

Program EVALPLOT  
(Version 2018-1)

by

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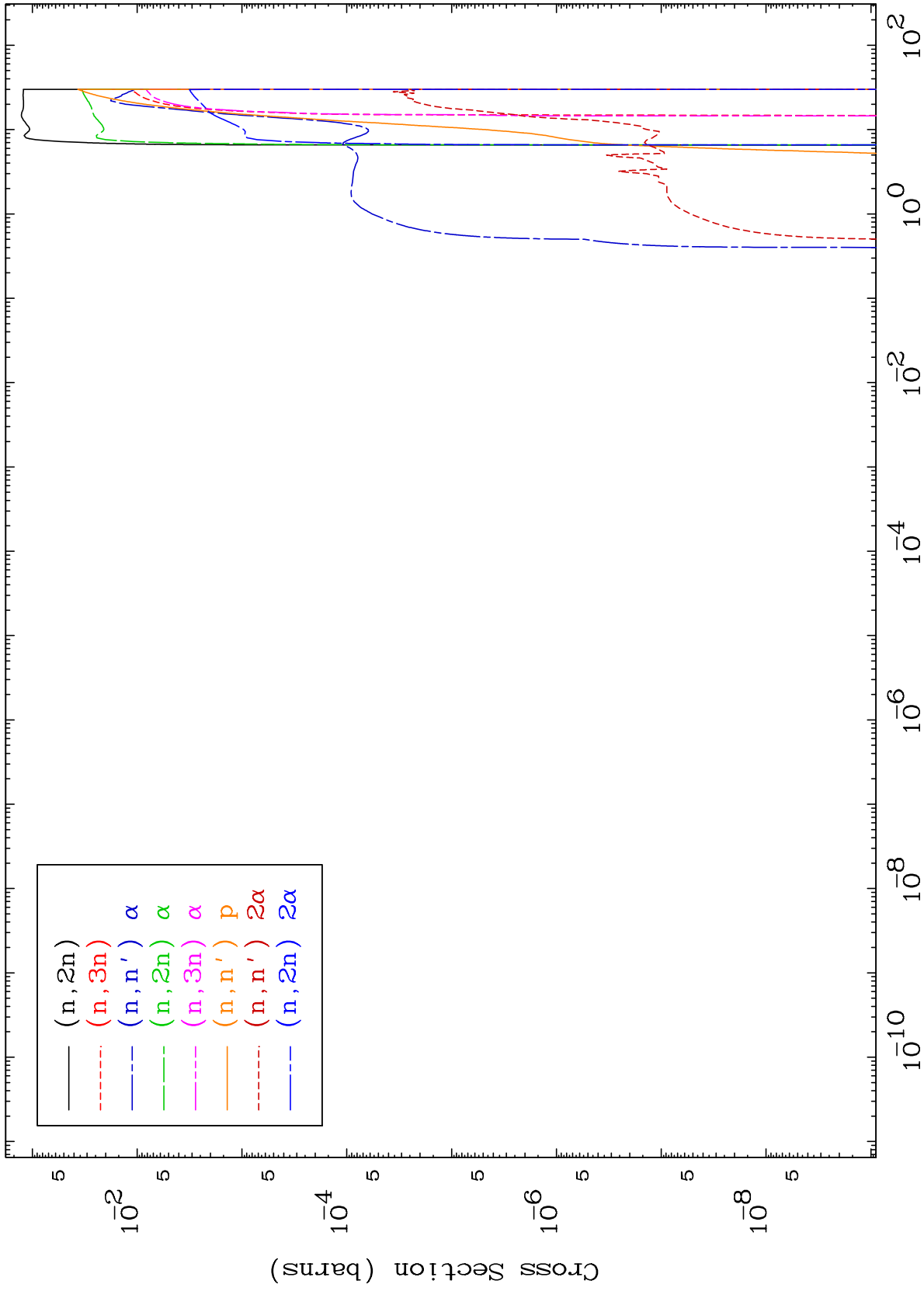
Press Mouse Button to Start



MAT 764

Neutron Production  
293 Kelvin Cross Sections

107-Bh-264



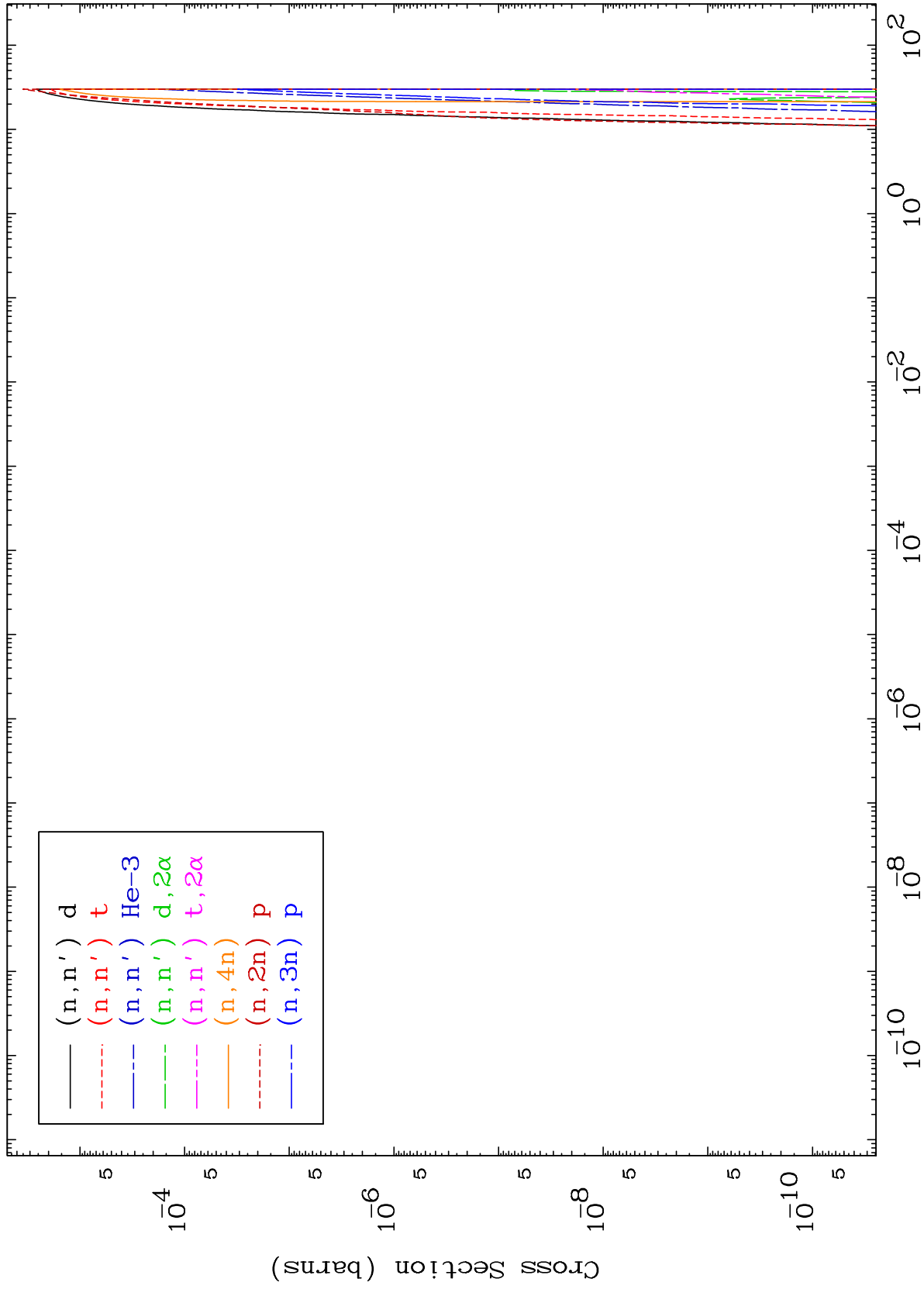
2

107-Bh-264

MAT 764

Neutron Production  
293 Kelvin Cross Sections

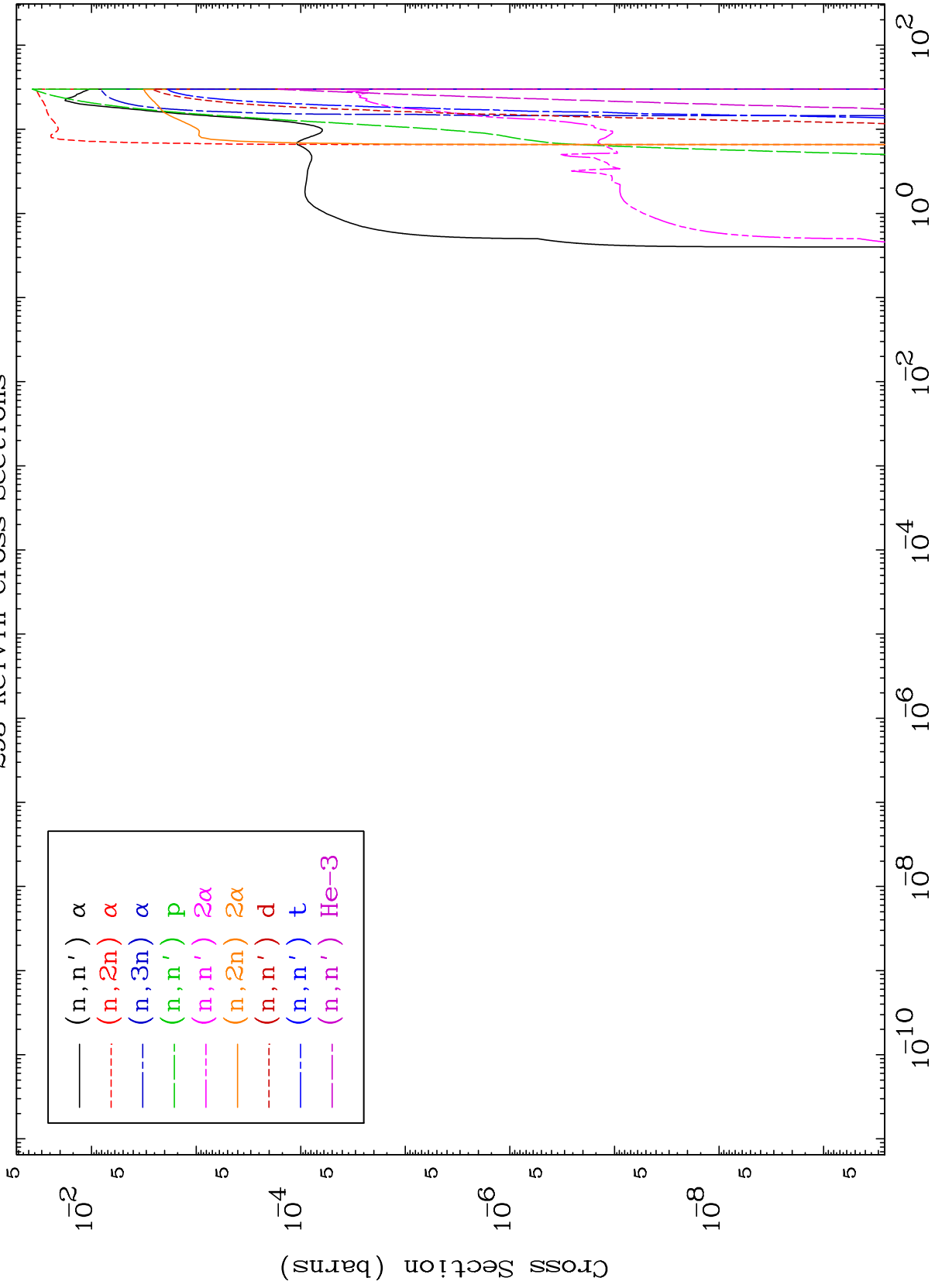
107-Bh-264



MAT 764

293 Kelvin Cross Sections

107-Bh-264

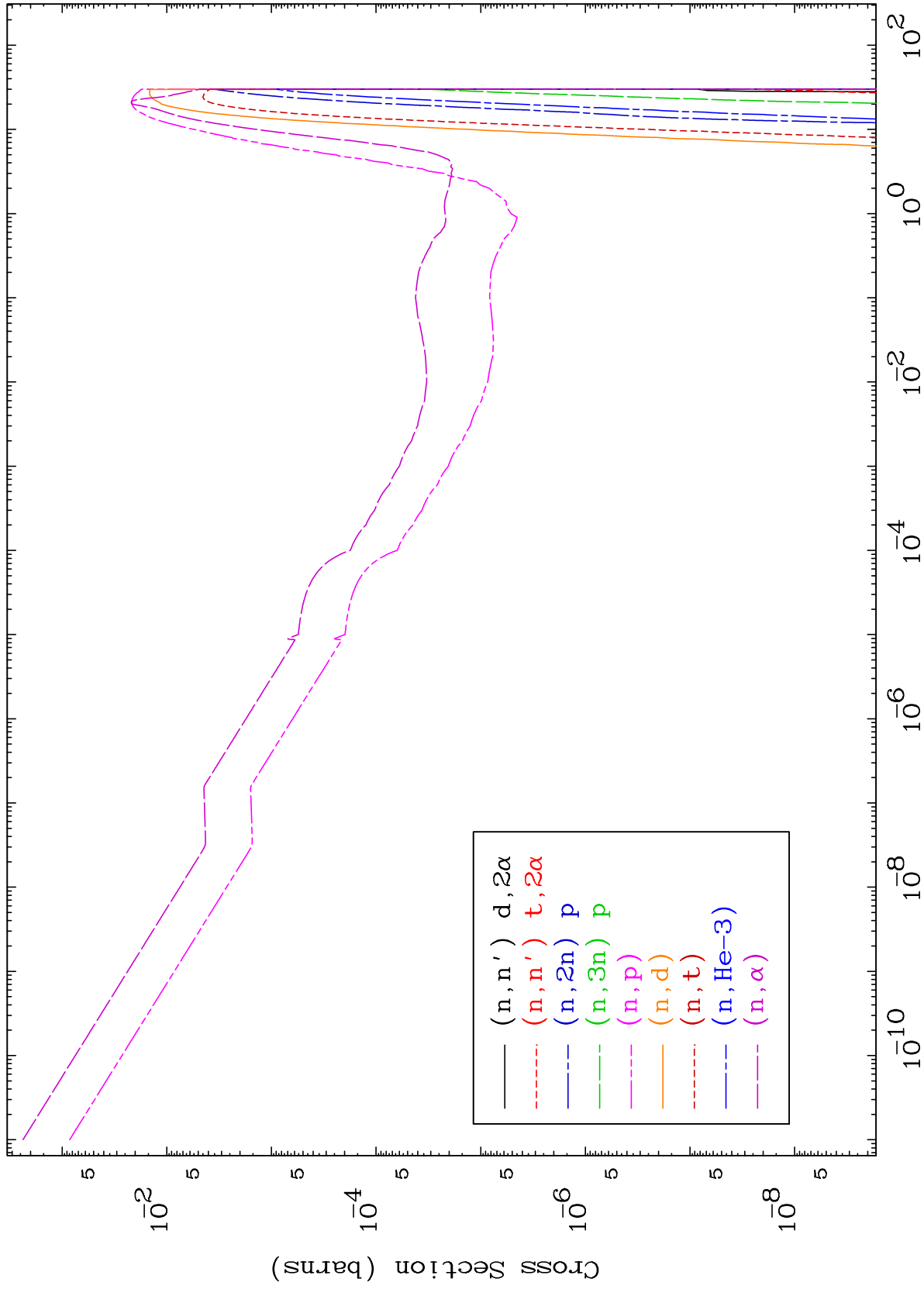


107-Bh-264

MAT 764

Charged Particle  
293 Kelvin Cross Sections

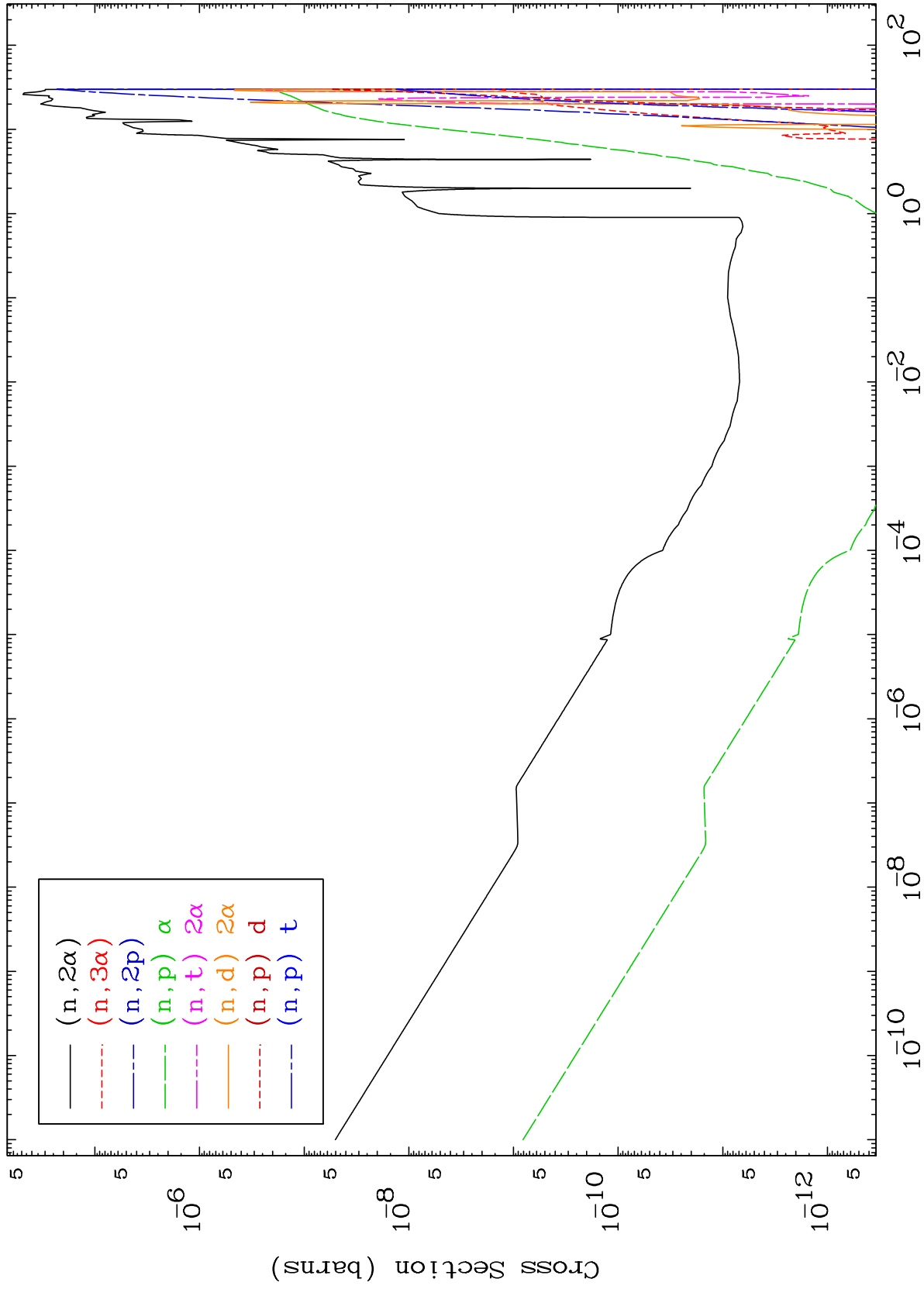
107-Bh-264



MAT 764

293 Kelvin Cross Sections

107-Bh-264



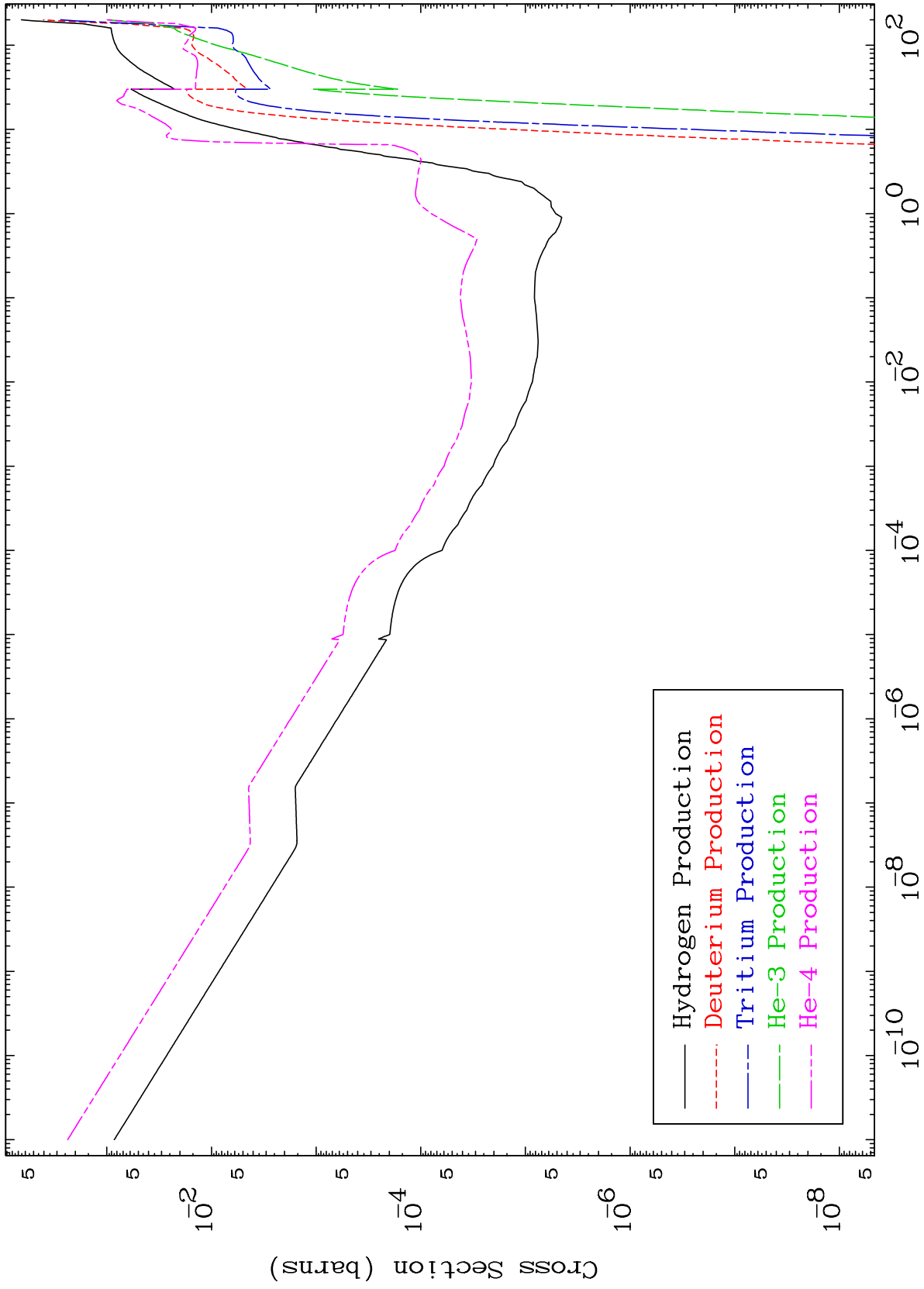
6

107-Bh-264

MAT 764

Particle Production  
293 Kelvin Cross Sections

107-Bh-264

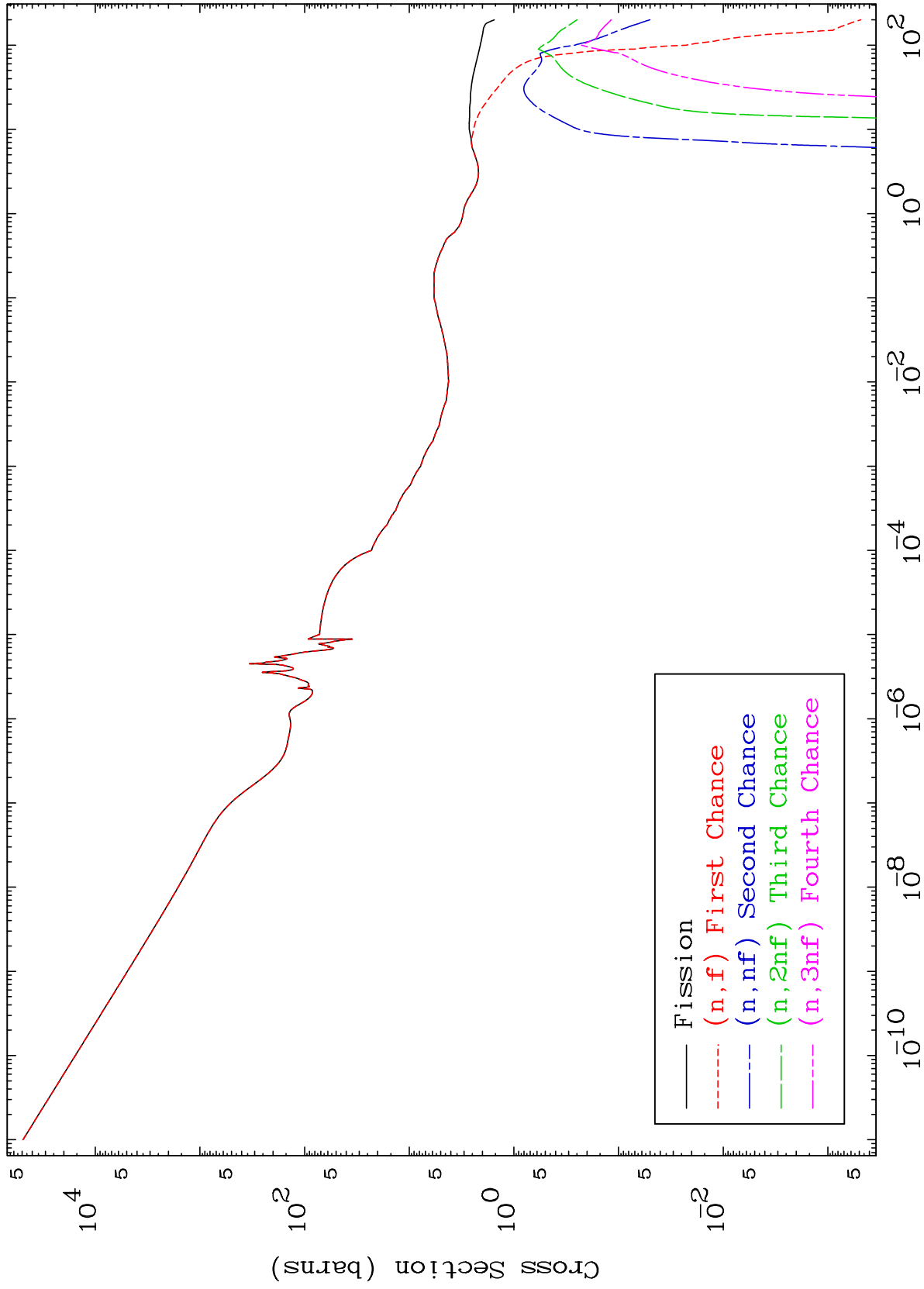




MAT 764

Fission  
293 Kelvin Cross Sections

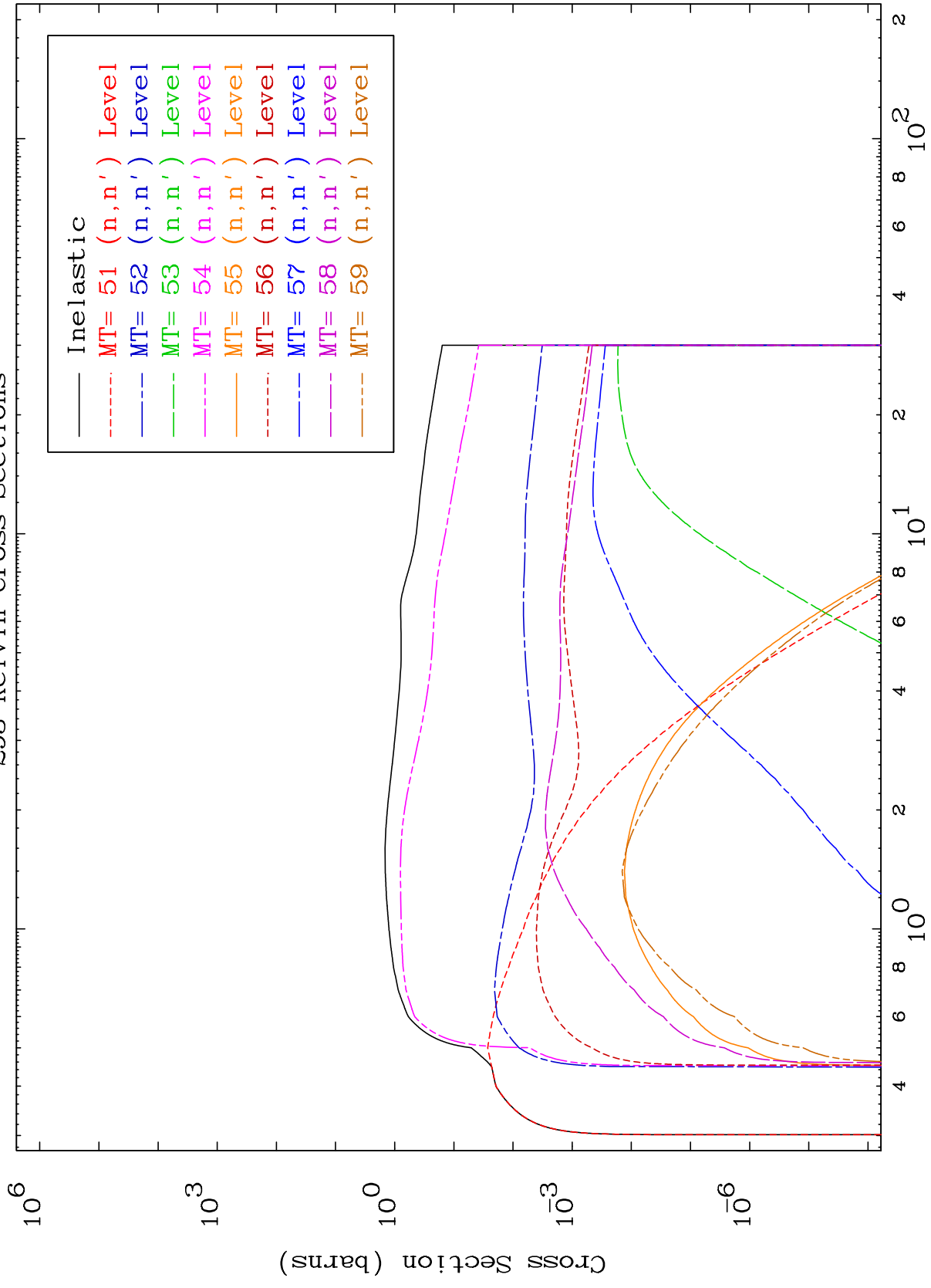
107-Bh-264



MAT 764

(n,n') Level  
293 Kelvin Cross Sections

107-Bh-264

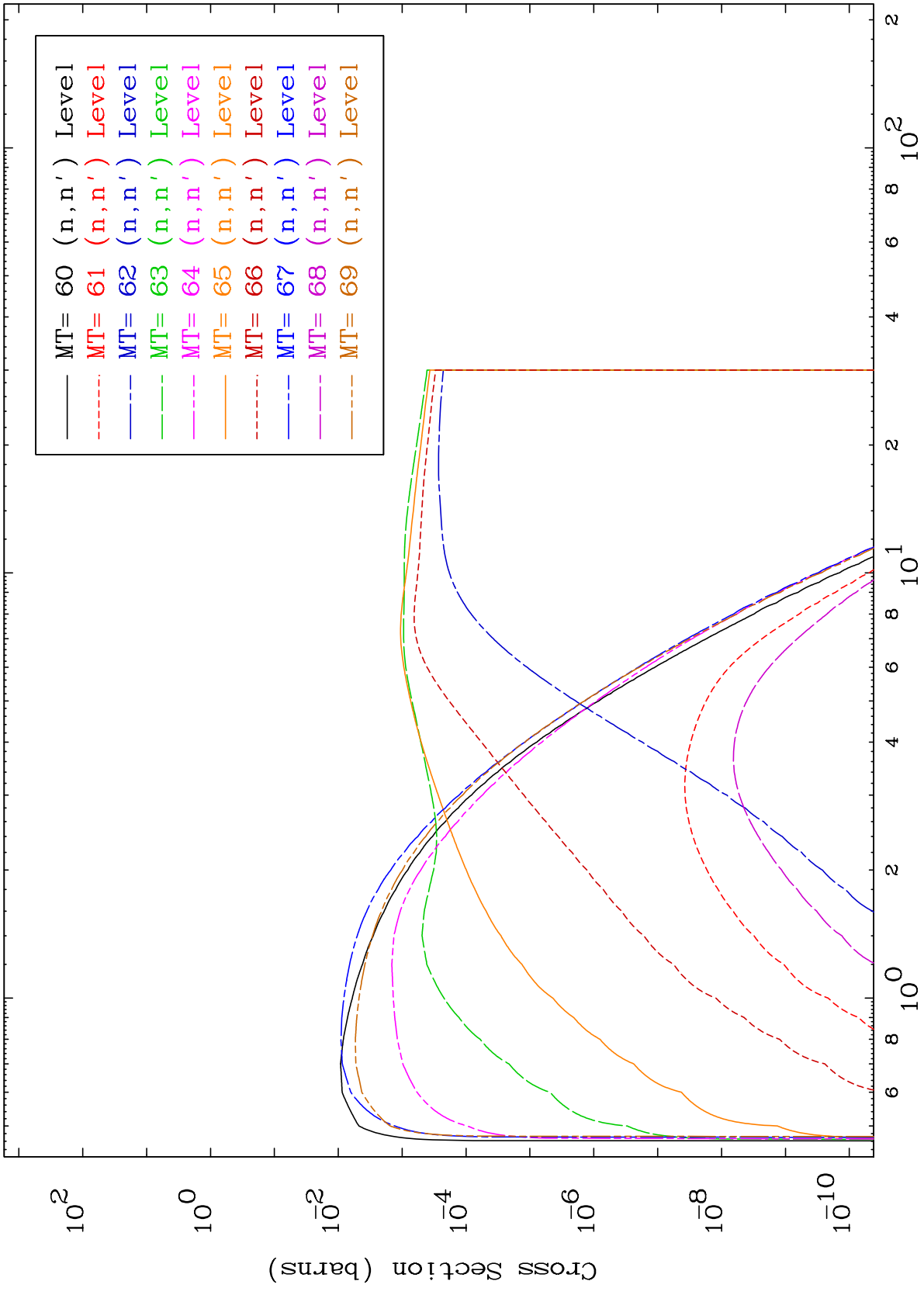


MAT 764

(n,n') Level

107-Bh-264

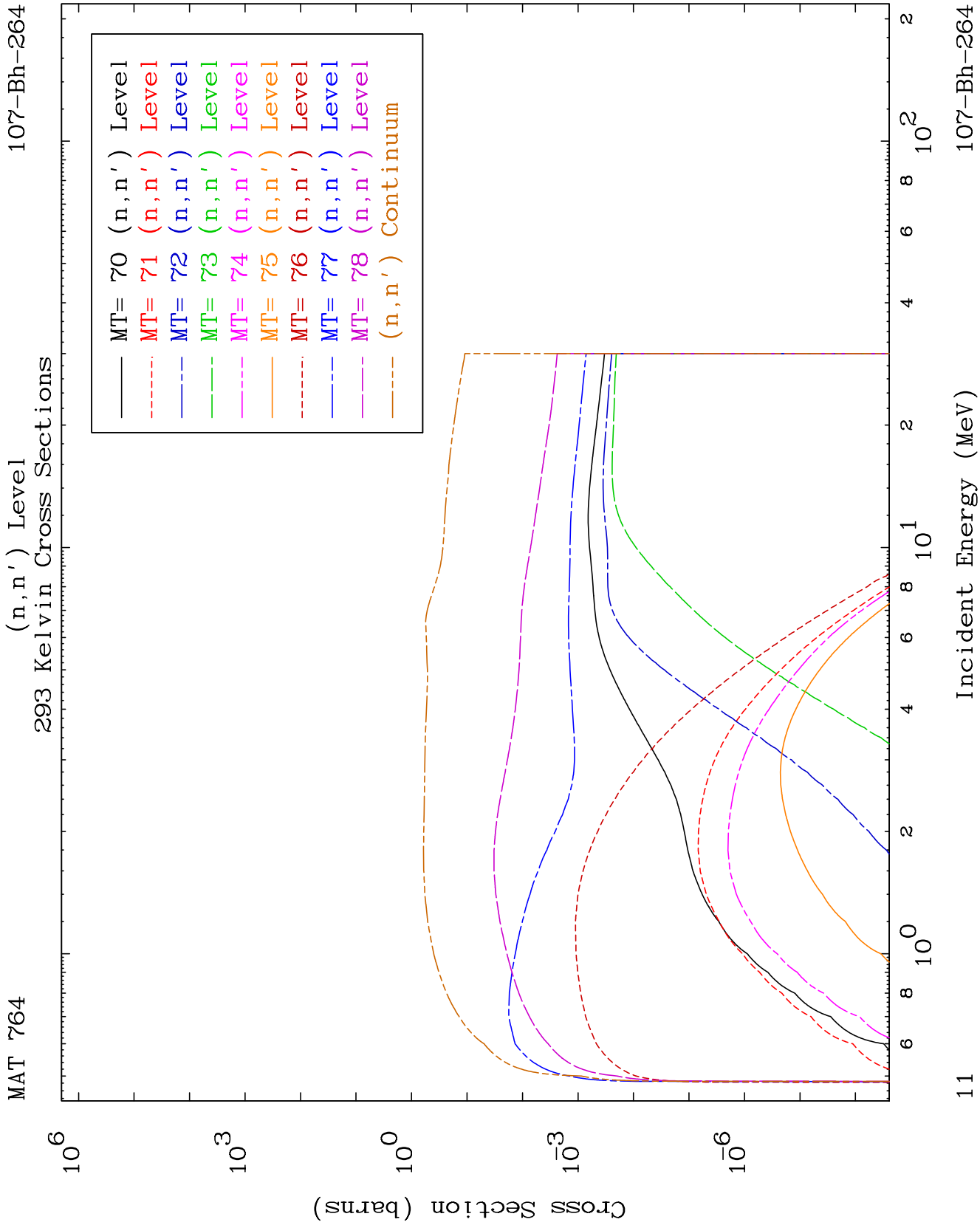
293 Kelvin Cross Sections

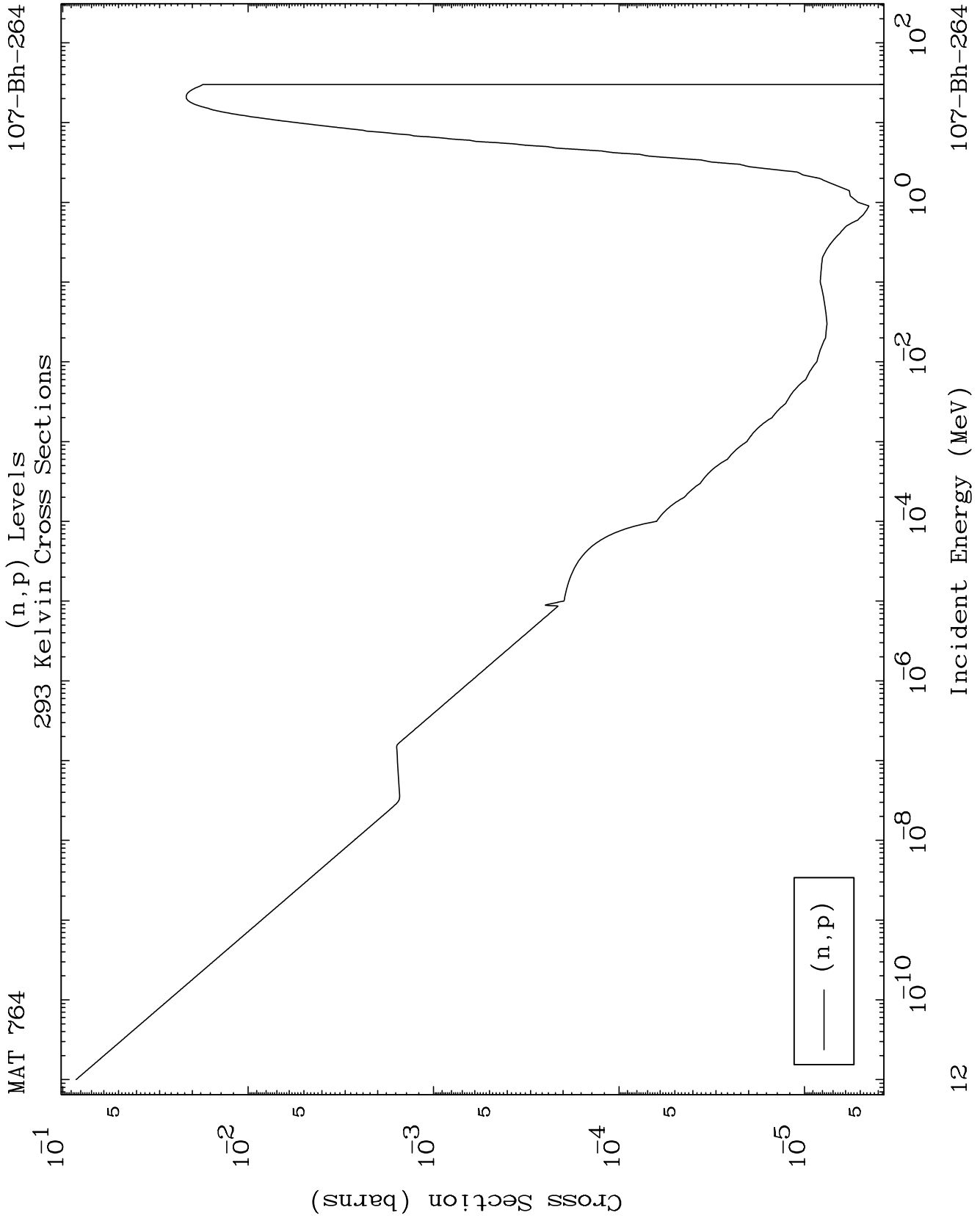


10

Incident Energy (MeV)

107-Bh-264

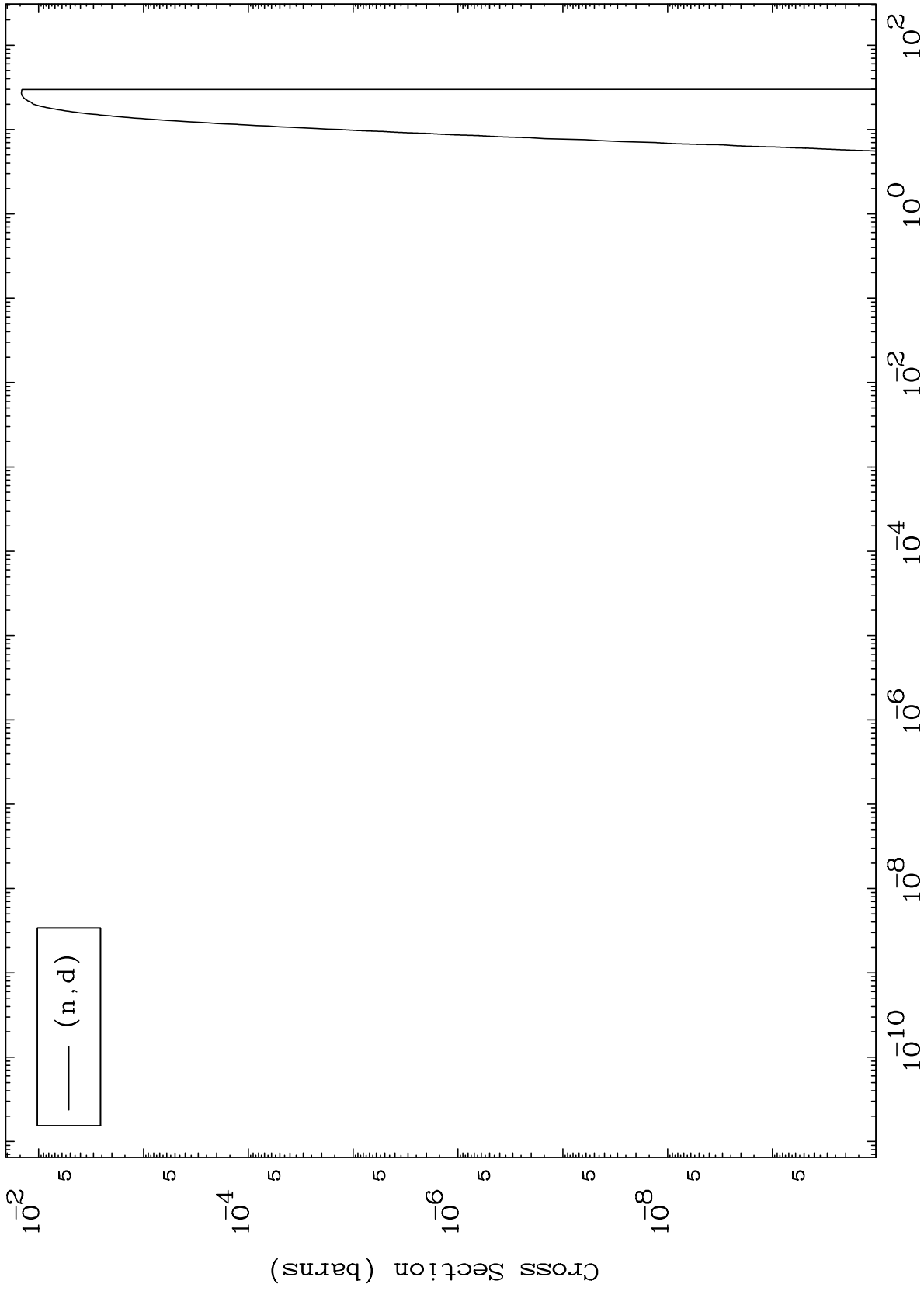




MAT 764

(n,d) Levels  
293 Kelvin Cross Sections

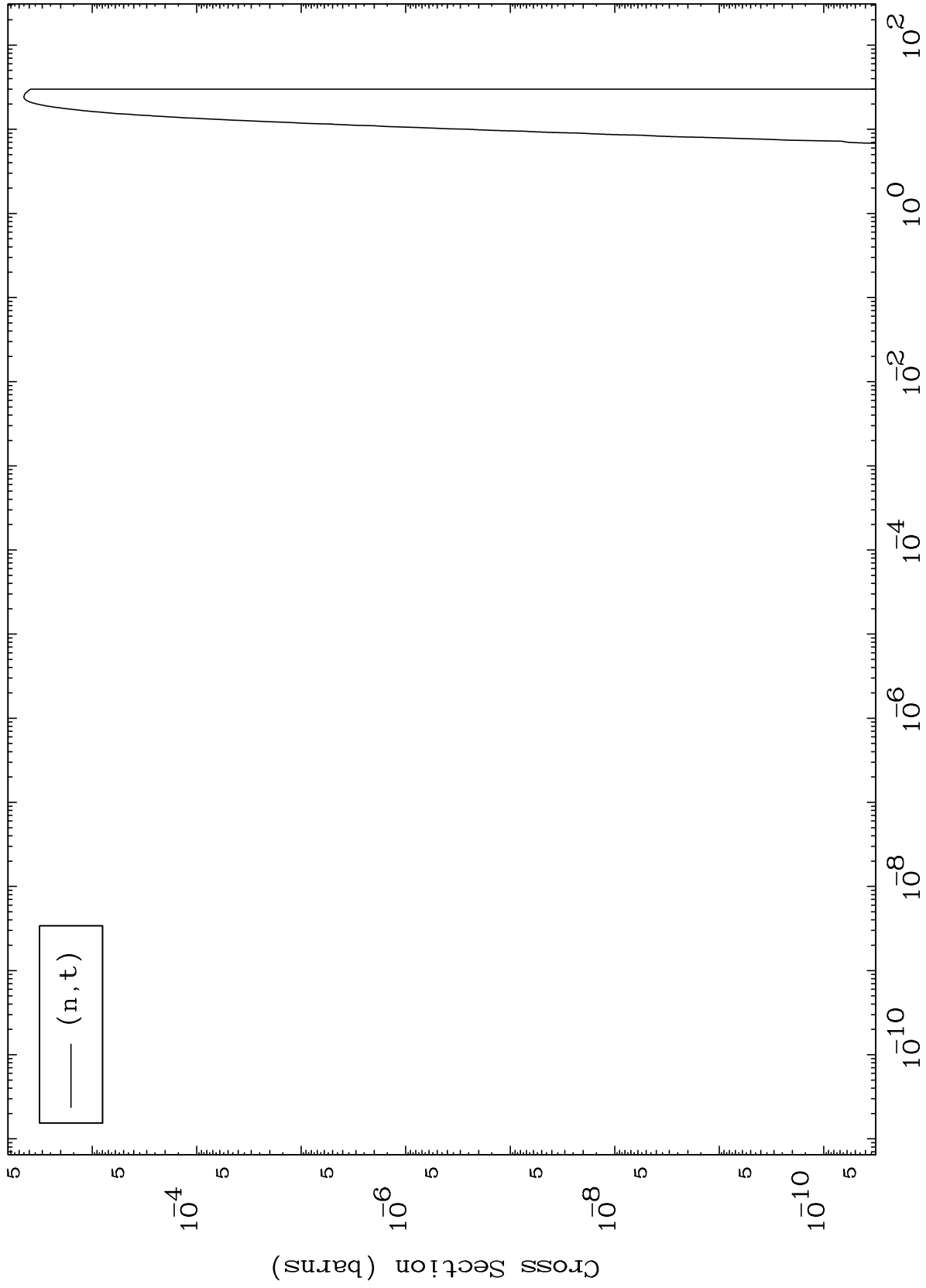
107-Bh-264



MAT 764

(n,t) Levels  
293 Kelvin Cross Sections

107-Bh-264



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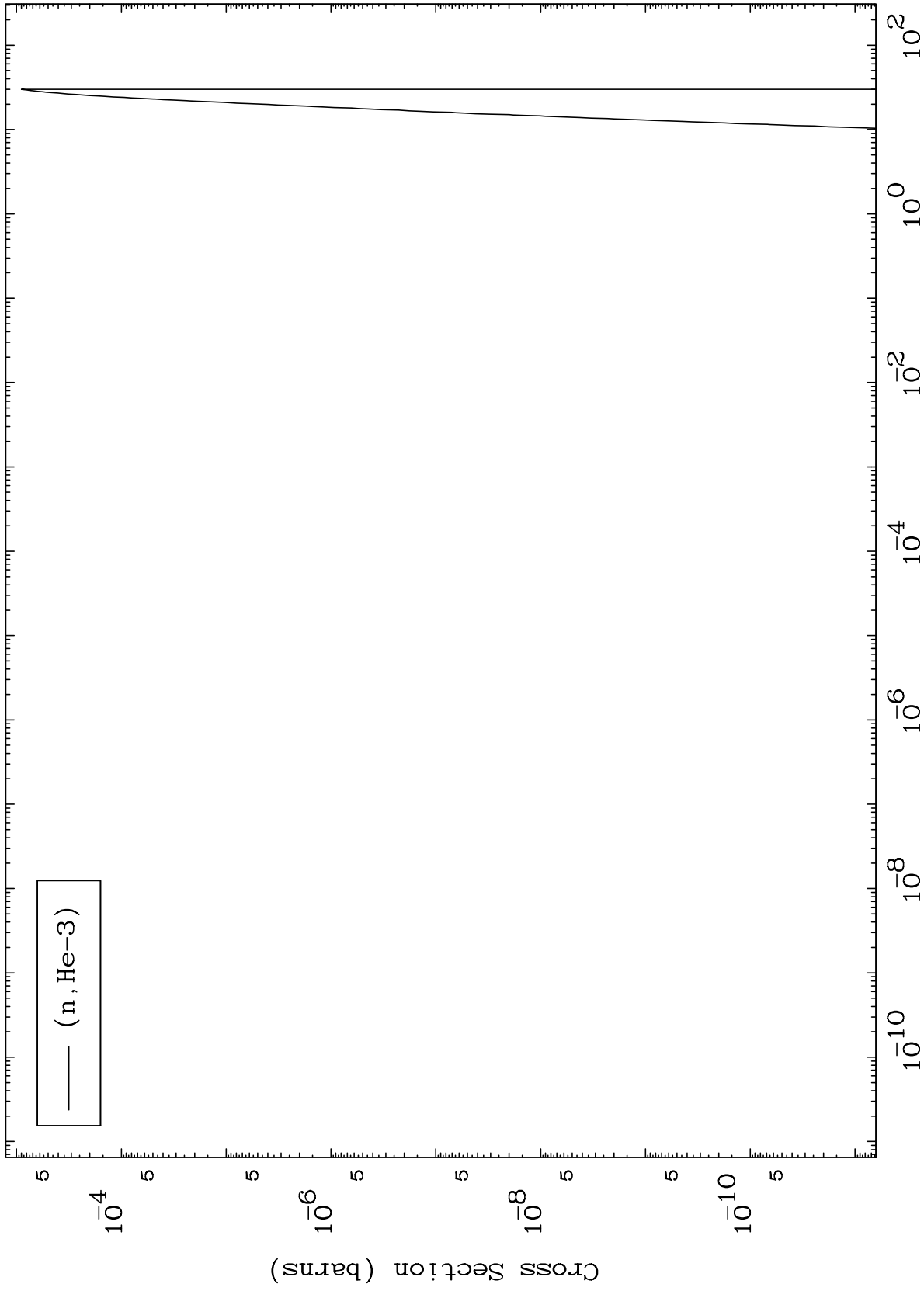
Incident Energy (MeV)

107-Bh-264

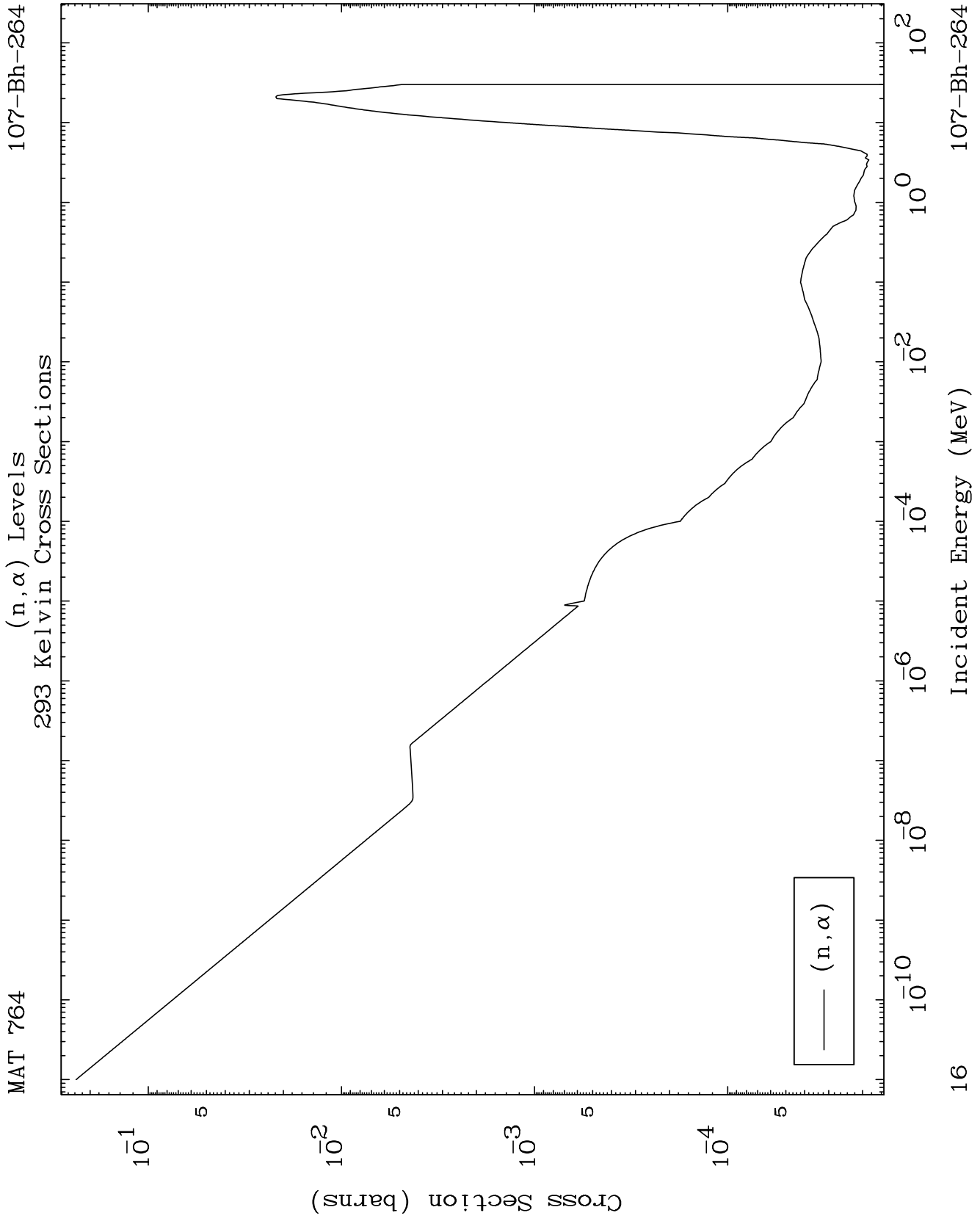
MAT 764

(n,He3) Levels  
293 Kelvin Cross Sections

107-Bh-264



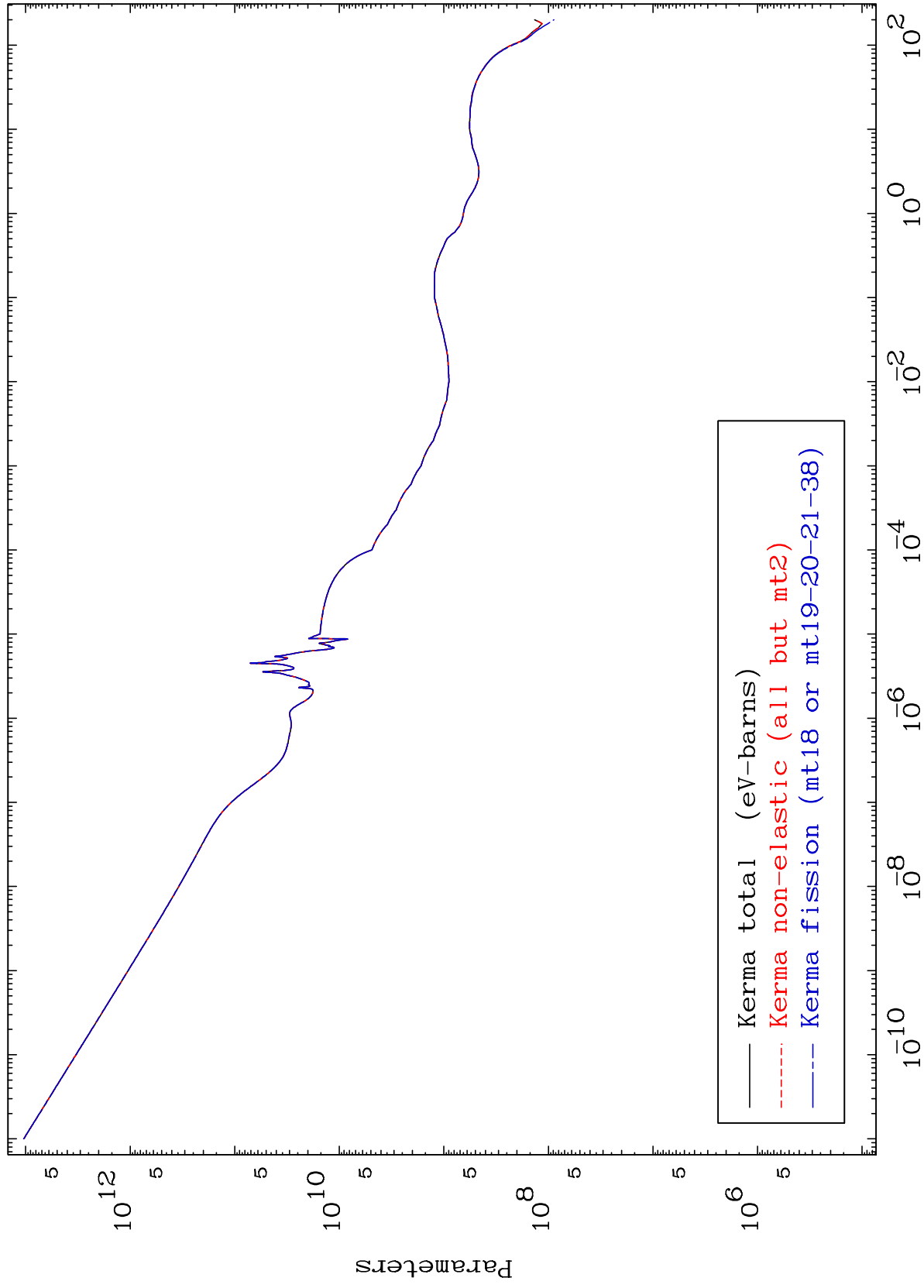




MAT 764

Energy Release  
Parameters

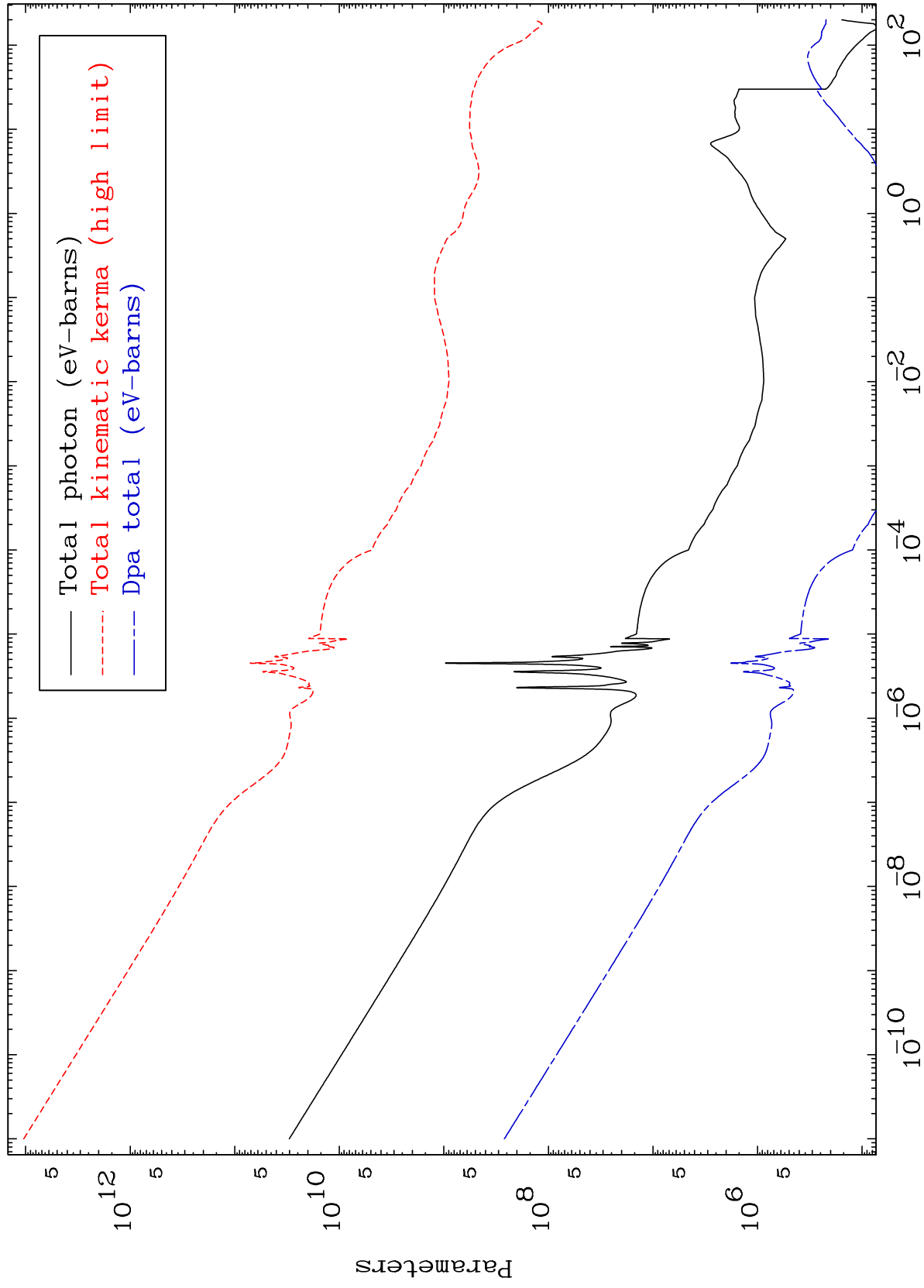
107-Bh-264



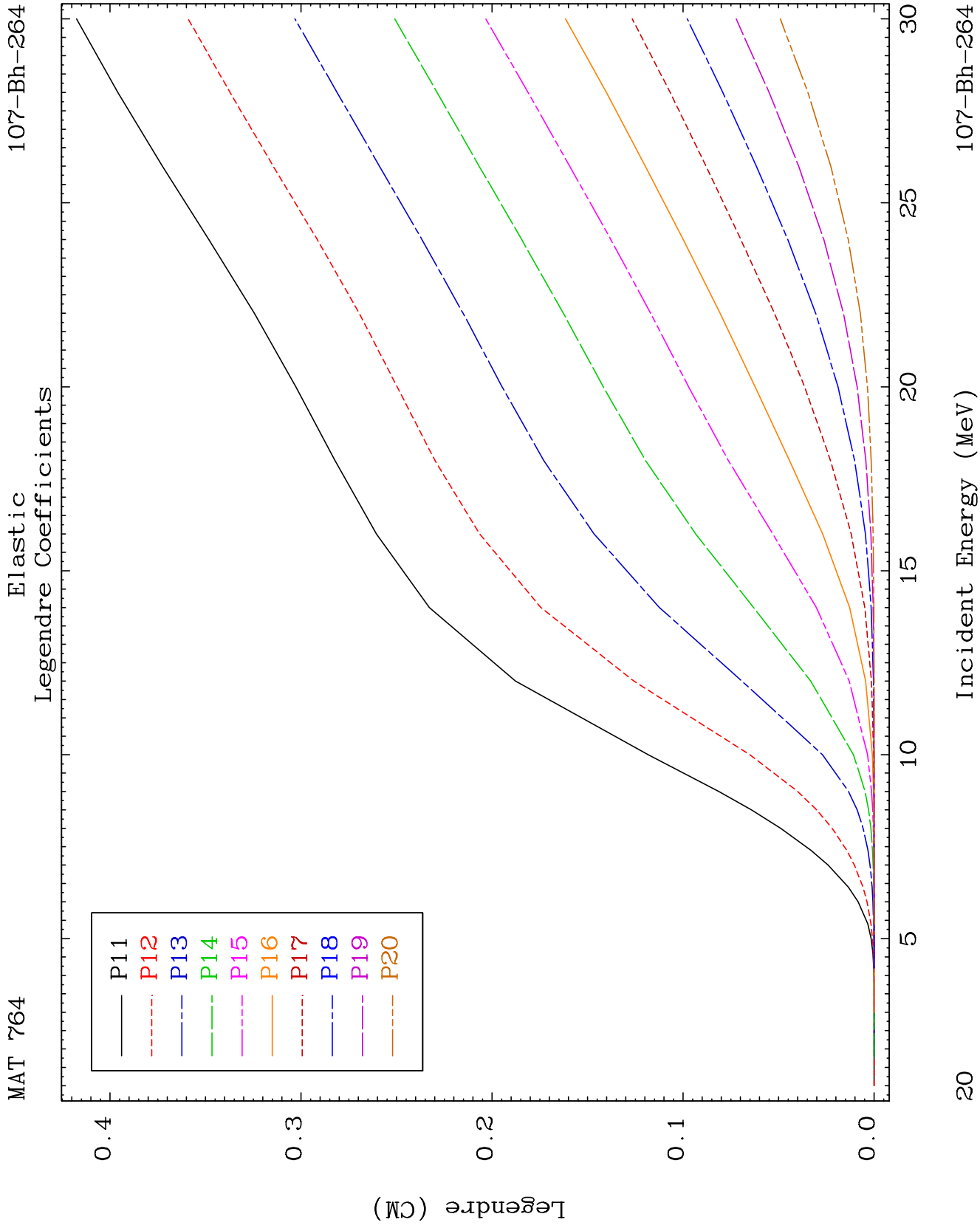
17

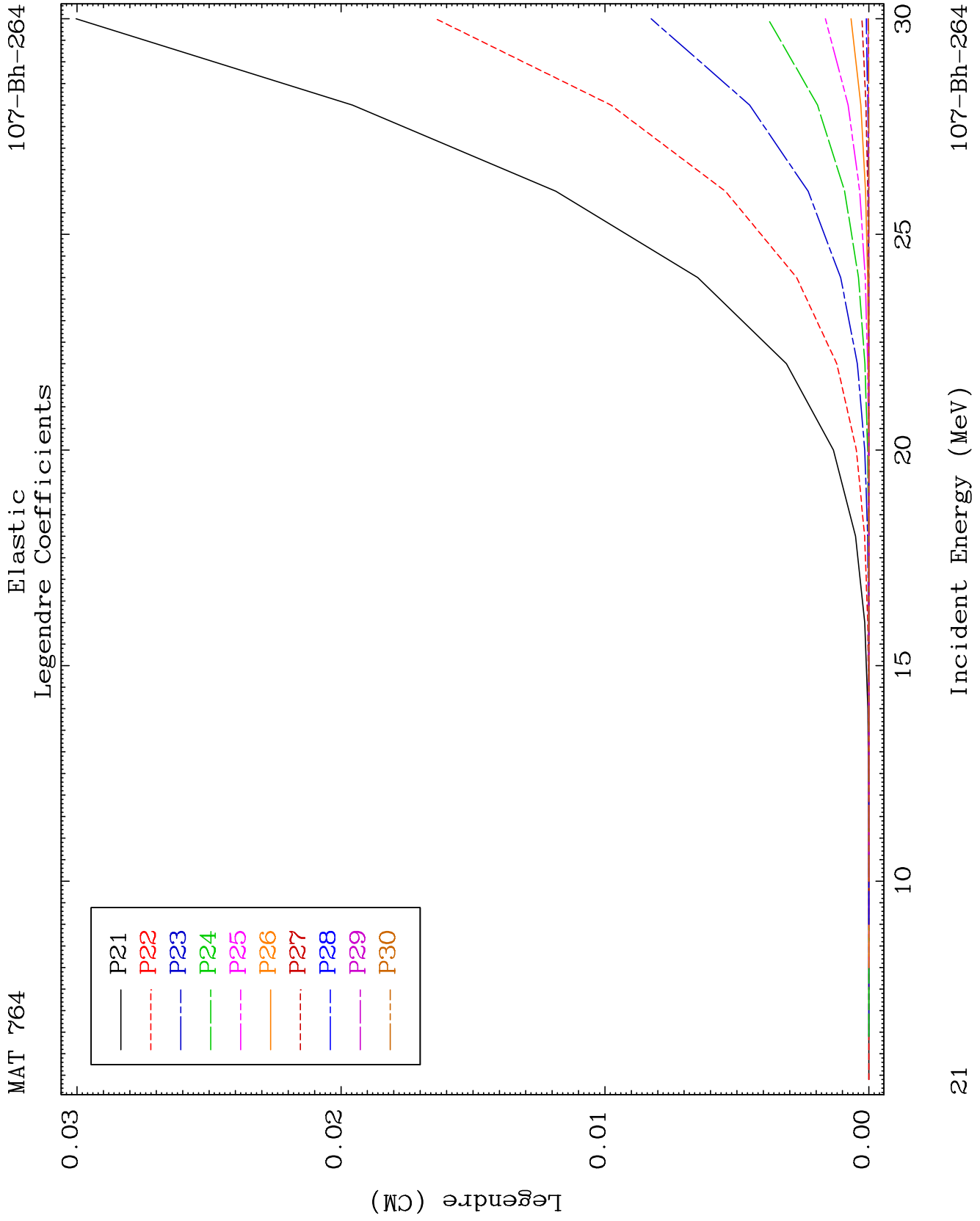
Incident Energy (MeV)

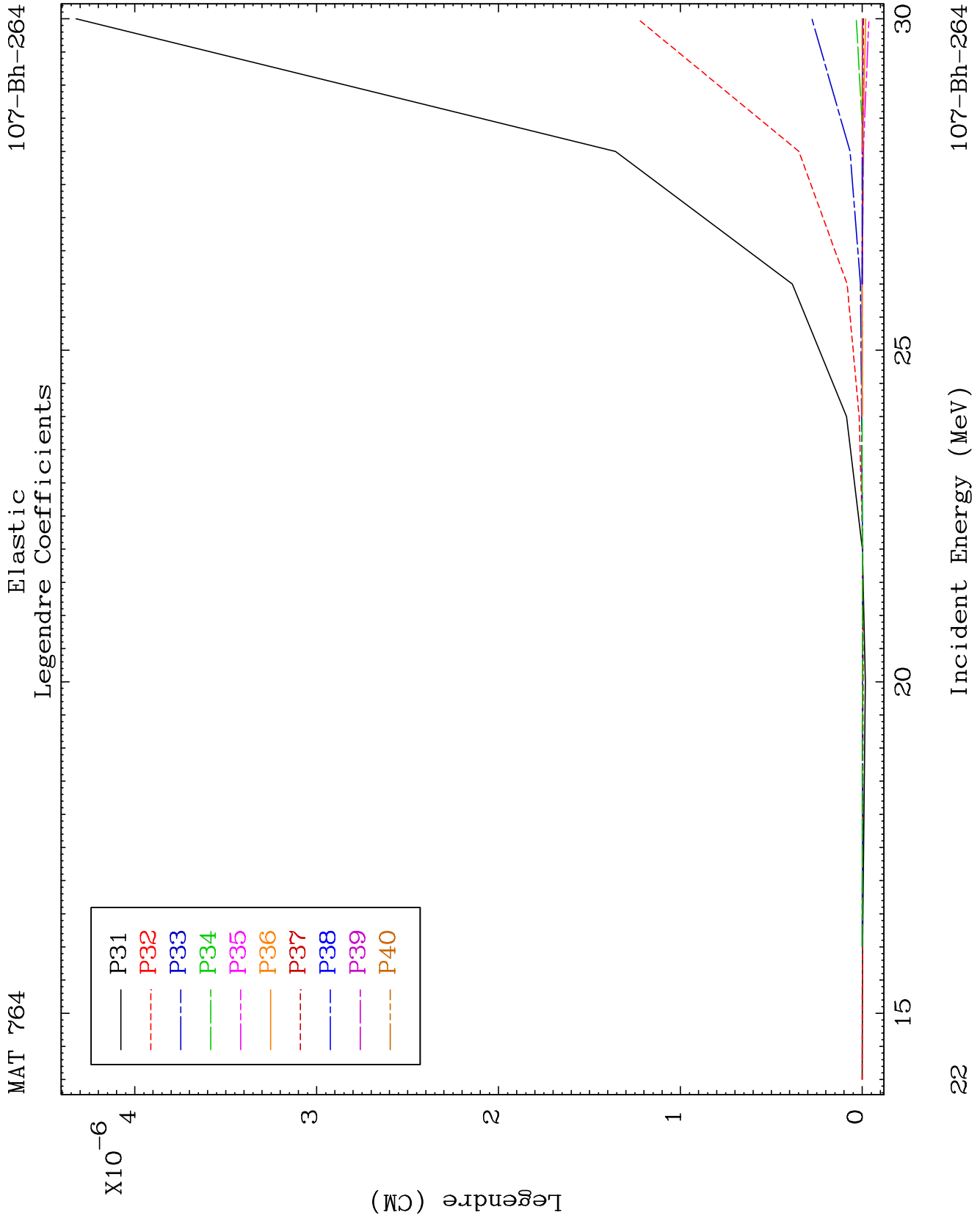
107-Bh-264







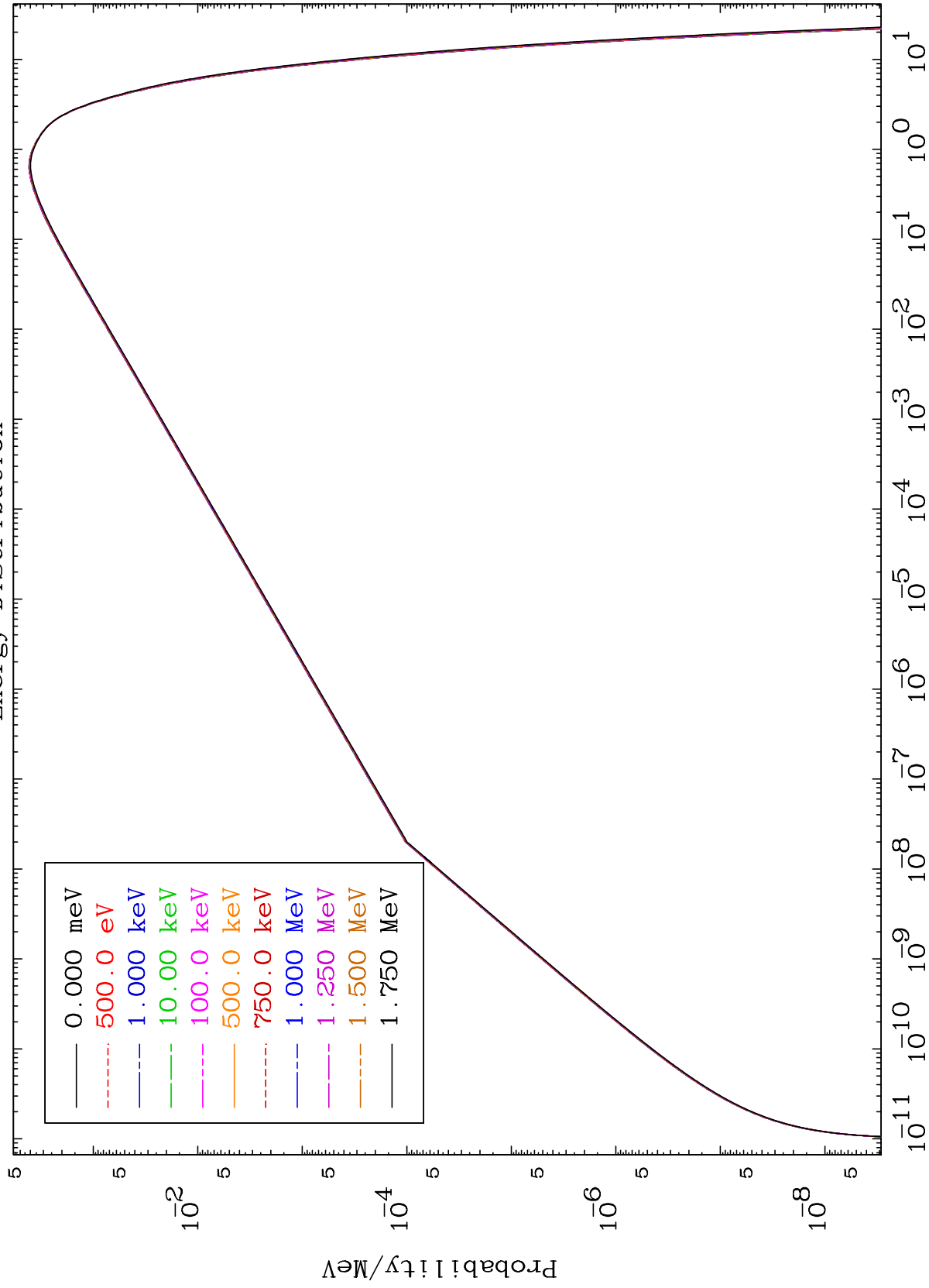




MAT 764

### Fission Energy Distribution

<sup>107</sup>Bh-264



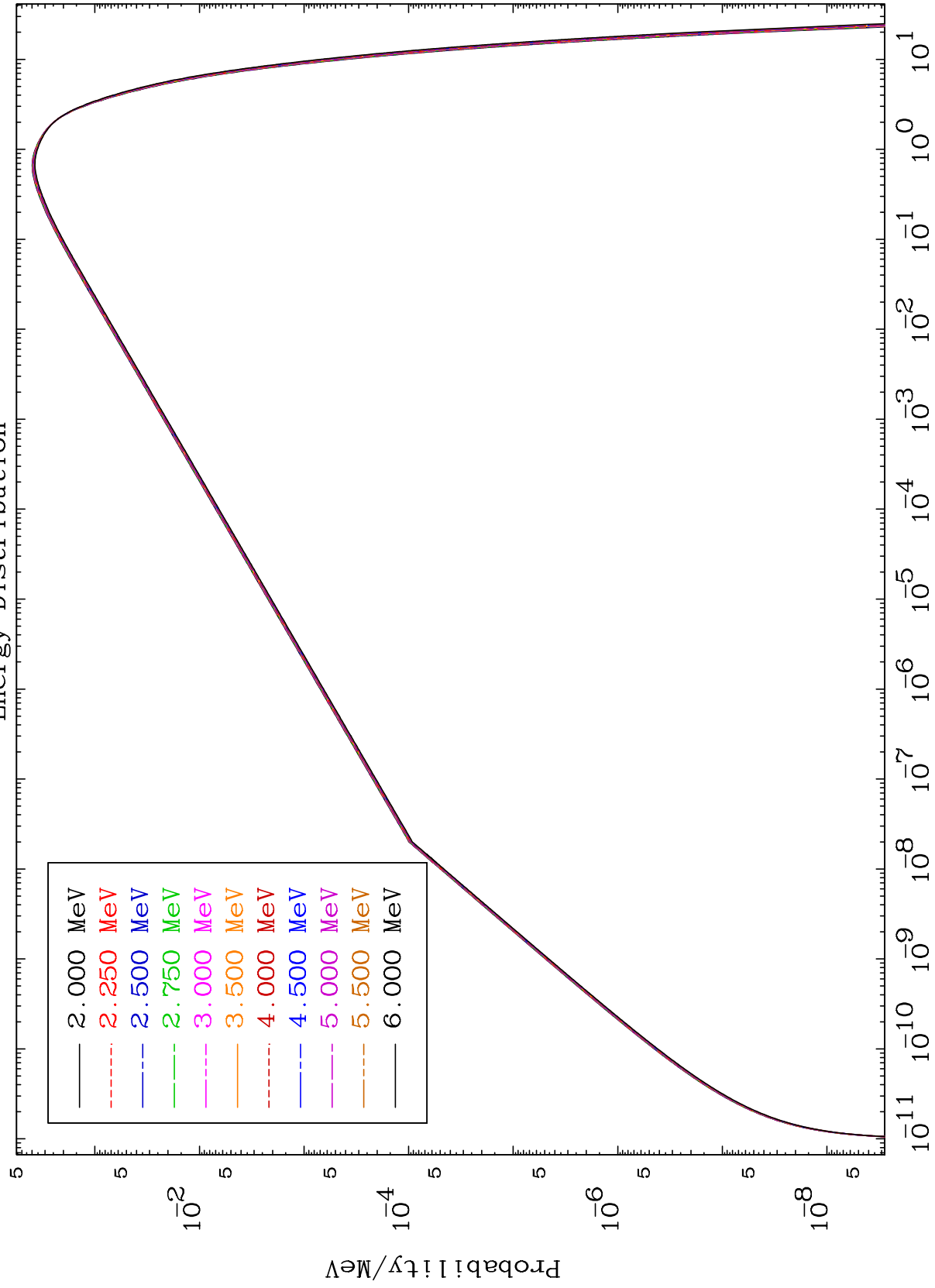
<sup>107</sup>Bh-264



MAT 764

# Fission Energy Distribution

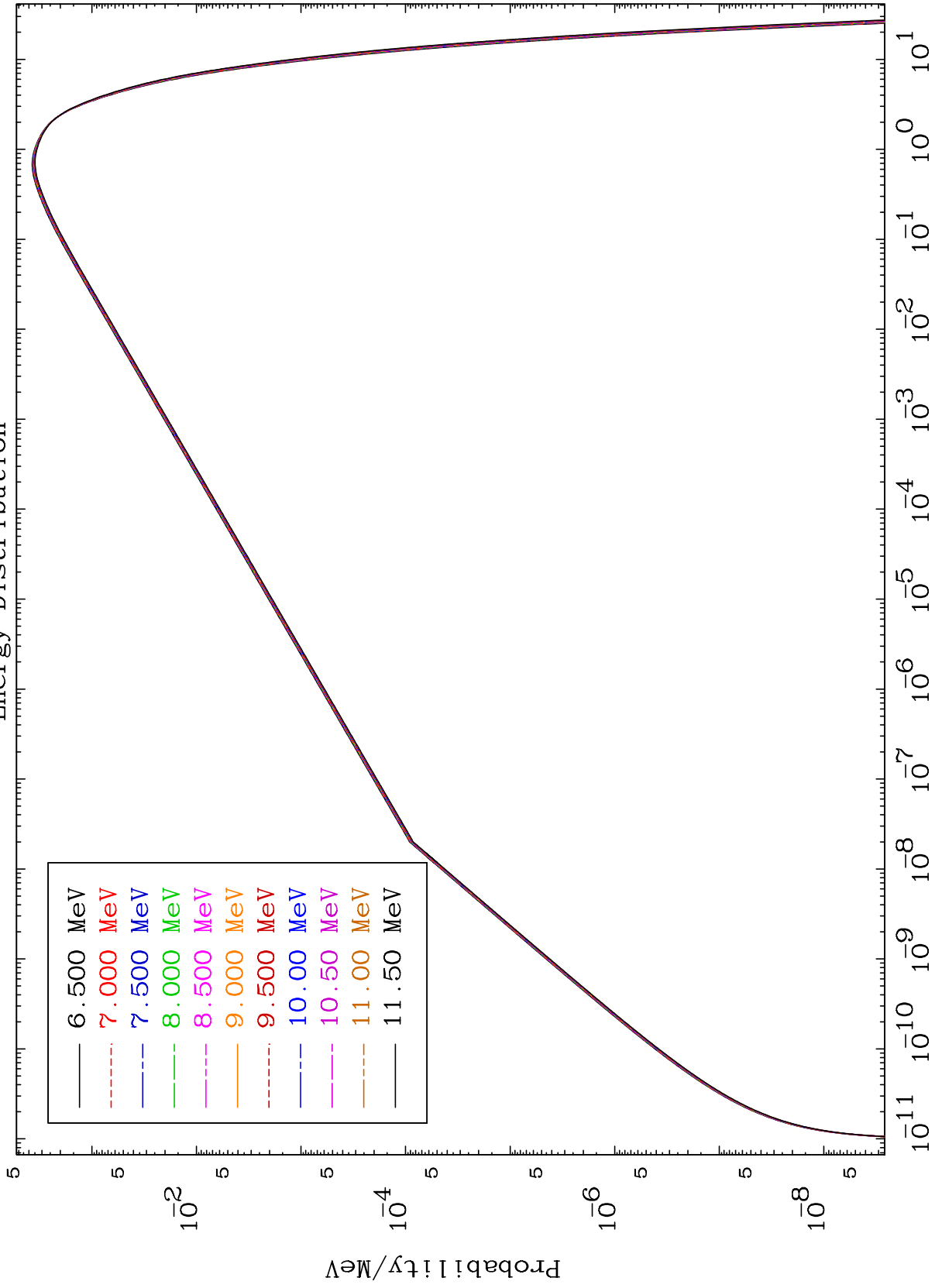
$^{107}\text{Bh-264}$



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# Fission Energy Distribution

107-Bh-264

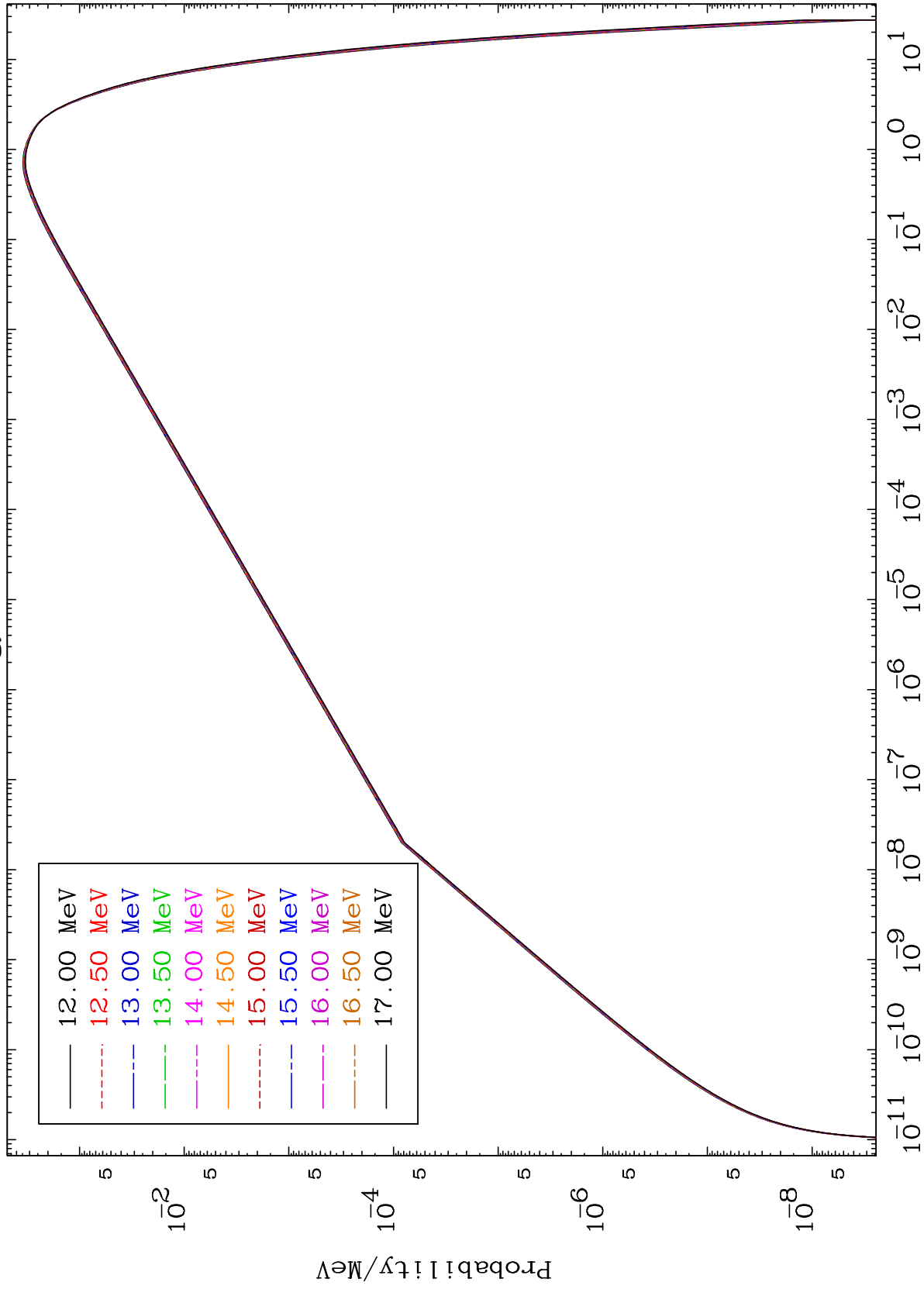


107-Bh-264

MAT 764

# Fission Energy Distribution

$^{107}\text{Bh-264}$



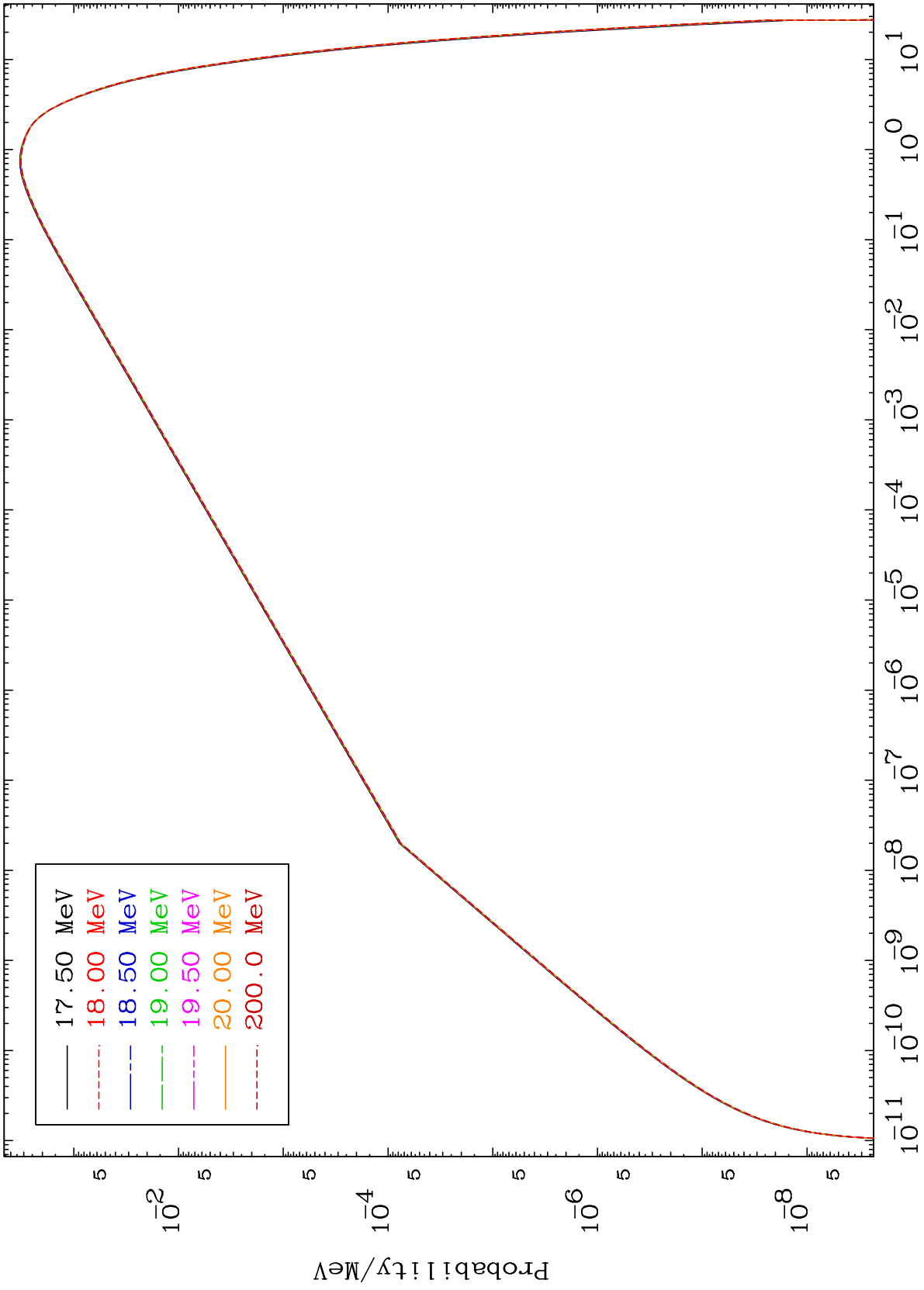
26

$^{107}\text{Bh-264}$

MAT 764

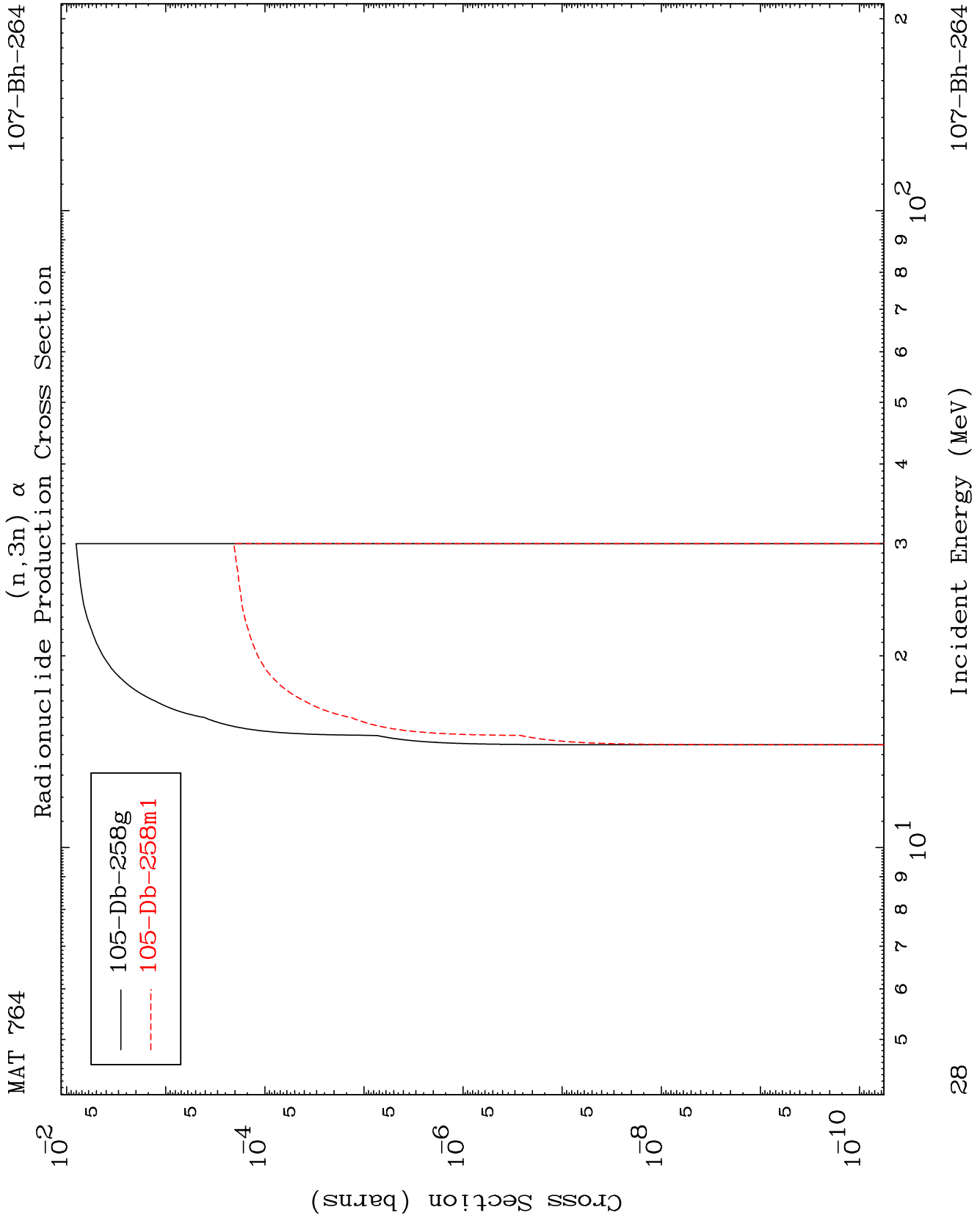
# Fission Energy Distribution

107-Bh-264



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107-Bh-264



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(n,2n) 2α

107-Bh-264

Radionuclide Production Cross Section

