

Program EVALPLOT
(Version 2018-1)

by

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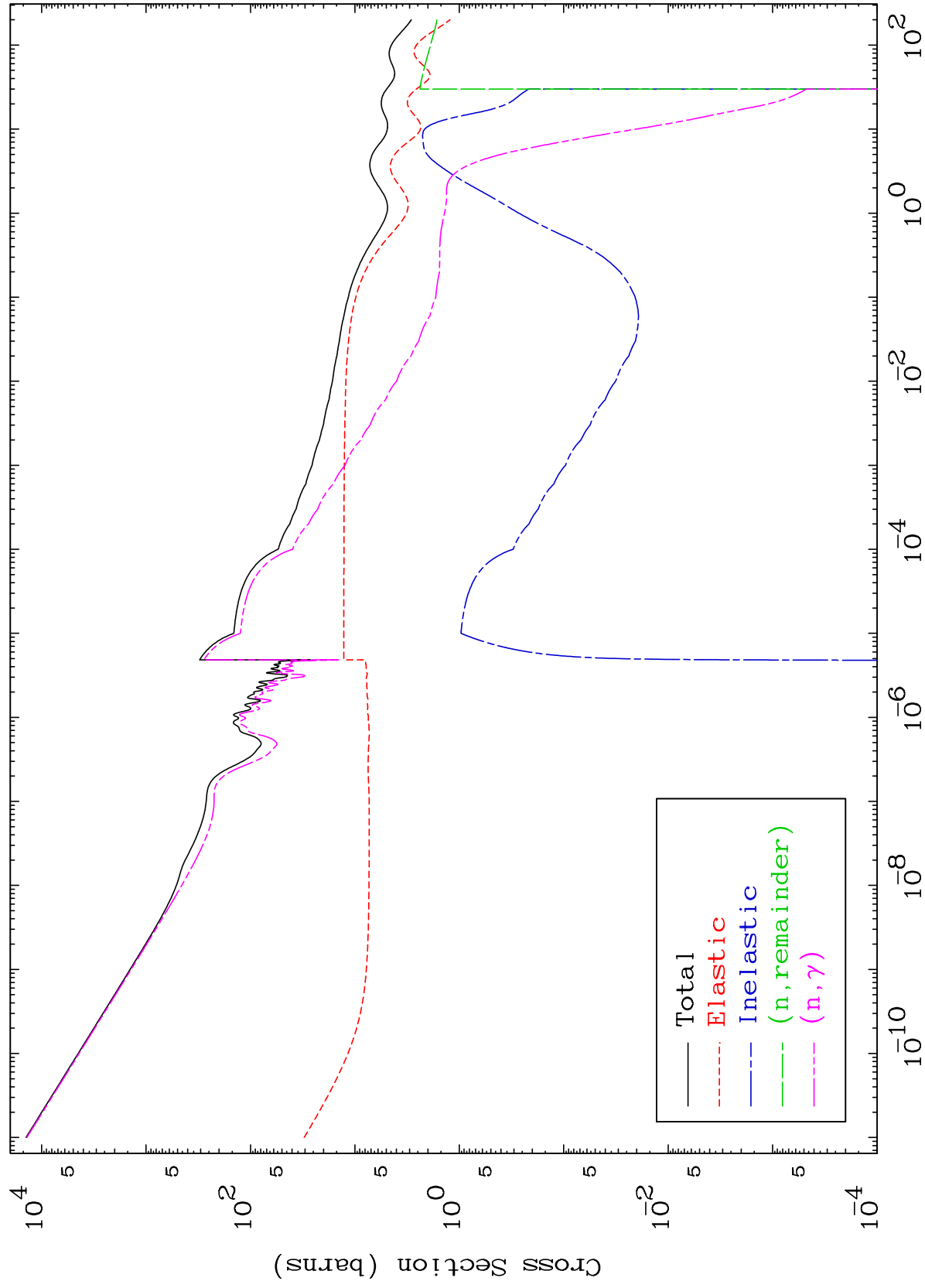
Press Mouse Button to Start

MAT 8282

Major

293 Kelvin Cross Sections

83-Bi-194



1

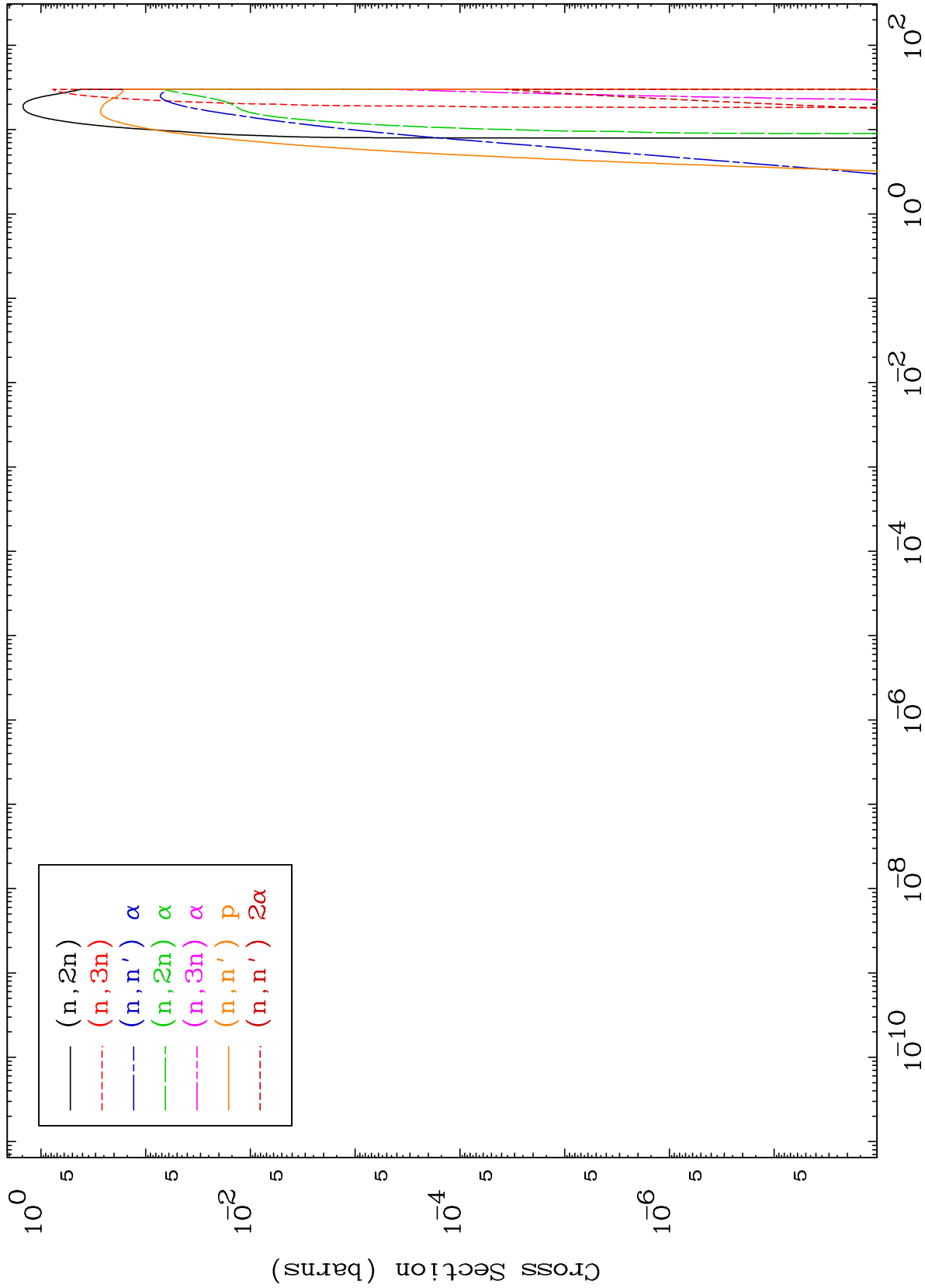
Incident Energy (MeV)

83-Bi-194

MAT 8282

Neutron Production
293 Kelvin Cross Sections

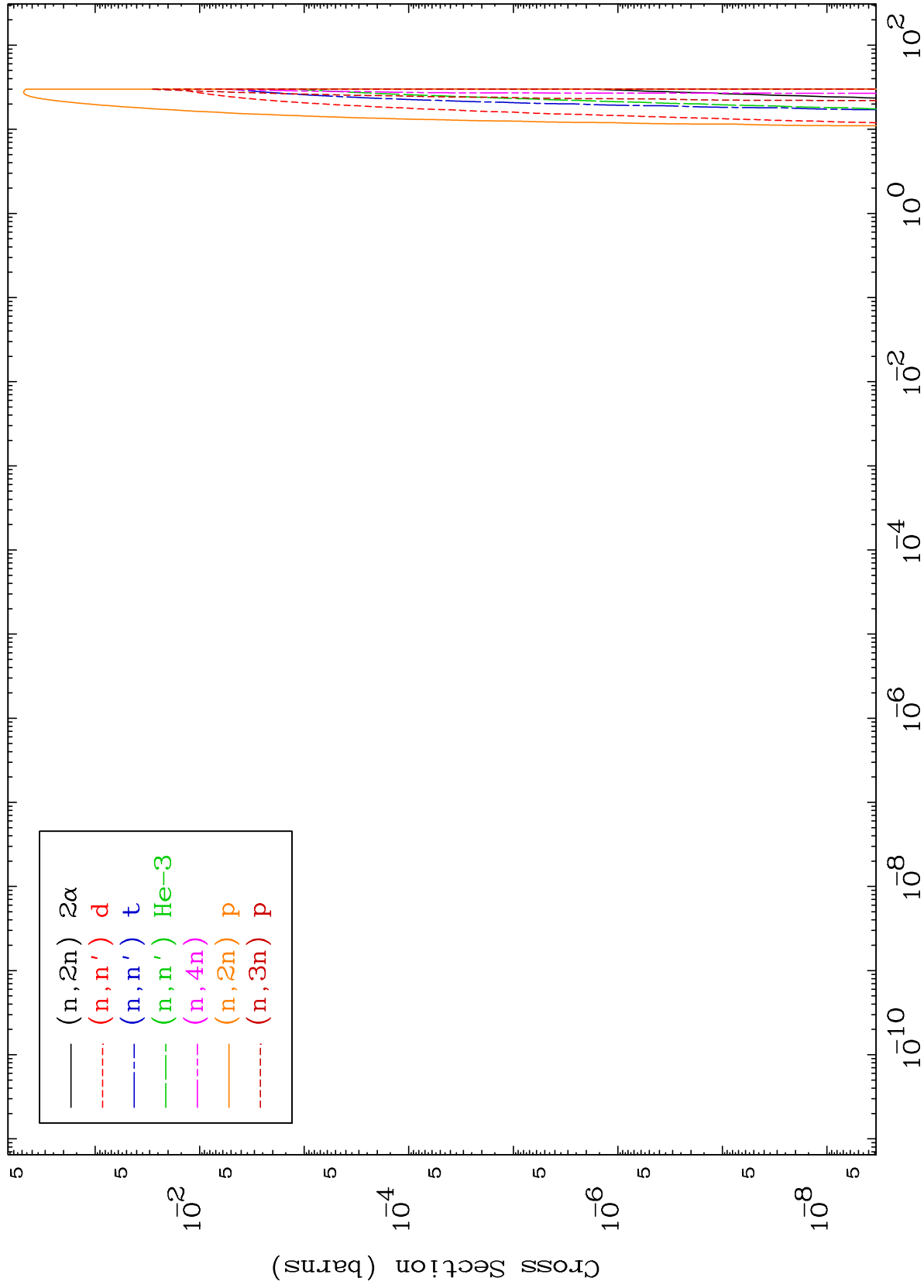
83-Bi-194



MAT 8282

Neutron Production
293 Kelvin Cross Sections

83-Bi-194



3

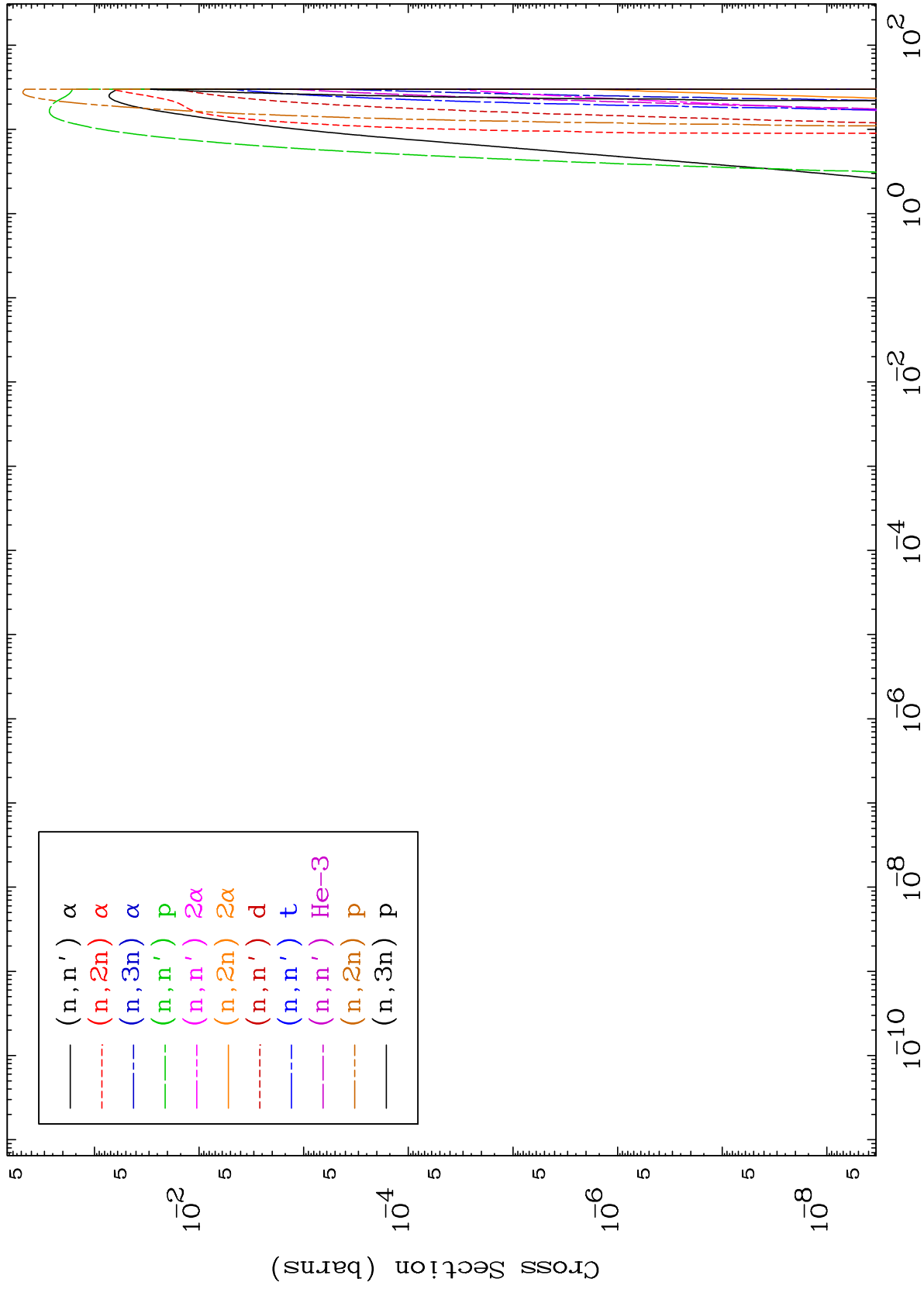
Incident Energy (MeV)

83-Bi-194

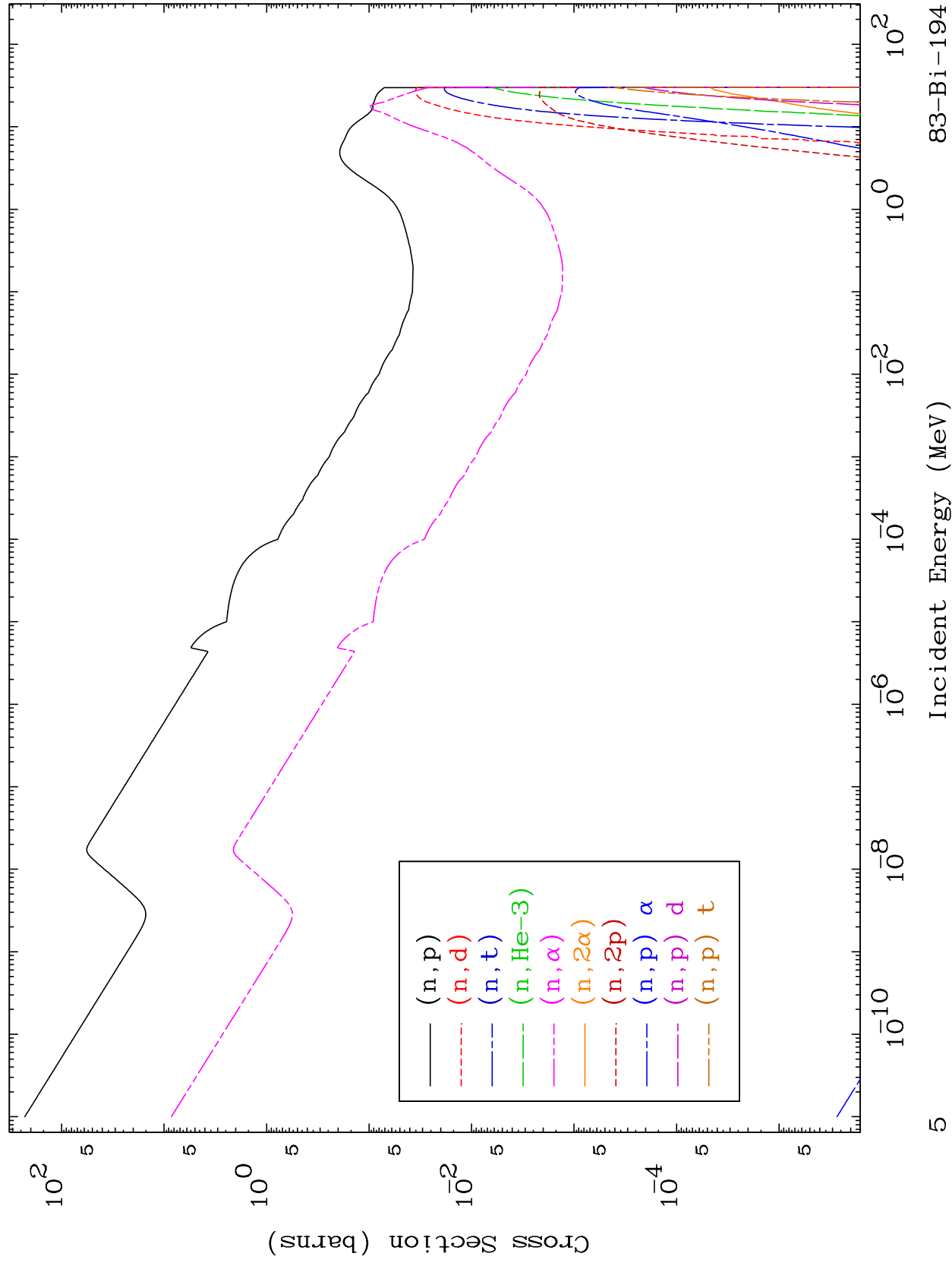
MAT 8282

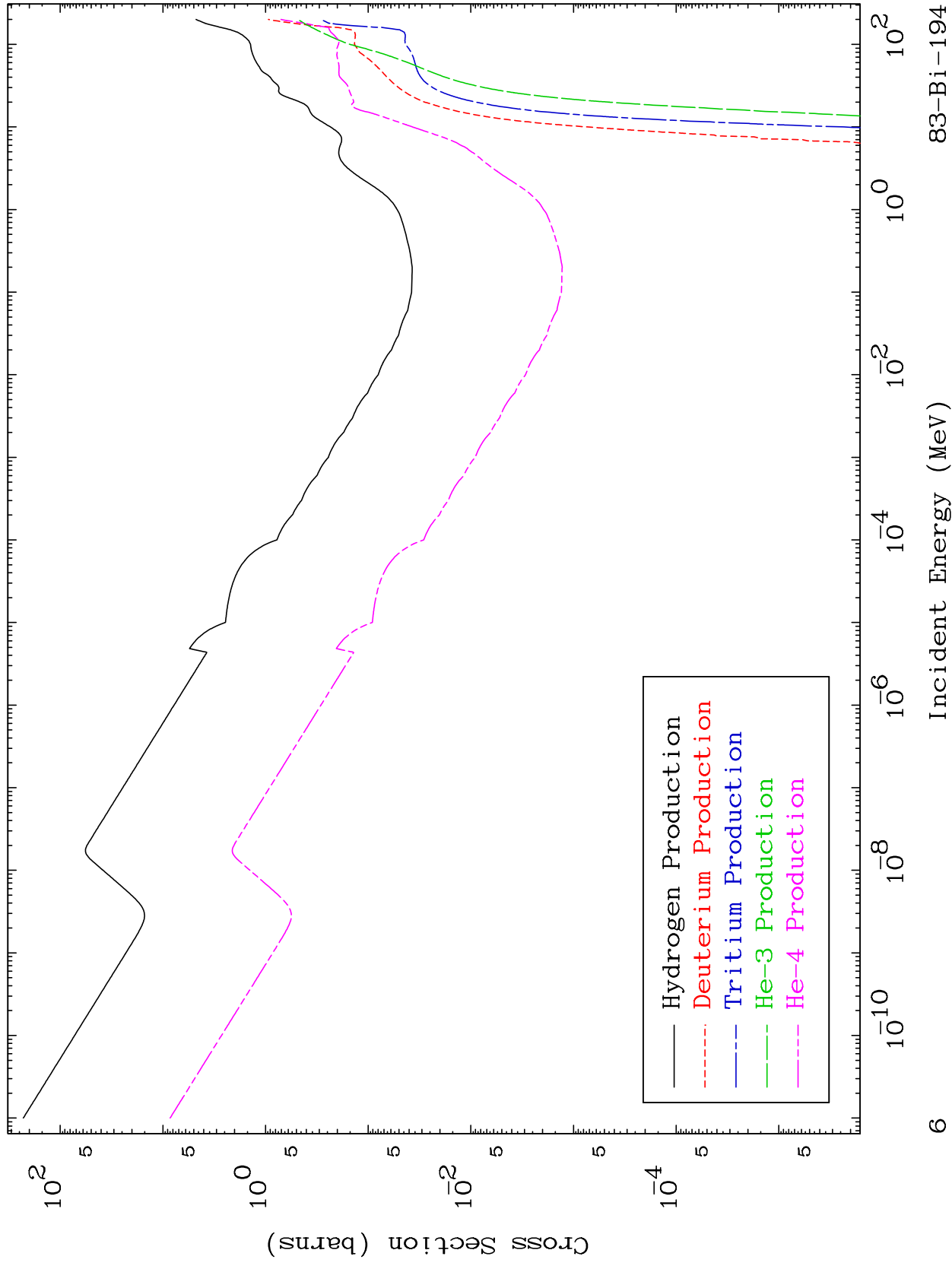
Charged Particle
293 Kelvin Cross Sections

83-Bi-194



83-Bi-194

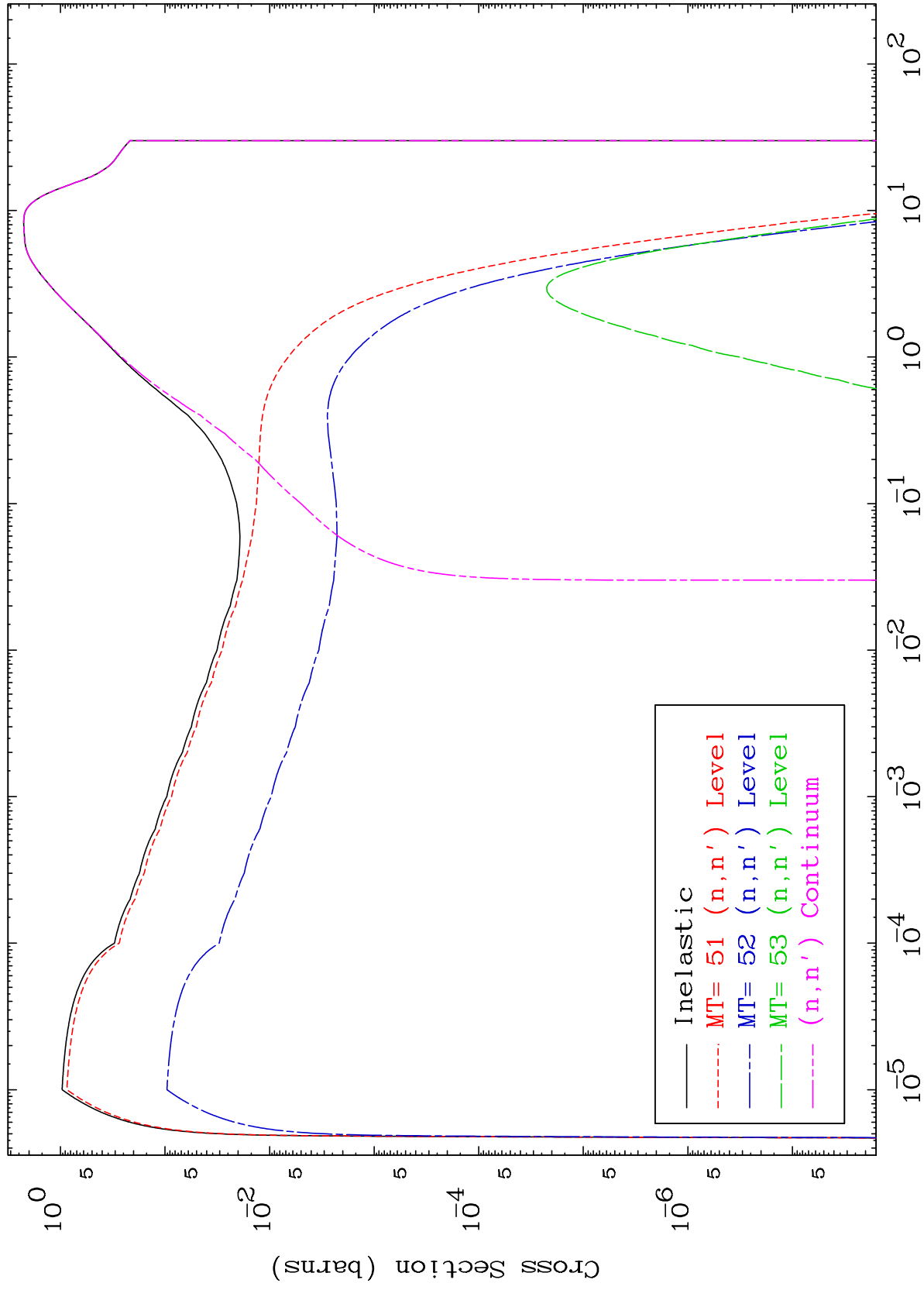




MAT 8282

(n,n') Level
293 Kelvin Cross Sections

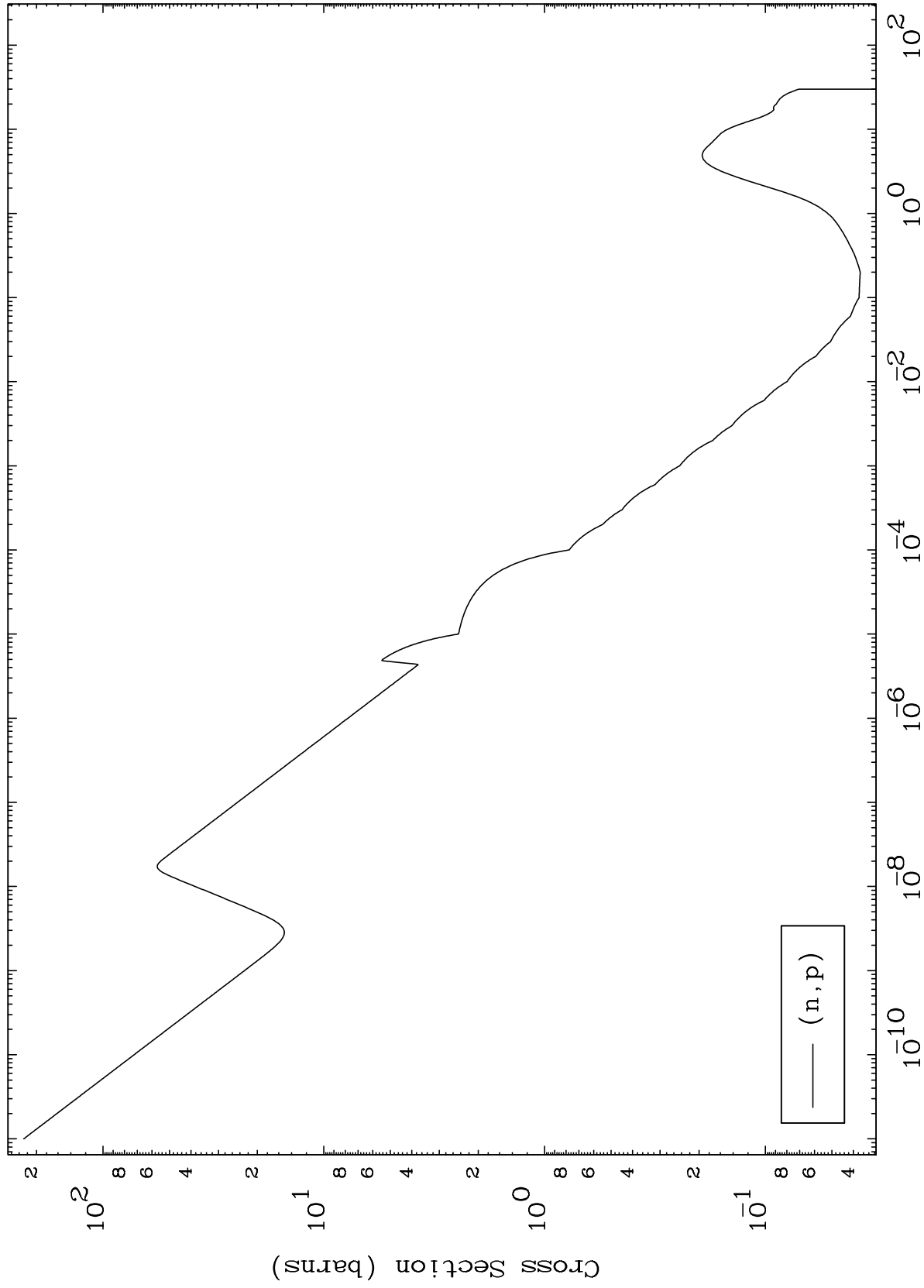
83-Bi-194



MAT 8282

(n,p) Levels
293 Kelvin Cross Sections

83-Bi-194



8

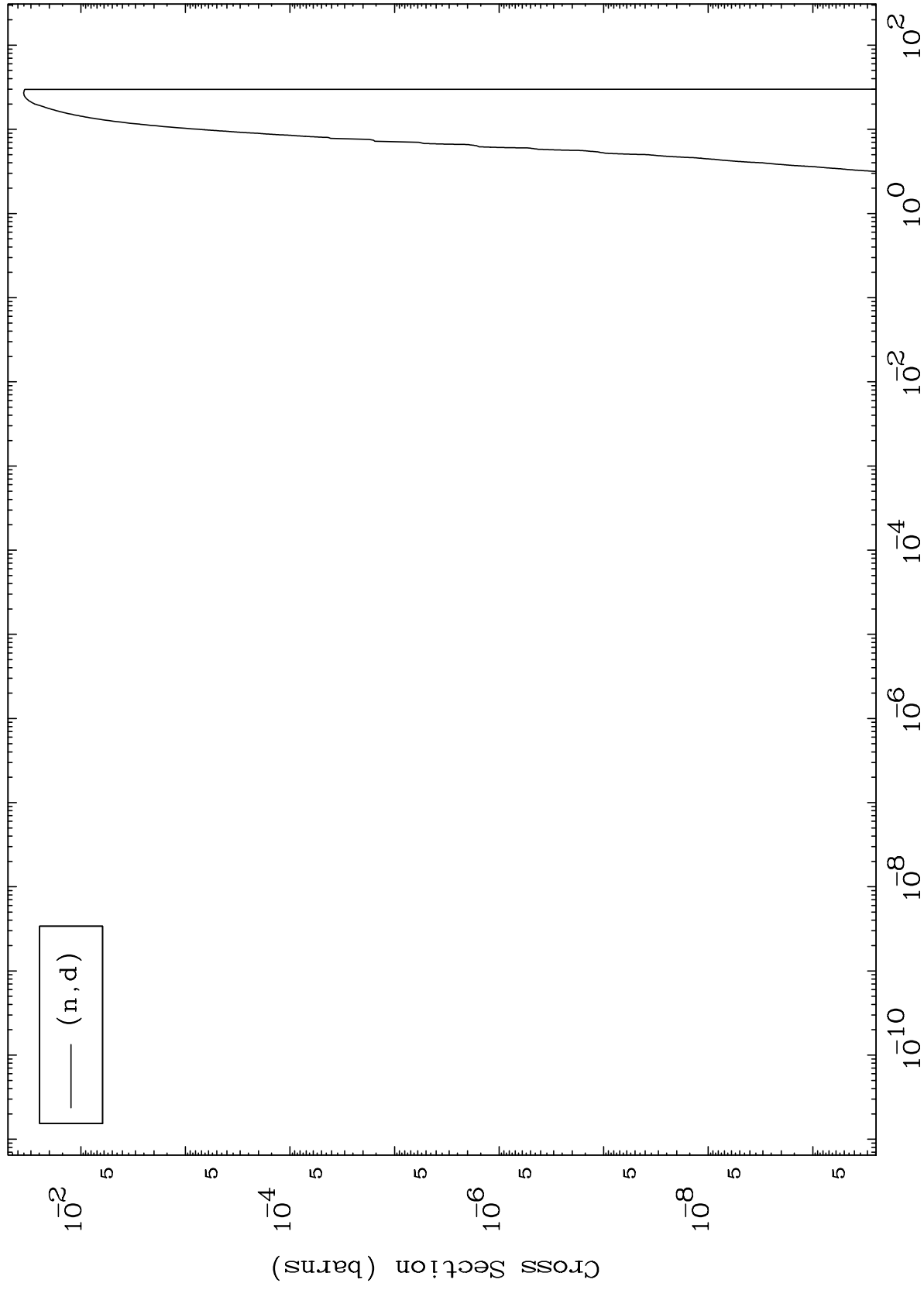
Incident Energy (MeV)

83-Bi-194

MAT 8282

(n,d) Levels
293 Kelvin Cross Sections

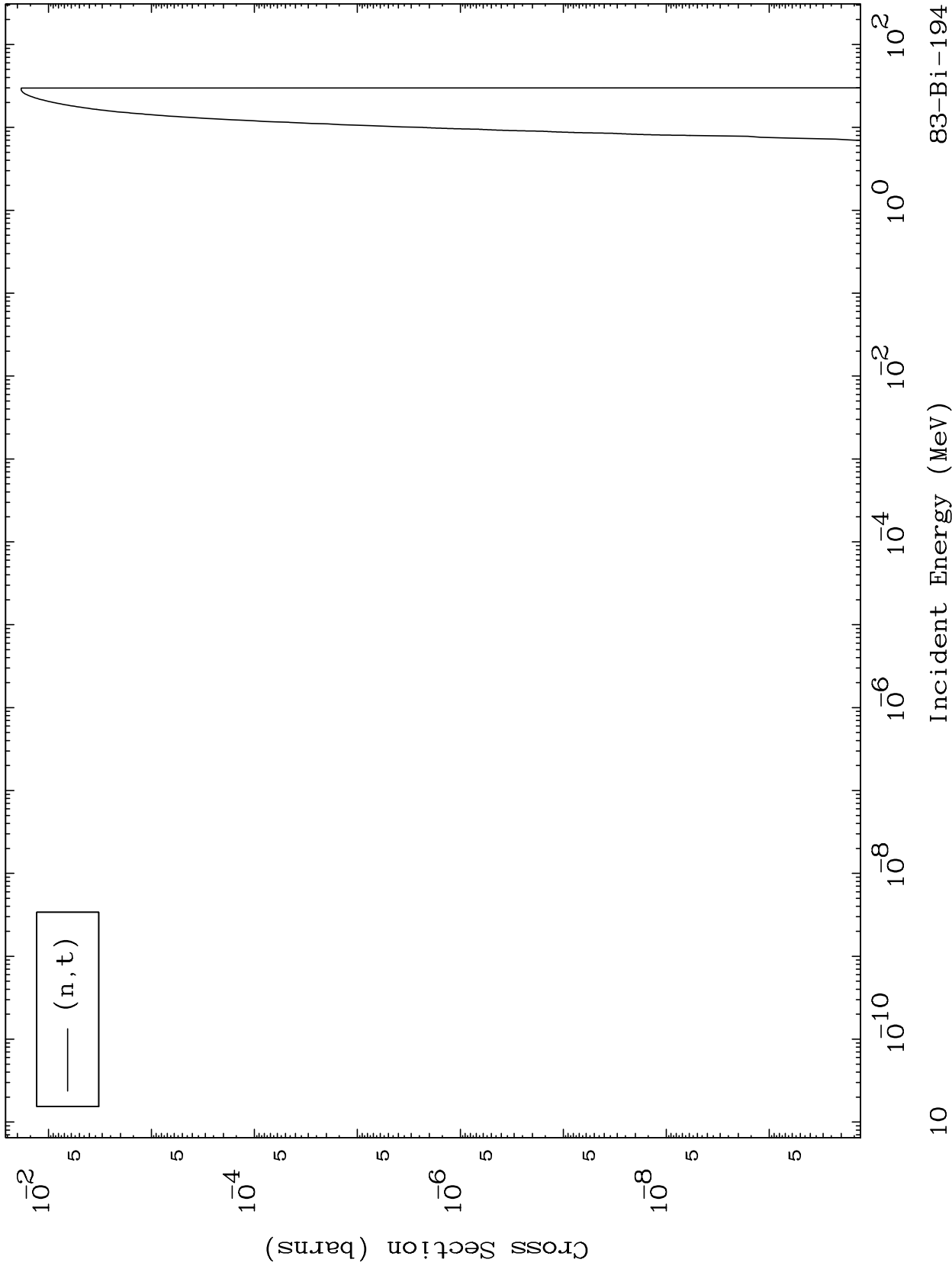
83-Bi-194



MAT 8282

(n,t) Levels
293 Kelvin Cross Sections

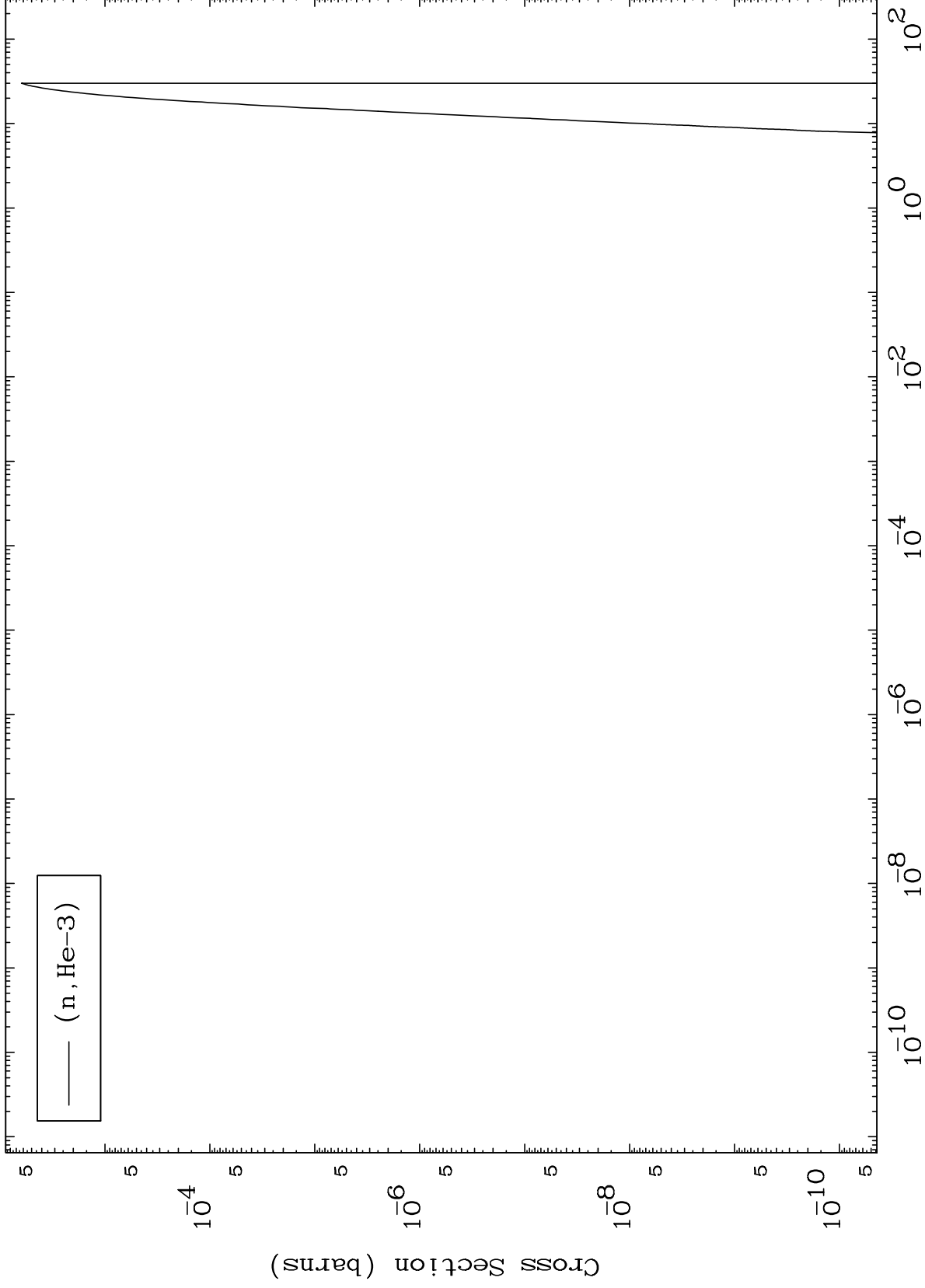
83-Bi-194



MAT 8282

(n,He3) Levels
293 Kelvin Cross Sections

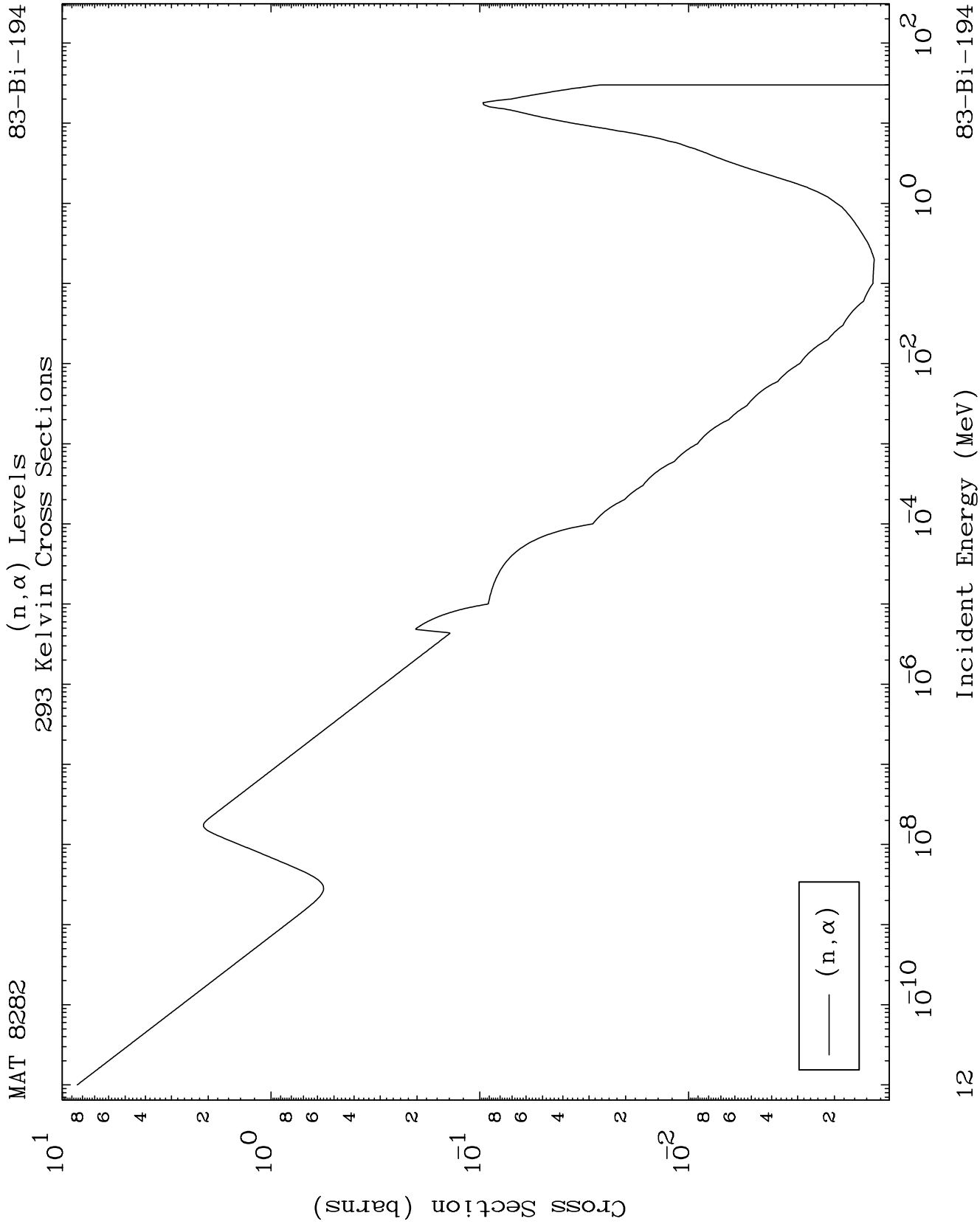
83-Bi-194

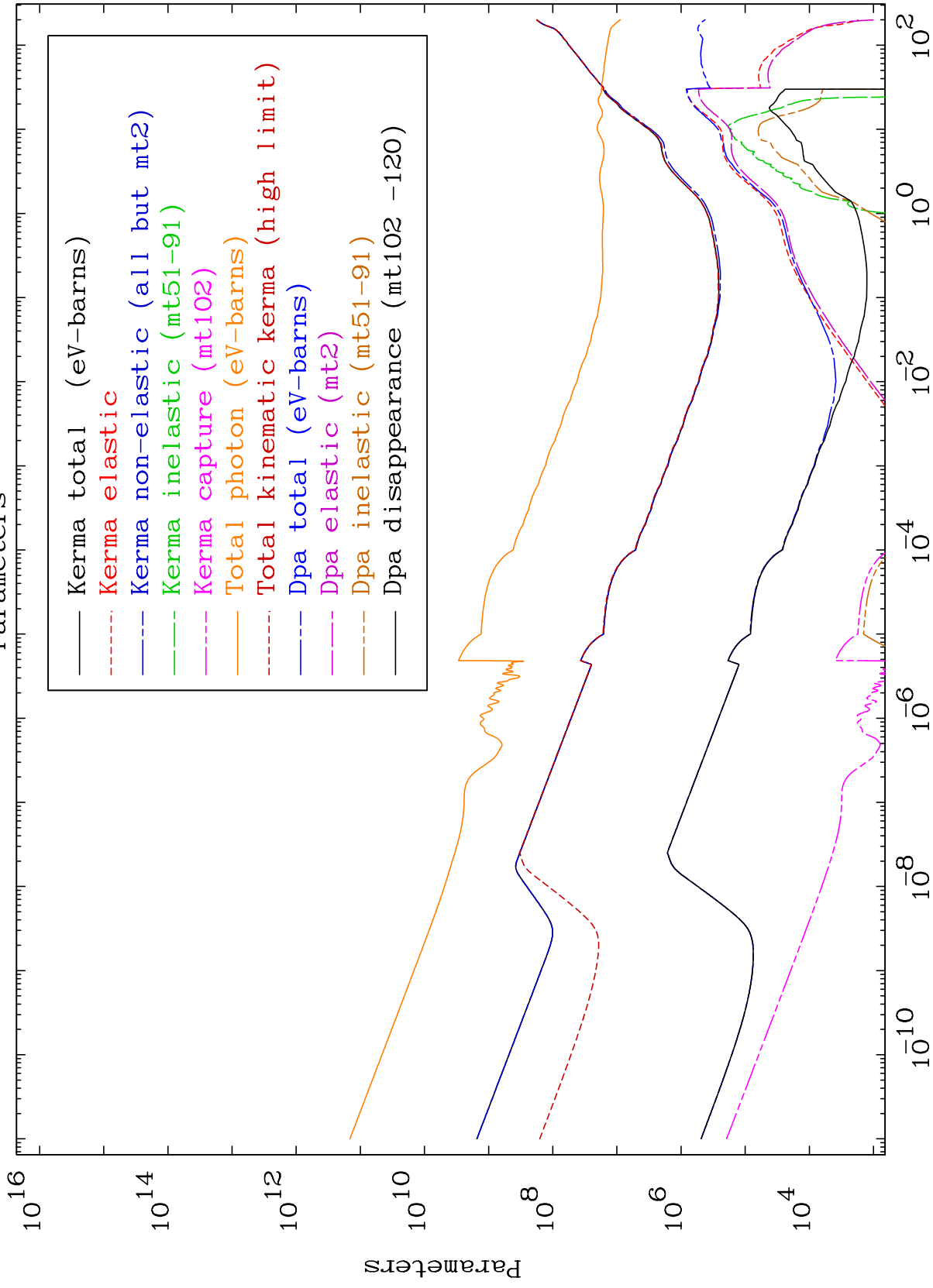


11

Incident Energy (MeV)

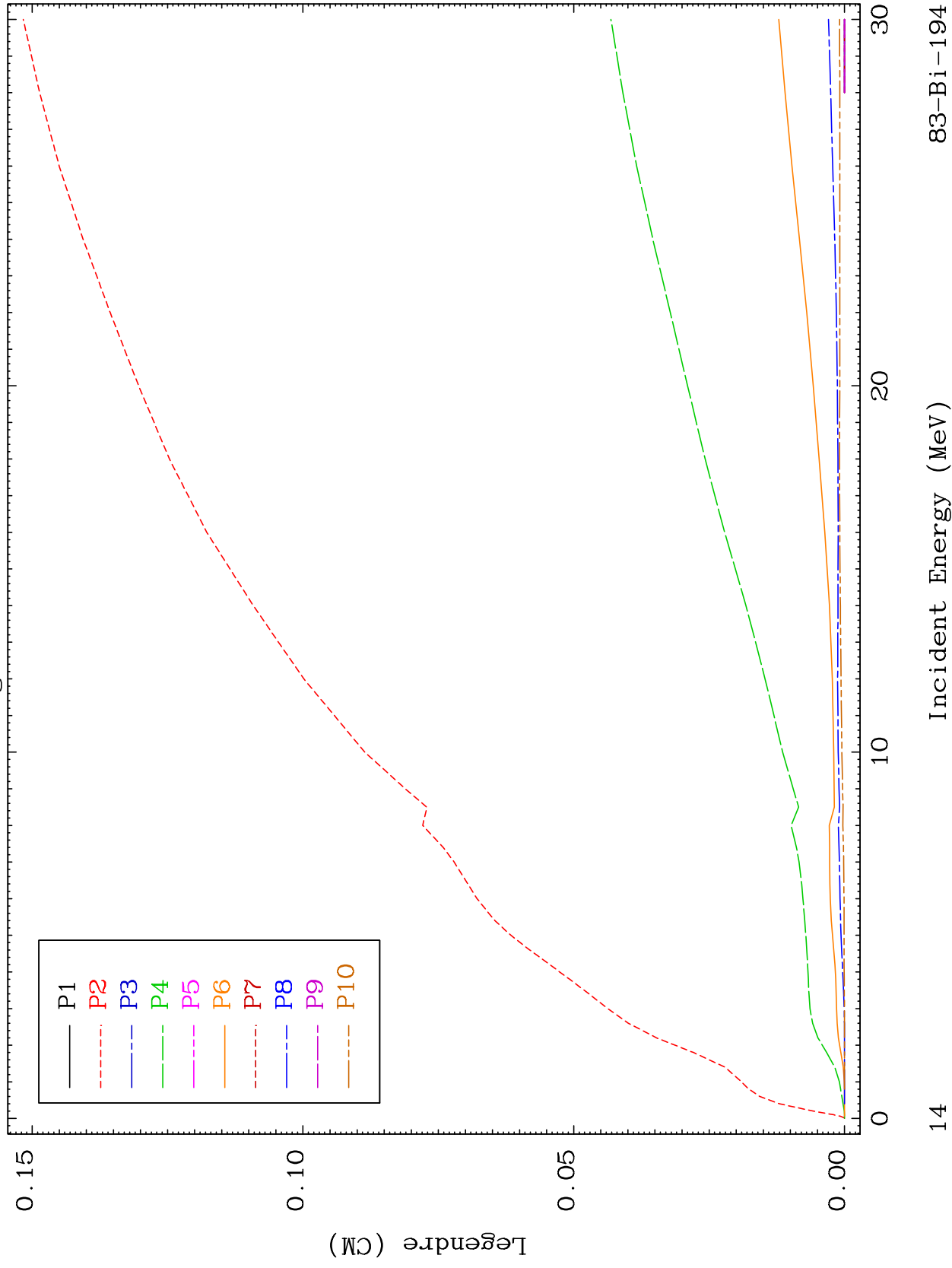
83-Bi-194





83-Bi-194

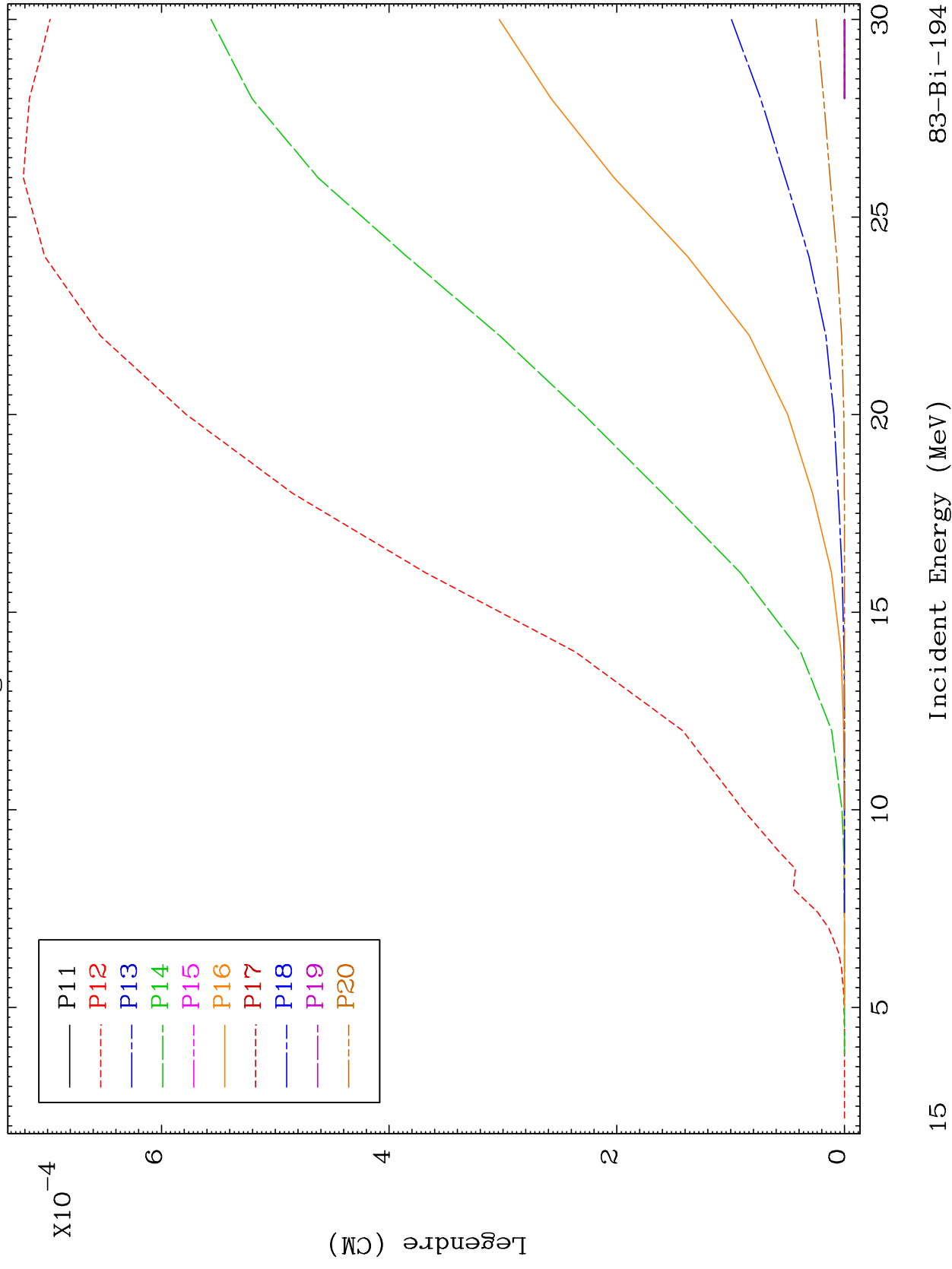
83-Bi-194



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Elastic Legendre Coefficients

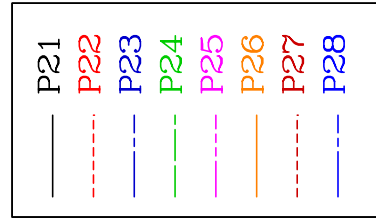
83-Bi-194



MAT 8282

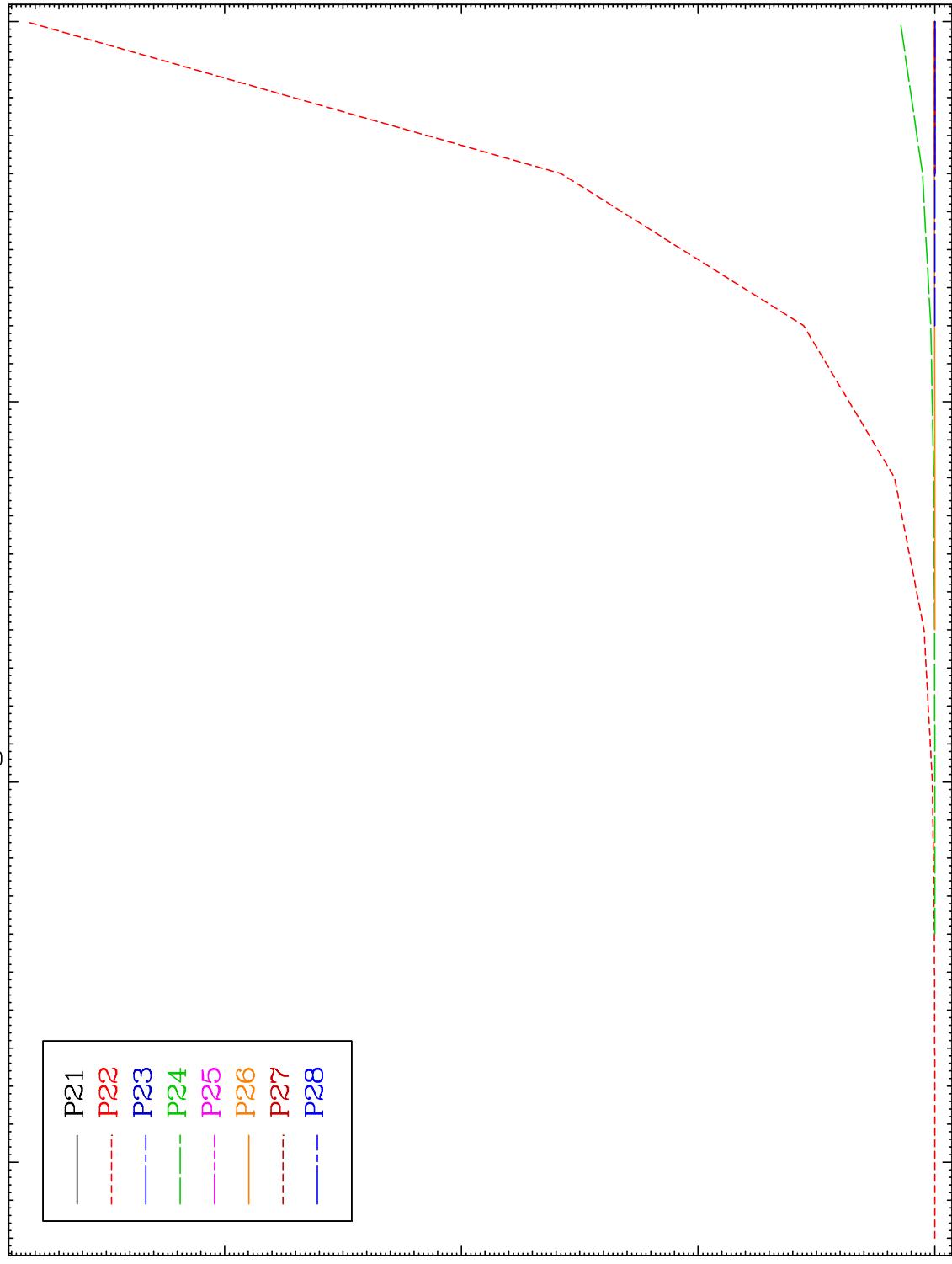
Elastic
Legendre Coefficients

83-Bi-194



$\times 10^{-6}$

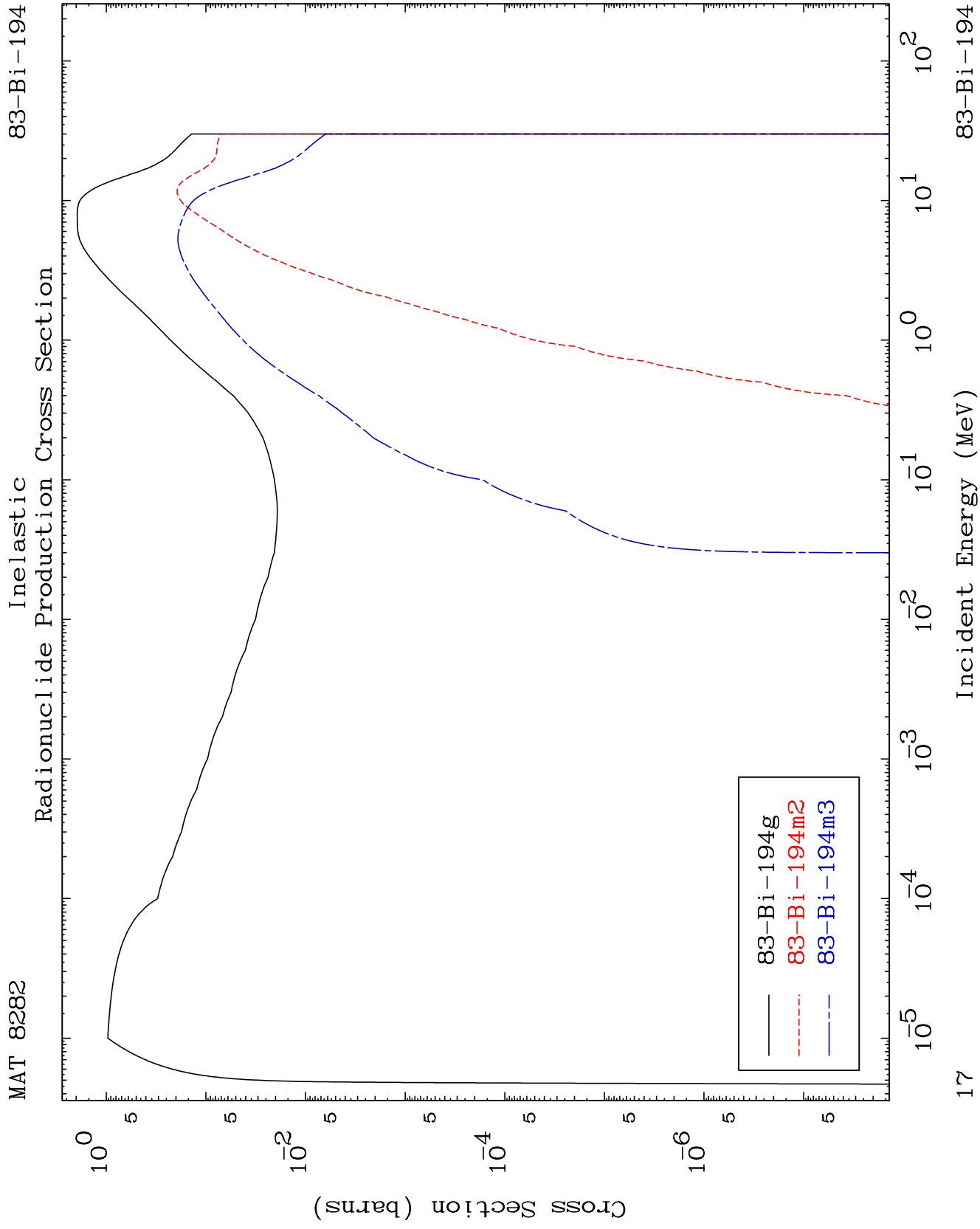
Legendre (CM)

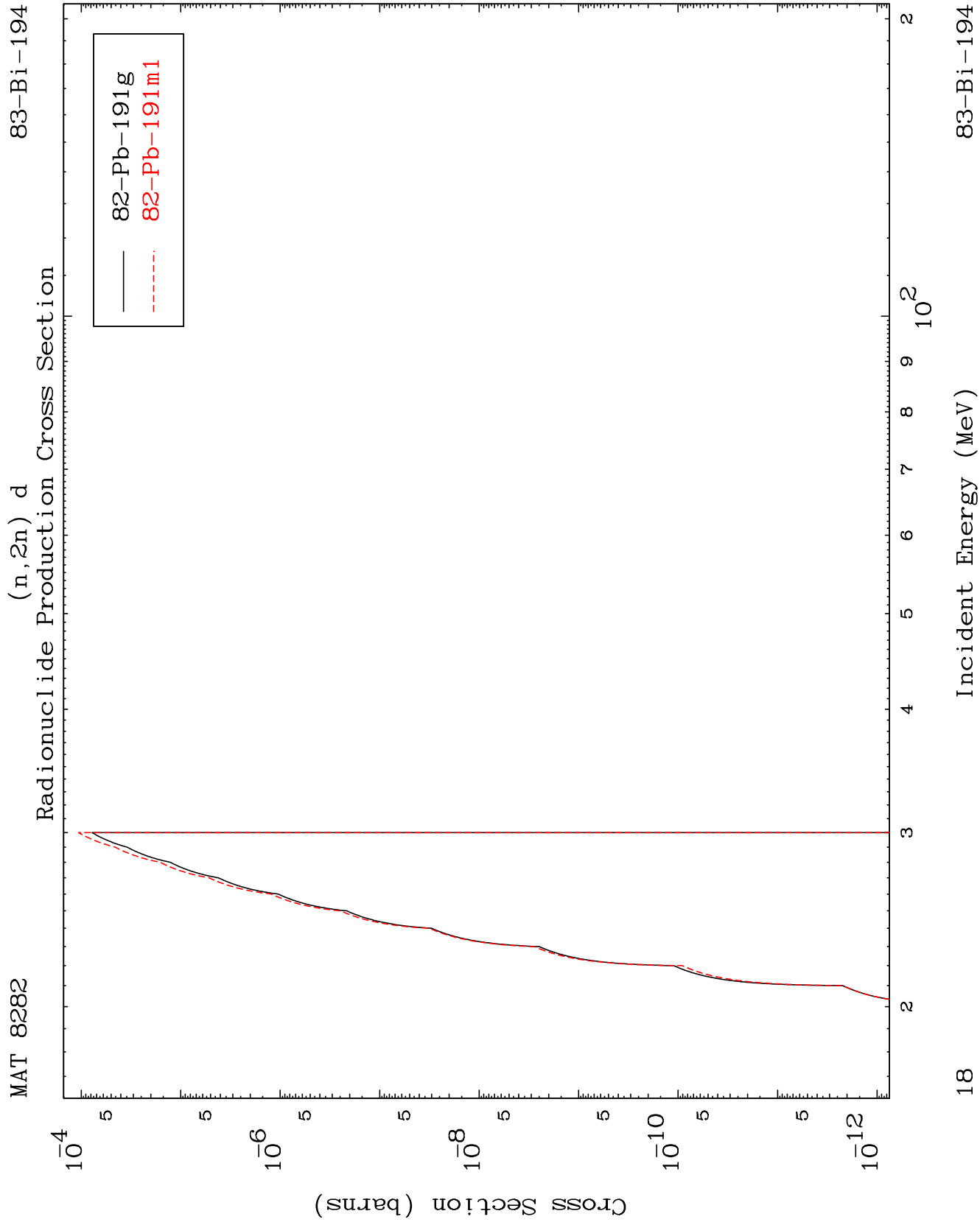


16

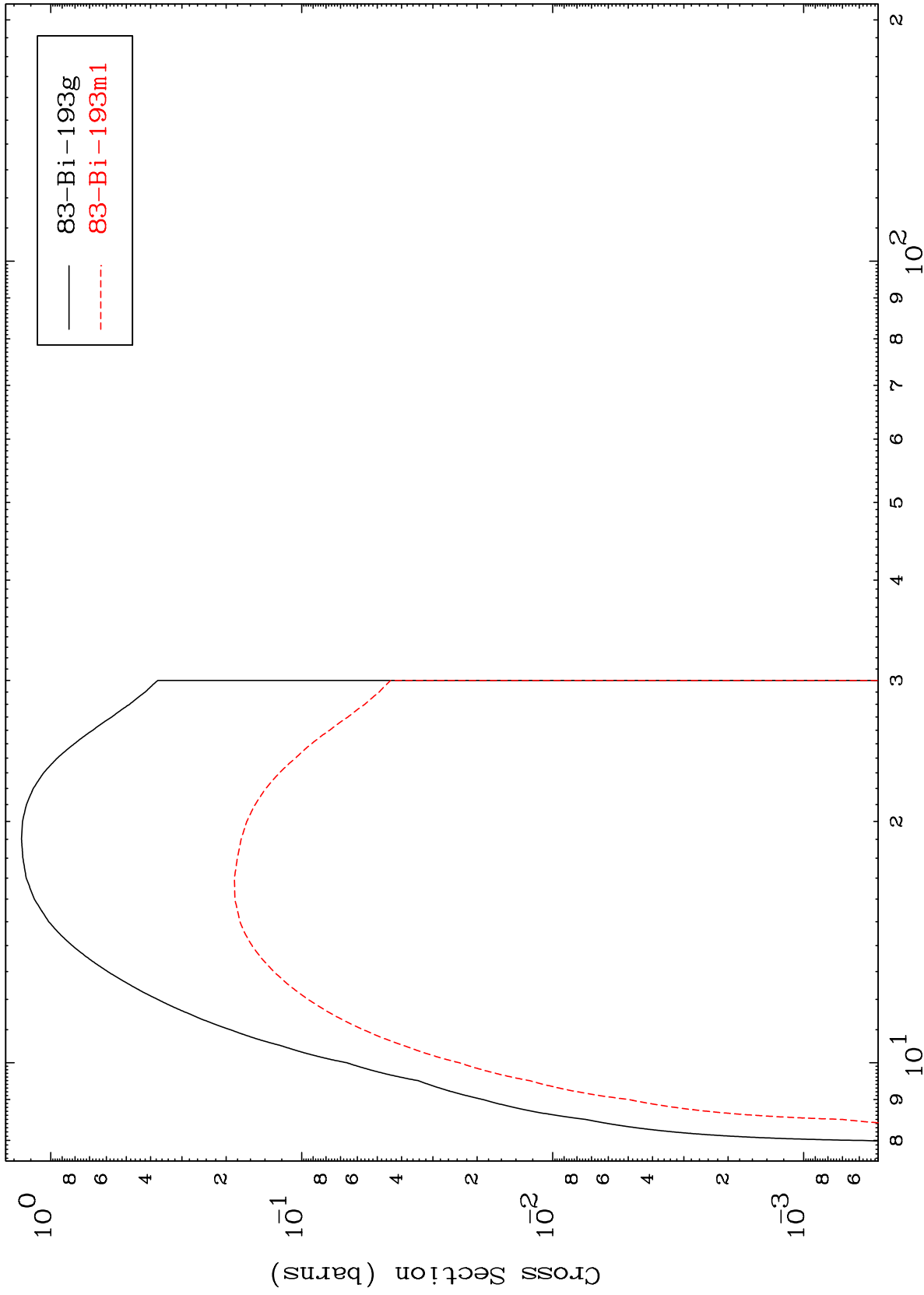
Incident Energy (MeV)

83-Bi-194





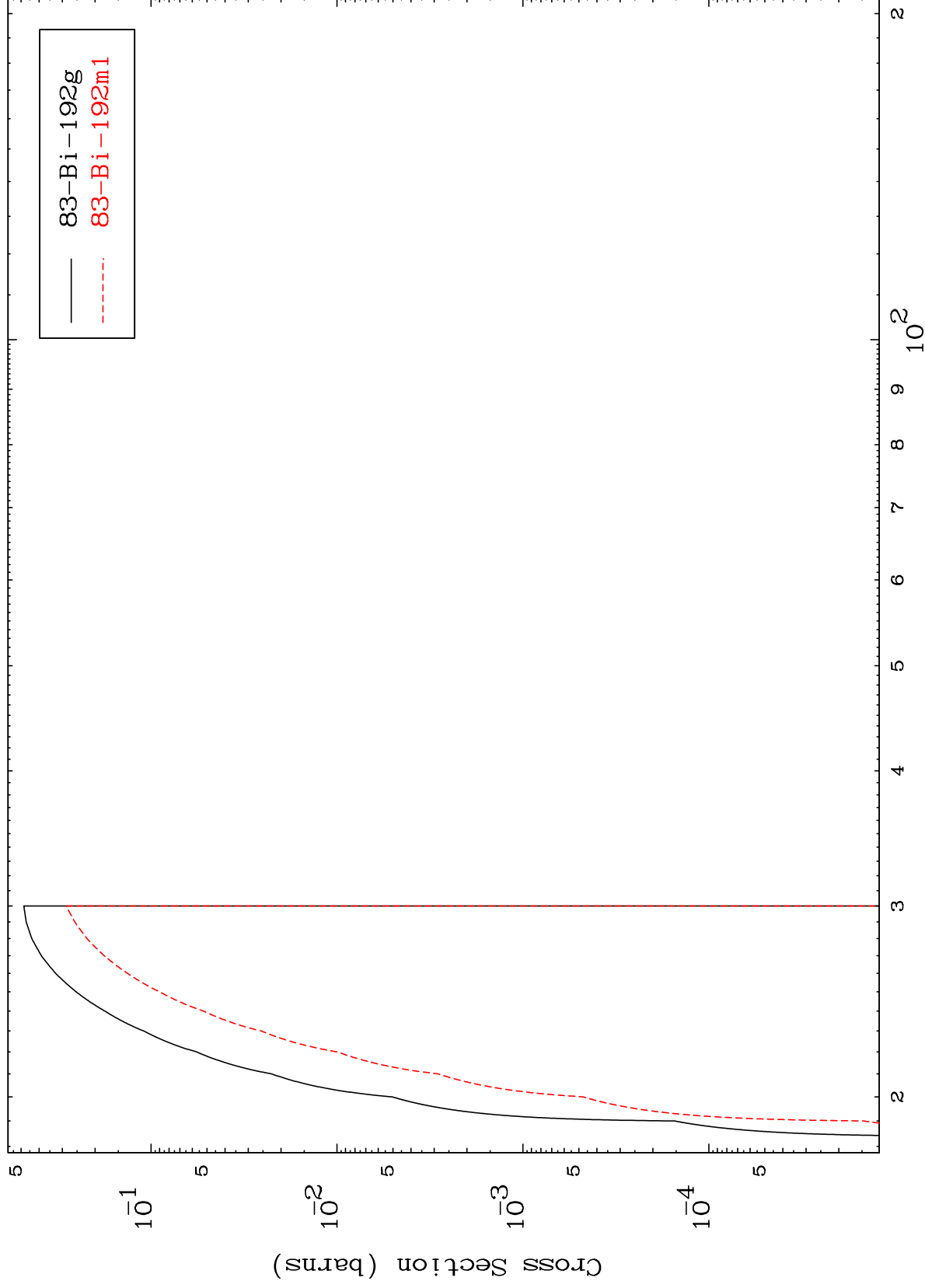
(n,2n)
Radionuclide Production Cross Section



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$^{83}\text{Bi}-194$

$(n,3n)$
Radionuclide Production Cross Section



20

Incident Energy (MeV)

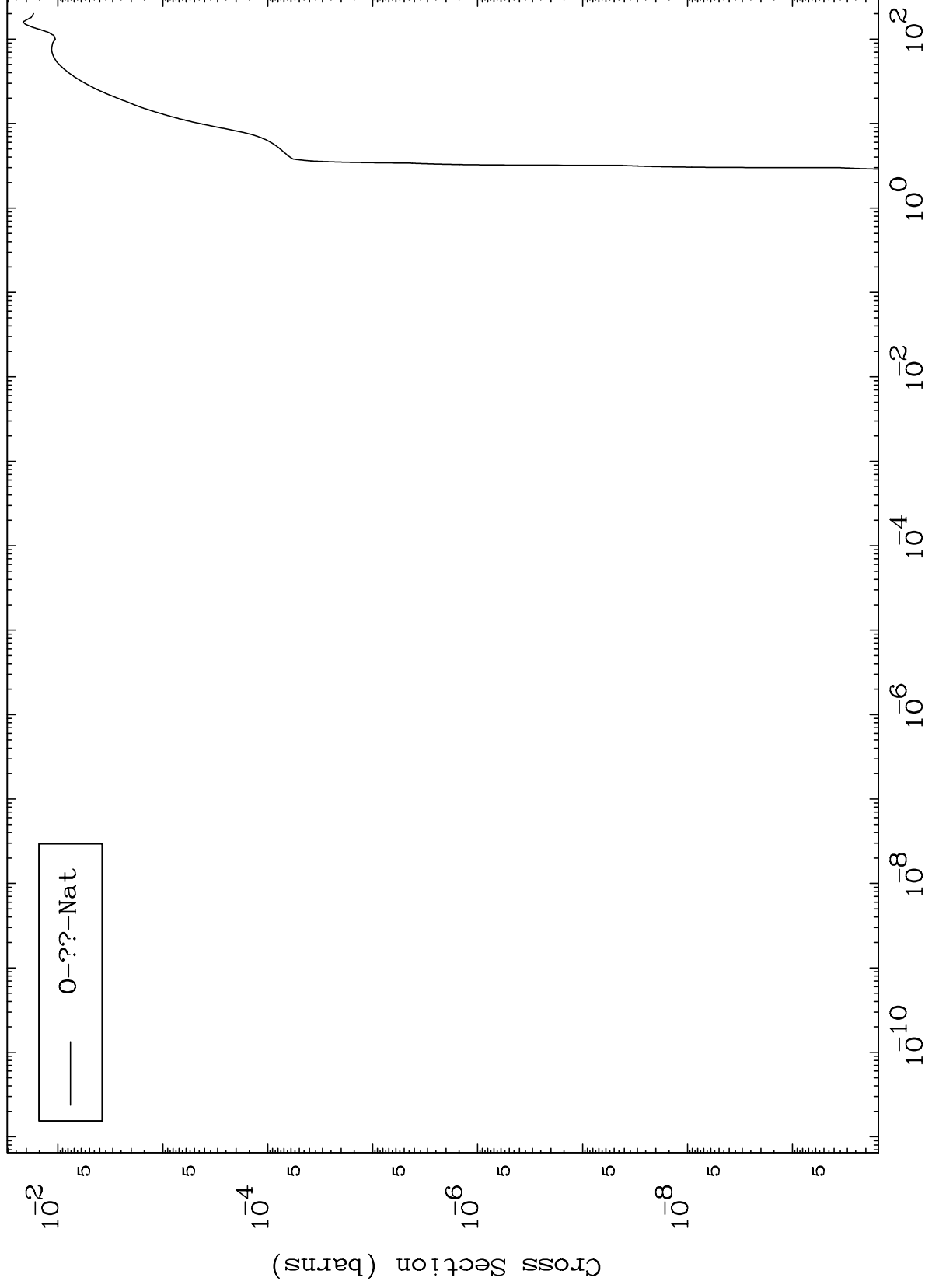
$^{83}\text{Bi}-194$

MAT 8282

Fission

83-Bi-194

Radionuclide Production Cross Section



0-??-Nat

21

Incident Energy (MeV)

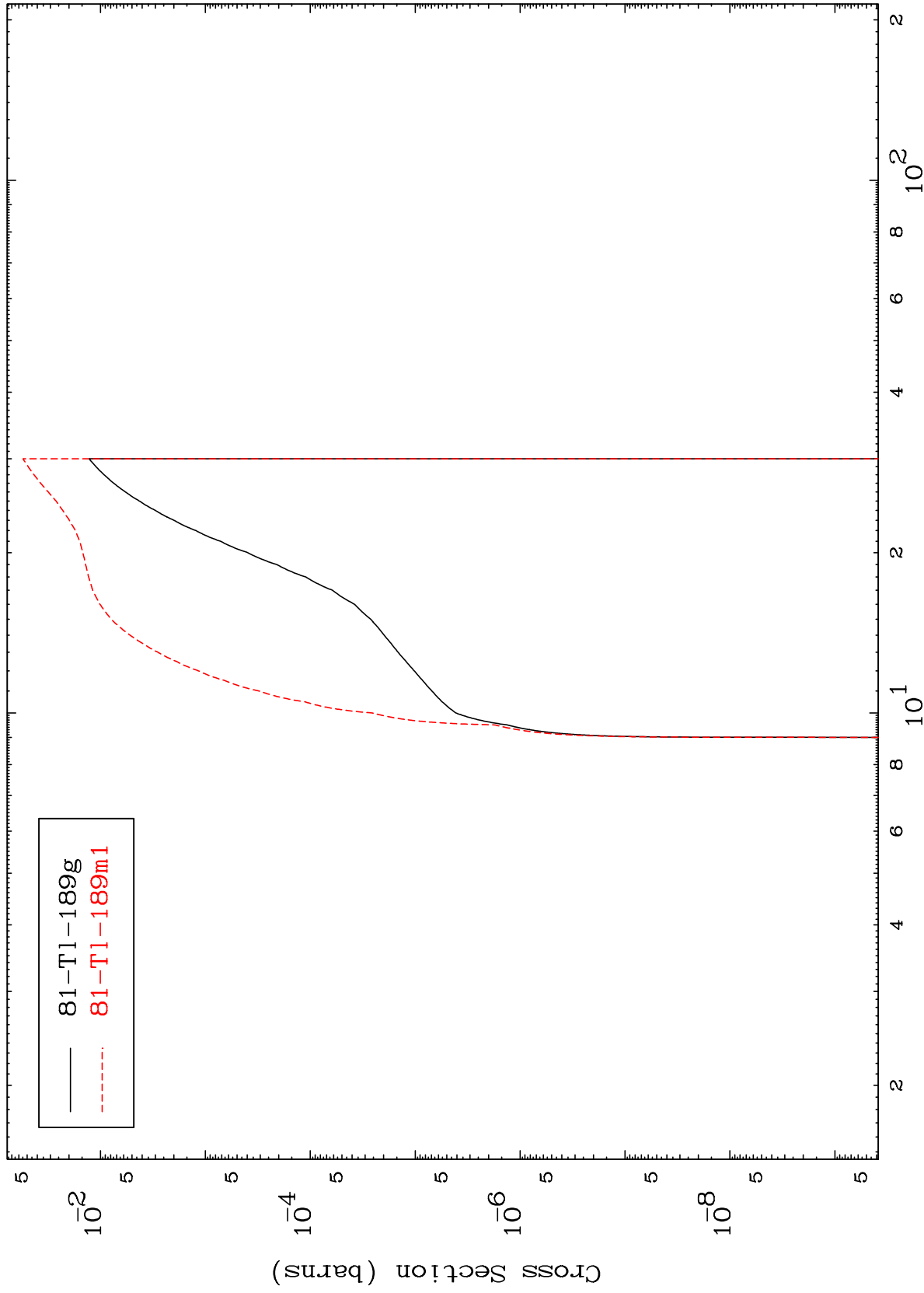
83-Bi-194

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$(n,2n) \alpha$

83-Bi-194

Radionuclide Production Cross Section

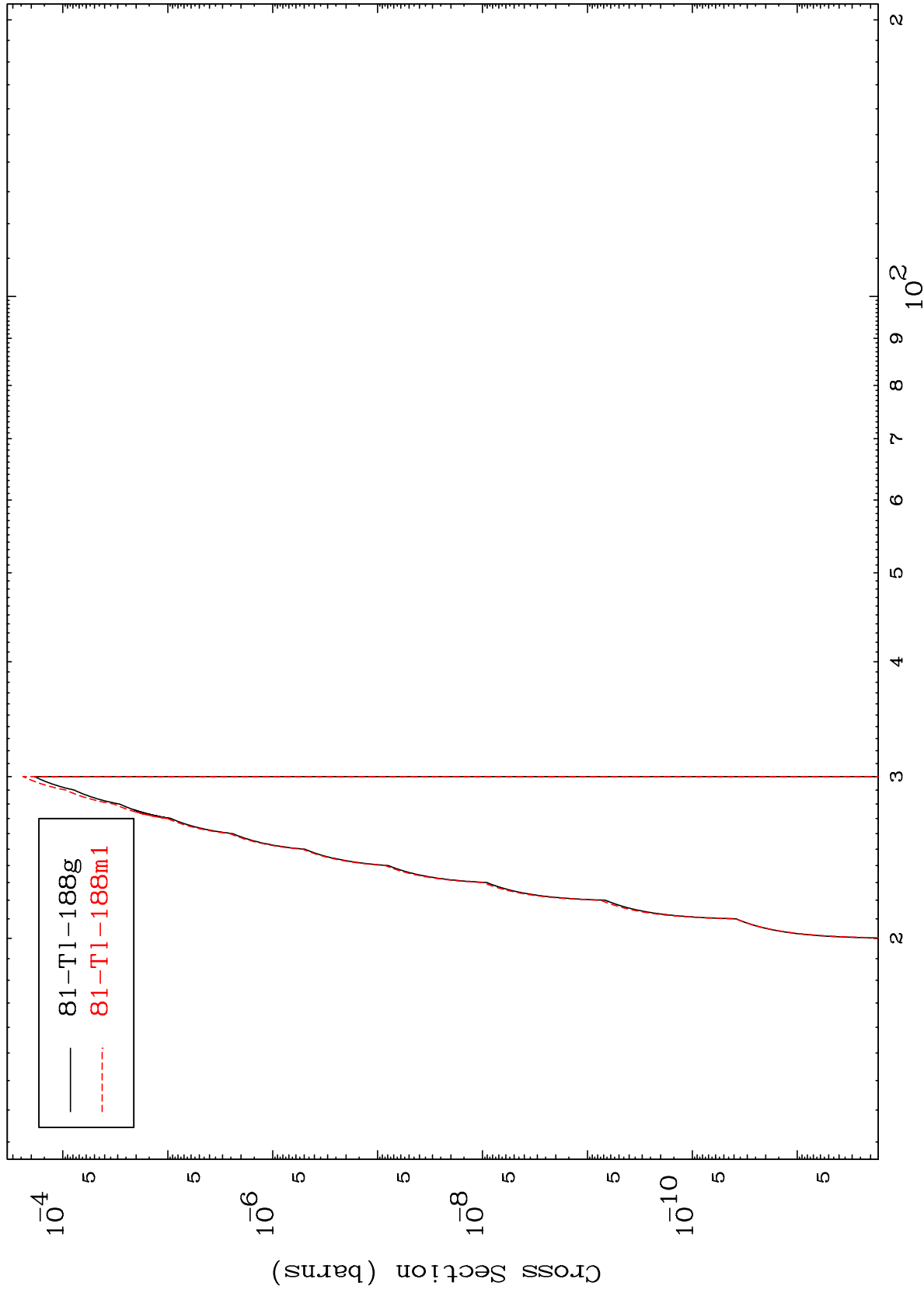


22

Incident Energy (MeV)

83-Bi-194

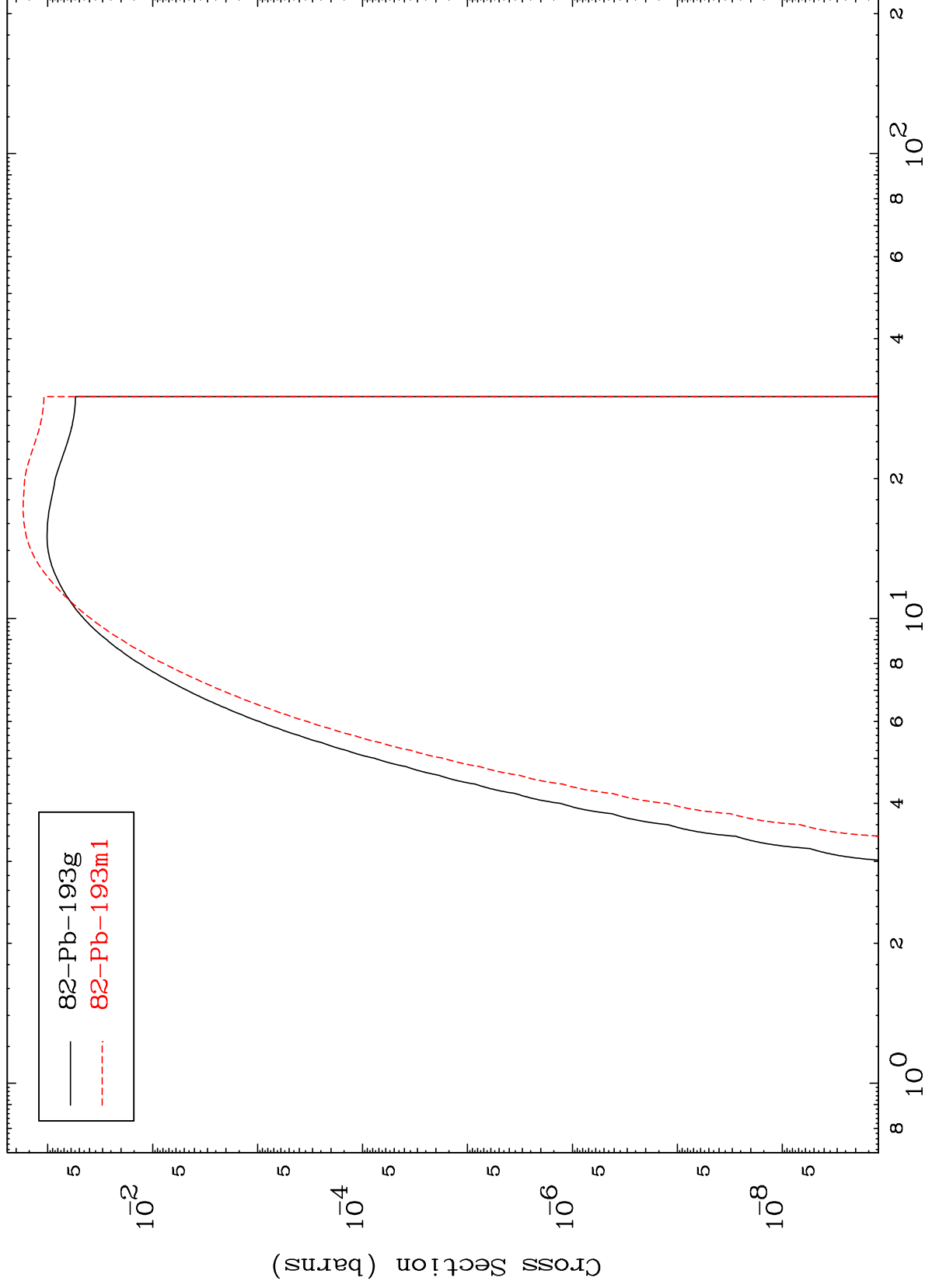
Radionuclide Production Cross Section



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83-Bi-194

(n,n') p
Radionuclide Production Cross Section



24

Incident Energy (MeV)

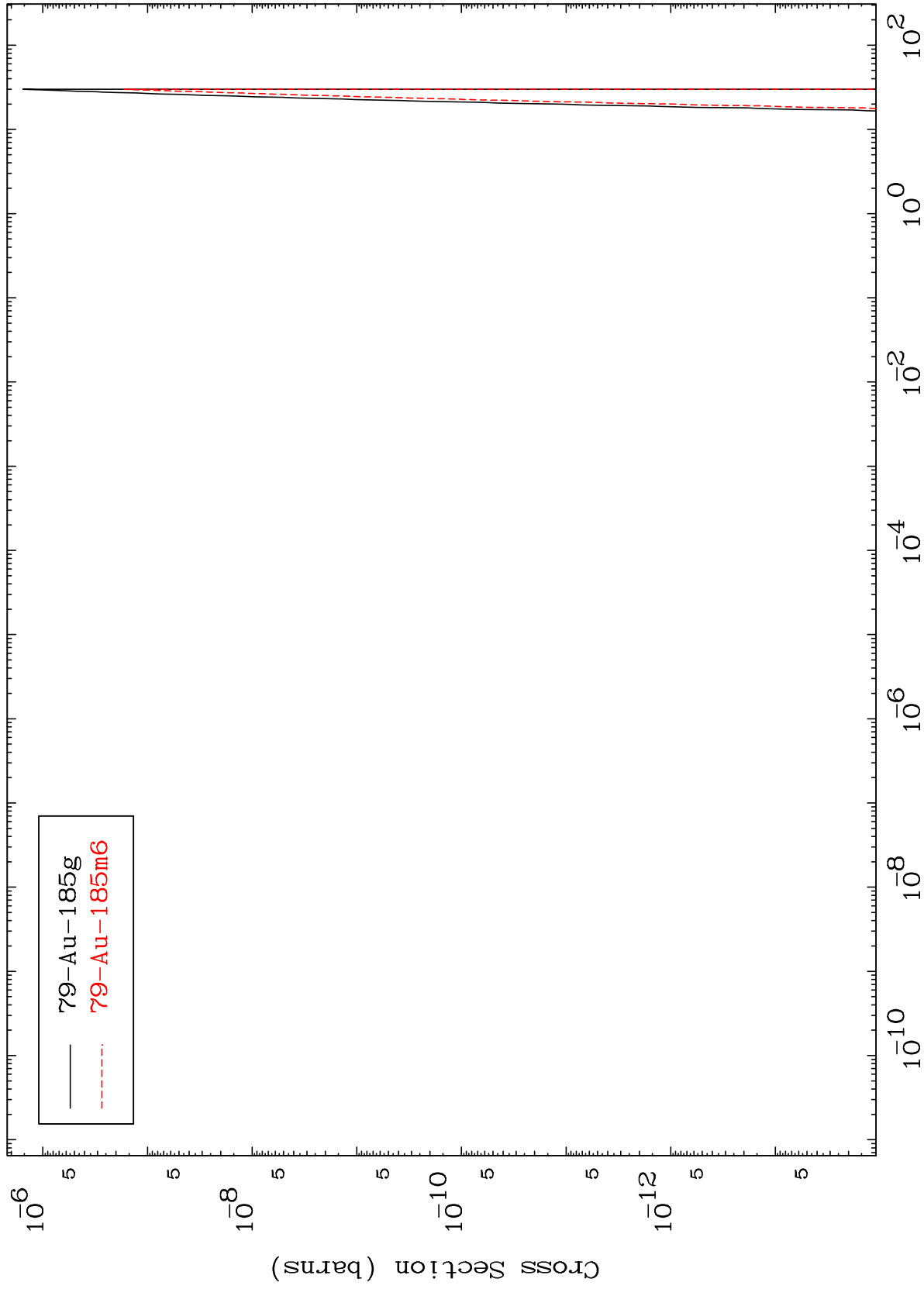
83-Bi-194

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(n,2n) 2α

83-Bi-194

Radionuclide Production Cross Section

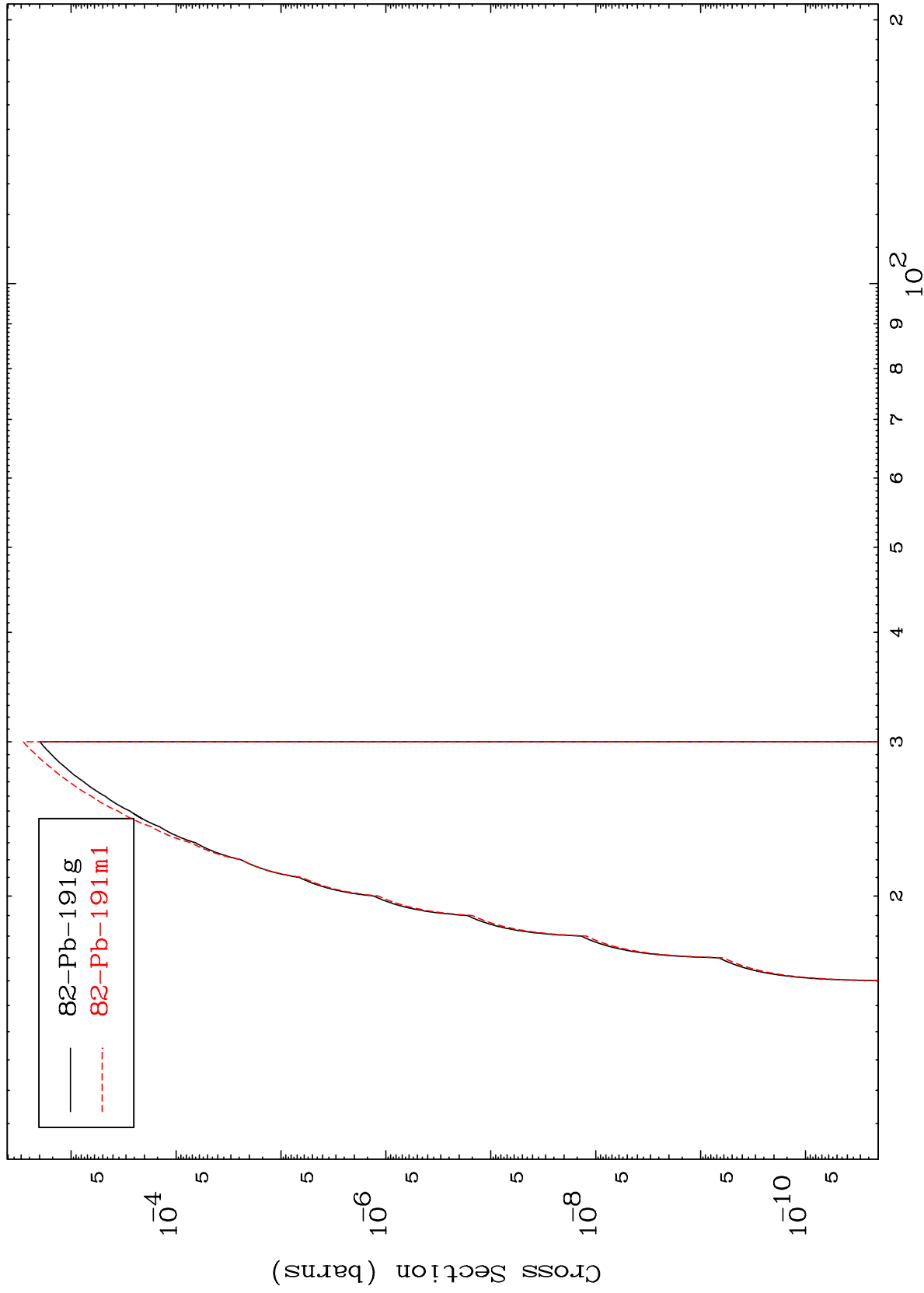


25

Incident Energy (MeV)

83-Bi-194

(n,n') t
Radionuclide Production Cross Section

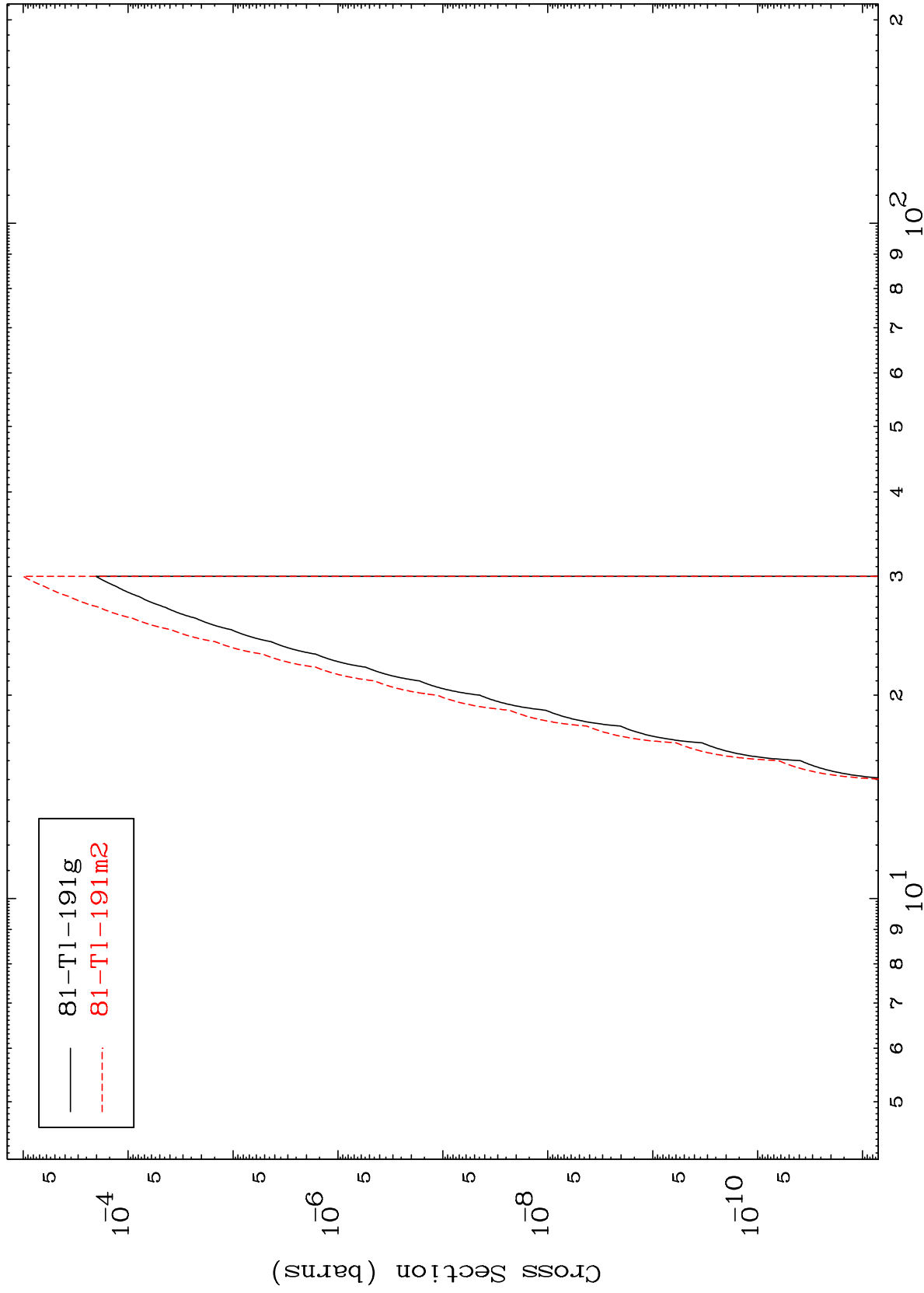


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(n,n') He-3

83-Bi-194

Radionuclide Production Cross Section



27

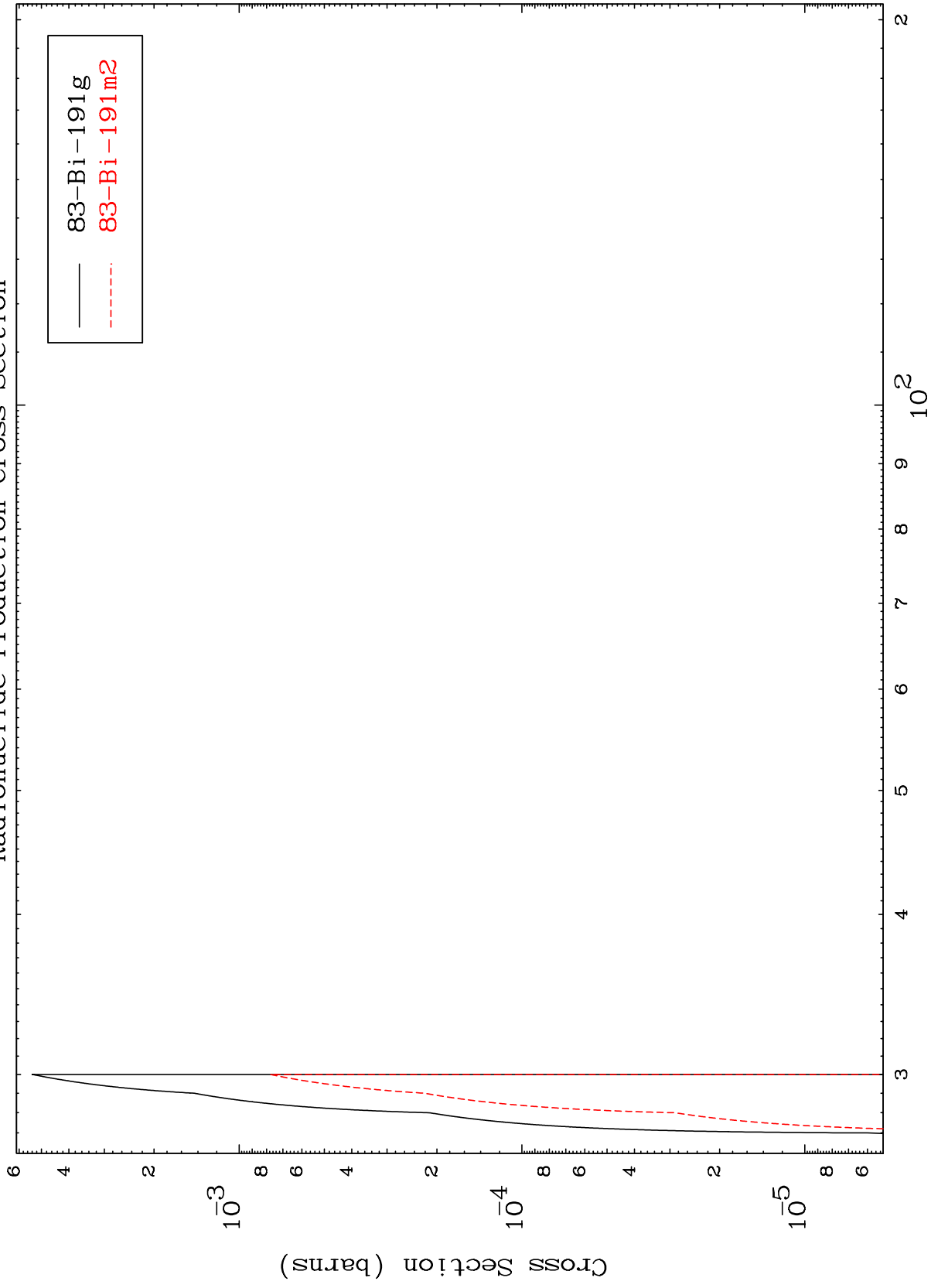
Incident Energy (MeV)

83-Bi-194

MAT 8282

$^{83}\text{Bi}-^{194}$

$(n,4n)$
Radionuclide Production Cross Section



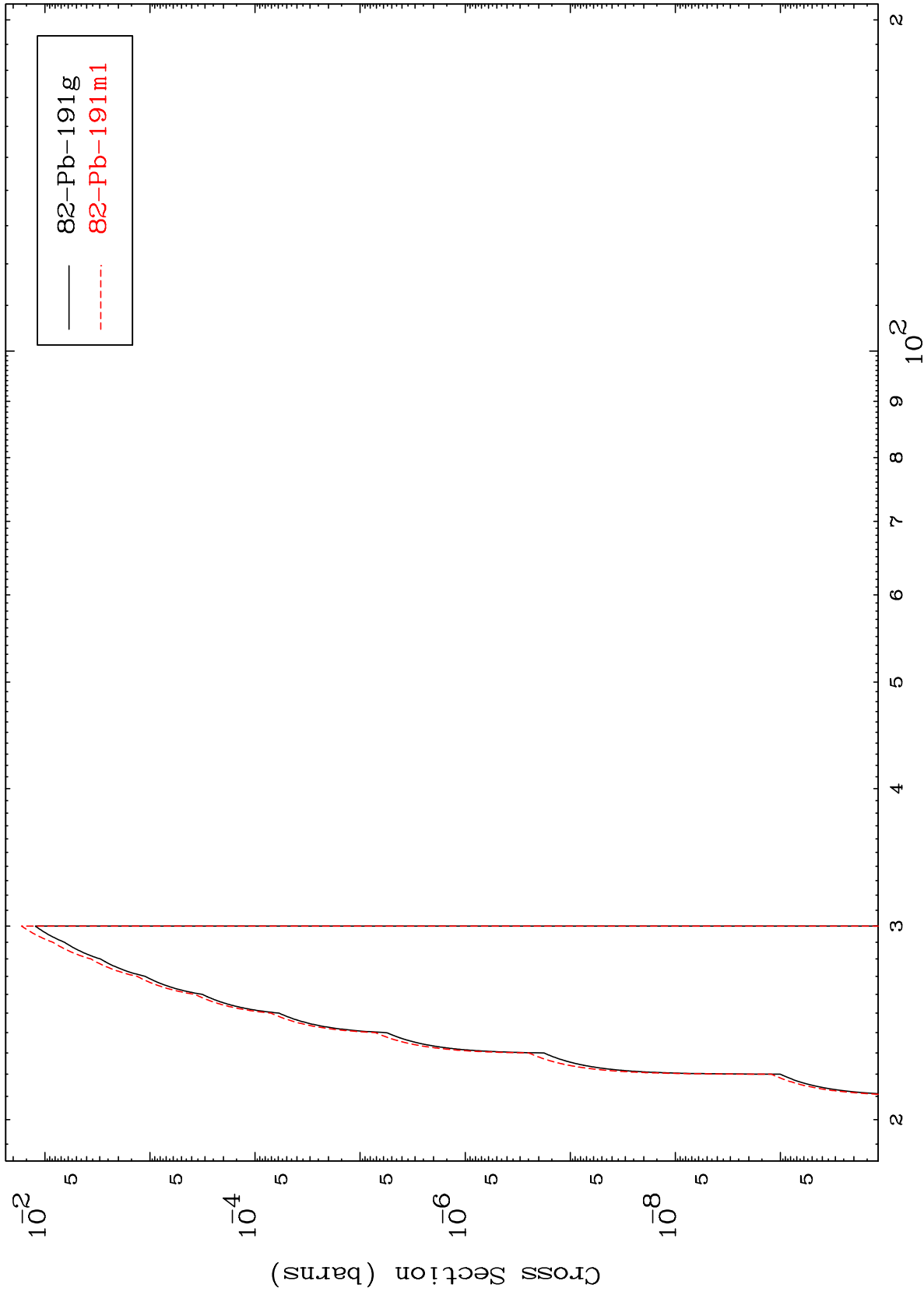
28

$^{83}\text{Bi}-^{194}$

MAT 8282

83-Bi-194

(n,3n) p
Radionuclide Production Cross Section



29

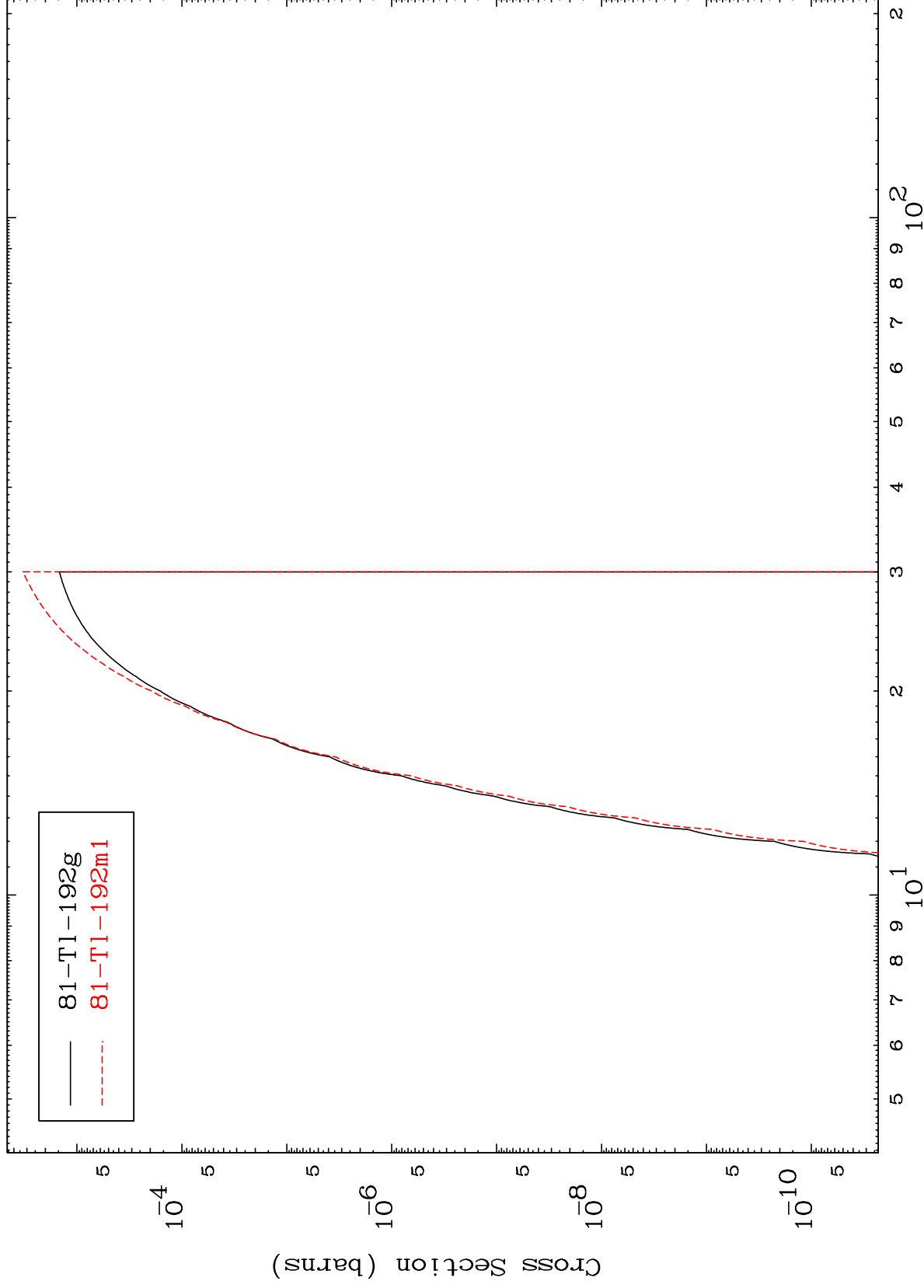
Incident Energy (MeV)

83-Bi-194

MAT 8282

83-Bi-194

(n,2n) p
Radionuclide Production Cross Section



30

Incident Energy (MeV)

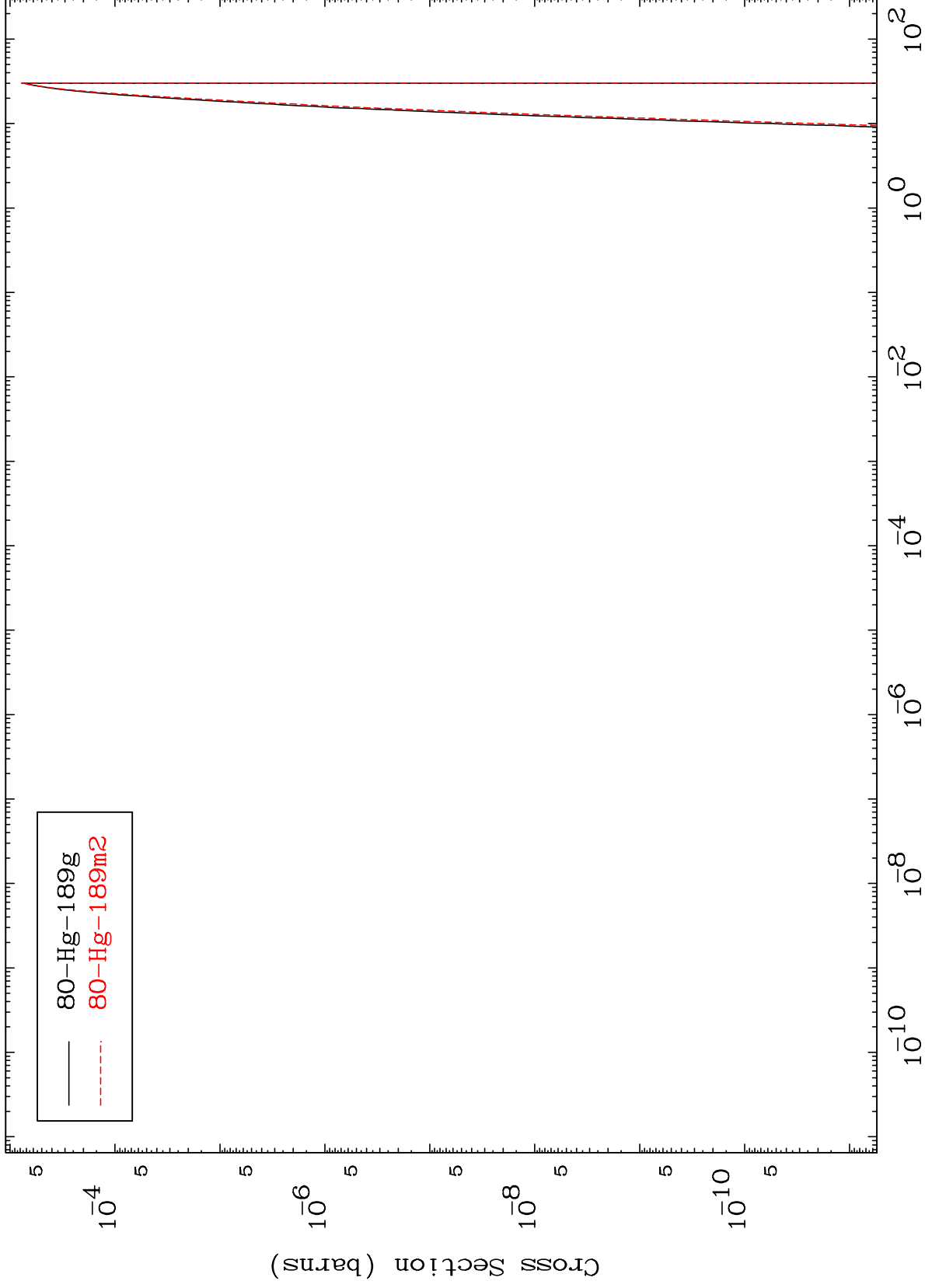
83-Bi-194

MAT 8282

(n,n') p α

83-Bi-194

Radionuclide Production Cross Section



31

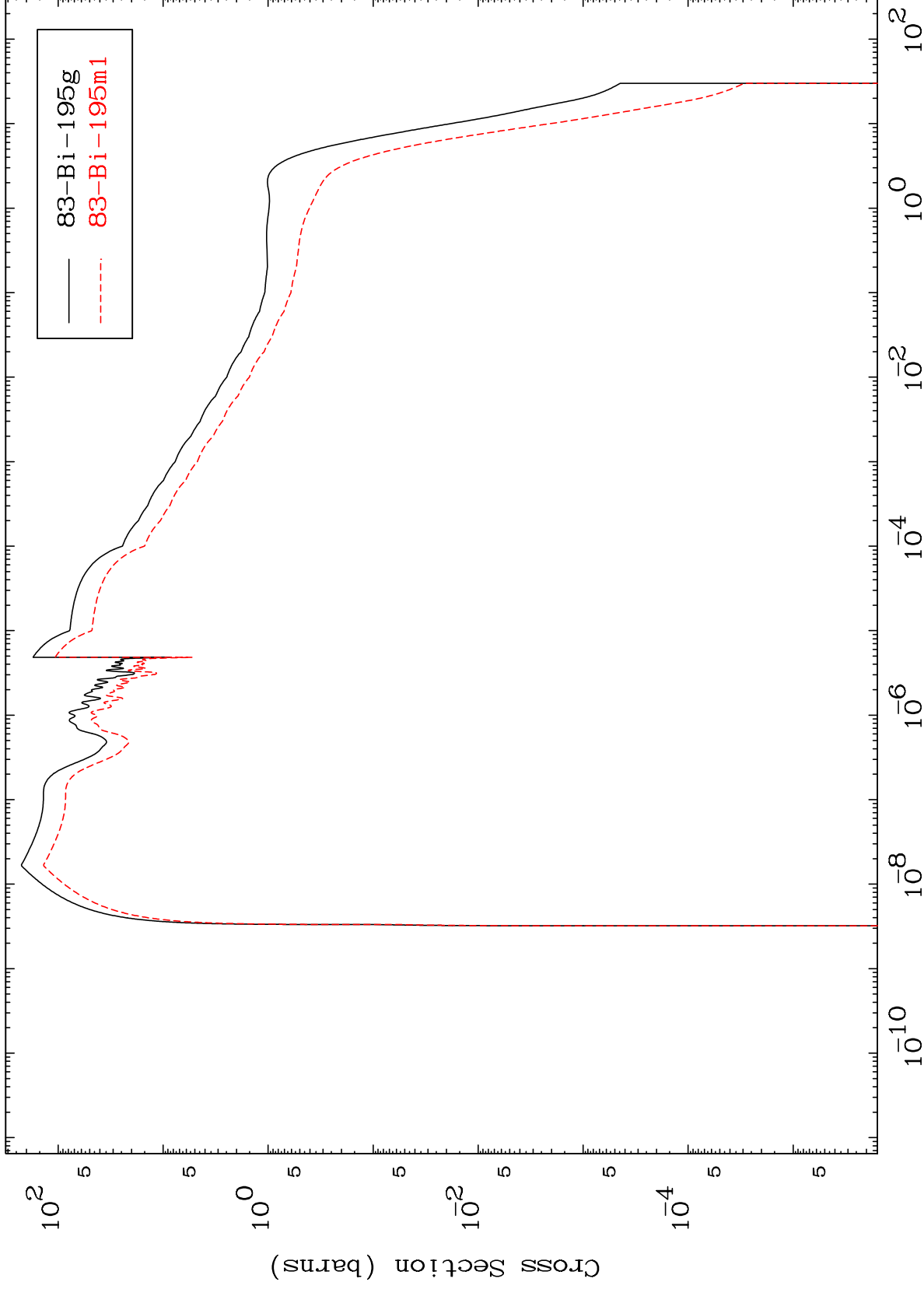
Incident Energy (MeV)

83-Bi-194

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83-Bi-194

Radionuclide Production Cross Section
(n, γ)



32

Incident Energy (MeV)

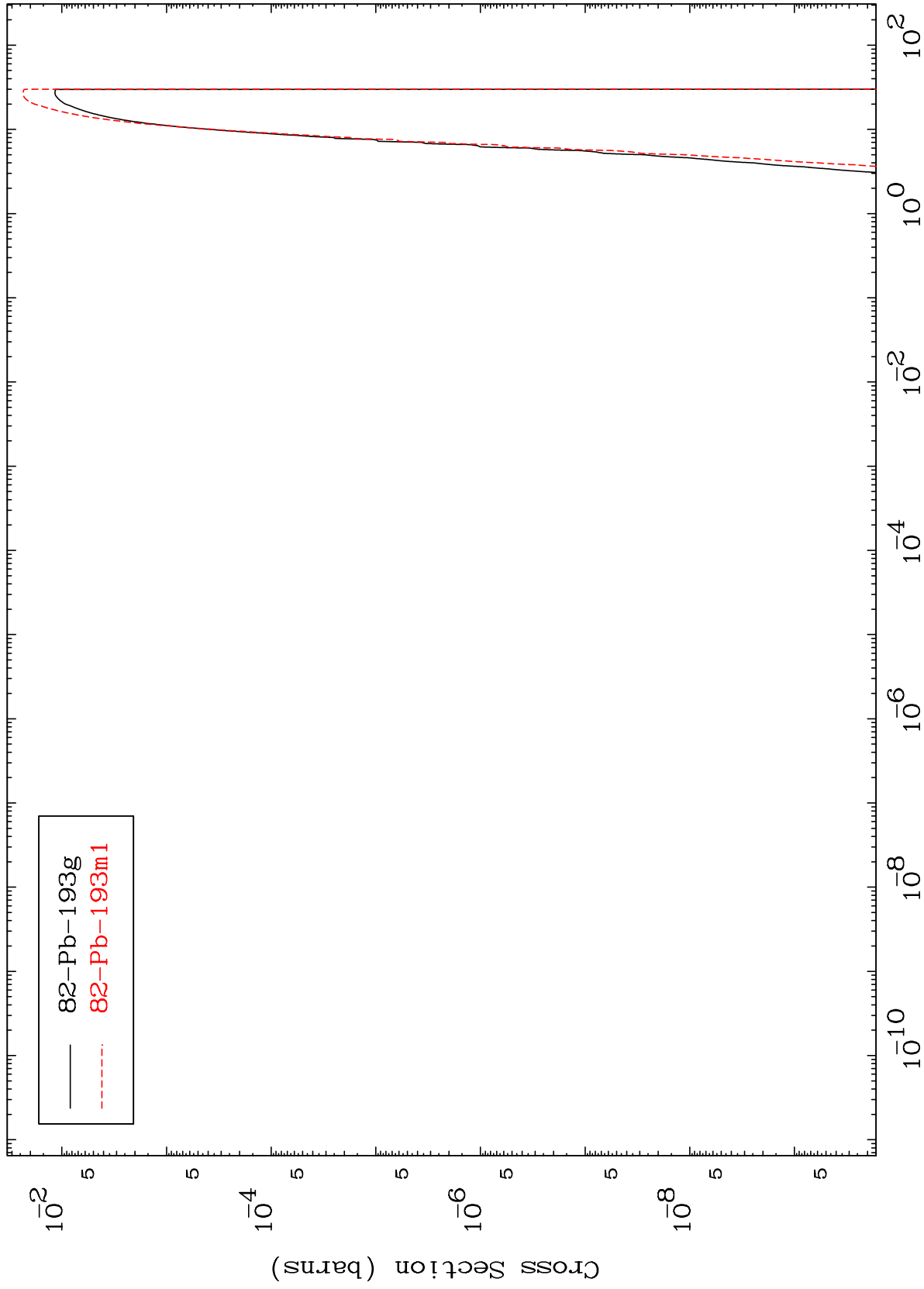
83-Bi-194

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(n,d)

83-Bi-194

Radionuclide Production Cross Section



33

Incident Energy (MeV)

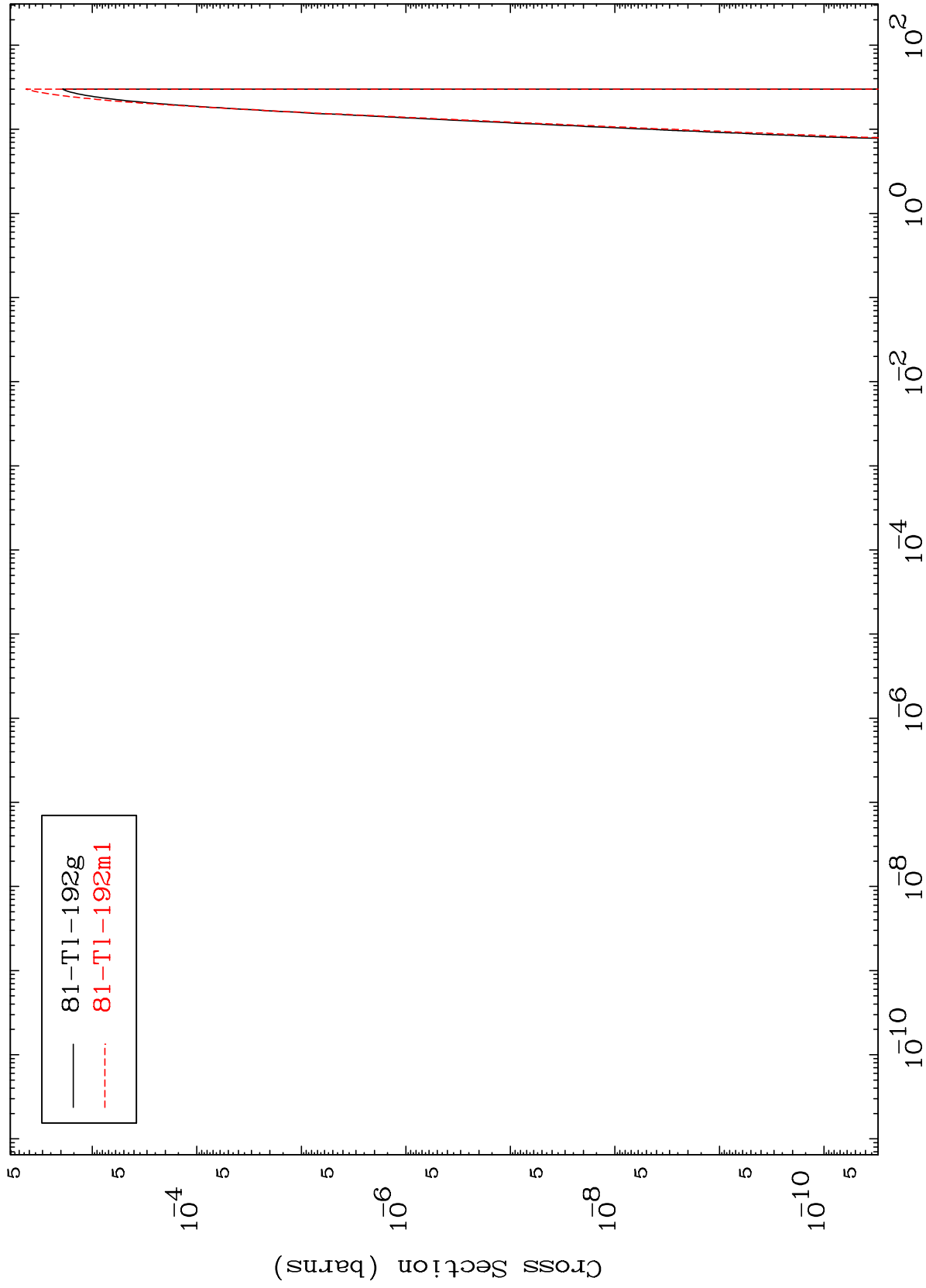
83-Bi-194

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(n,He-3)

83-Bi-194

Radionuclide Production Cross Section



34

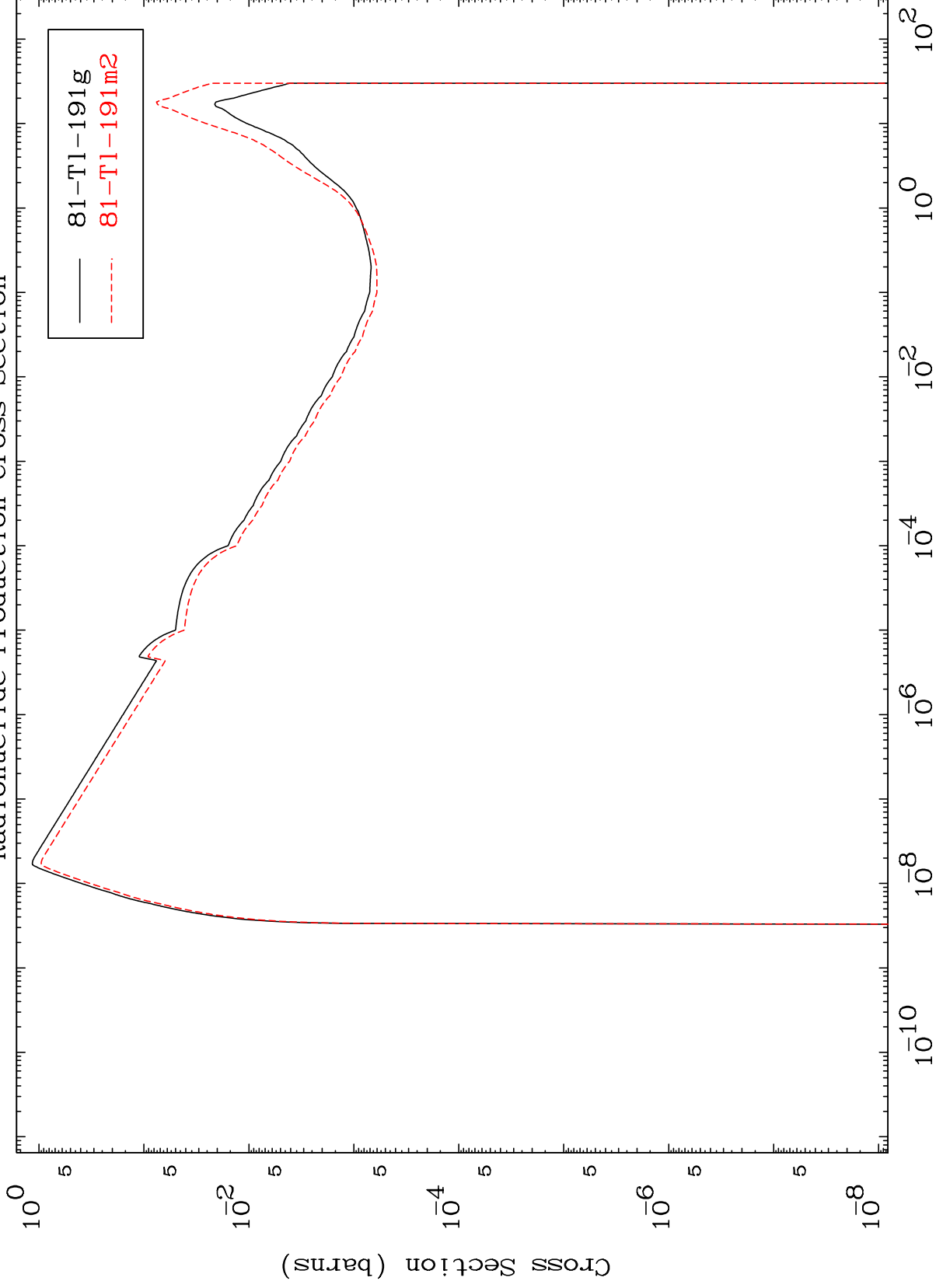
Incident Energy (MeV)

83-Bi-194

MAT 8282

83-Bi-194

(n,α)
Radionuclide Production Cross Section



35

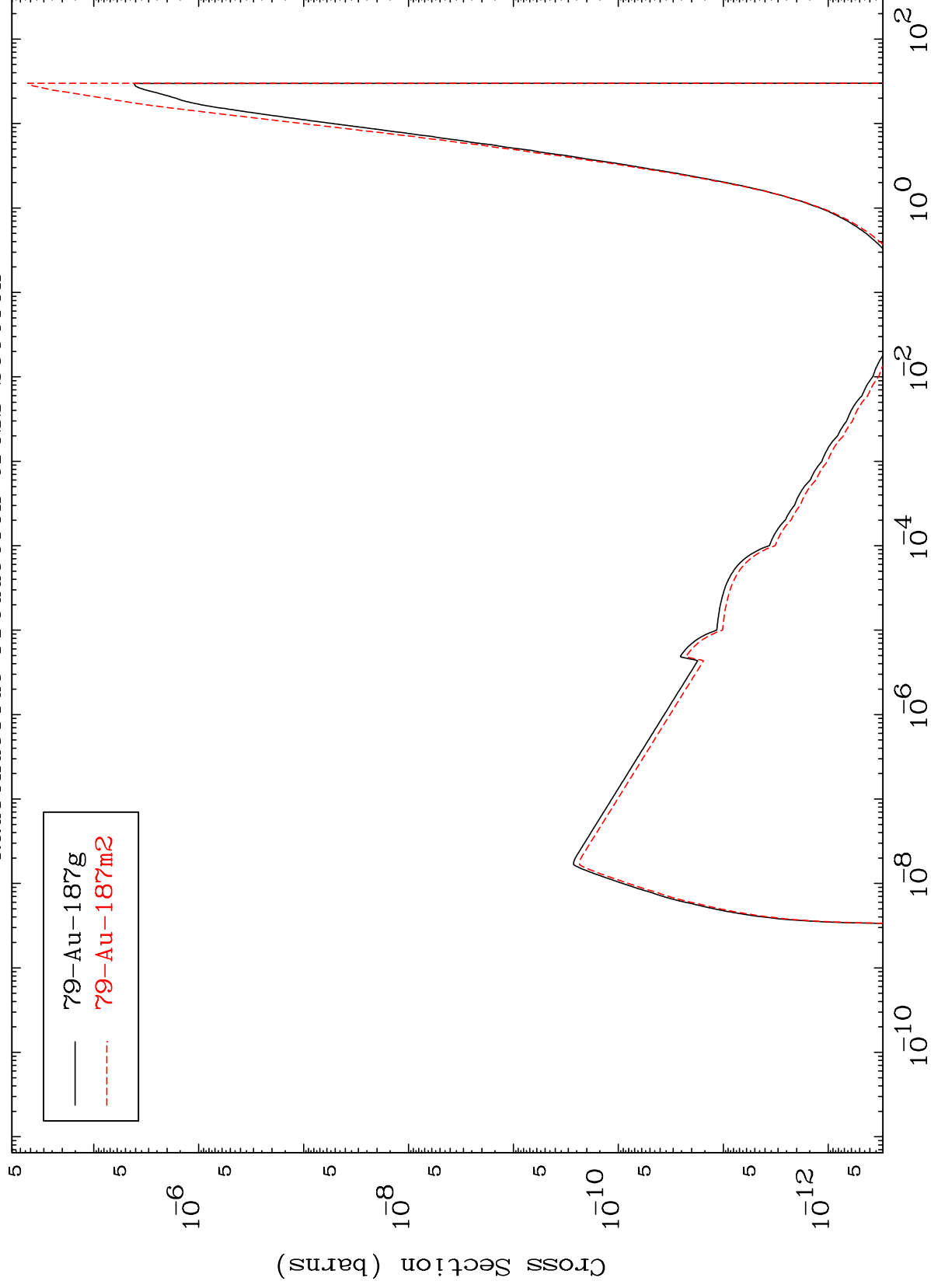
Incident Energy (MeV)

83-Bi-194

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Radionuclide Production Cross Section
(n,2α)

83-Bi-194



36

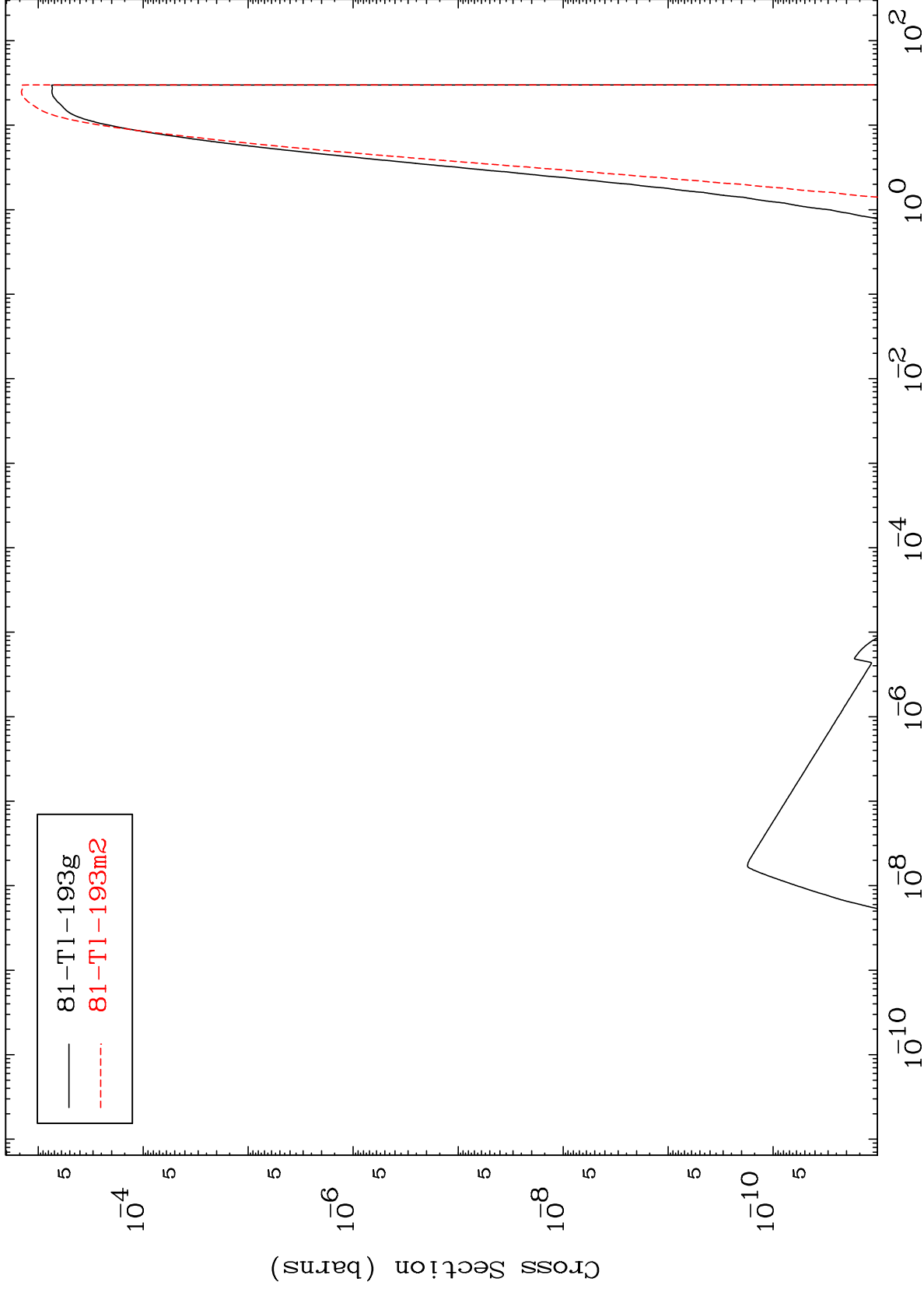
Incident Energy (MeV)

83-Bi-194

MAT 8282

Radionuclide Production Cross Section
(n,2p)

83-Bi-194



37

Incident Energy (MeV)

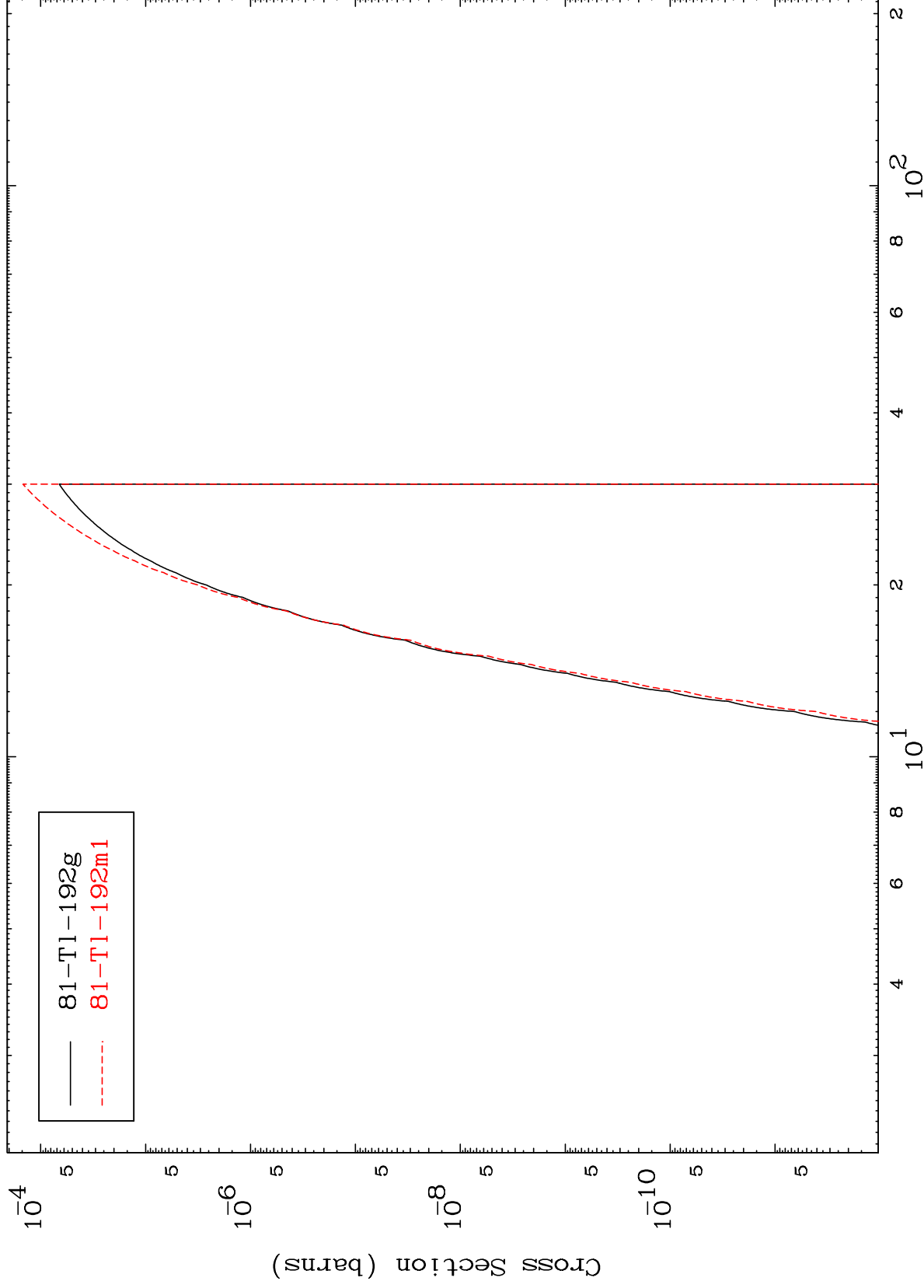
83-Bi-194

MAT 8282

(n,p) d

83-Bi-194

Radionuclide Production Cross Section



38

Incident Energy (MeV)

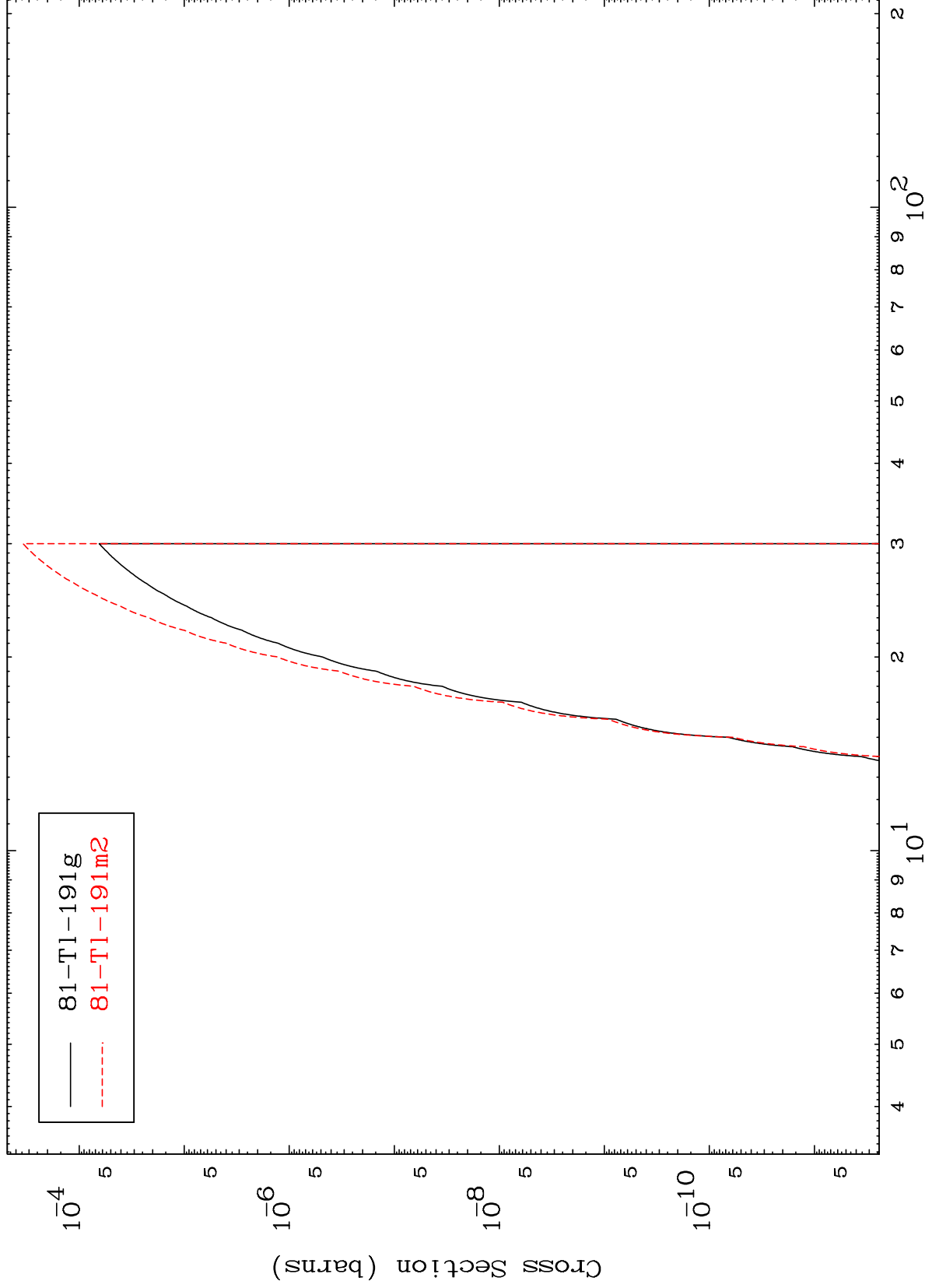
83-Bi-194

MAT 8282

(n,p) t

83-Bi-194

Radionuclide Production Cross Section



39

Incident Energy (MeV)

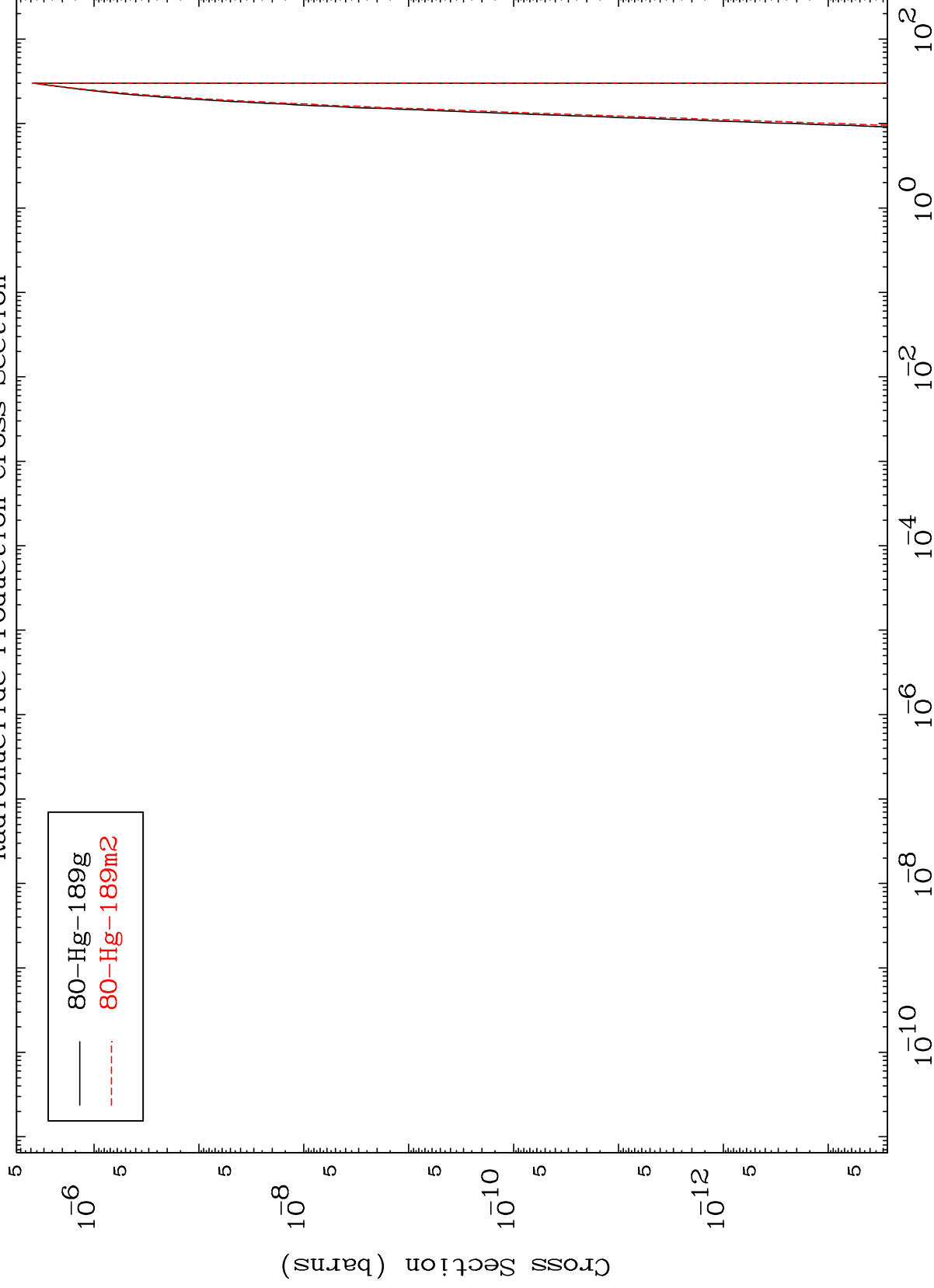
83-Bi-194

MAT 8282

(n,d) α

83-Bi-194

Radionuclide Production Cross Section



40

Incident Energy (MeV)

83-Bi-194