

Program EVALPLOT
(Version 2018-1)

by

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(Present Contact Information)

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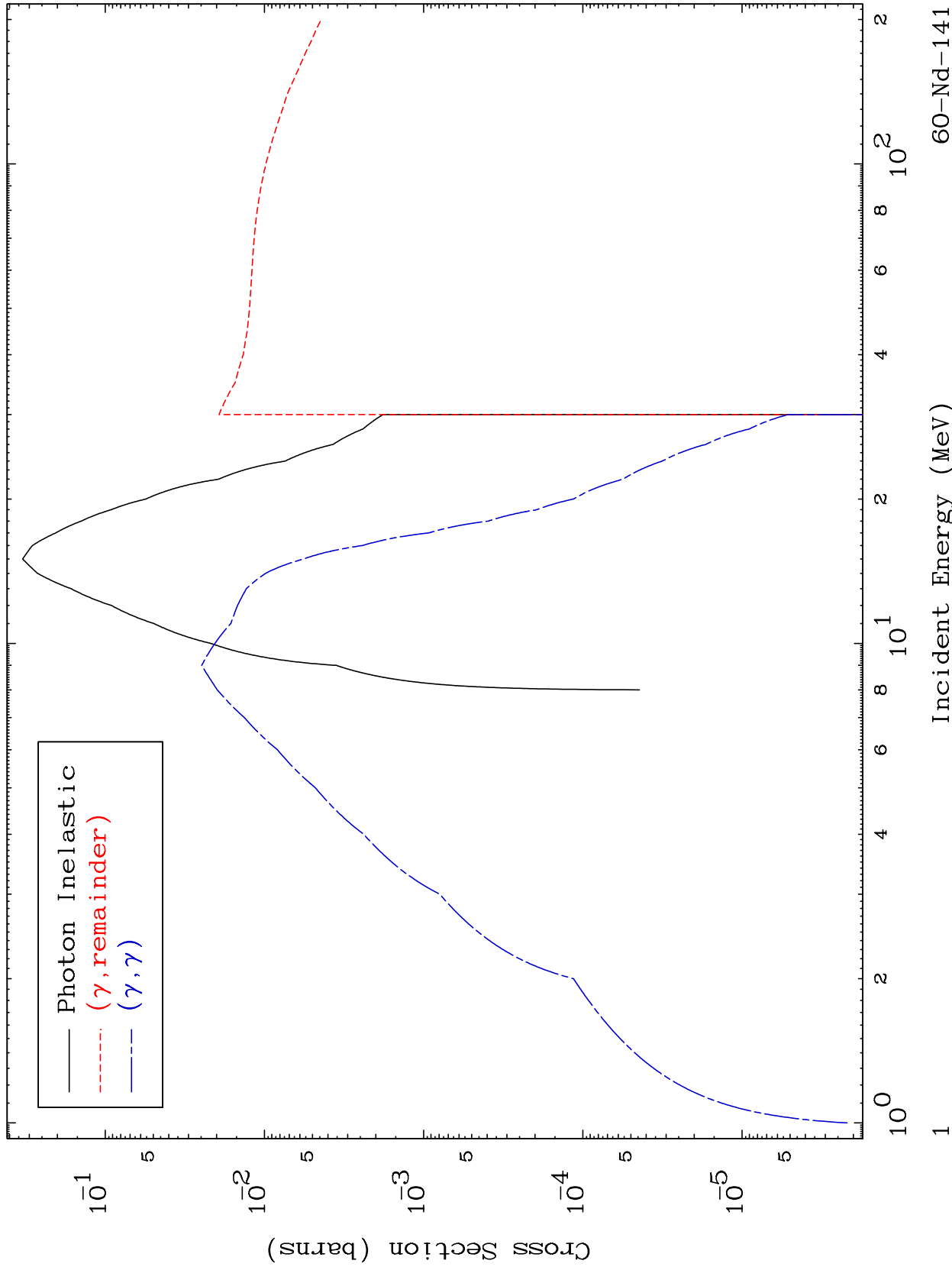
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 6023

Photon Major
0 Kelvin Cross Sections

60-Nd-141

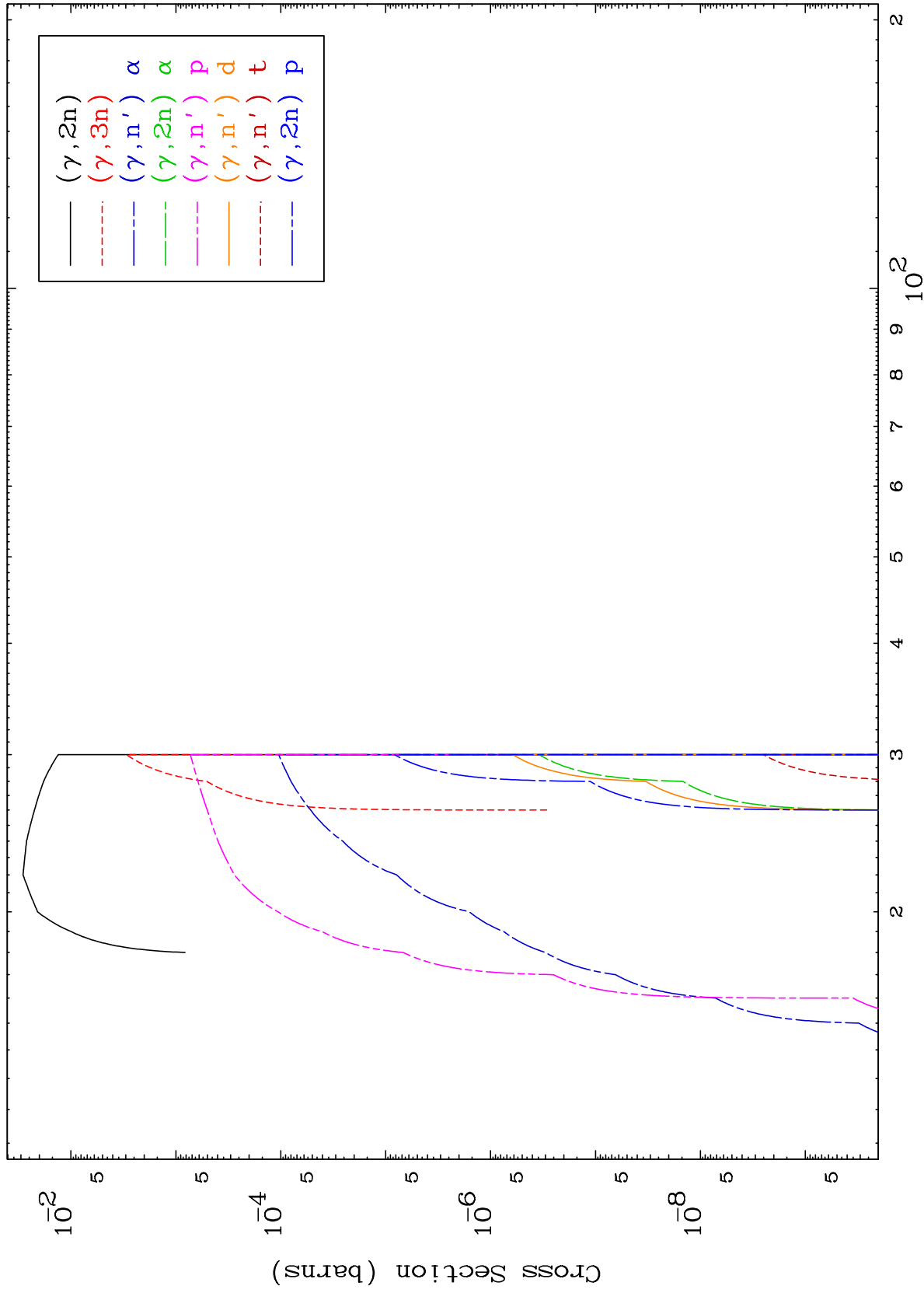


60-Nd-141

MAT 6023

Photon Neutron Production
0 Kelvin Cross Sections

60-Nd-141



2

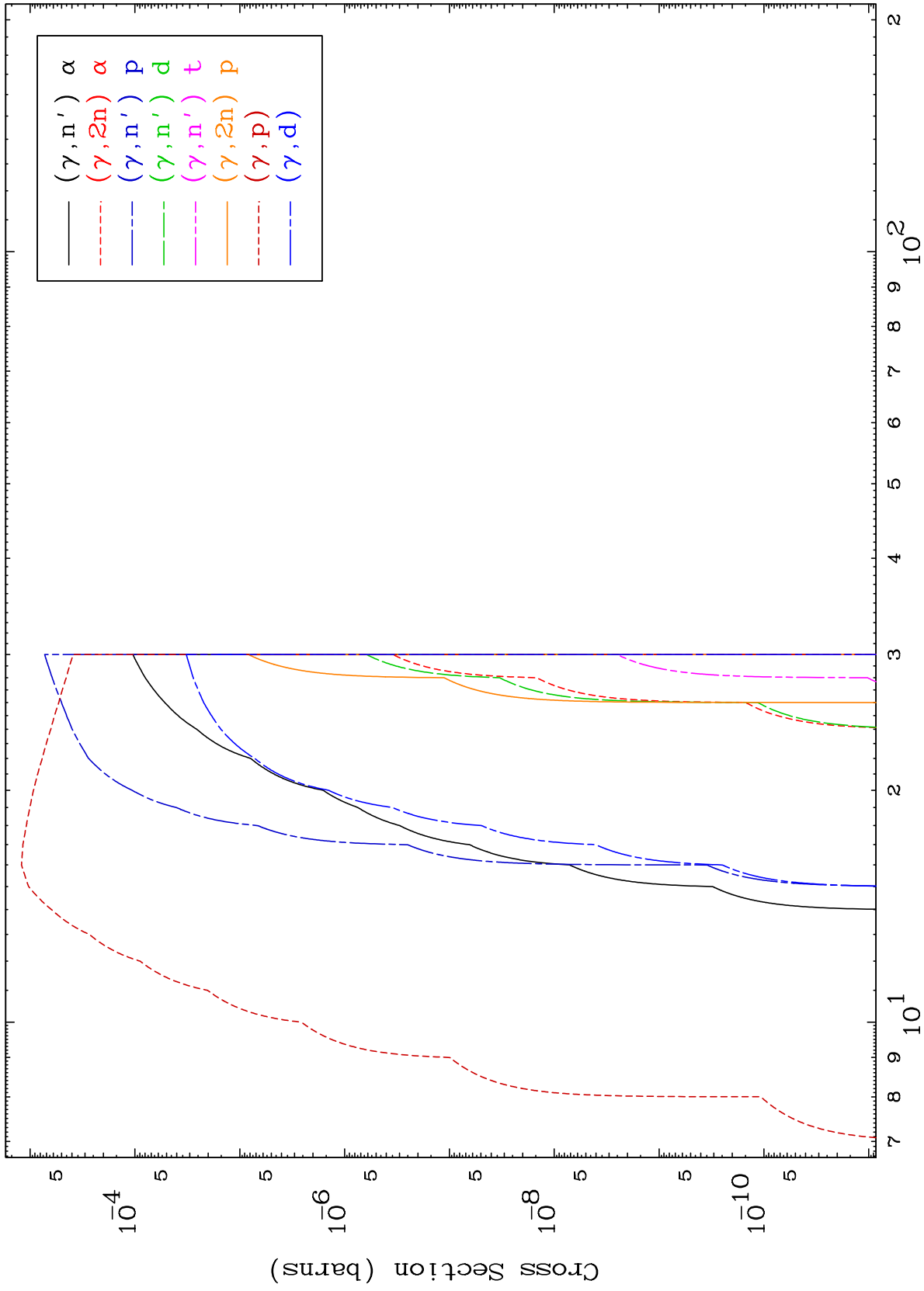
Incident Energy (MeV)

60-Nd-141

MAT 6023

Photon Charged Particle
0 Kelvin Cross Sections

60-Nd-141



3

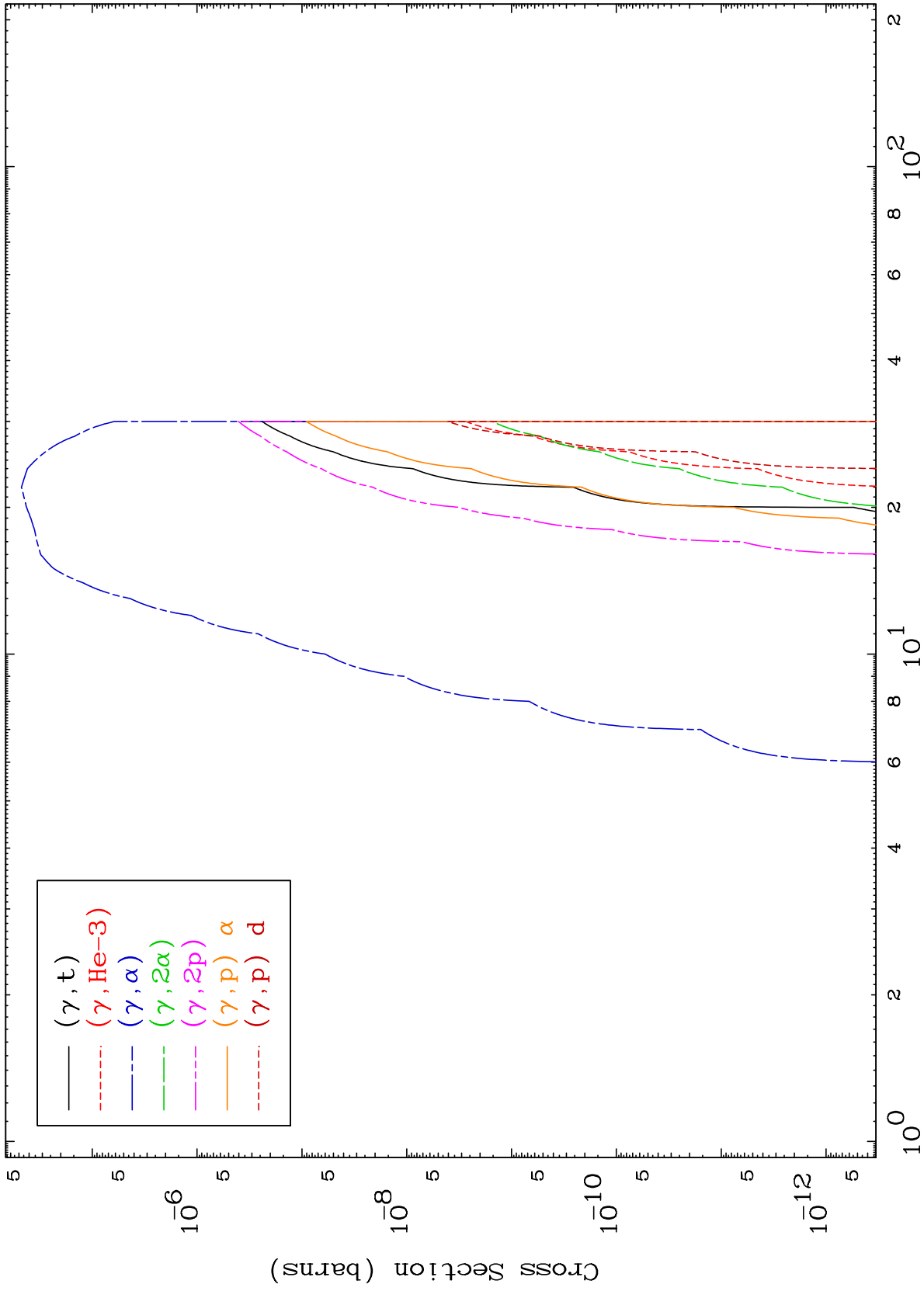
Incident Energy (MeV)

60-Nd-141

MAT 6023

Photon Charged Particle
0 Kelvin Cross Sections

60-Nd-141



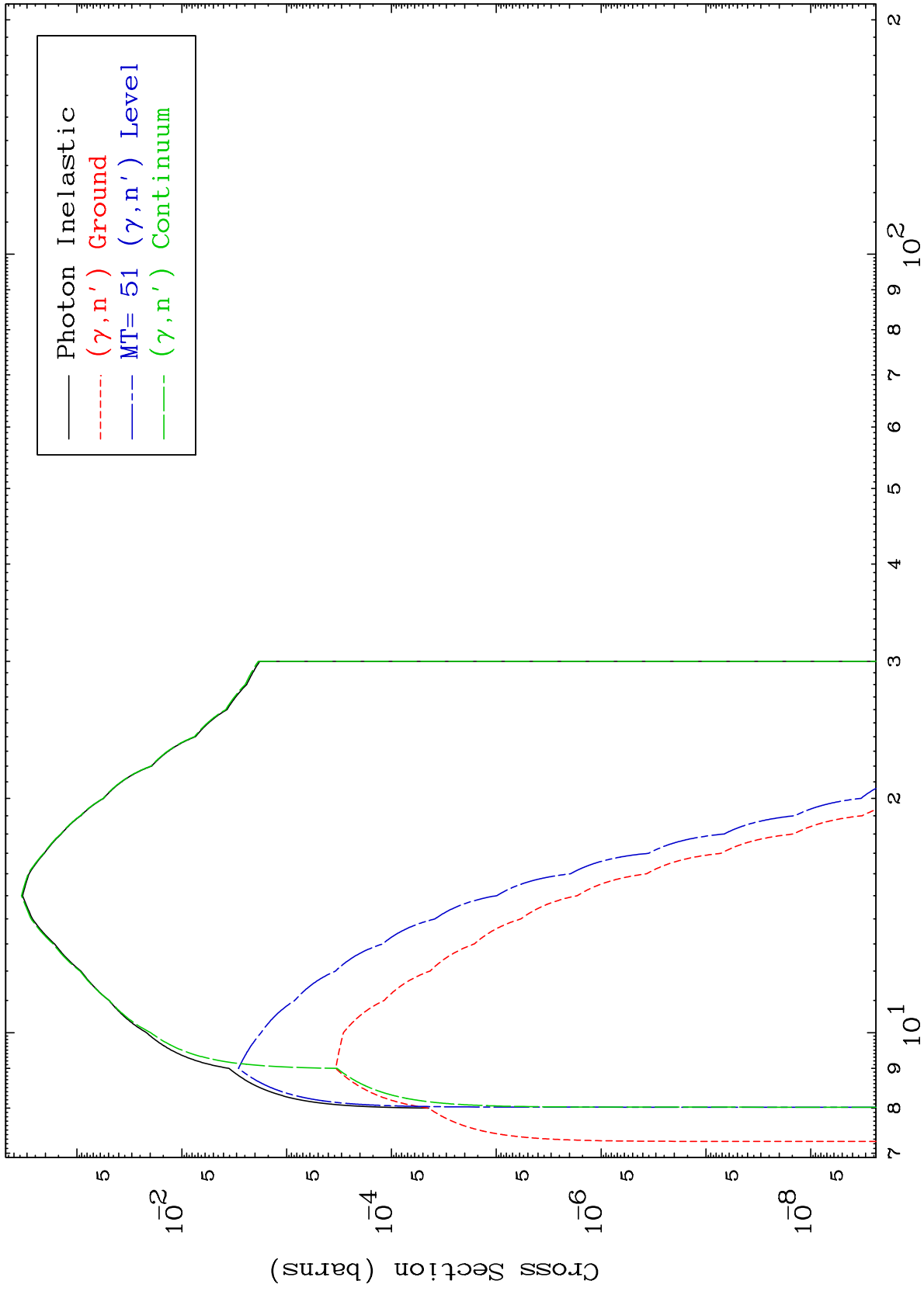
Incident Energy (MeV)

60-Nd-141

MAT 6023

(γ, n') Level
0 Kelvin Cross Sections

60-Nd-141



5

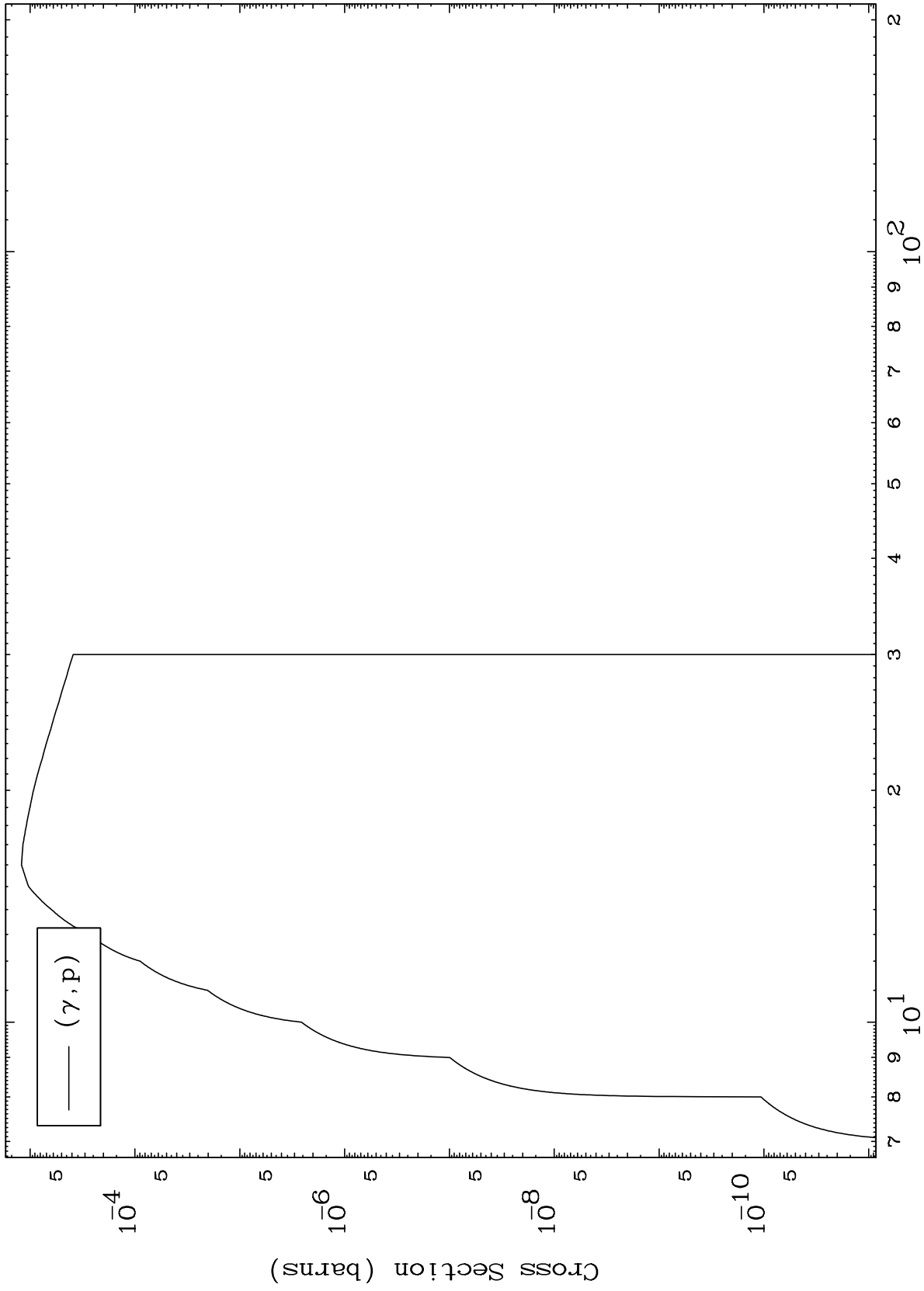
Incident Energy (MeV)

60-Nd-141

MAT 6023

60-Nd-141

(γ, p) Levels
0 Kelvin Cross Sections



6

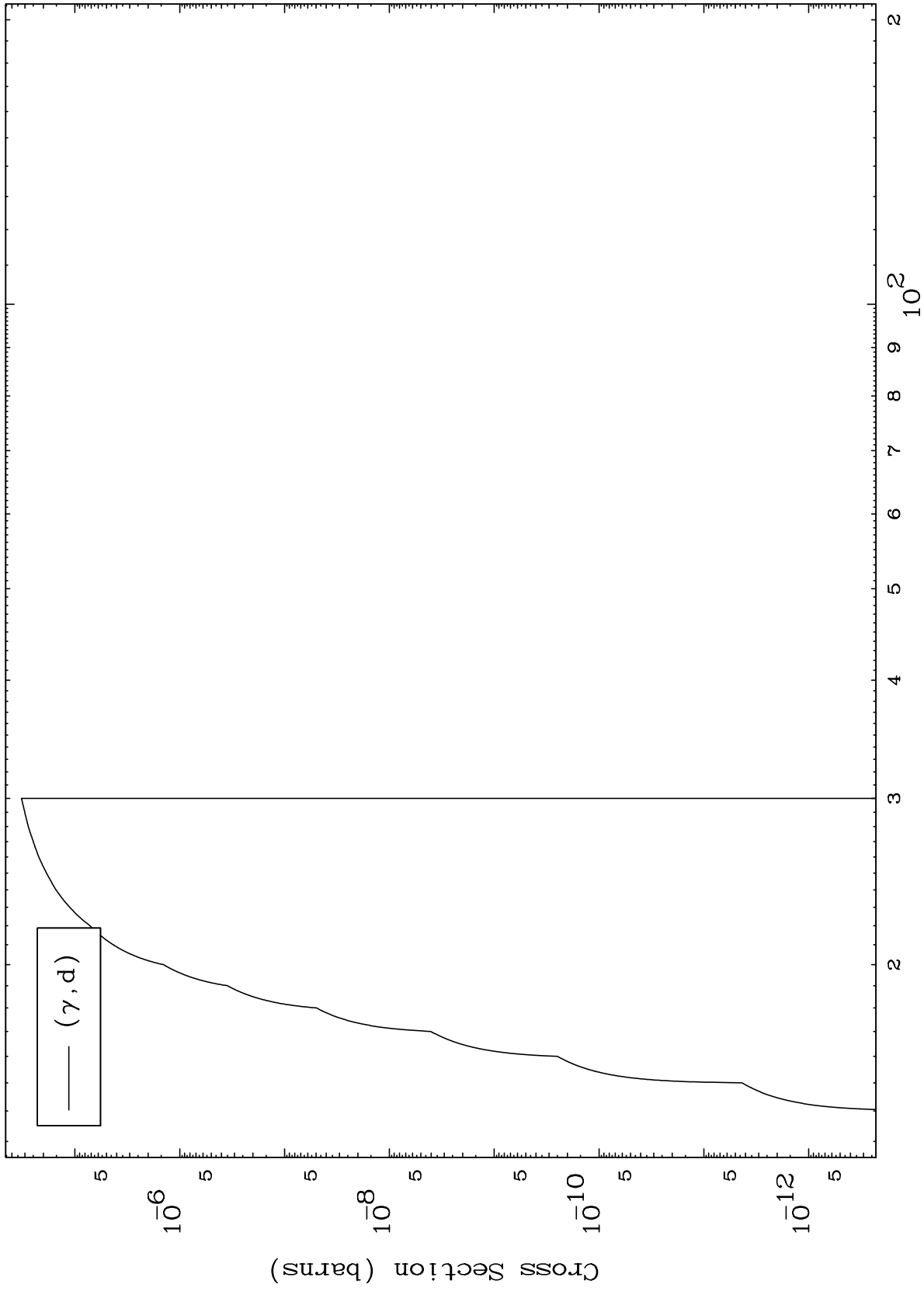
Incident Energy (MeV)

60-Nd-141

MAT 6023

(γ, d) Levels
0 Kelvin Cross Sections

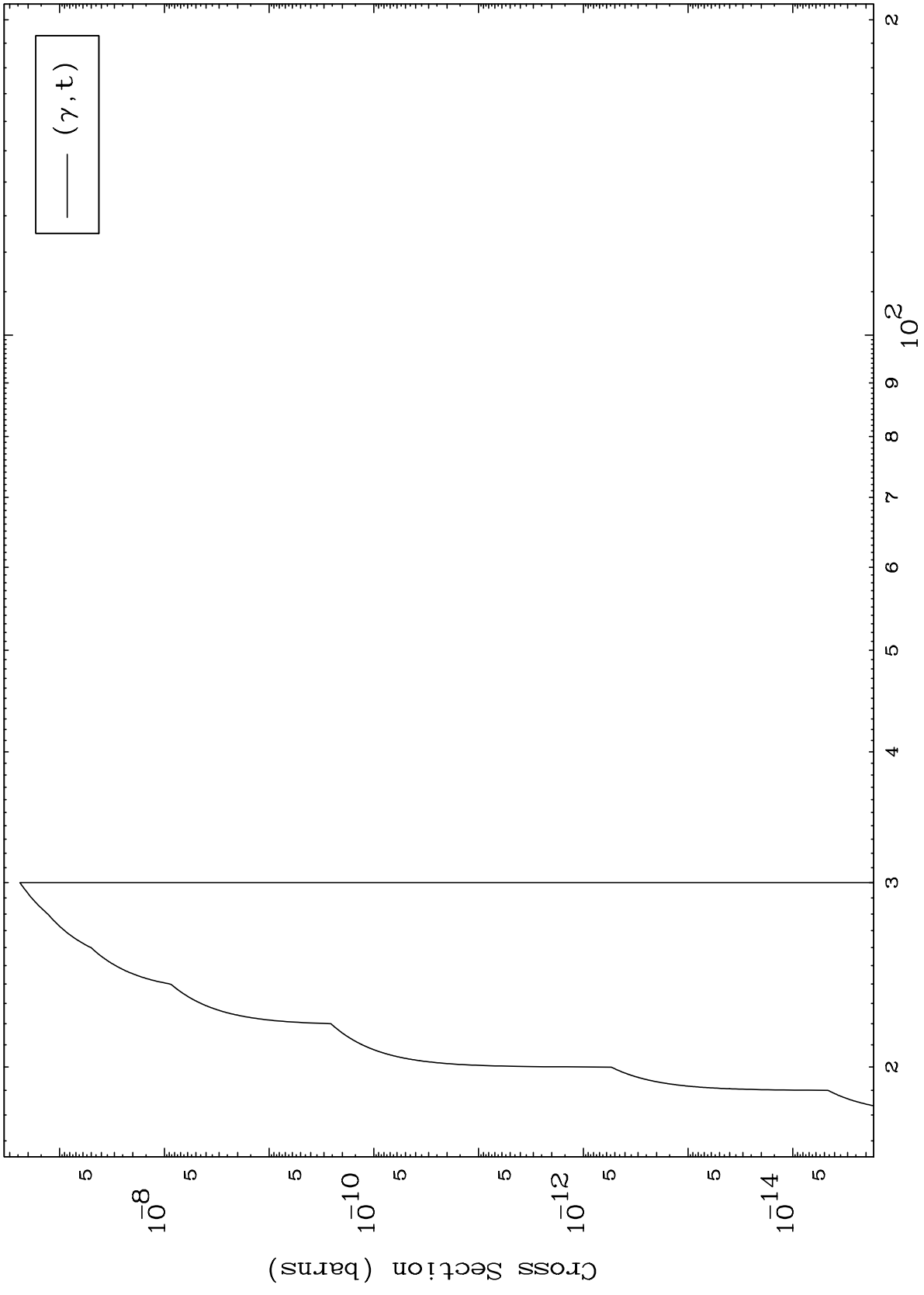
60-Nd-141

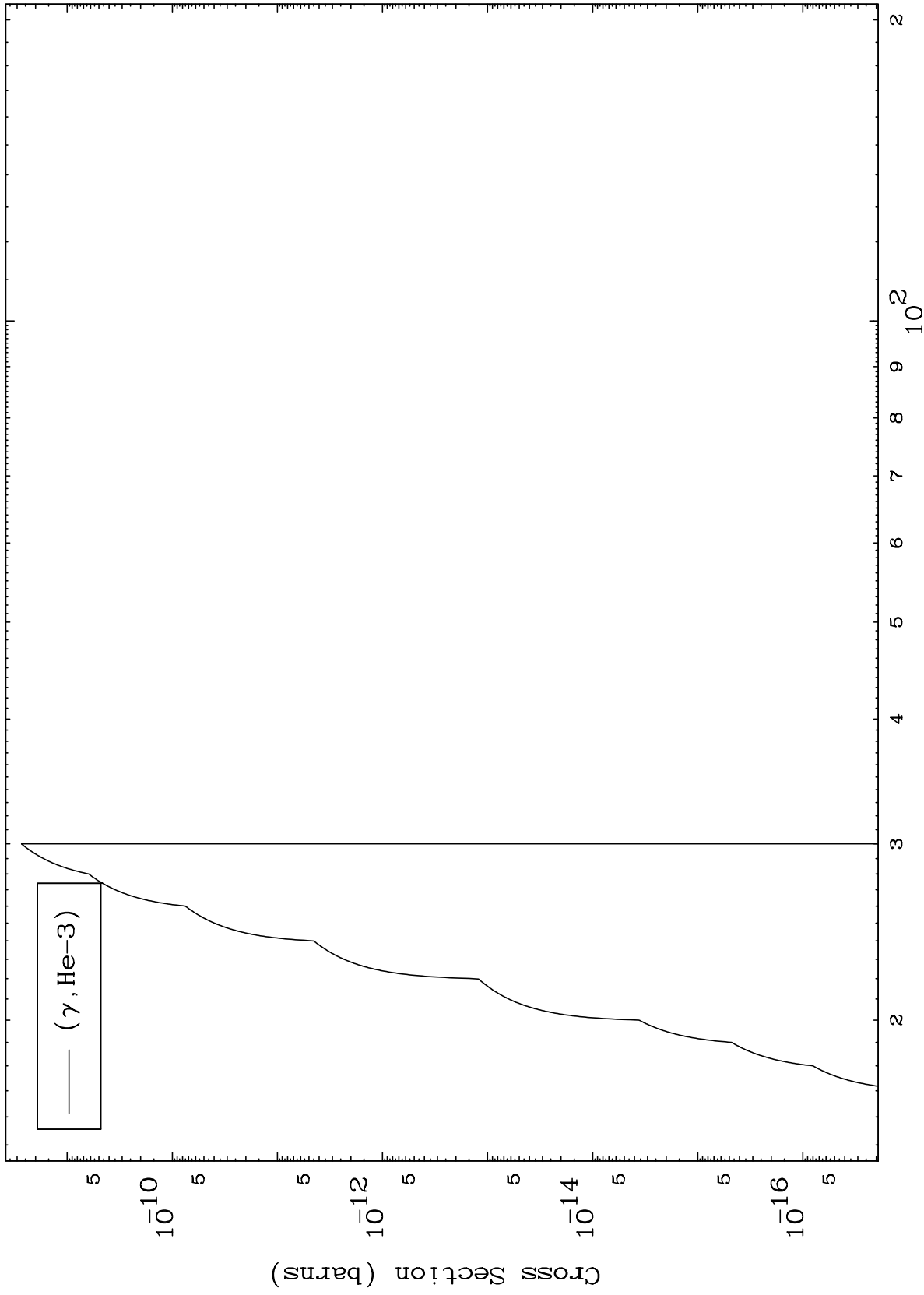


7

Incident Energy (MeV)

60-Nd-141

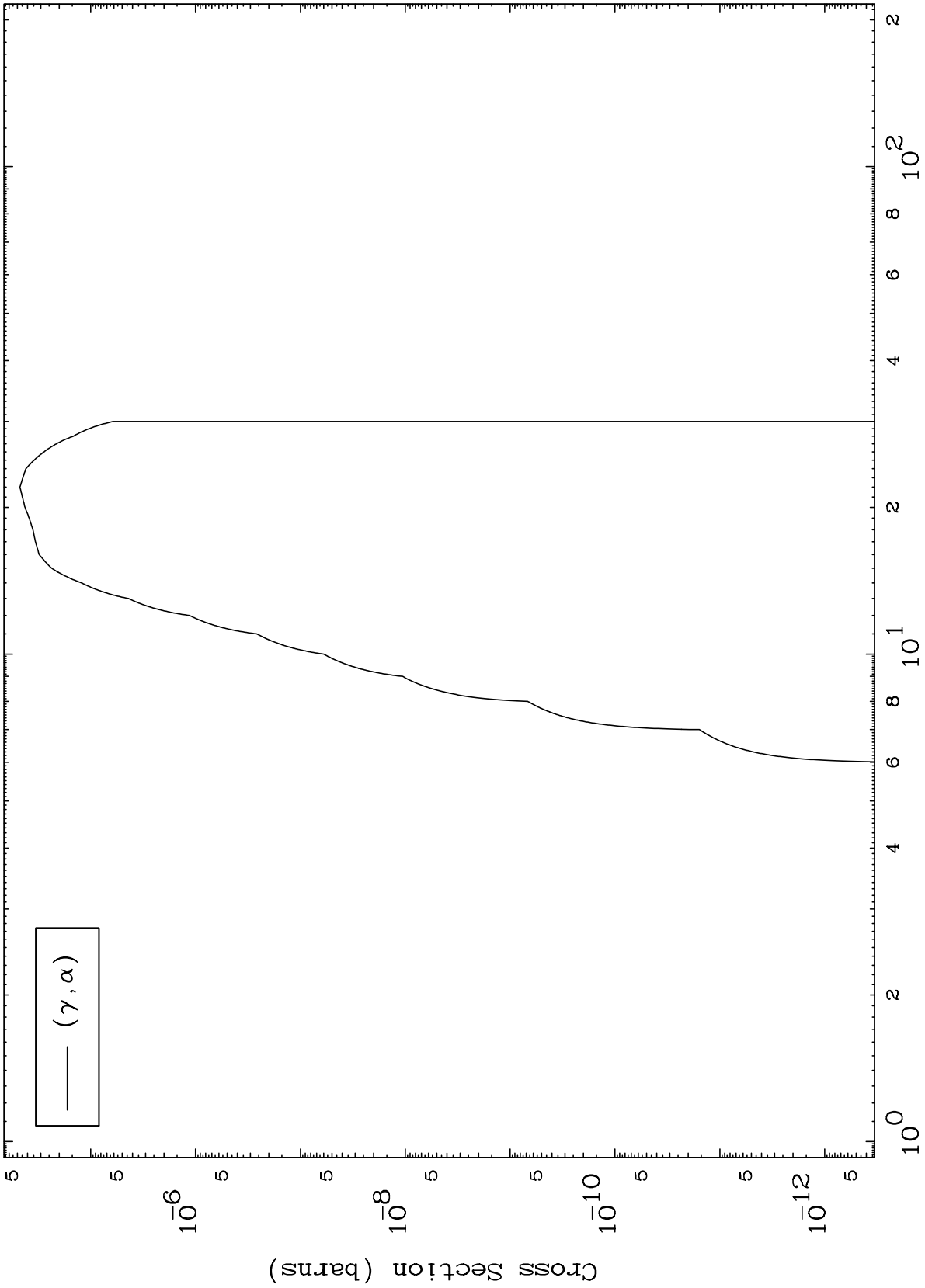




MAT 6023

(γ, α) Levels
0 Kelvin Cross Sections

60-Nd-141



(γ, α)

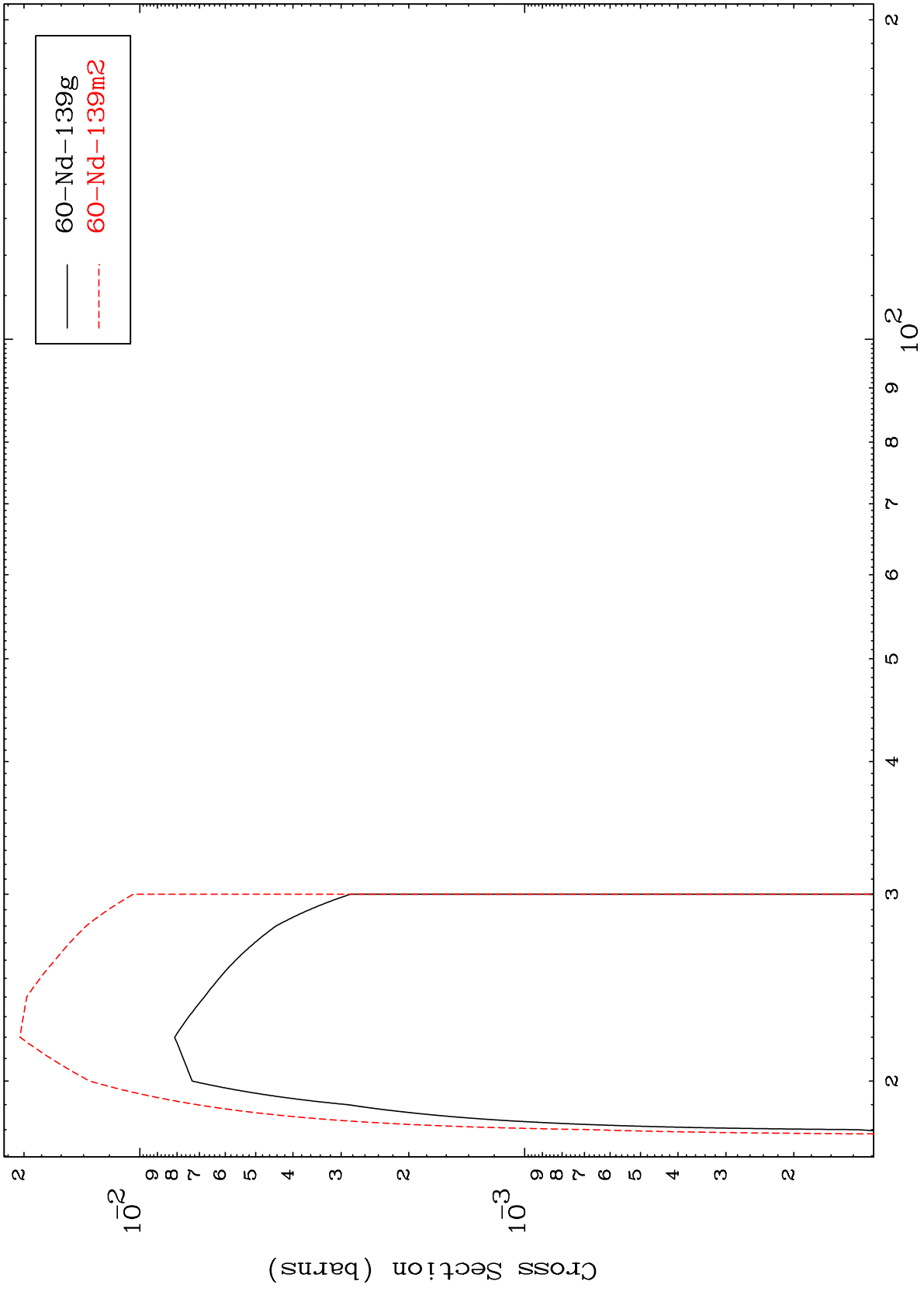
Incident Energy (MeV)

60-Nd-141

MAT 6023

60-Nd-141

Radionuclide Production Cross Section
($\gamma, 2n$)



11

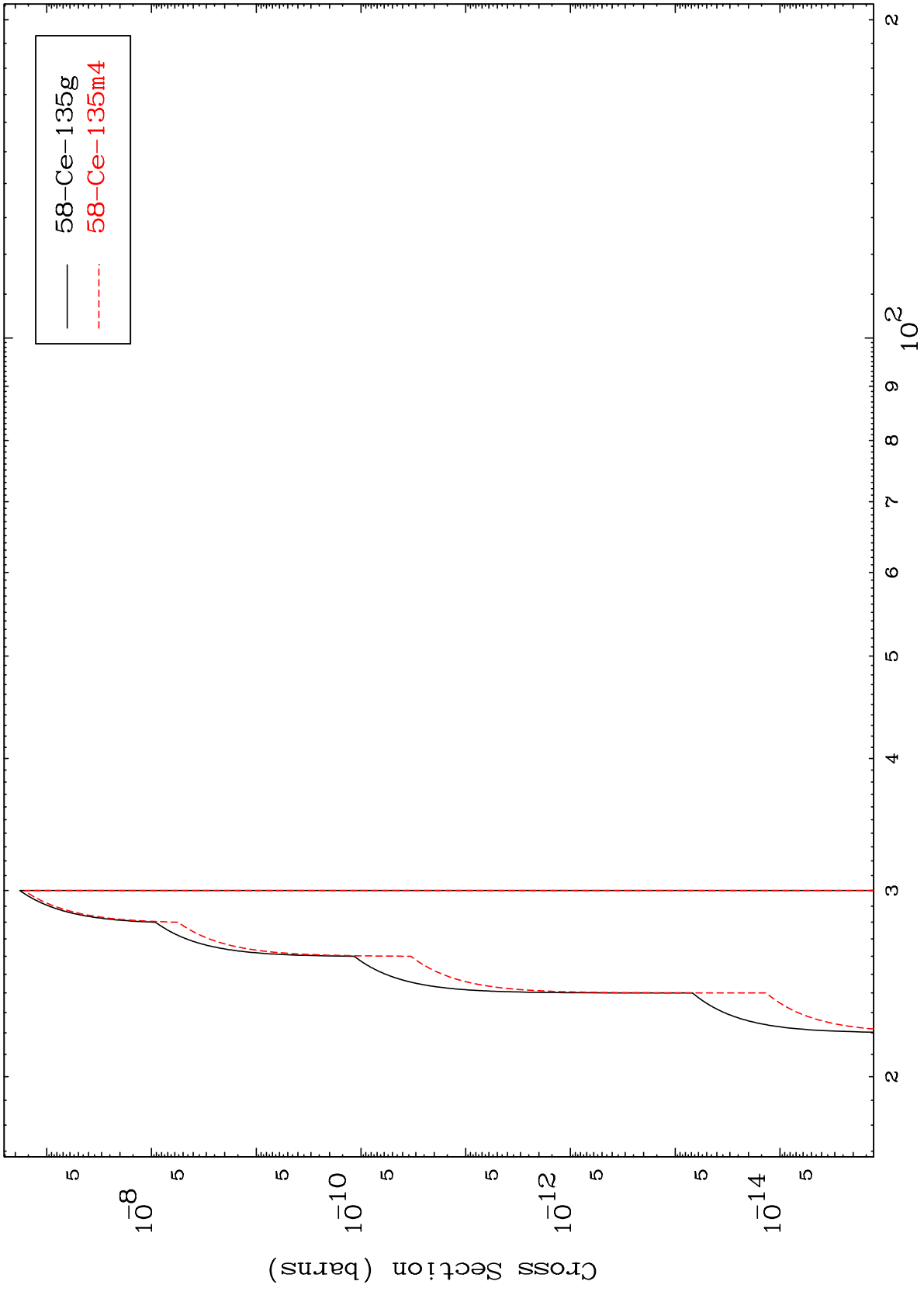
60-Nd-141

MAT 6023

$(\gamma, 2n) \alpha$

$^{60}\text{Nd}-141$

Radionuclide Production Cross Section



12

Incident Energy (MeV)

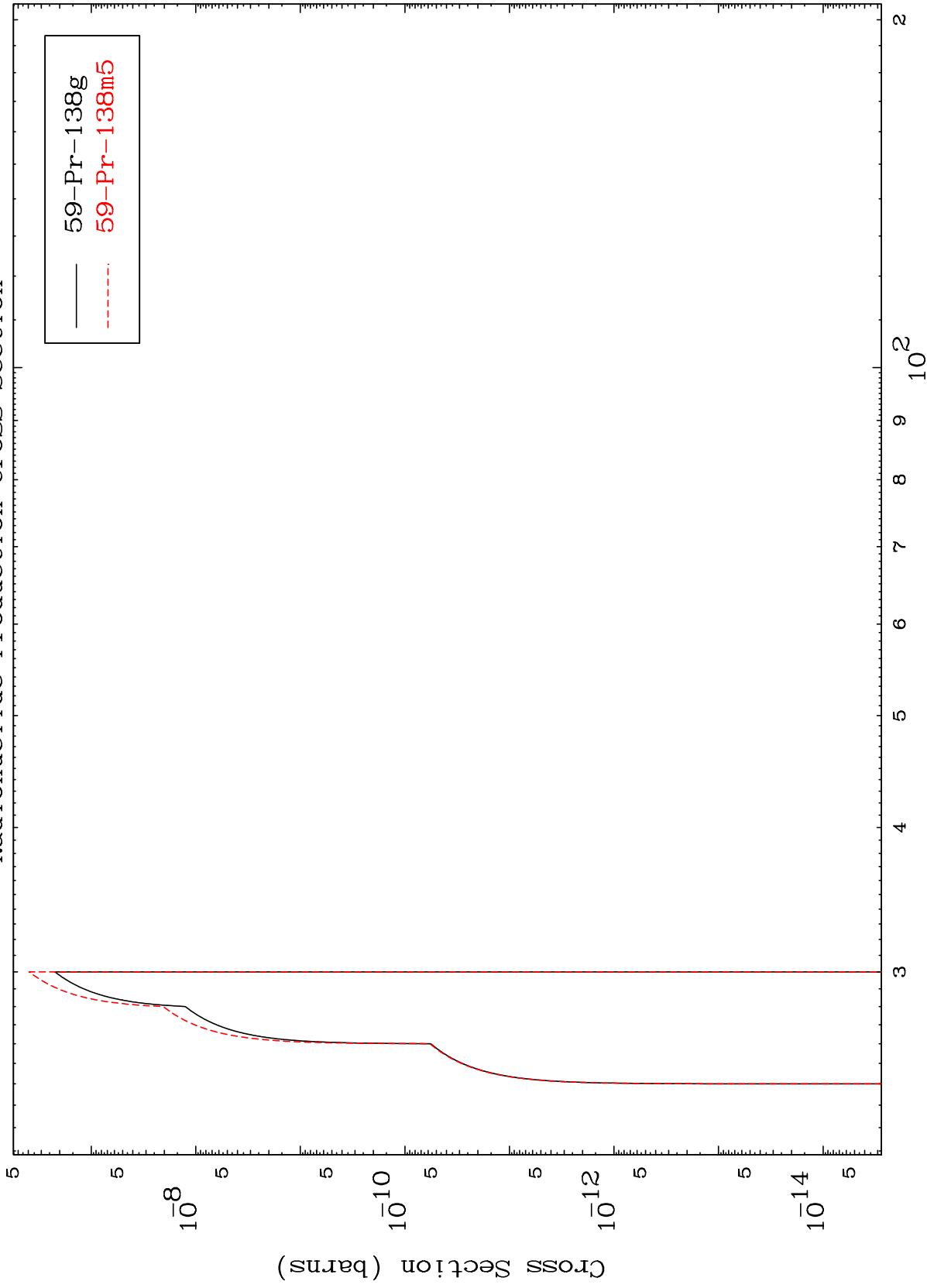
$^{60}\text{Nd}-141$

MAT 6023

(γ, n') d

$^{60}\text{Nd}-^{141}$

Radionuclide Production Cross Section



13

Incident Energy (MeV)

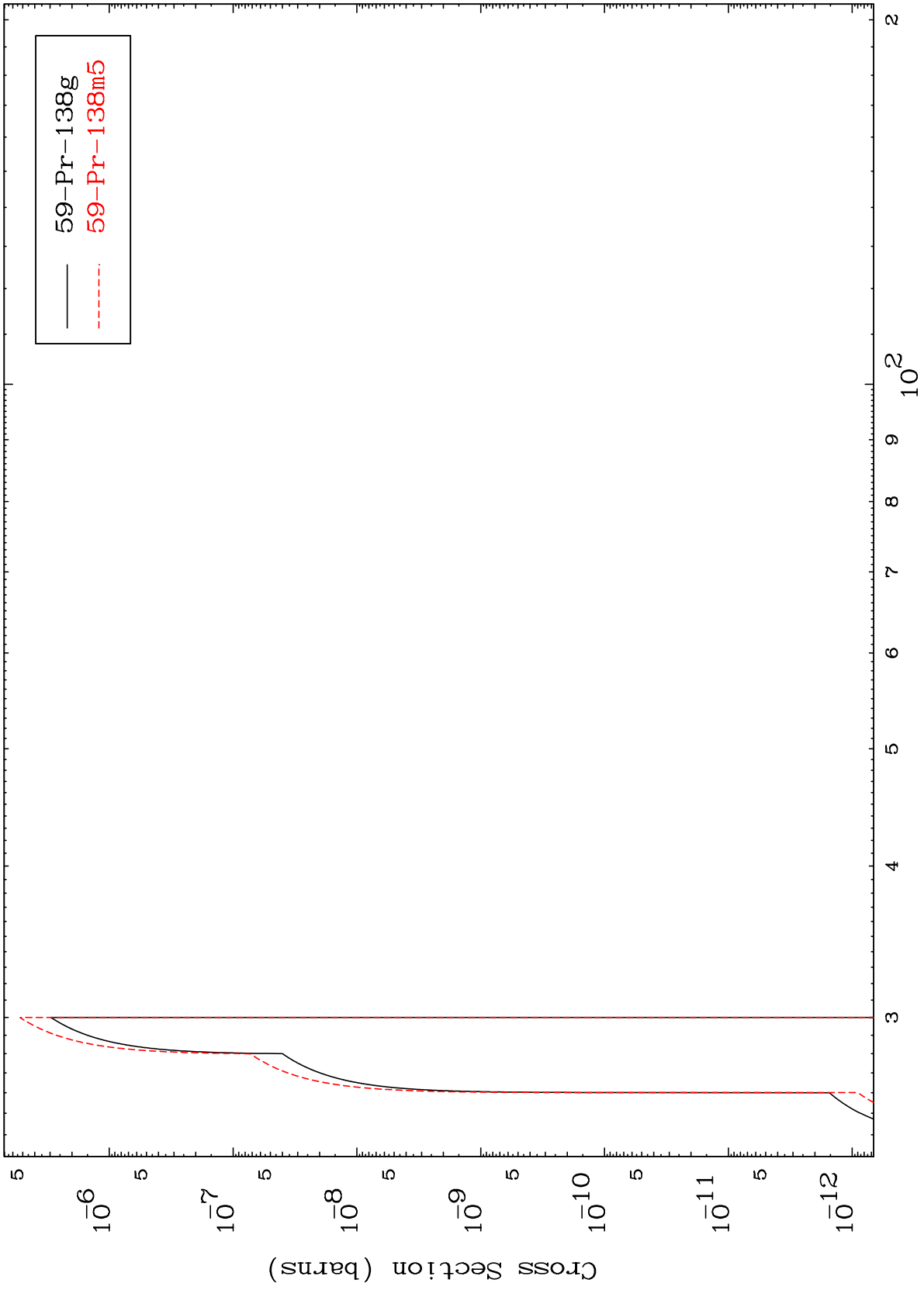
$^{60}\text{Nd}-^{141}$

MAT 6023

($\gamma, 2n$) p

60-Nd-141

Radionuclide Production Cross Section



14

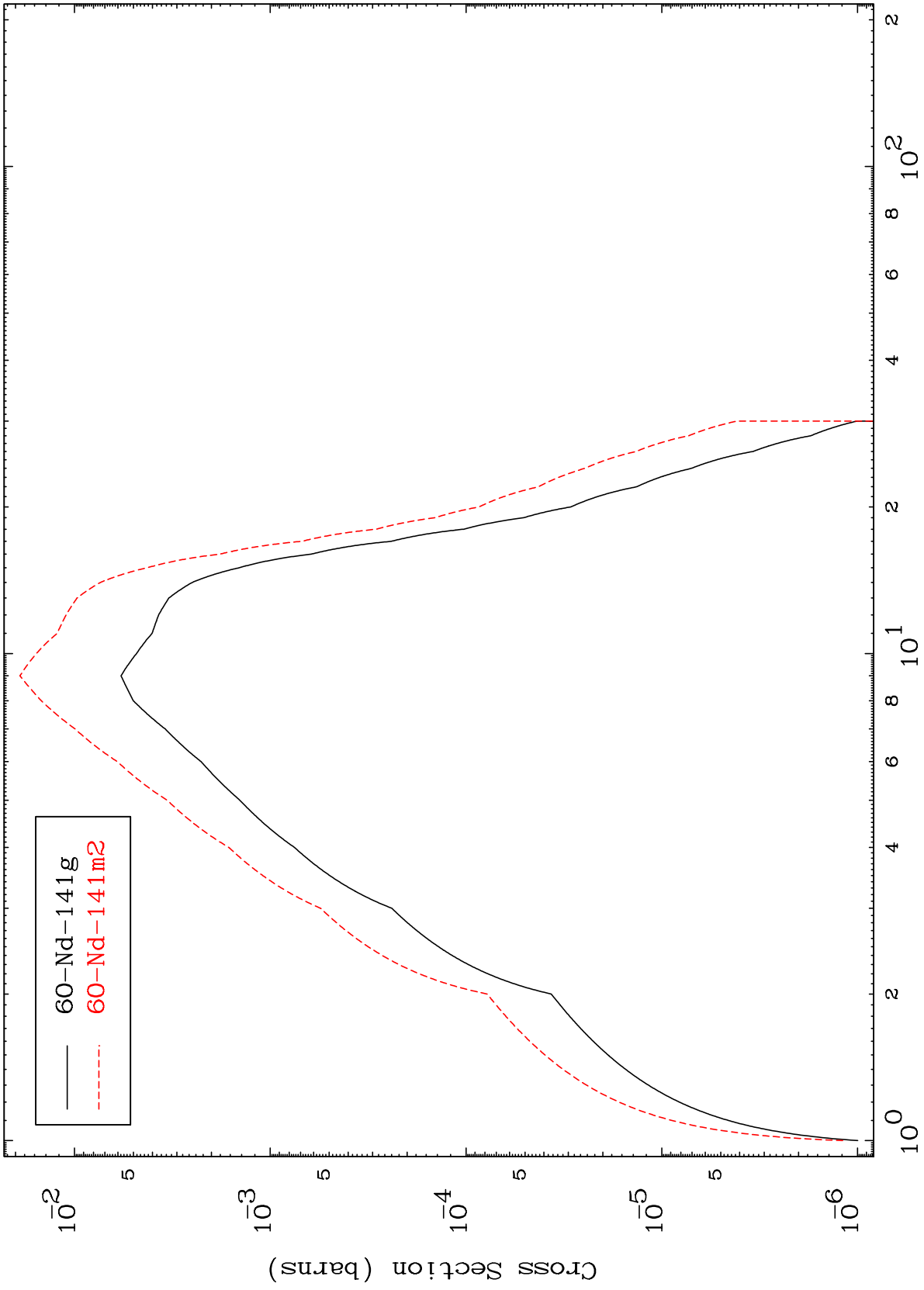
Incident Energy (MeV)

60-Nd-141

MAT 6023

Radionuclide Production Cross Section
(γ, γ)

$^{60}\text{Nd}-141$



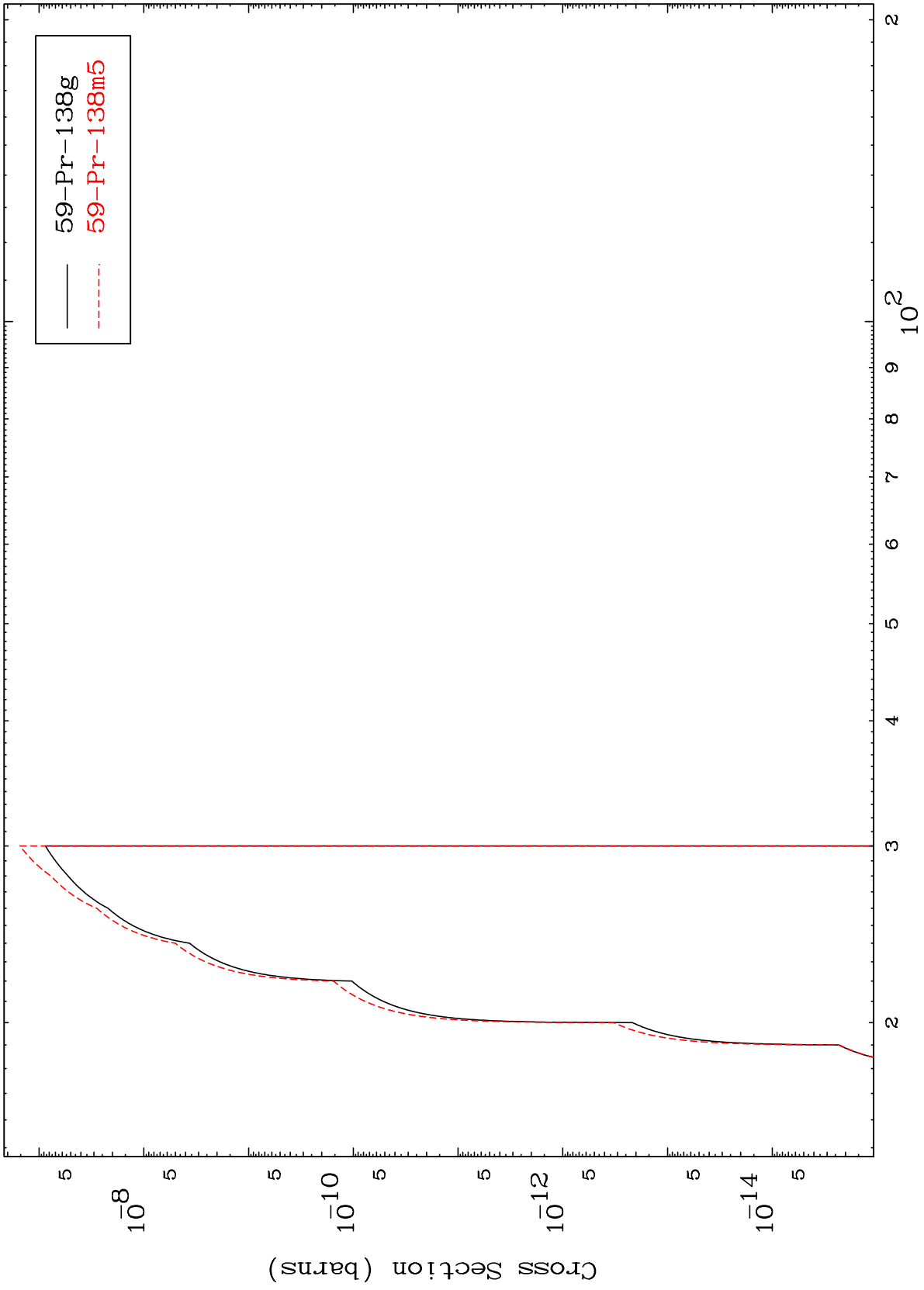
Incident Energy (MeV)

$^{60}\text{Nd}-141$

MAT 6023

60-Nd-141

(γ, t)
Radionuclide Production Cross Section



16

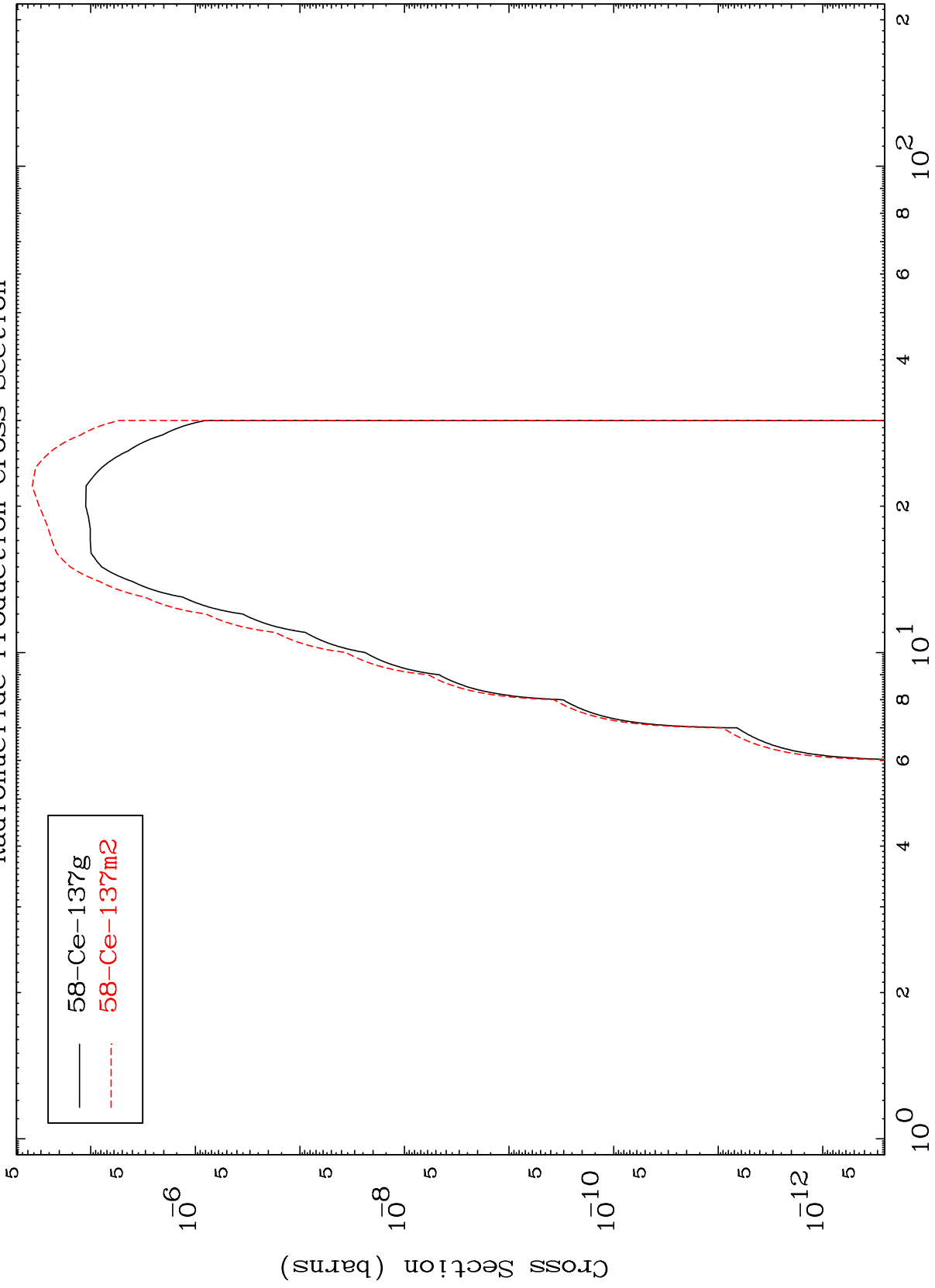
Incident Energy (MeV)

60-Nd-141

MAT 6023

Radionuclide Production Cross Section
(γ, α)

60-Nd-141



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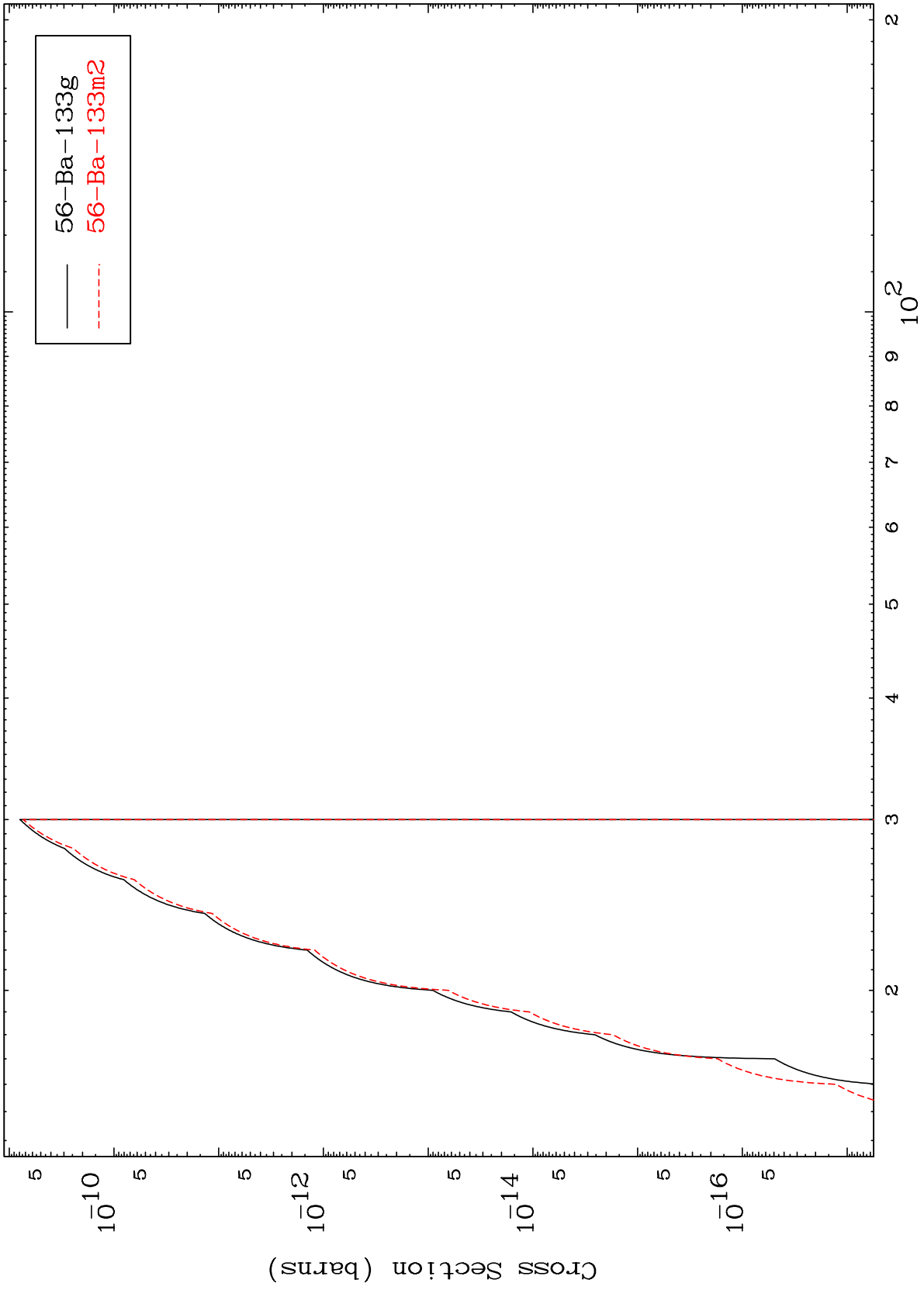
Incident Energy (MeV)

60-Nd-141

MAT 6023

60-Nd-141

($\gamma, 2\alpha$)
Radionuclide Production Cross Section



18

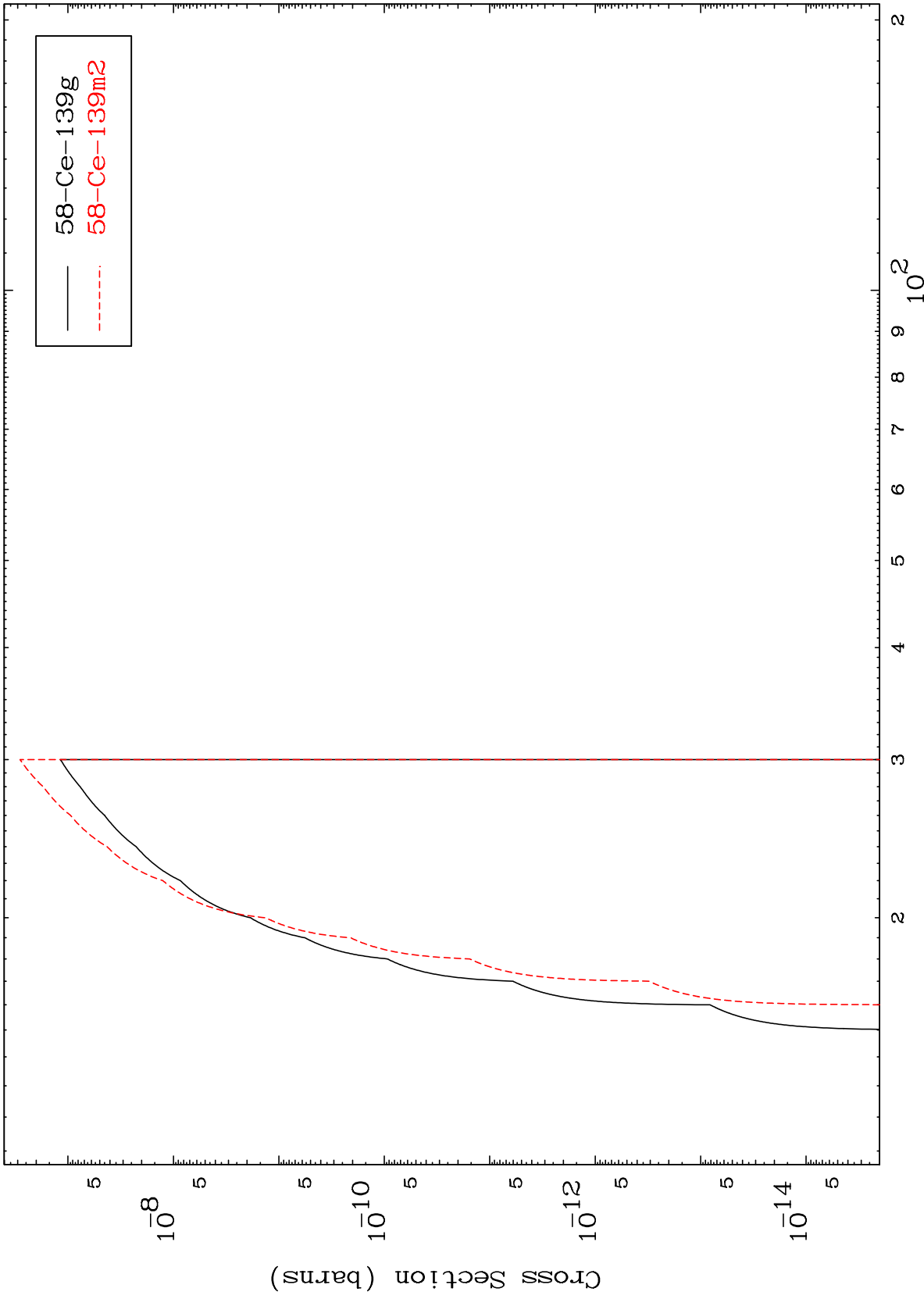
Incident Energy (MeV)

60-Nd-141

MAT 6023

60-Nd-141

($\gamma, 2p$)
Radionuclide Production Cross Section



19

Incident Energy (MeV)

60-Nd-141