

Program EVALPLOT
(Version 2018-1)

by

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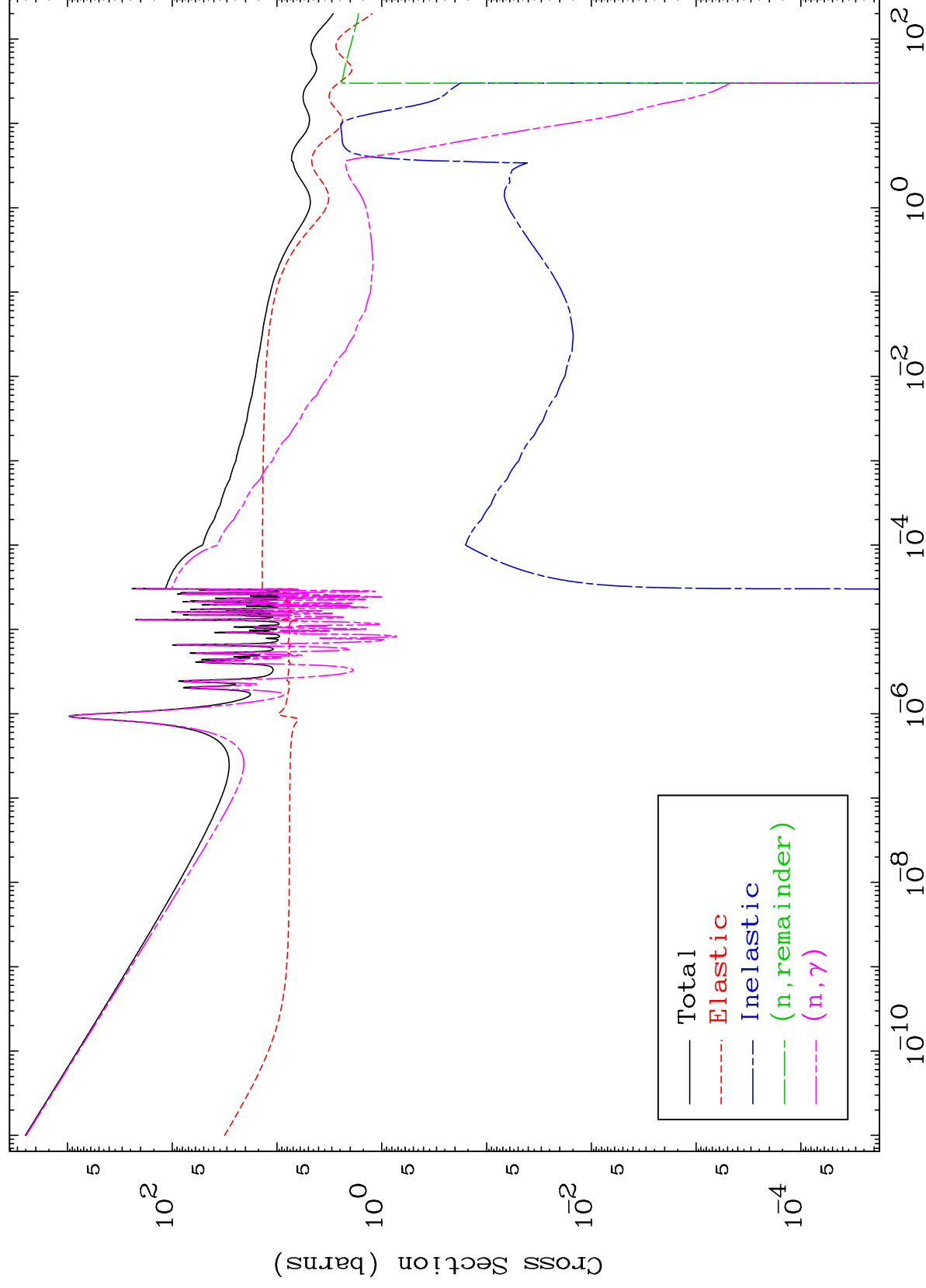
Press Mouse Button to Start

MAT 8290

Major

293 Kelvin Cross Sections

83-Bi-197



1

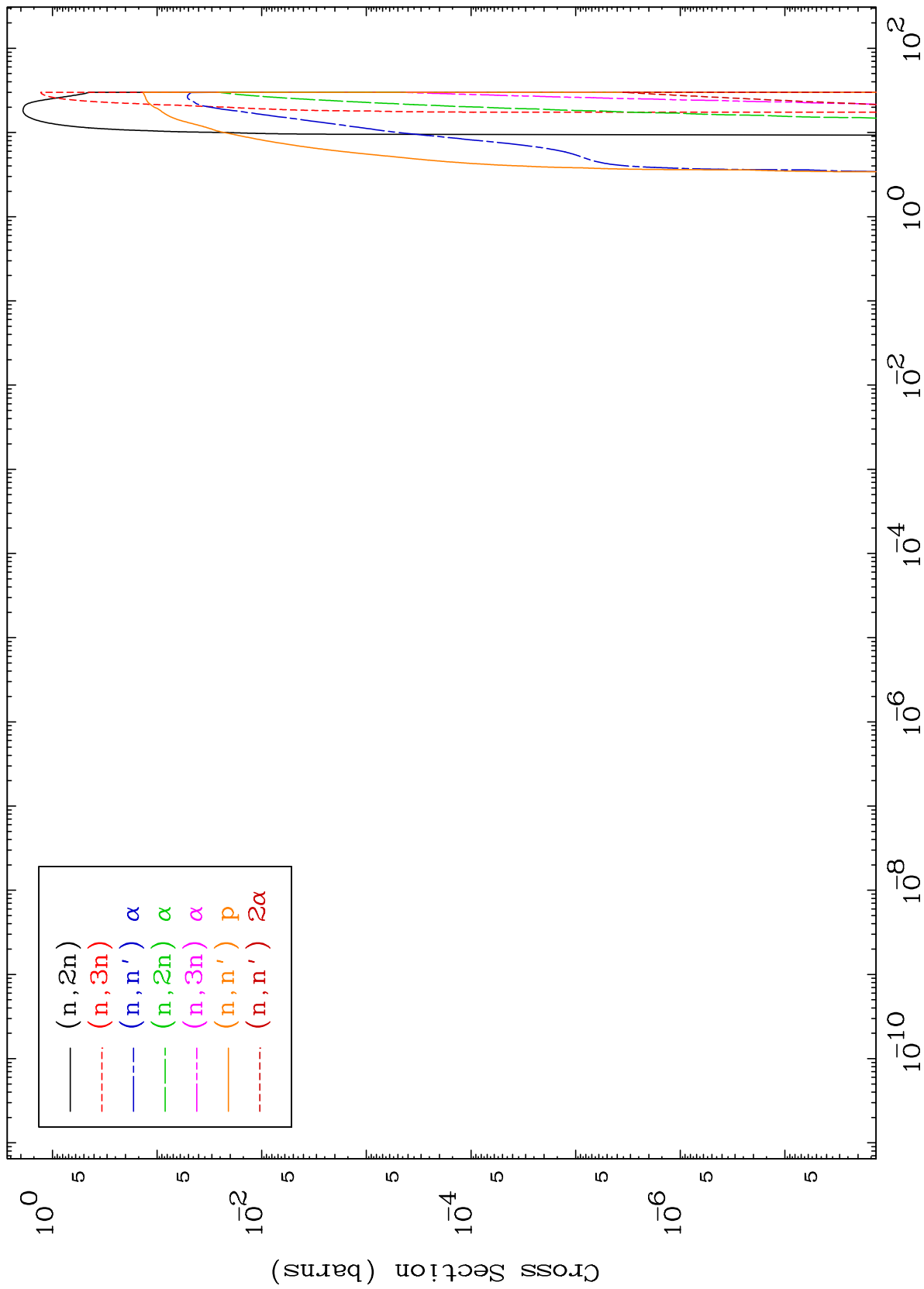
Incident Energy (MeV)

83-Bi-197

MAT 8290

Neutron Production
293 Kelvin Cross Sections

83-Bi-197



2

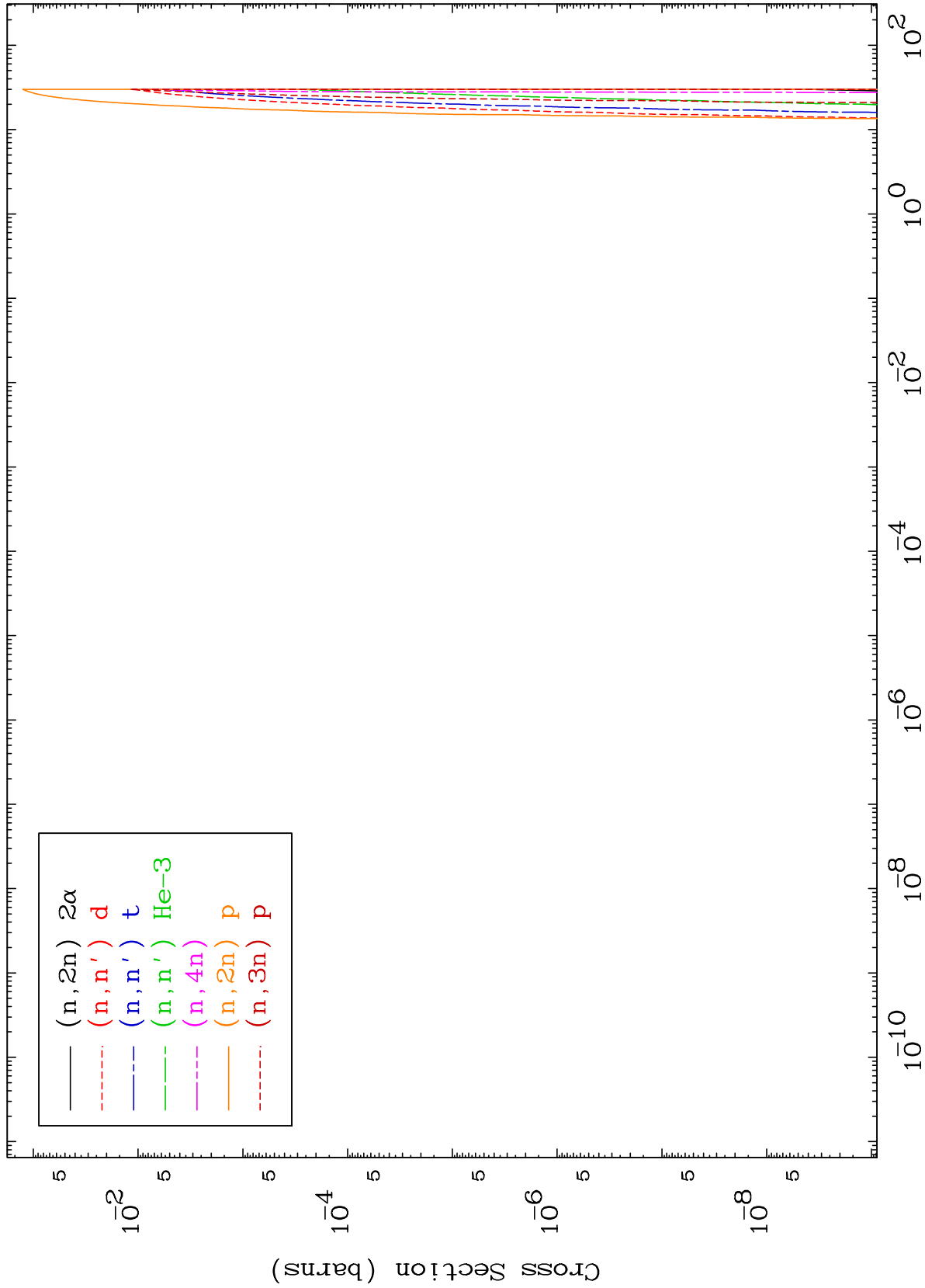
Incident Energy (MeV)

83-Bi-197

MAT 8290

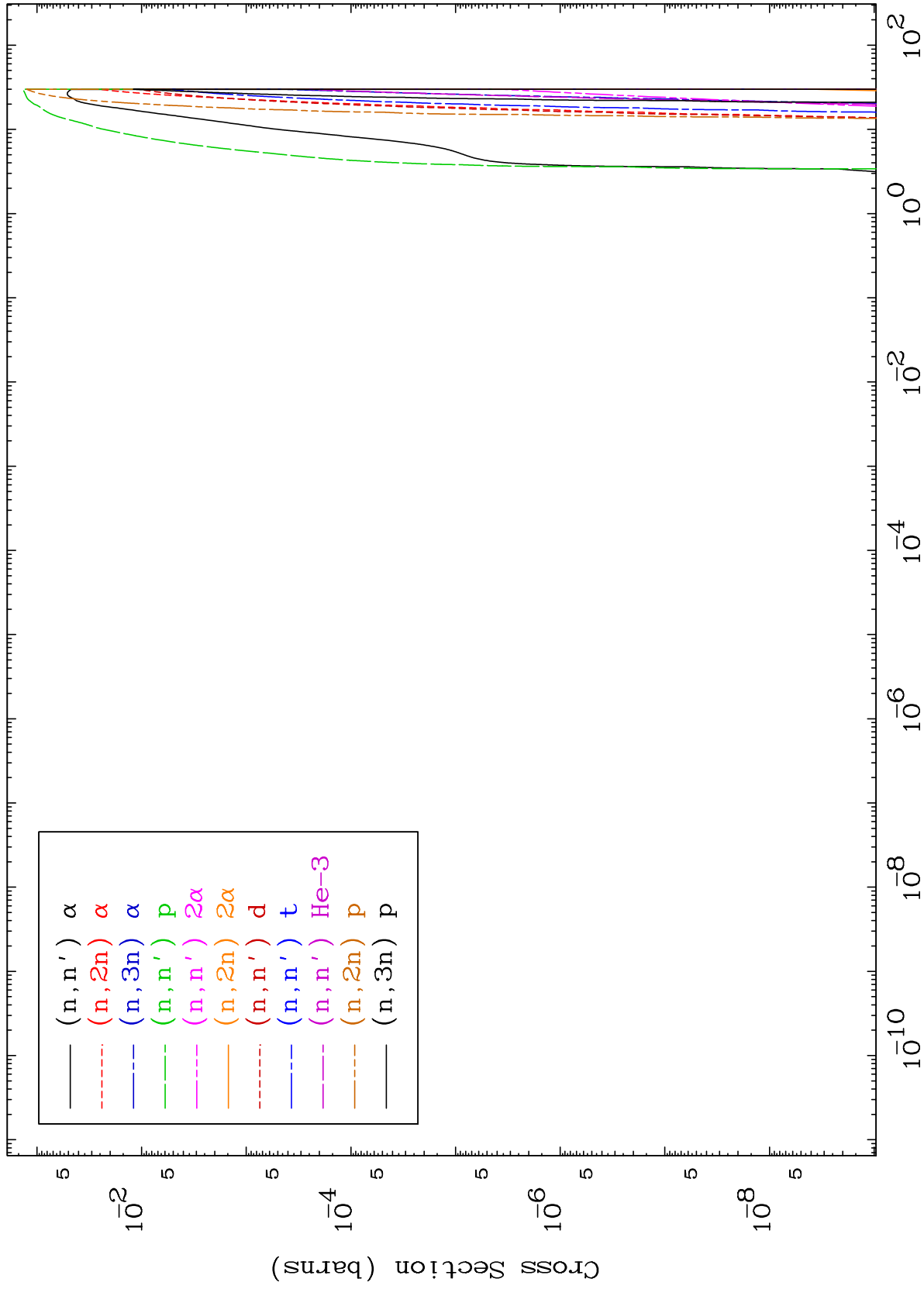
Neutron Production
293 Kelvin Cross Sections

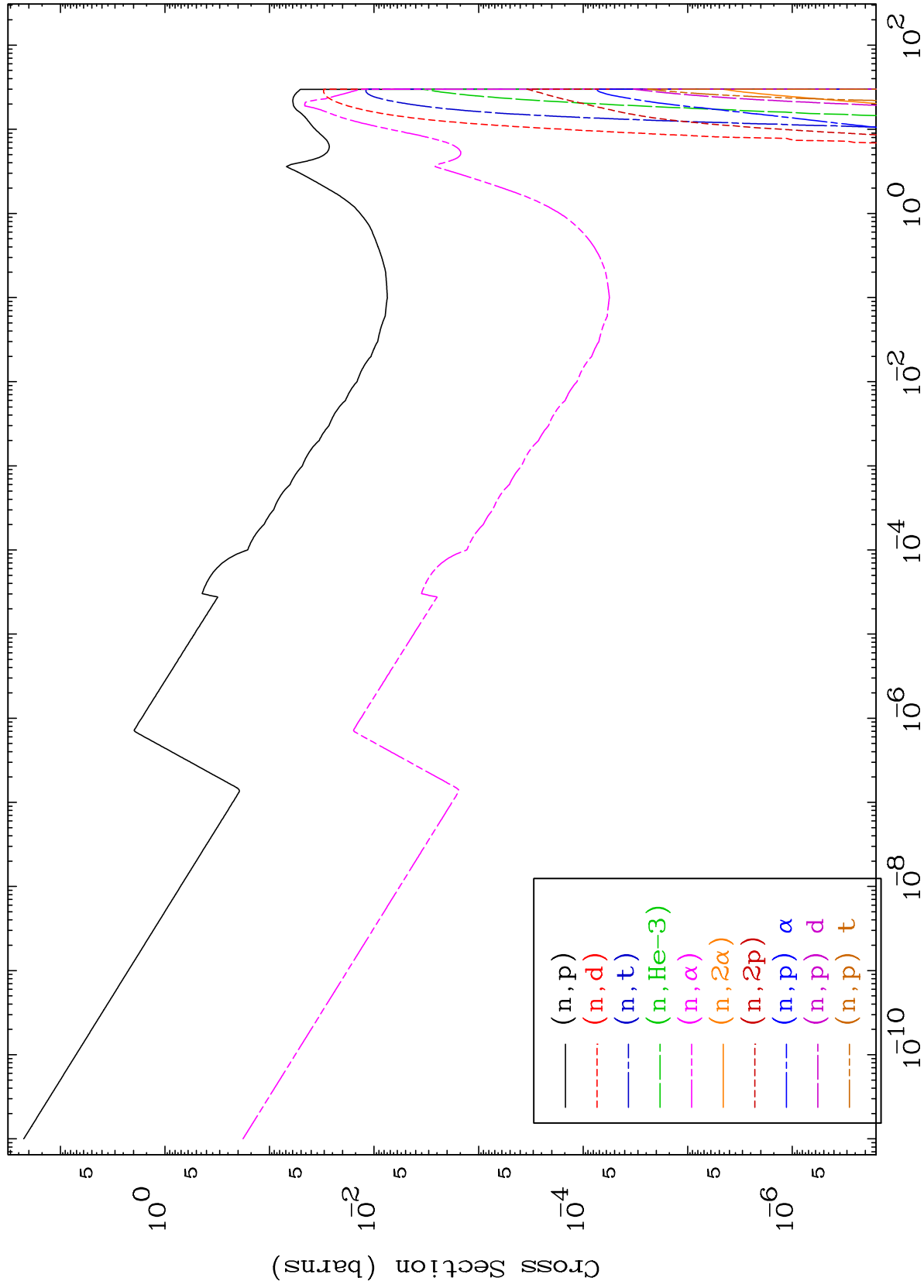
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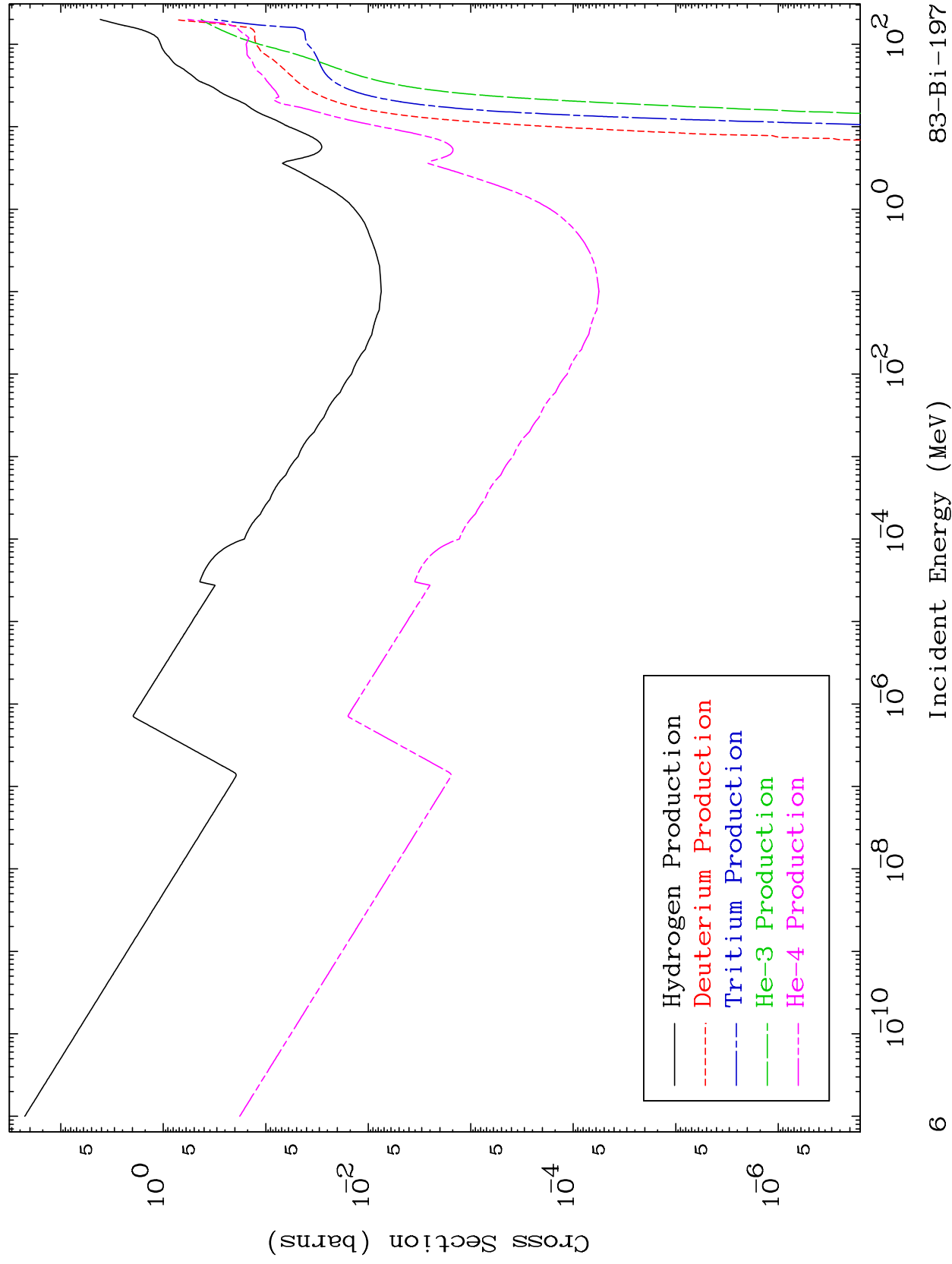


3

83-Bi-197





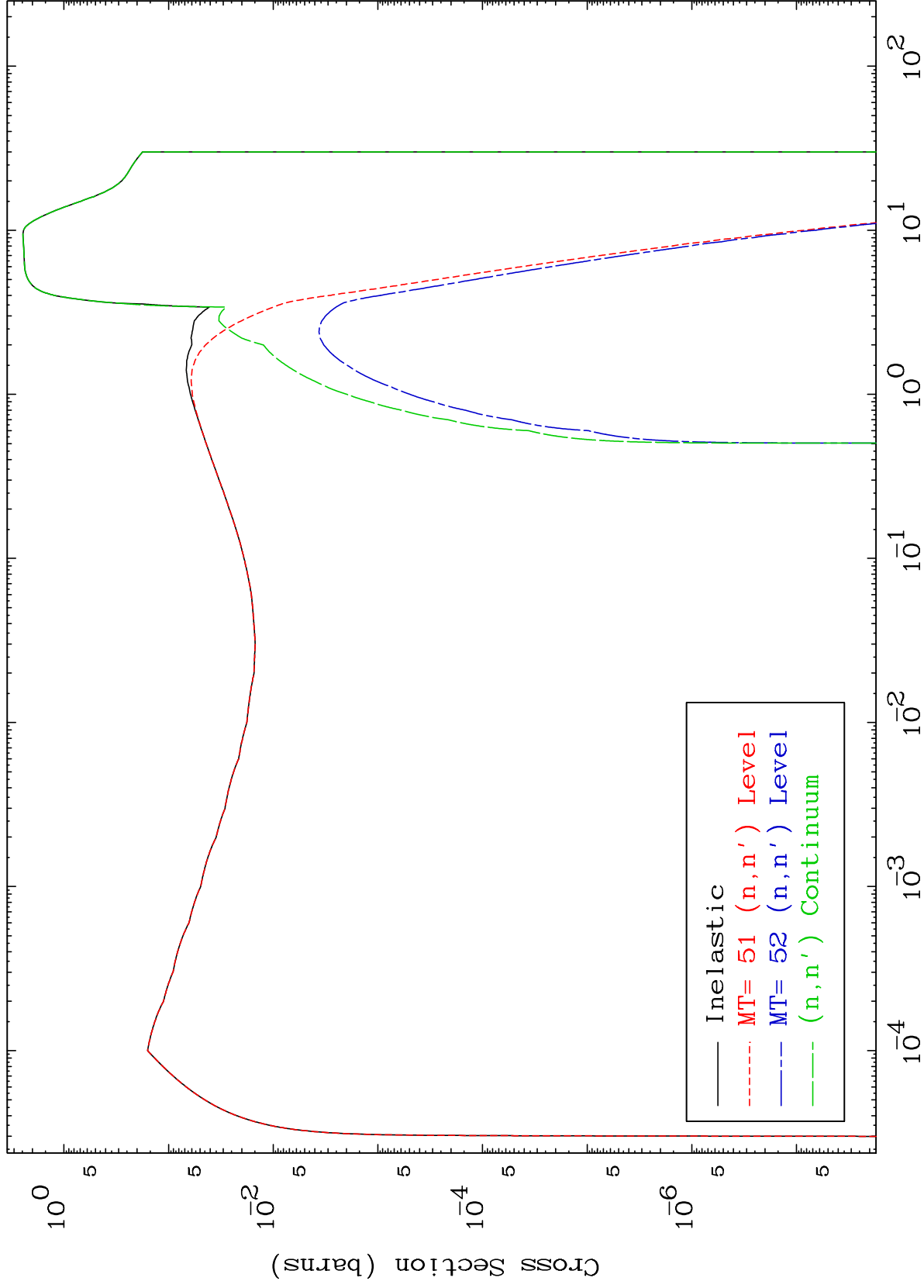


MAT 8290

(n,n') Level

293 Kelvin Cross Sections

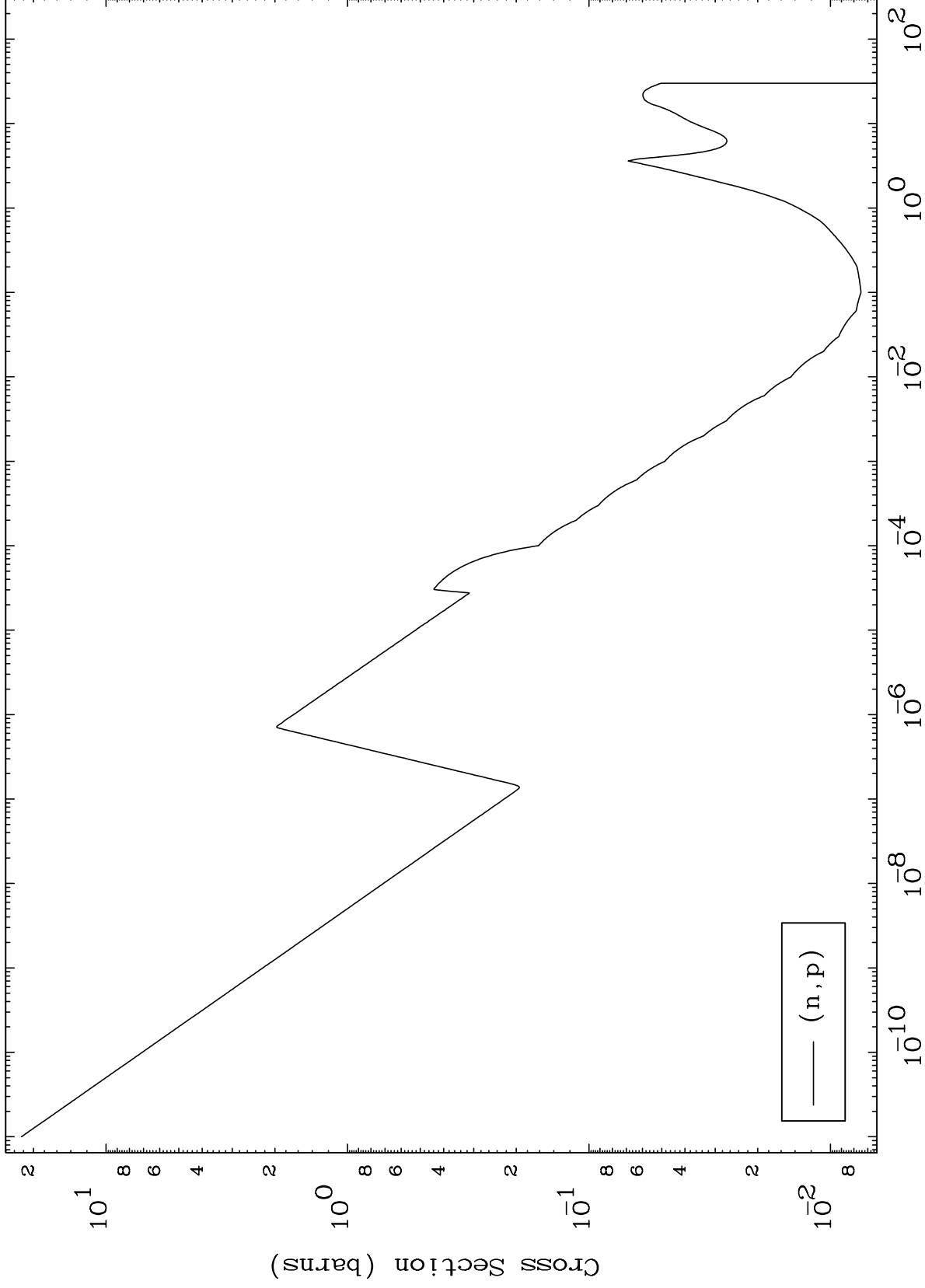
83-Bi-197



MAT 8290

(n,p) Levels
293 Kelvin Cross Sections

83-Bi-197



8

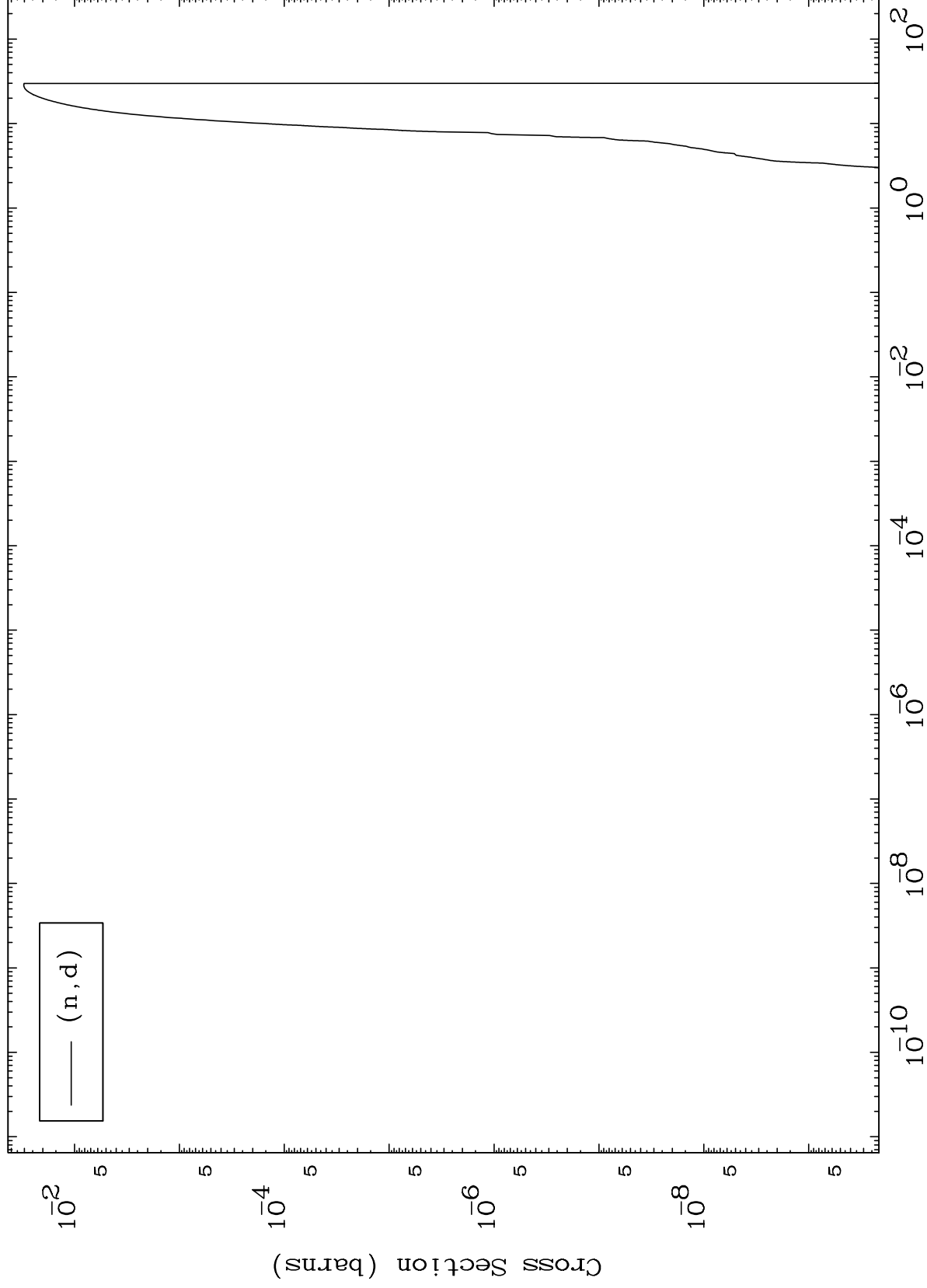
Incident Energy (MeV)

83-Bi-197

MAT 8290

(n,d) Levels
293 Kelvin Cross Sections

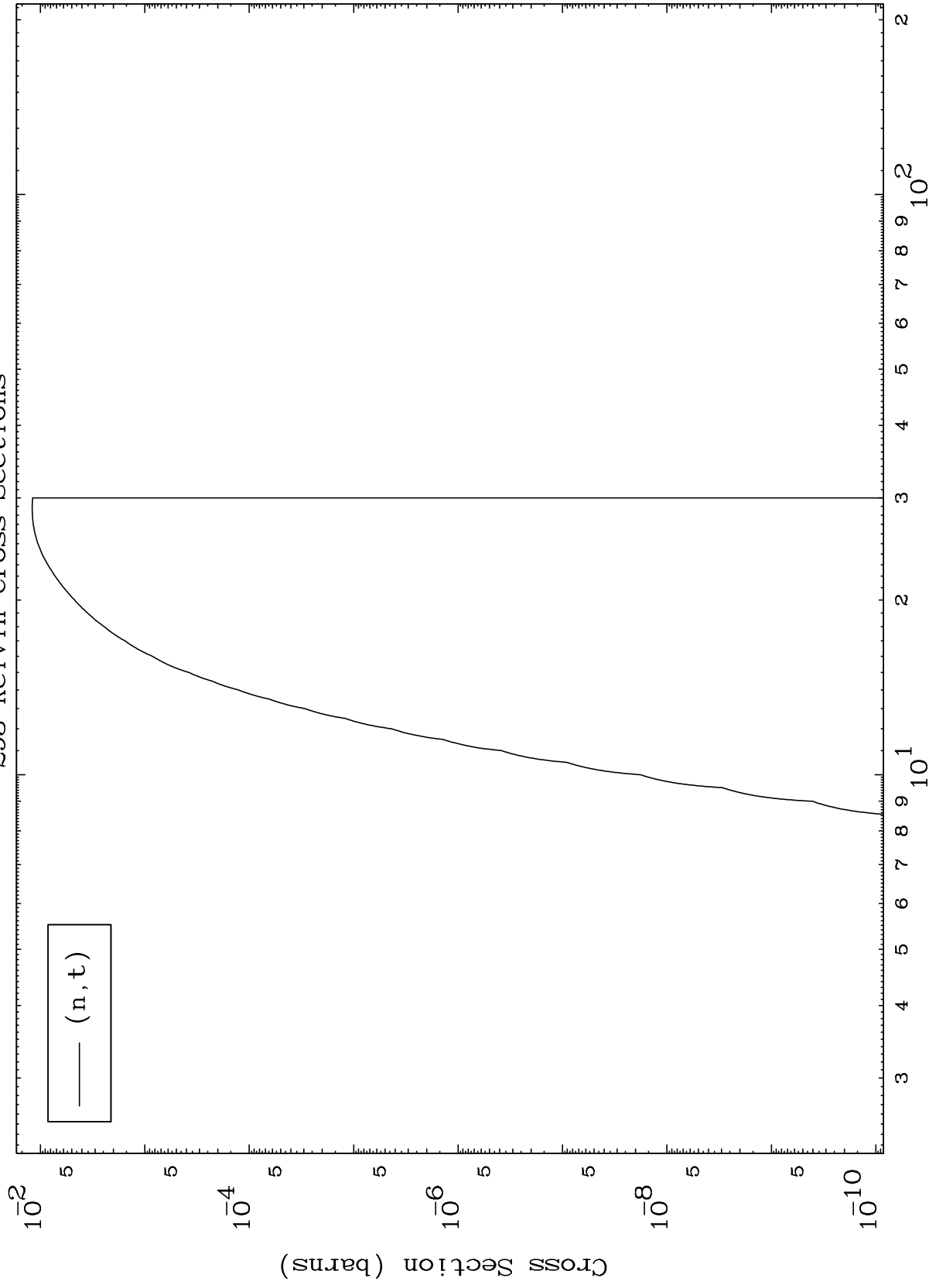
83-Bi-197



MAT 8290

(n,t) Levels
293 Kelvin Cross Sections

83-Bi-197



10

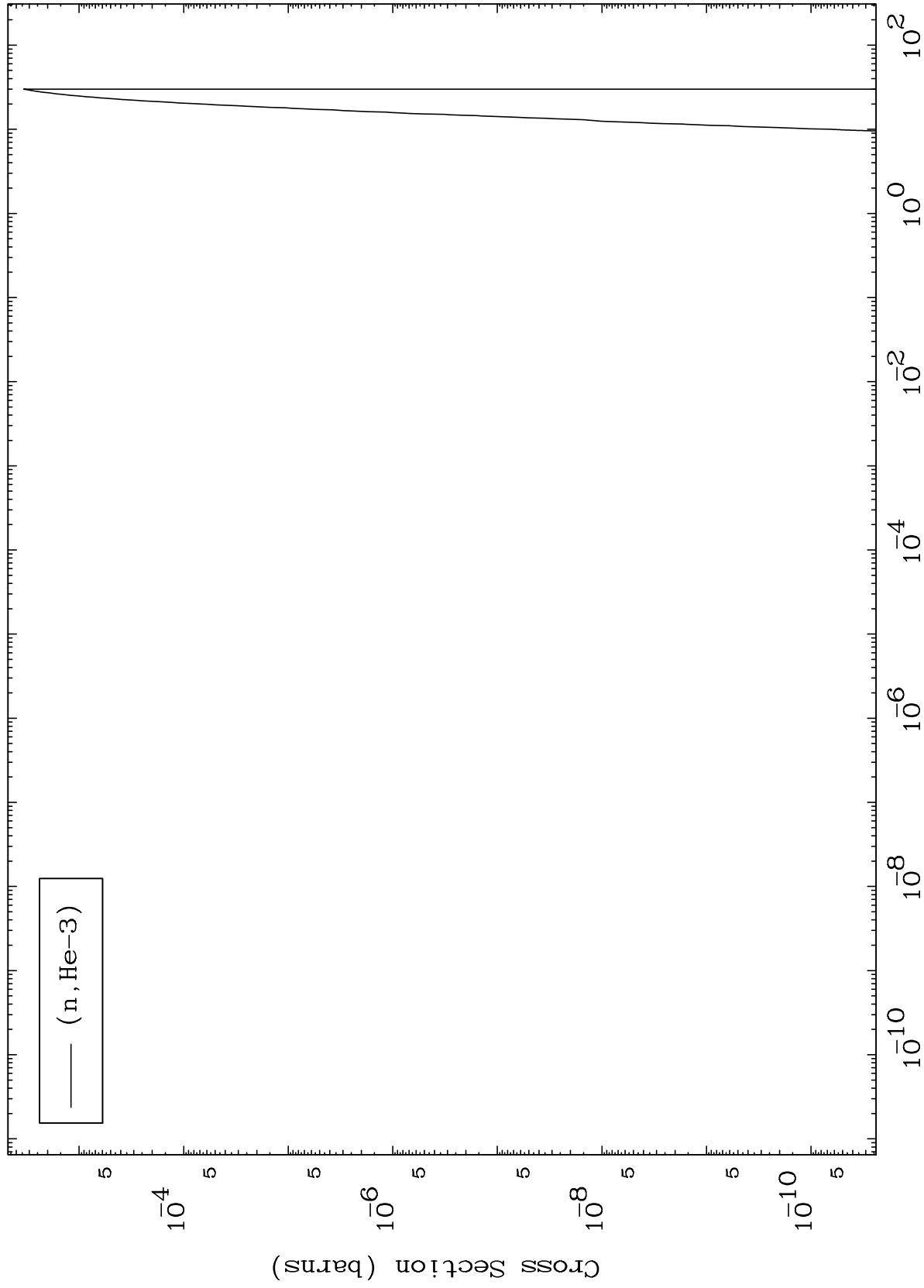
Incident Energy (MeV)

83-Bi-197

MAT 8290

(n,He3) Levels
293 Kelvin Cross Sections

83-Bi-197



11

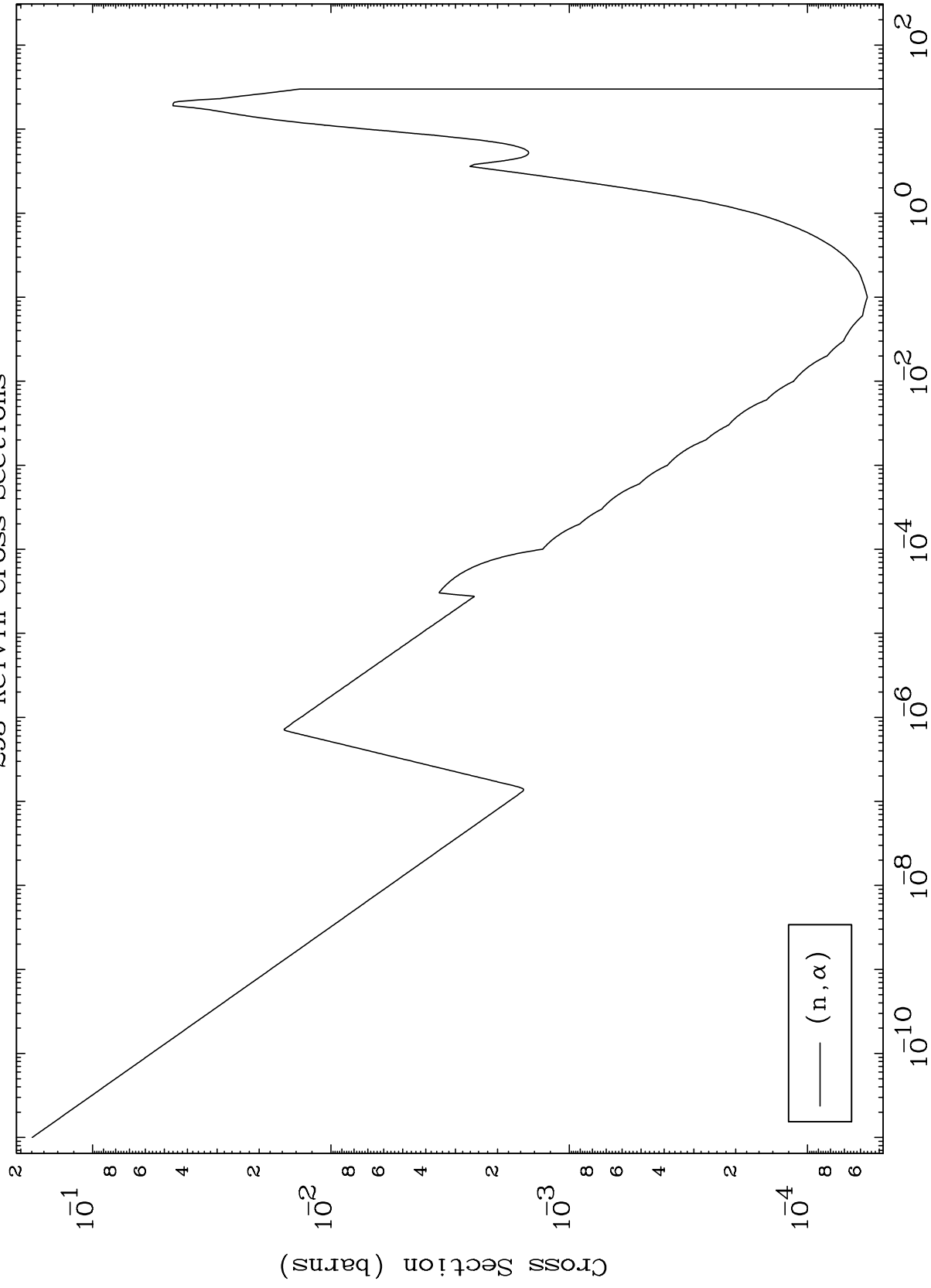
Incident Energy (MeV)

83-Bi-197

MAT 8290

(n, α) Levels
293 Kelvin Cross Sections

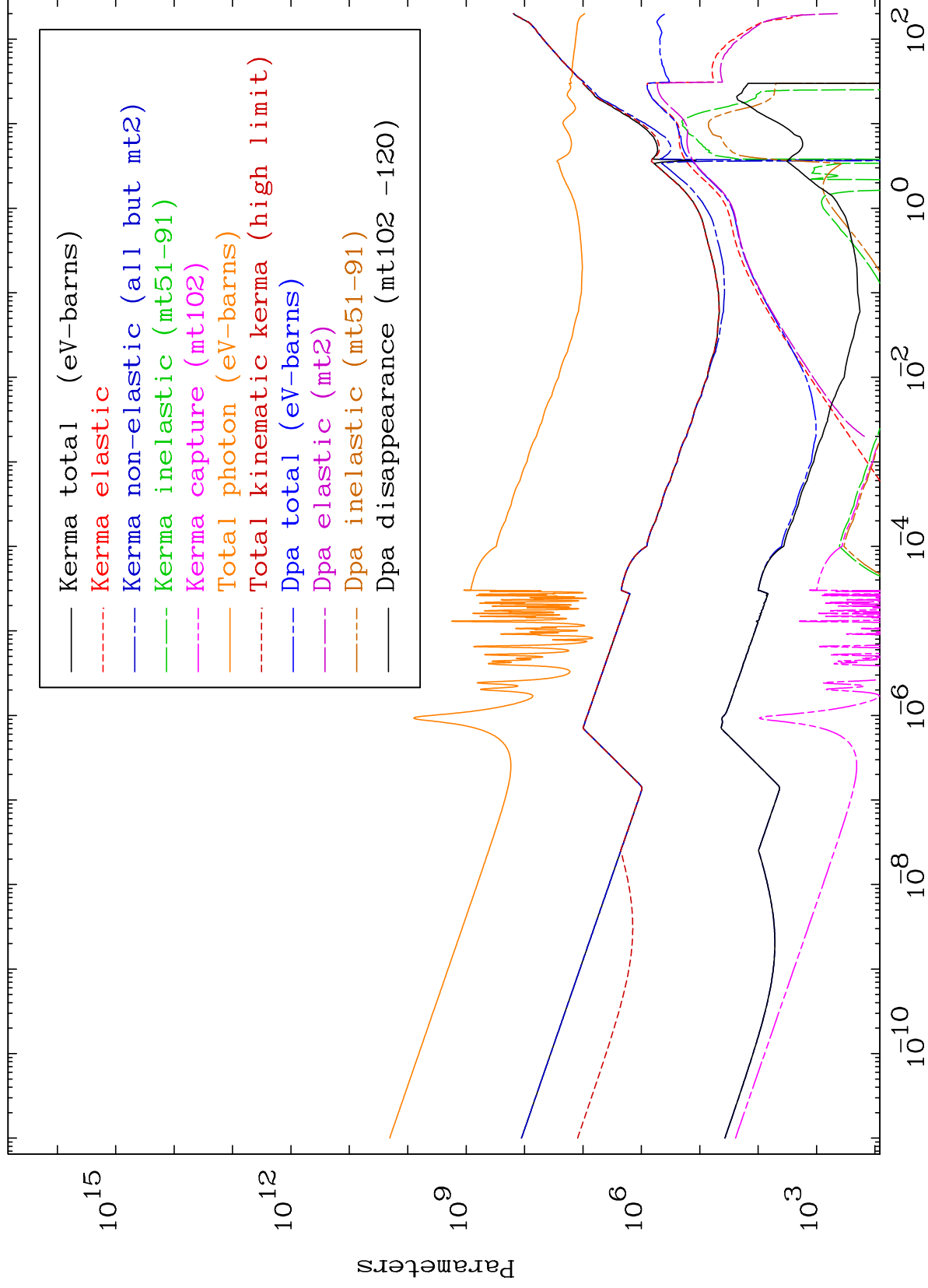
83-Bi-197



12

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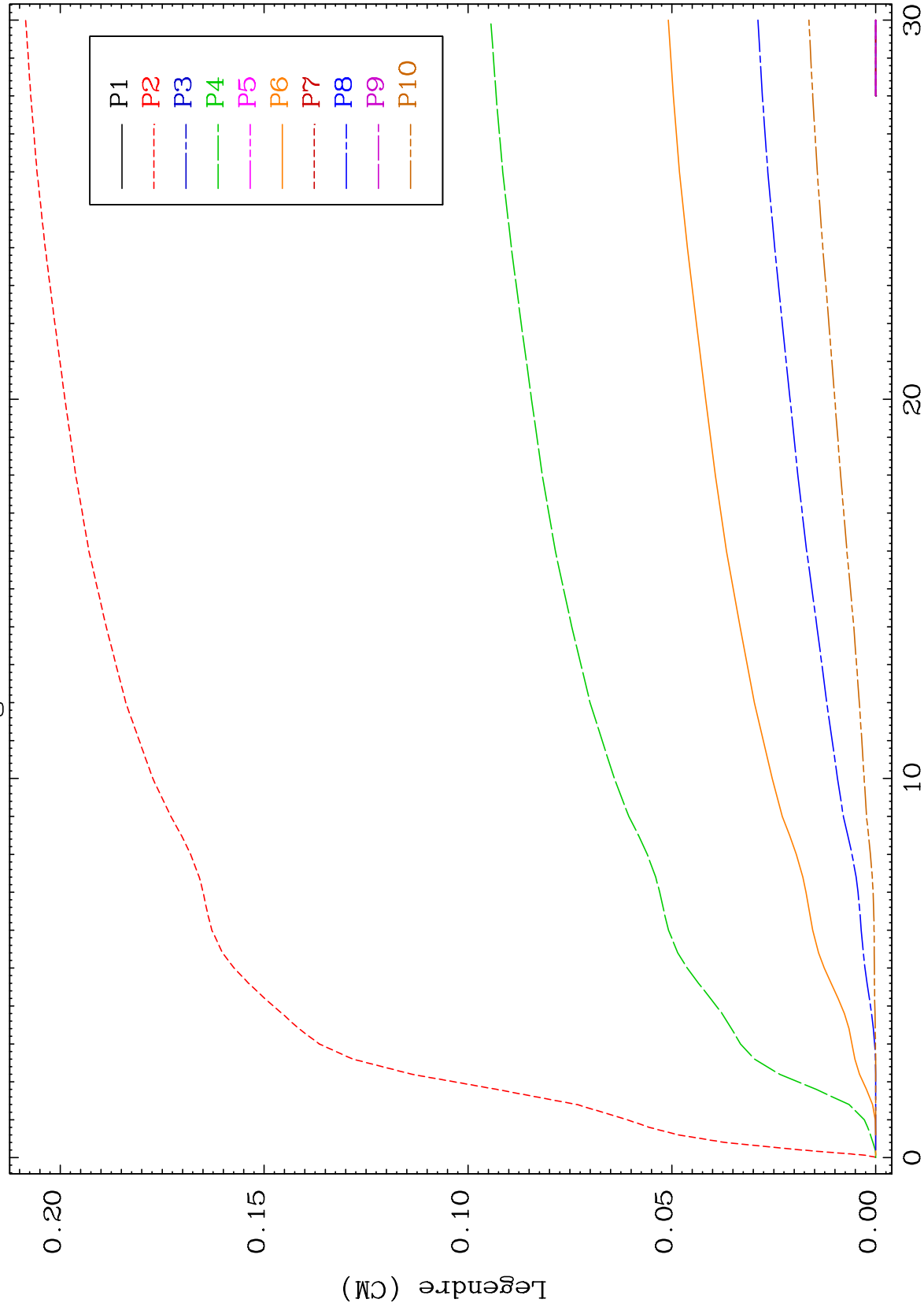
83-Bi-197



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83-Bi-197

Elastic Legendre Coefficients



14

Incident Energy (MeV)

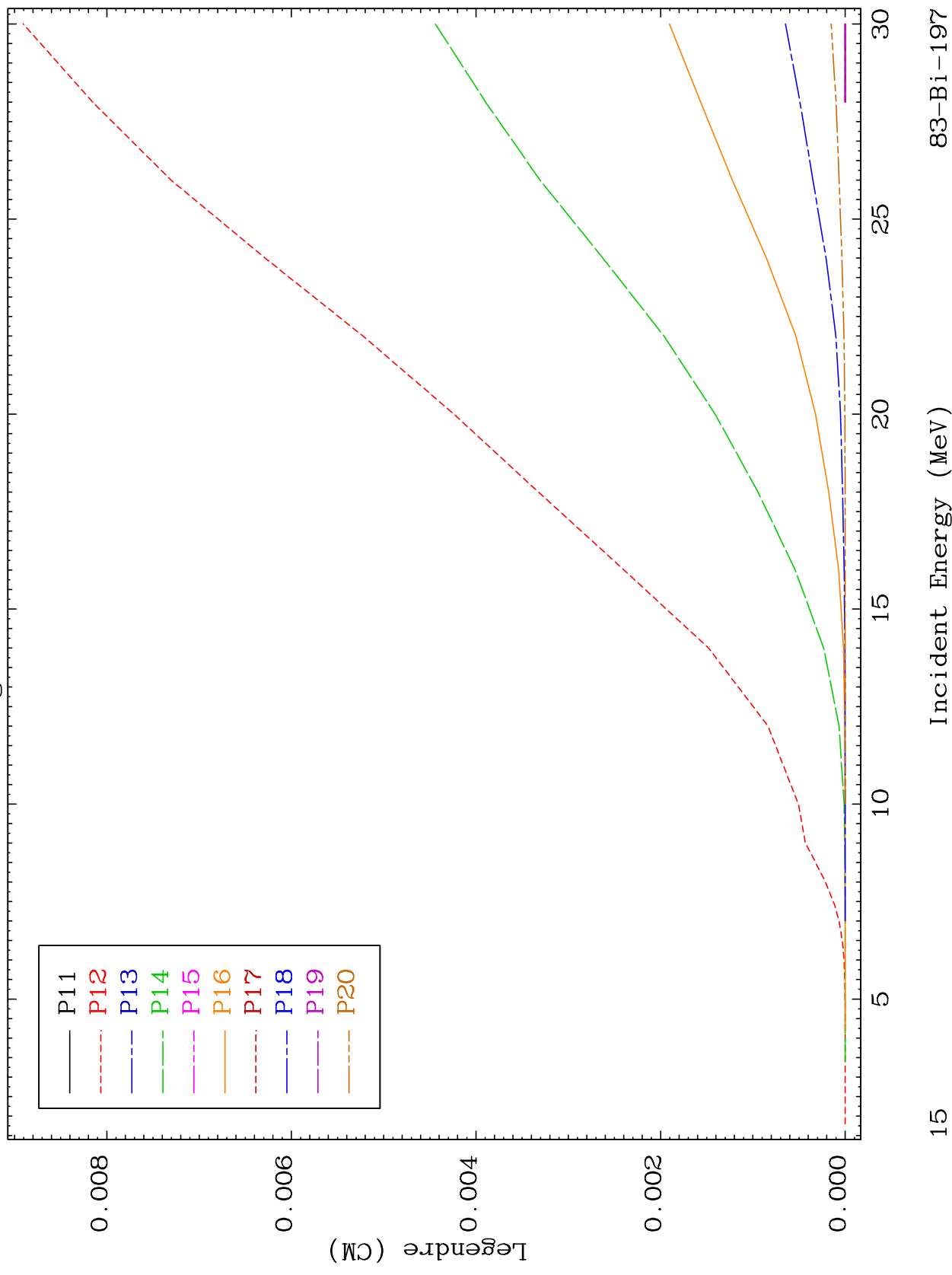
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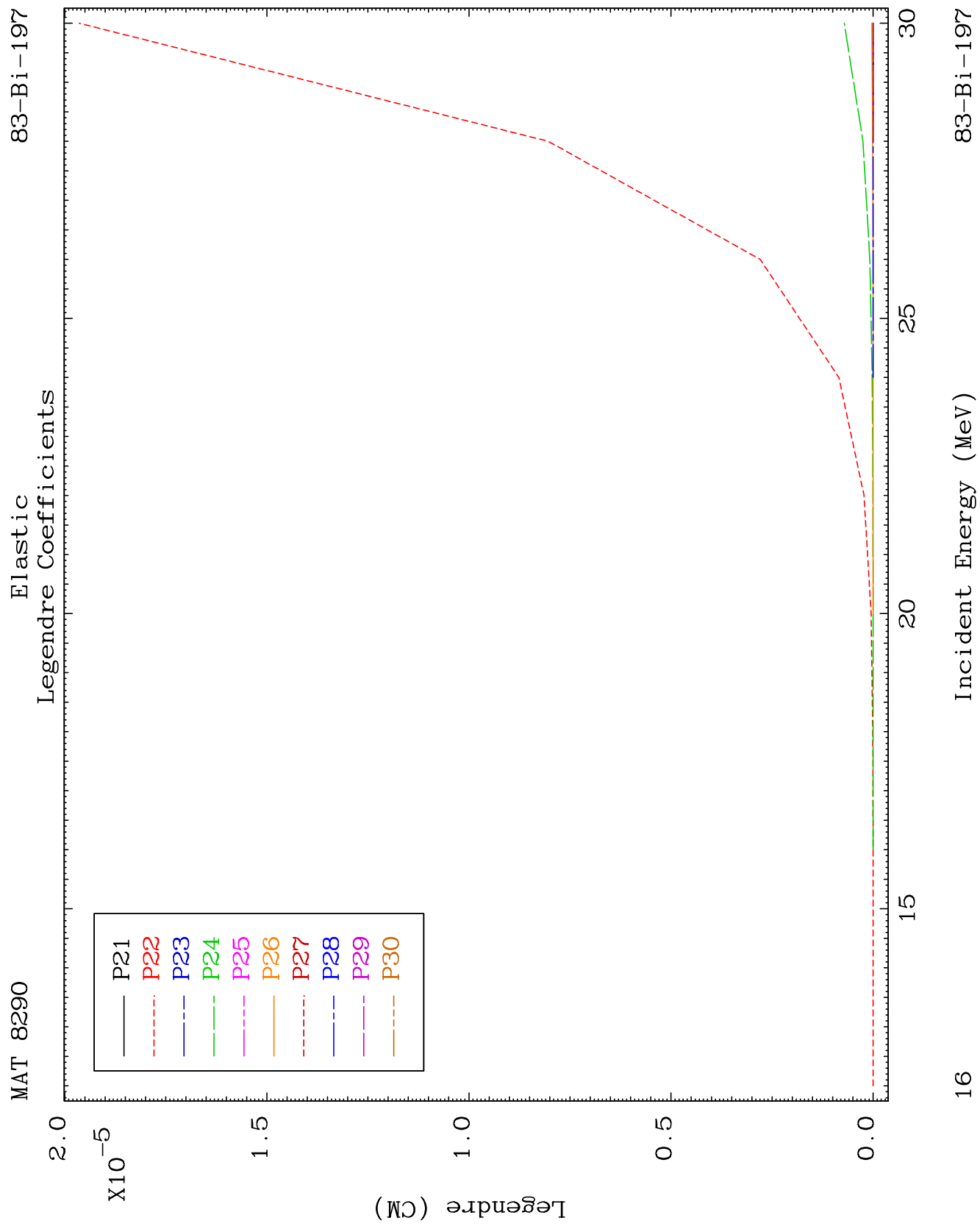
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Elastic

83-Bi-197

Legendre Coefficients

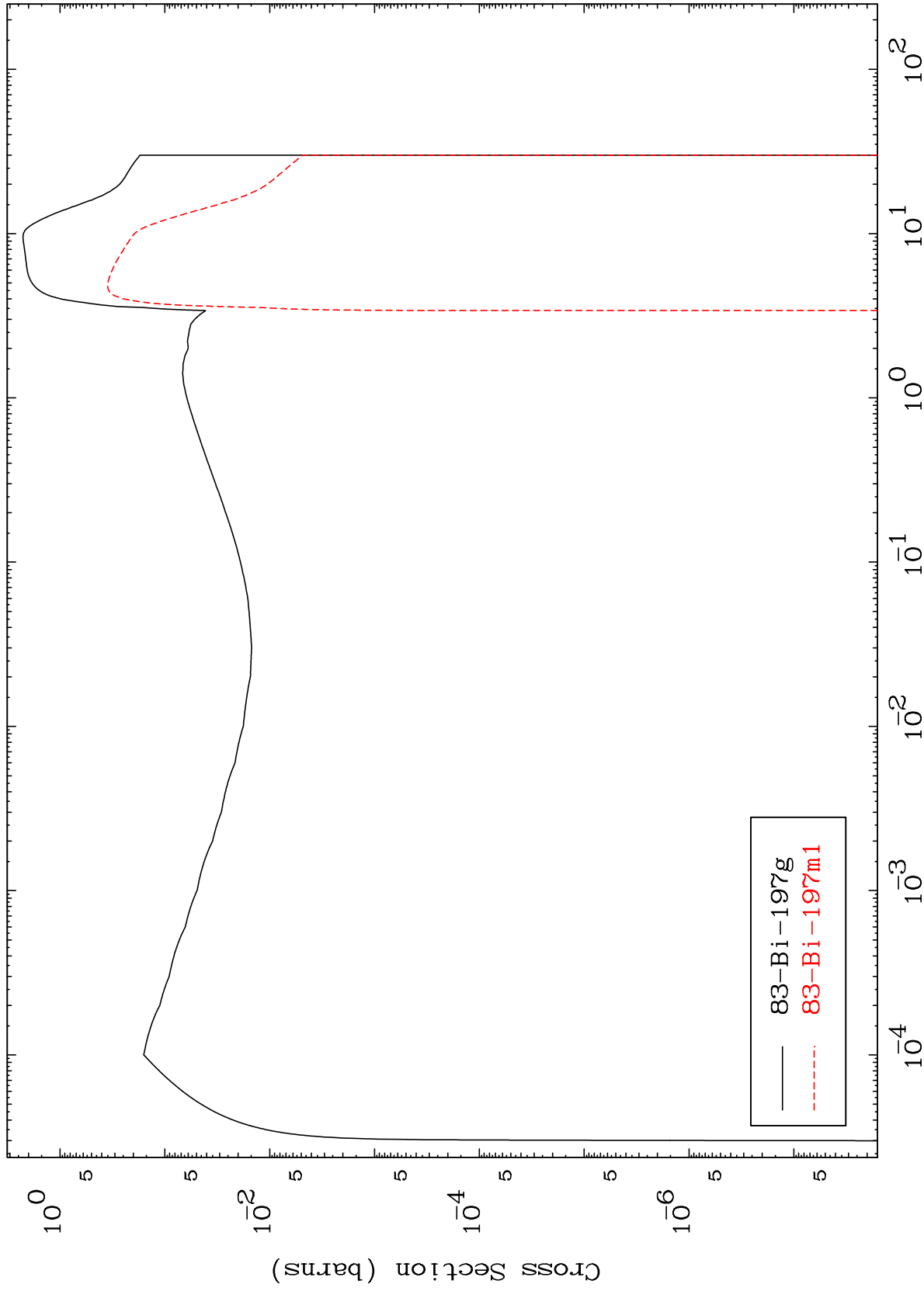




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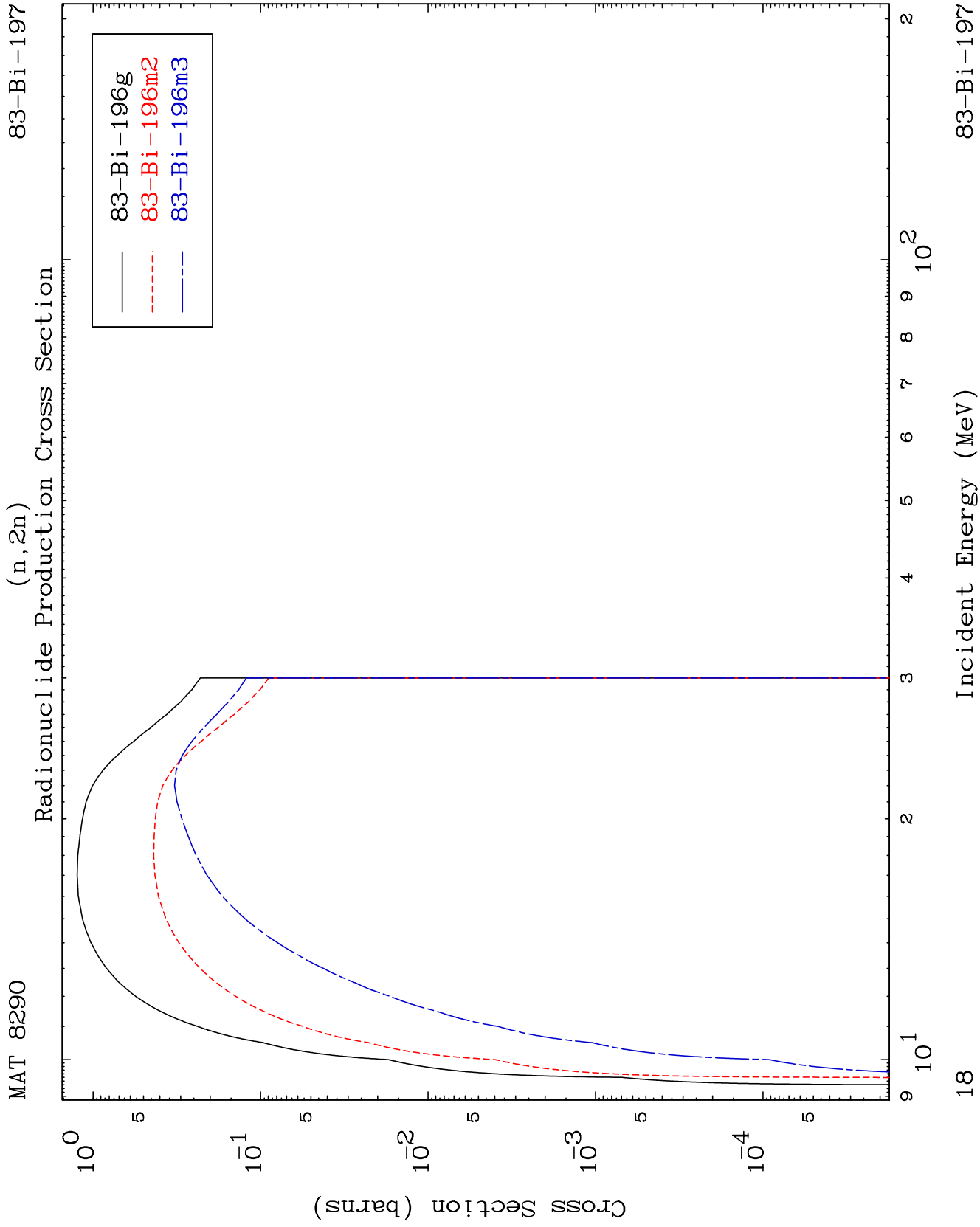
83-Bi-197

Inelastic
Radionuclide Production Cross Section



17

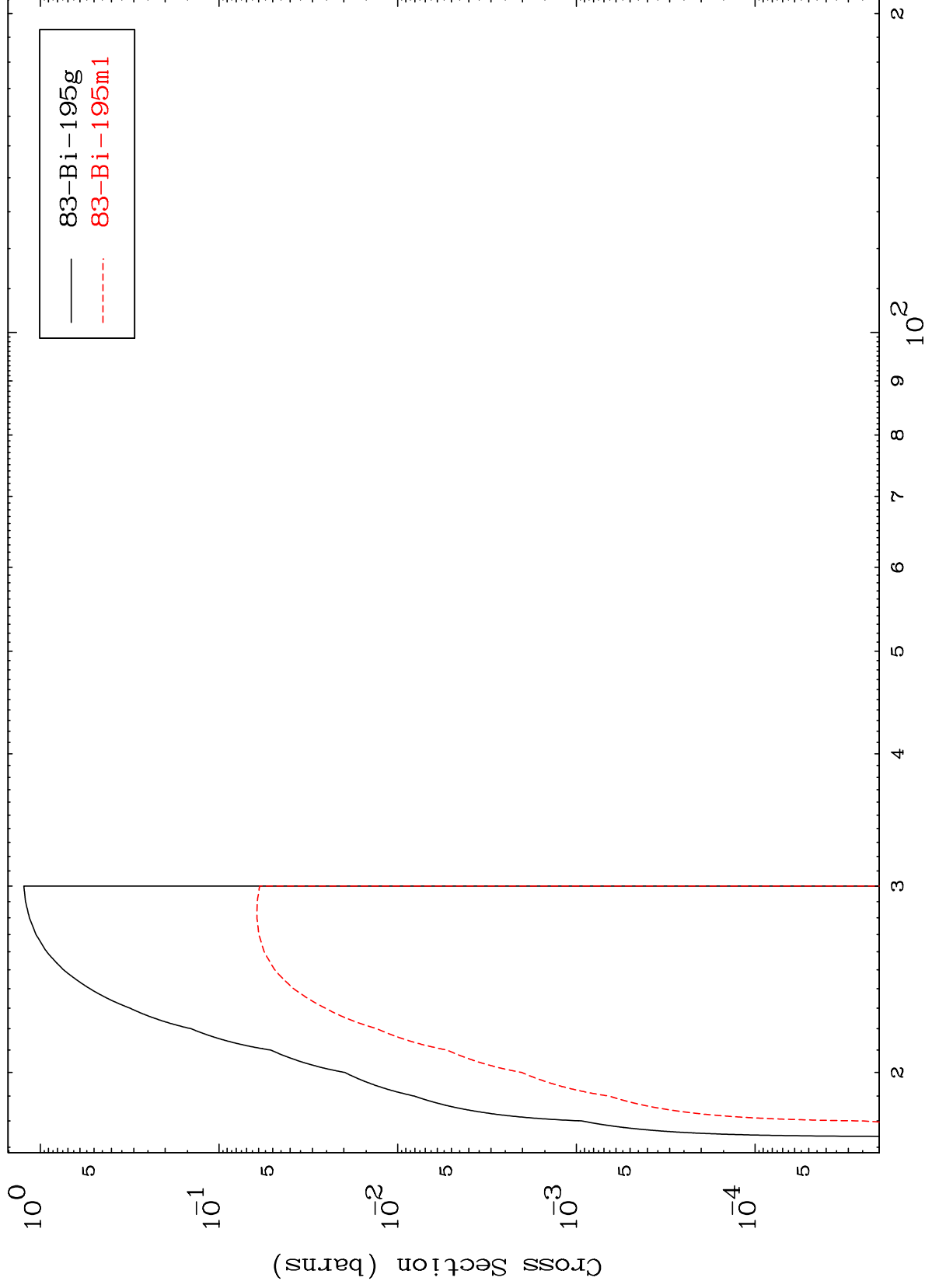
83-Bi-197



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83-Bi-197

(n,3n)
Radionuclide Production Cross Section



19

Incident Energy (MeV)

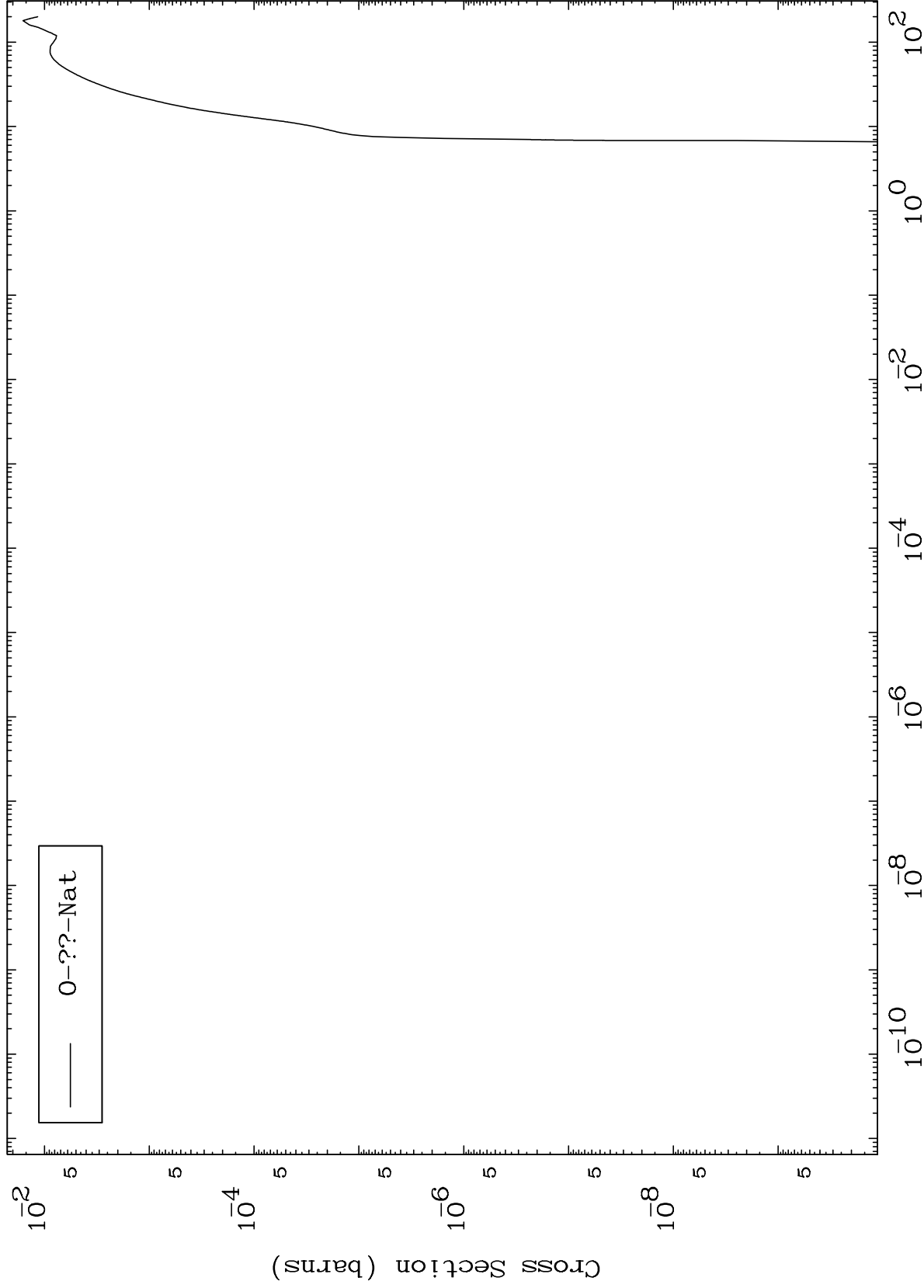
83-Bi-197

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Fission

83-Bi-197

Radionuclide Production Cross Section



0-??-Nat

20

Incident Energy (MeV)

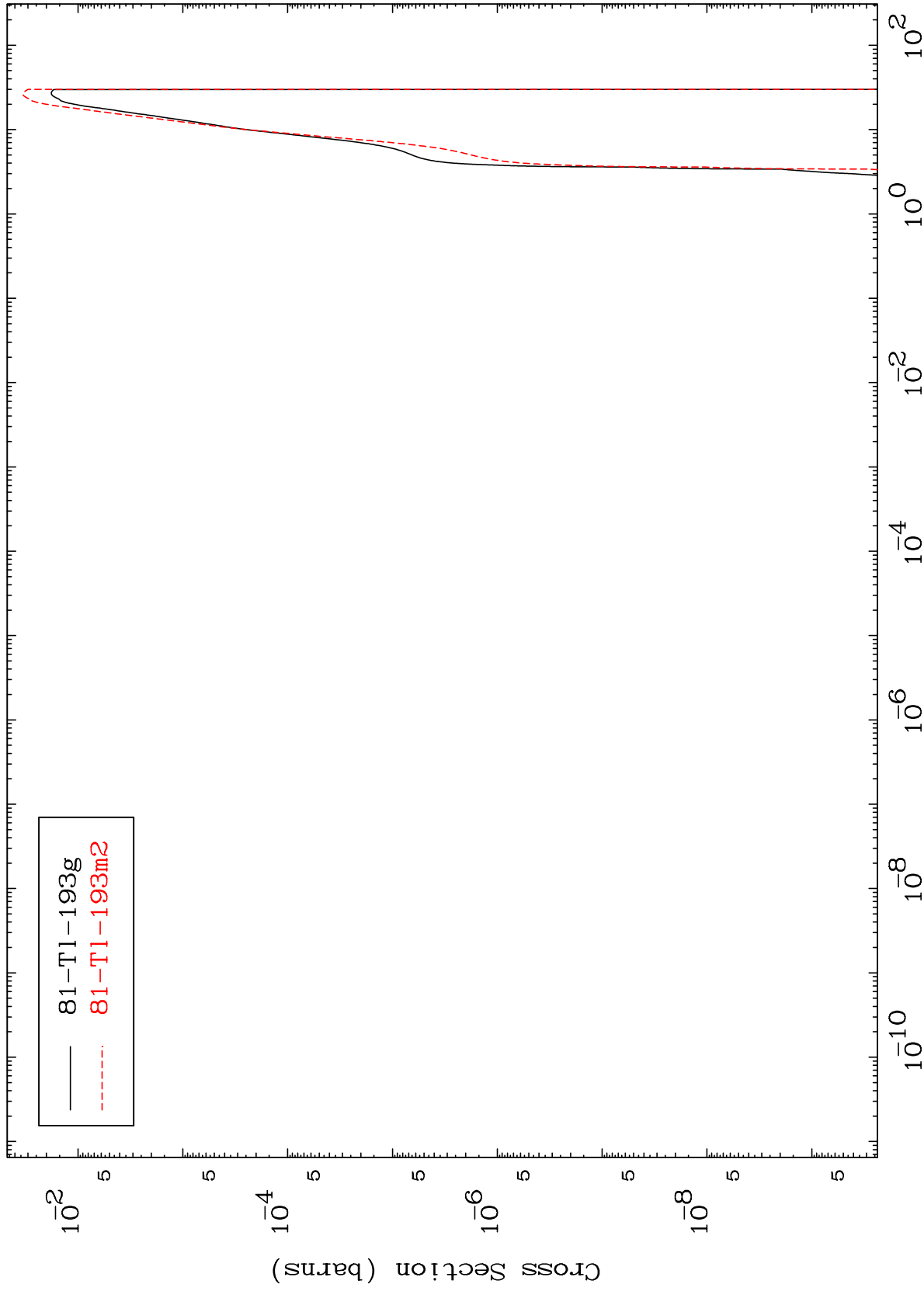
83-Bi-197

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$(n,n') \alpha$

83-Bi-197

Radionuclide Production Cross Section



21

Incident Energy (MeV)

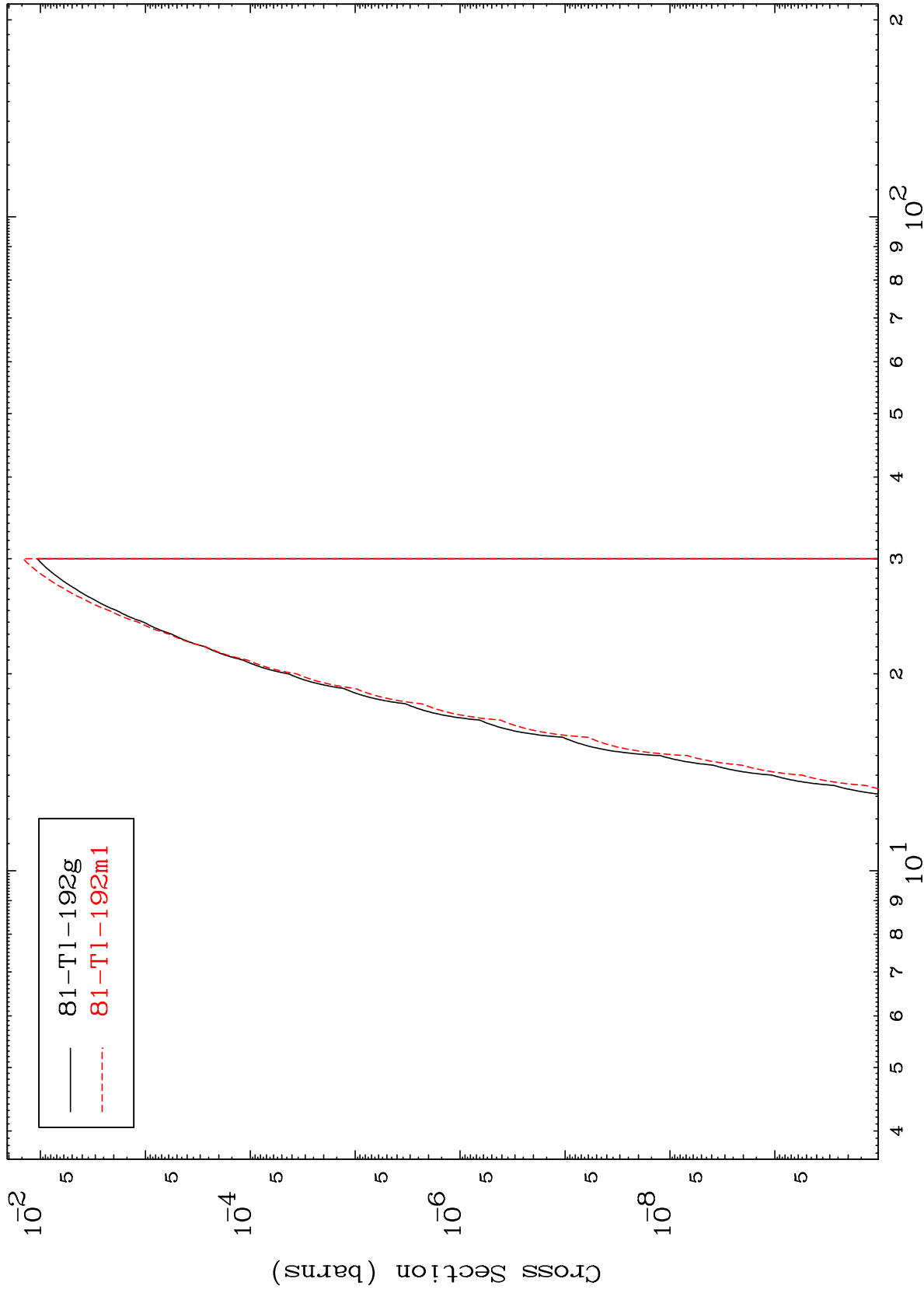
83-Bi-197

MAT 8290

$(n,2n) \alpha$

83-Bi-197

Radionuclide Production Cross Section

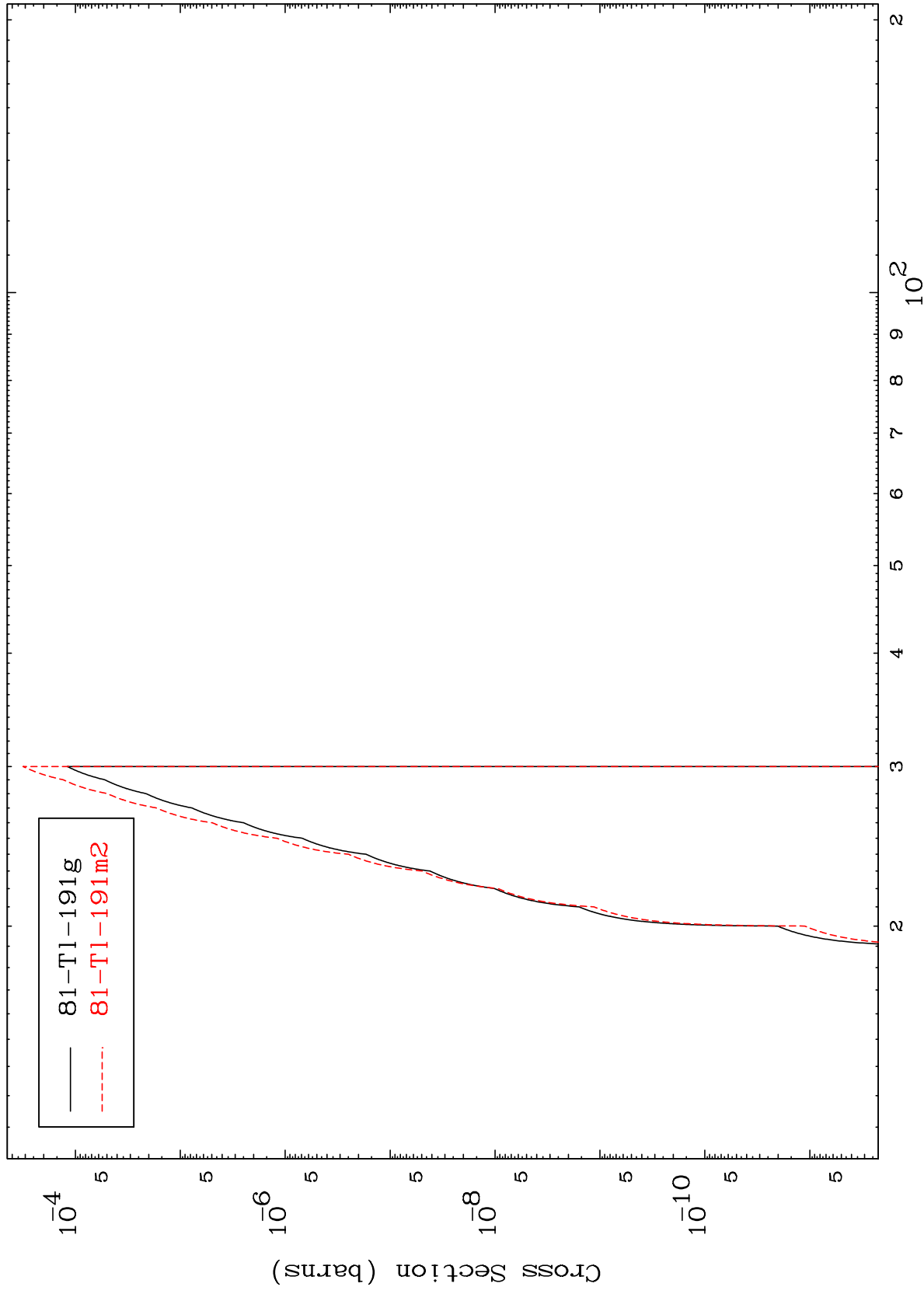


22

Incident Energy (MeV)

83-Bi-197

Radionuclide Production Cross Section

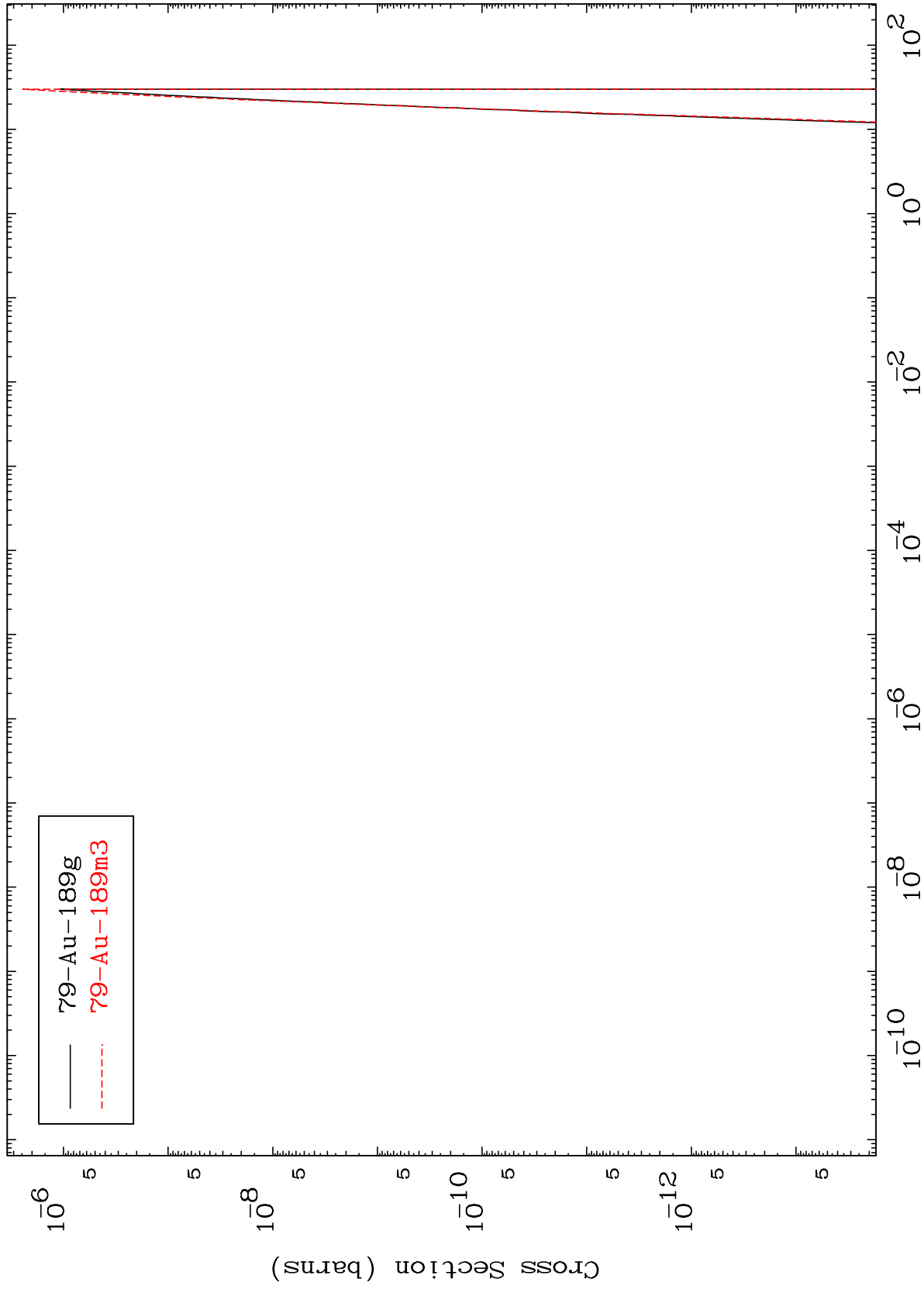


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(n,n') 2 α

83-Bi-197

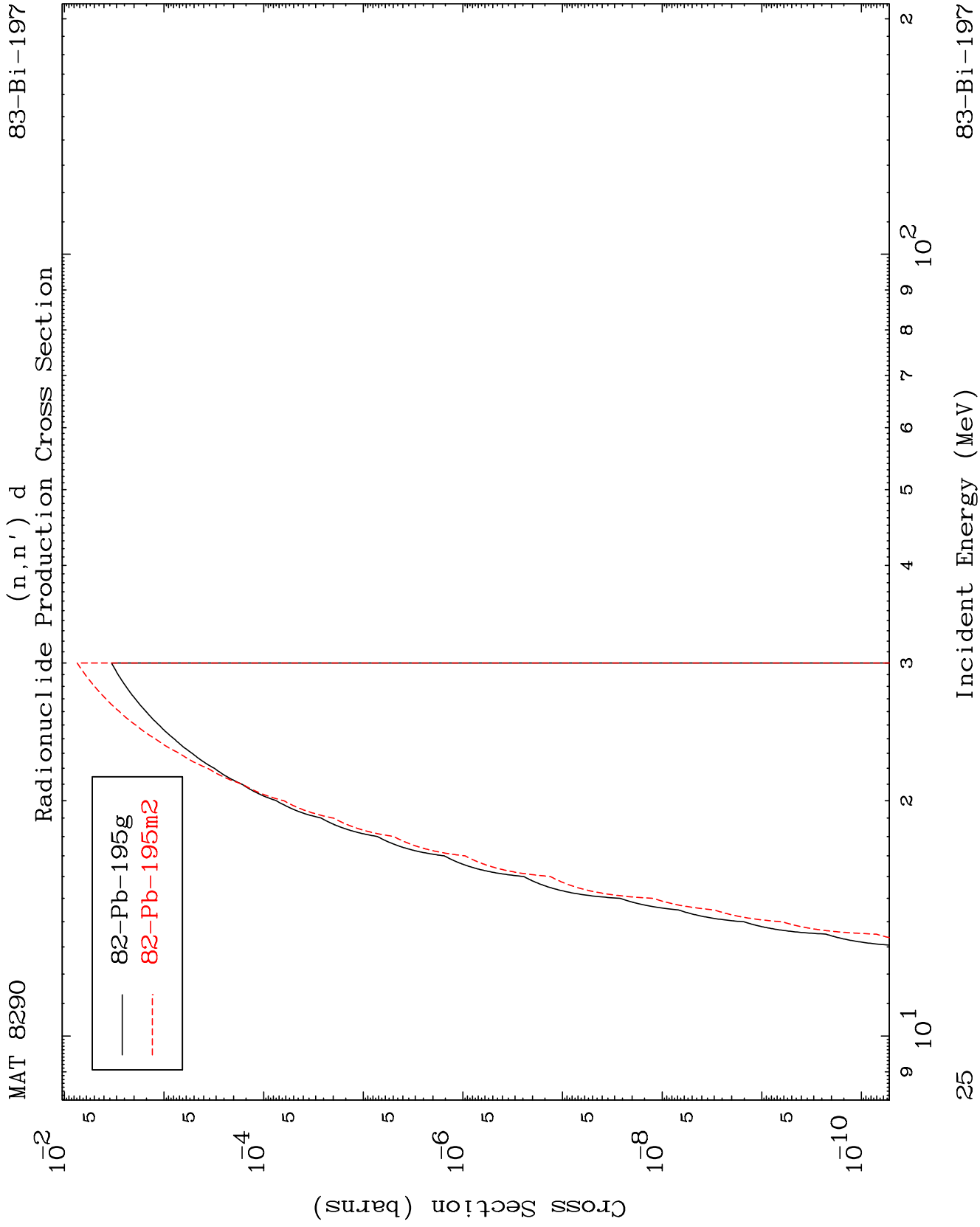
Radionuclide Production Cross Section



24

Incident Energy (MeV)

83-Bi-197

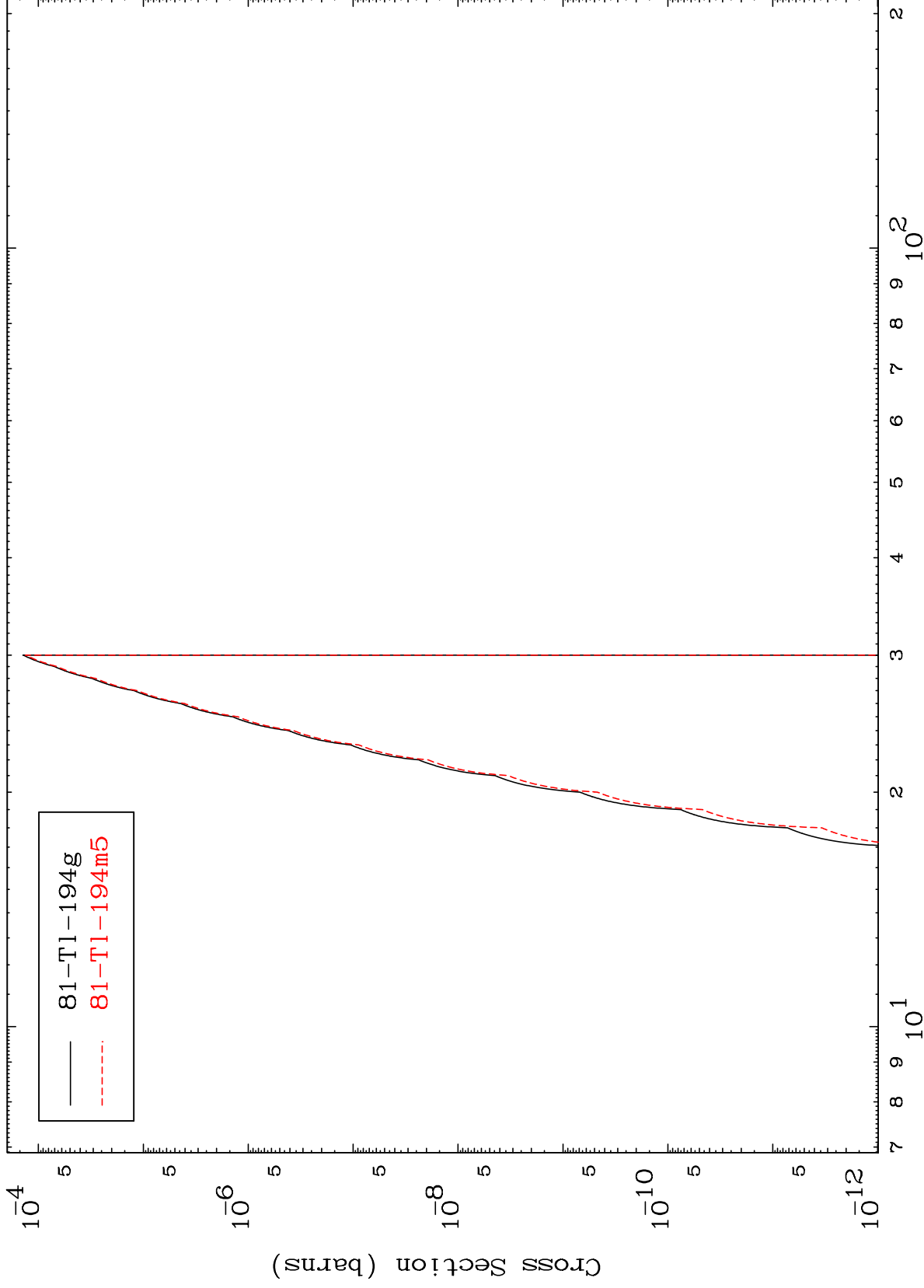


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(n,n') He-3

83-Bi-197

Radionuclide Production Cross Section



26

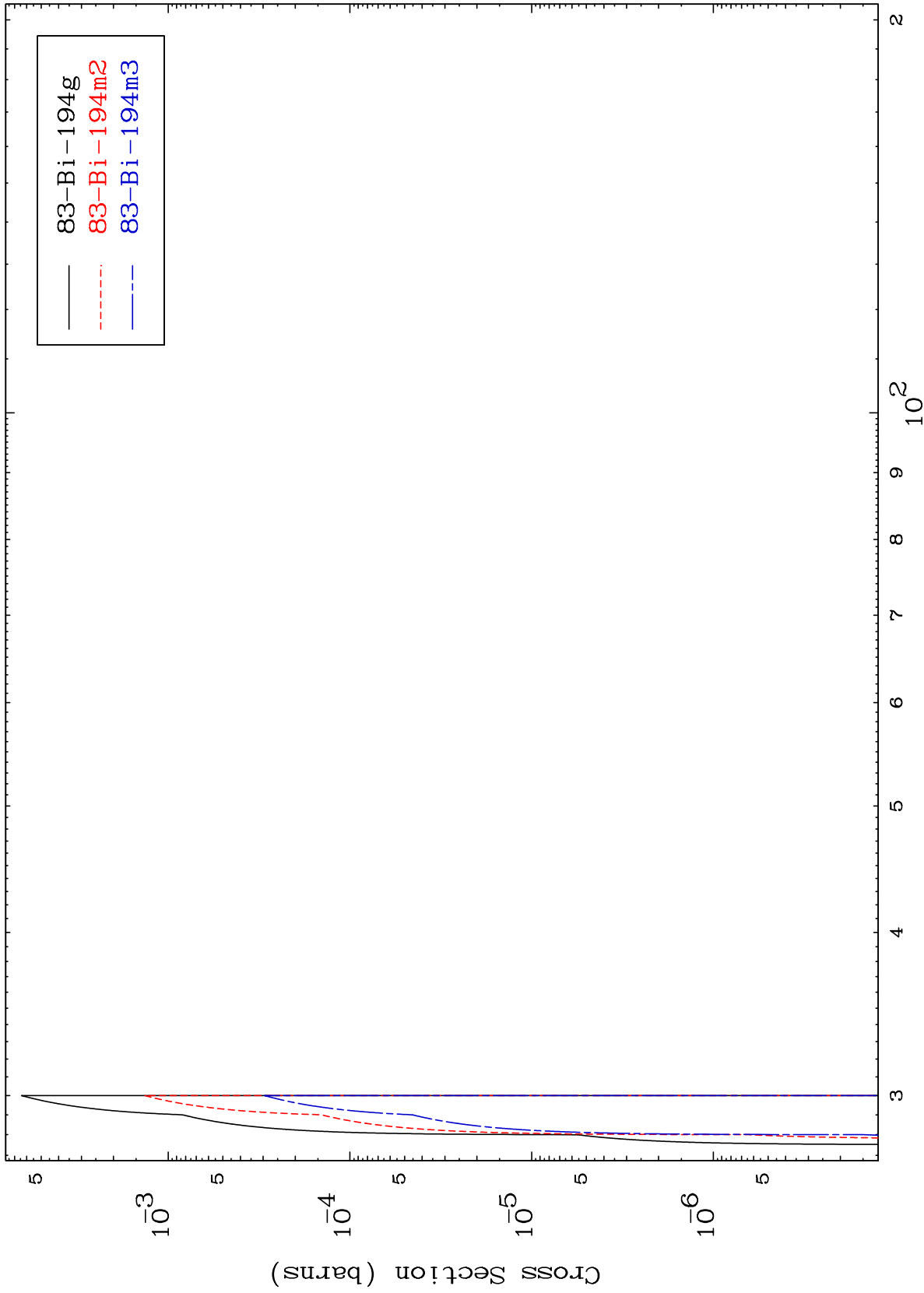
Incident Energy (MeV)

83-Bi-197

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$^{83}\text{Bi}-197$

$(n,4n)$
Radionuclide Production Cross Section

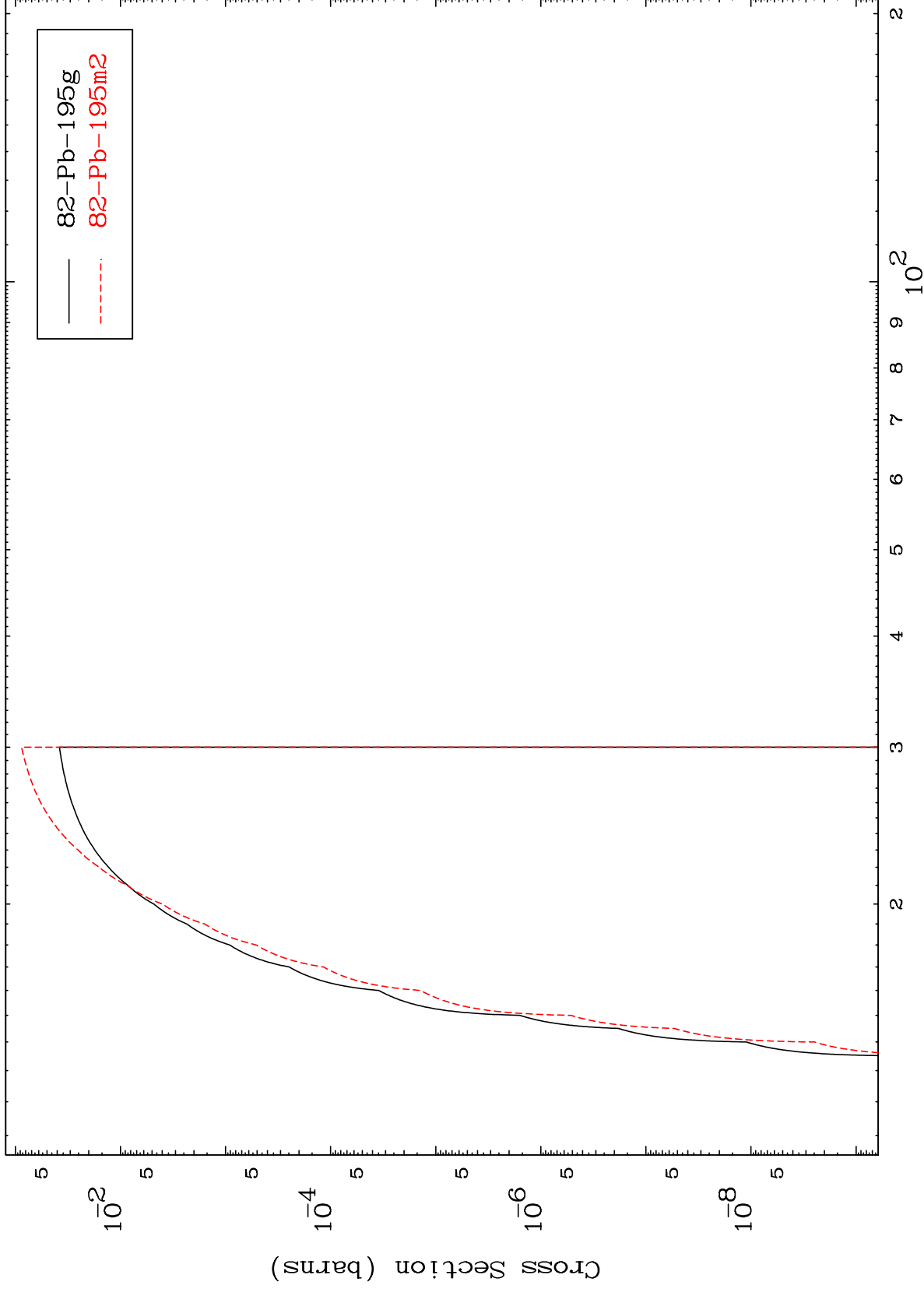


27

Incident Energy (MeV)

$^{83}\text{Bi}-197$

Radionuclide Production Cross Section

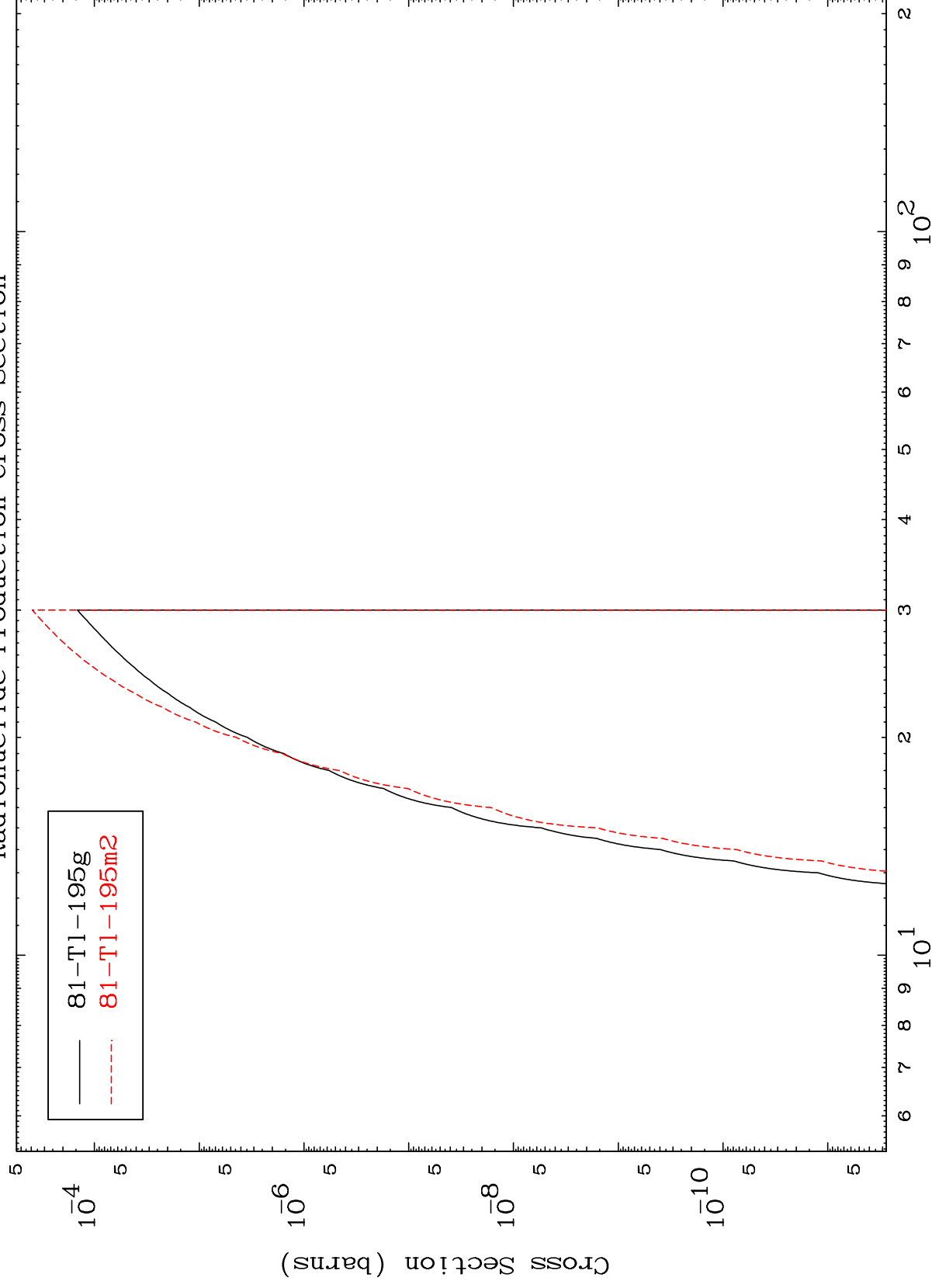


MAT 8290

(n,2n) p

83-Bi-197

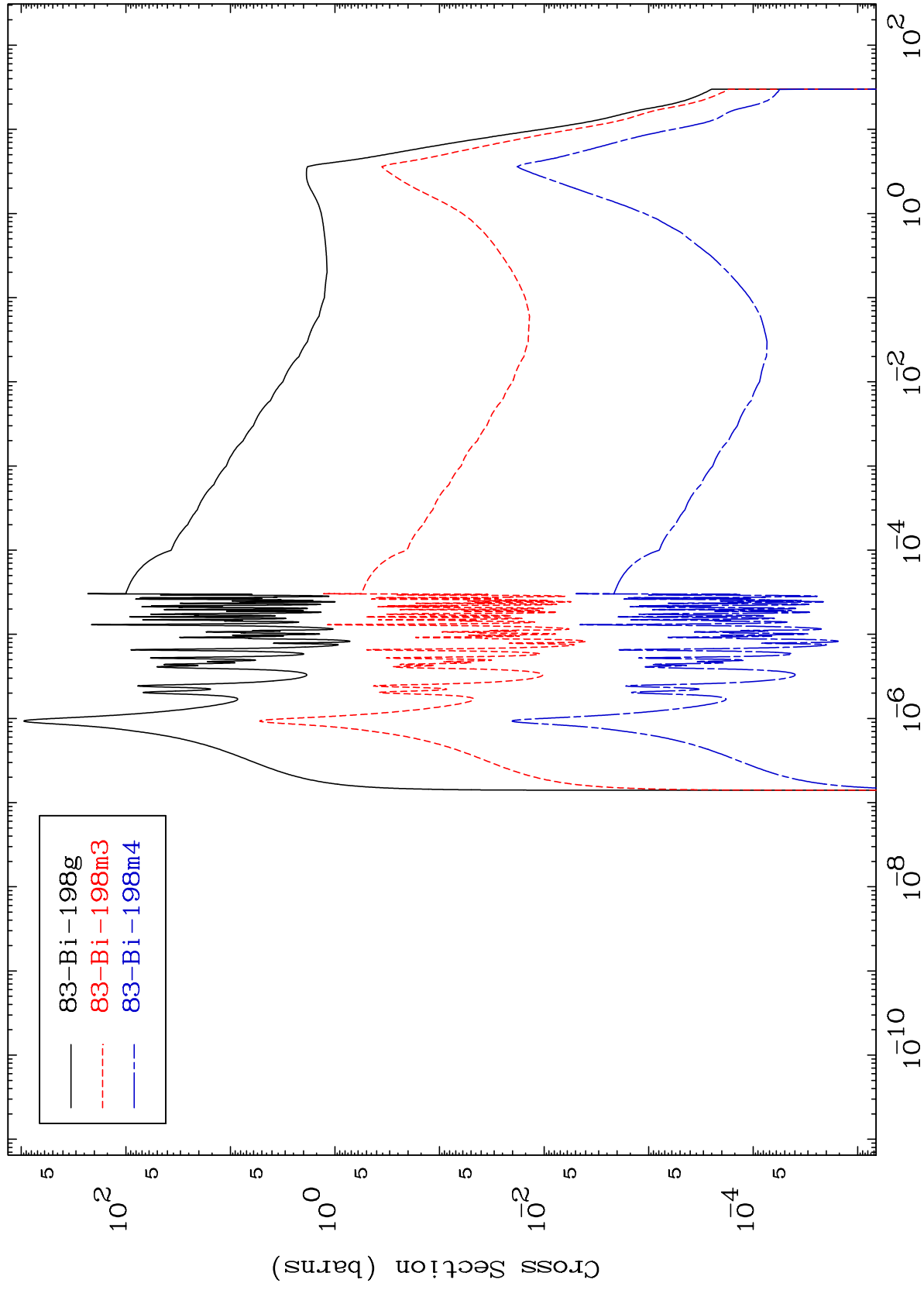
Radionuclide Production Cross Section



29

Incident Energy (MeV)

83-Bi-197

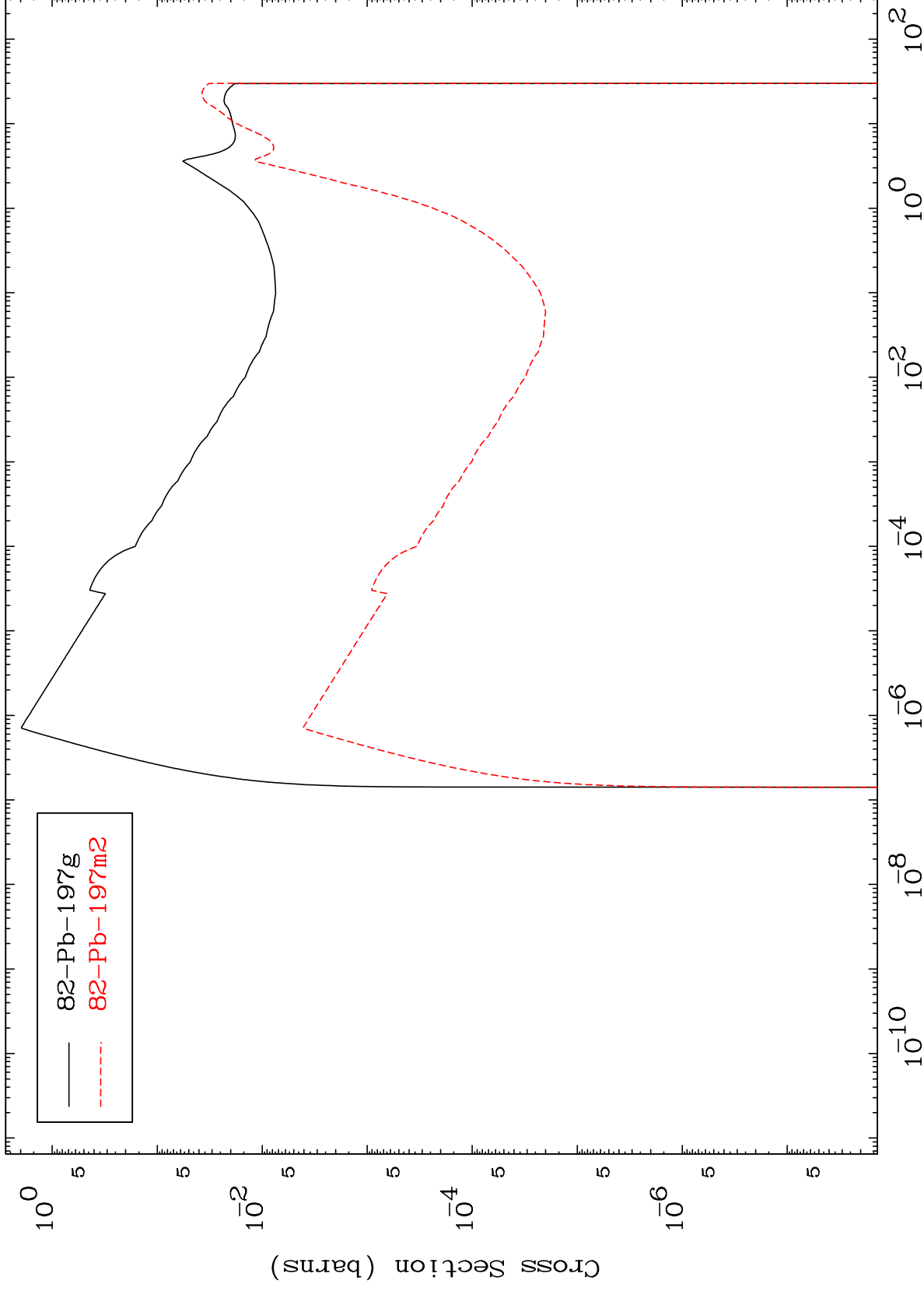
(n, γ)
Radionuclide Production Cross Section

MAT 8290

(n,p)

83-Bi-197

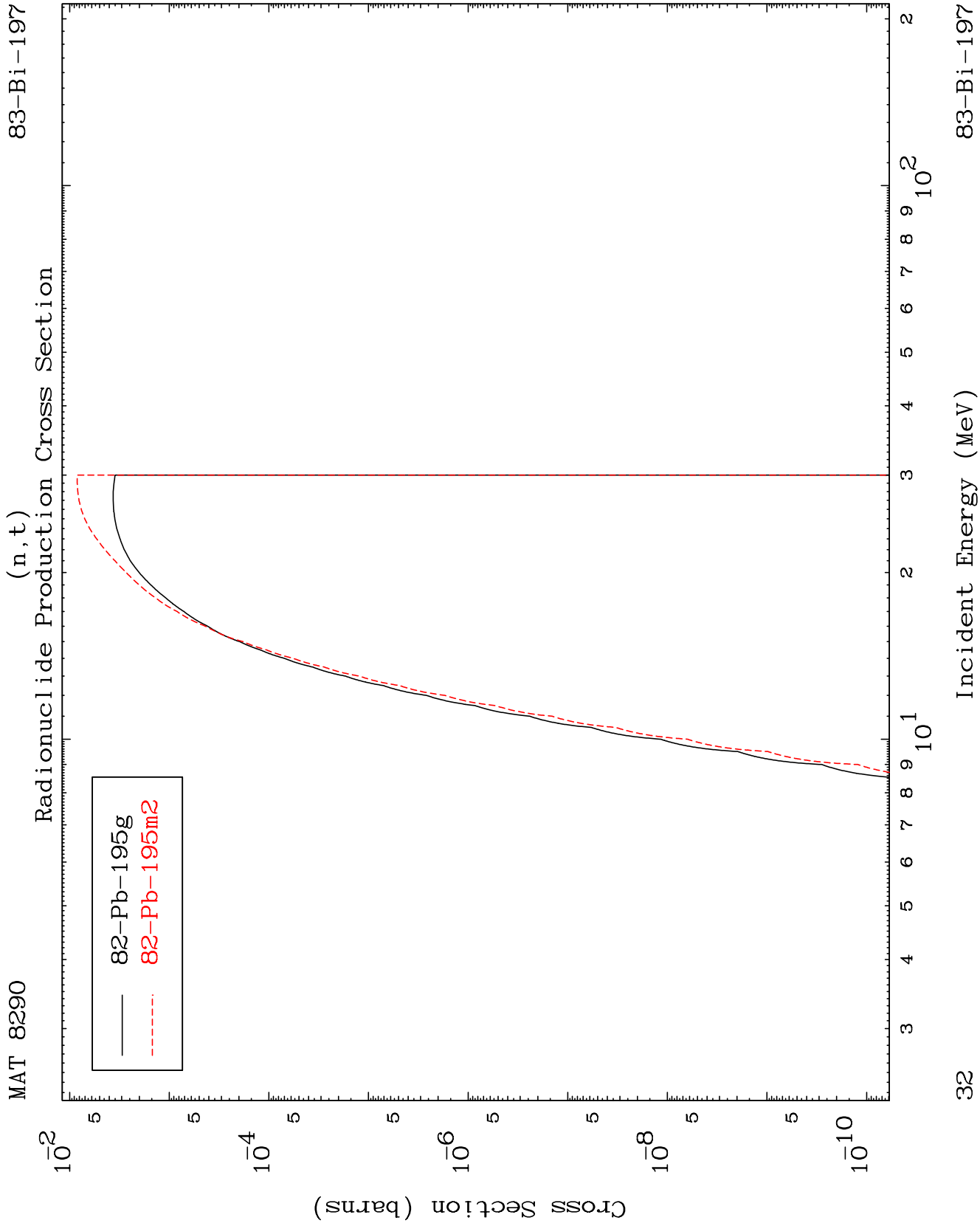
Radionuclide Production Cross Section



31

Incident Energy (MeV)

83-Bi-197

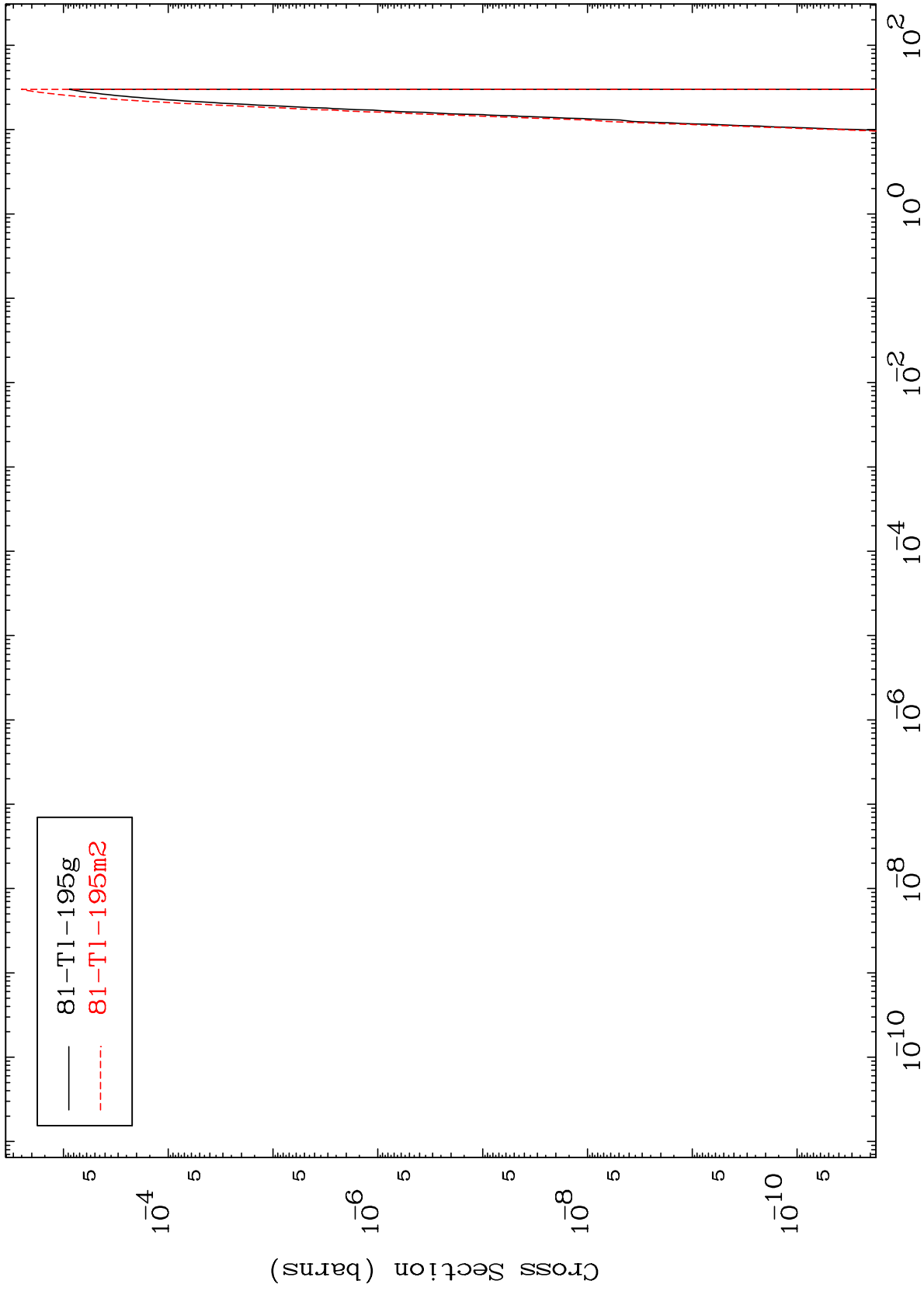


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(n,He-3)

83-Bi-197

Radionuclide Production Cross Section



33

Incident Energy (MeV)

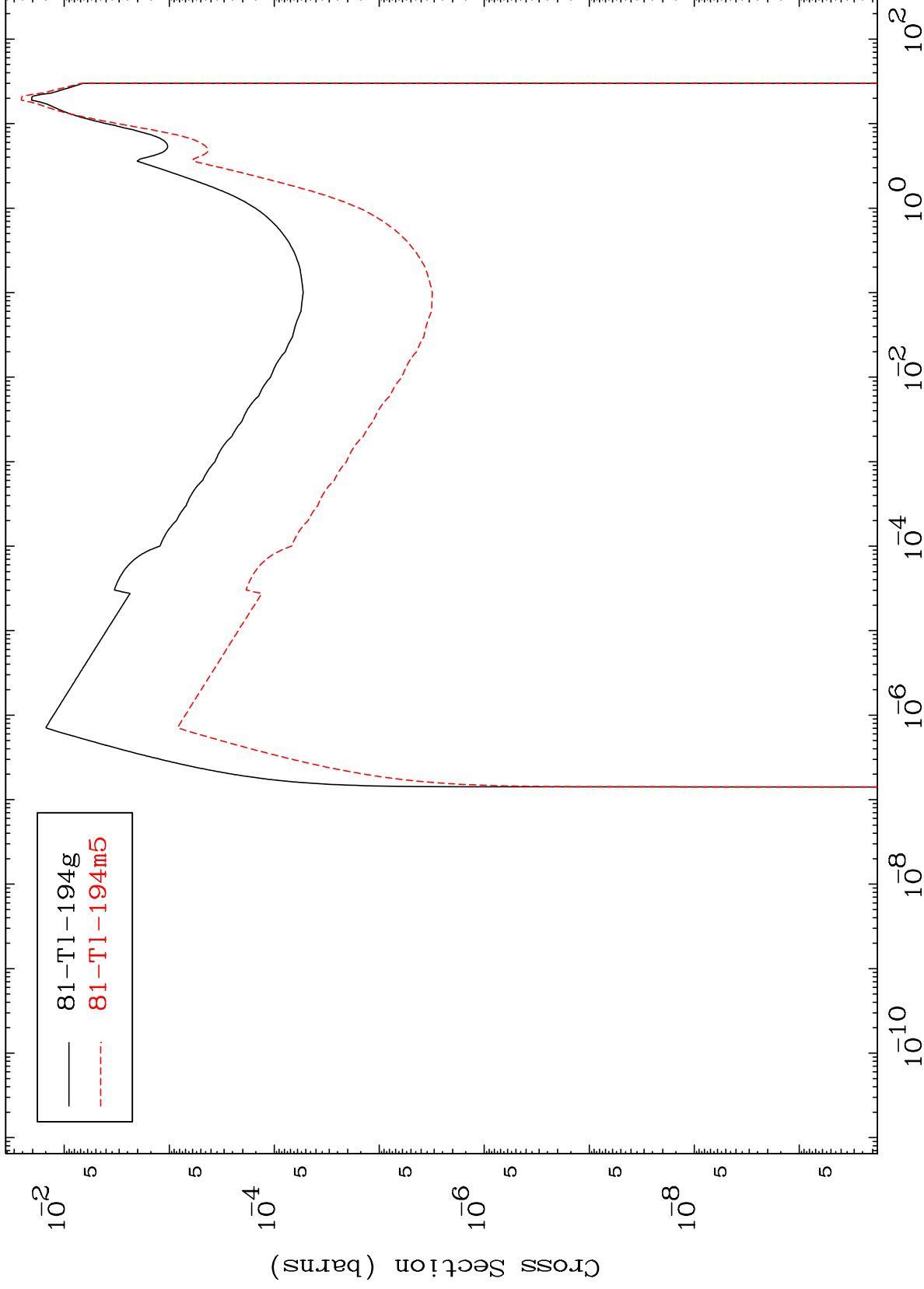
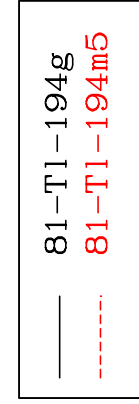
83-Bi-197

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(n, α)

83-Bi-197

Radionuclide Production Cross Section



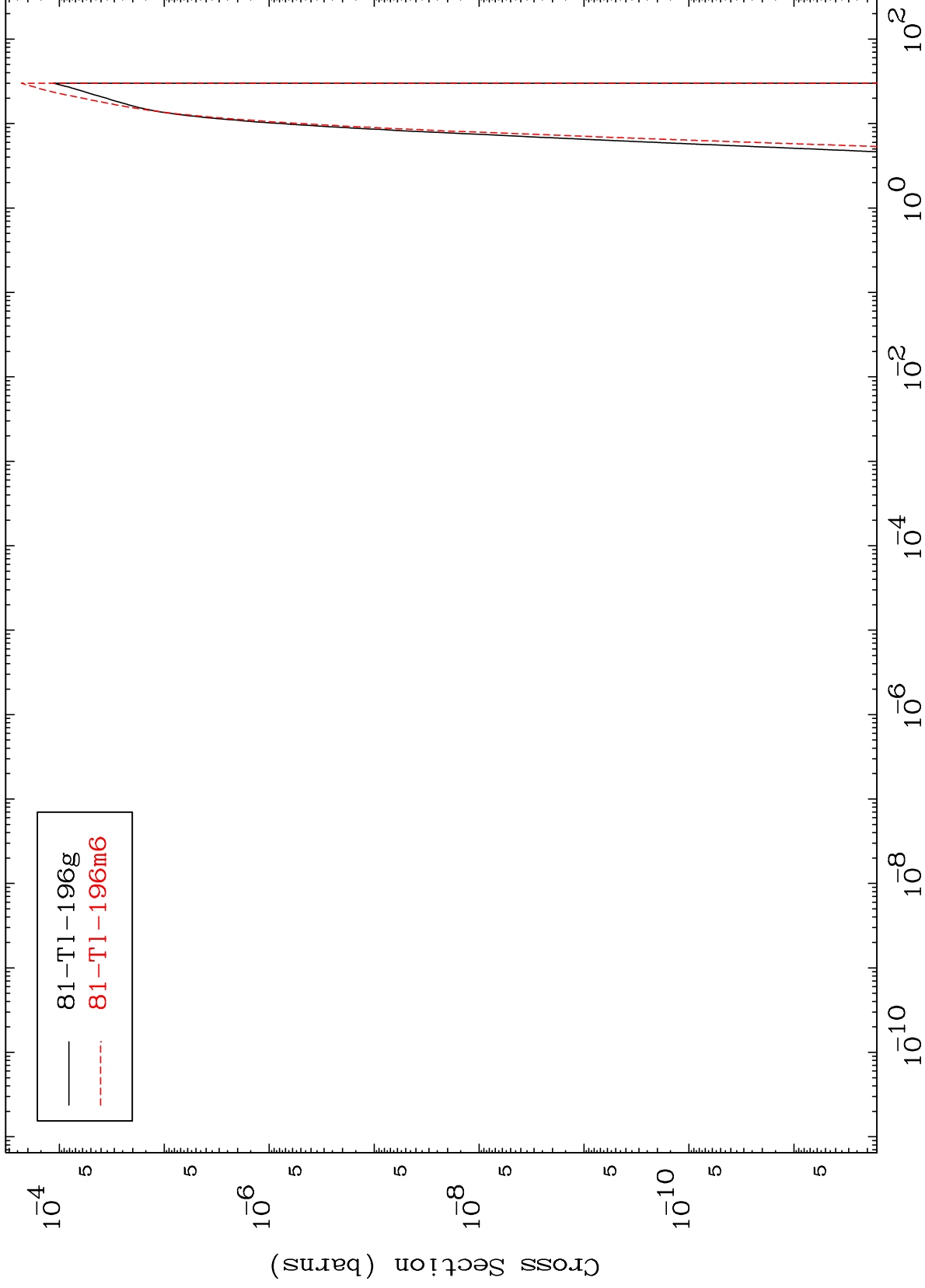
34

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(n,2p)

83-Bi-197

Radionuclide Production Cross Section



35

Incident Energy (MeV)

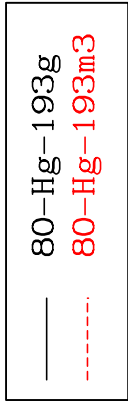
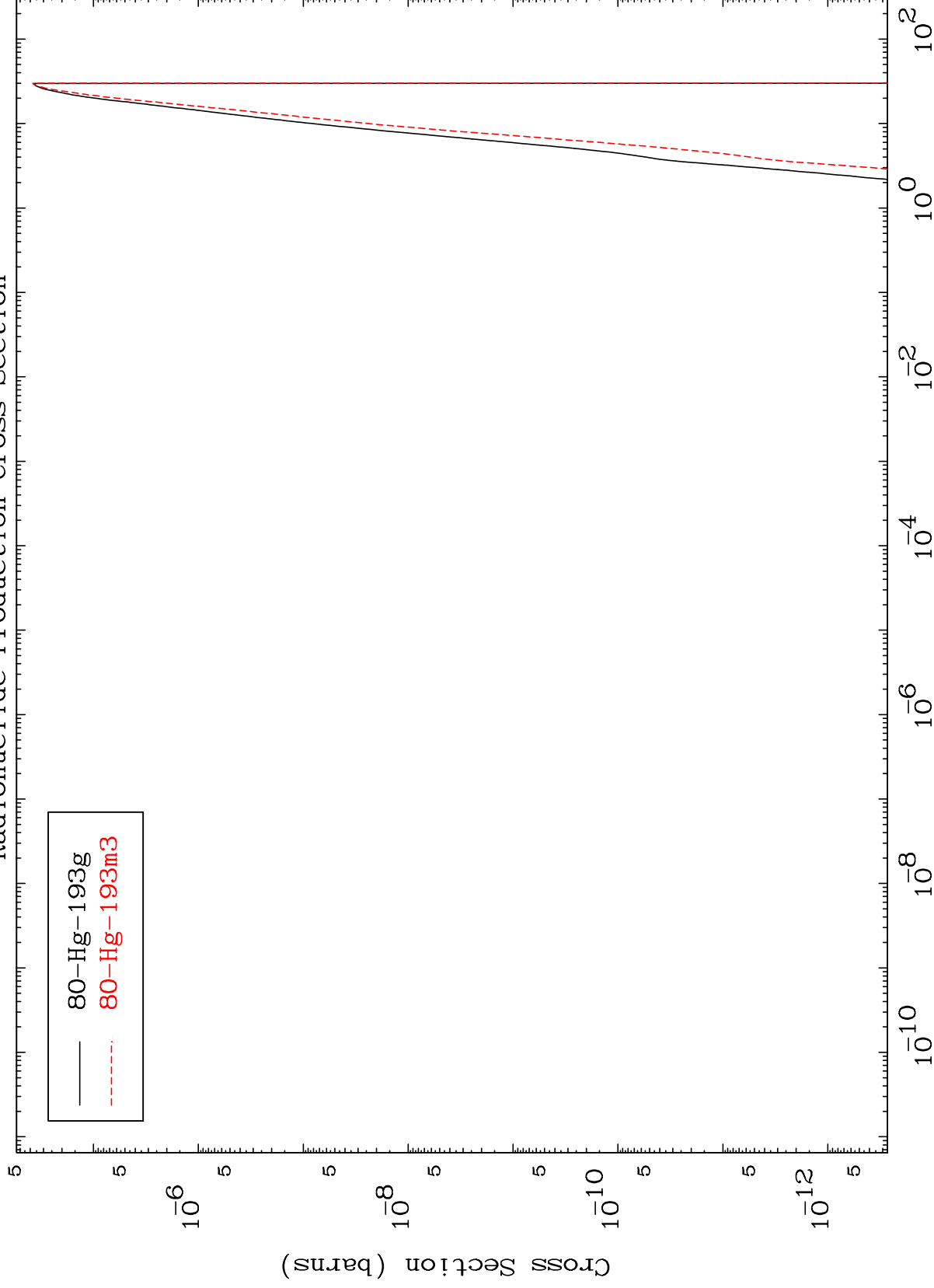
83-Bi-197

MAT 8290

(n,p) α

83-Bi-197

Radionuclide Production Cross Section



36

Incident Energy (MeV)

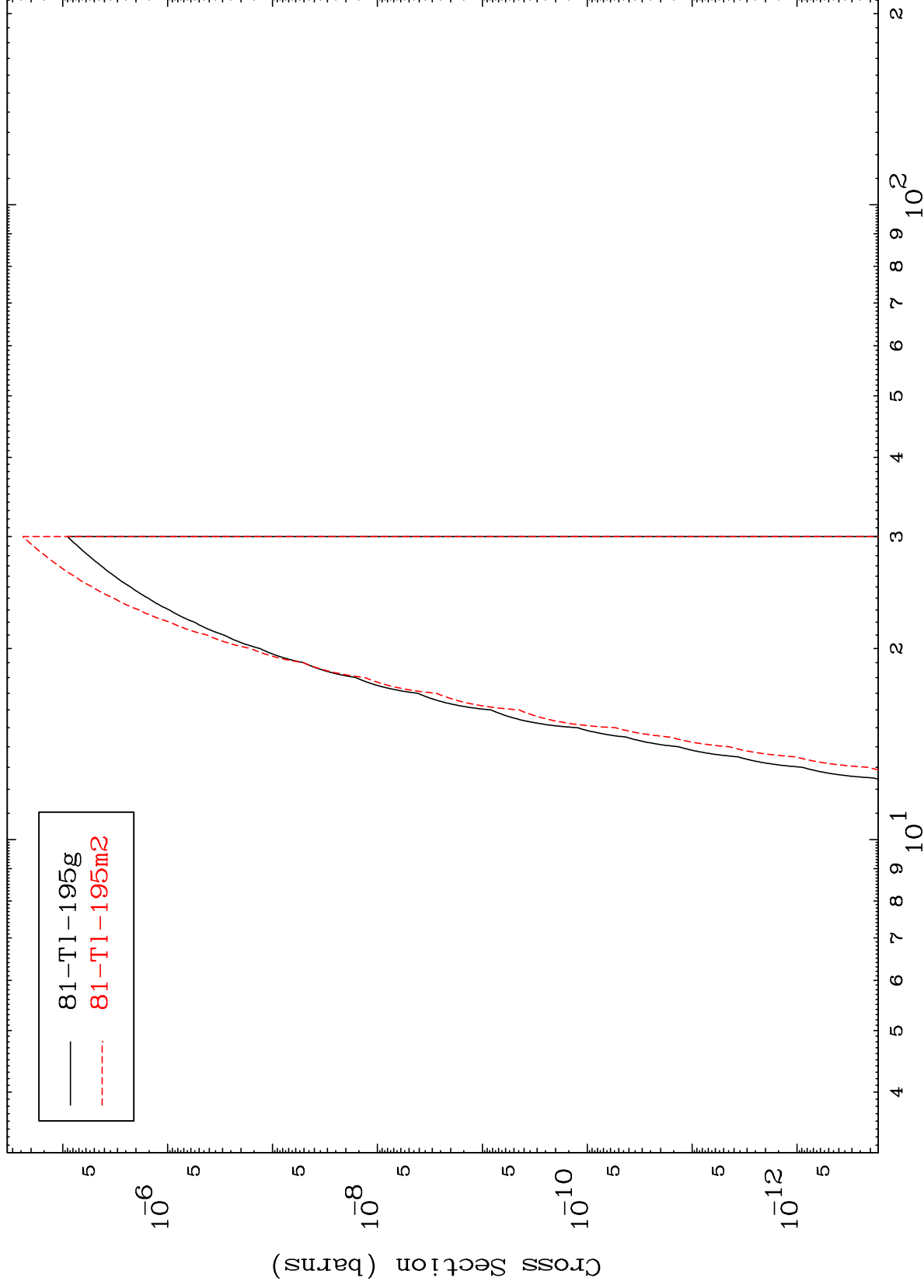
83-Bi-197

MAT 8290

(n,p) d

83-Bi-197

Radionuclide Production Cross Section



37

Incident Energy (MeV)

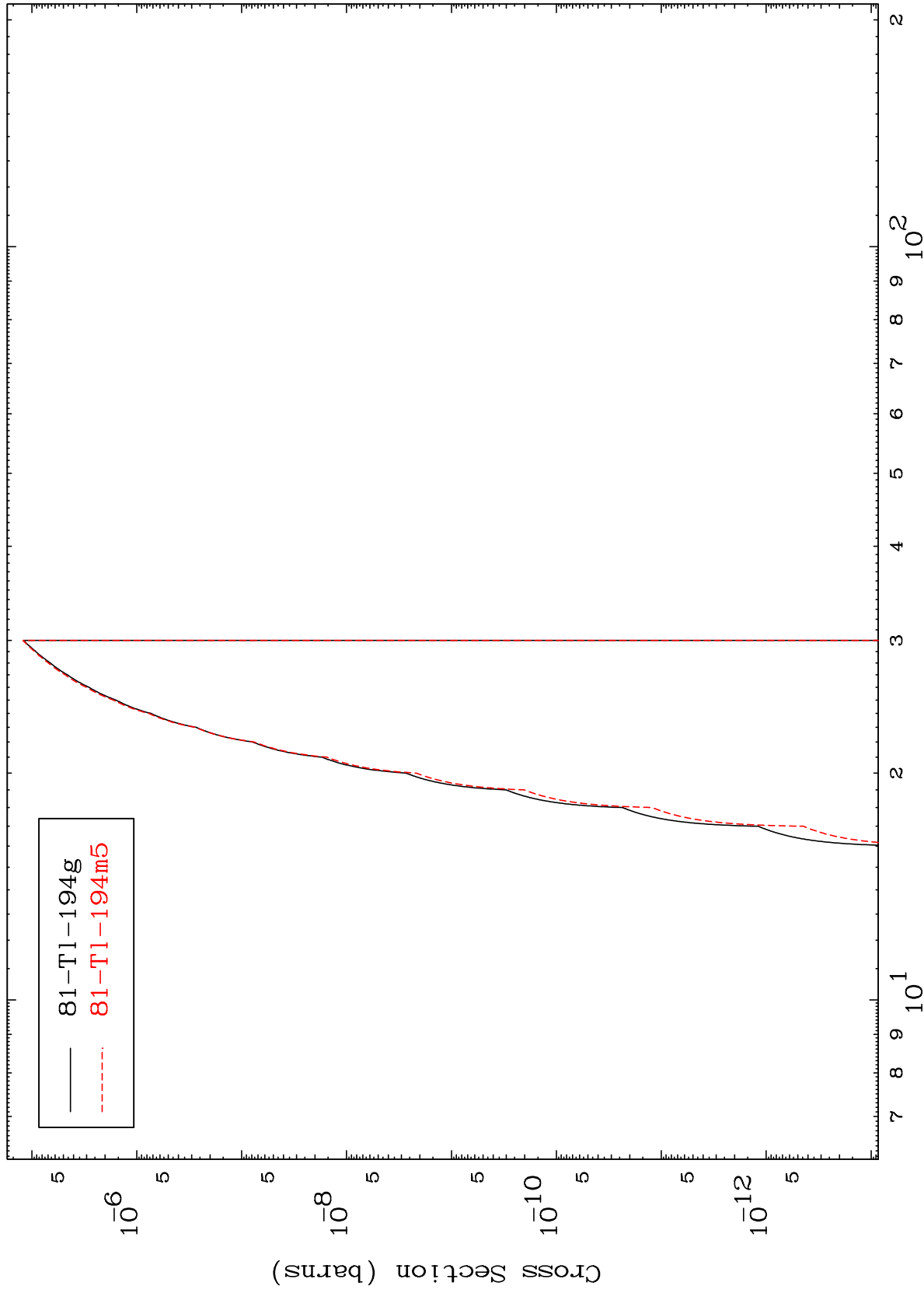
83-Bi-197

MAT 8290

(n,p) t

83-Bi-197

Radionuclide Production Cross Section



38

Incident Energy (MeV)

83-Bi-197