

Program EVALPLOT
(Version 2018-1)

by

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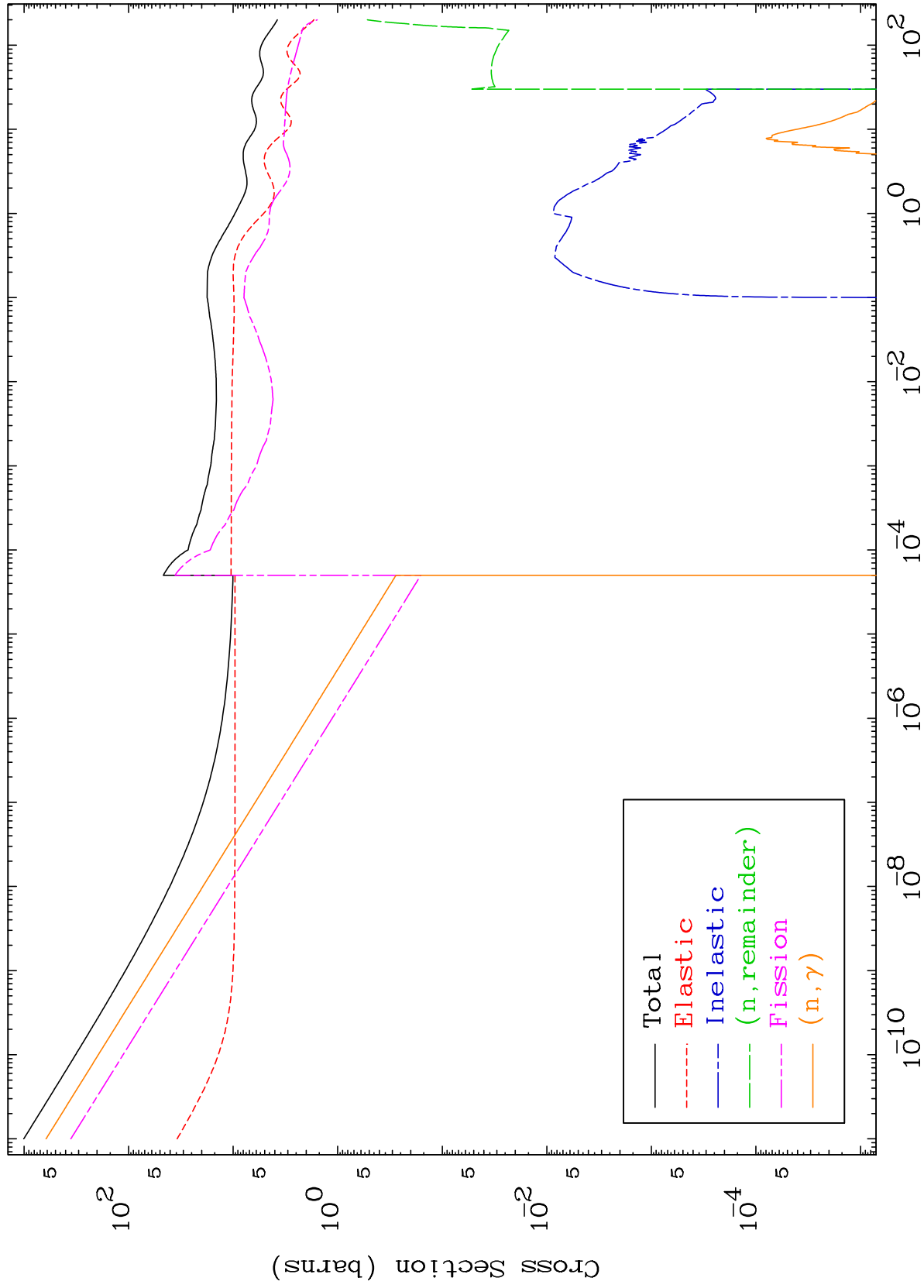
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 773

Major
293 Kelvin Cross Sections

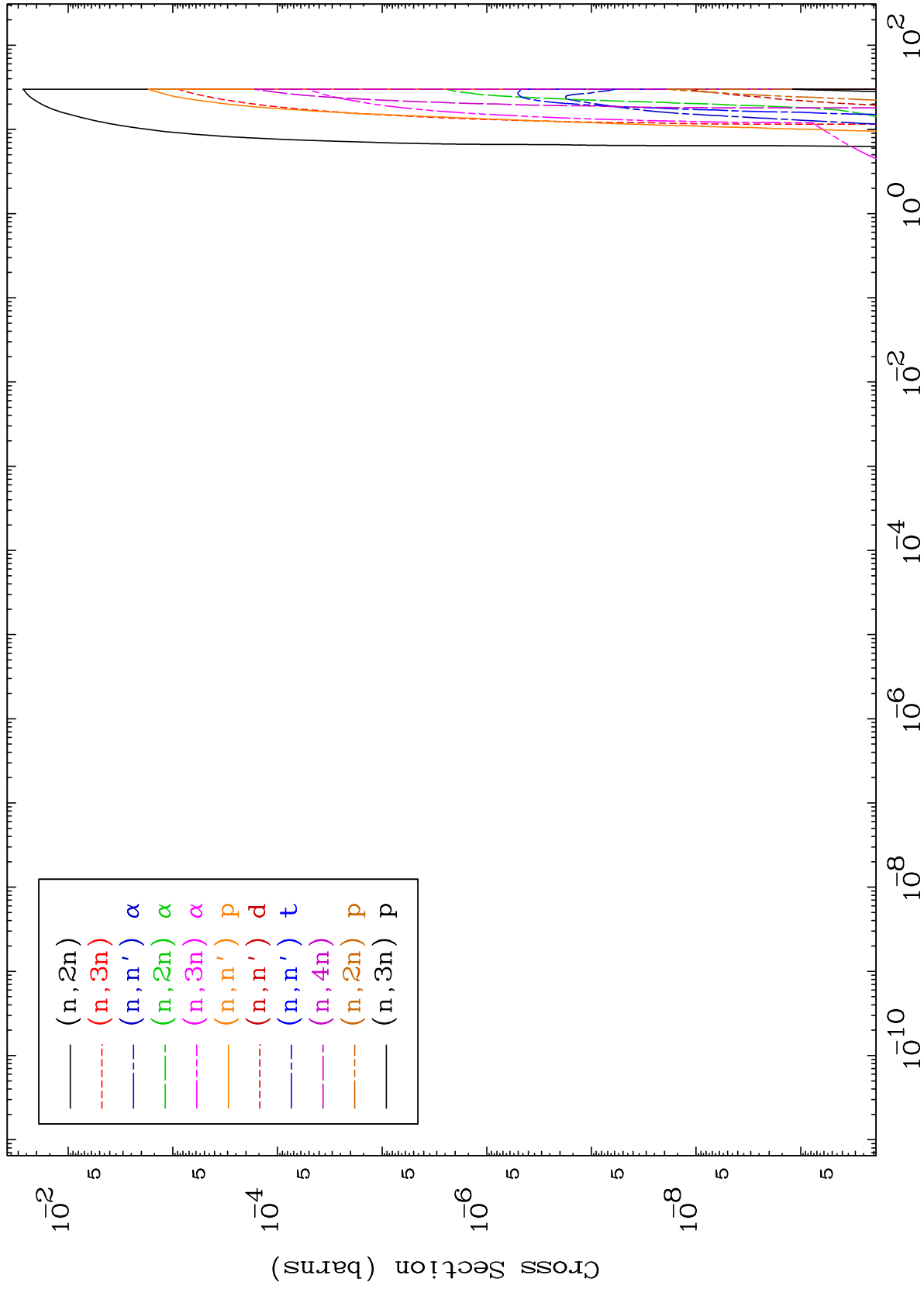
107-Bh-273



MAT 773

Neutron Production
293 Kelvin Cross Sections

107-Bh-273



2

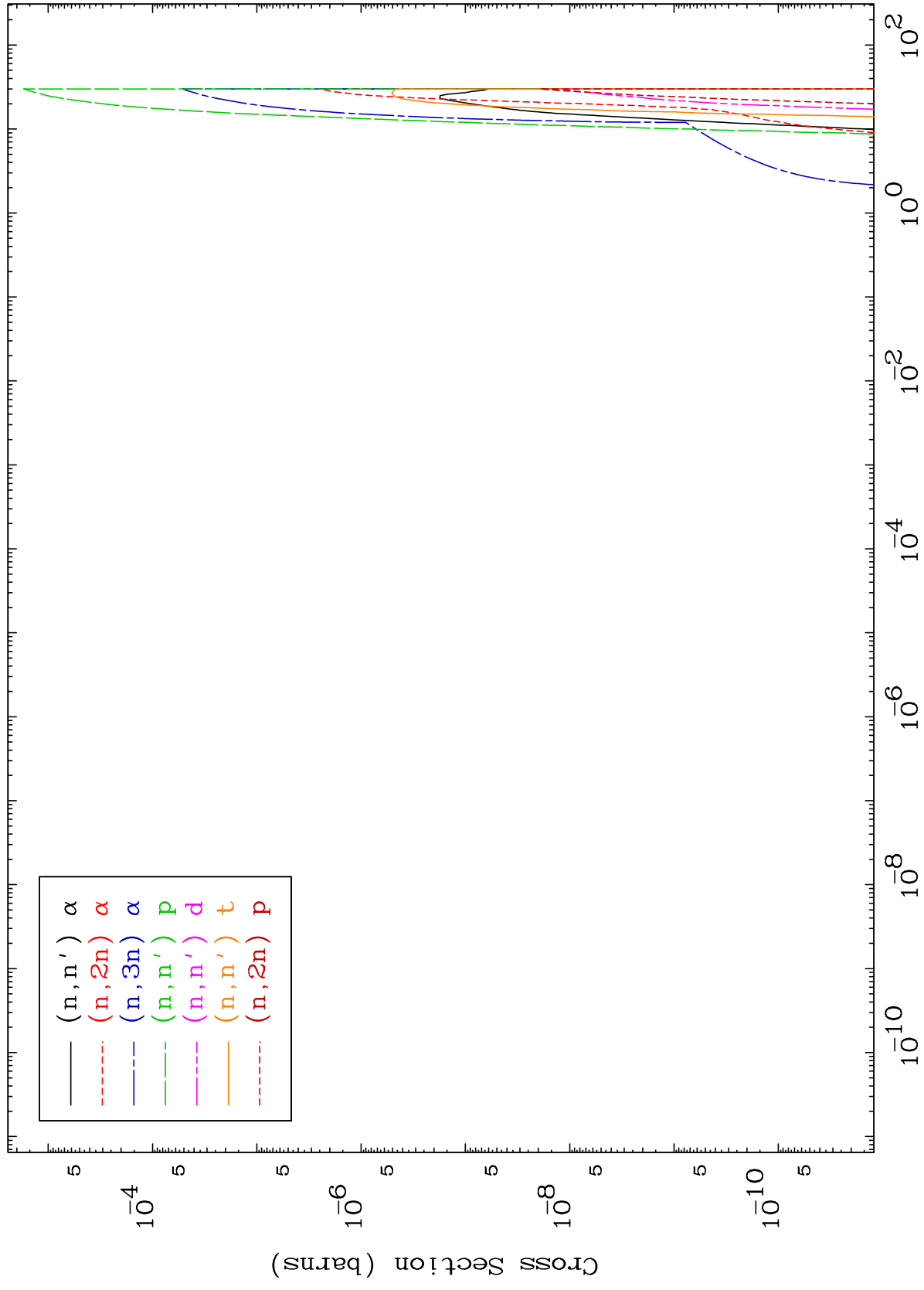
Incident Energy (MeV)

107-Bh-273

MAT 773

Charged Particle
293 Kelvin Cross Sections

107-Bh-273



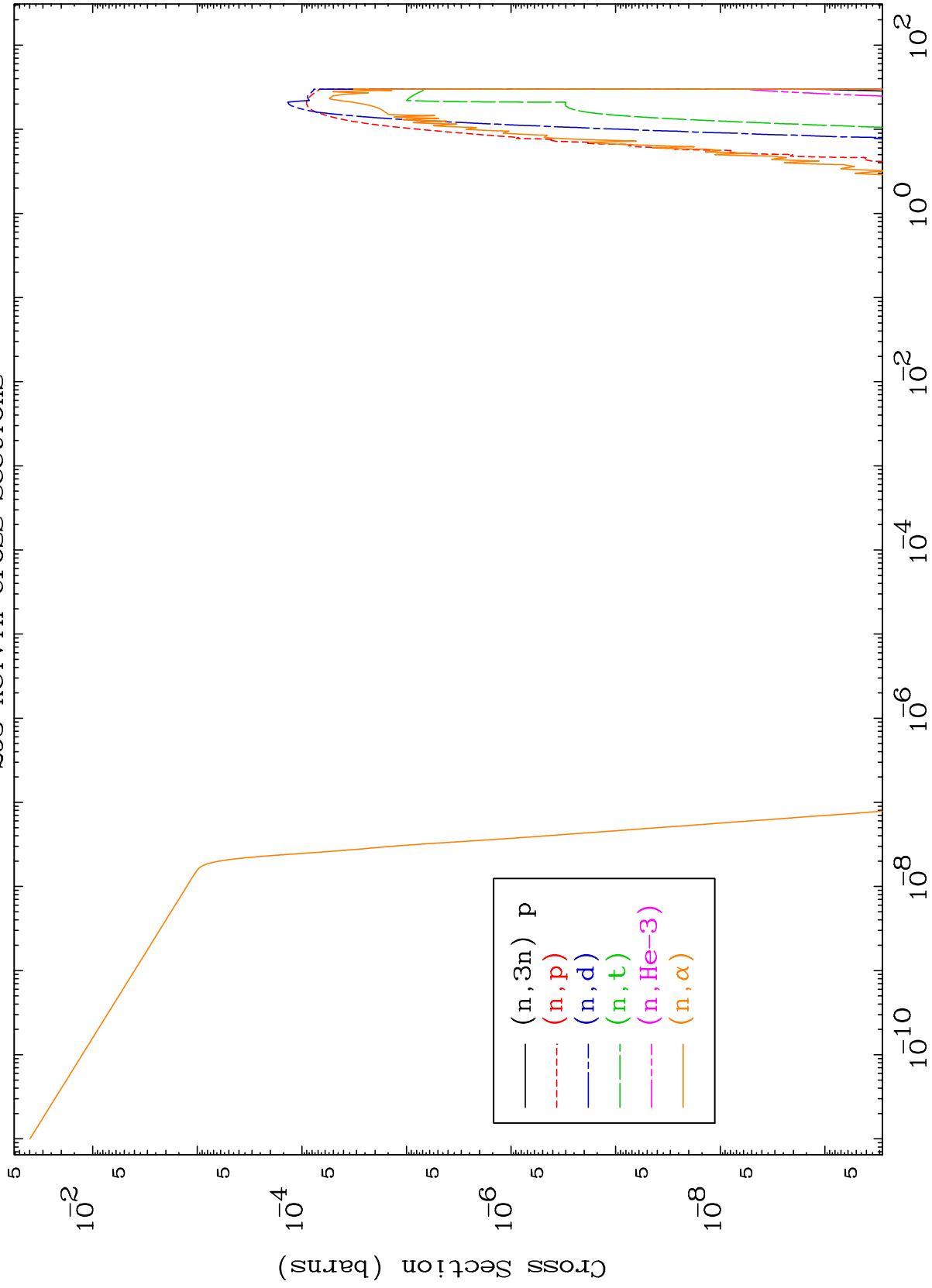
107-Bh-273

Incident Energy (MeV)

MAT 773

Charged Particle
293 Kelvin Cross Sections

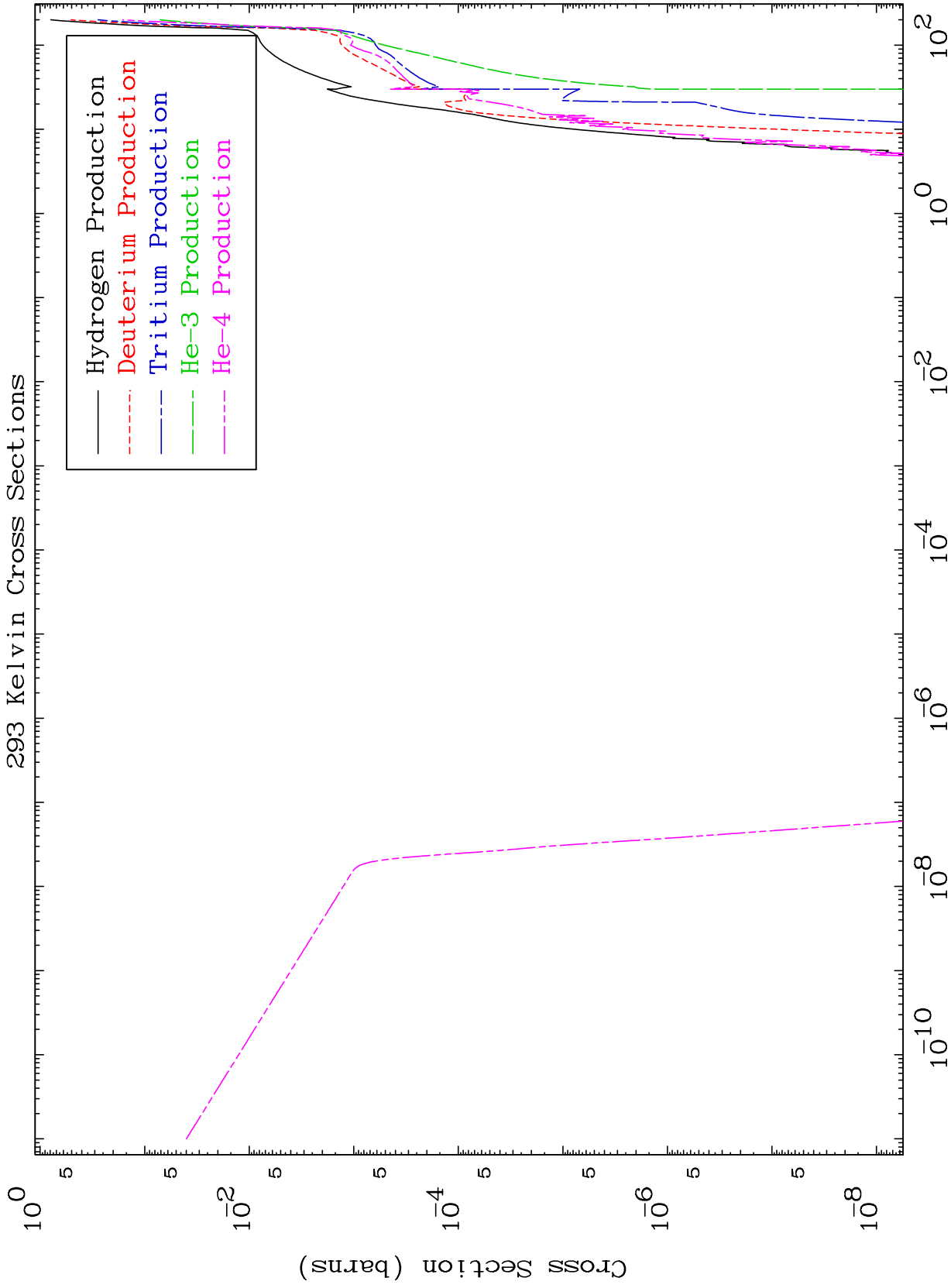
107-Bh-273



MAT 773

Particle Production
293 Kelvin Cross Sections

107-Bh-273



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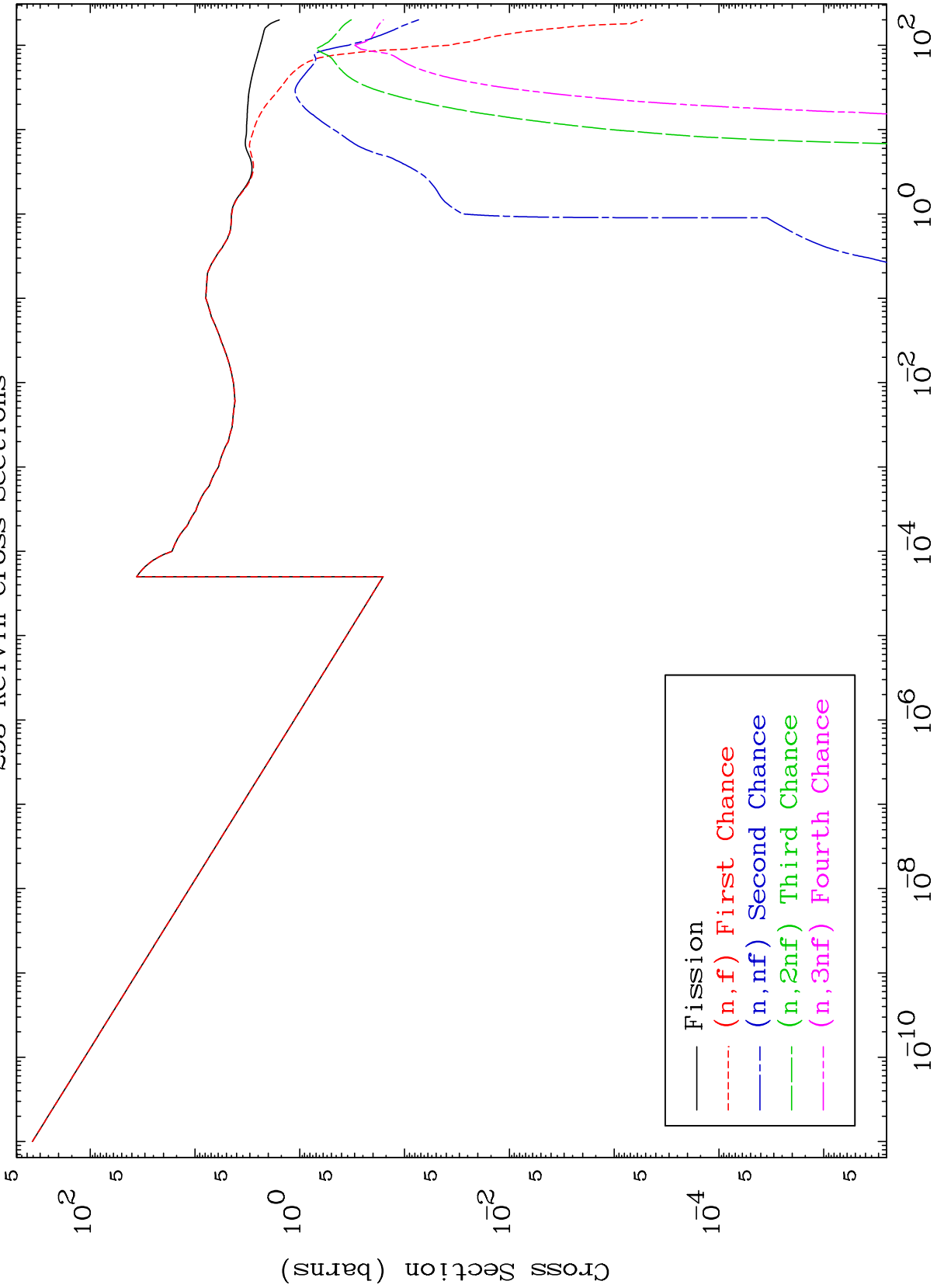
Incident Energy (MeV)

107-Bh-273

MAT 773

Fission
293 Kelvin Cross Sections

107-Bh-273



107-Bh-273

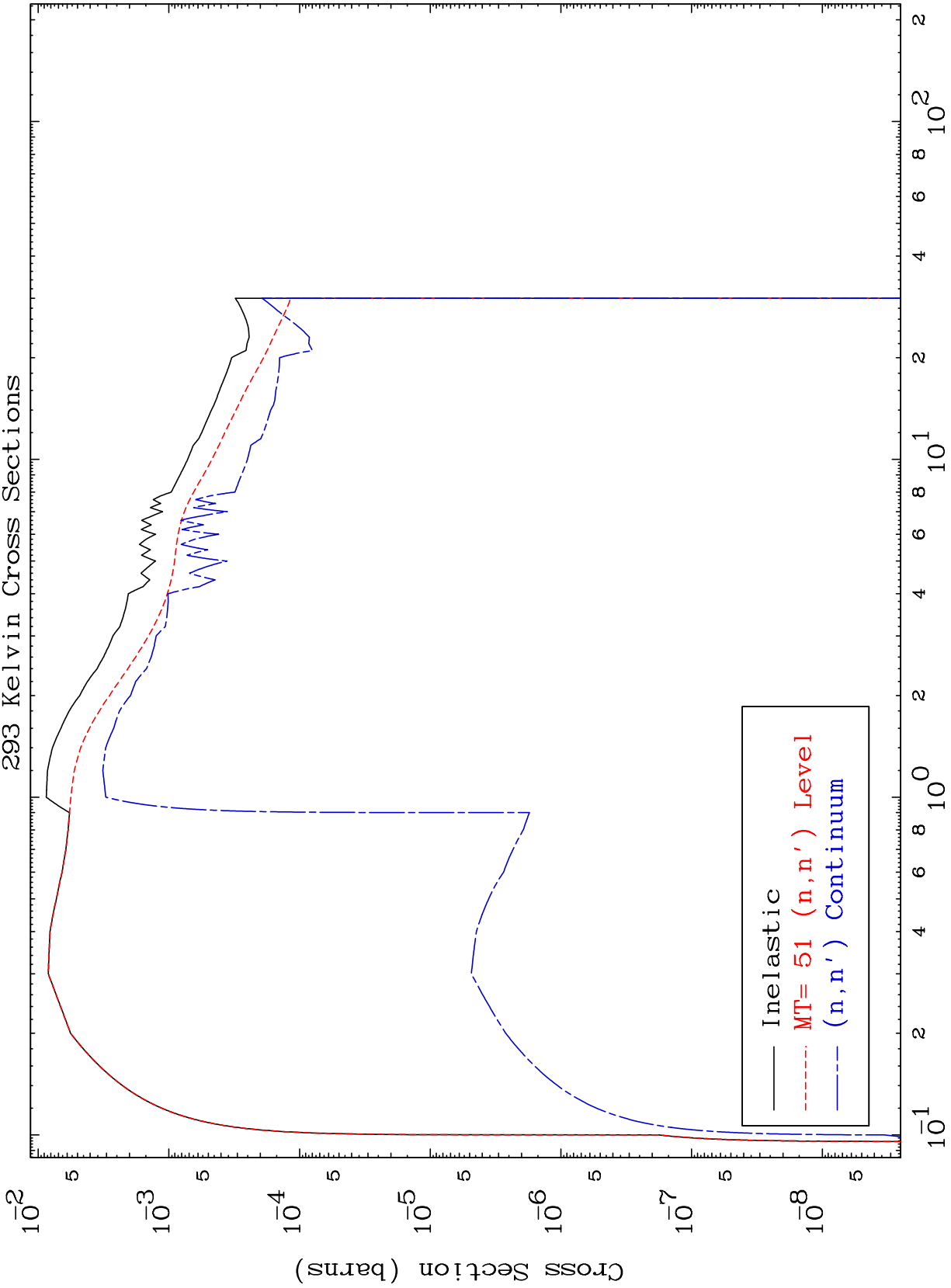
Incident Energy (MeV)

6

MAT 773

(n,n') Level
293 Kelvin Cross Sections

107-Bh-273



107-Bh-273

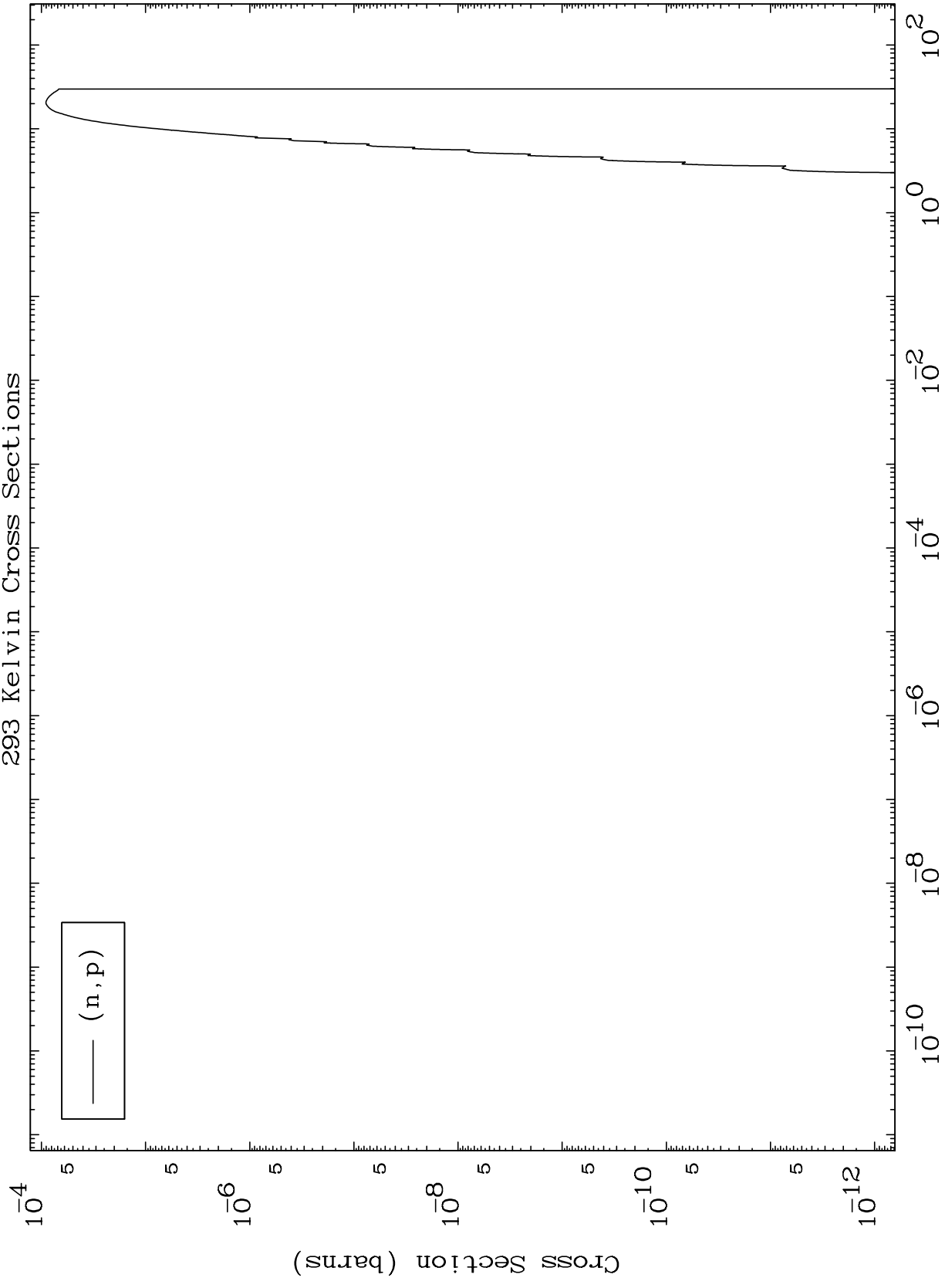
Incident Energy (MeV)

7

MAT 773

(n,p) Levels
293 Kelvin Cross Sections

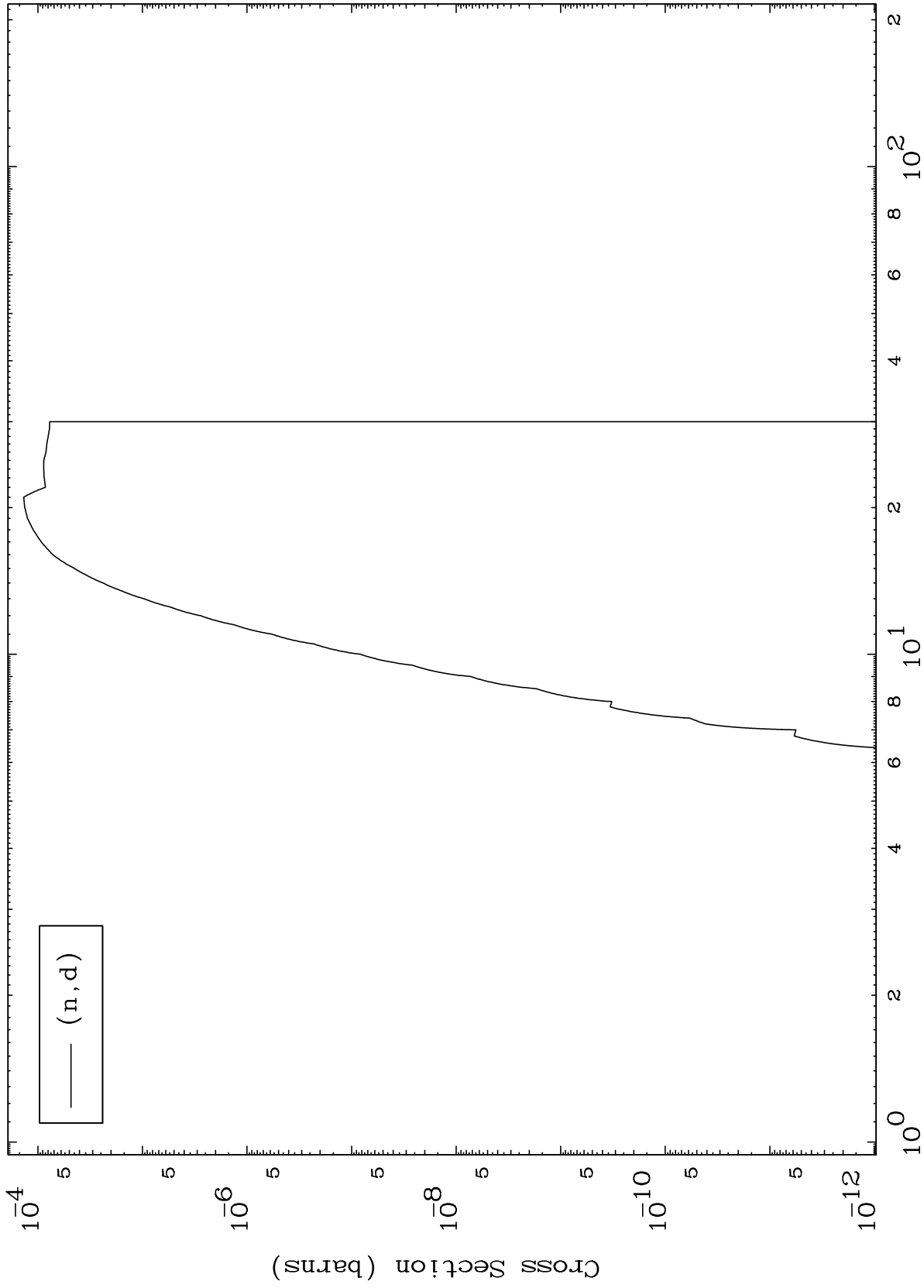
107-Bh-273



MAT 773

(n,d) Levels
293 Kelvin Cross Sections

107-Bh-273



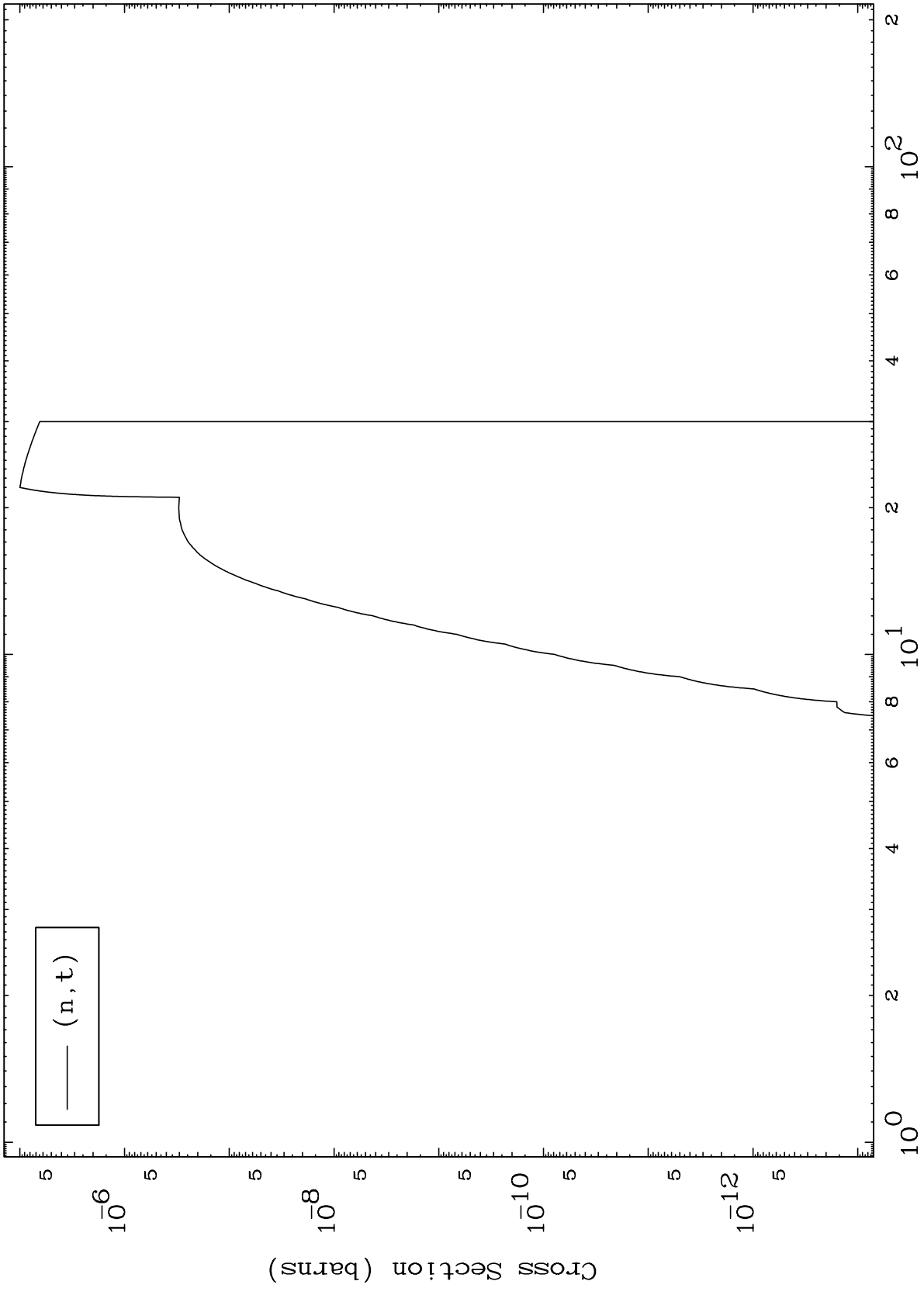
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MAT 773

(n,t) Levels
293 Kelvin Cross Sections

107-Bh-273



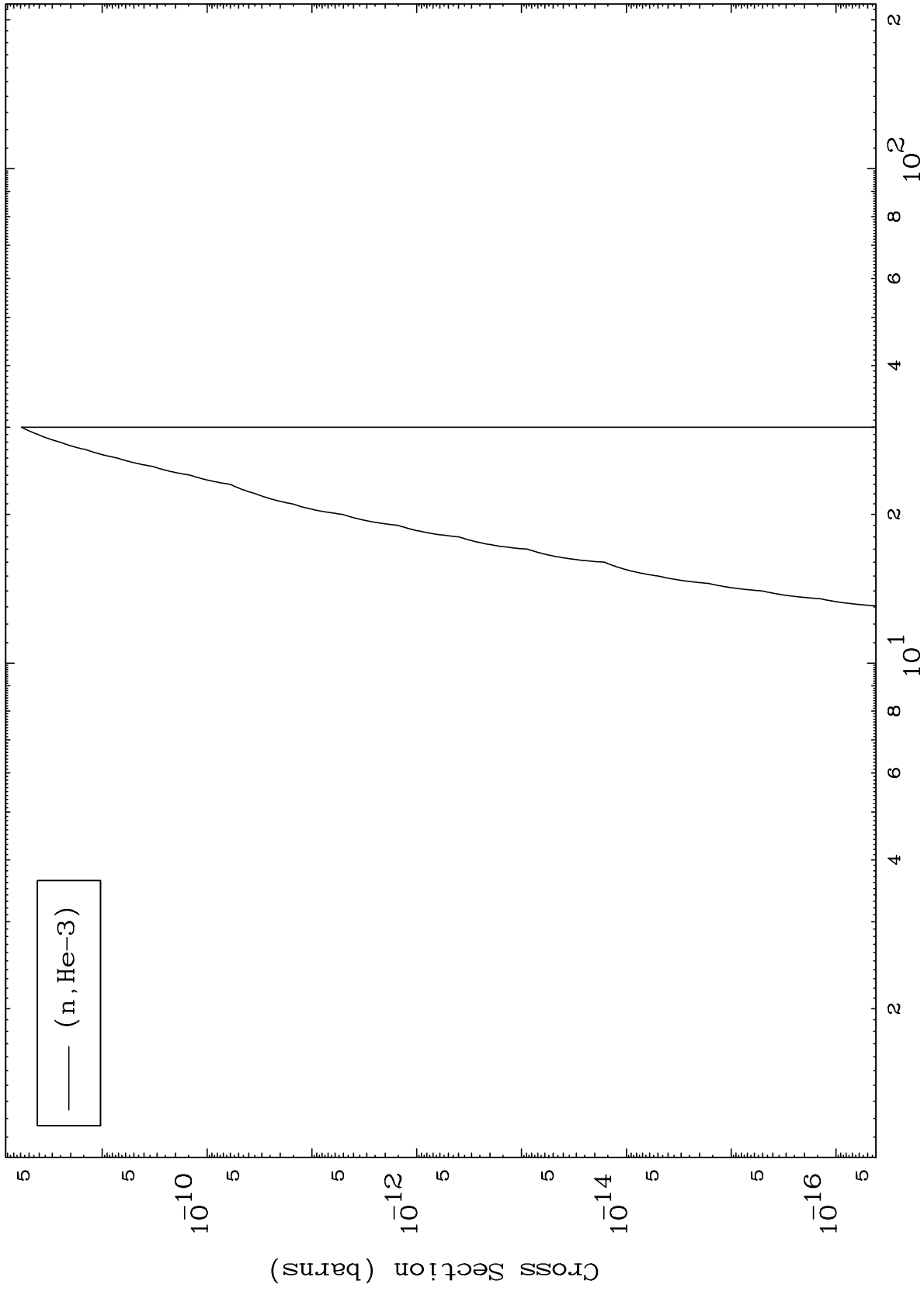
Incident Energy (MeV)

107-Bh-273

MAT 773

(n,He3) Levels
293 Kelvin Cross Sections

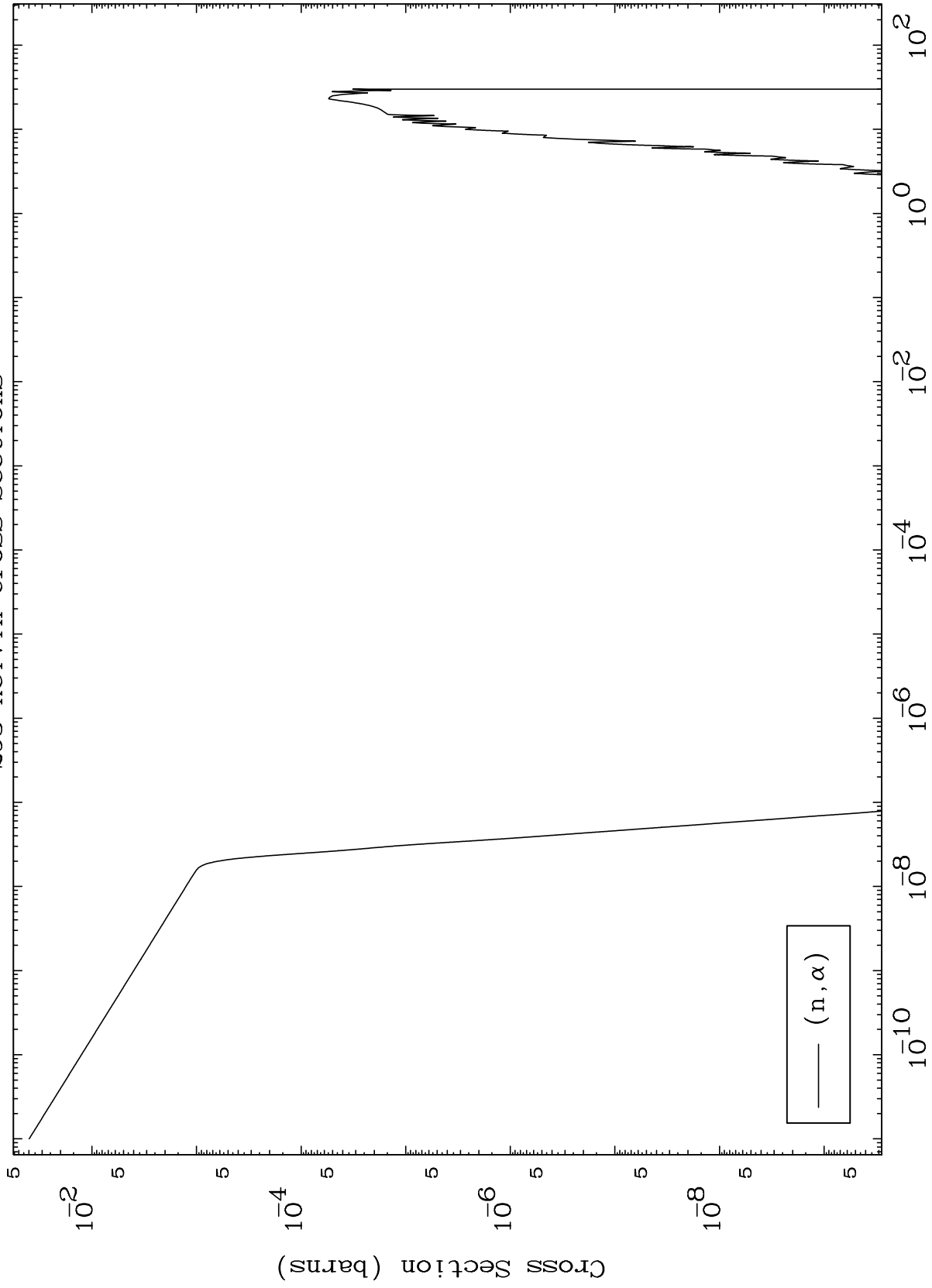
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MAT 773

(n,α) Levels
293 Kelvin Cross Sections

107-Bh-273



12

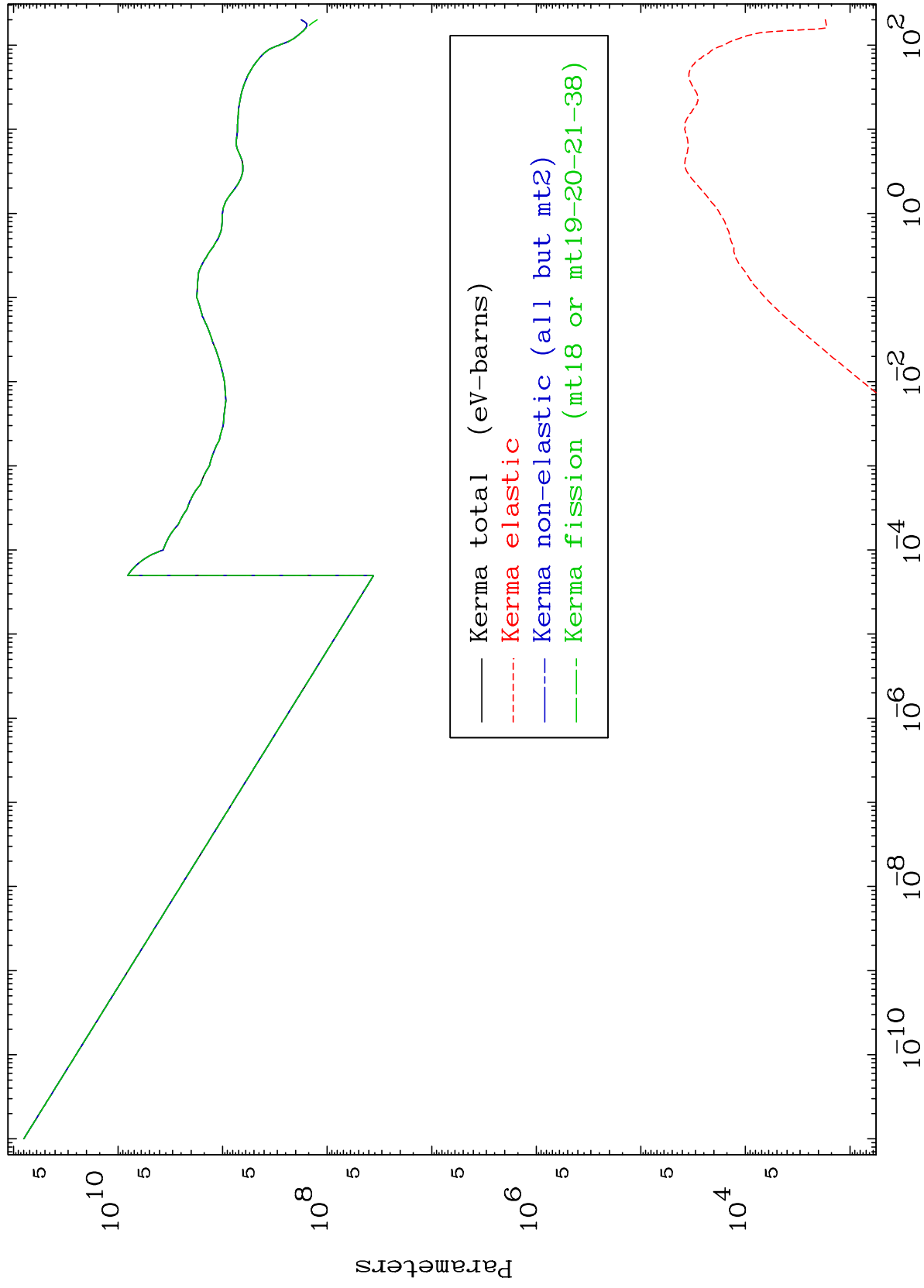
Incident Energy (MeV)

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MAT 773

Energy Release
Parameters

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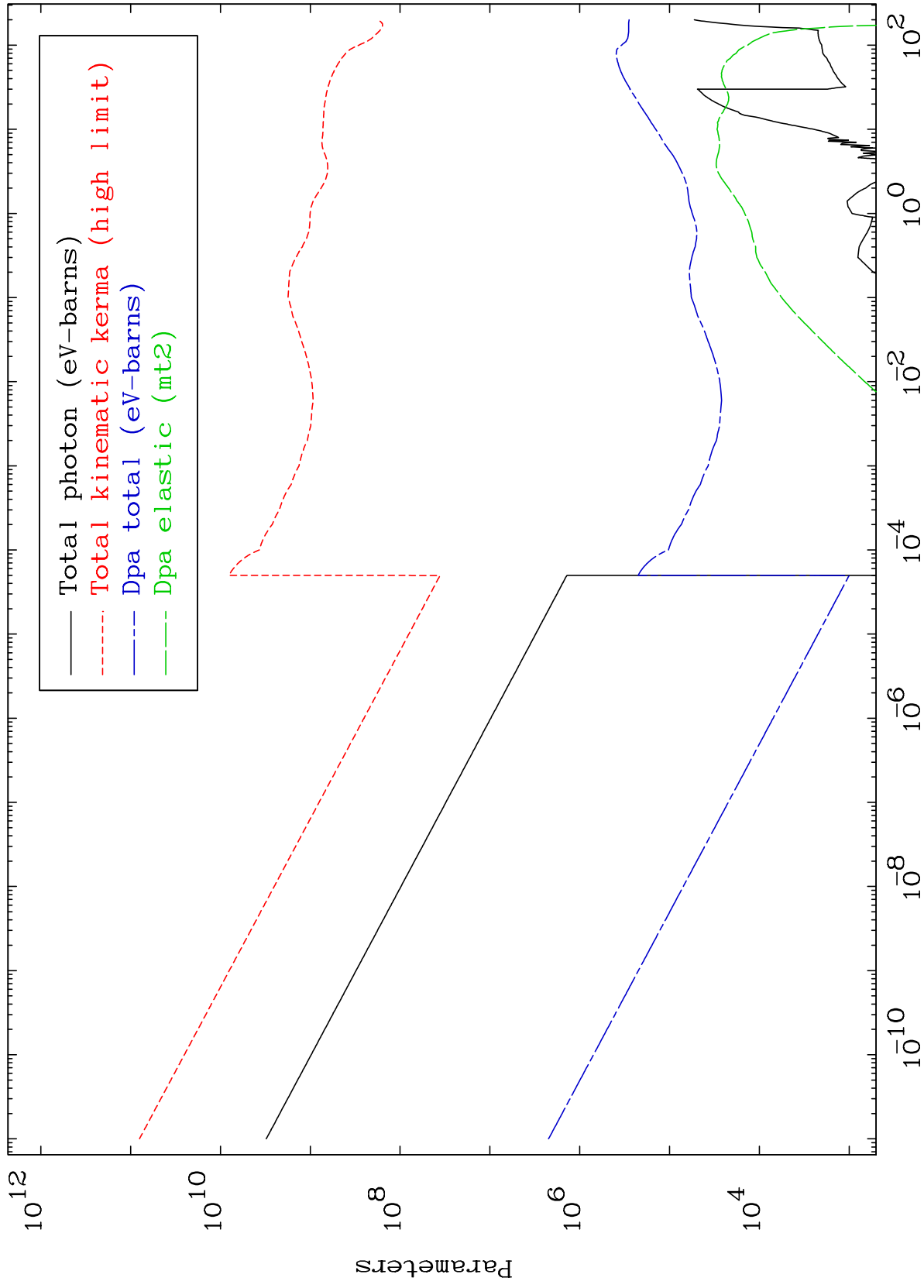
Incident Energy (MeV)

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MAT 773

Energy Release
Parameters

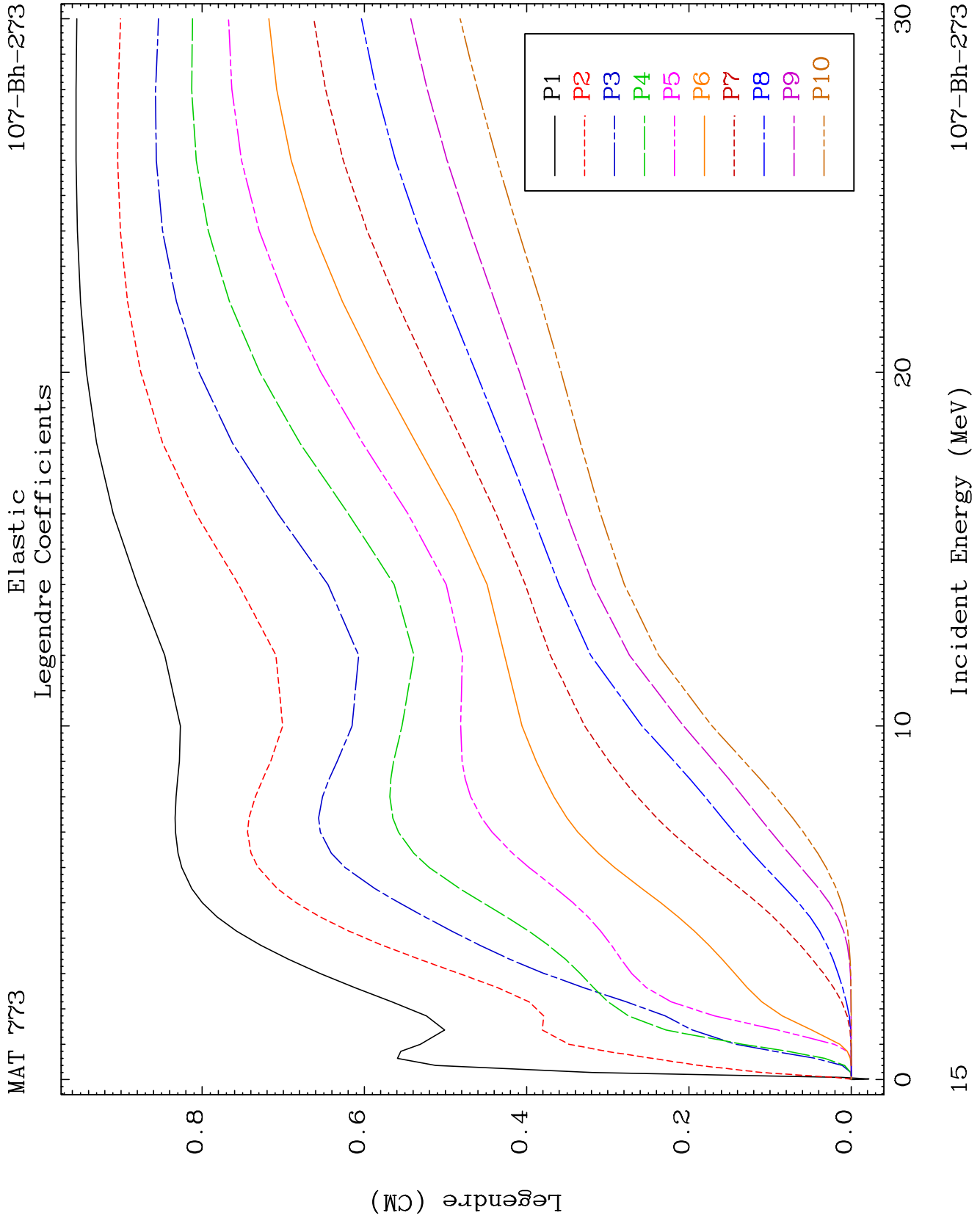
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Incident Energy (MeV)

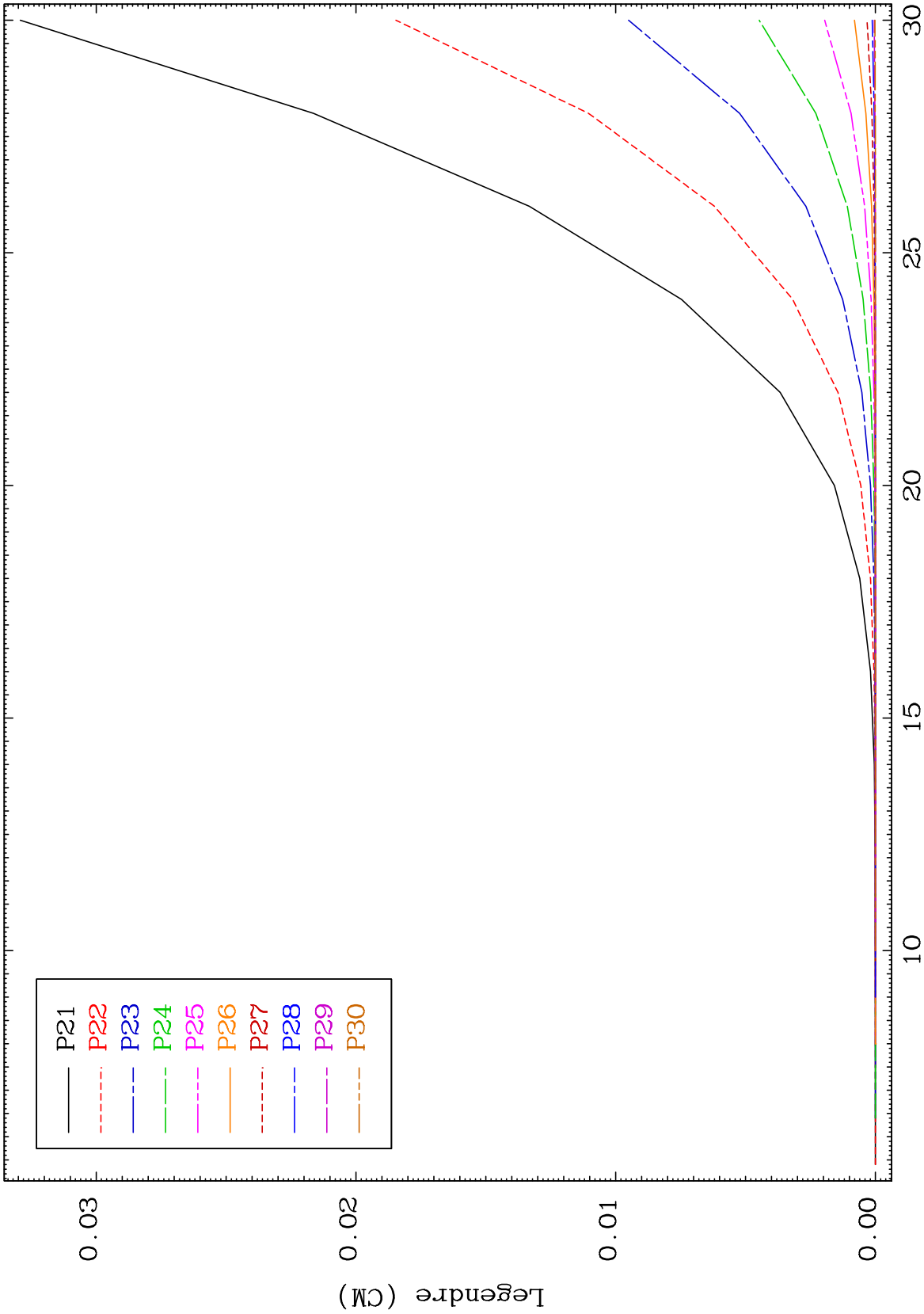
107-Bh-273



MAT 773

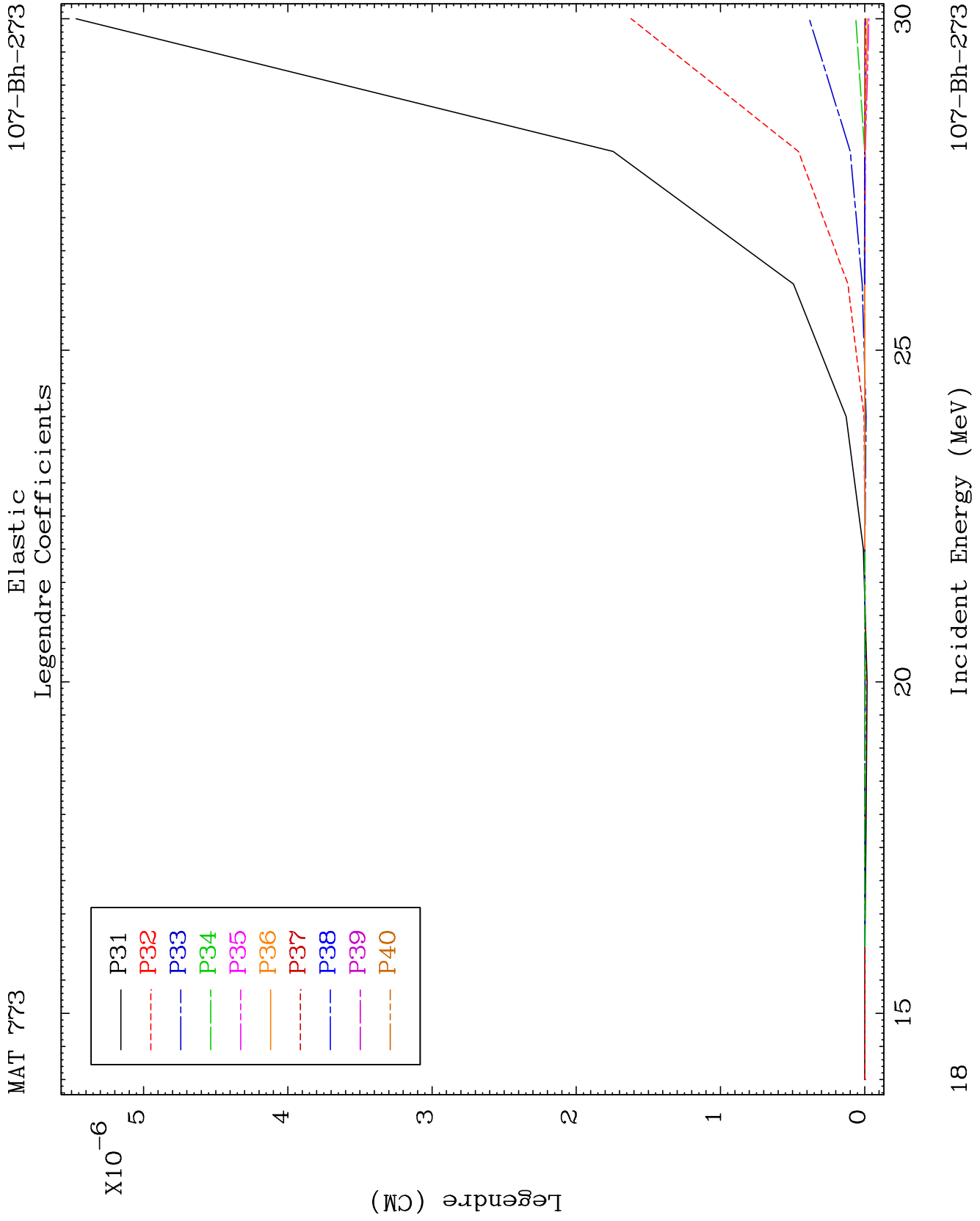
Elastic Legendre Coefficients

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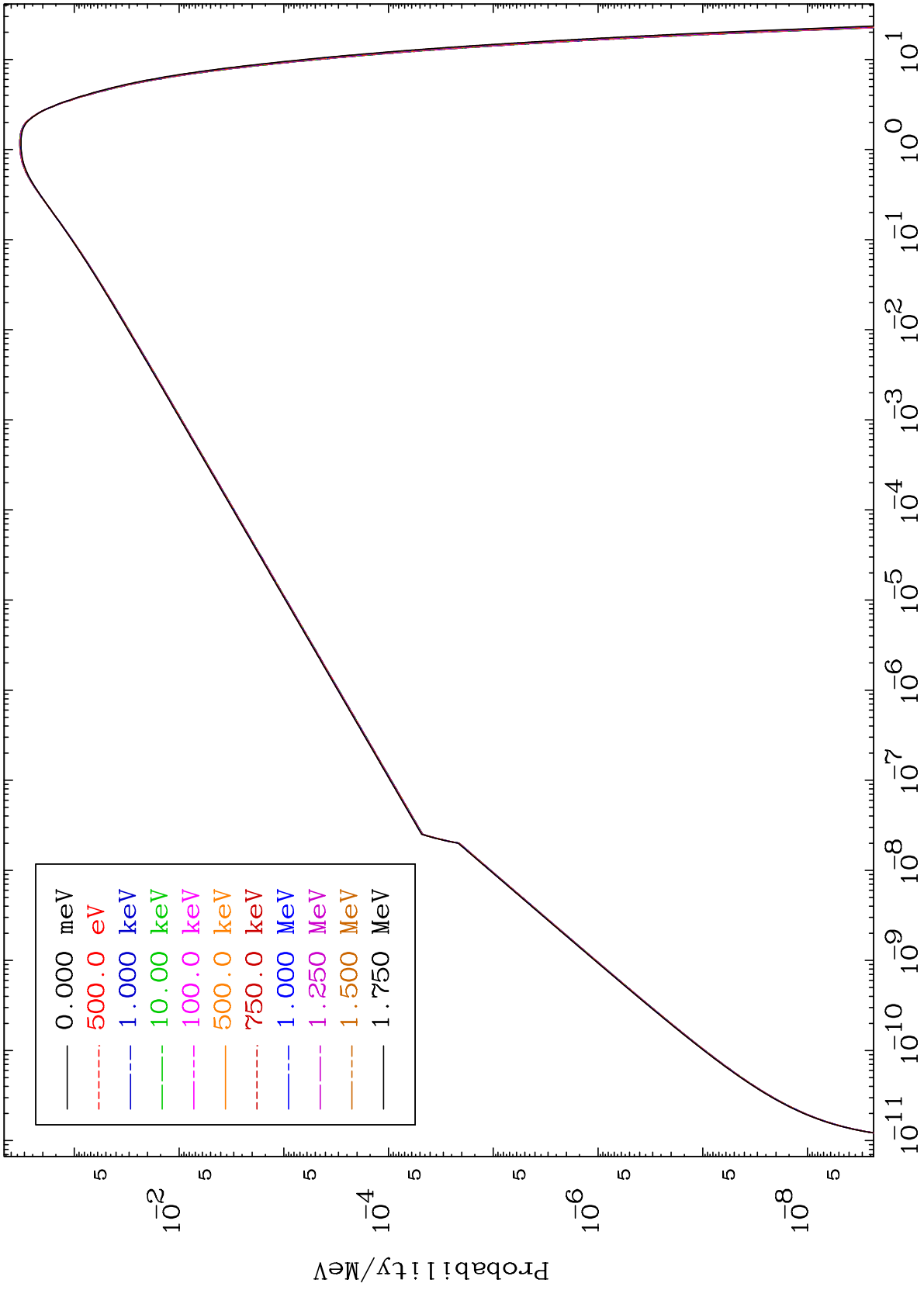
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MAT 773

Fission Energy Distribution

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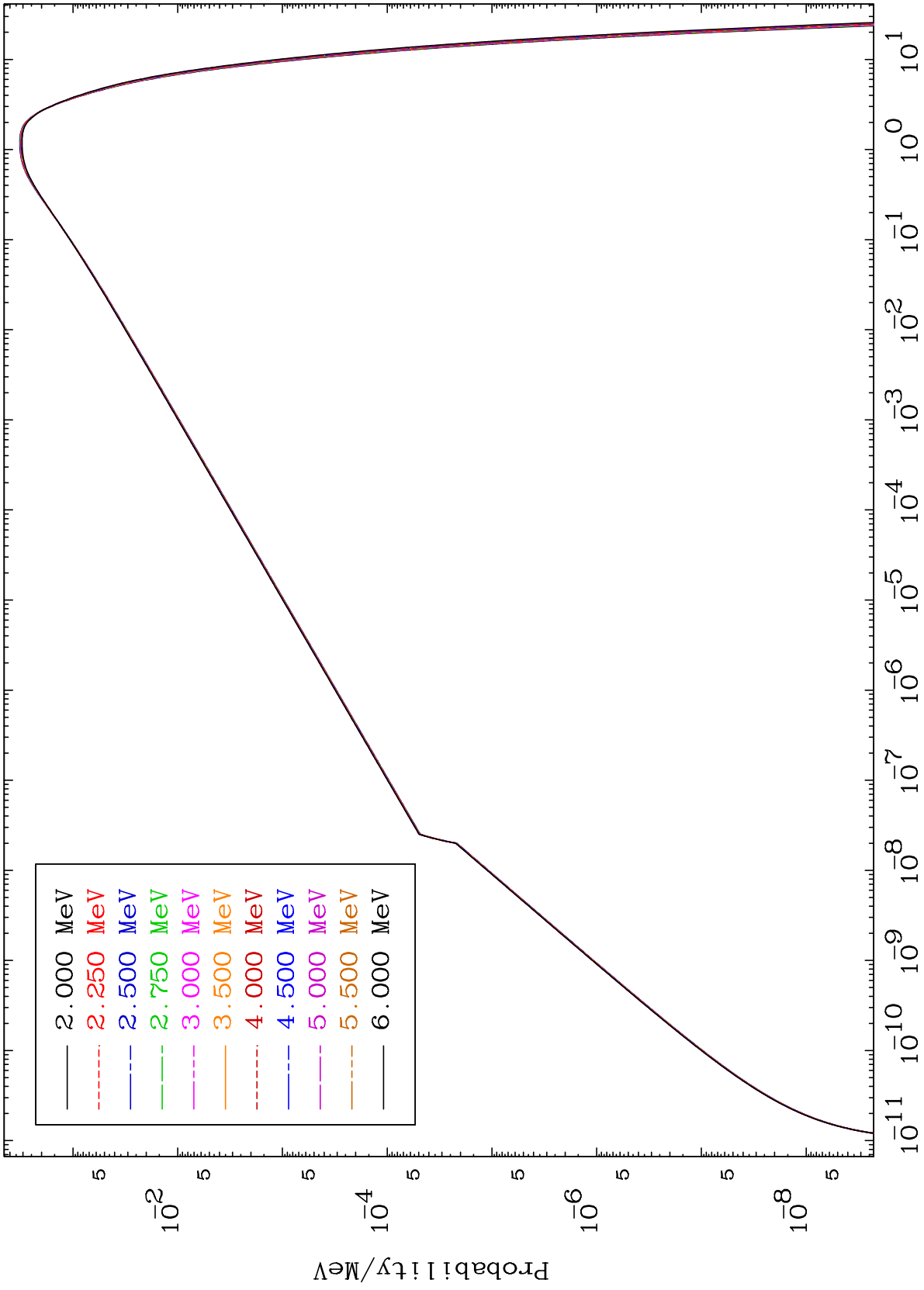


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MAT 773

Fission Energy Distribution

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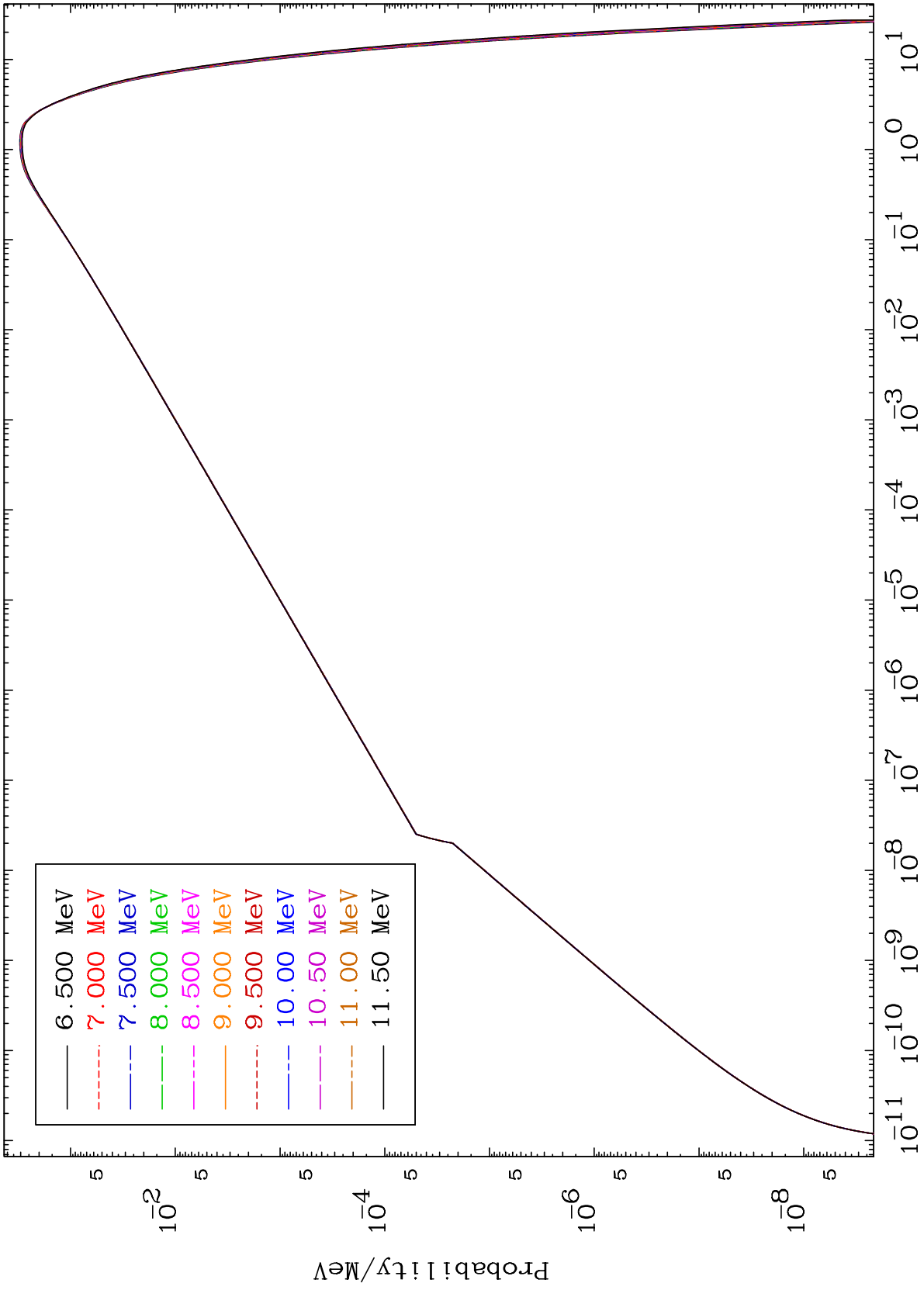


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MAT 773

Fission Energy Distribution

107-Bh-273

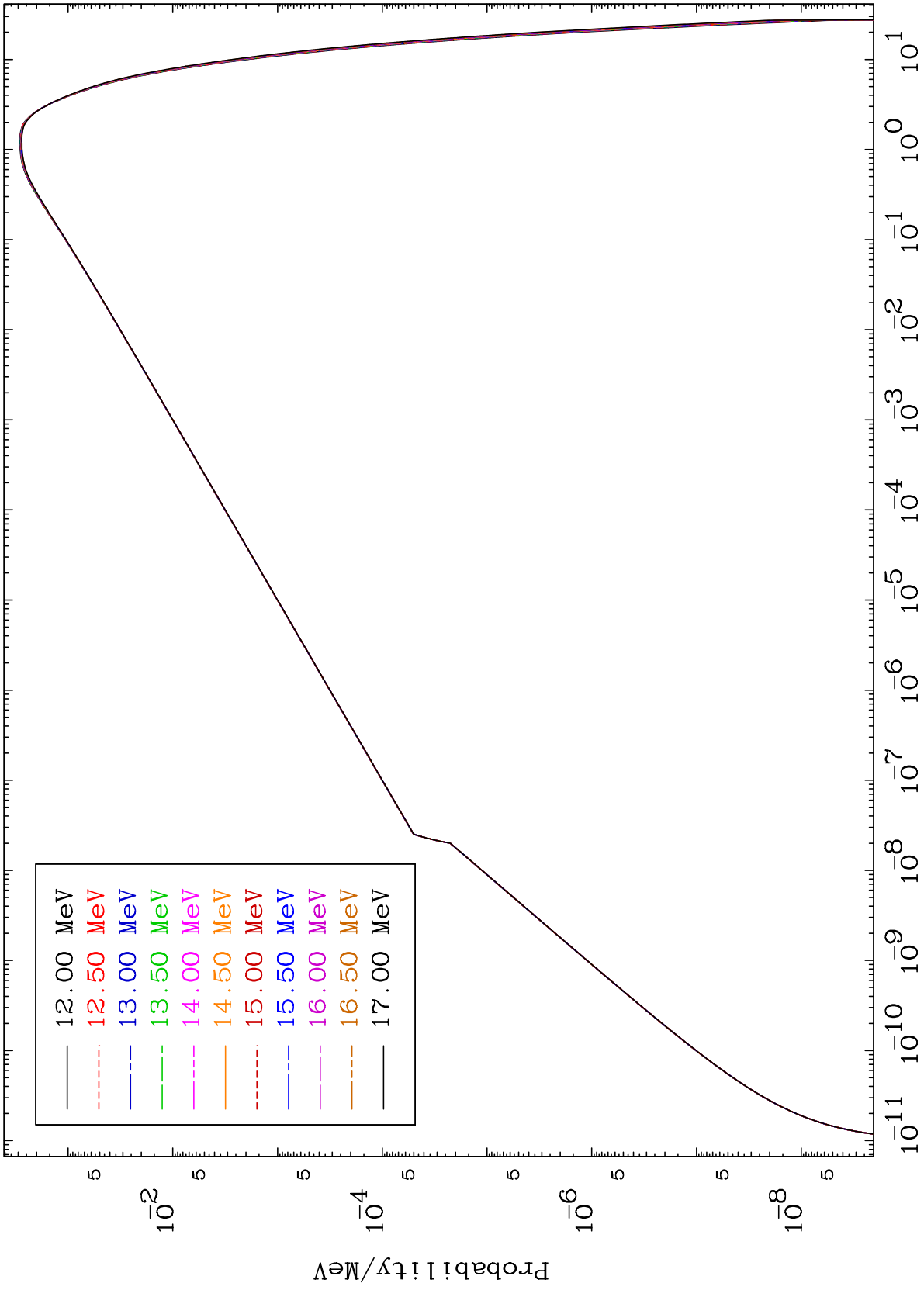


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MAT 773

Fission Energy Distribution

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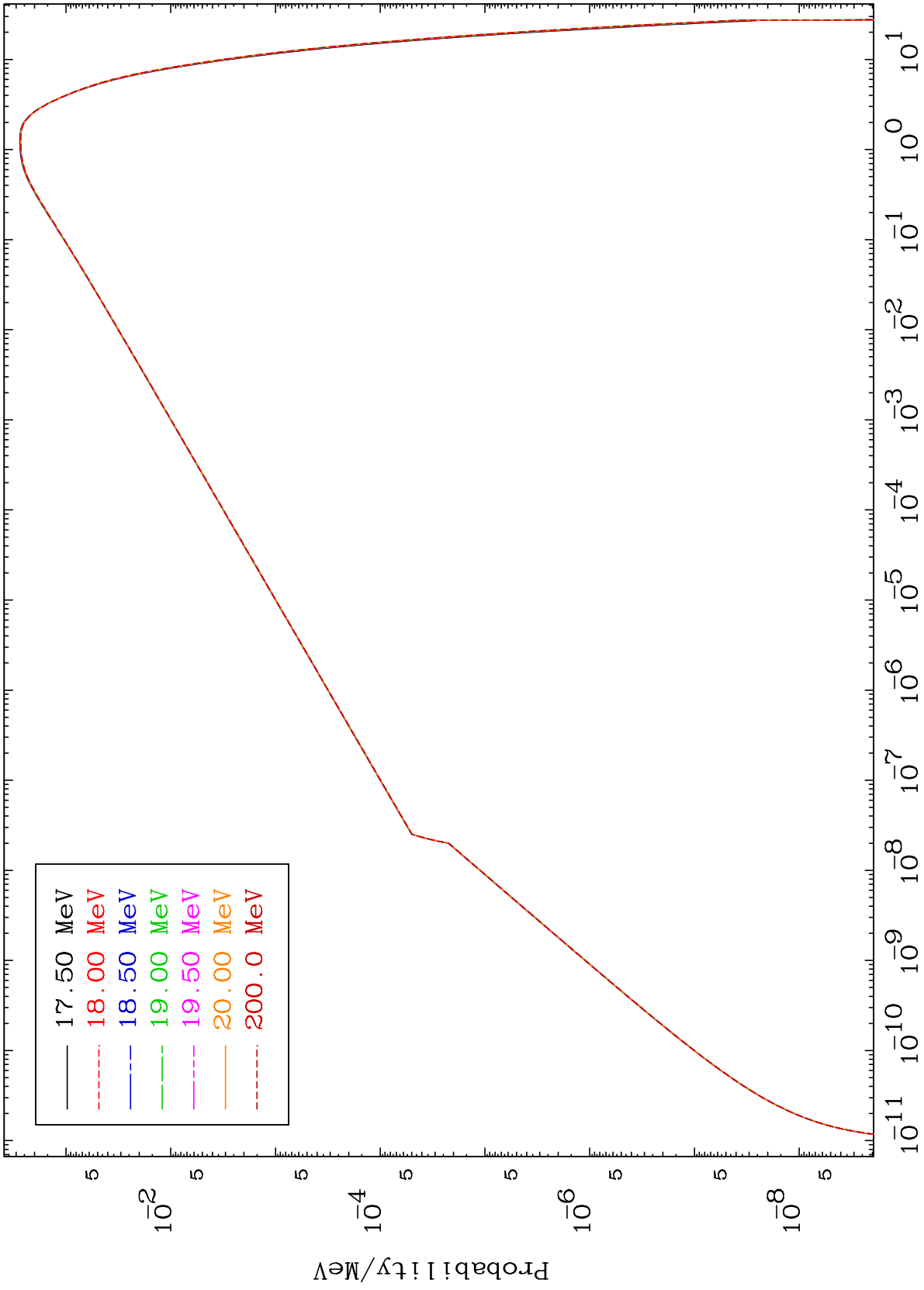


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MAT 773

Fission Energy Distribution

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—	17.50	MeV
- - -	18.00	MeV
- · - ·	18.50	MeV
- - -	19.00	MeV
- · - ·	19.50	MeV
- - -	20.00	MeV
- - -	200.0	MeV

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