

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

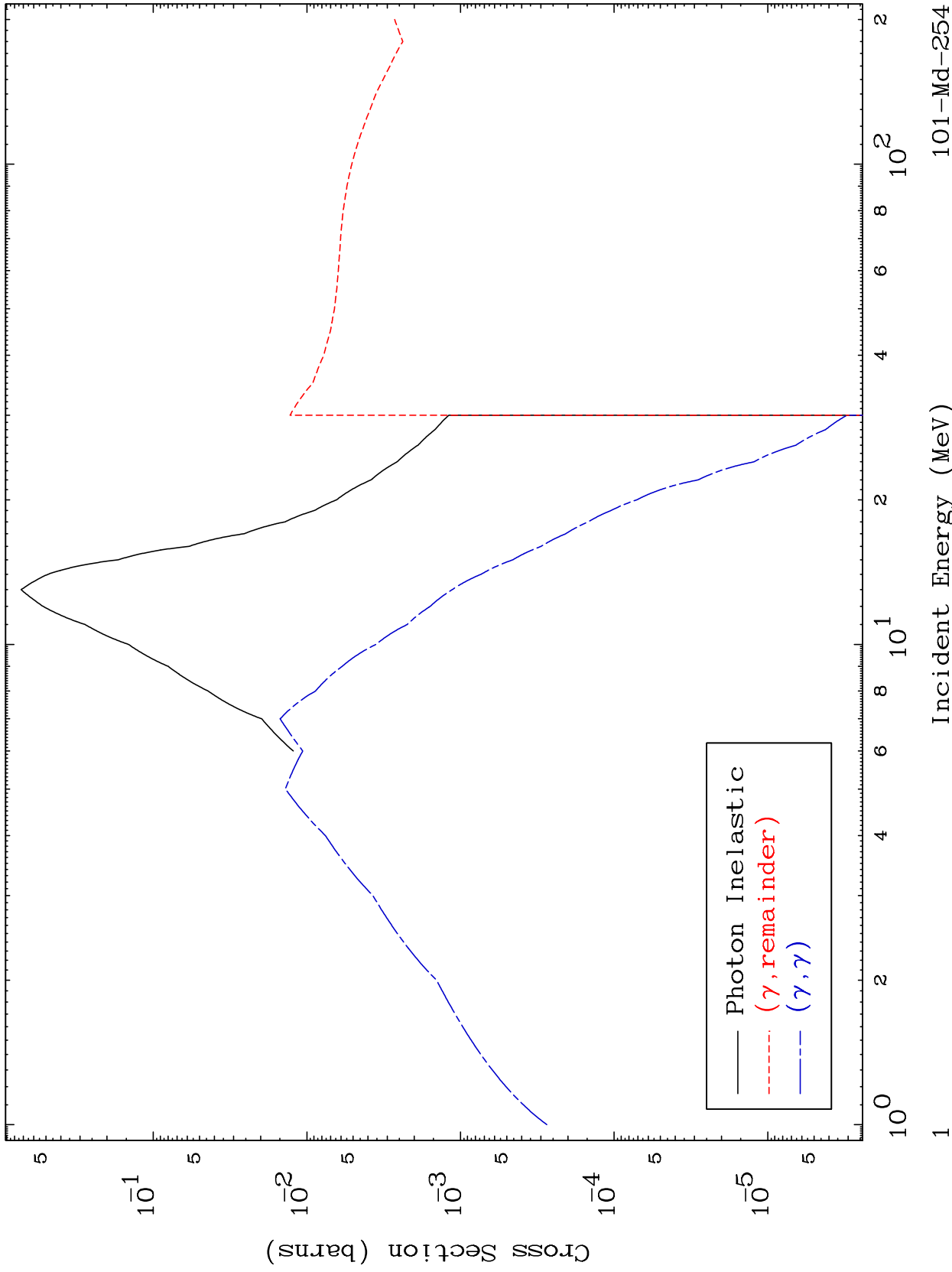
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9955

Photon Major  
0 Kelvin Cross Sections

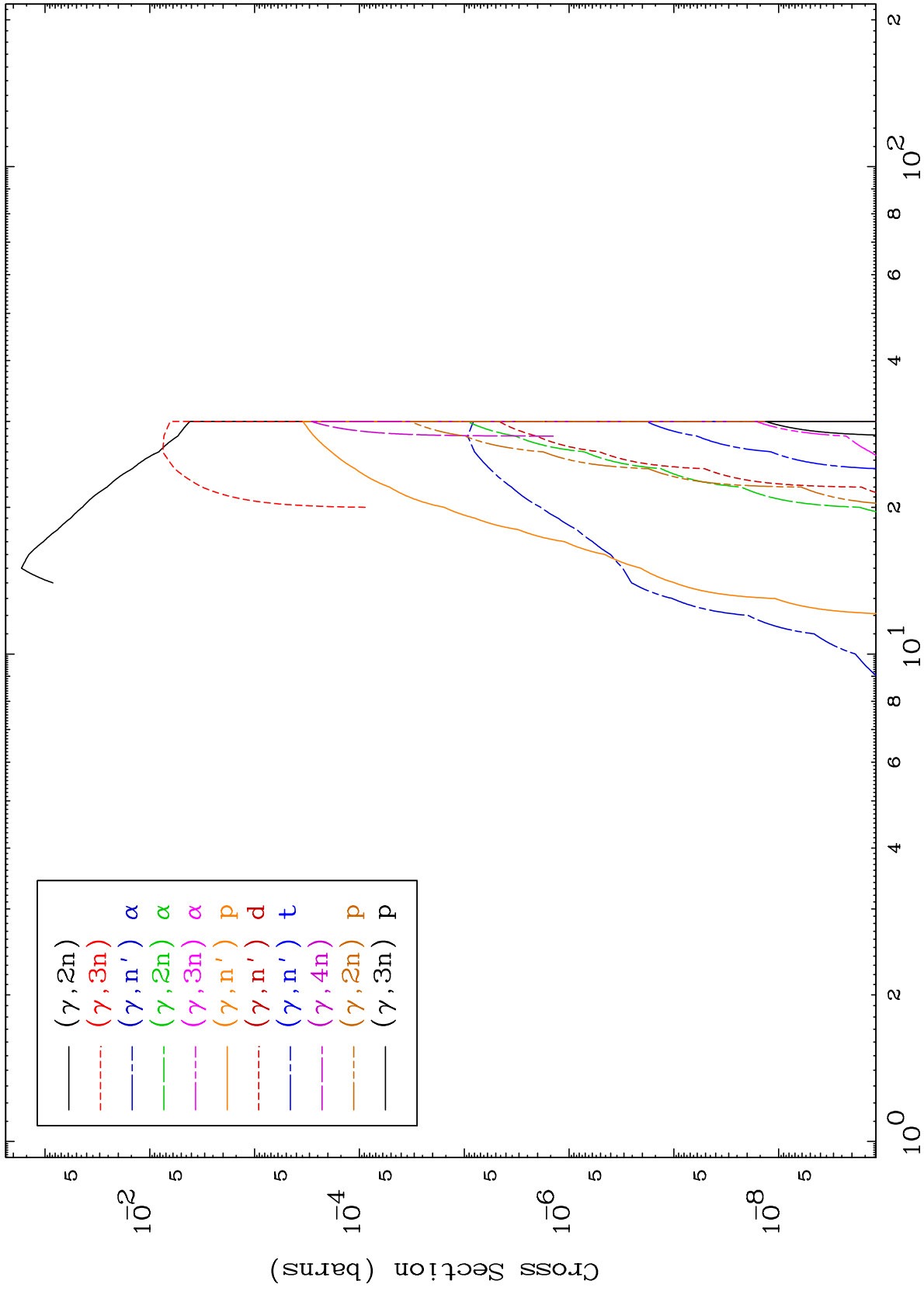
101-Md-254



MAT 9955

Photon Neutron Production  
0 Kelvin Cross Sections

101-Md-254



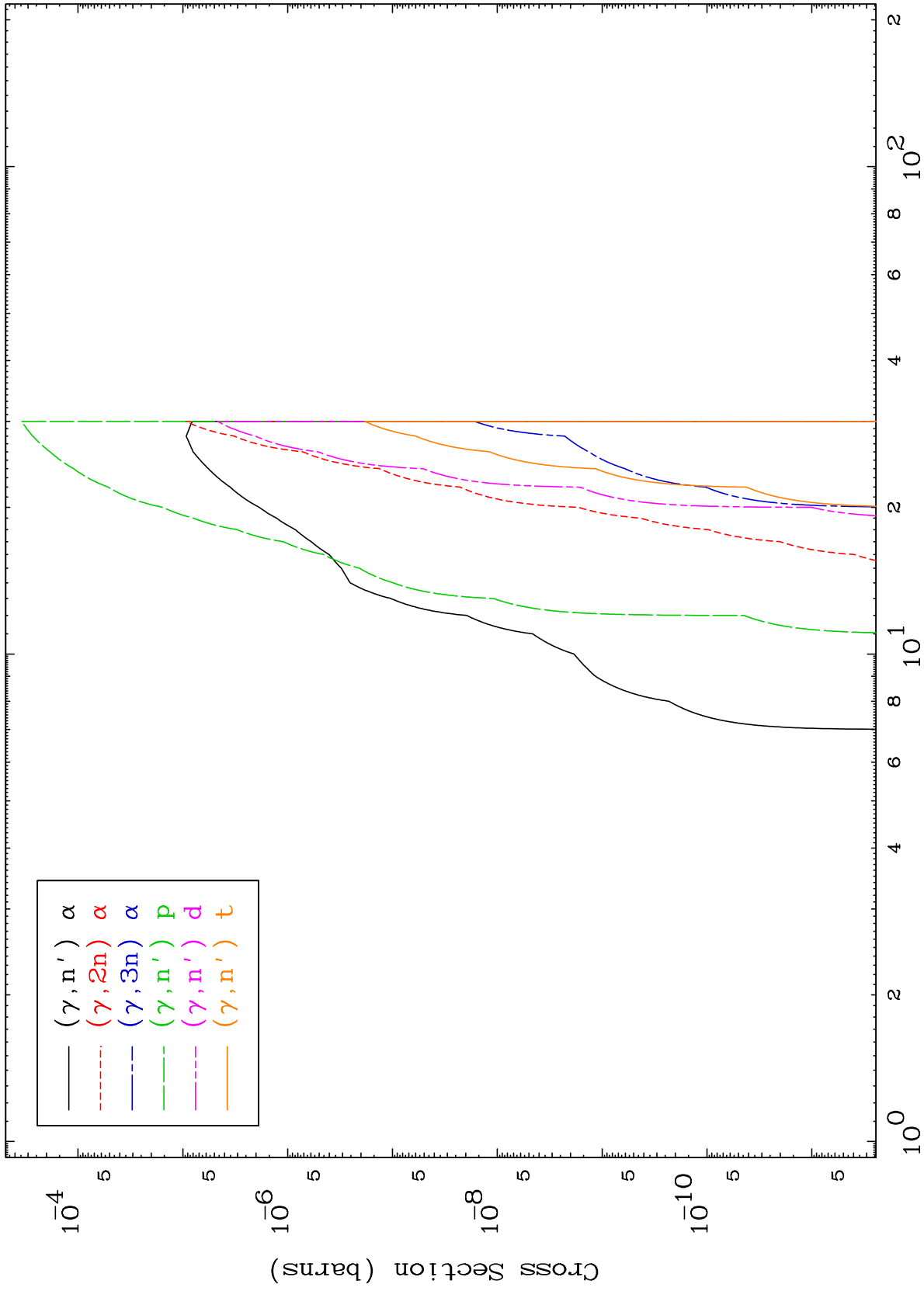
Incident Energy (MeV)

101-Md-254

MAT 9955

Photon Charged Particle  
0 Kelvin Cross Sections

101-Md-254



MAT 9955

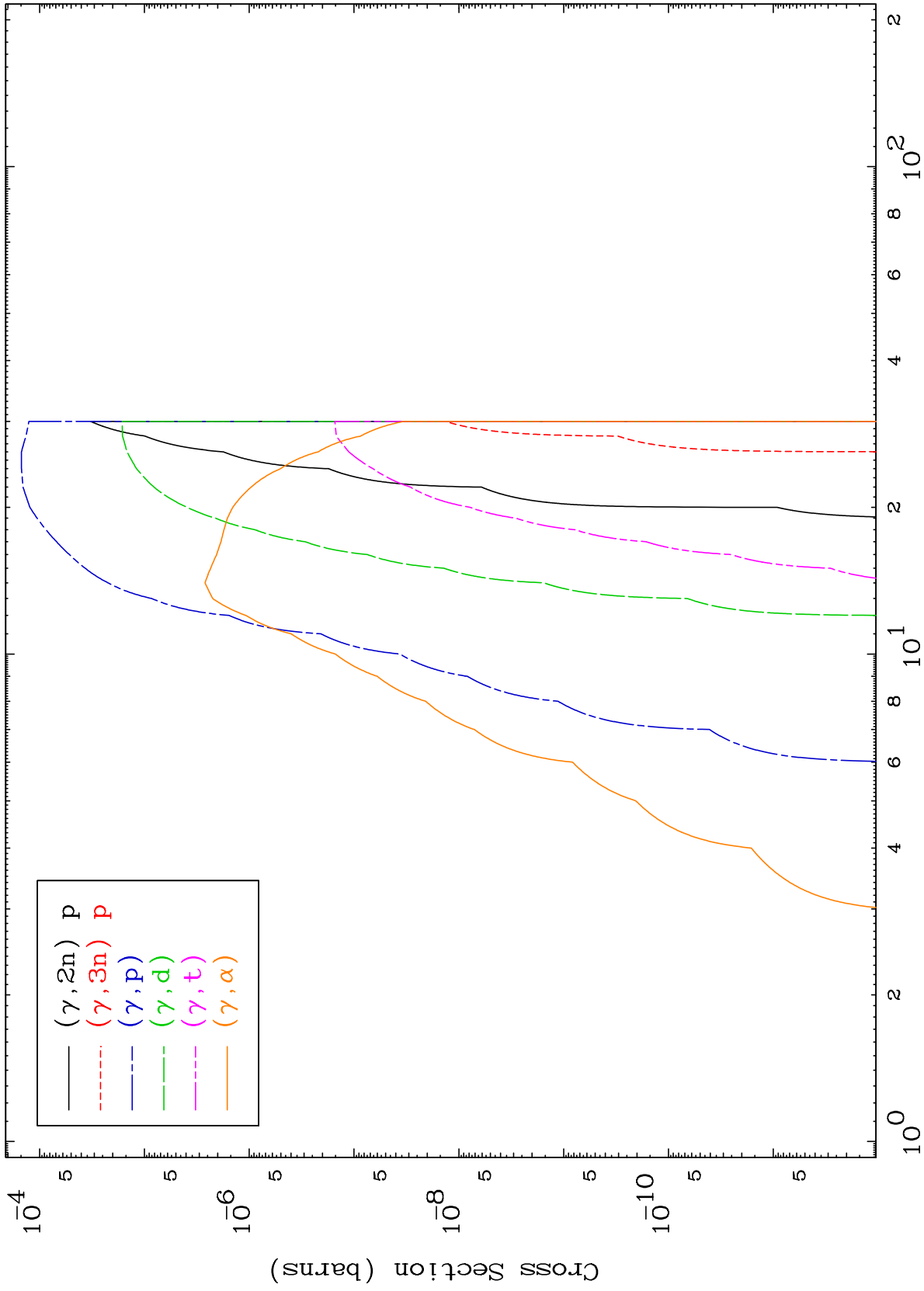
Photon Charged Particle  
0 Kelvin Cross Sections

101-Md-254

MAT 9955

Photon Charged Particle  
0 Kelvin Cross Sections

101-Md-254



101-Md-254

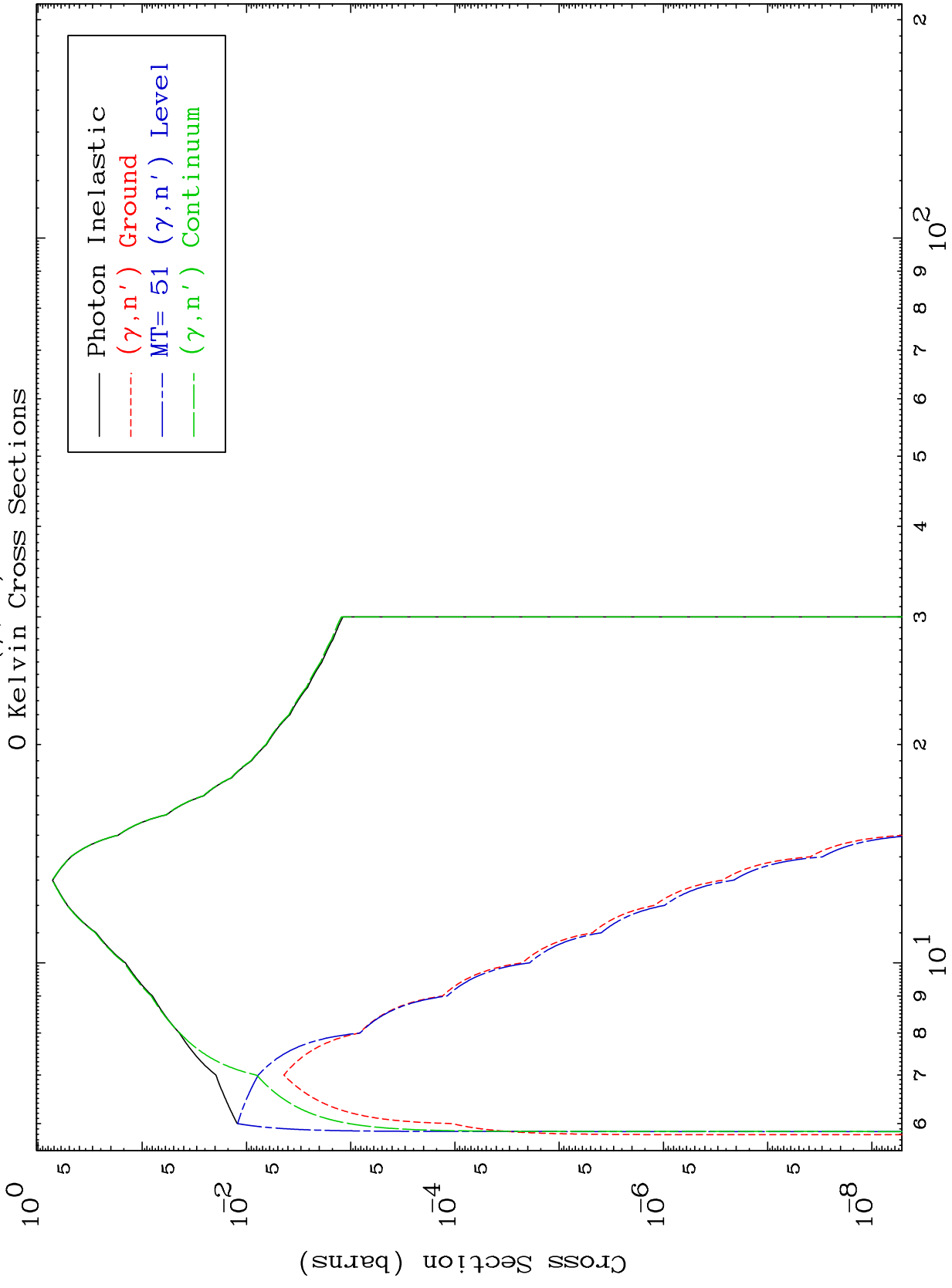
Incident Energy (MeV)

4

MAT 9955

$(\gamma, n')$  Level

101-Md-254



5

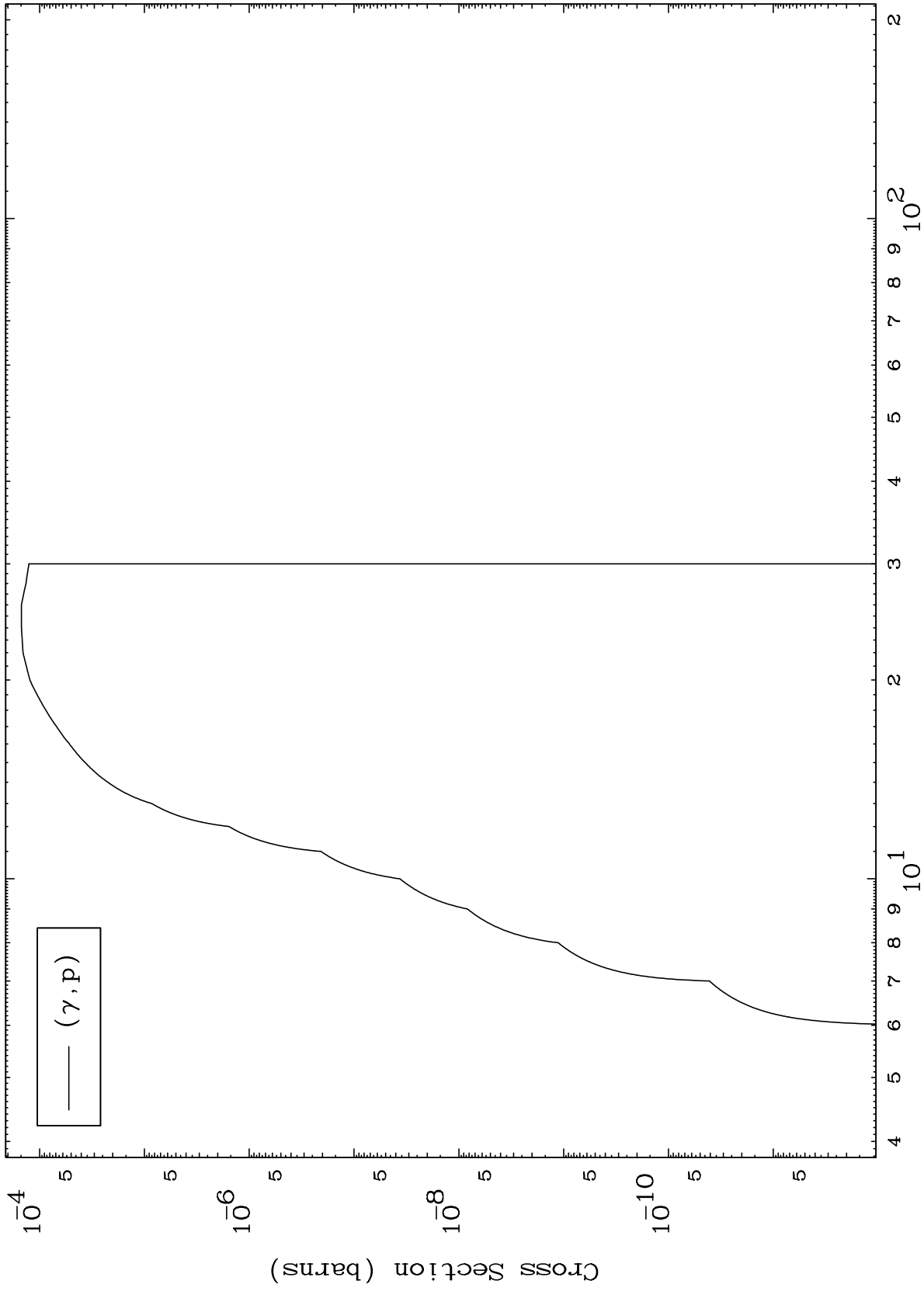
Incident Energy (MeV)

101-Md-254

MAT 9955

( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections

101-Md-254



6

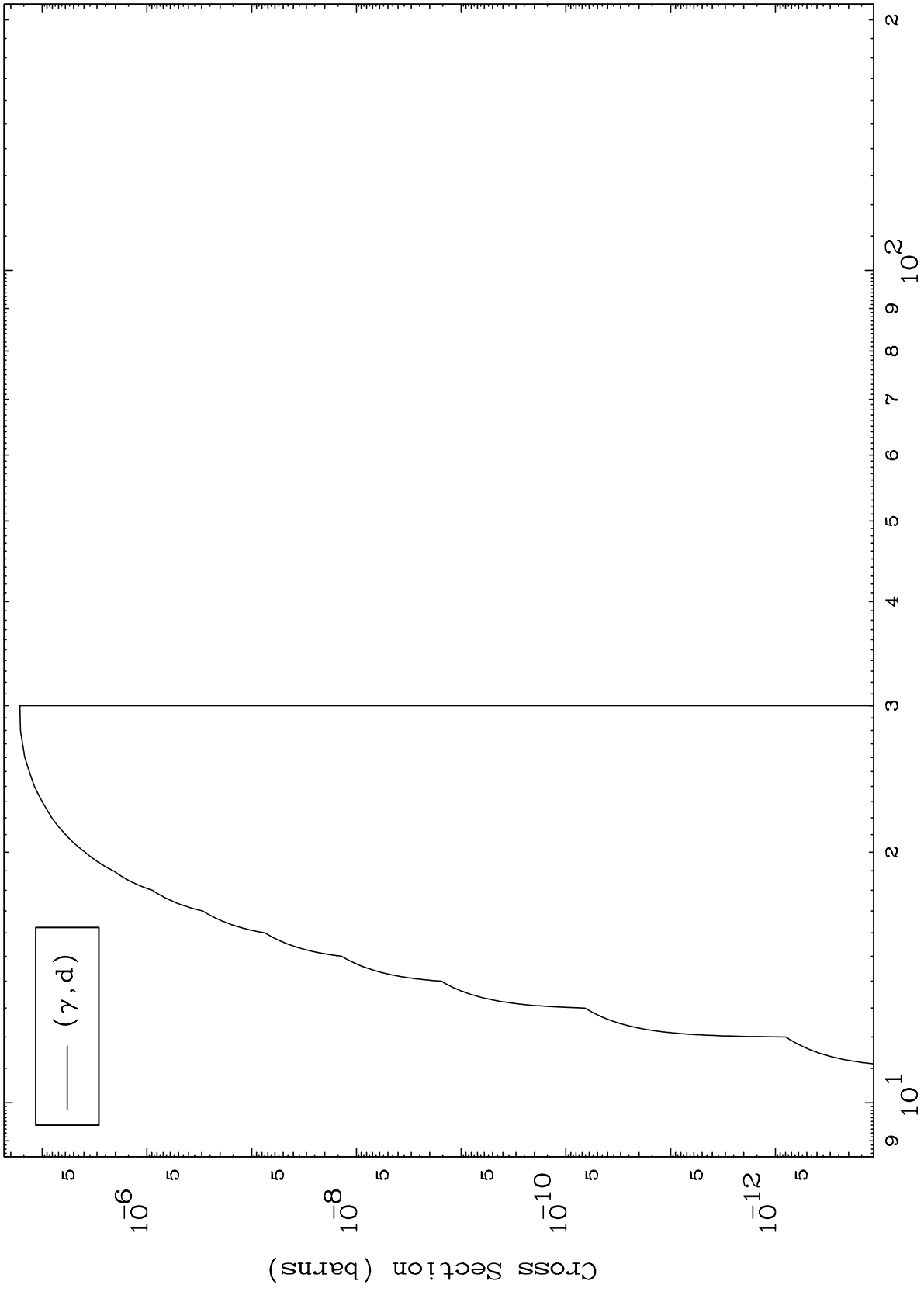
Incident Energy (MeV)

101-Md-254

MAT 9955

( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

101-Md-254



7

Incident Energy (MeV)

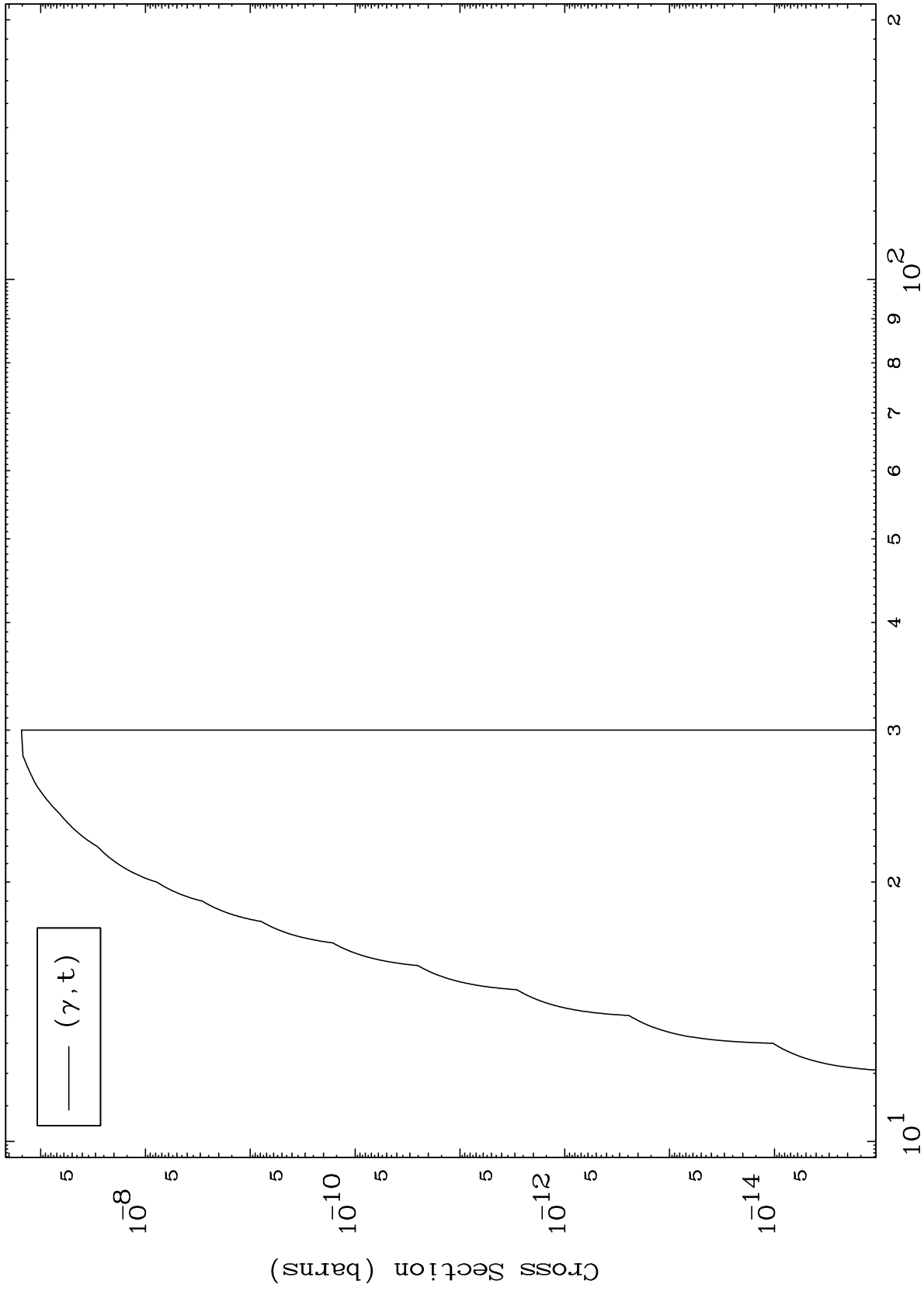
101-Md-254



MAT 9955

( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

101-Md-254



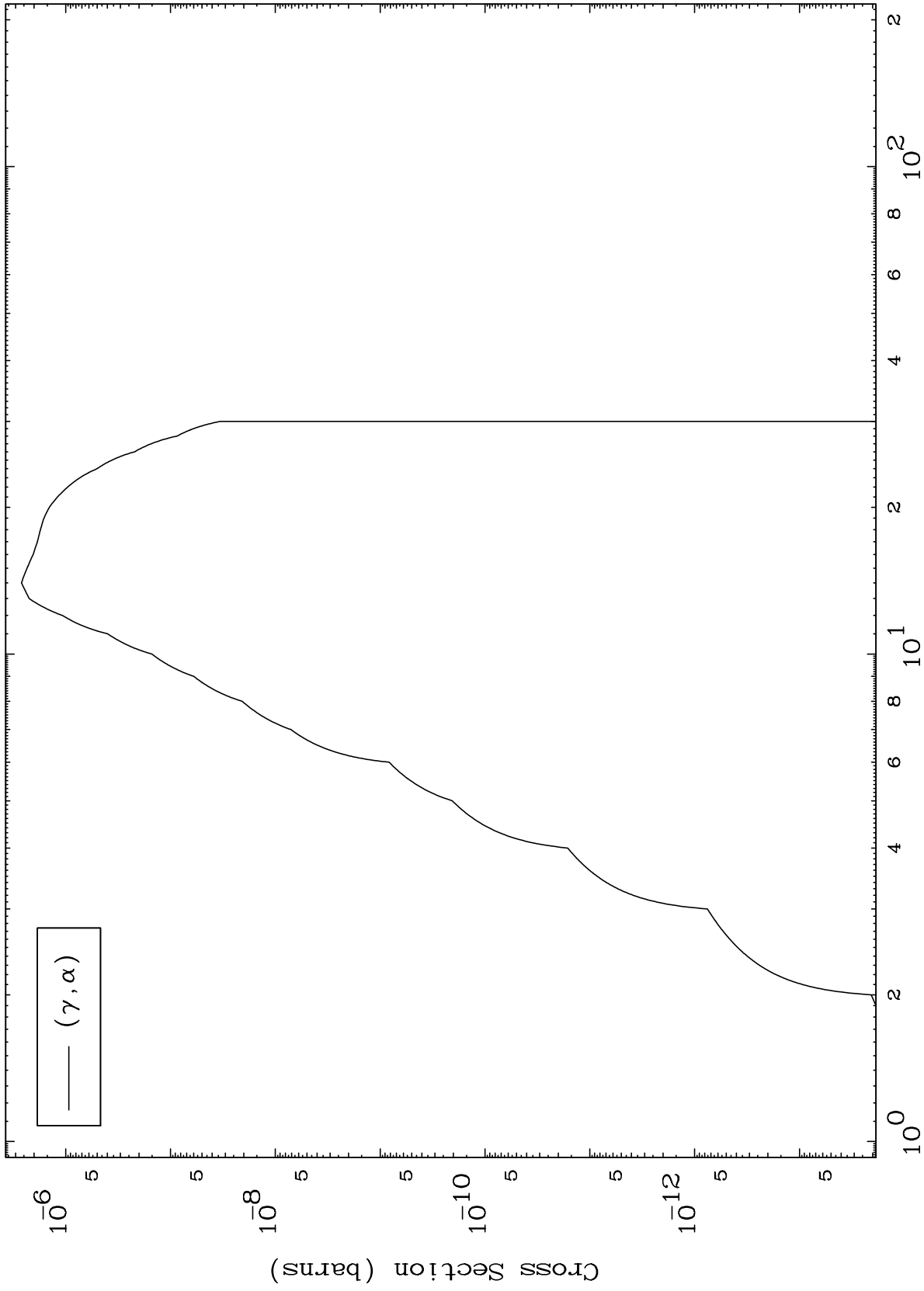
Incident Energy (MeV)

101-Md-254

MAT 9955

( $\gamma, \alpha$ ) Levels  
0 Kelvin Cross Sections

101-Md-254



Incident Energy (MeV)

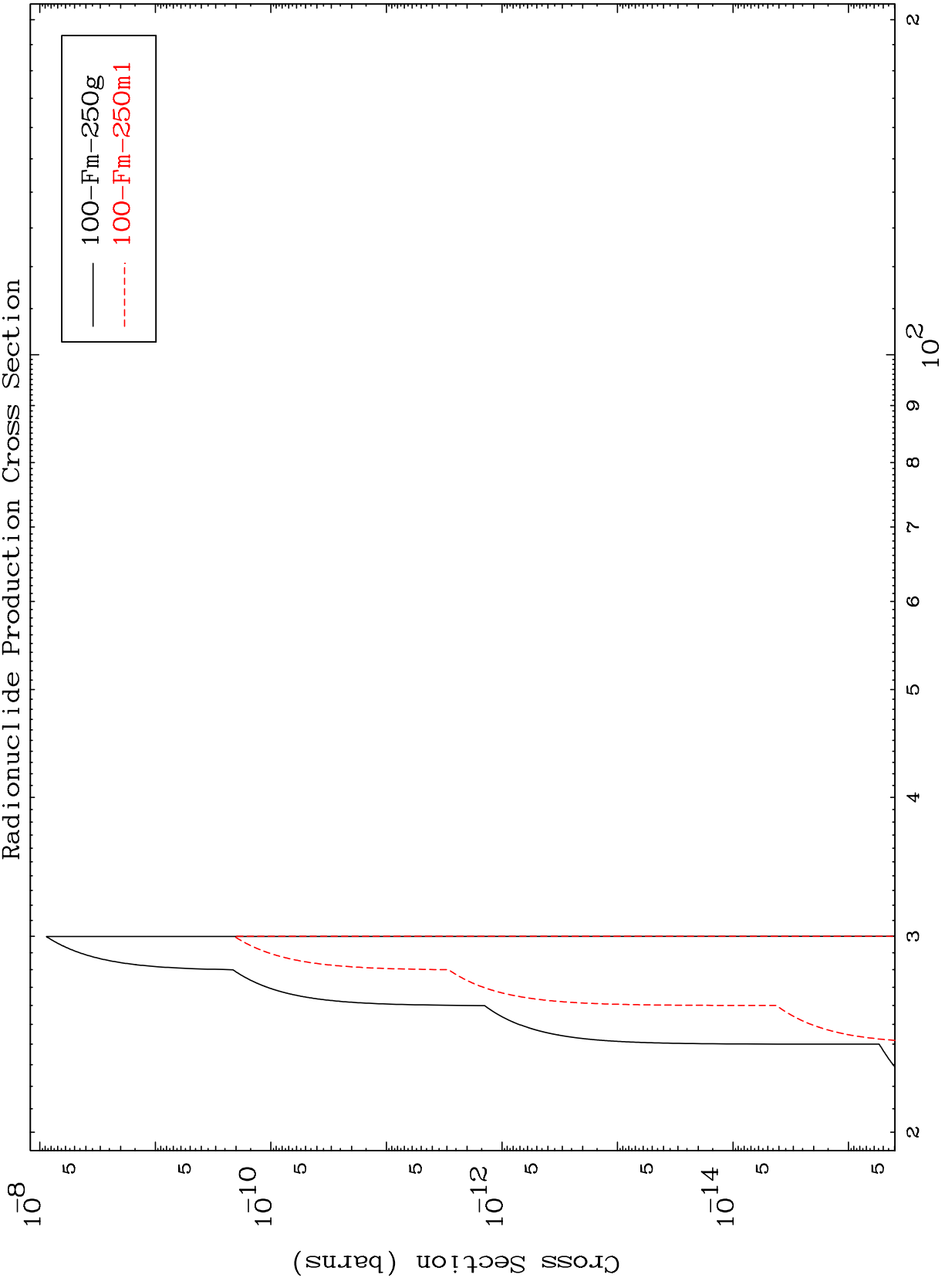
101-Md-254

MAT 9955

( $\gamma, 2n$ ) d

101-Md-254

Radionuclide Production Cross Section



10

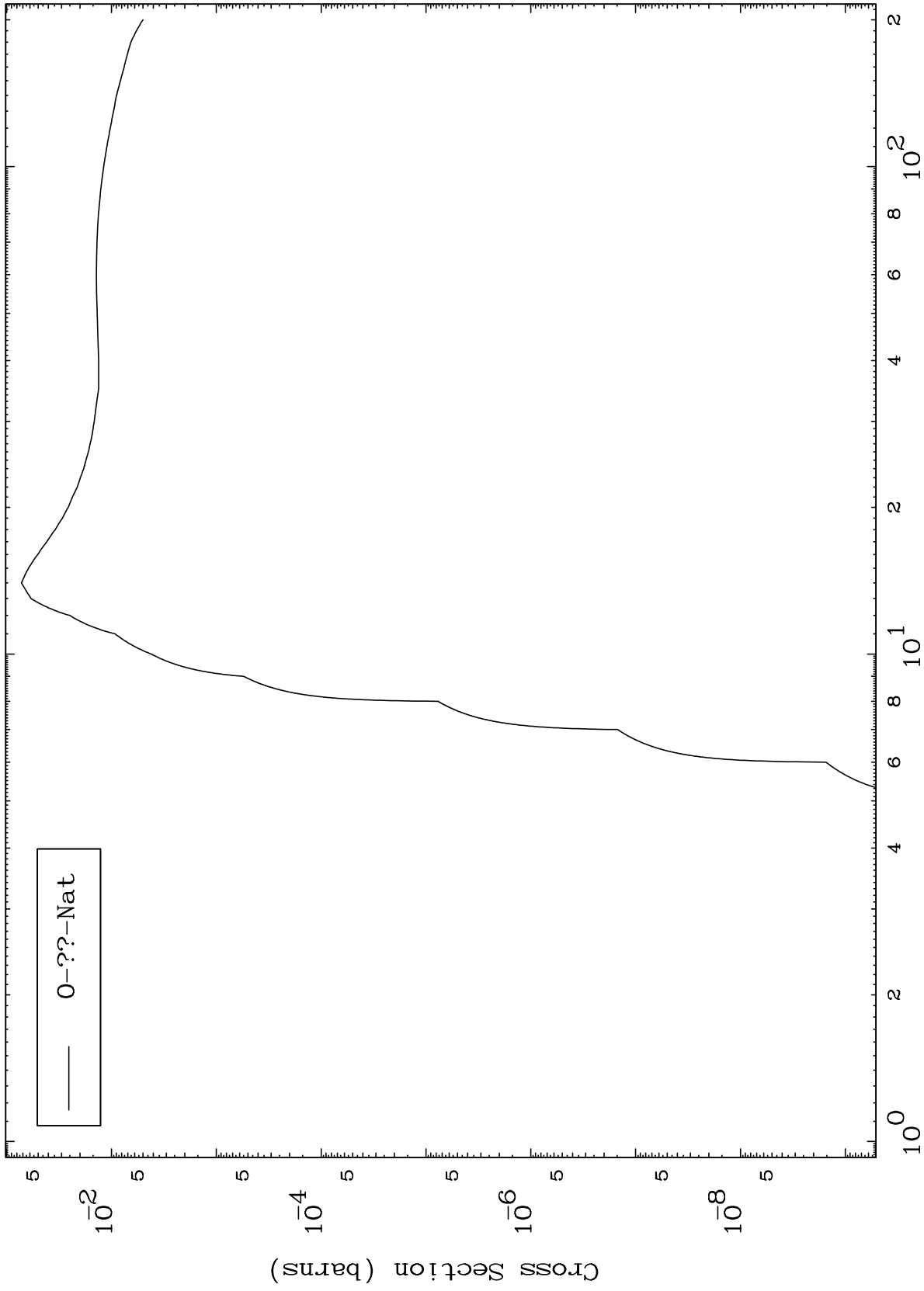
Incident Energy (MeV)

101-Md-254

MAT 9955

Photon Fission  
Radionuclide Production Cross Section

101-Md-254



11

Incident Energy (MeV)

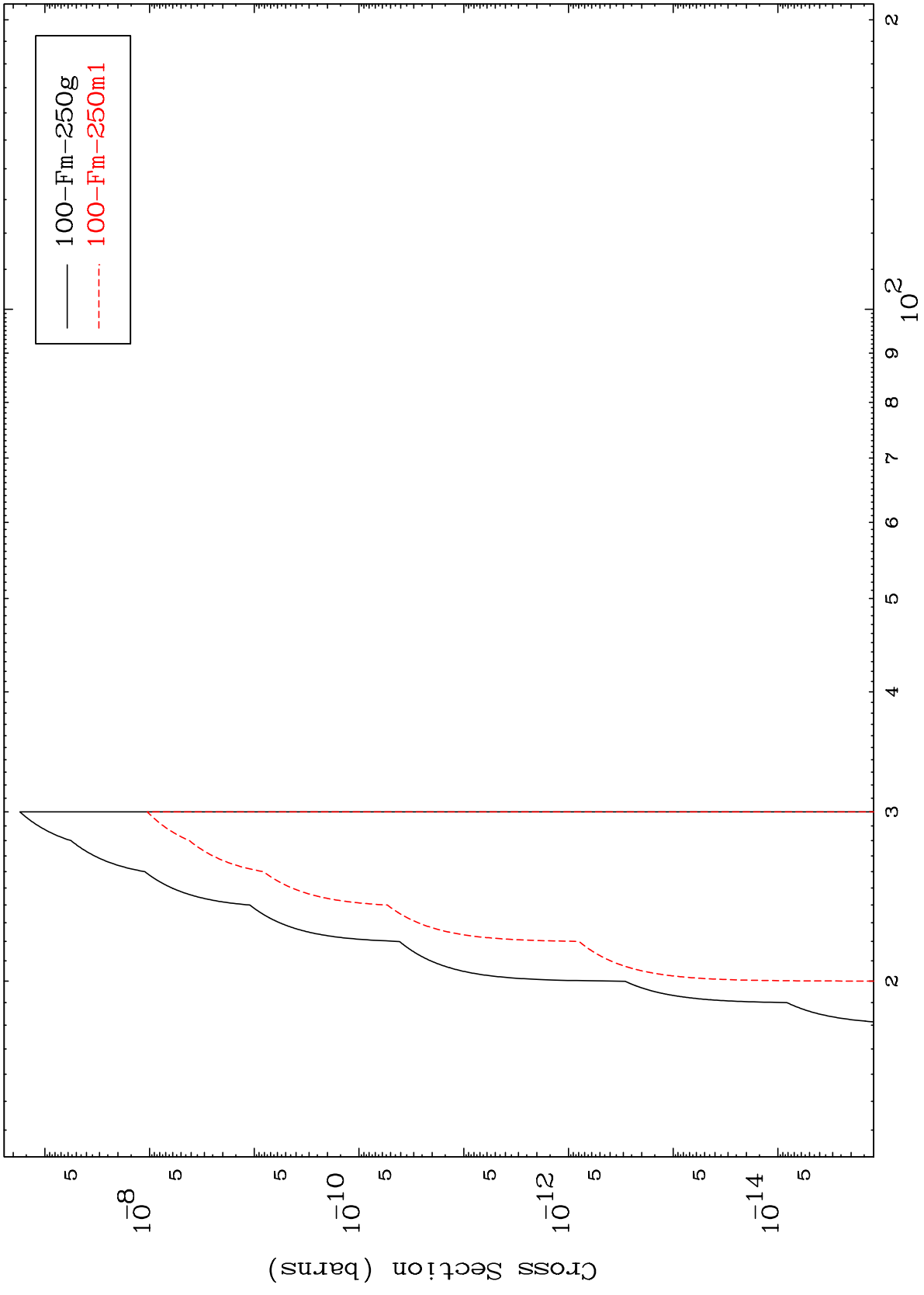
101-Md-254

MAT 9955

( $\gamma, n'$ ) t

101-Md-254

Radionuclide Production Cross Section



12

Incident Energy (MeV)

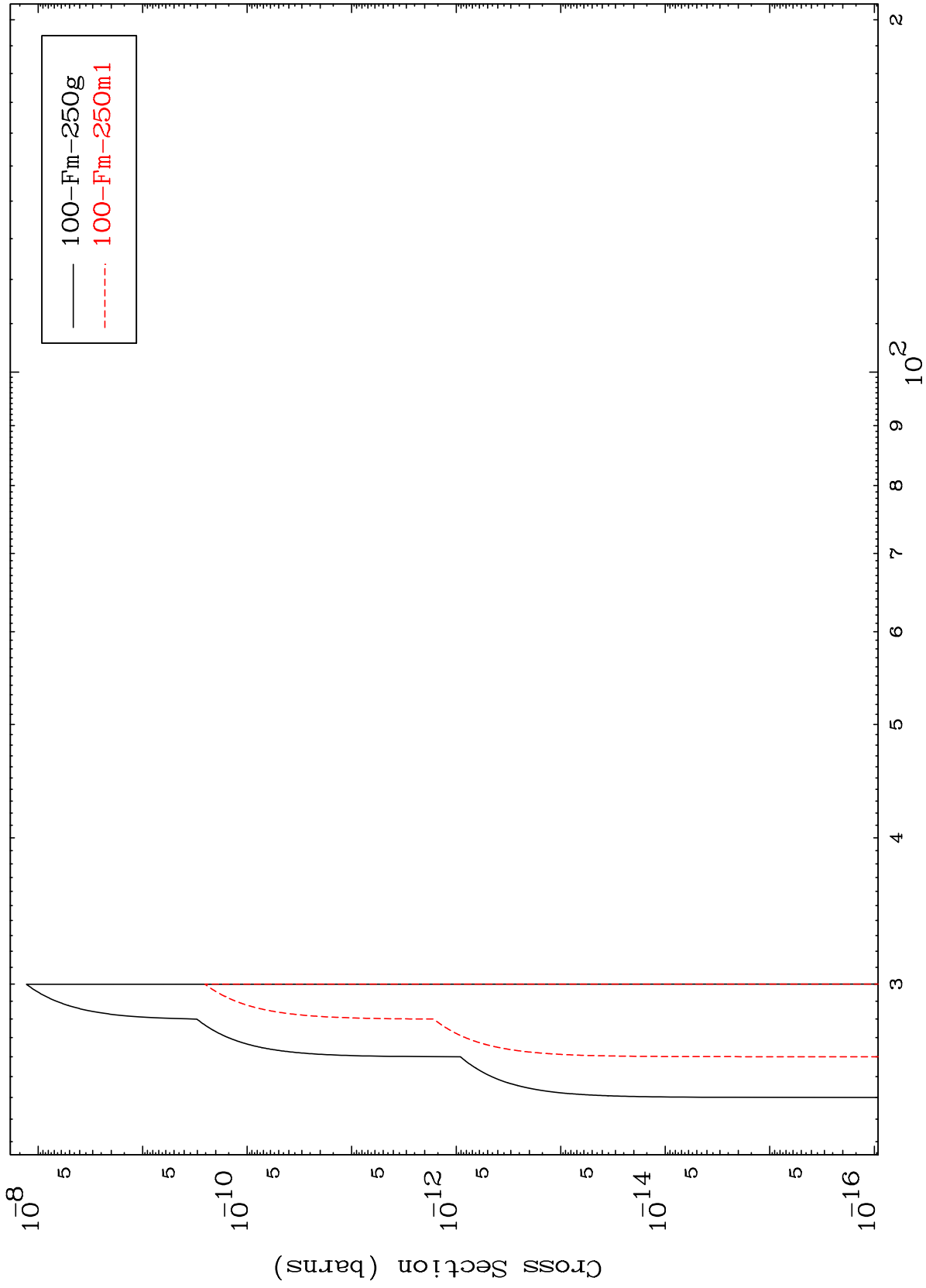
101-Md-254

MAT 9955

( $\gamma, 3n$ ) p

101-Md-254

Radionuclide Production Cross Section



13

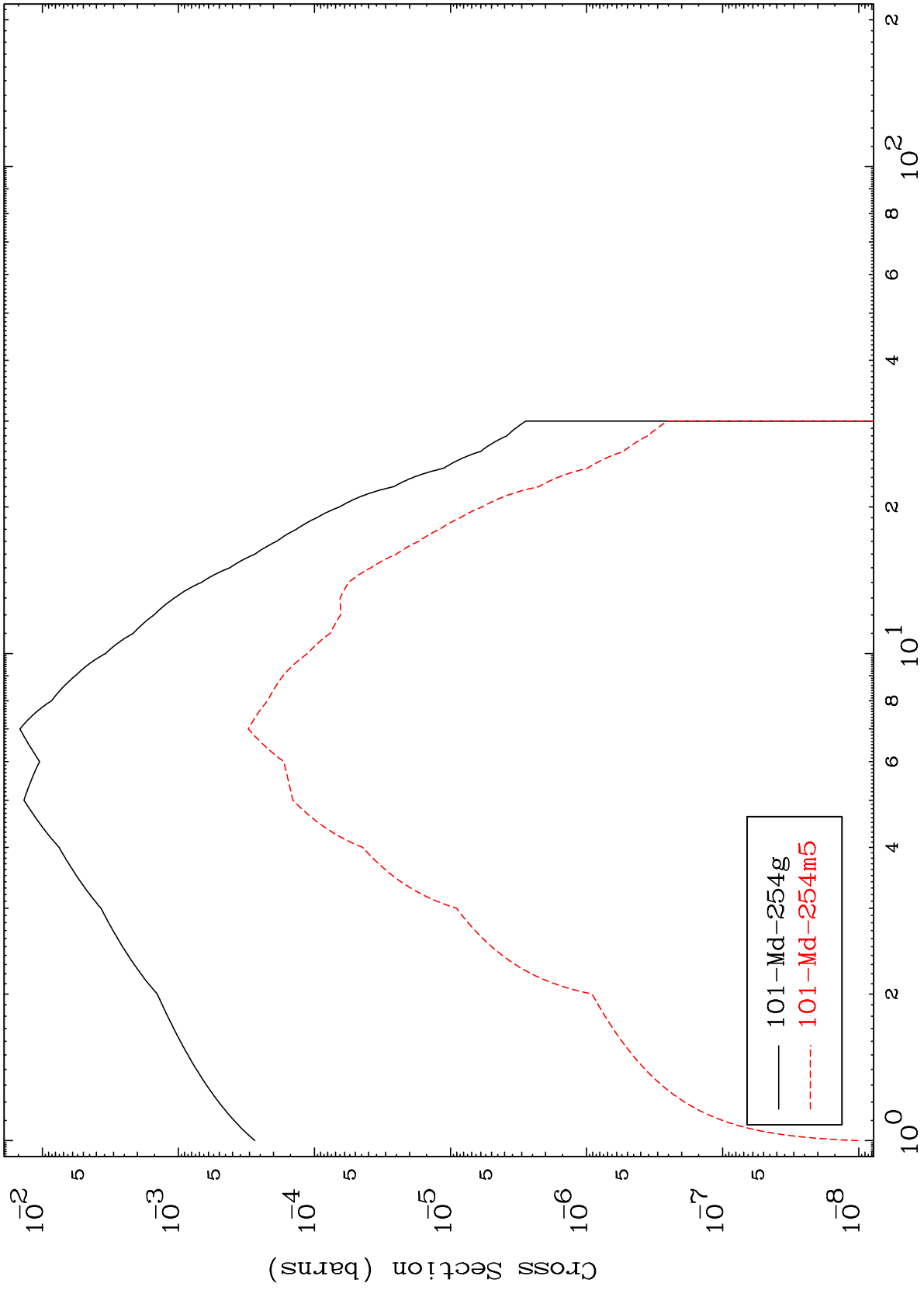
Incident Energy (MeV)

101-Md-254

MAT 9955

101-Md-254

Radionuclide Production Cross Section  
( $\gamma, \gamma$ )



101-Md-254

Incident Energy (MeV)

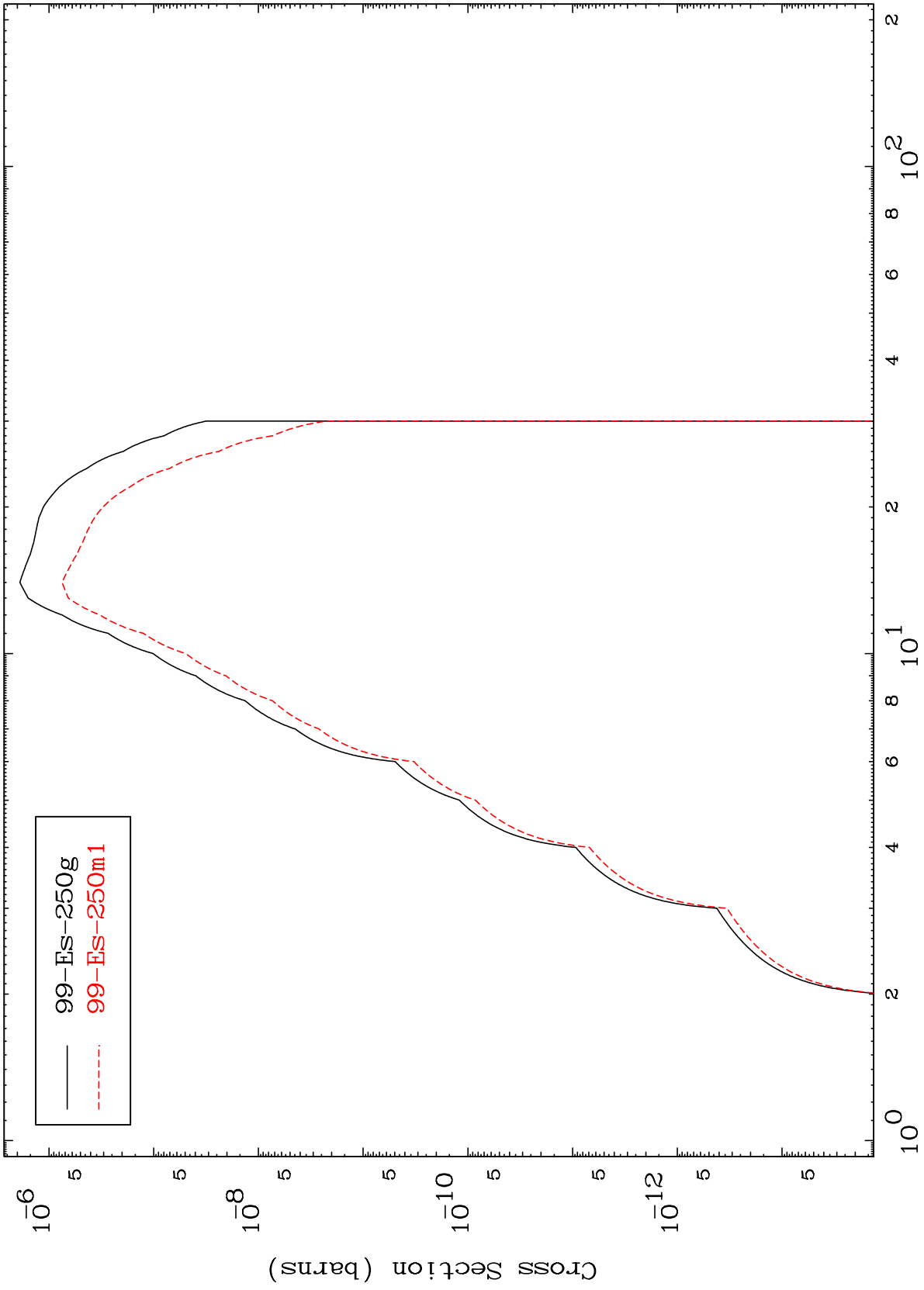
14

MAT 9955

101-Md-254

Radionuclide Production Cross Section

( $\gamma, \alpha$ )



99-Es-250g  
99-Es-250ml

Incident Energy (MeV)

101-Md-254