

Program EVALPLOT
(Version 2018-1)

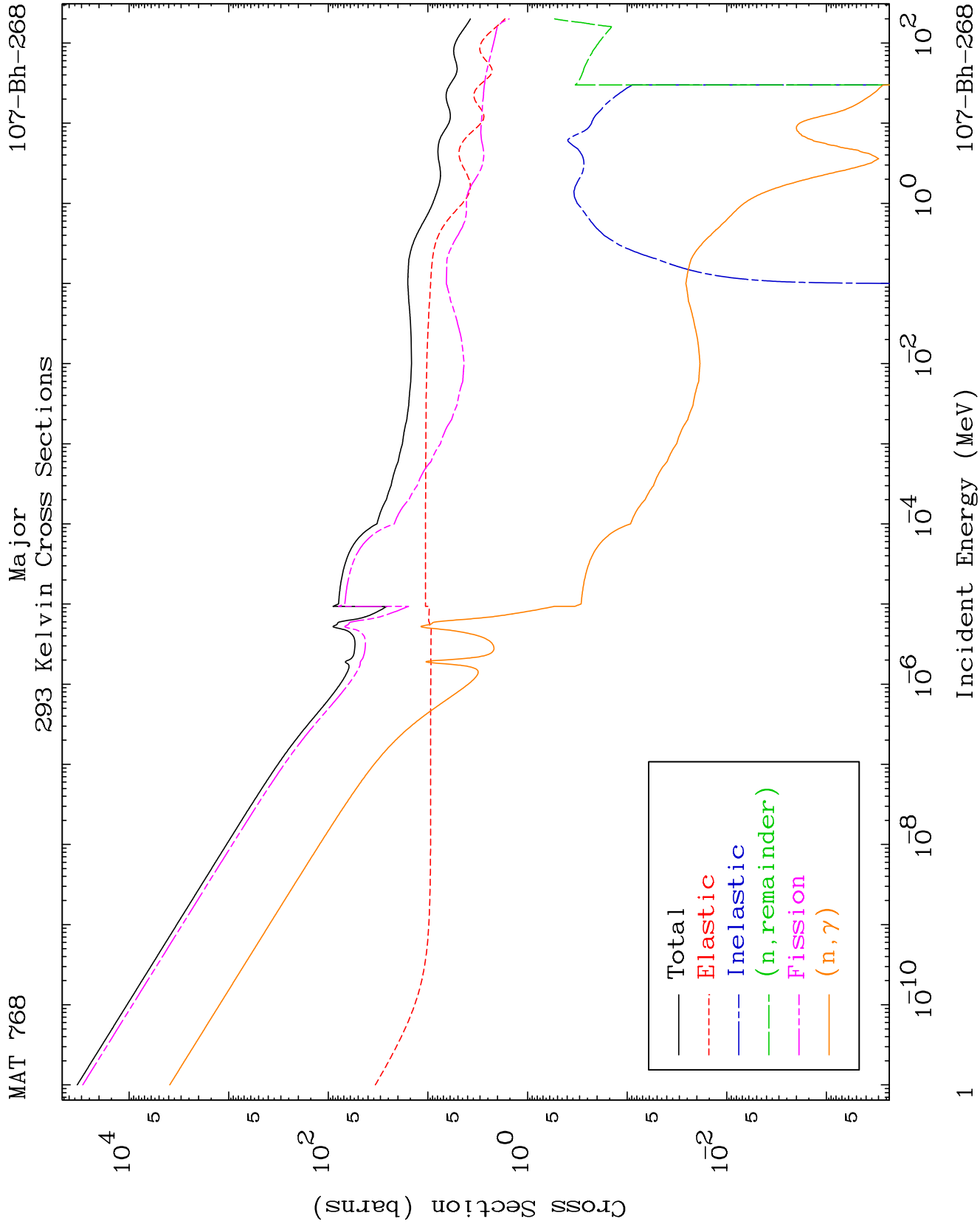
by

Dermott E. Cullen
(Present Contact Information)
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

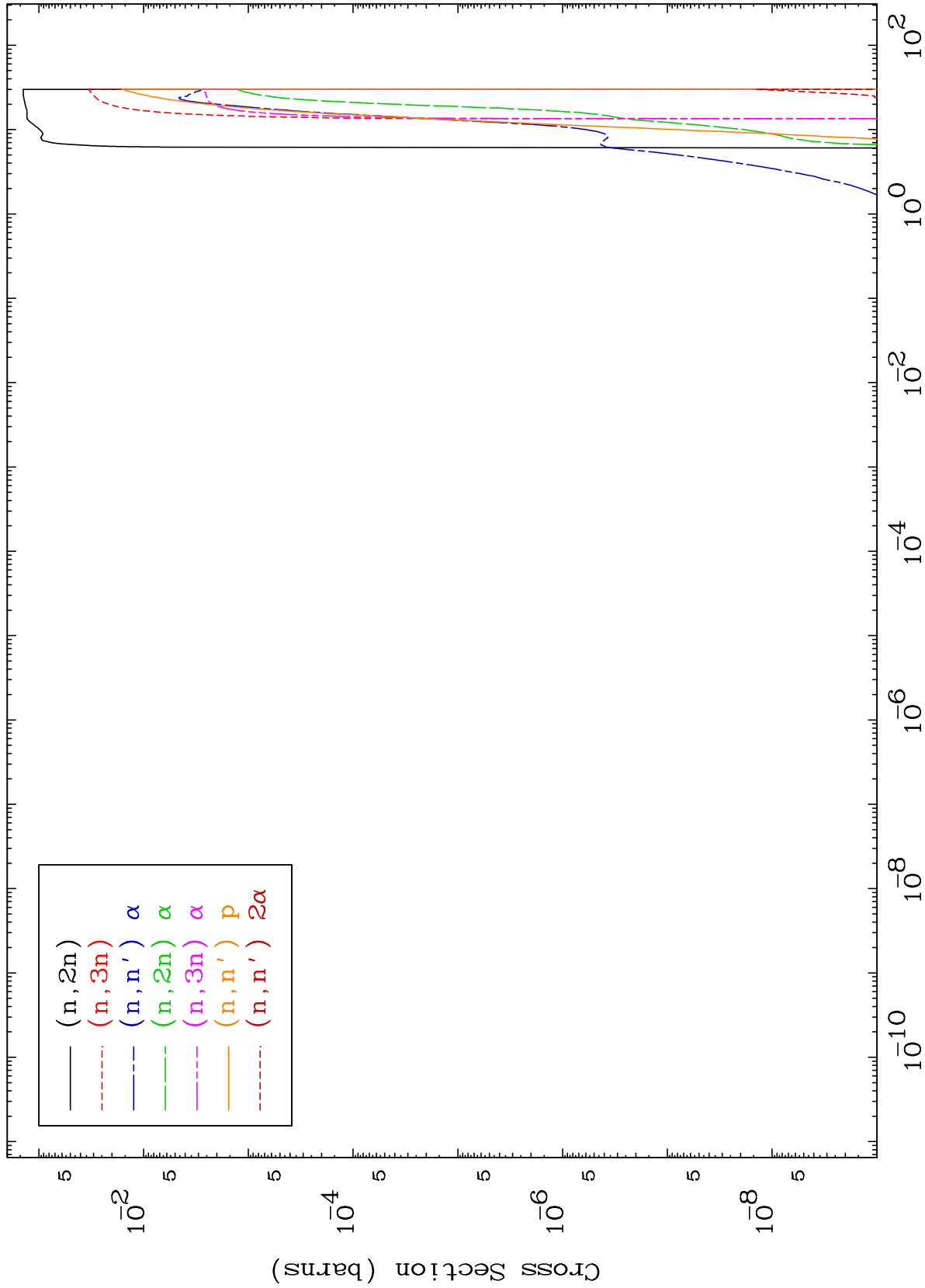
Press Mouse Button to Start



MAT 768

Neutron Production
293 Kelvin Cross Sections

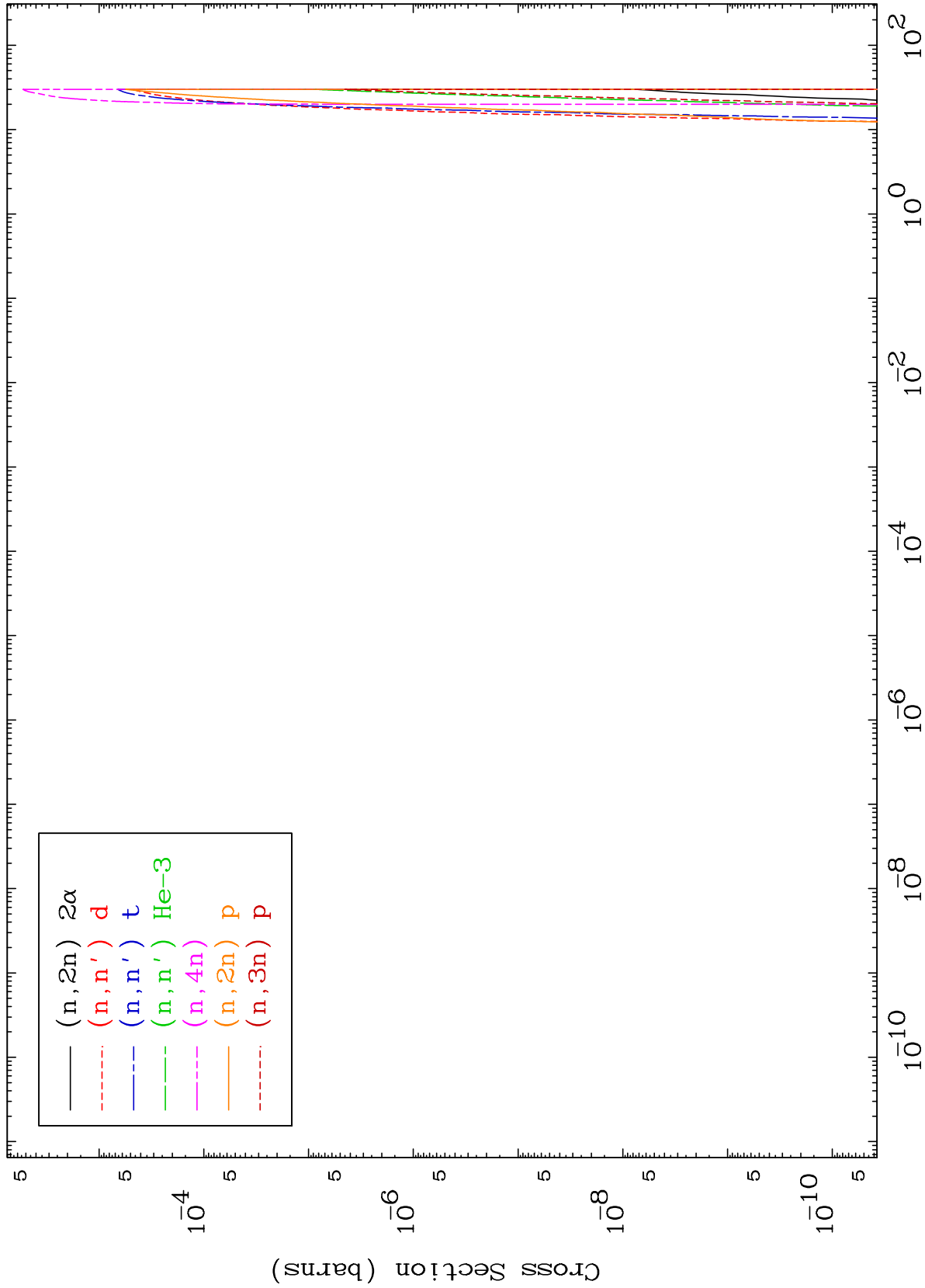
107-Bh-268



MAT 768

Neutron Production
293 Kelvin Cross Sections

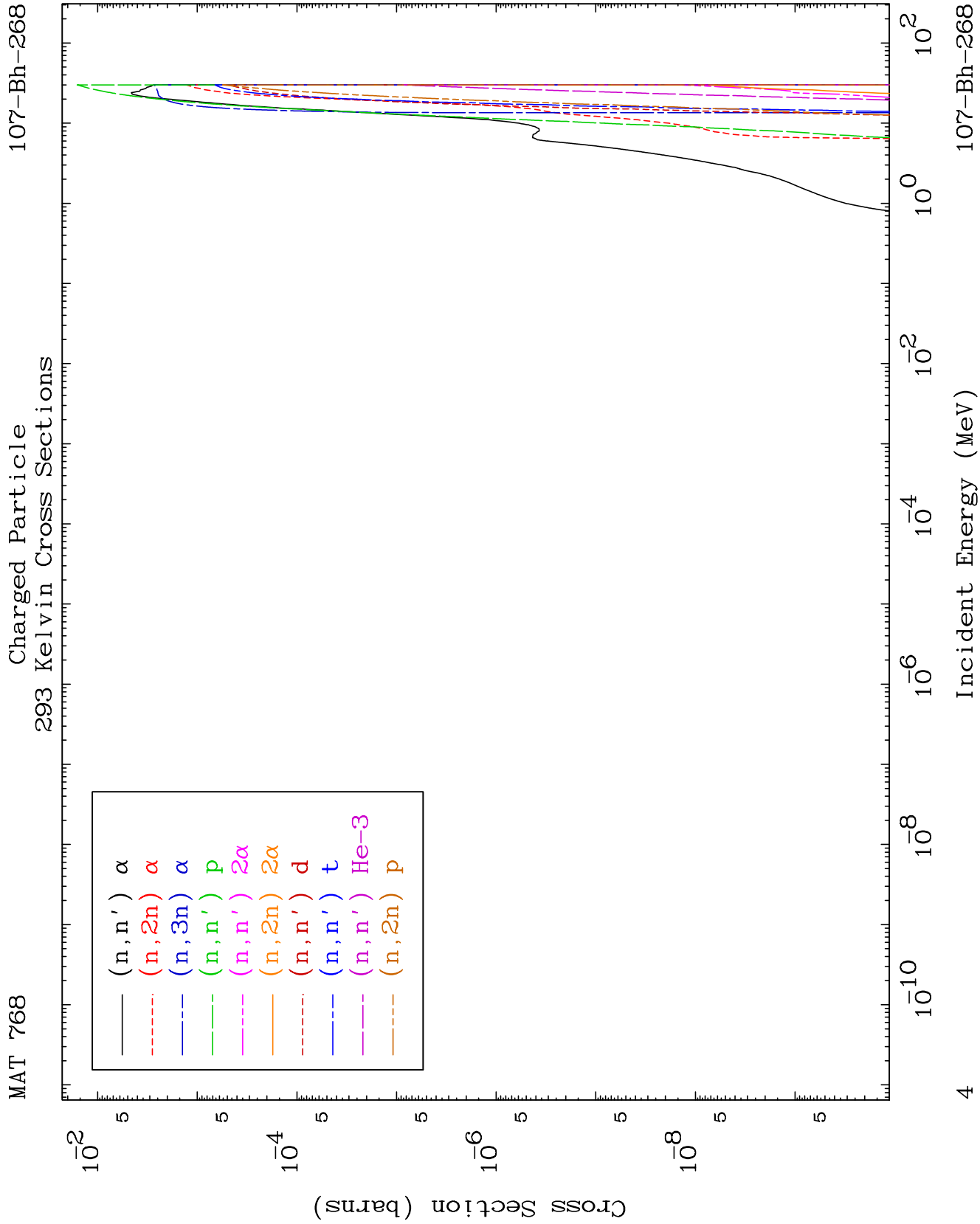
107-Bh-268

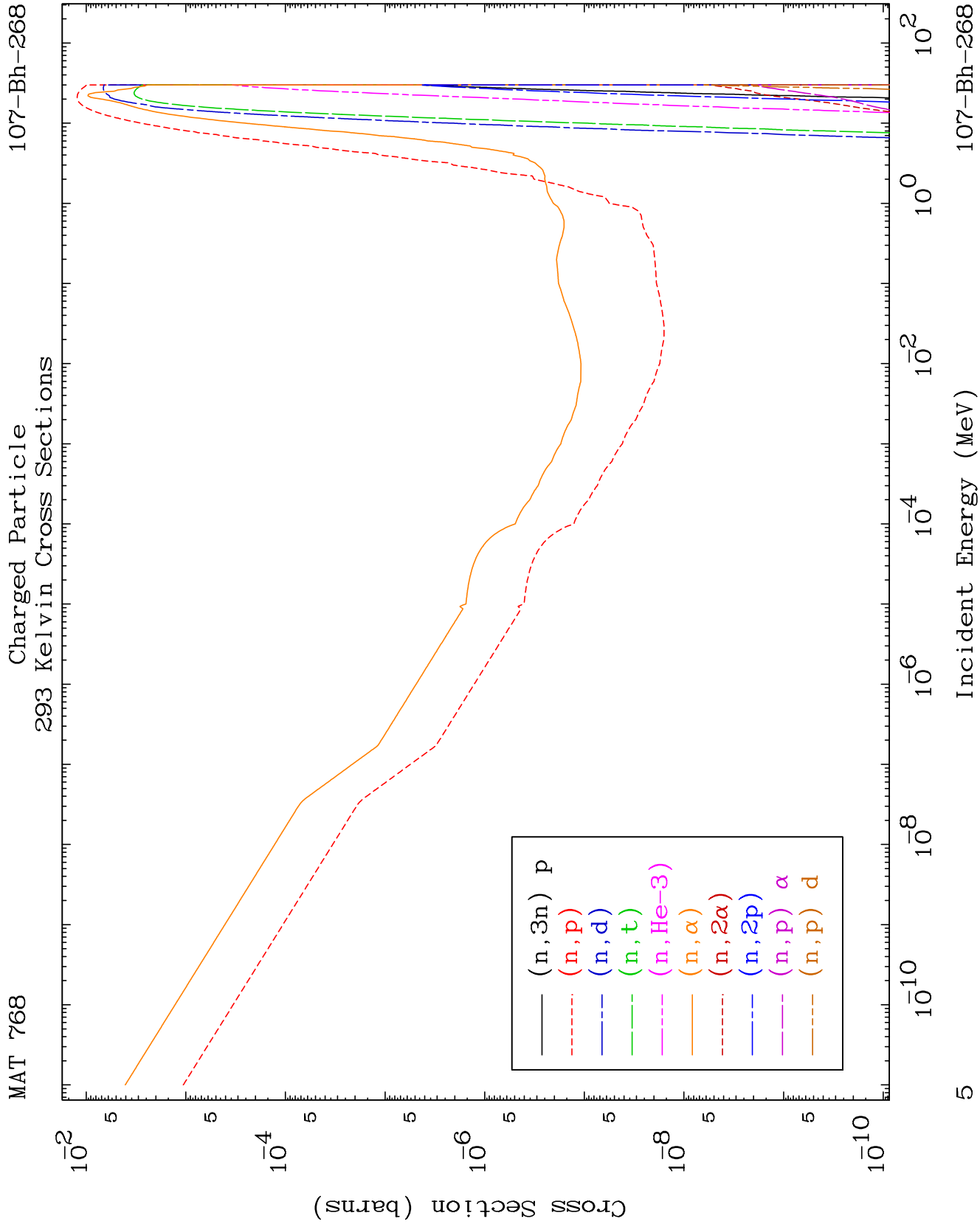


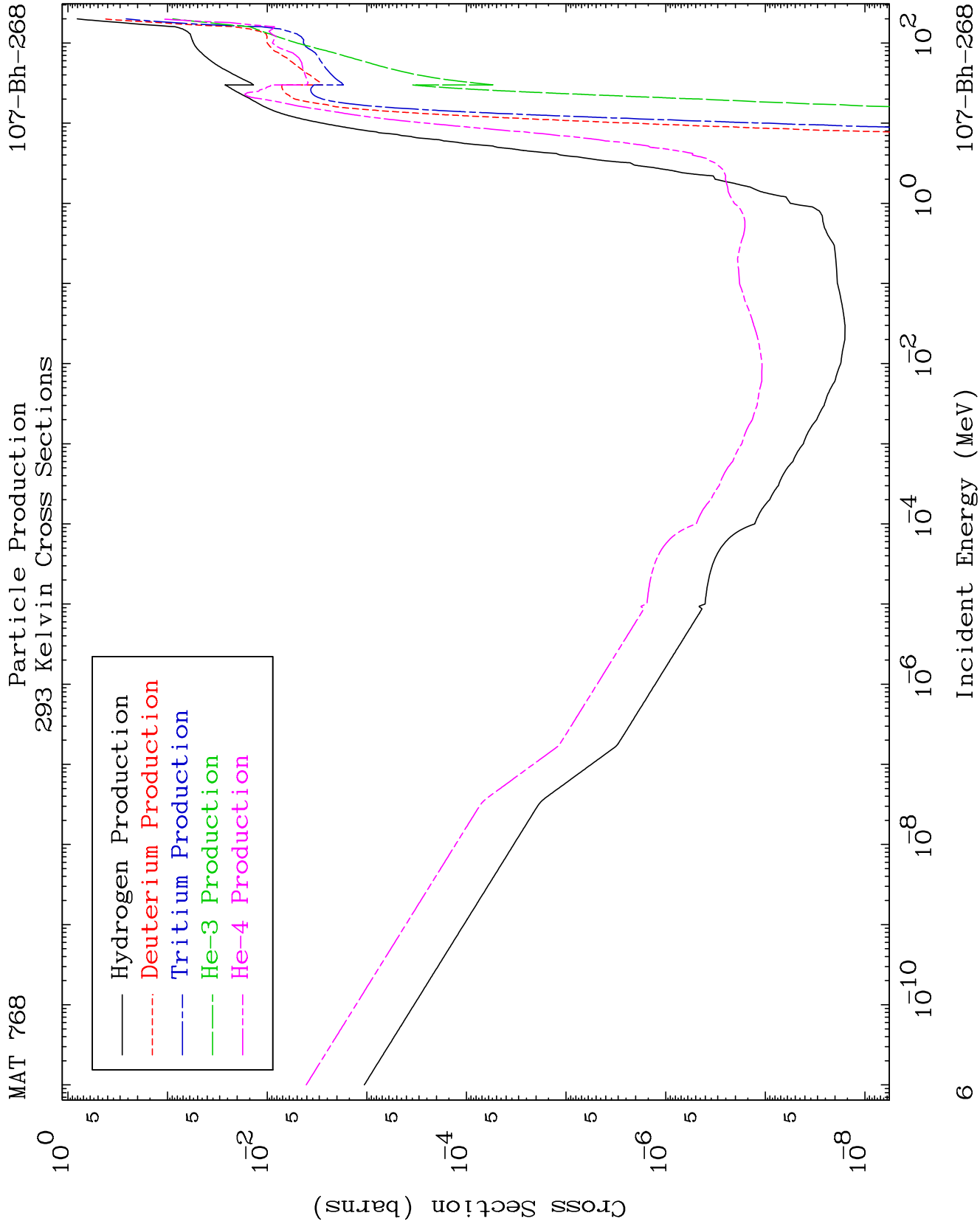
3

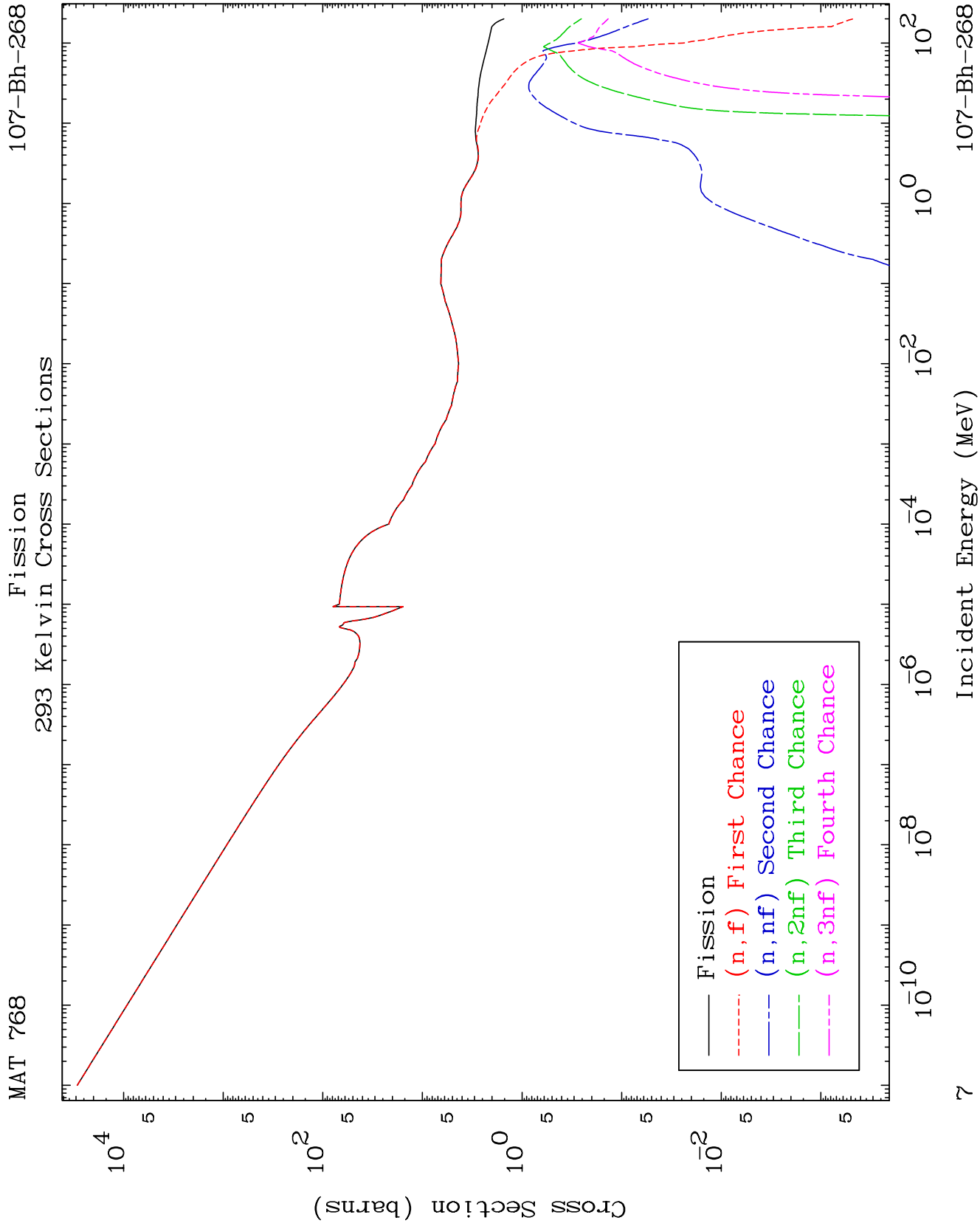
Incident Energy (MeV)

107-Bh-268





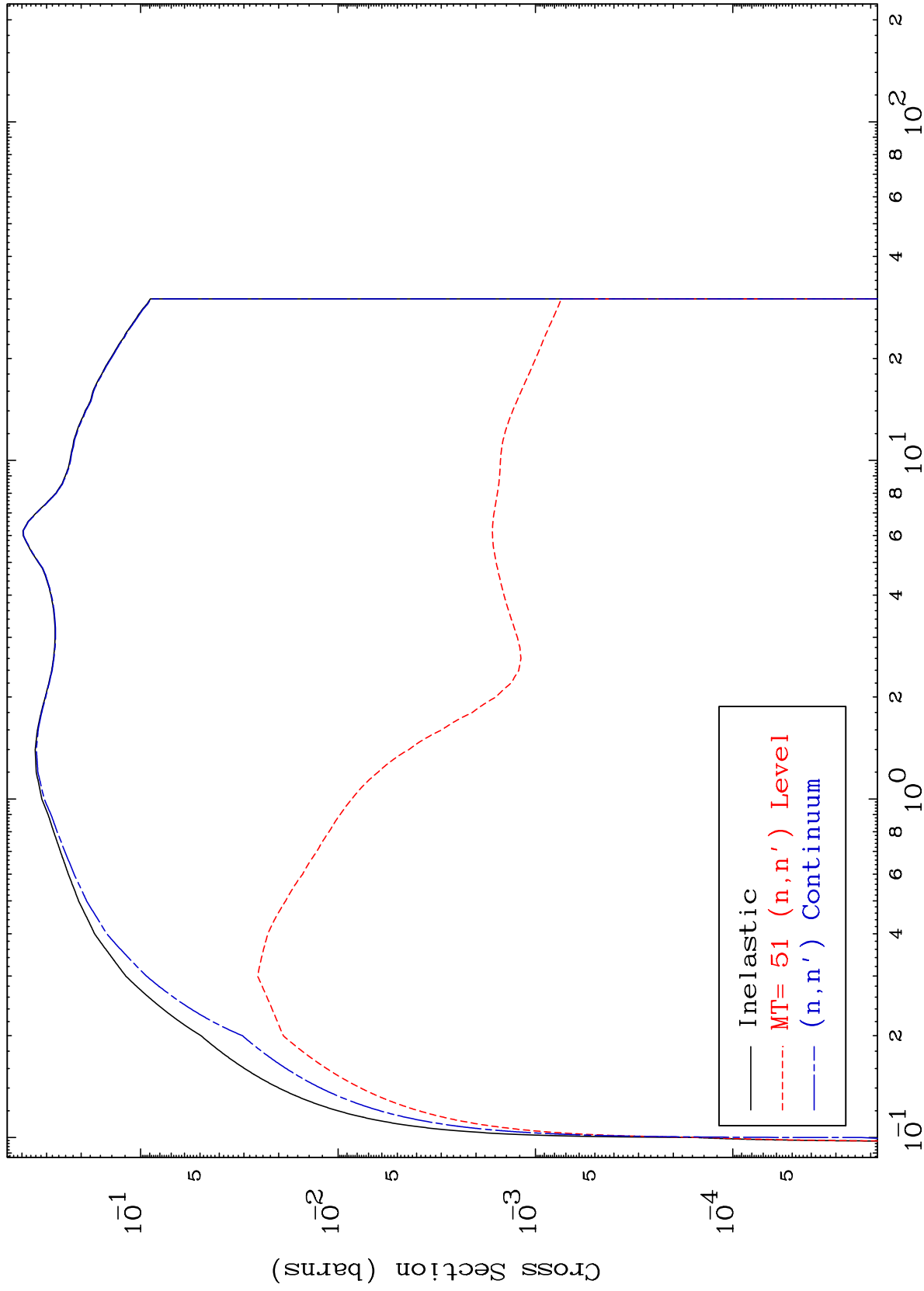




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(n,n') Level
293 Kelvin Cross Sections

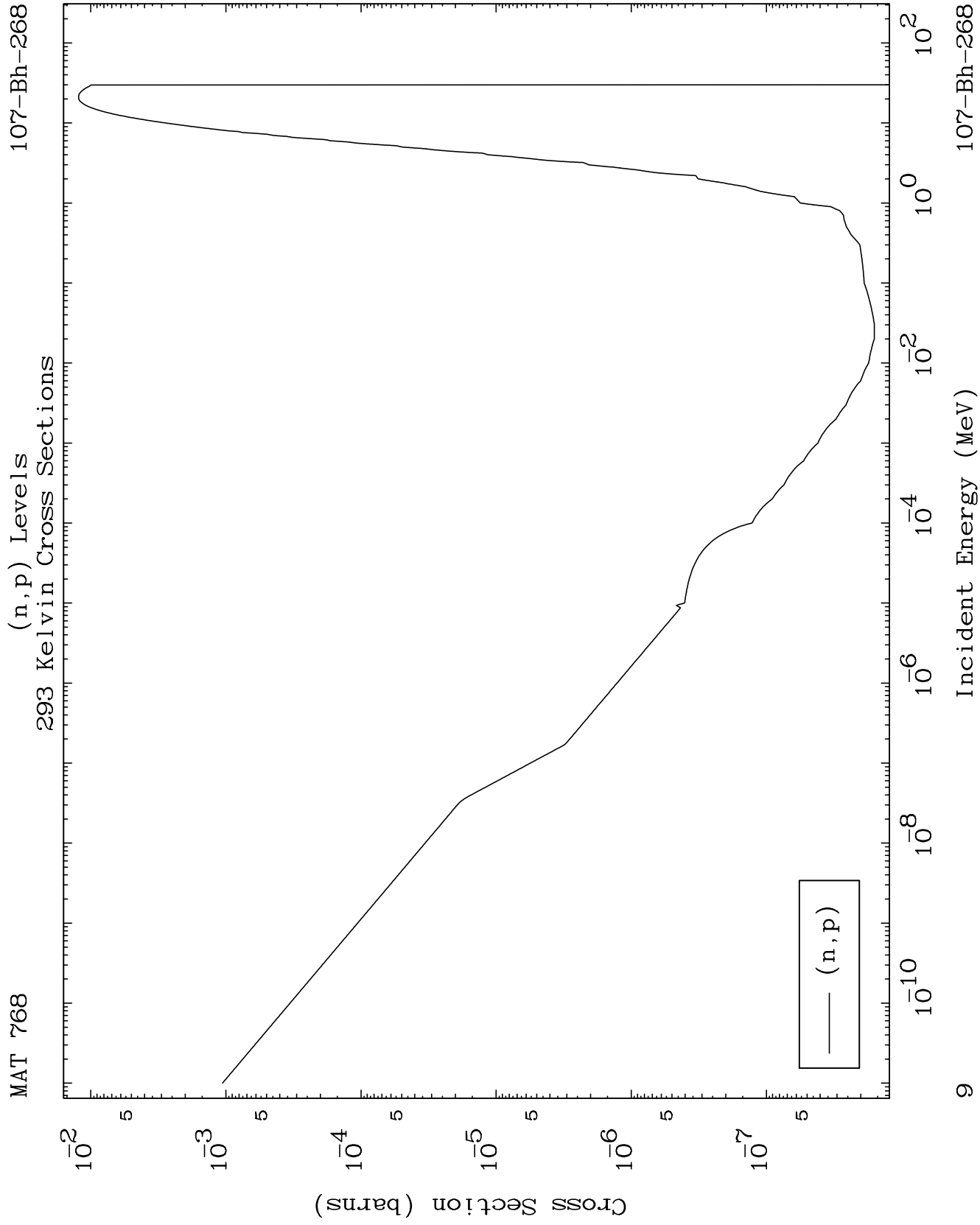
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Incident Energy (MeV)

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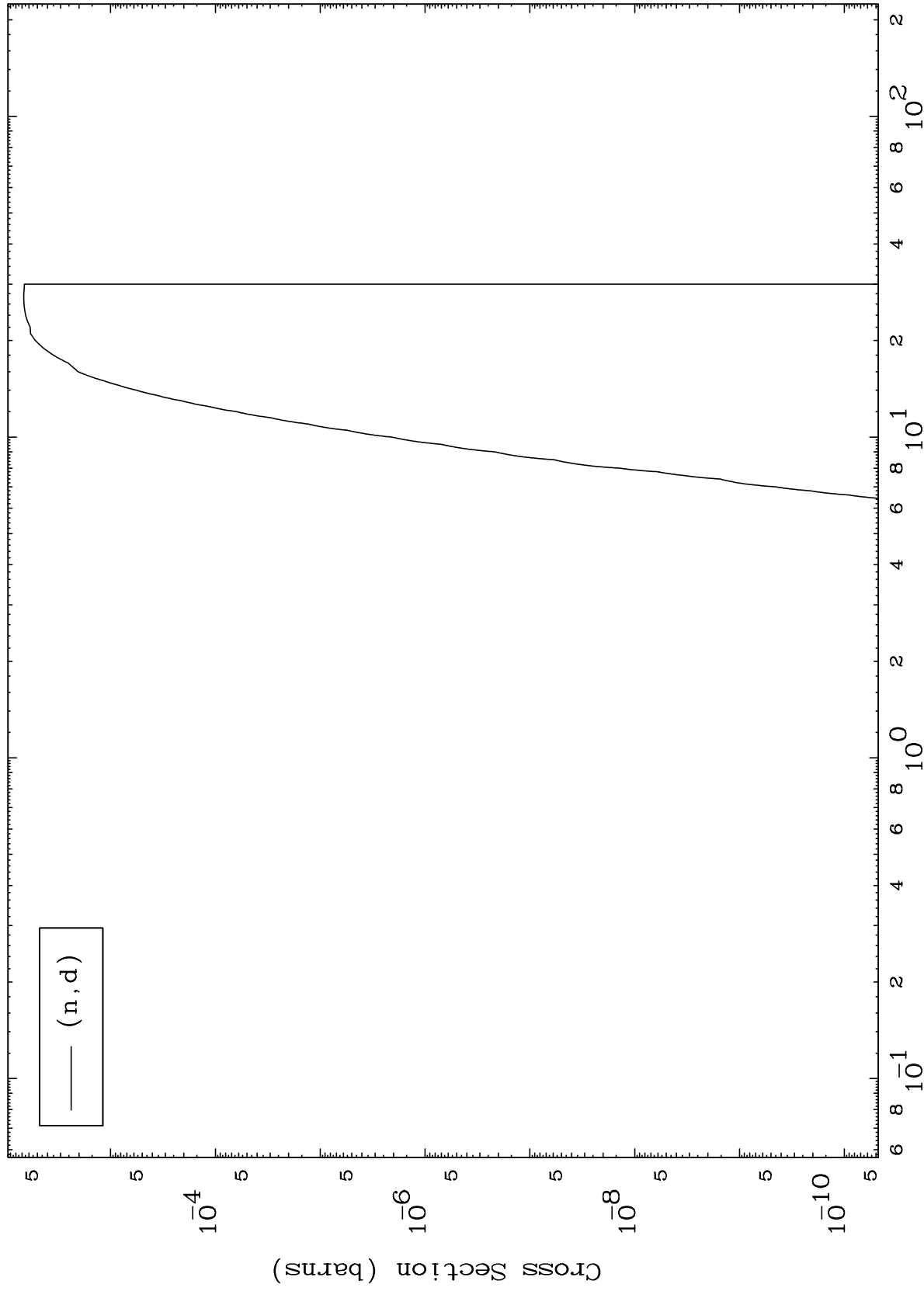


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(n,d) Levels

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293 Kelvin Cross Sections



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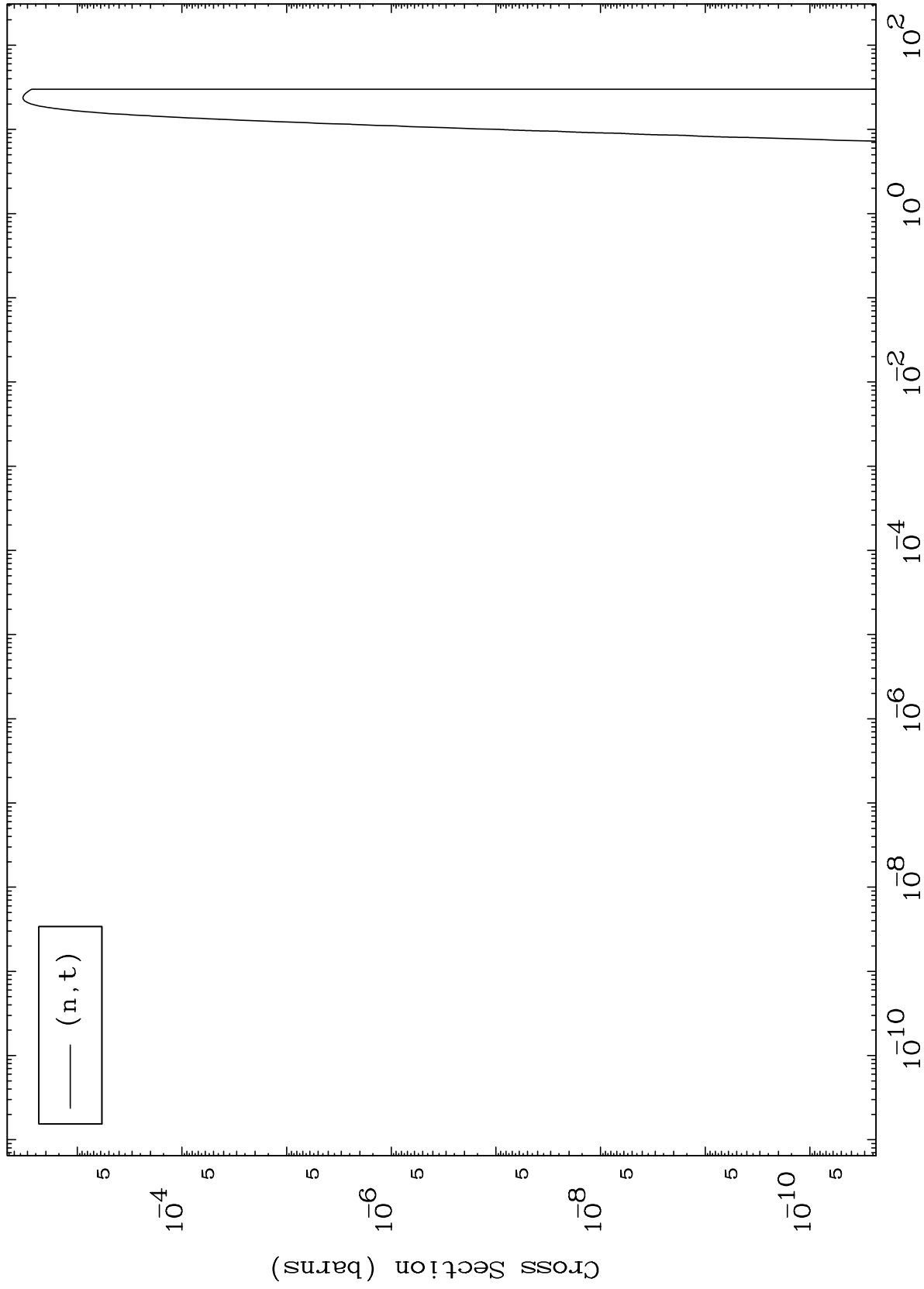
Incident Energy (MeV)

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(n,t) Levels
293 Kelvin Cross Sections

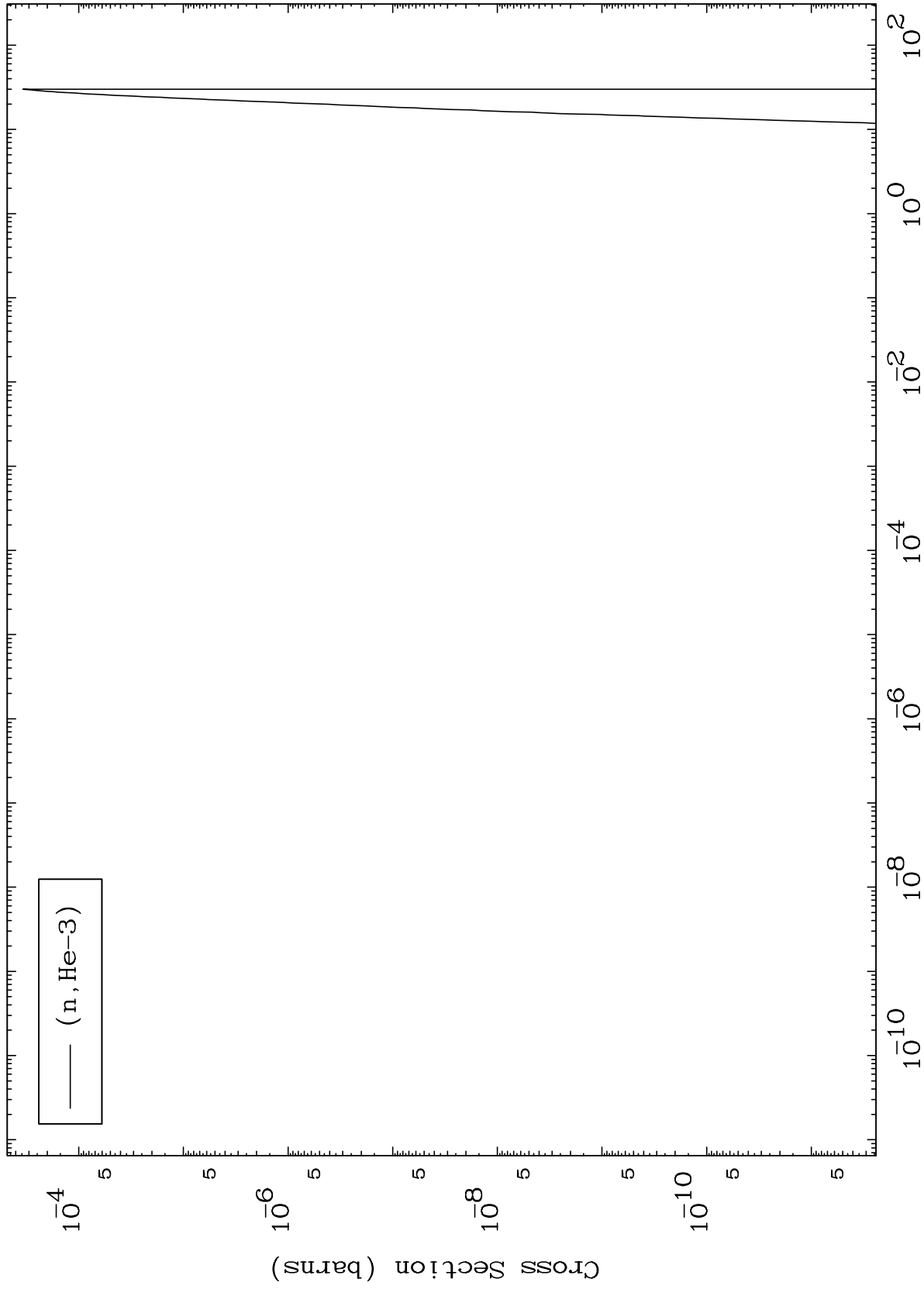
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(n,He3) Levels
293 Kelvin Cross Sections

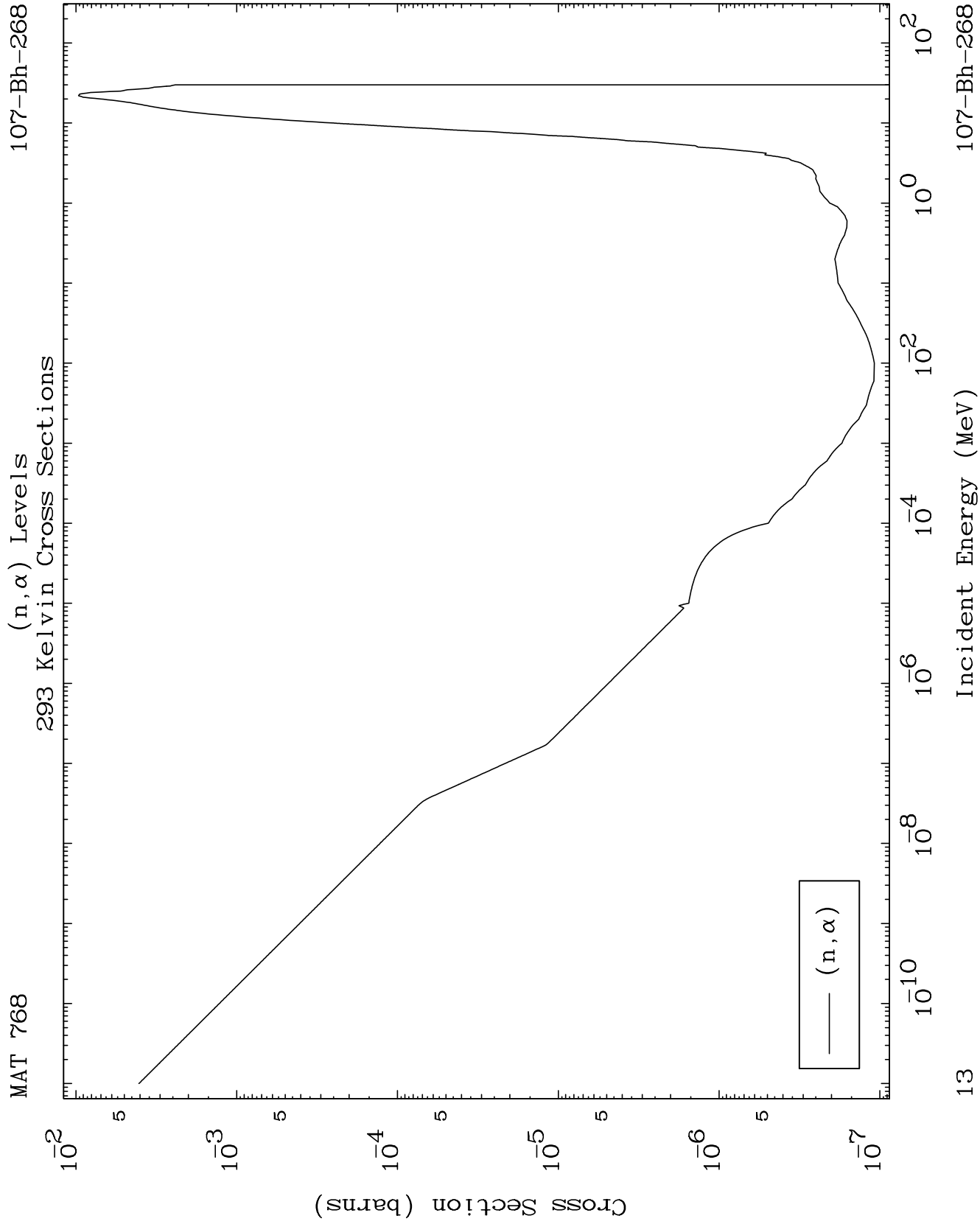
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Incident Energy (MeV)

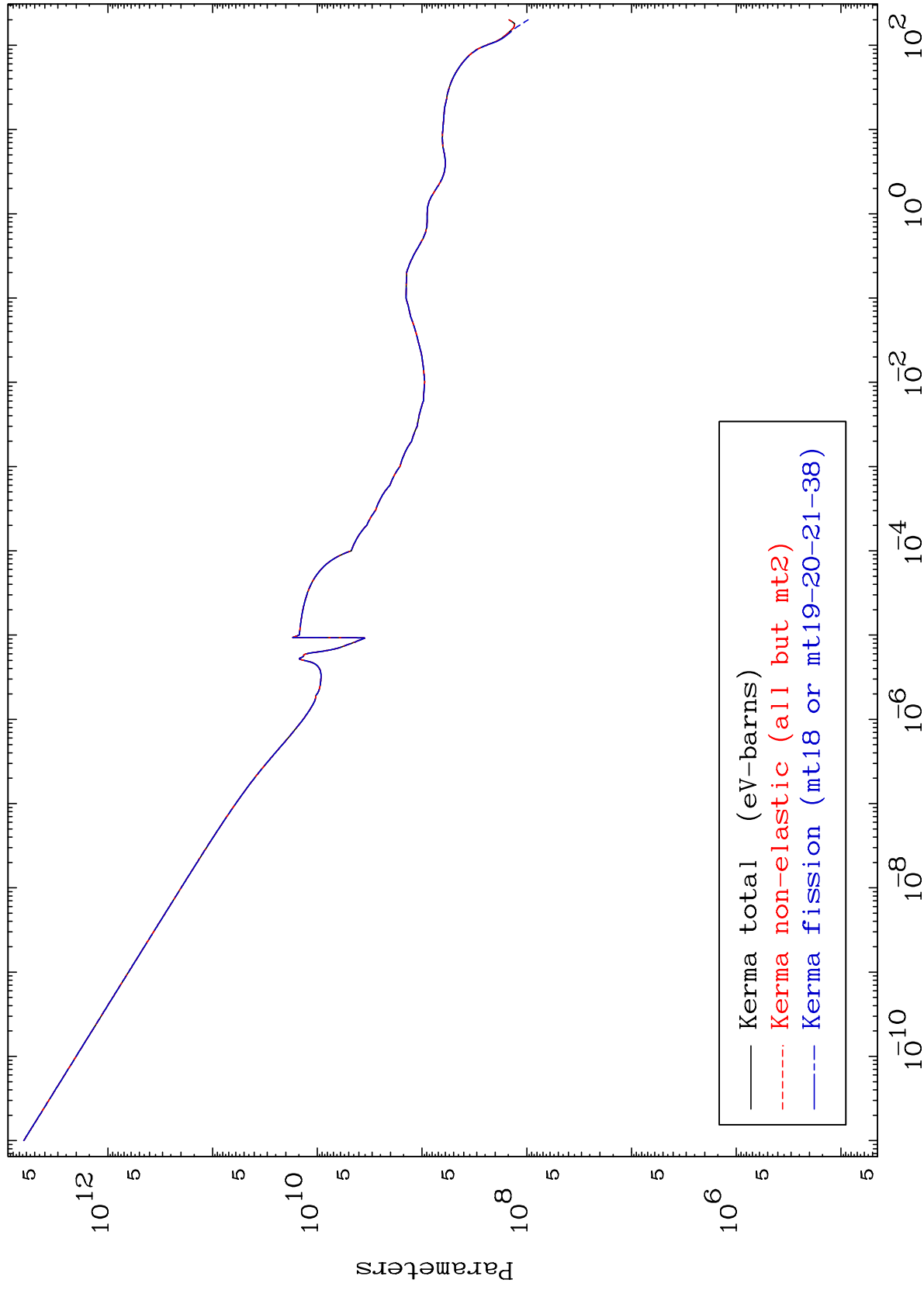
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Energy Release
Parameters

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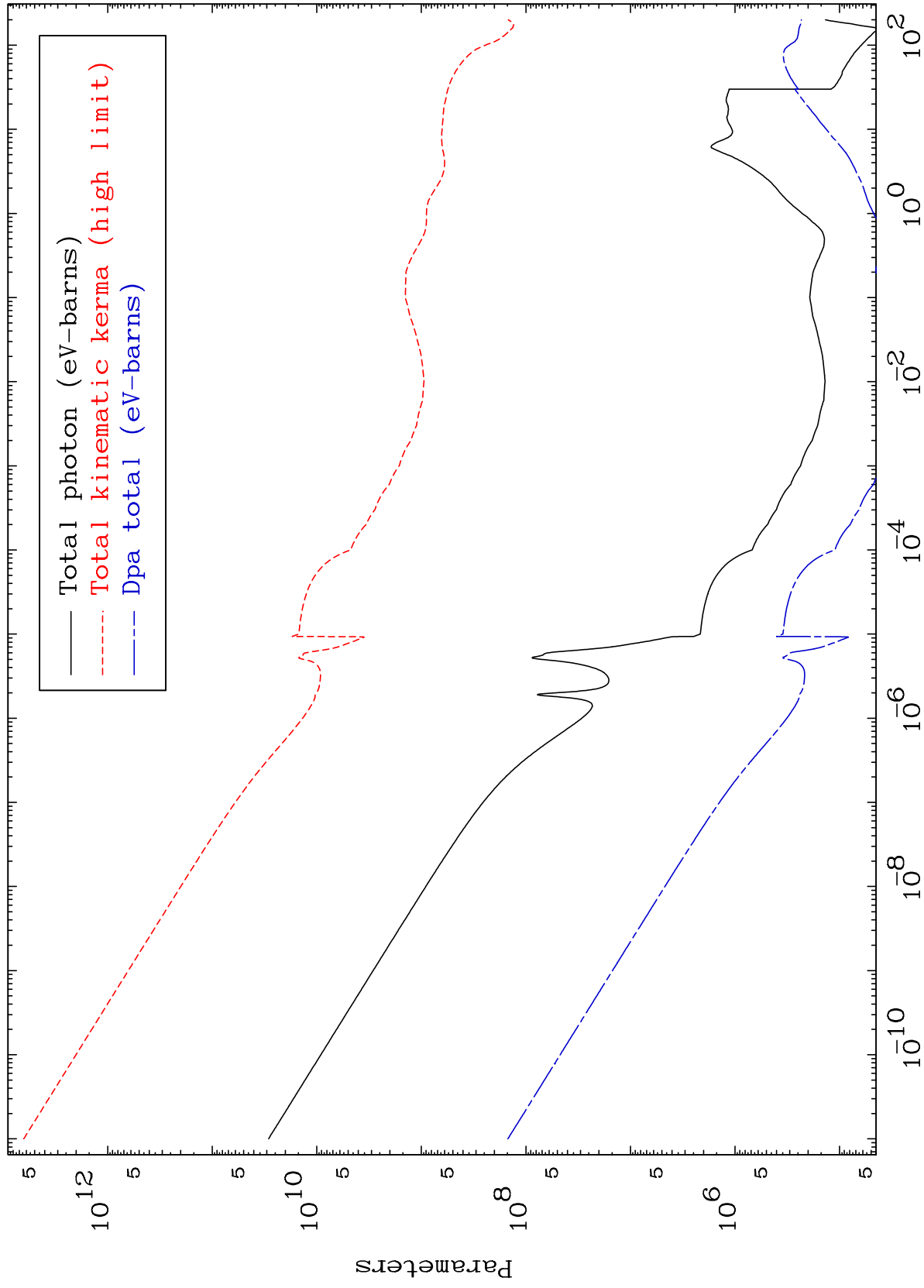
Incident Energy (MeV)

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MAT 768

Energy Release
Parameters

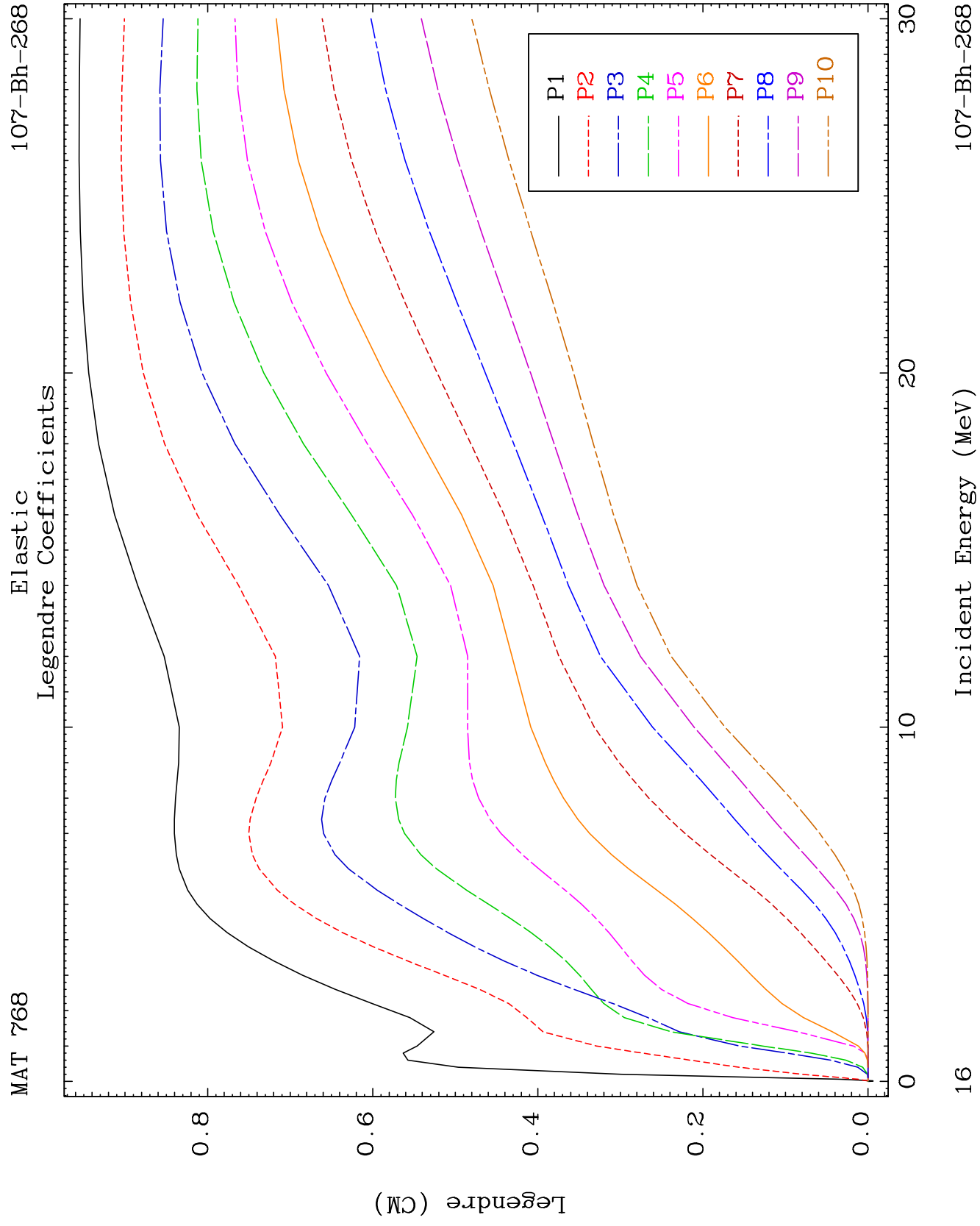
107-Bh-268

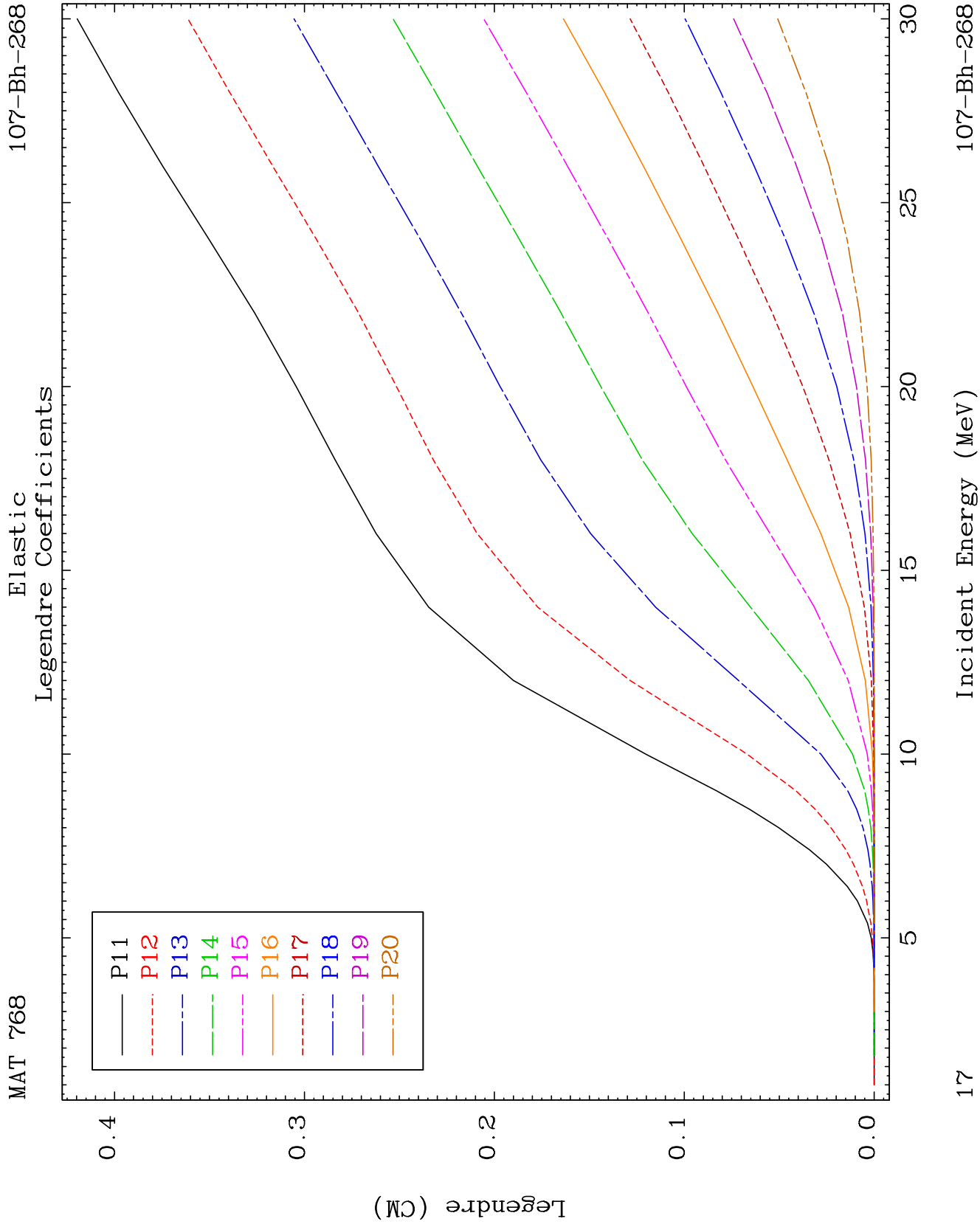


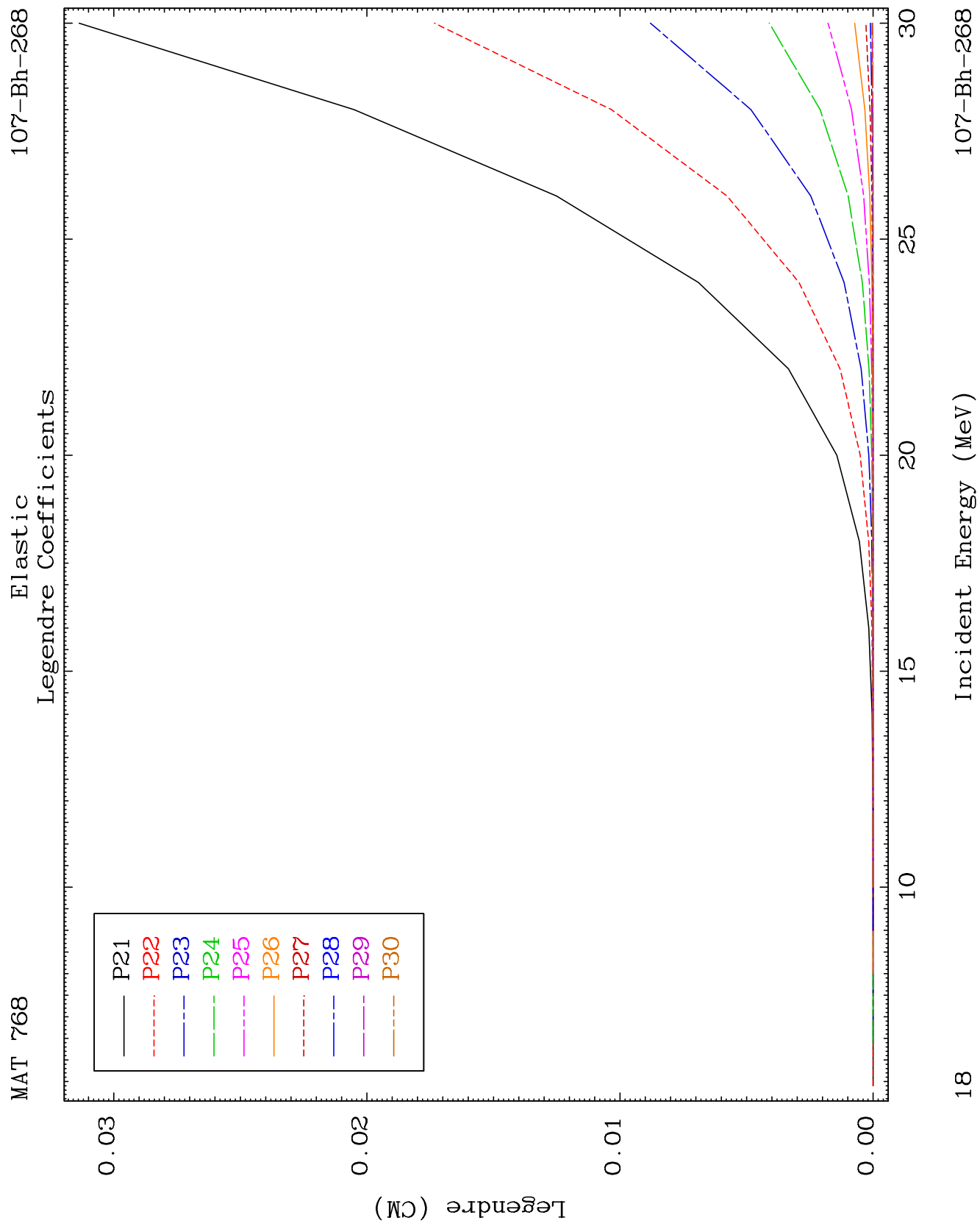
15

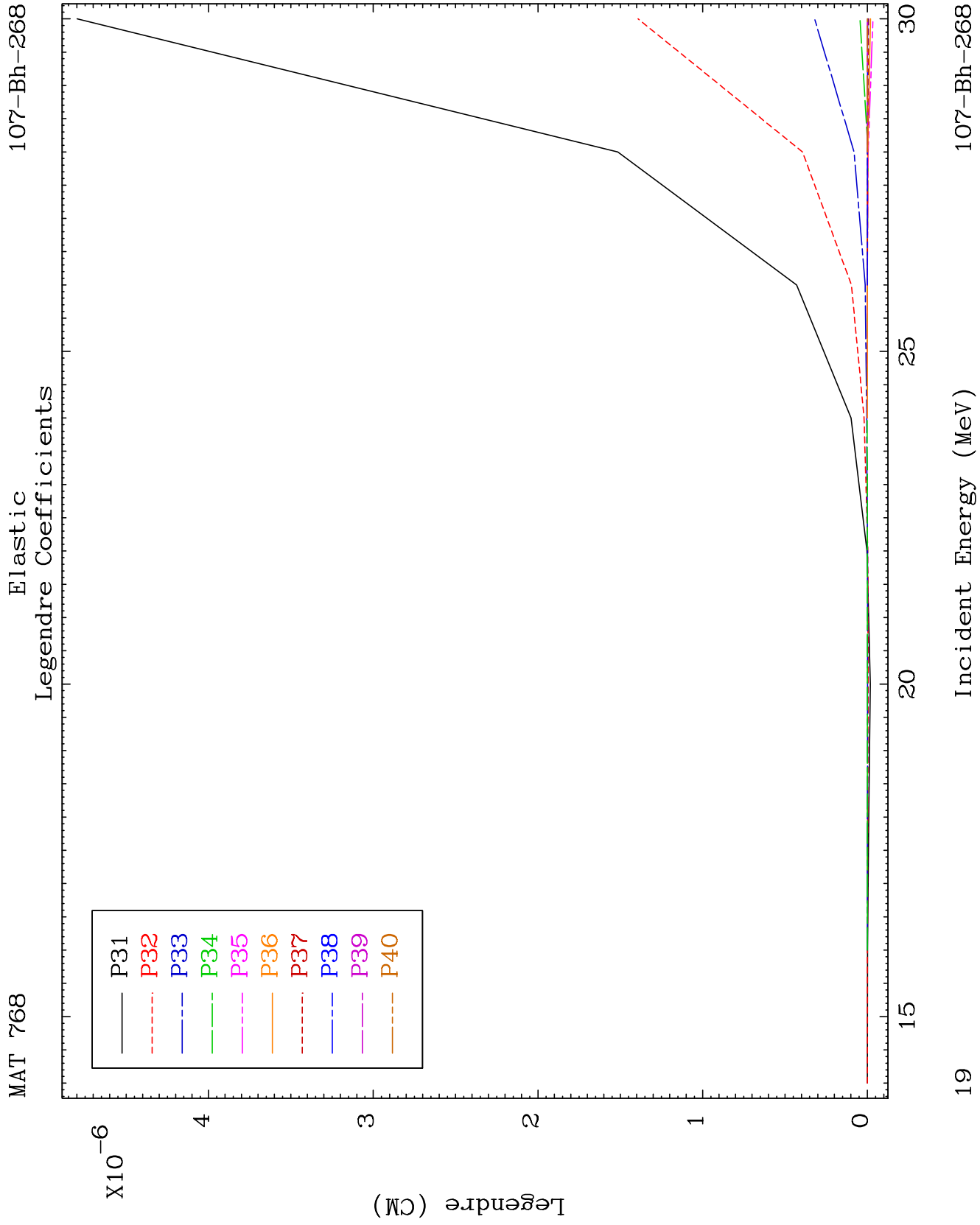
Incident Energy (MeV)

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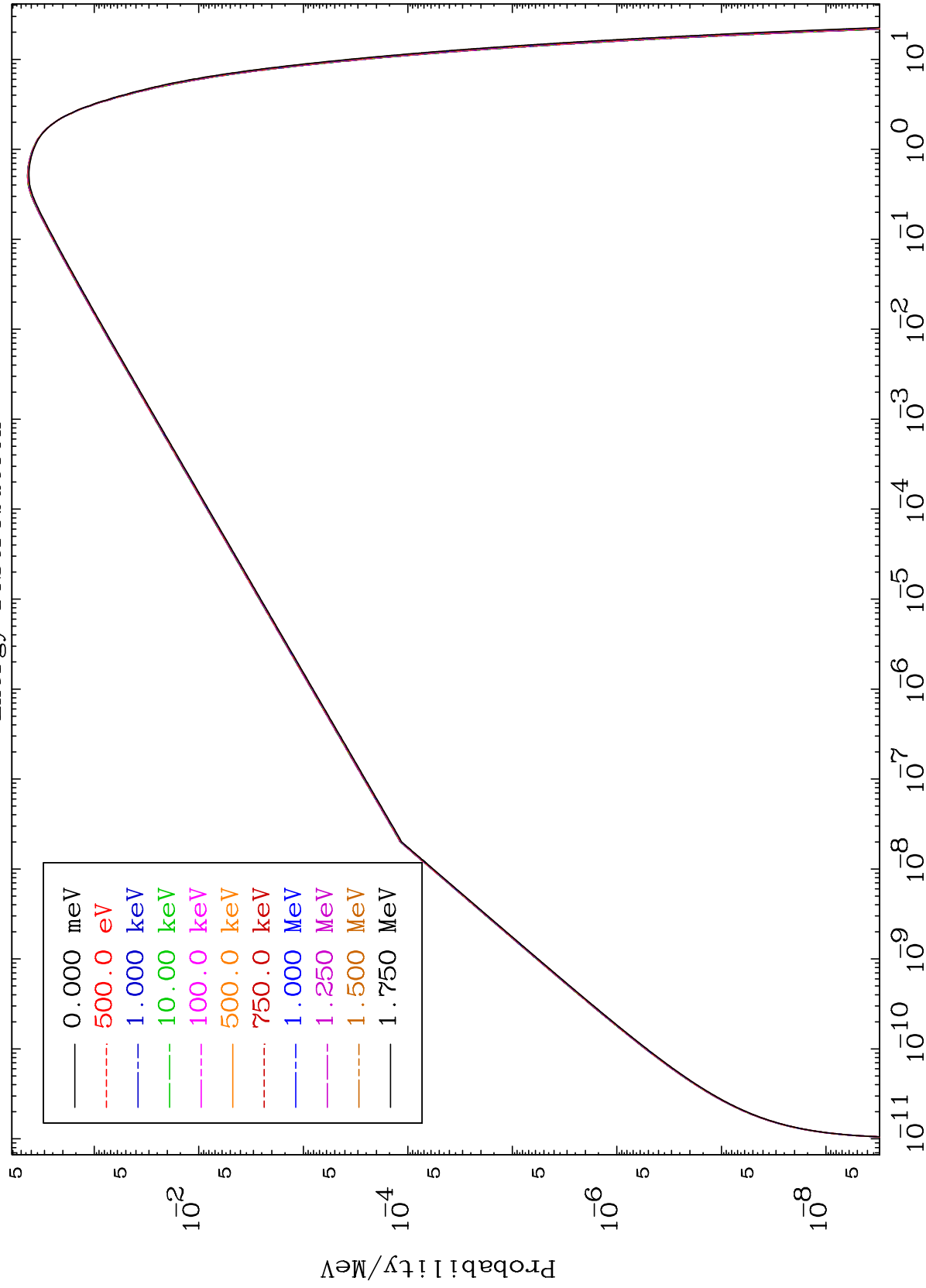


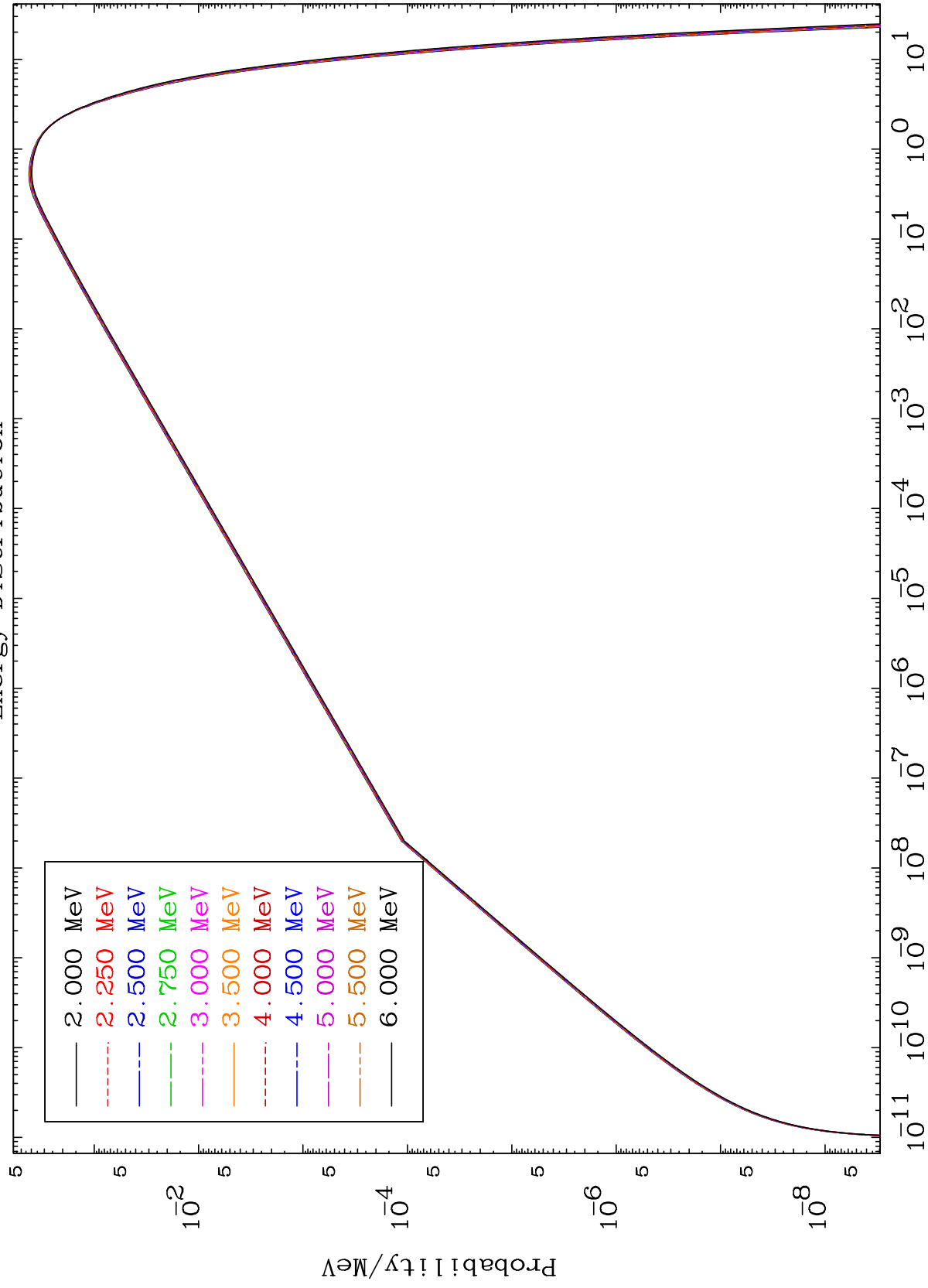


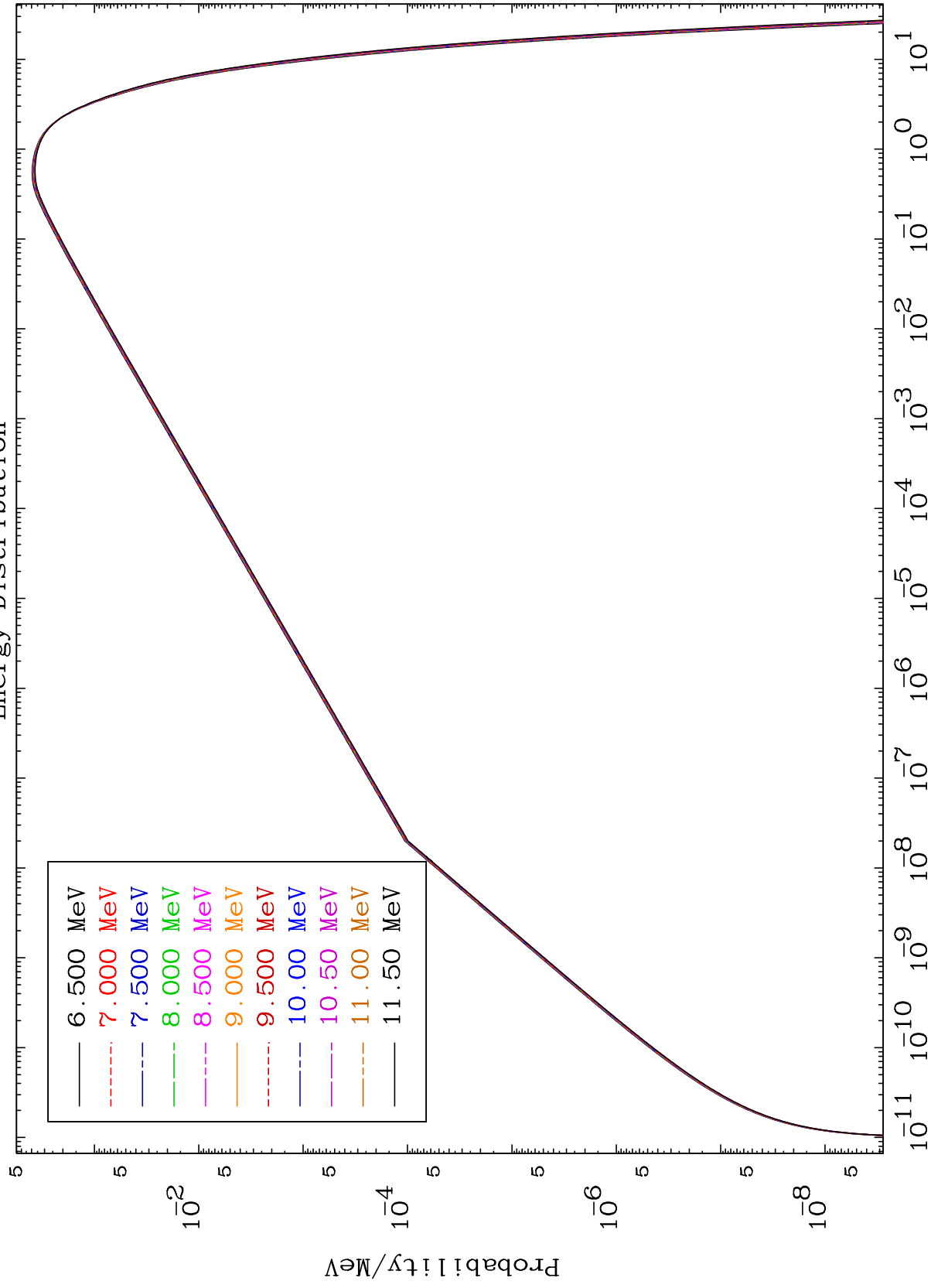
MAT 768

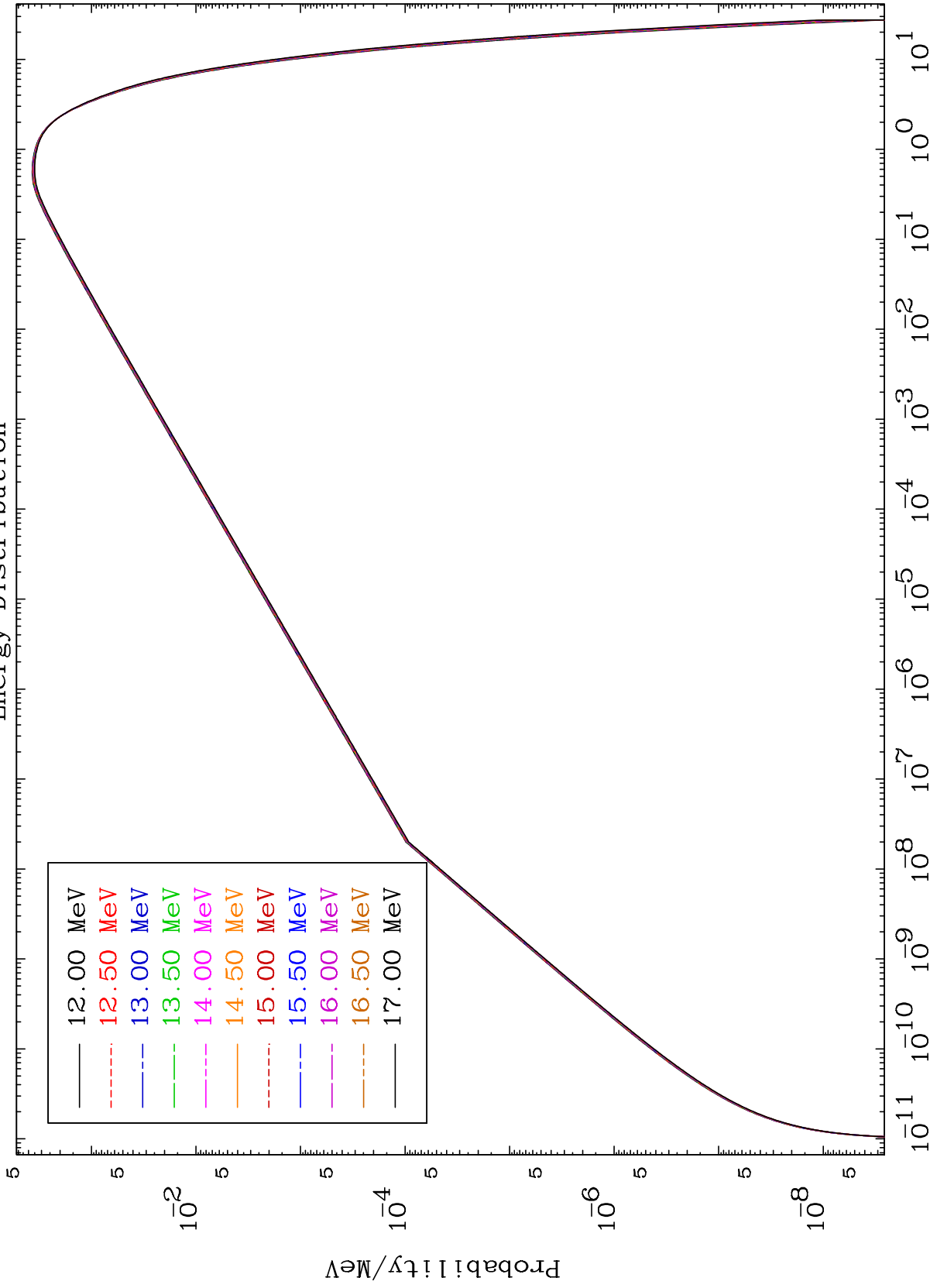
Fission
Energy Distribution

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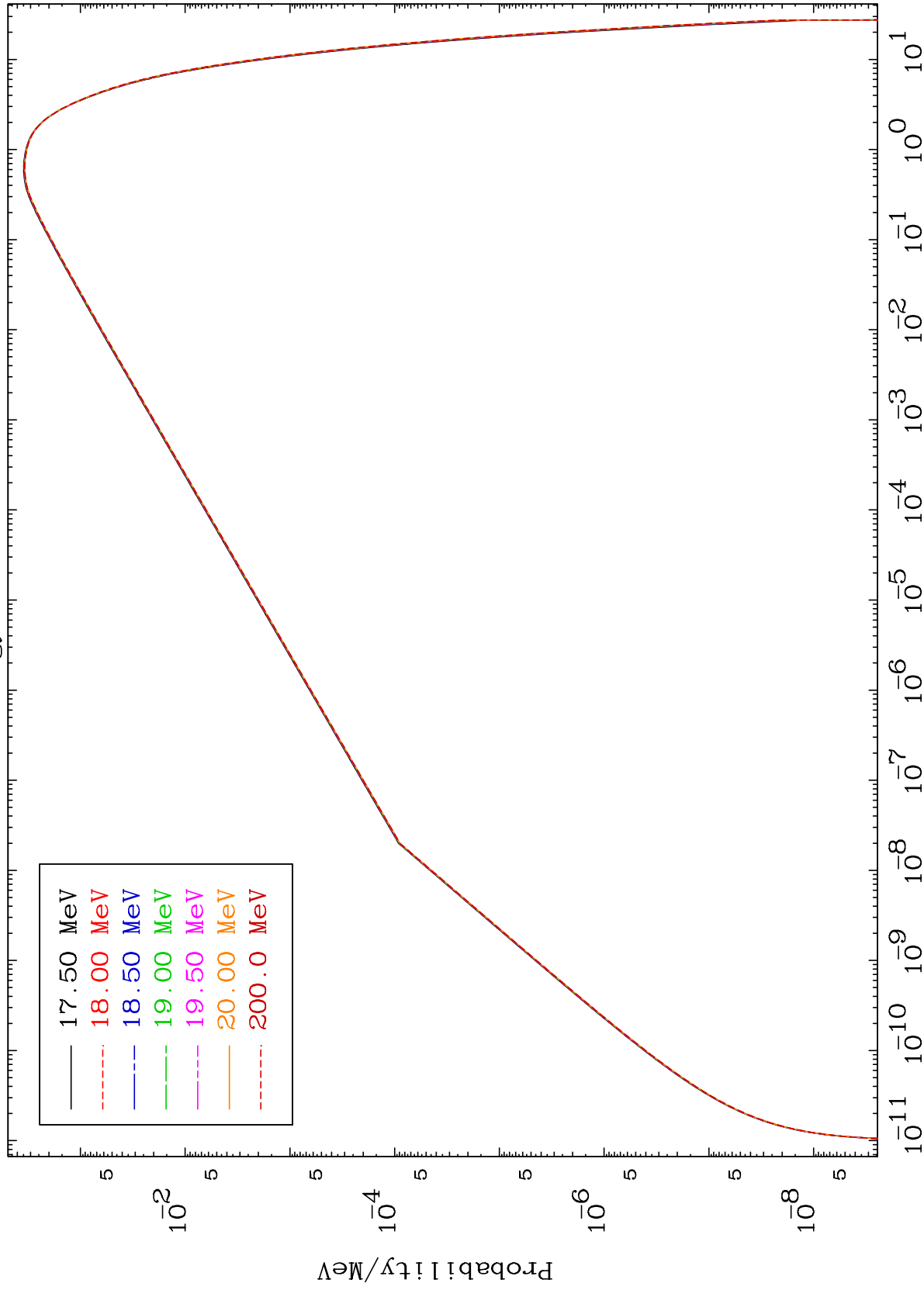




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Fission
Energy Distribution

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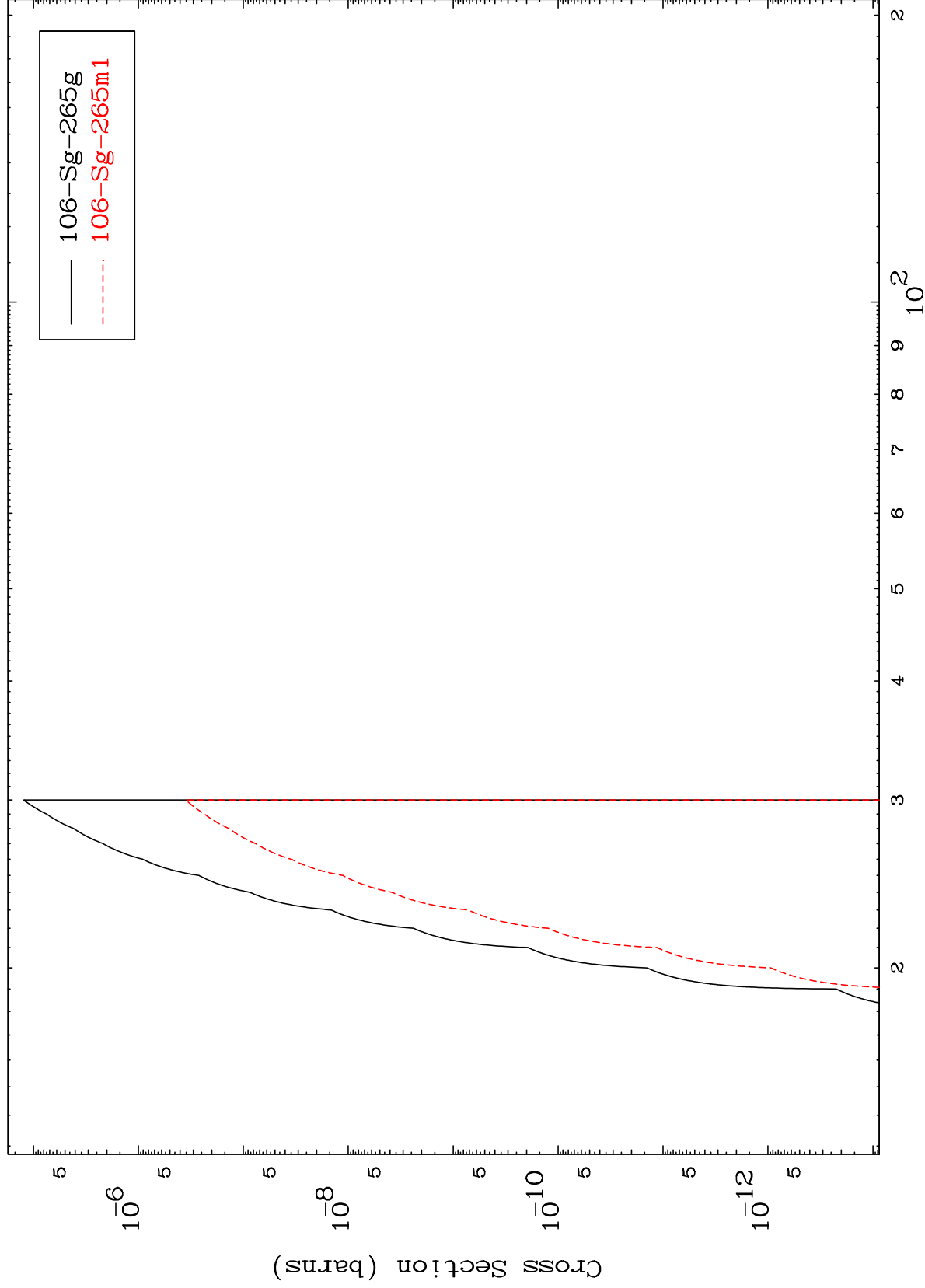


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Secondary Energy (MeV)

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Radionuclide Production Cross Section

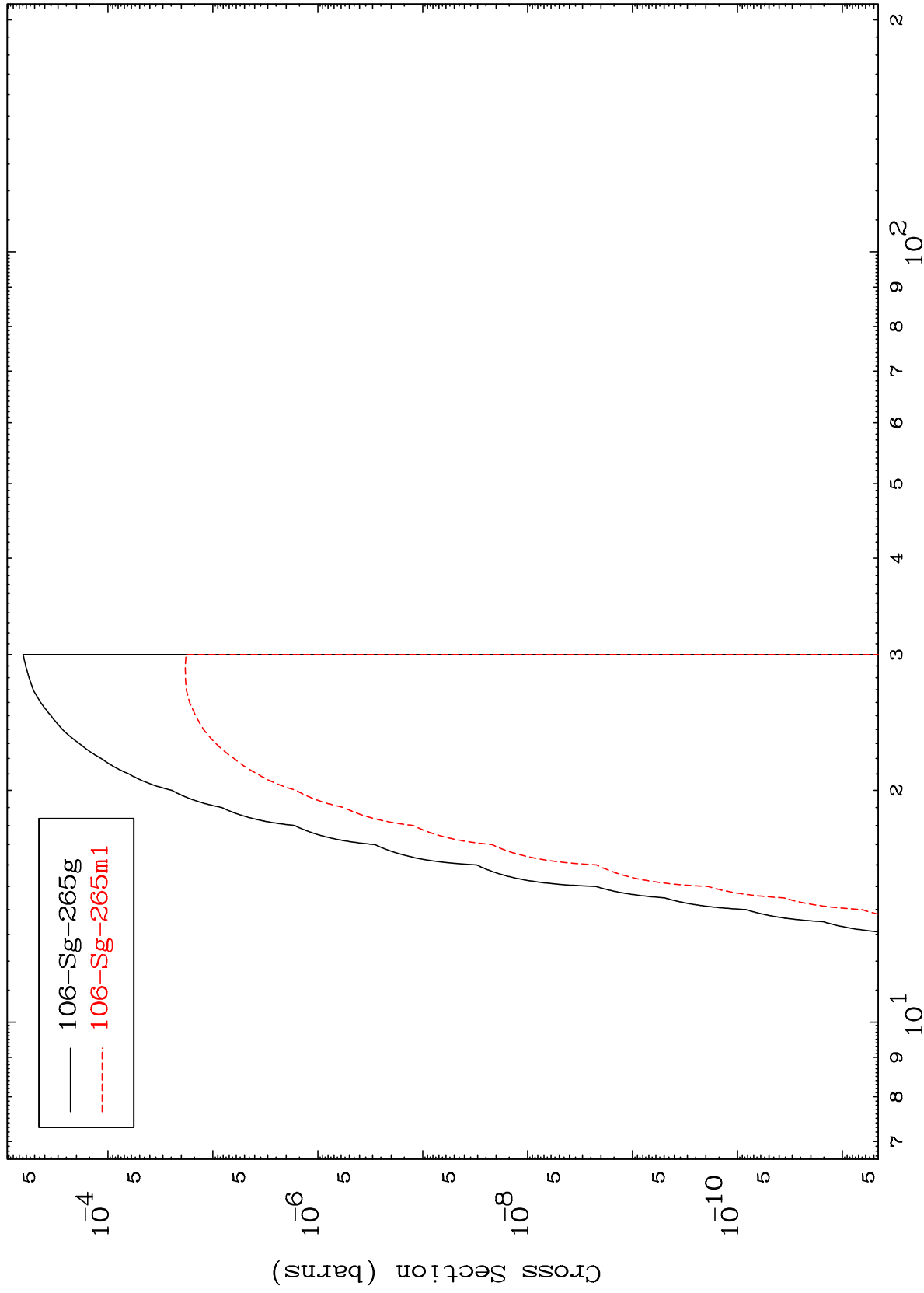


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(n,n') t

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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