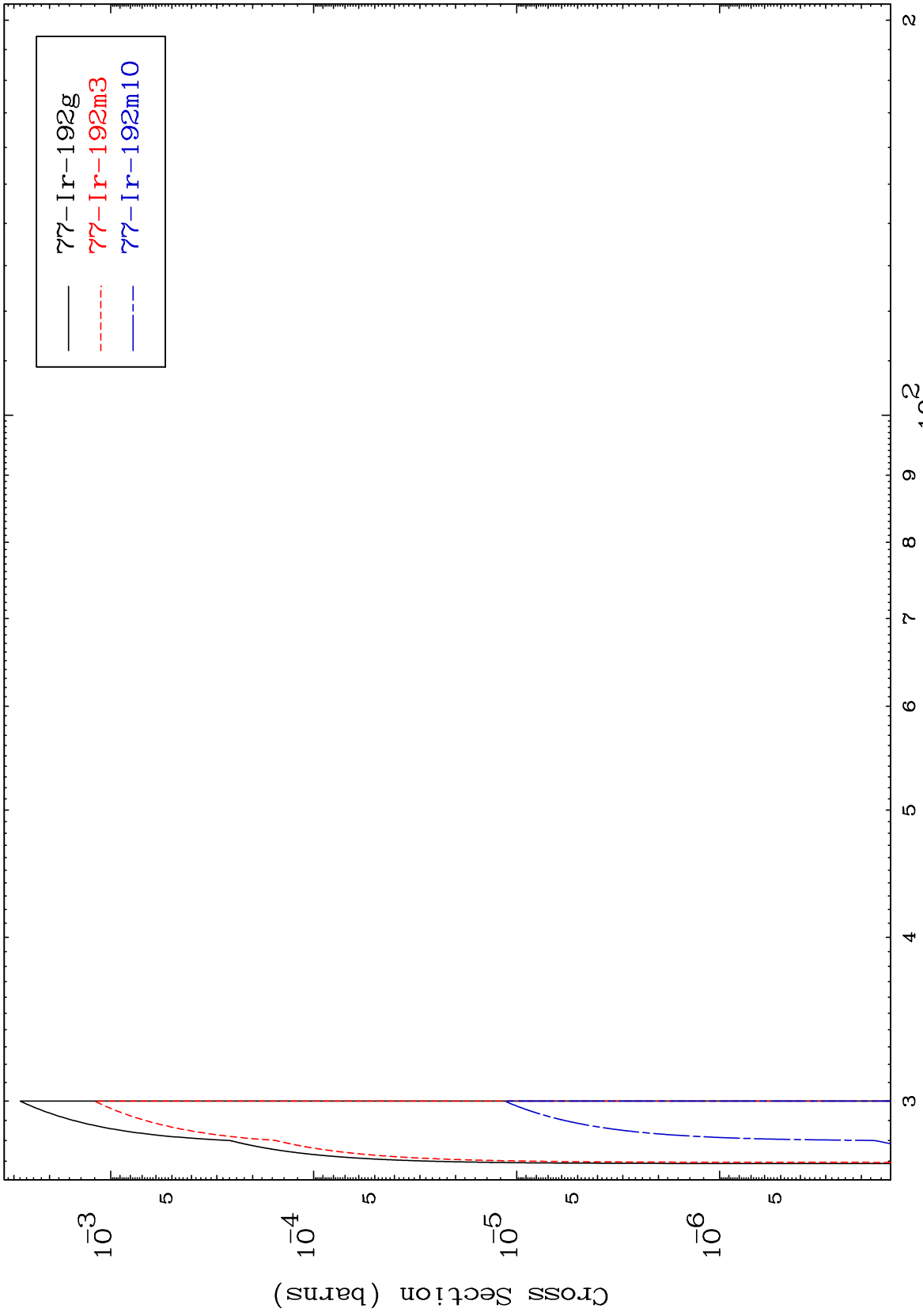
77-Ir-196



MAT 7740

77-Ir-196

( $\gamma, 4n$ )  
Radionuclide Production Cross Section



16

Incident Energy (MeV)

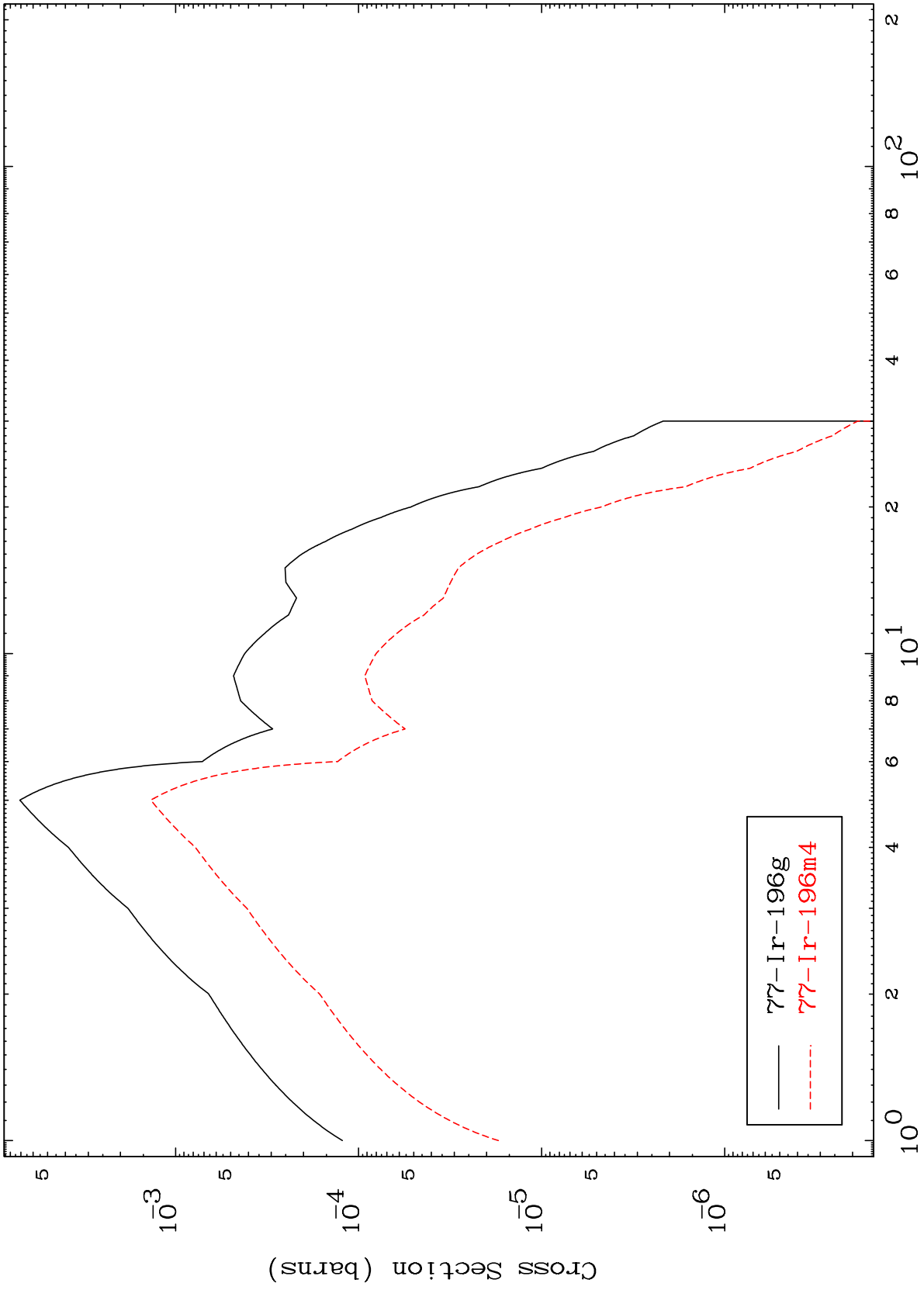
77-Ir-196

MAT 7740

77-Ir-196

Radionuclide Production Cross Section

( $\gamma, \gamma$ )



Incident Energy (MeV)

77-Ir-196

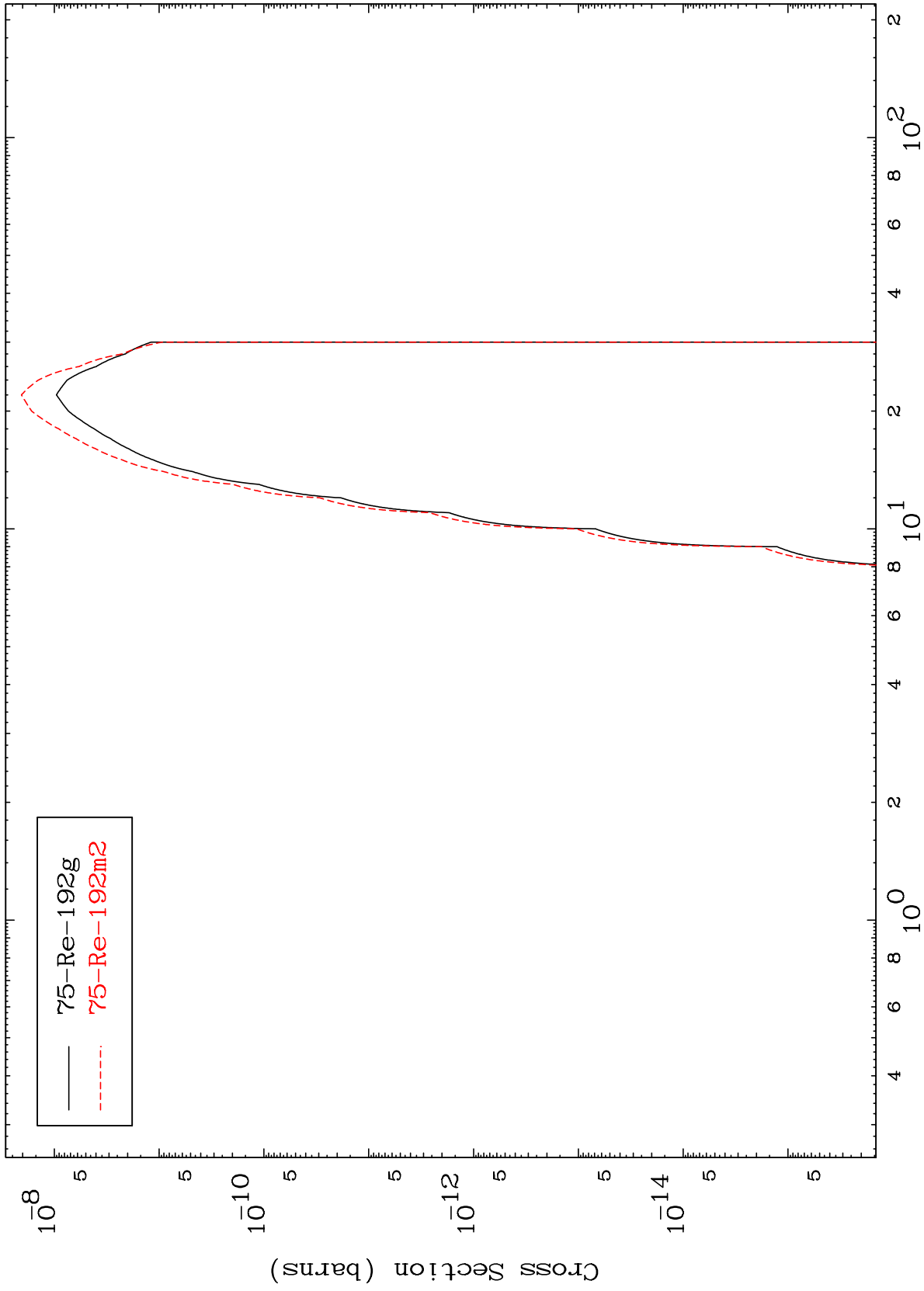
— 77-Ir-196g  
- - - 77-Ir-196m4

MAT 7740

( $\gamma, \alpha$ )

77-Ir-196

Radionuclide Production Cross Section



18

Incident Energy (MeV)

77-Ir-196