

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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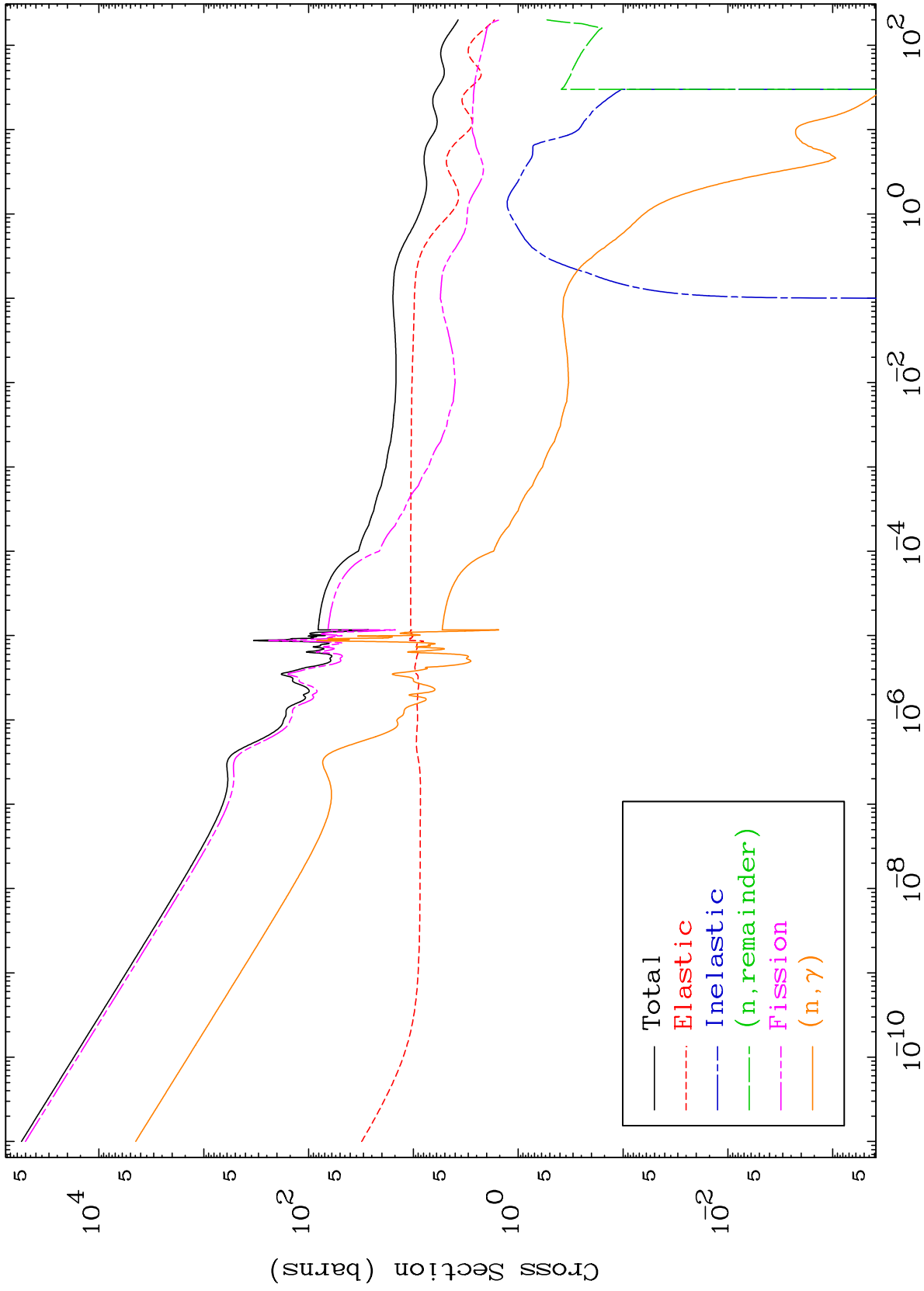
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 766

Major  
293 Kelvin Cross Sections

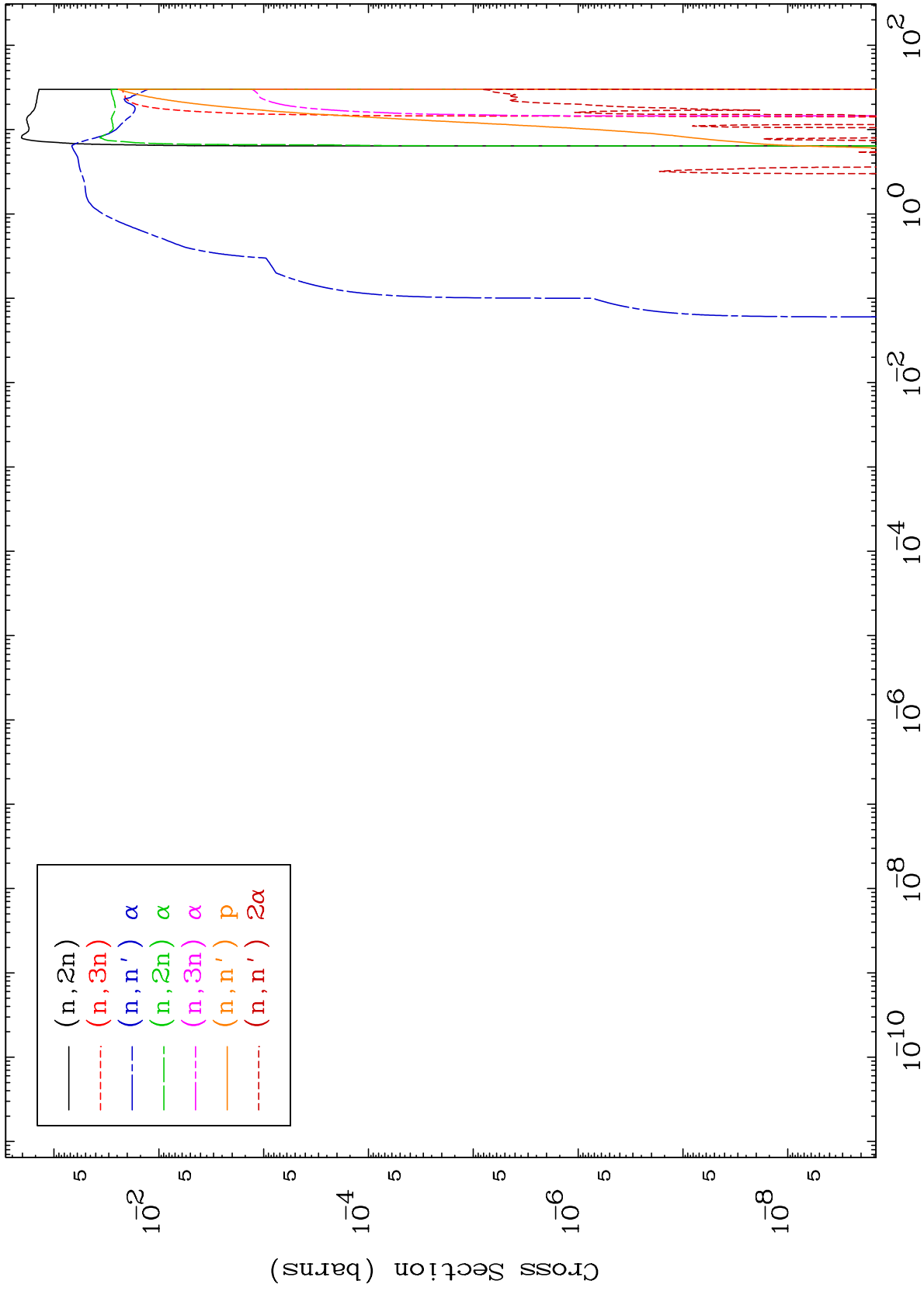
107-Bh-266



MAT 766

Neutron Production  
293 Kelvin Cross Sections

107-Bh-266

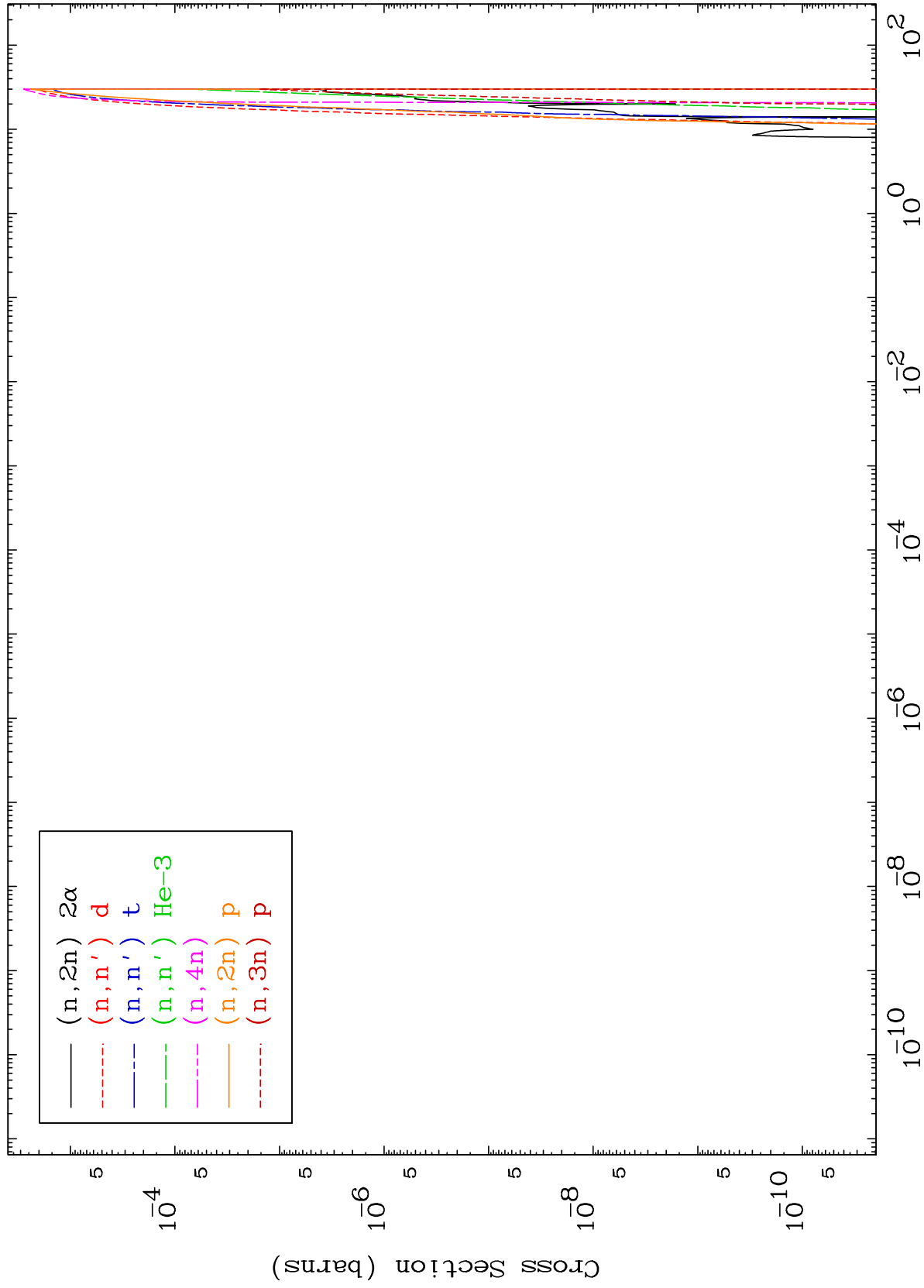


107-Bh-266

MAT 766

Neutron Production  
293 Kelvin Cross Sections

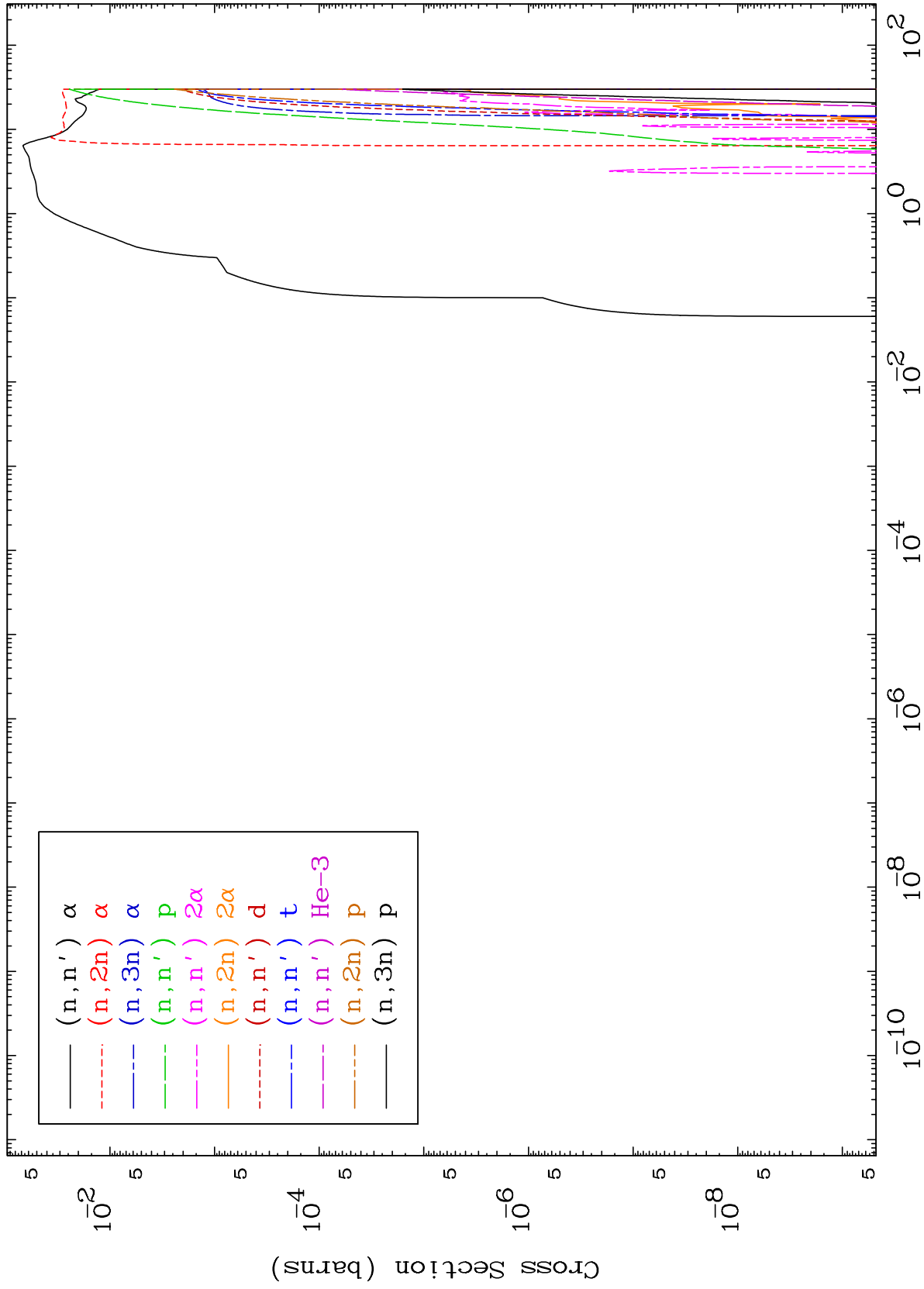
107-Bh-266



MAT 766

Charged Particle  
293 Kelvin Cross Sections

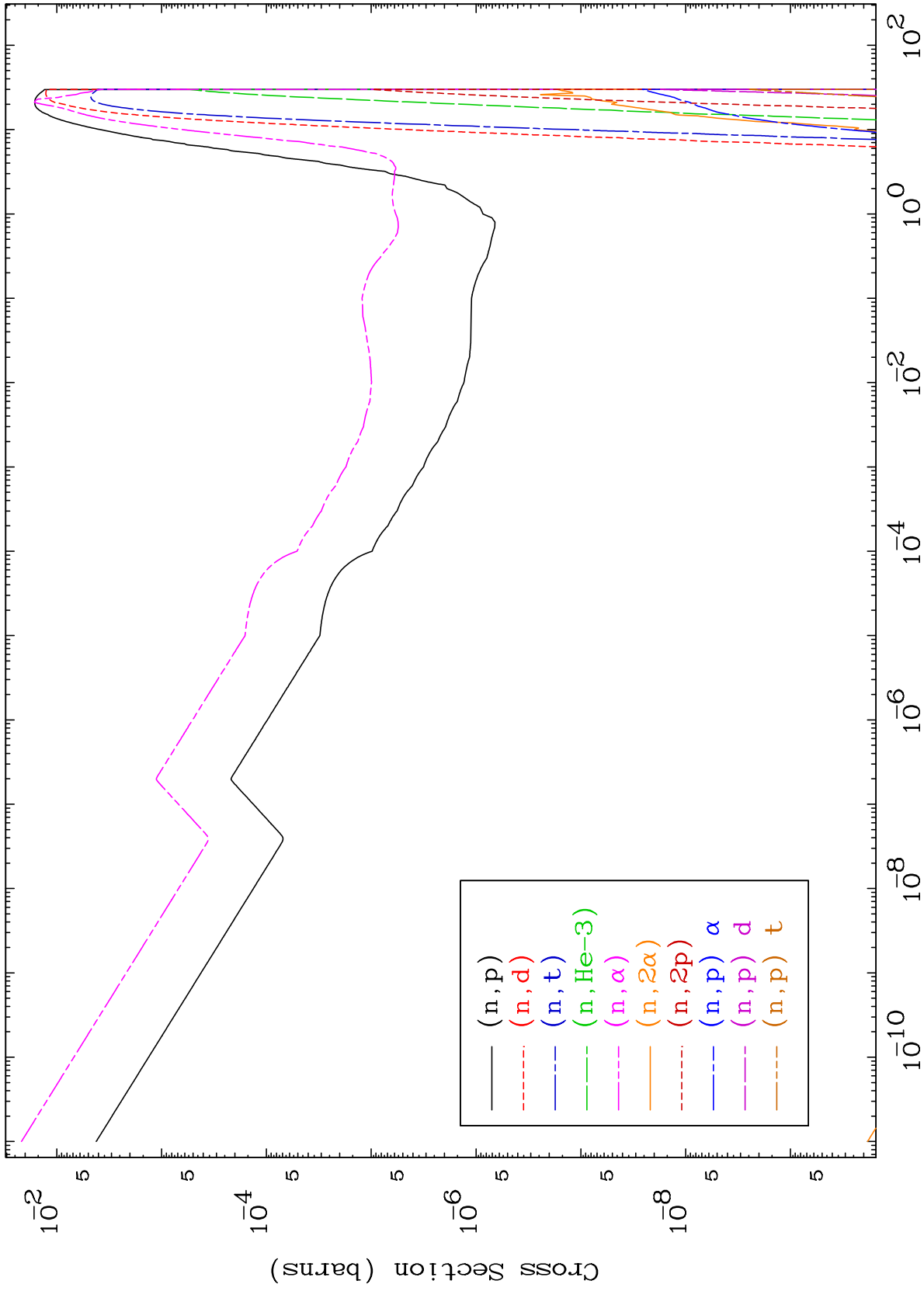
107-Bh-266



MAT 766

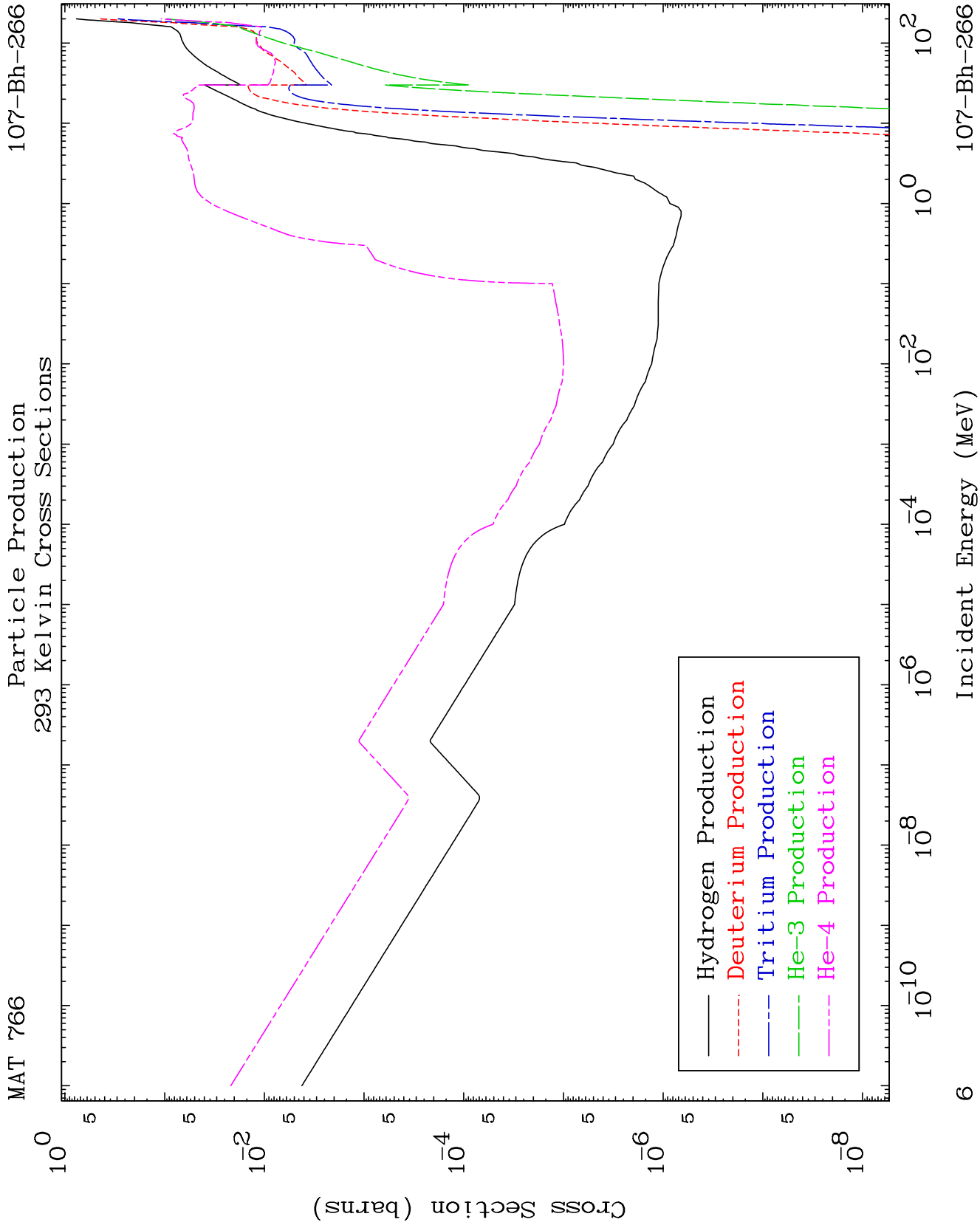
Charged Particle  
293 Kelvin Cross Sections

107-Bh-266



5

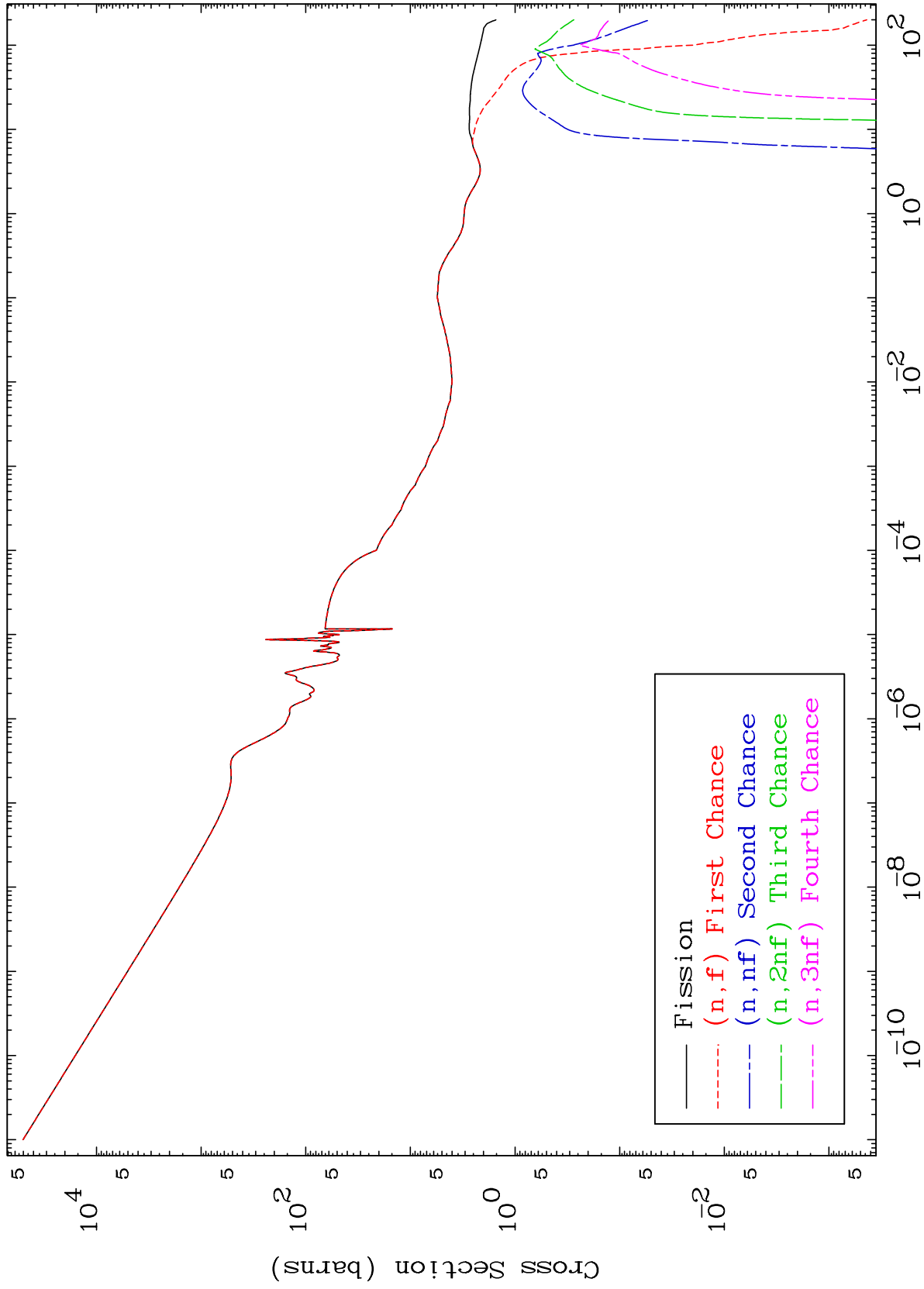
107-Bh-266



MAT 766

Fission  
293 Kelvin Cross Sections

107-Bh-266



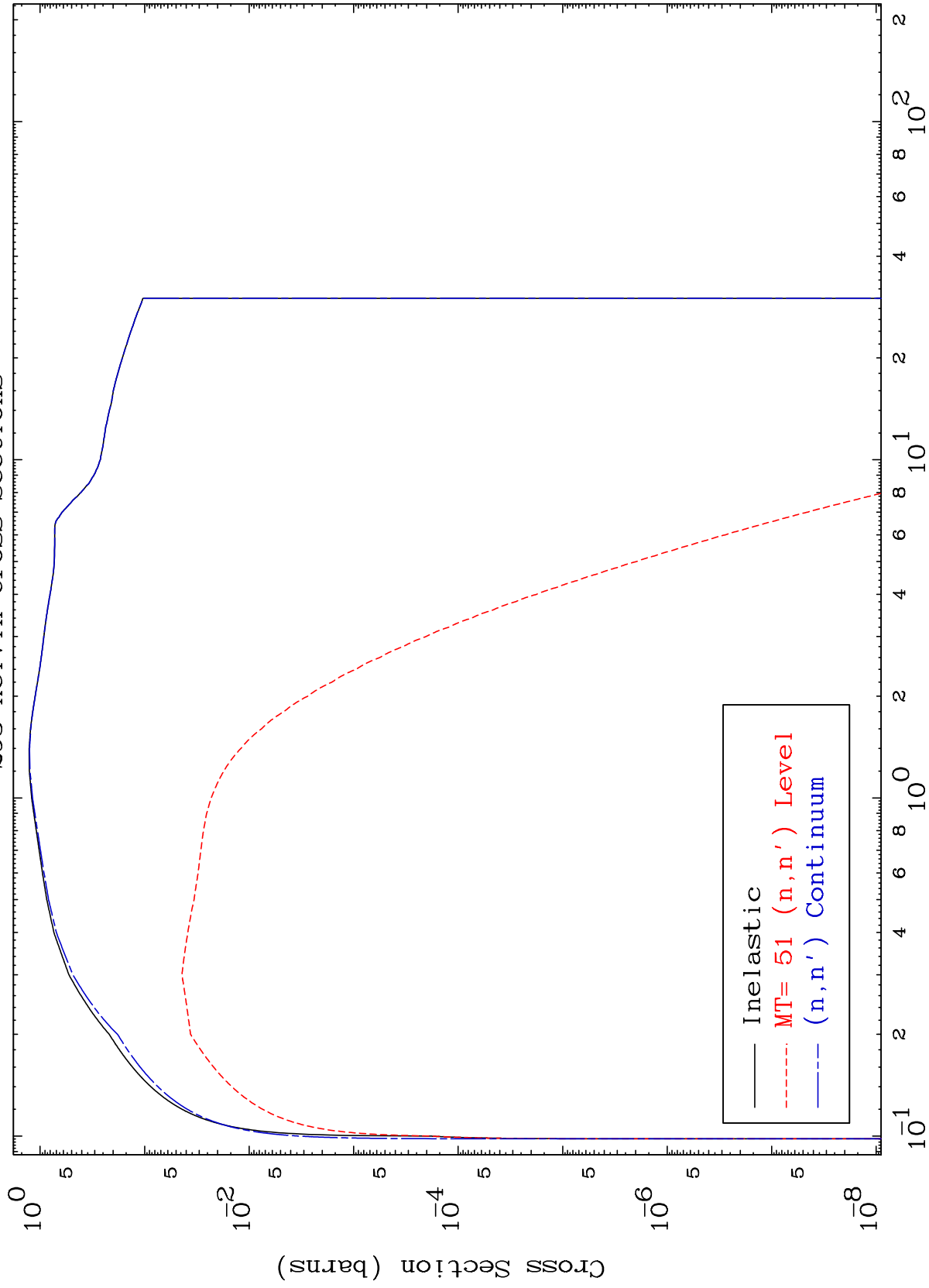
107-Bh-266



MAT 766

(n,n') Level  
293 Kelvin Cross Sections

107-Bh-266



8

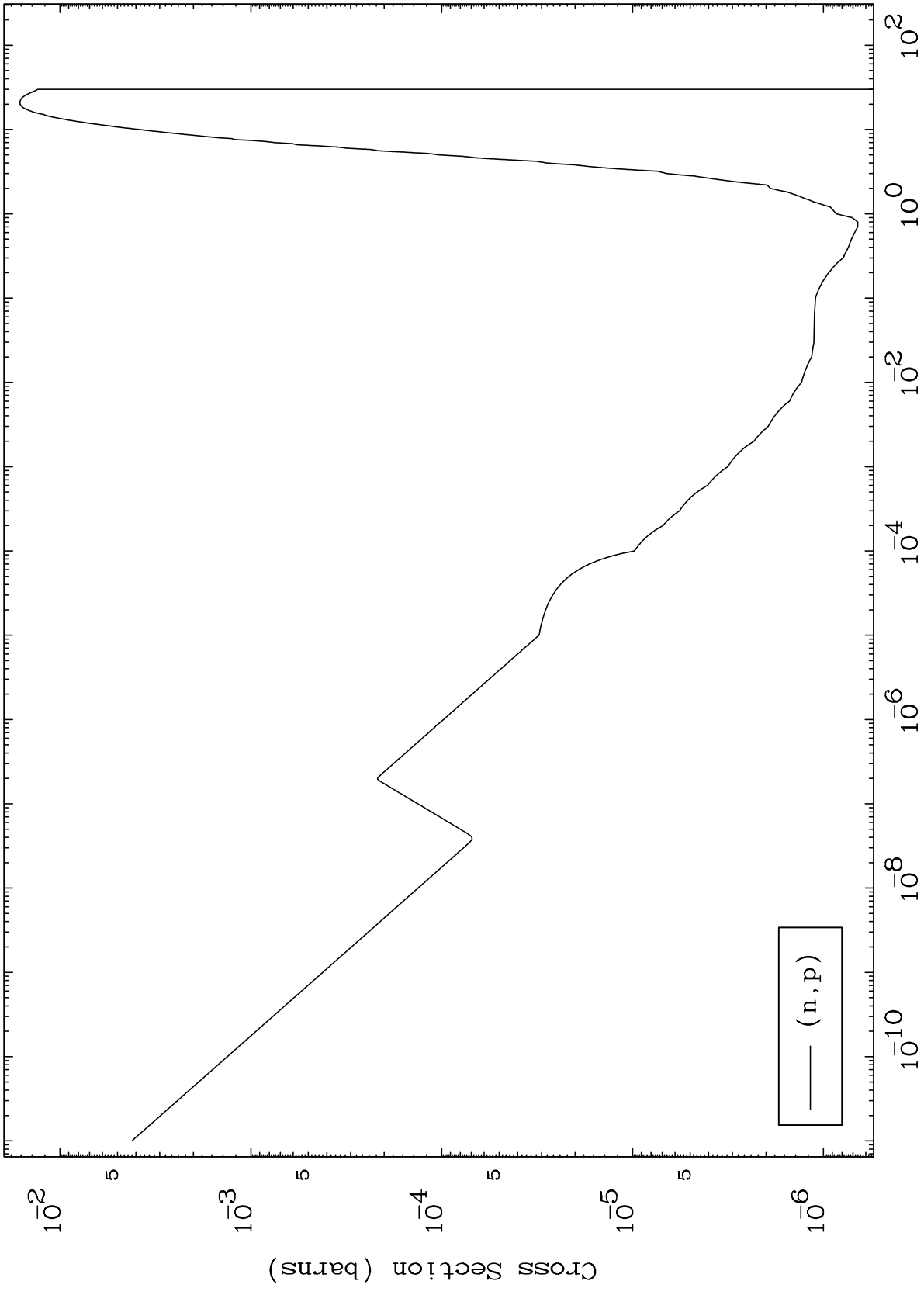
Incident Energy (MeV)

107-Bh-266

MAT 766

(n,p) Levels  
293 Kelvin Cross Sections

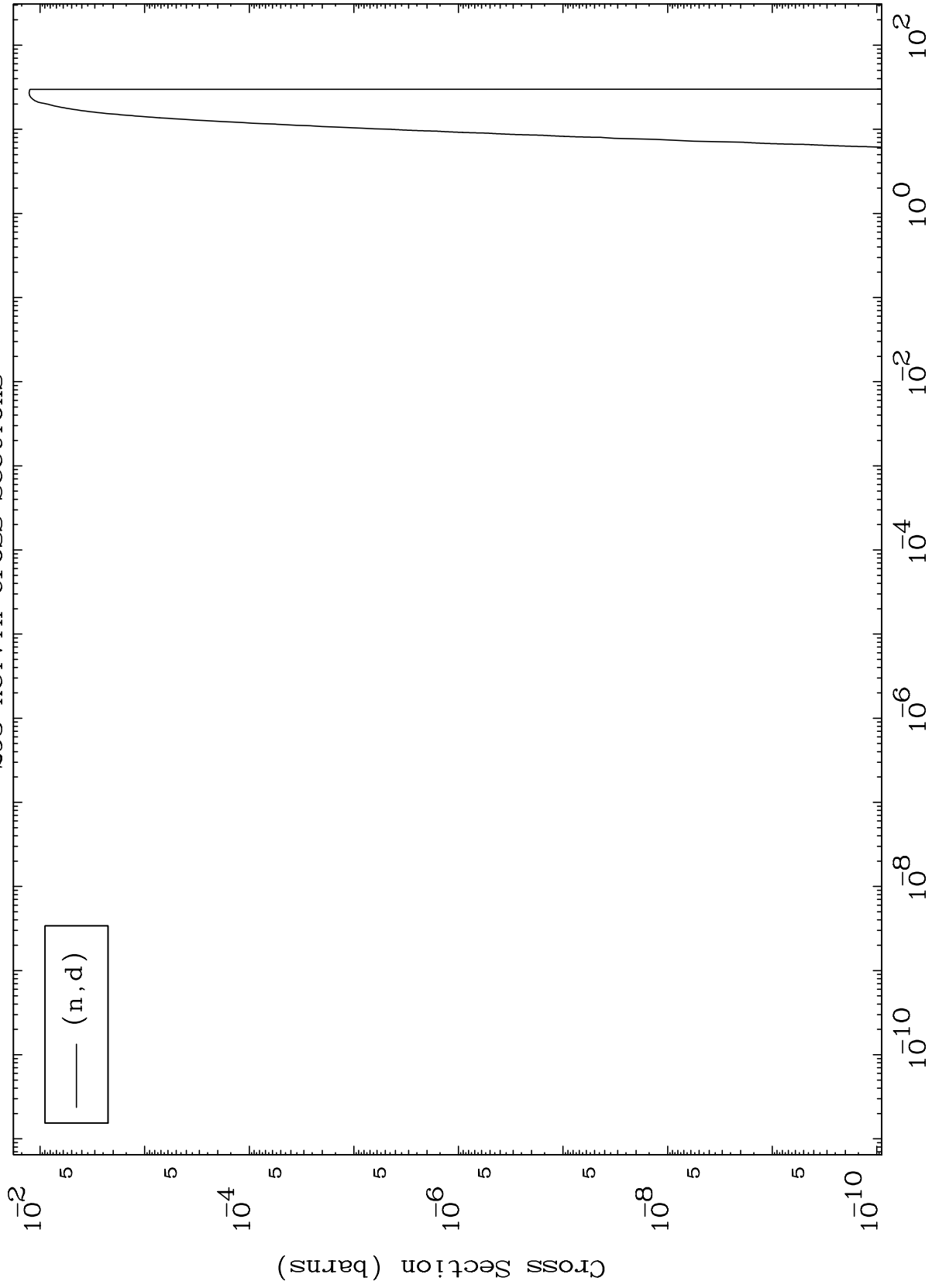
107-Bh-266



MAT 766

(n,d) Levels  
293 Kelvin Cross Sections

107-Bh-266



10

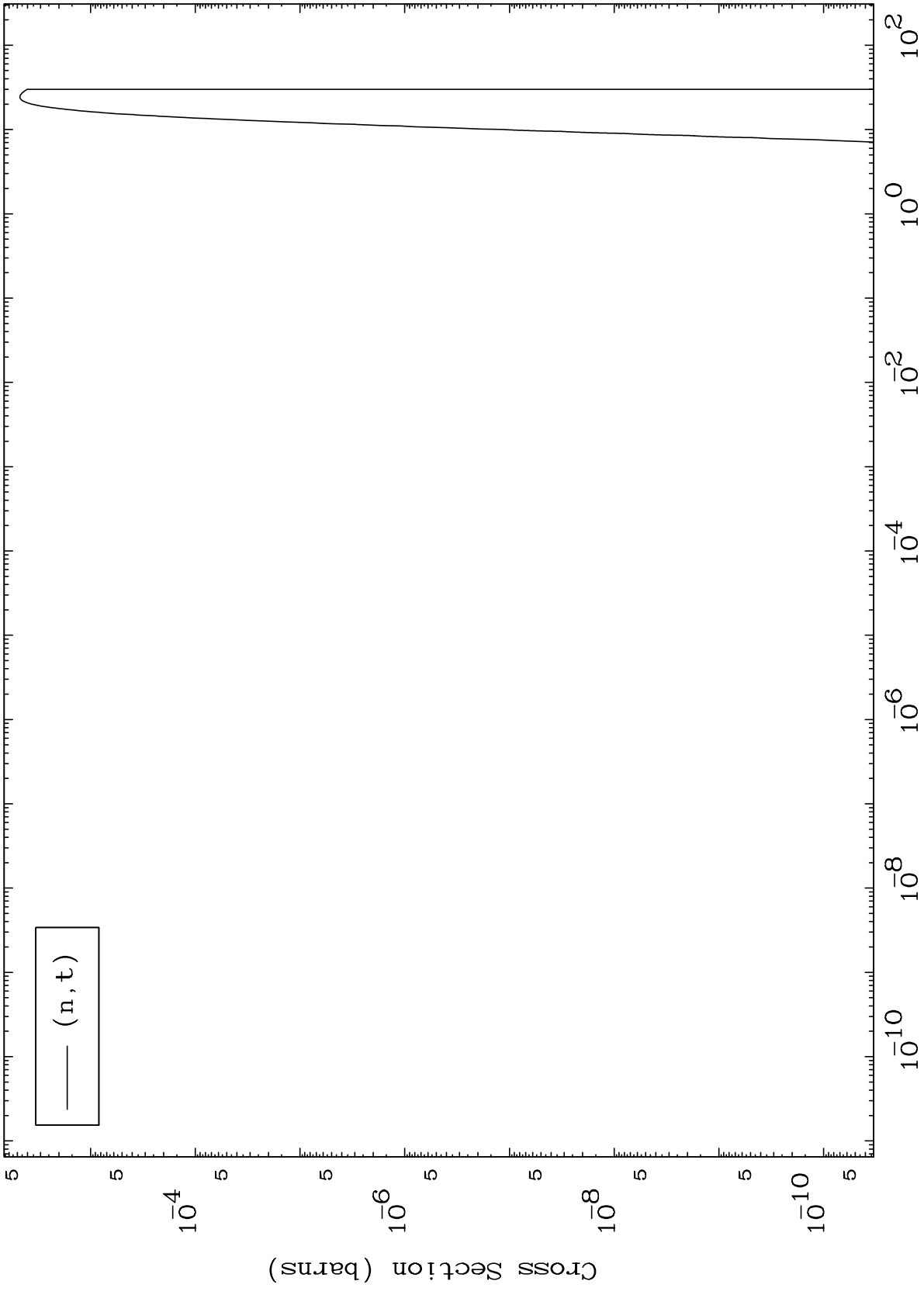
Incident Energy (MeV)

107-Bh-266

MAT 766

(n,t) Levels  
293 Kelvin Cross Sections

107-Bh-266

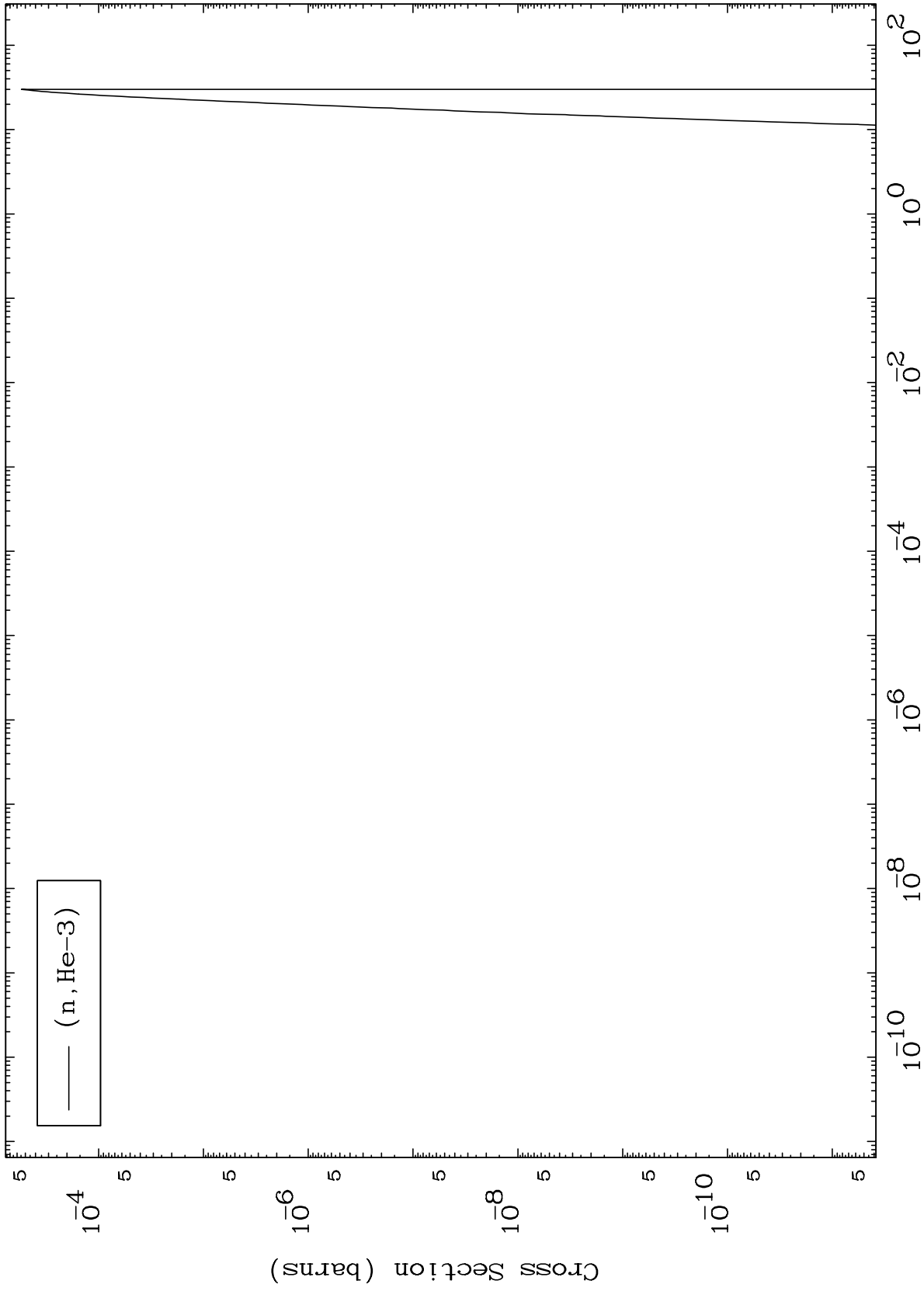


107-Bh-266

MAT 766

(n,He3) Levels  
293 Kelvin Cross Sections

107-Bh-266



12

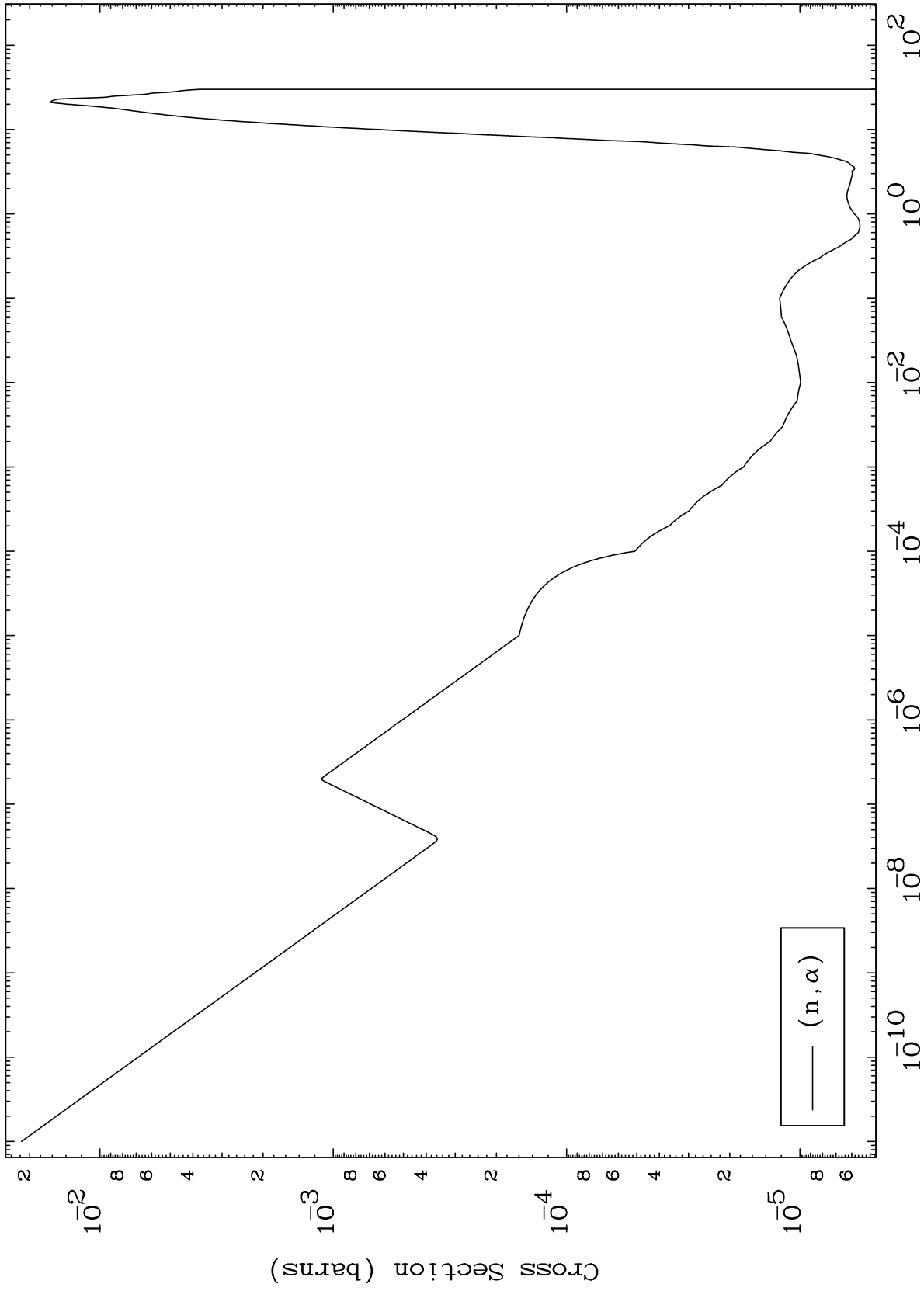
Incident Energy (MeV)

107-Bh-266

MAT 766

(n,α) Levels  
293 Kelvin Cross Sections

107-Bh-266



13

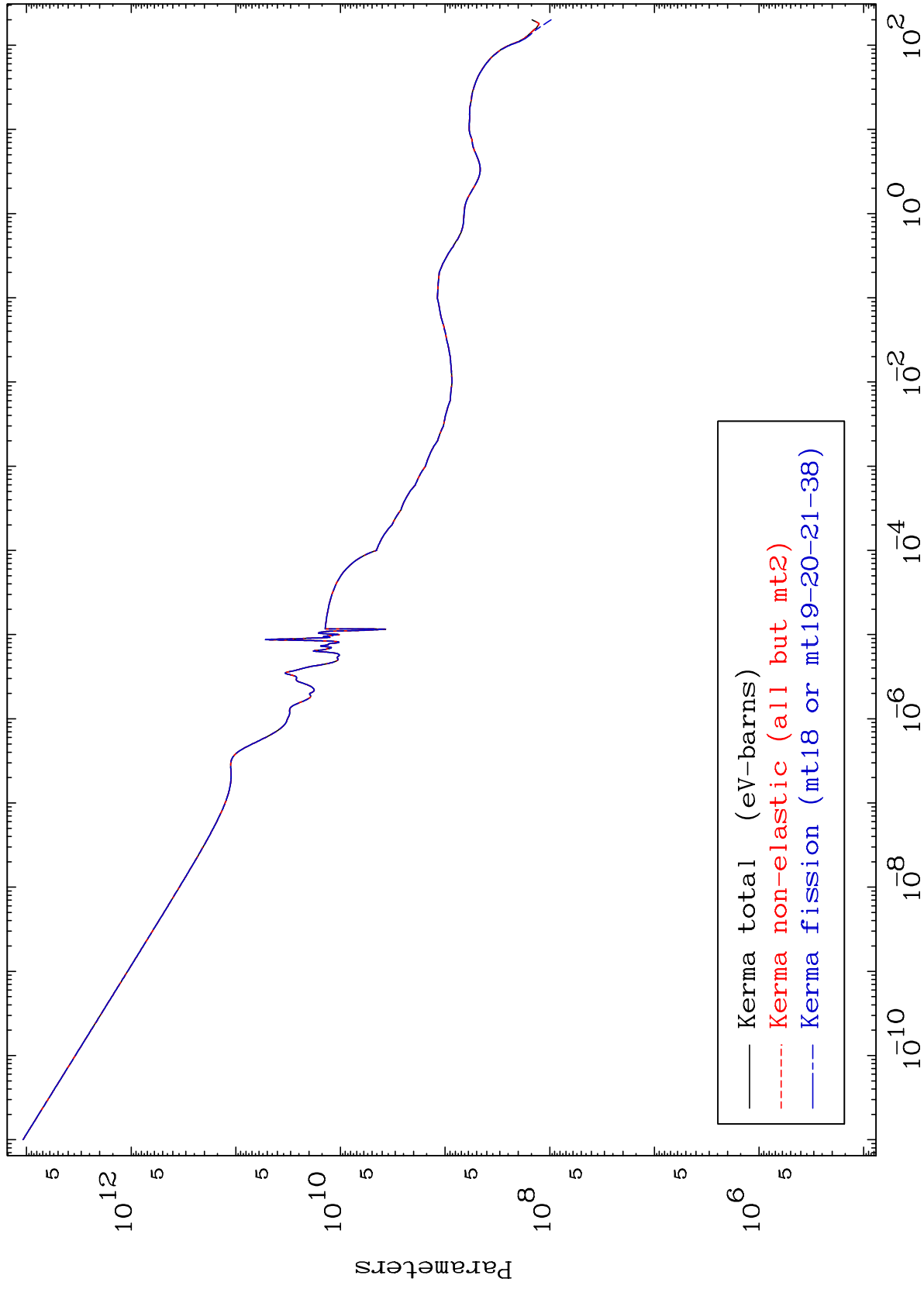
Incident Energy (MeV)

107-Bh-266

MAT 766

Energy Release  
Parameters

107-Bh-266



14

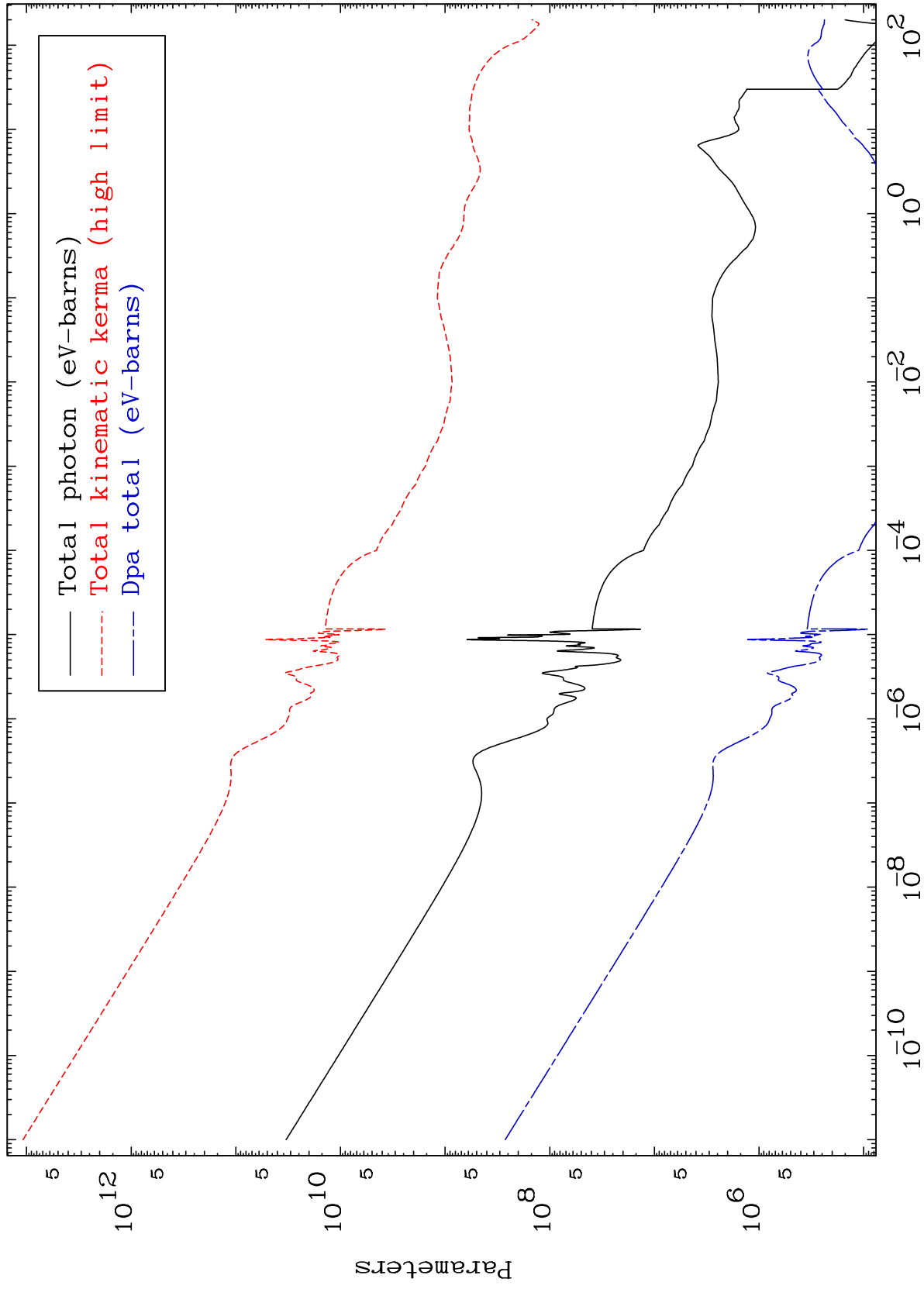
Incident Energy (MeV)

107-Bh-266

MAT 766

Energy Release  
Parameters

107-Bh-266

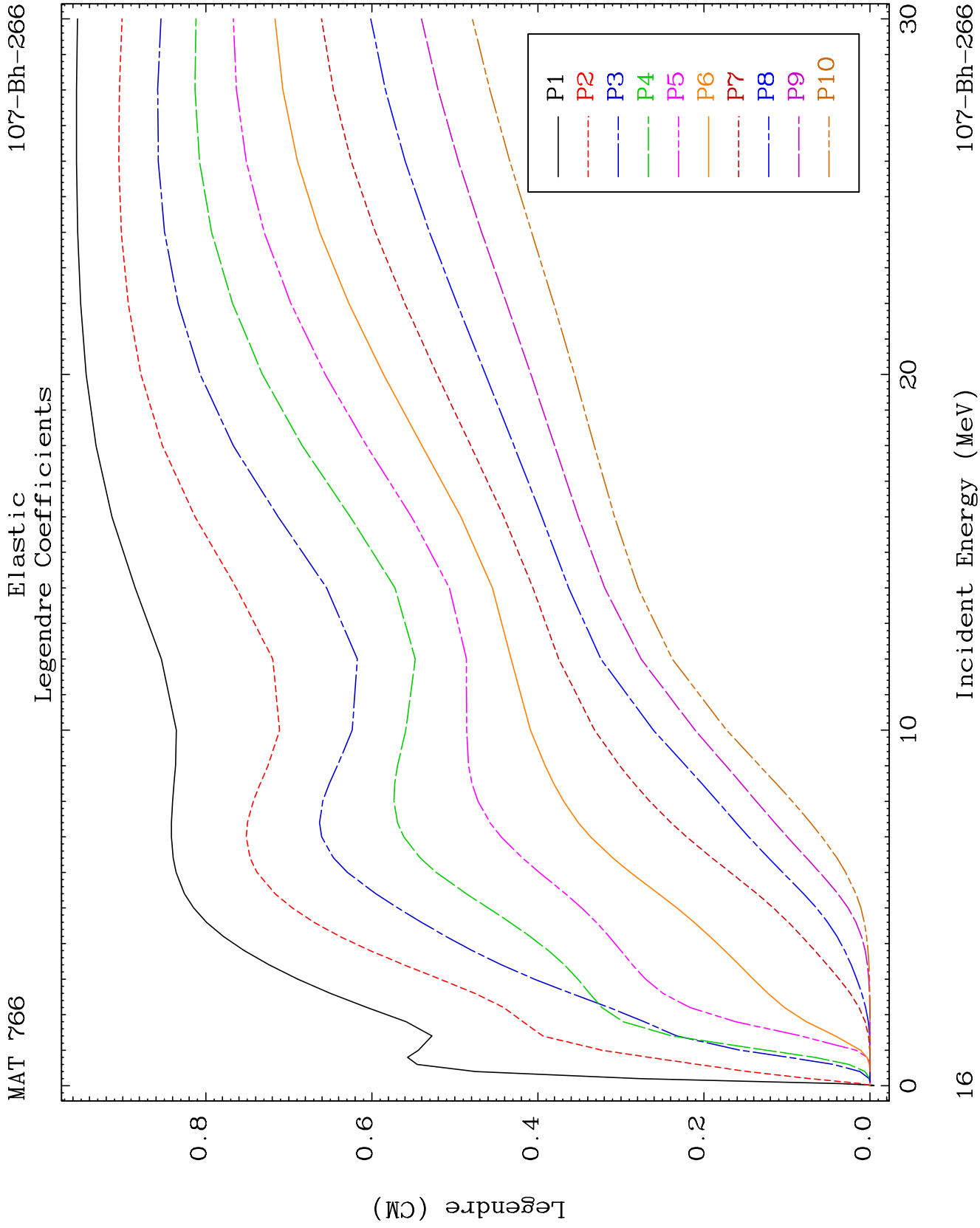


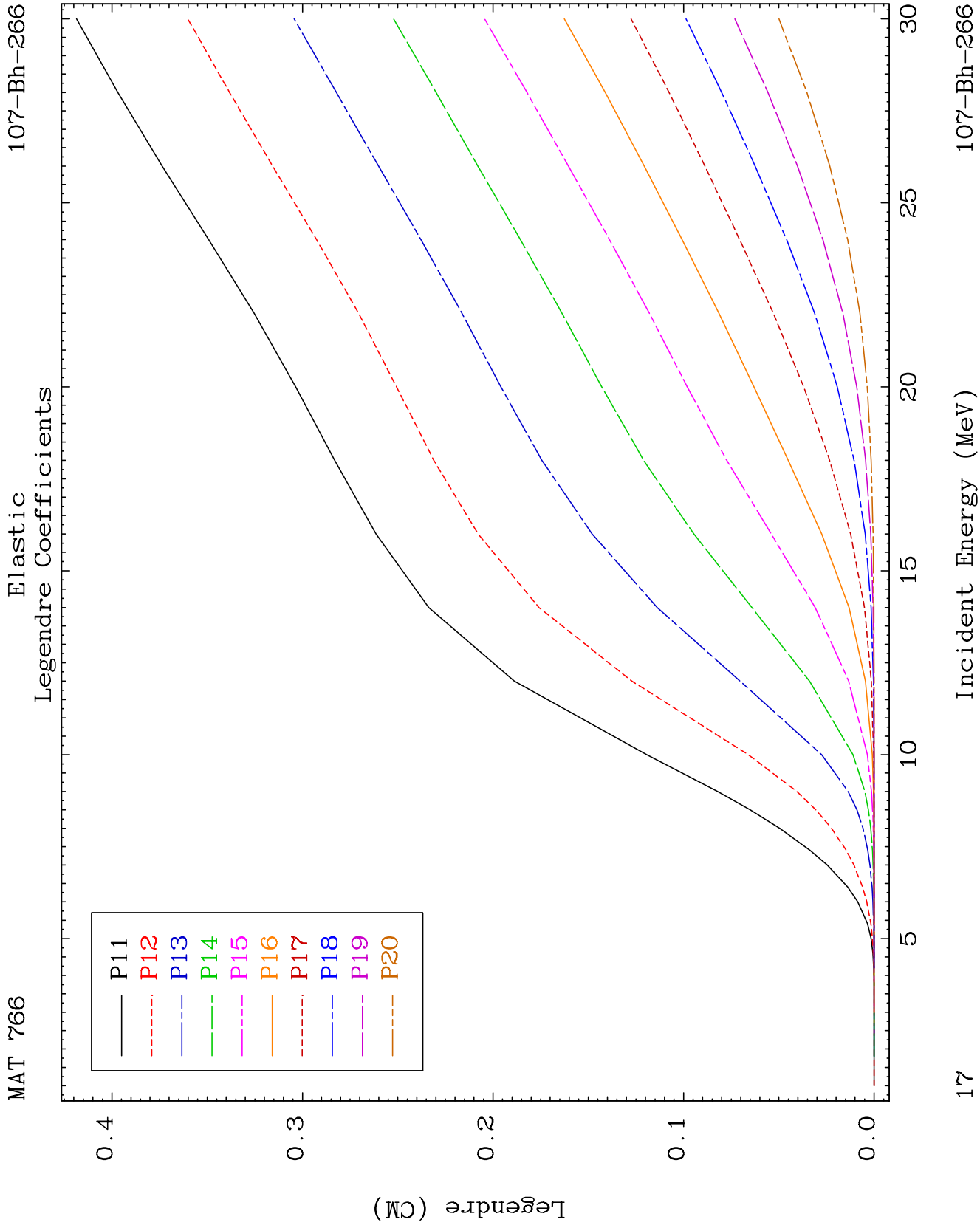
15

Incident Energy (MeV)

107-Bh-266



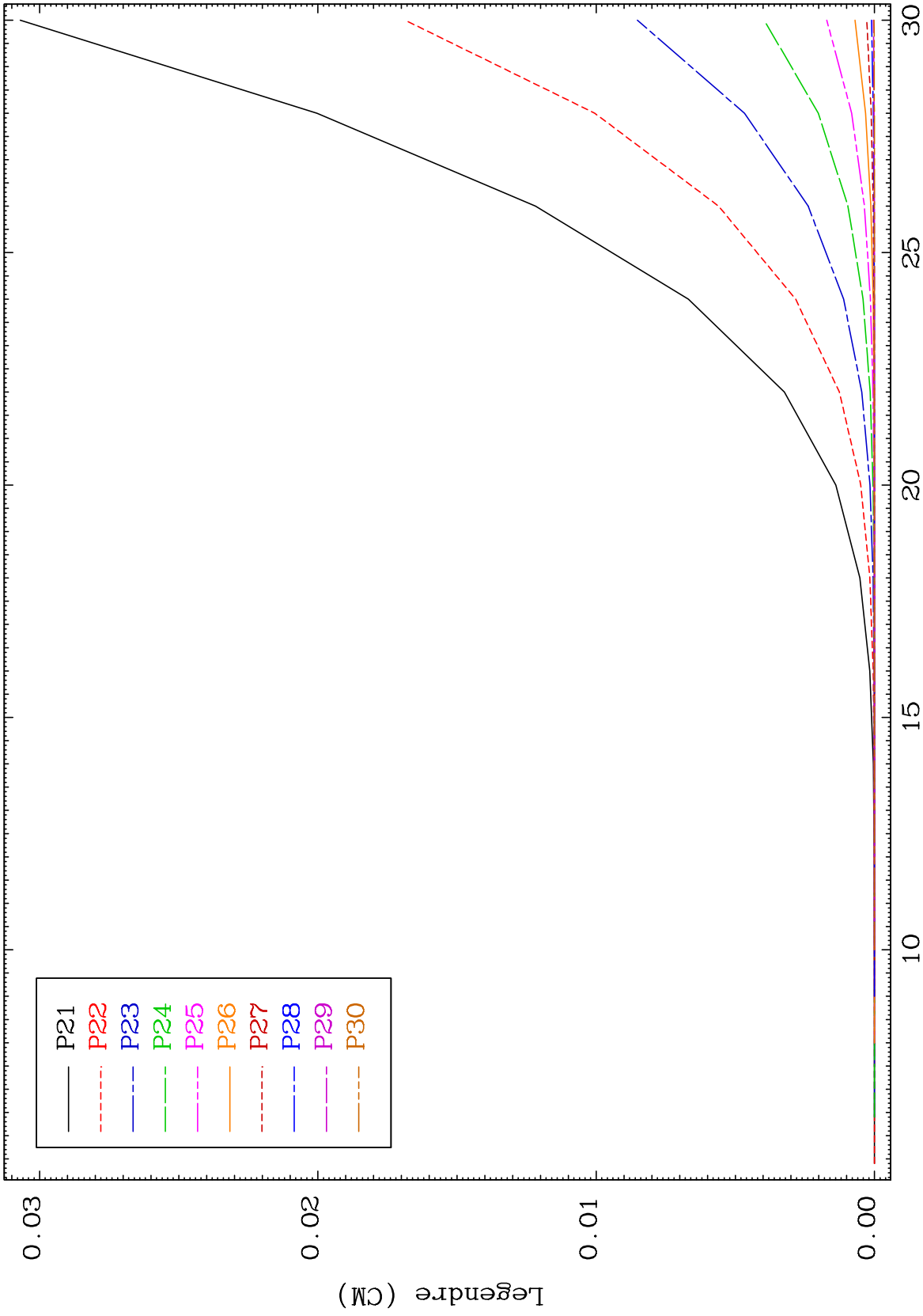




MAT 766

Elastic Legendre Coefficients

107-Bh-266



18

Incident Energy (MeV)

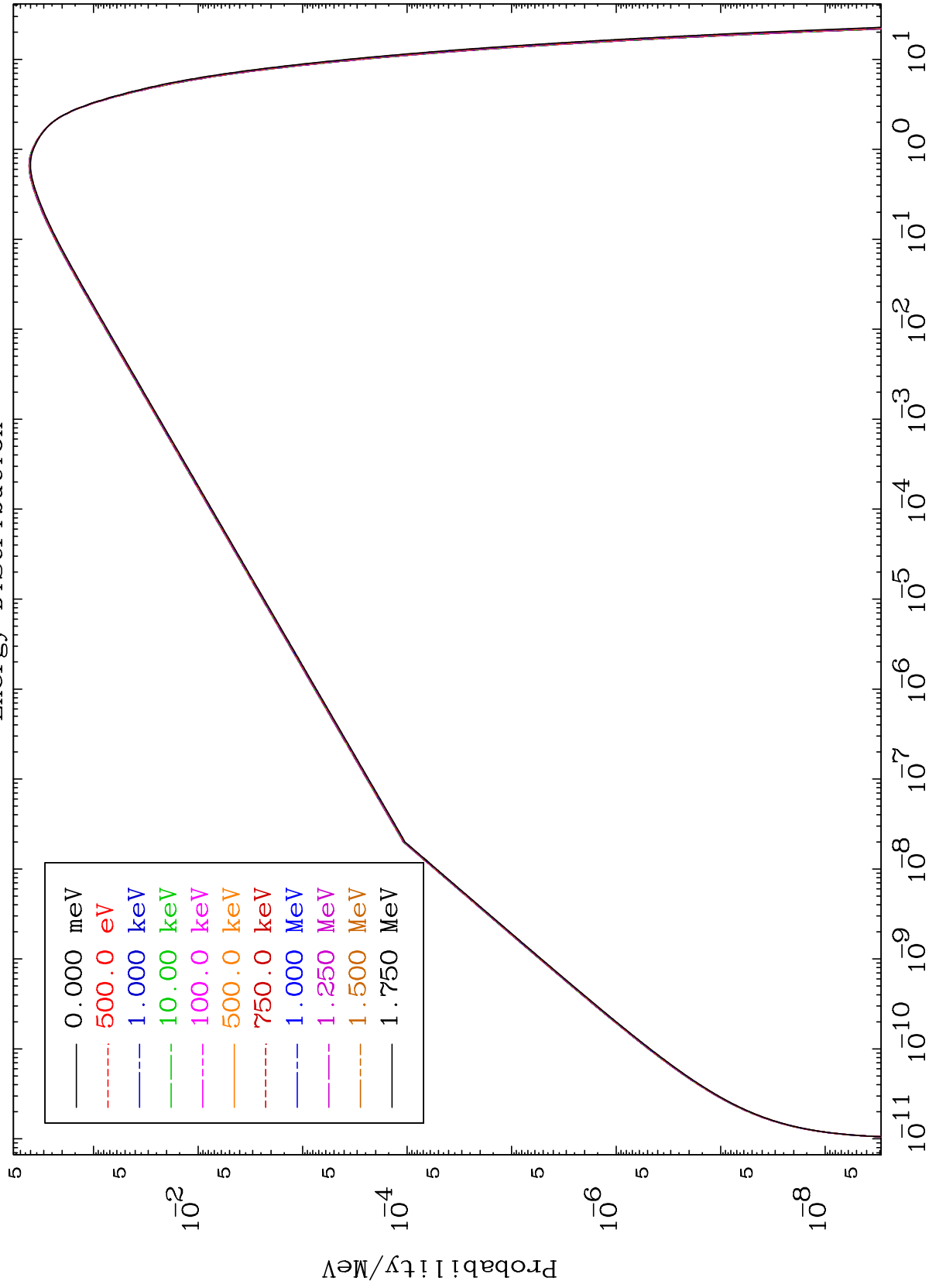
107-Bh-266



MAT 766

# Fission Energy Distribution

$^{107}\text{Bh-266}$

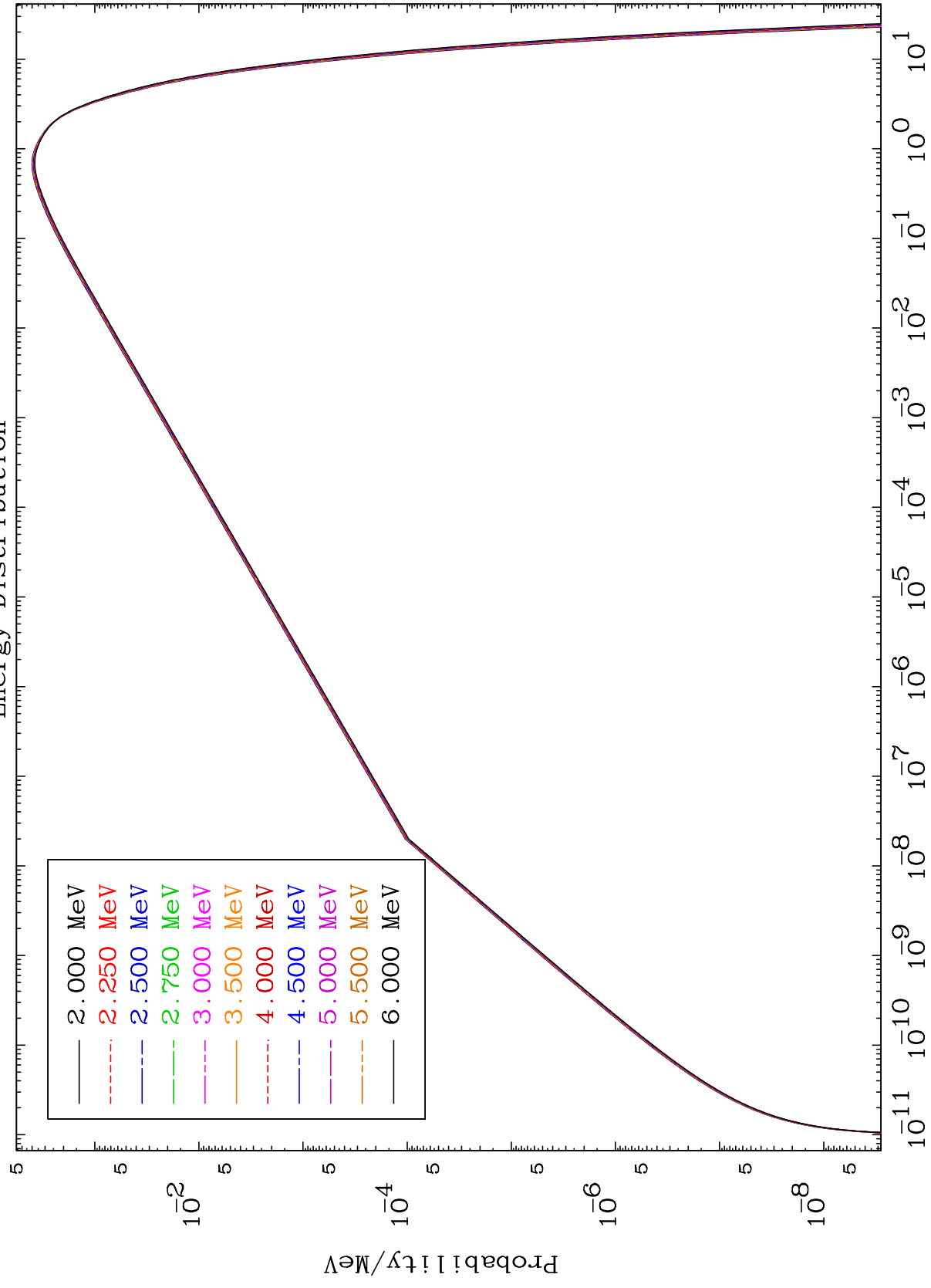


$^{107}\text{Bh-266}$

MAT 766

### Fission Energy Distribution

<sup>107</sup>Bh-266

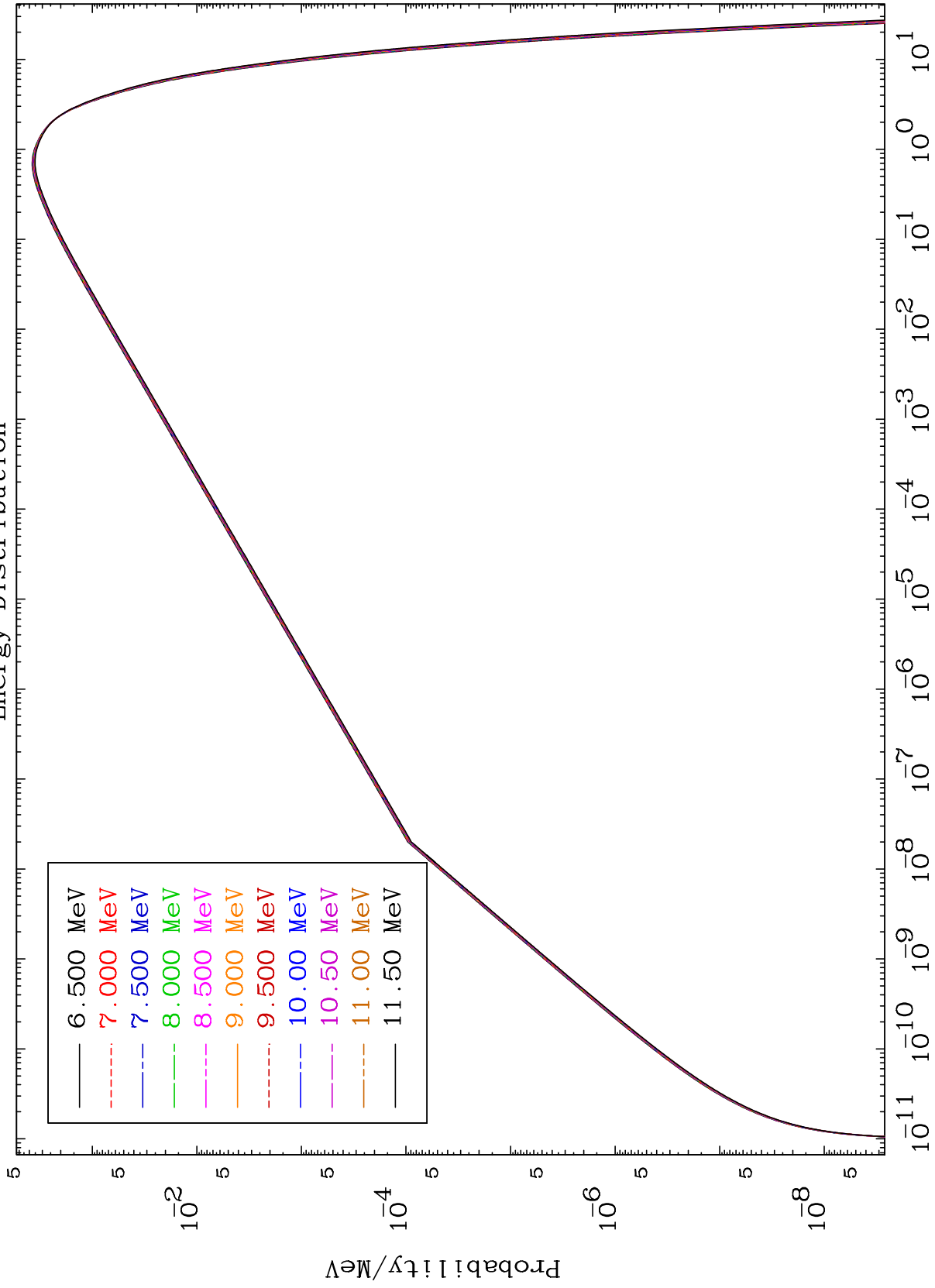


<sup>107</sup>Bh-266

MAT 766

# Fission Energy Distribution

$^{107}\text{Bh-266}$

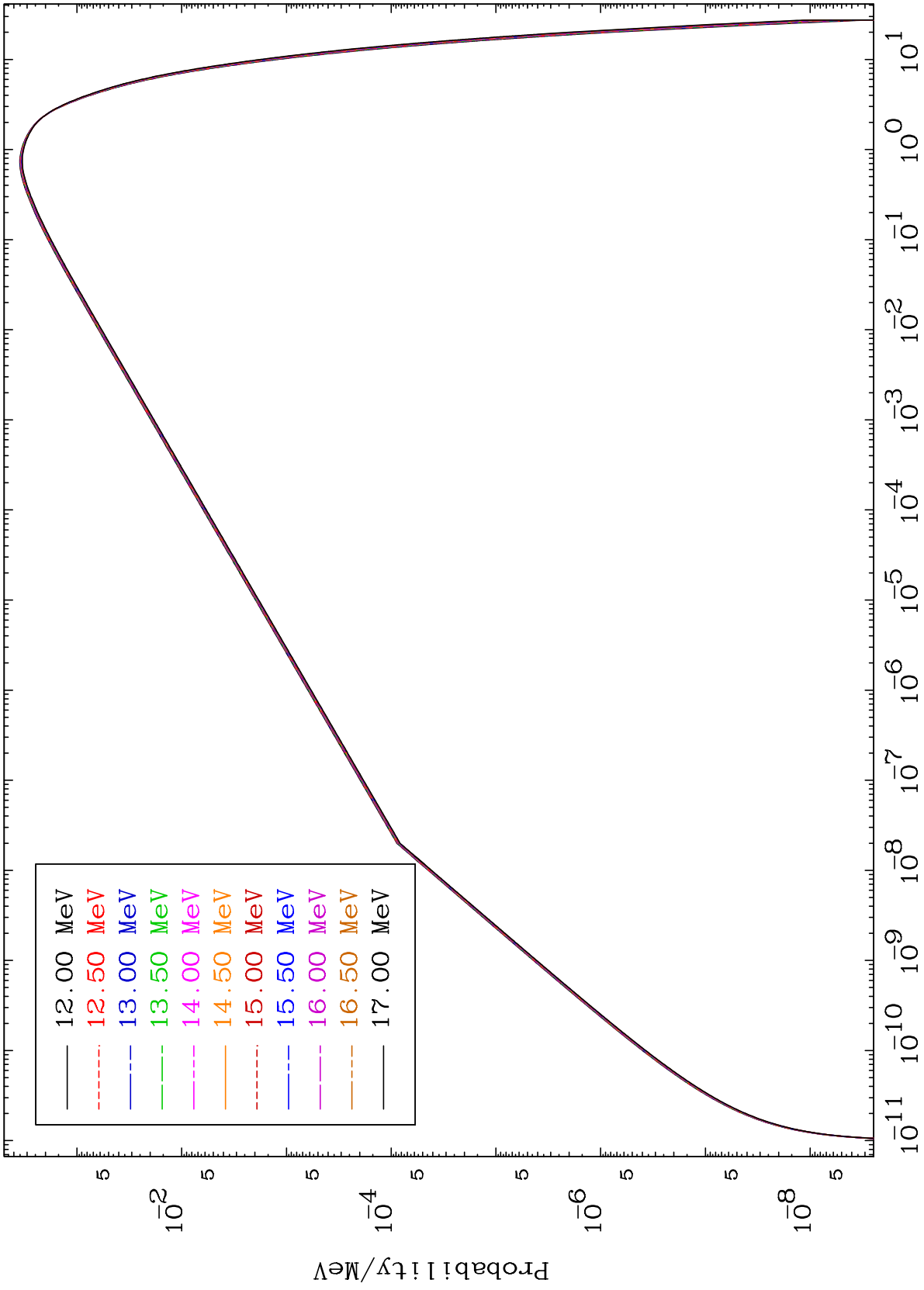


$^{107}\text{Bh-266}$

MAT 766

# Fission Energy Distribution

$^{107}\text{Bh-266}$



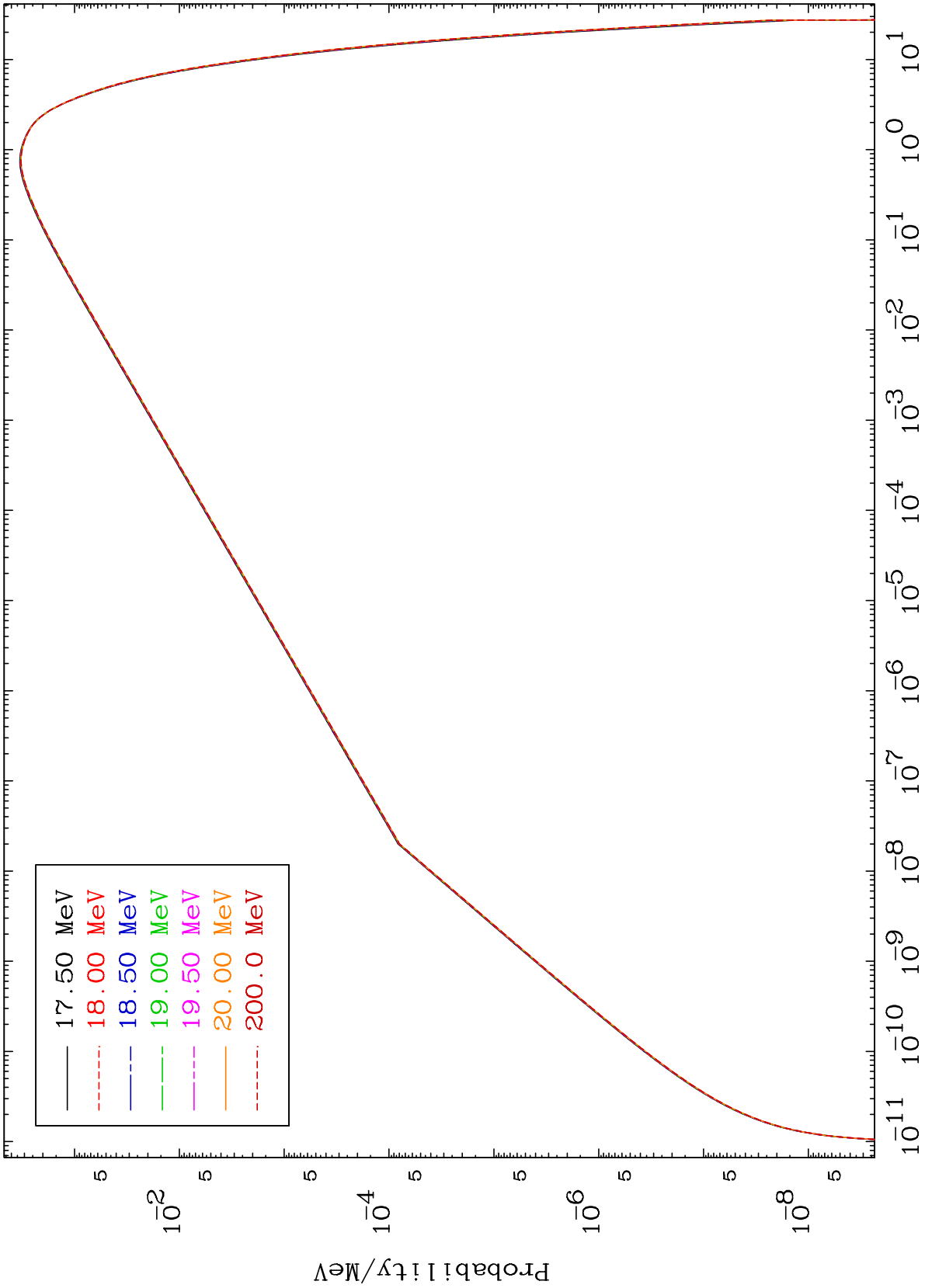
$^{107}\text{Bh-266}$



MAT 766

### Fission Energy Distribution

<sup>107</sup>Bh-266



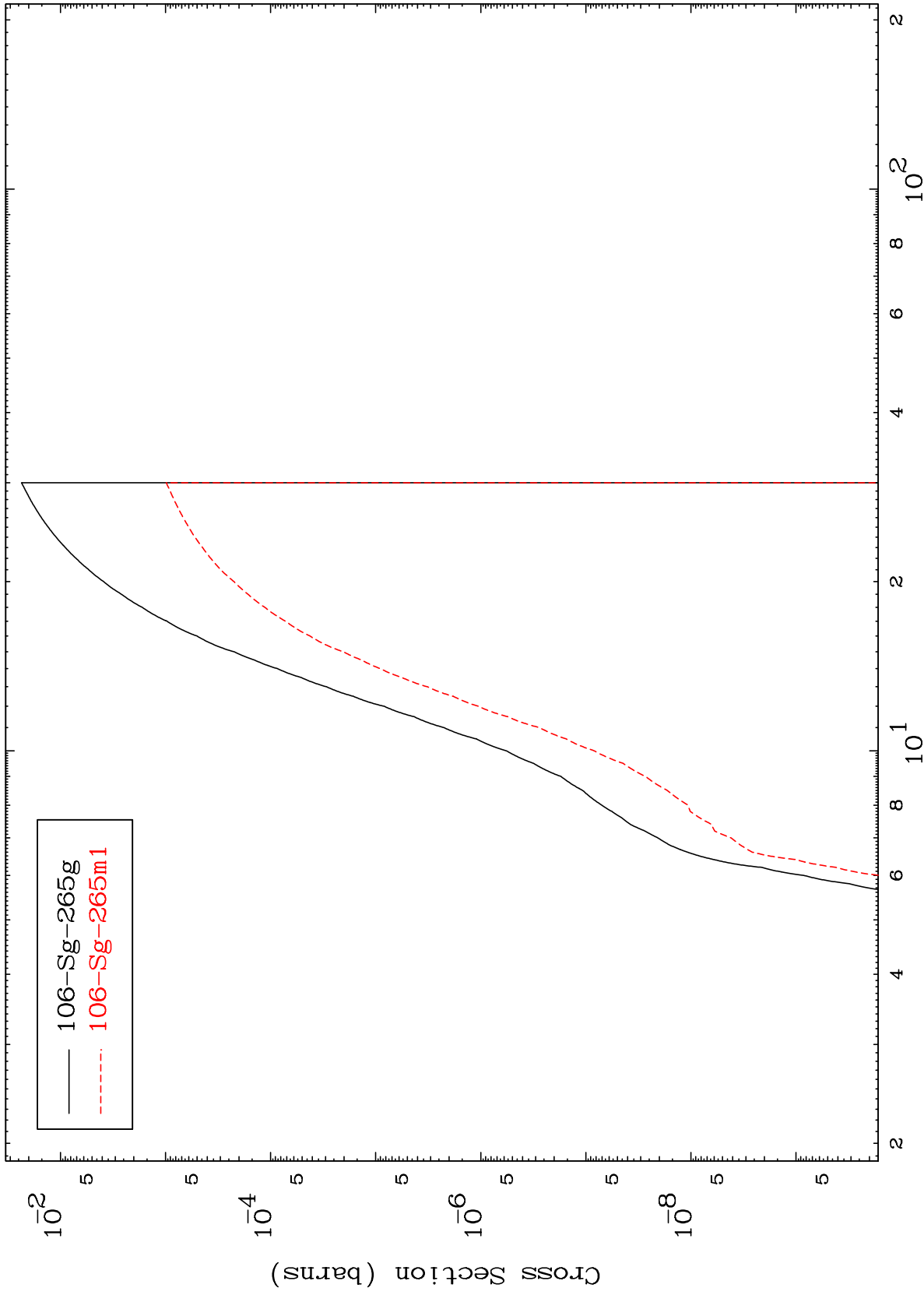
<sup>107</sup>Bh-266

MAT 766

(n,n') p

107-Bh-266

Radionuclide Production Cross Section



25

Incident Energy (MeV)

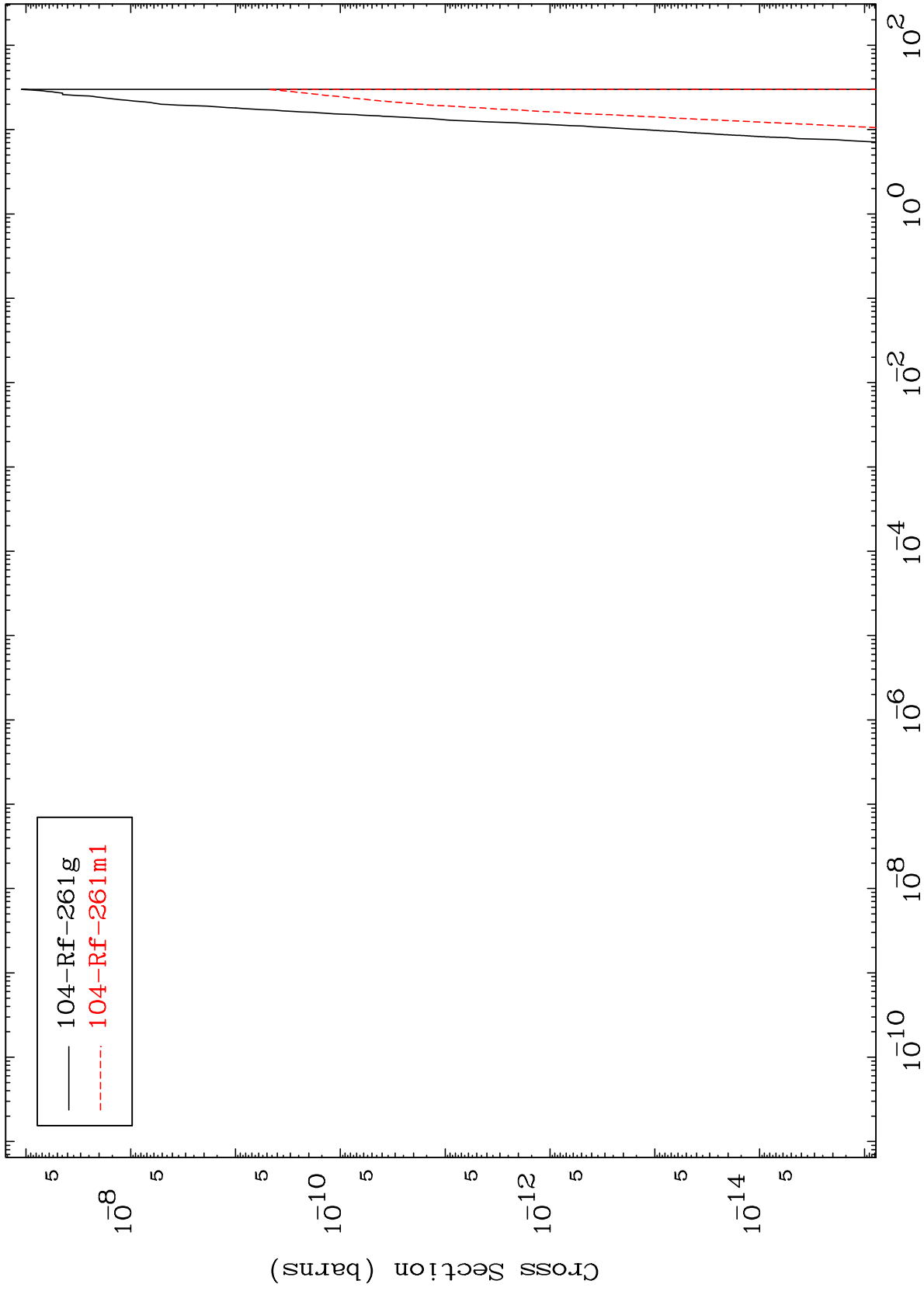
107-Bh-266

MAT 766

(n,n') p  $\alpha$

107-Bh-266

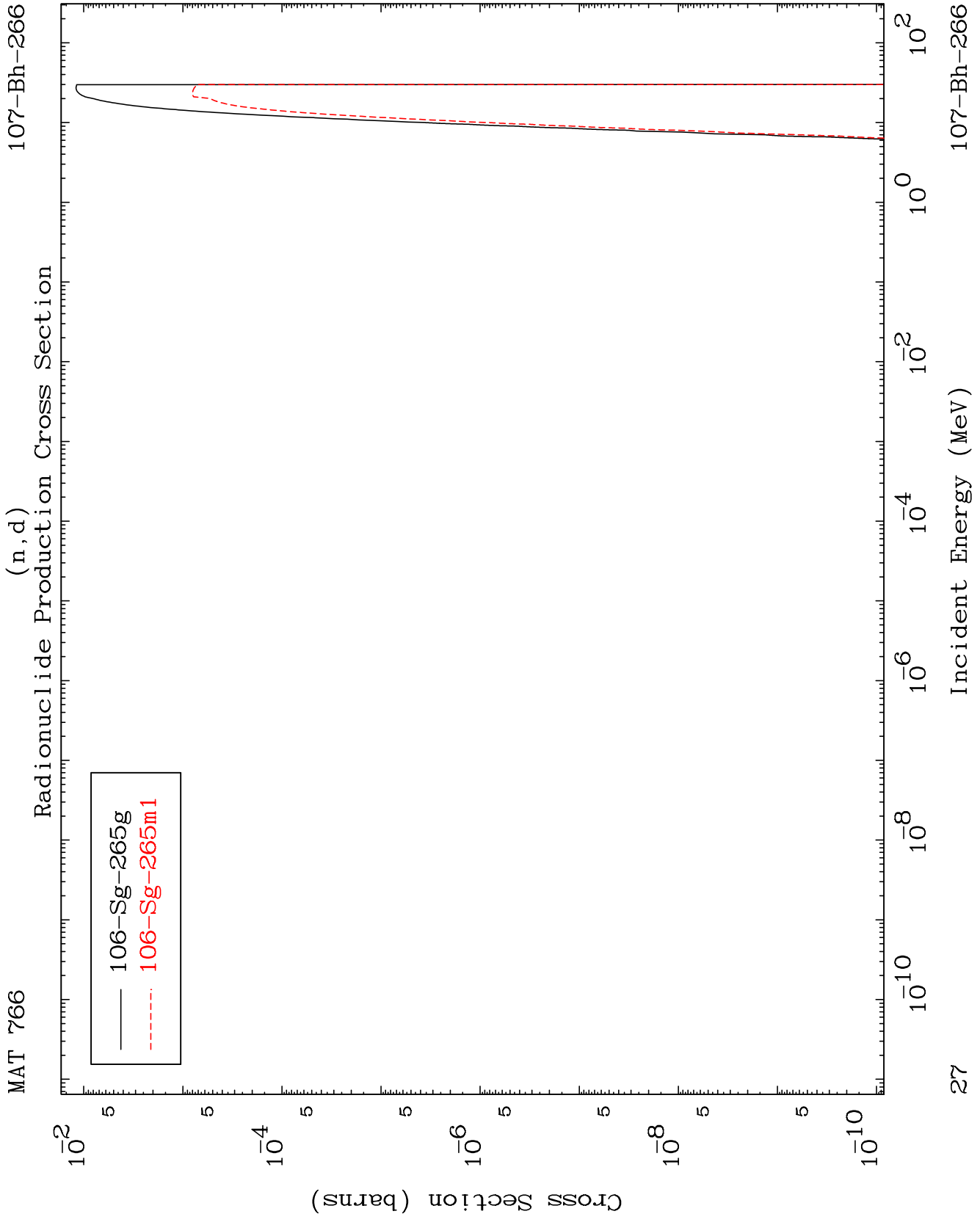
Radionuclide Production Cross Section



26

Incident Energy (MeV)

107-Bh-266

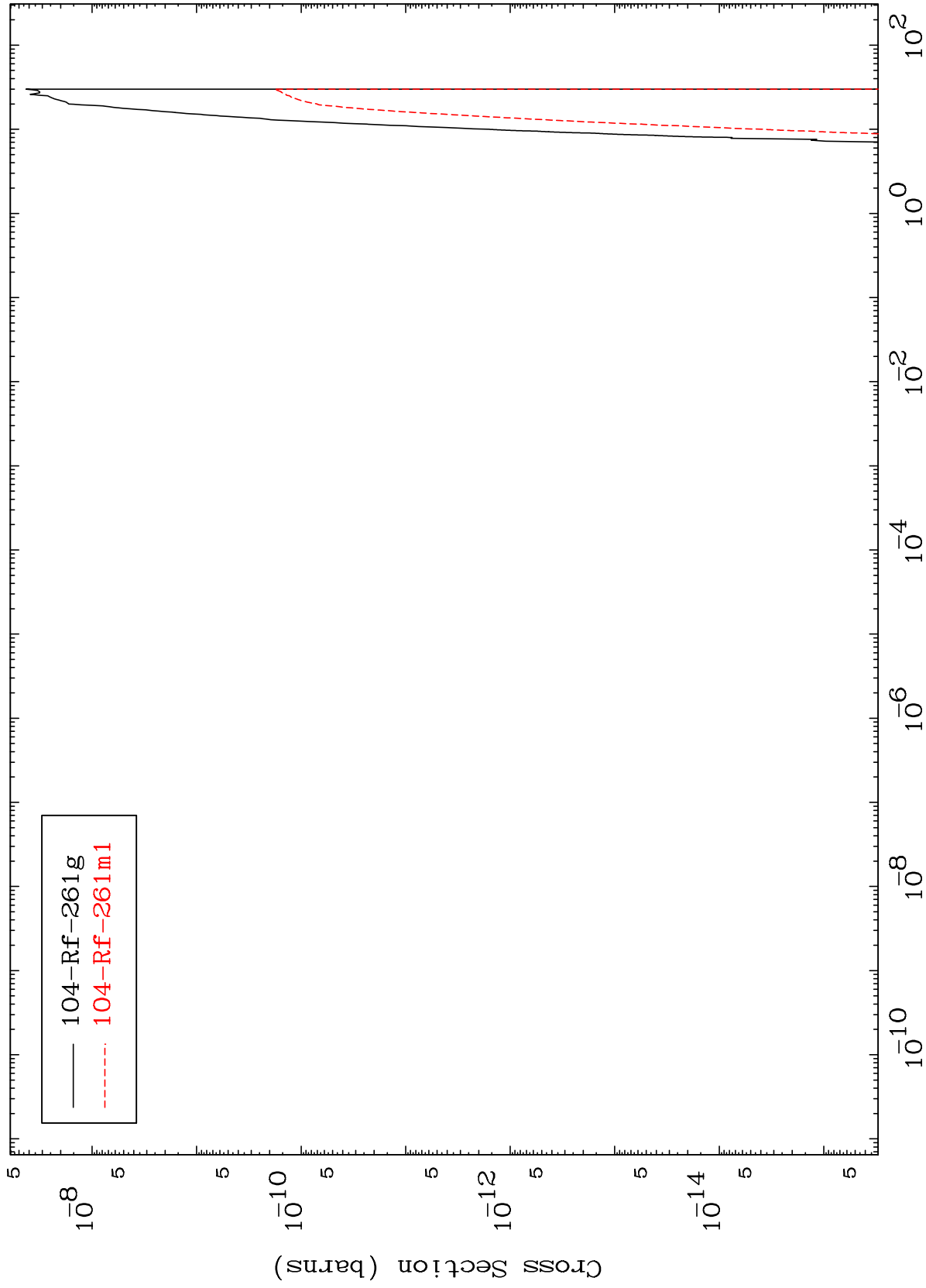


MAT 766

(n,d)  $\alpha$

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Radionuclide Production Cross Section



28

Incident Energy (MeV)

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