

Program EVALPLOT
(Version 2018-1)

by

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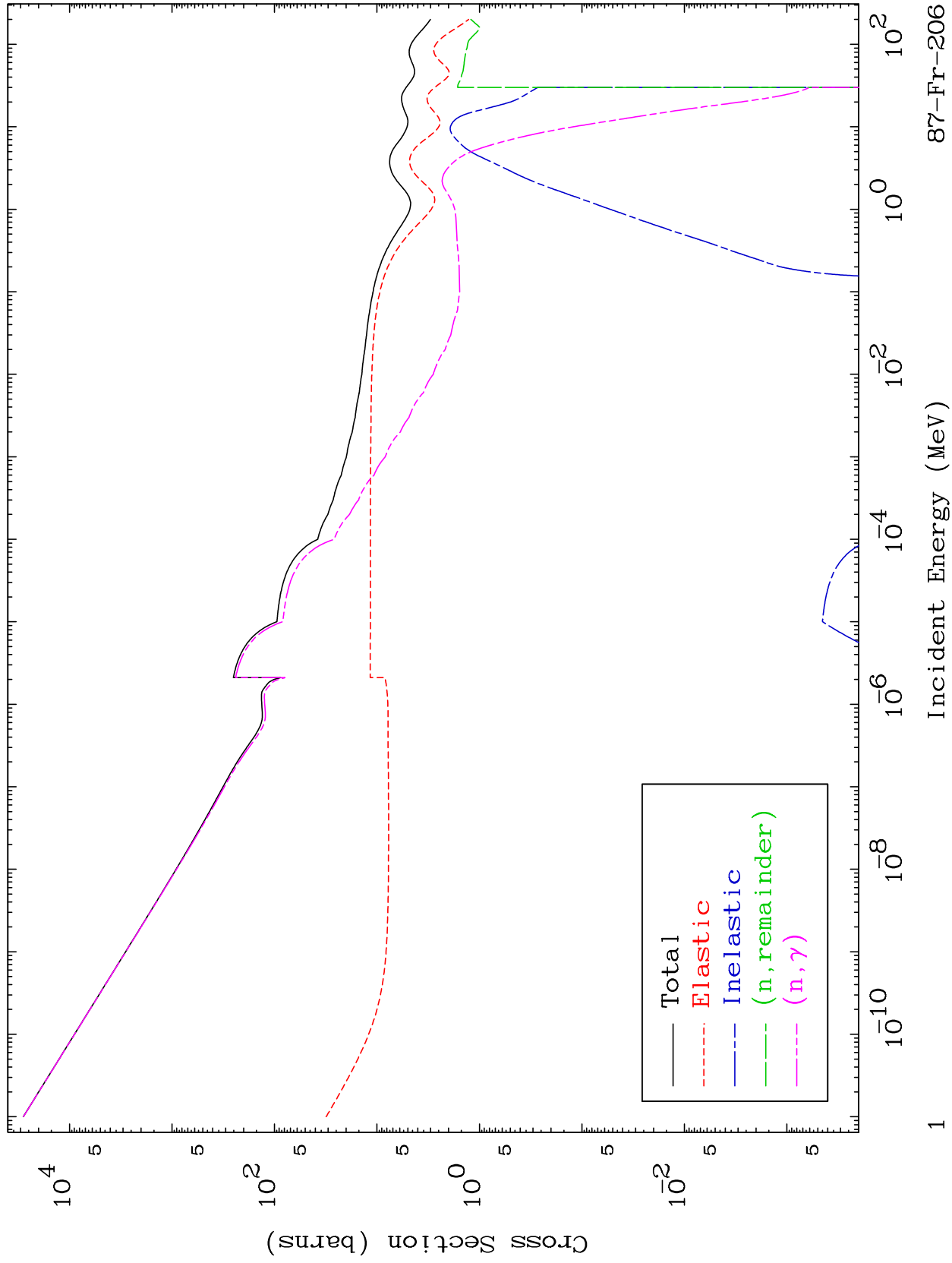
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8708

Major
293 Kelvin Cross Sections

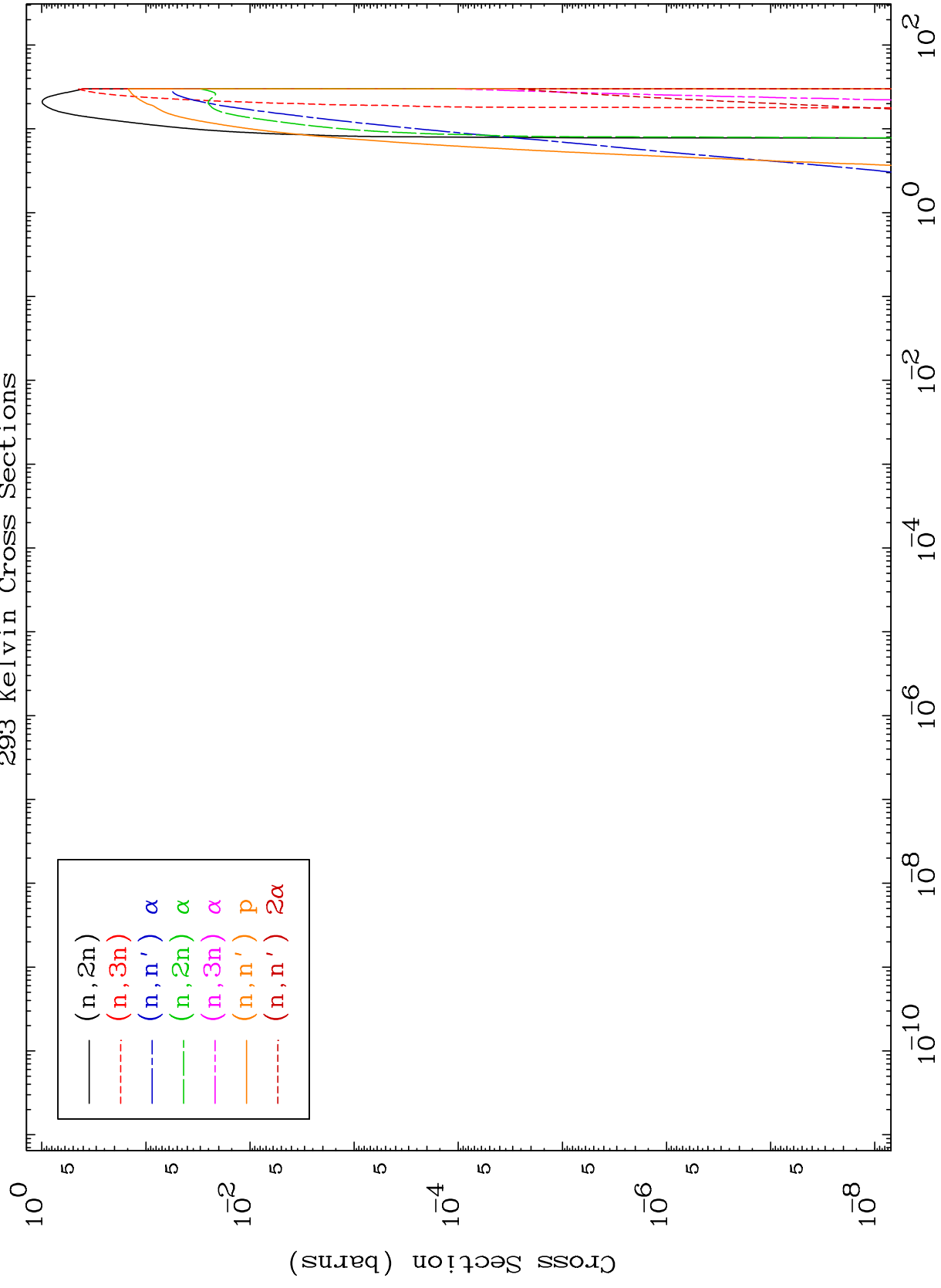
87-Fr-206



MAT 8708

Neutron Production
293 Kelvin Cross Sections

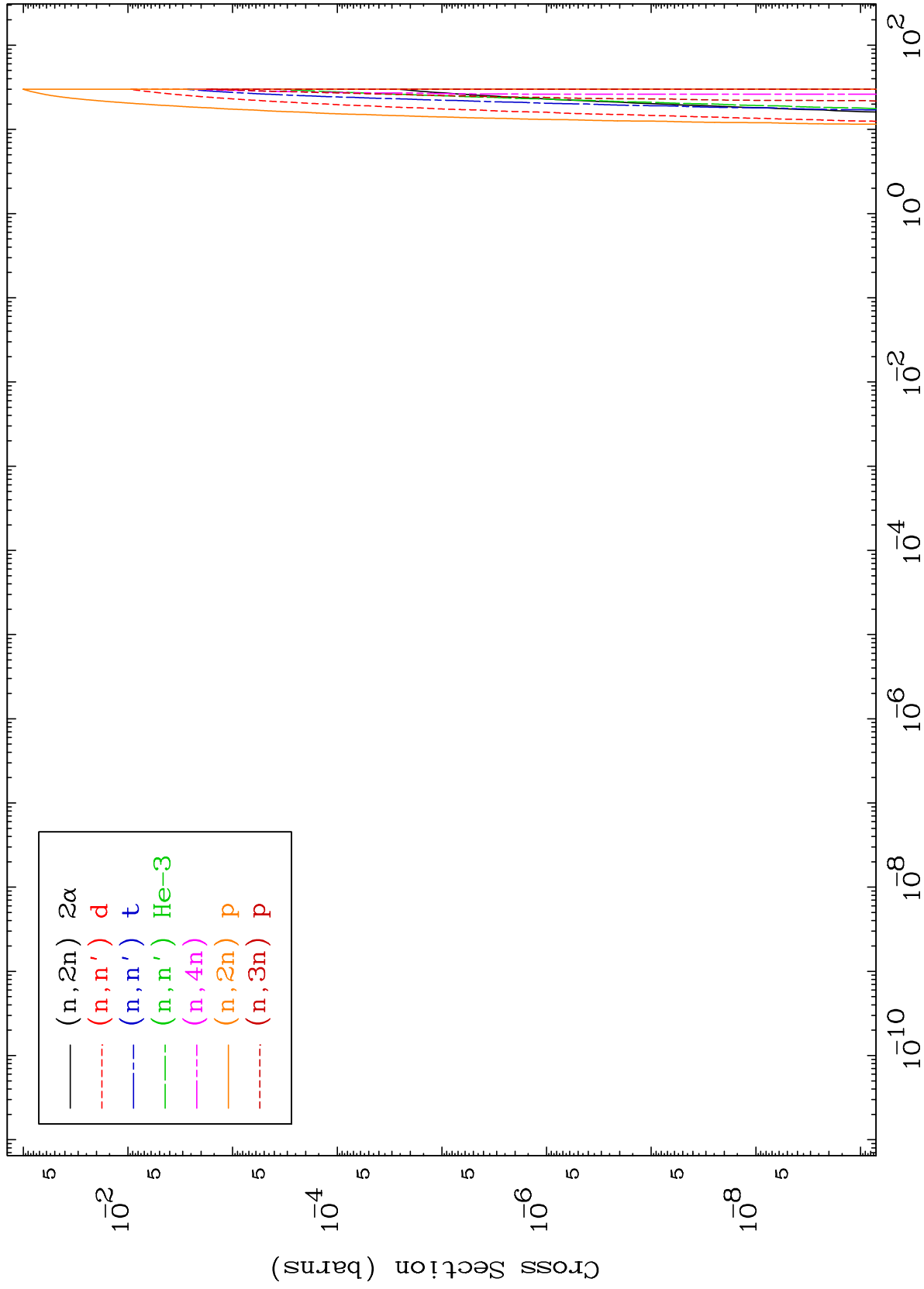
87-Fr-206



MAT 8708

Neutron Production
293 Kelvin Cross Sections

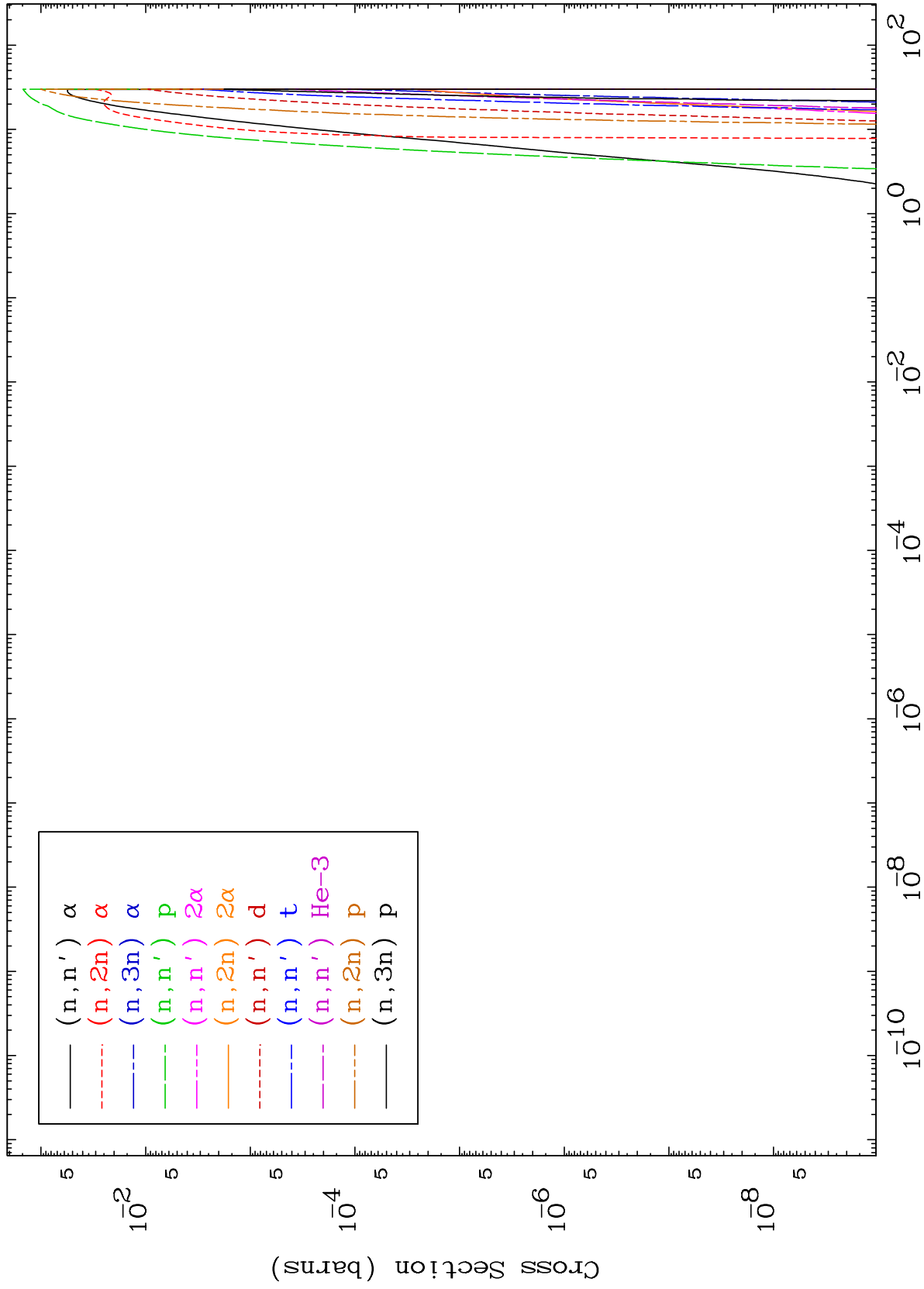
87-Fr-206



MAT 8708

Charged Particle
293 Kelvin Cross Sections

87-Fr-206

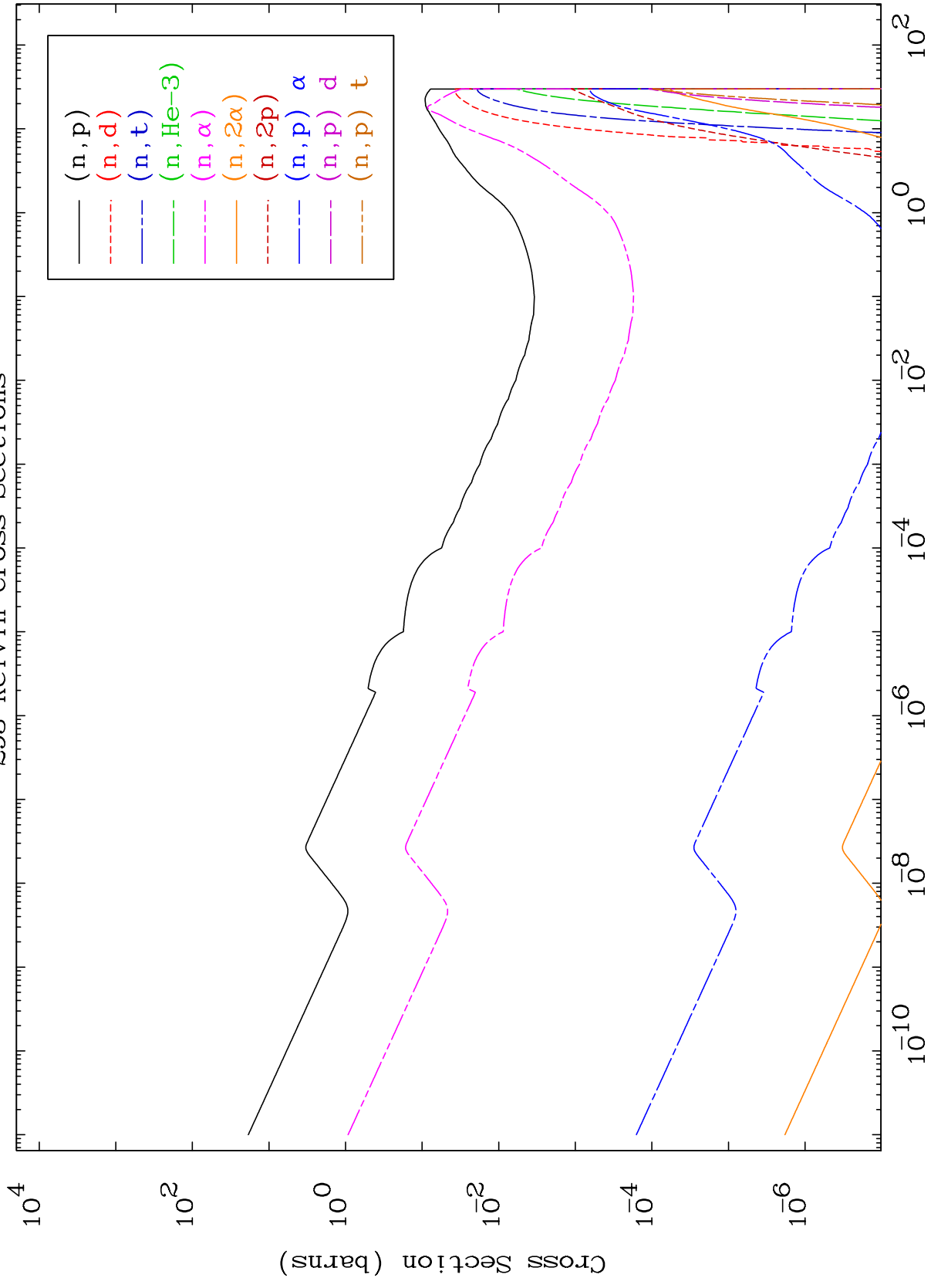


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MAT 8708

293 Kelvin Cross Sections

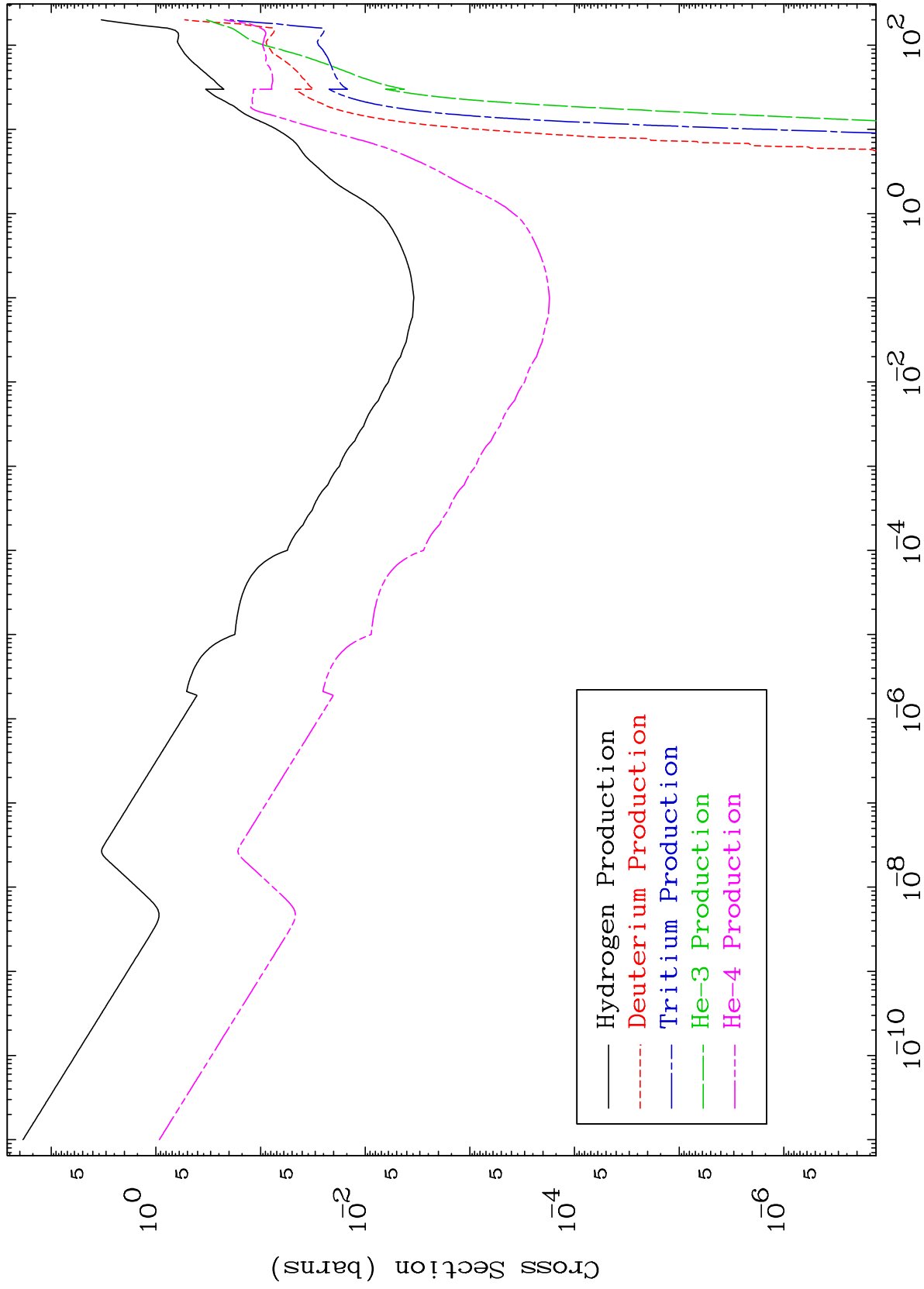
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MAT 8708

Particle Production
293 Kelvin Cross Sections

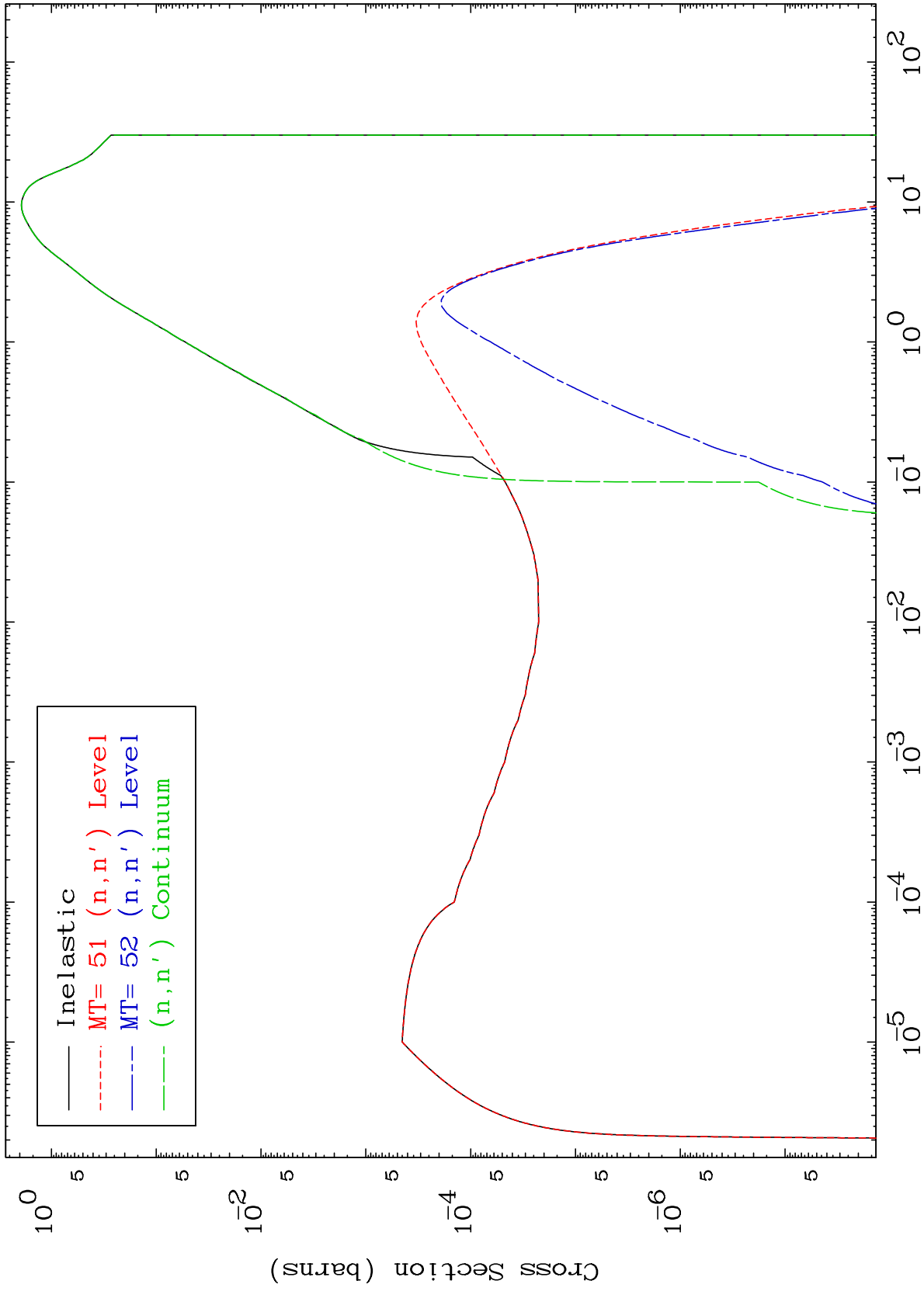
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MAT 8708

293 Kelvin Cross Sections

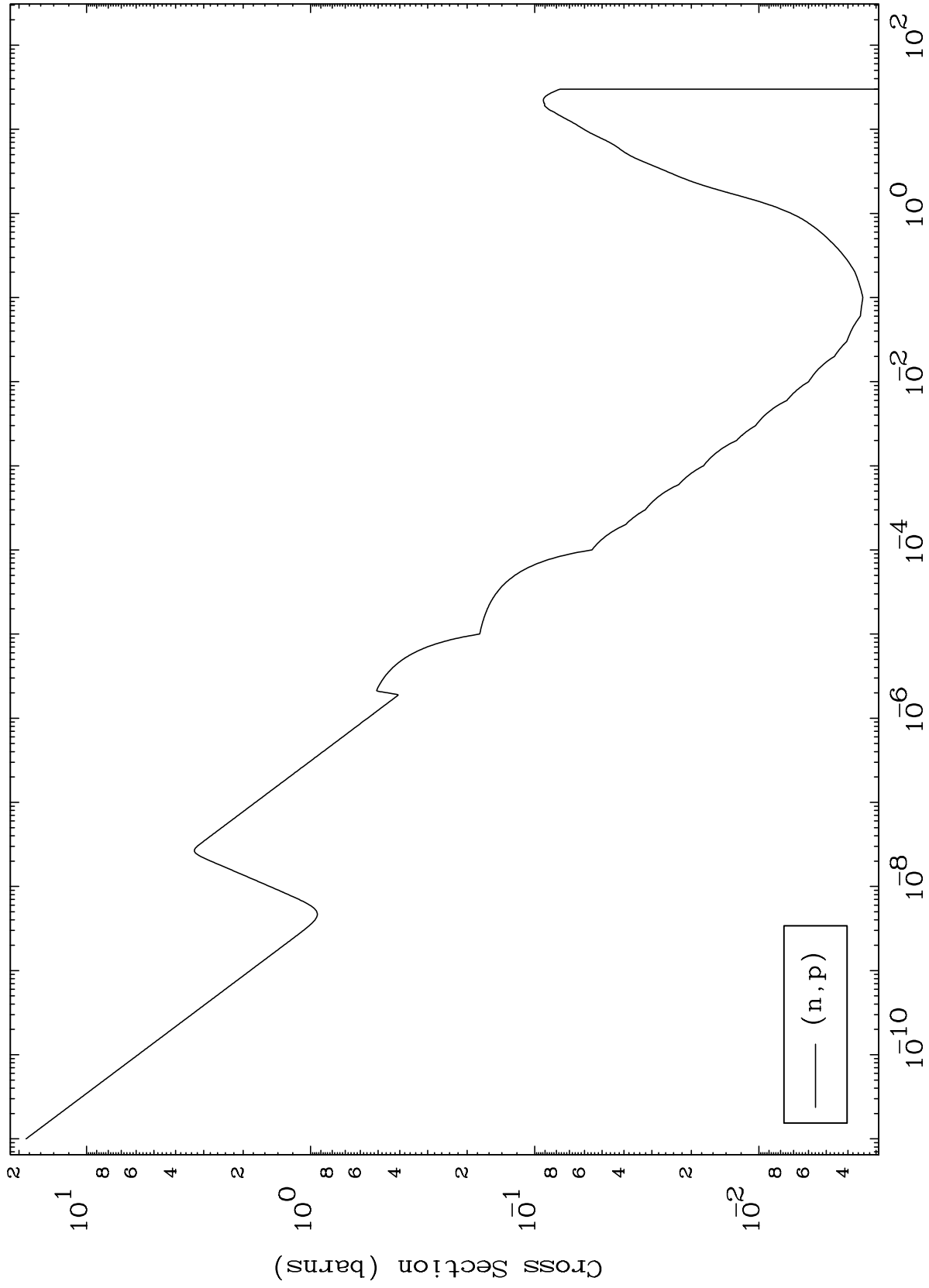
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MAT 8708

(n,p) Levels
293 Kelvin Cross Sections

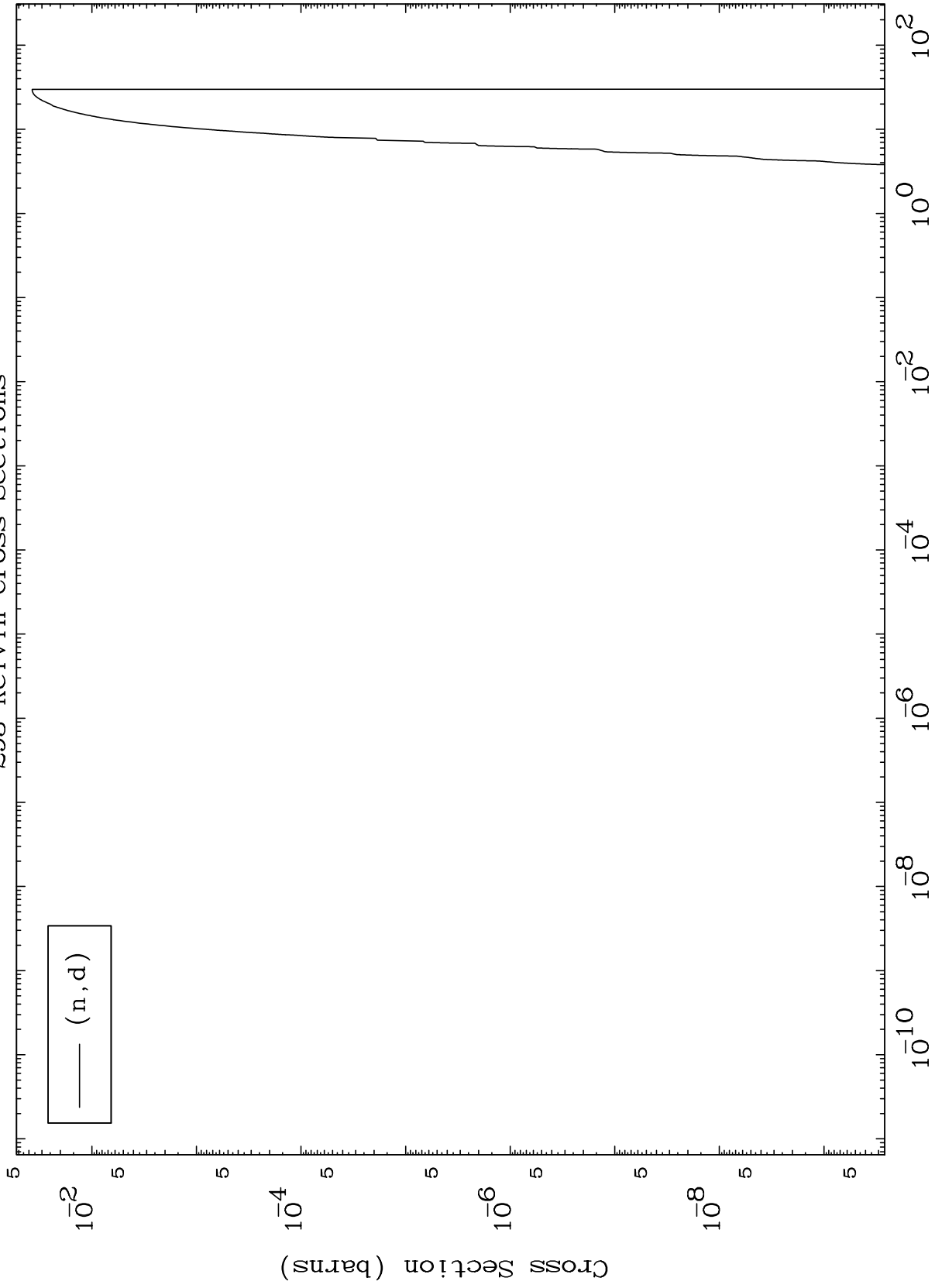
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MAT 8708

(n,d) Levels
293 Kelvin Cross Sections

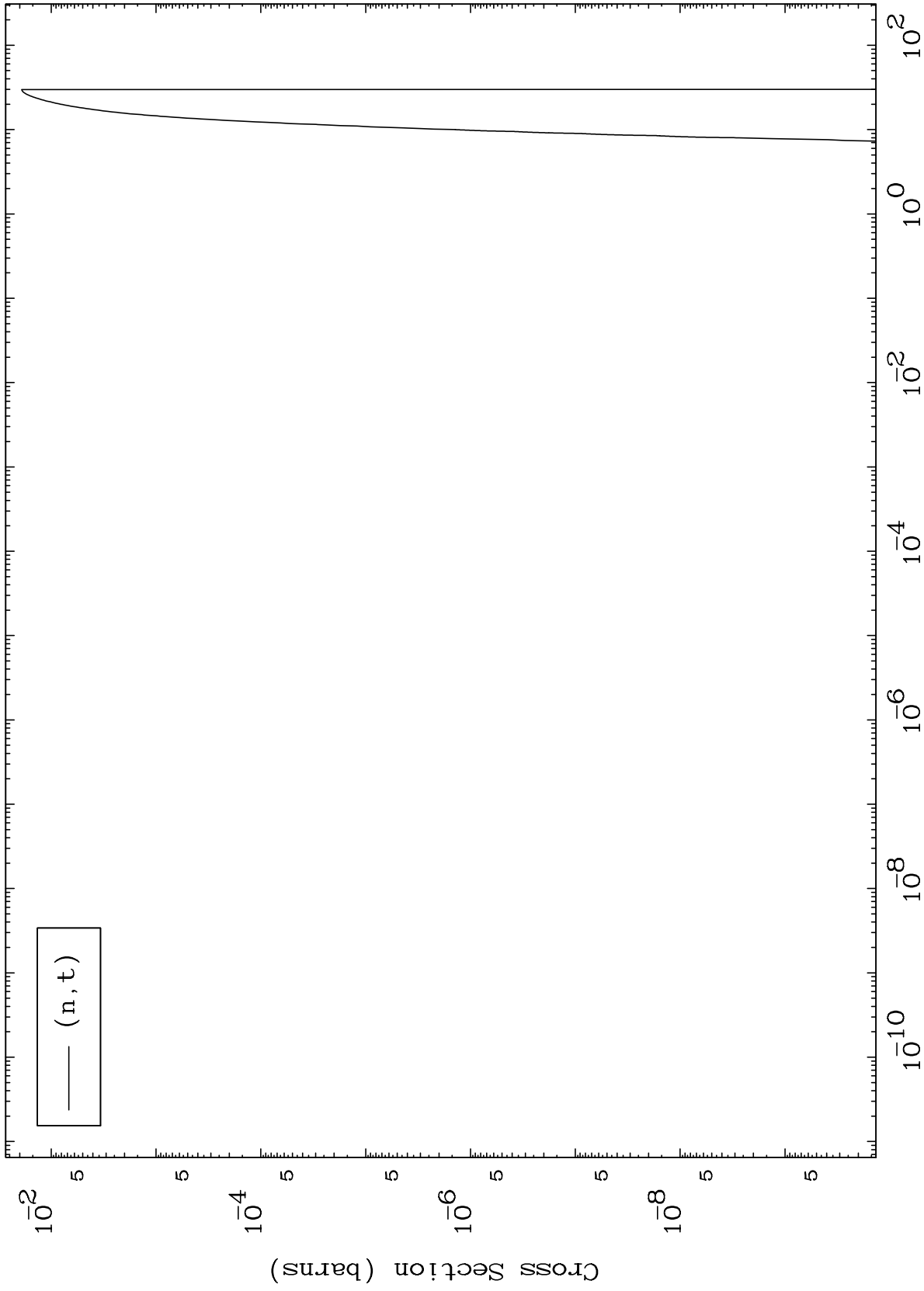
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(n,t) Levels
293 Kelvin Cross Sections

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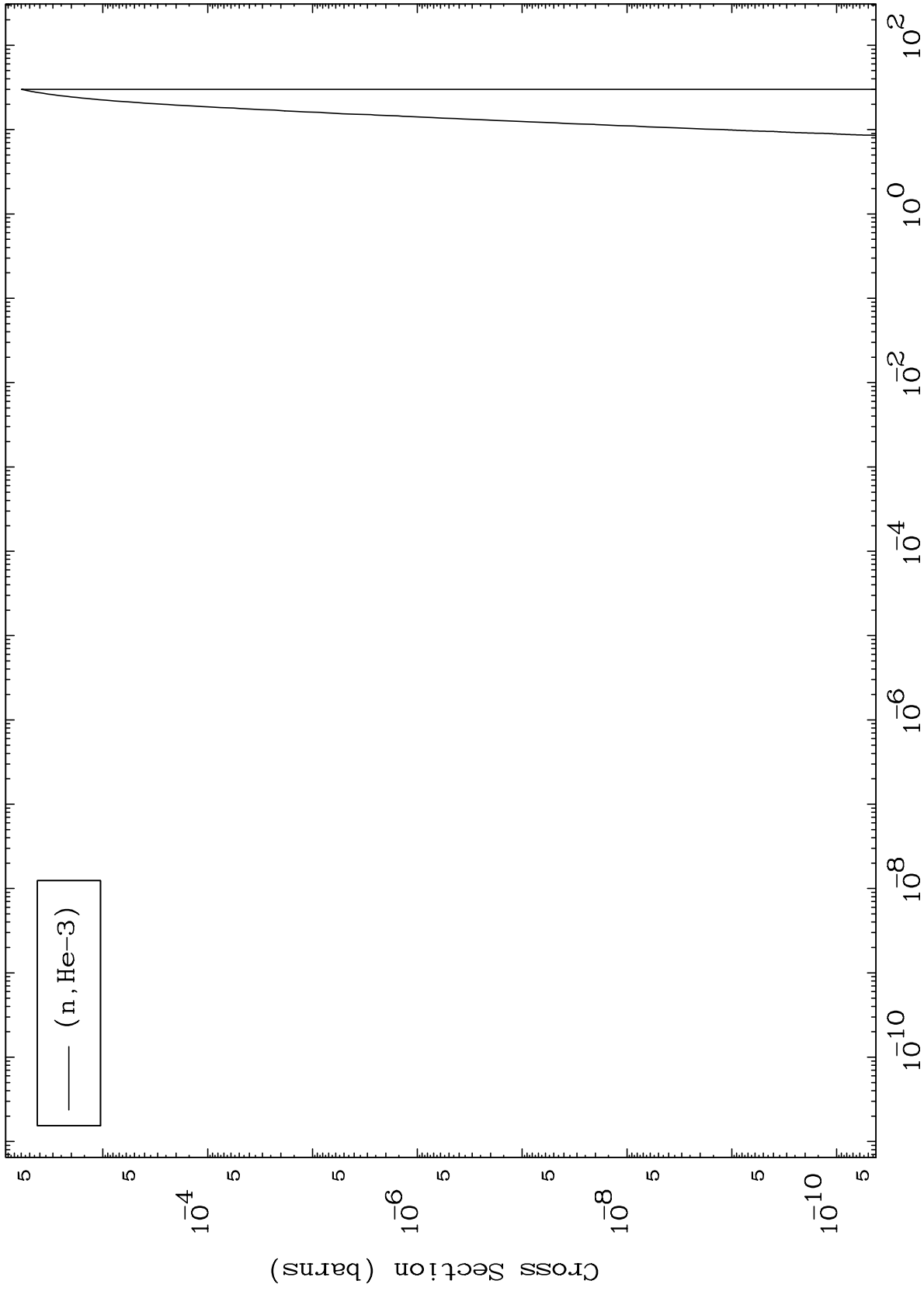
Incident Energy (MeV)

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MAT 8708

(n,He3) Levels
293 Kelvin Cross Sections

87-Fr-206



11

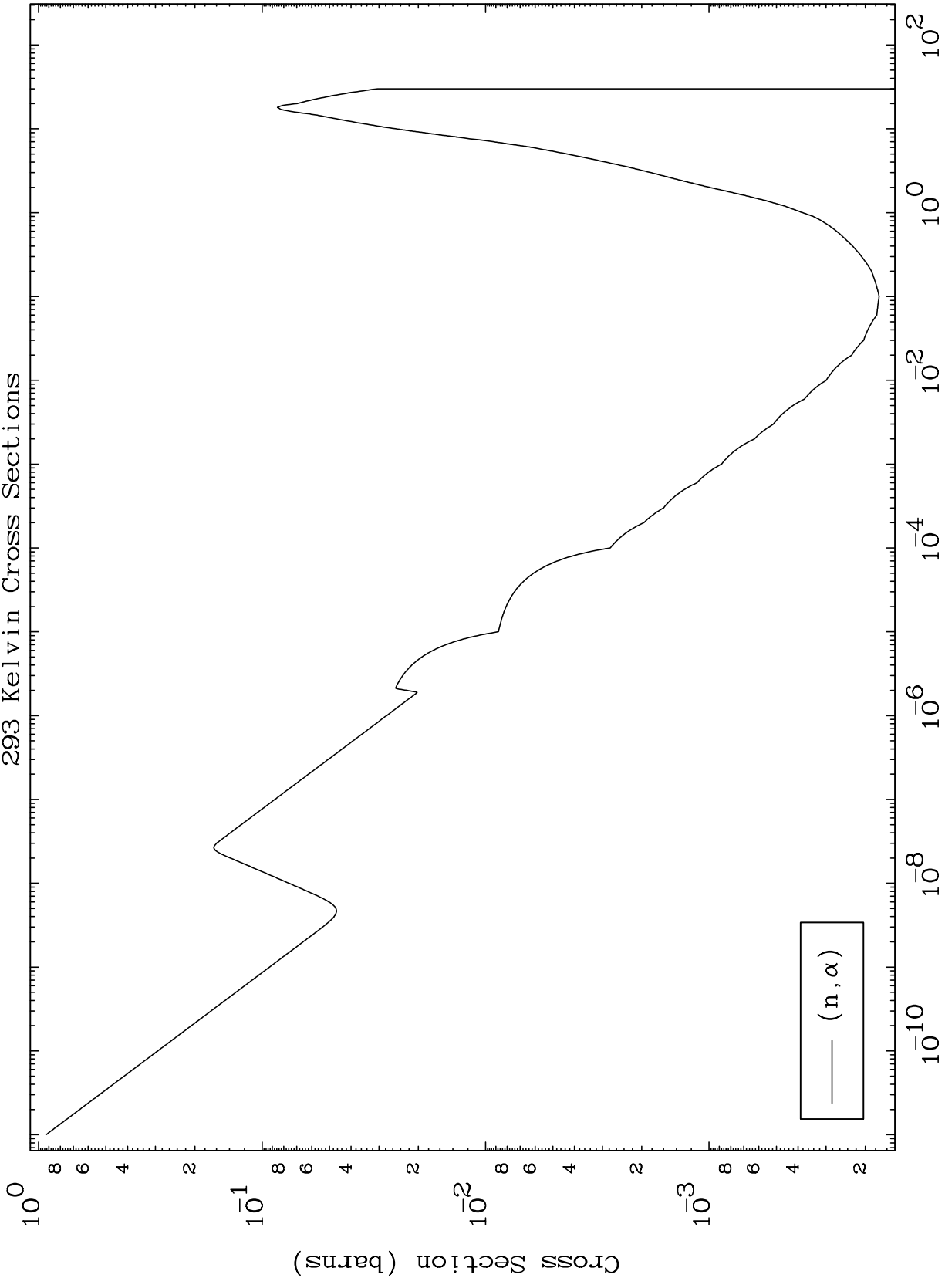
Incident Energy (MeV)

87-Fr-206

MAT 8708

(n, α) Levels
293 Kelvin Cross Sections

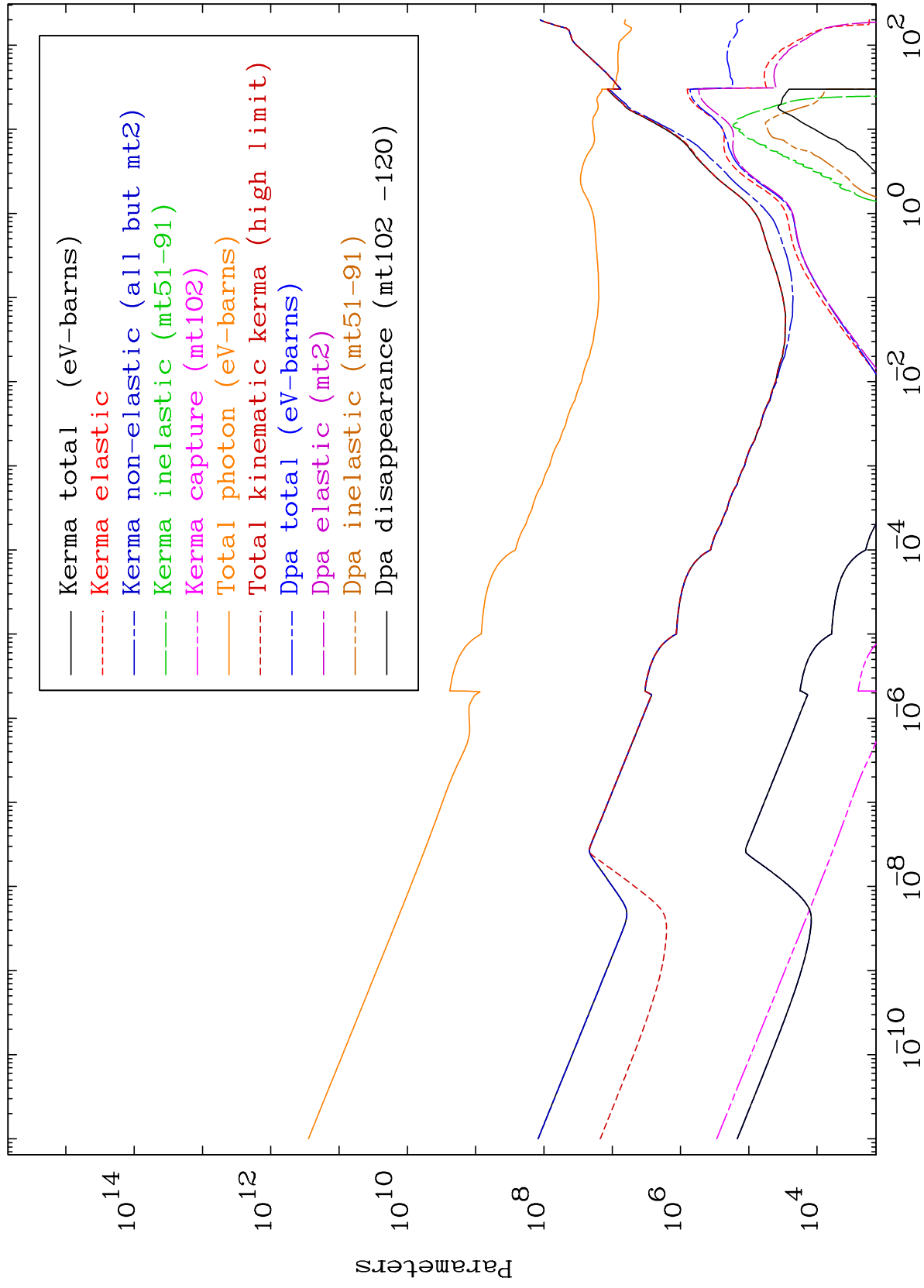
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Incident Energy (MeV)

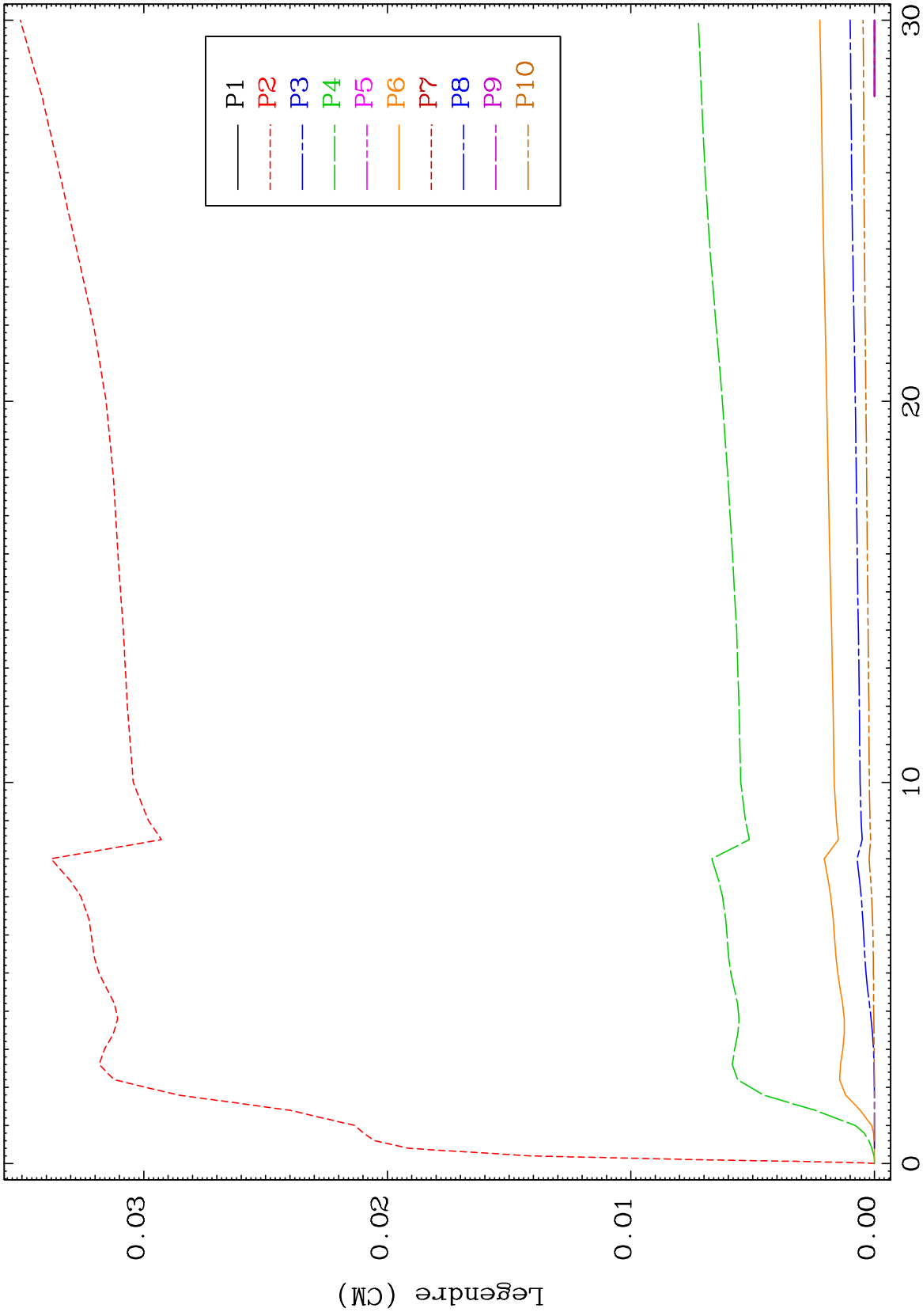
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Elastic Legendre Coefficients

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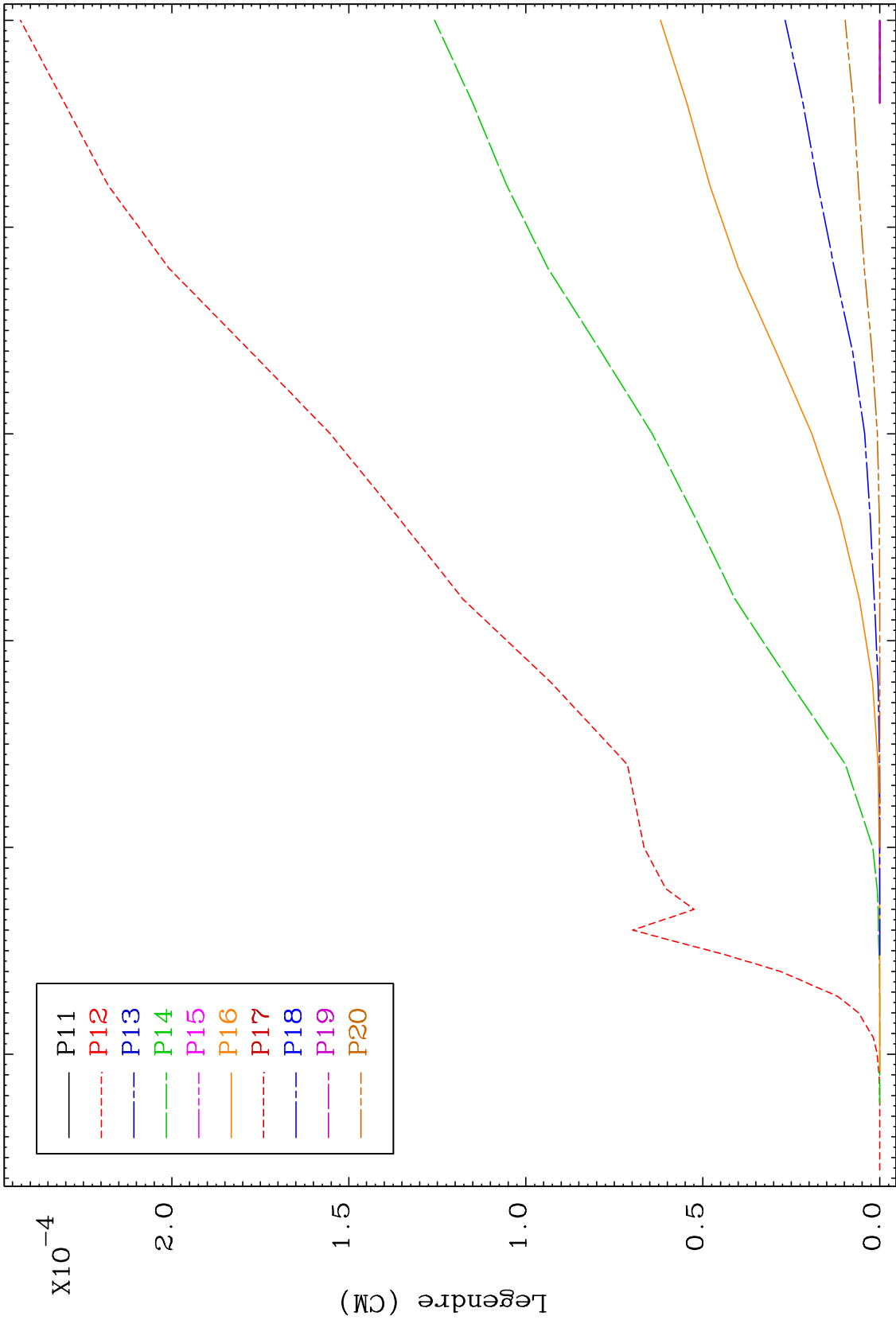
Incident Energy (MeV)

87-Fr-206

MAT 8708

Elastic Legendre Coefficients

87-Fr-206



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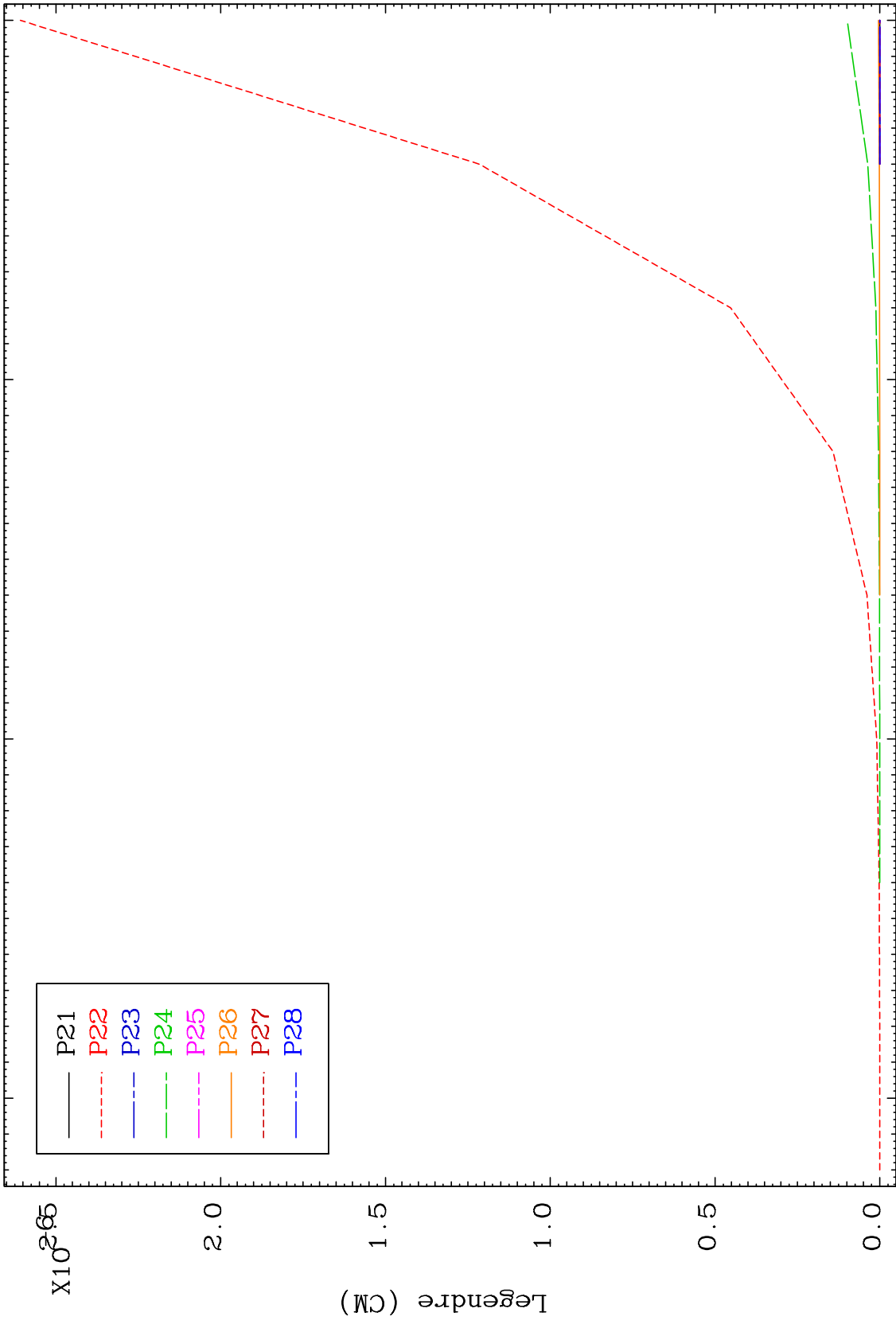
Incident Energy (MeV)

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Elastic Legendre Coefficients

87-Fr-206



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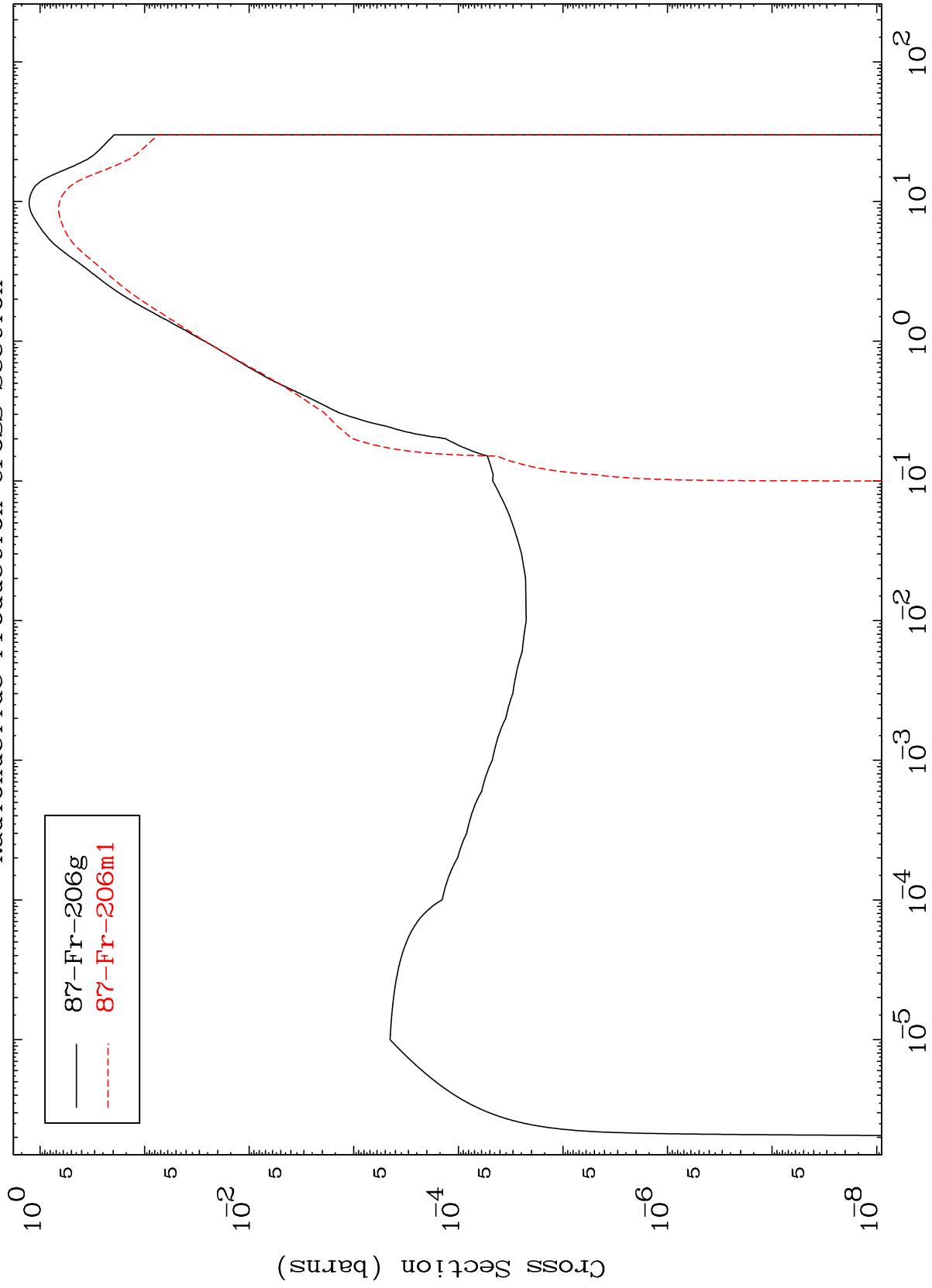
Incident Energy (MeV)

87-Fr-206

MAT 8708

Inelastic
Radionuclide Production Cross Section

87-Fr-206



17

Incident Energy (MeV)

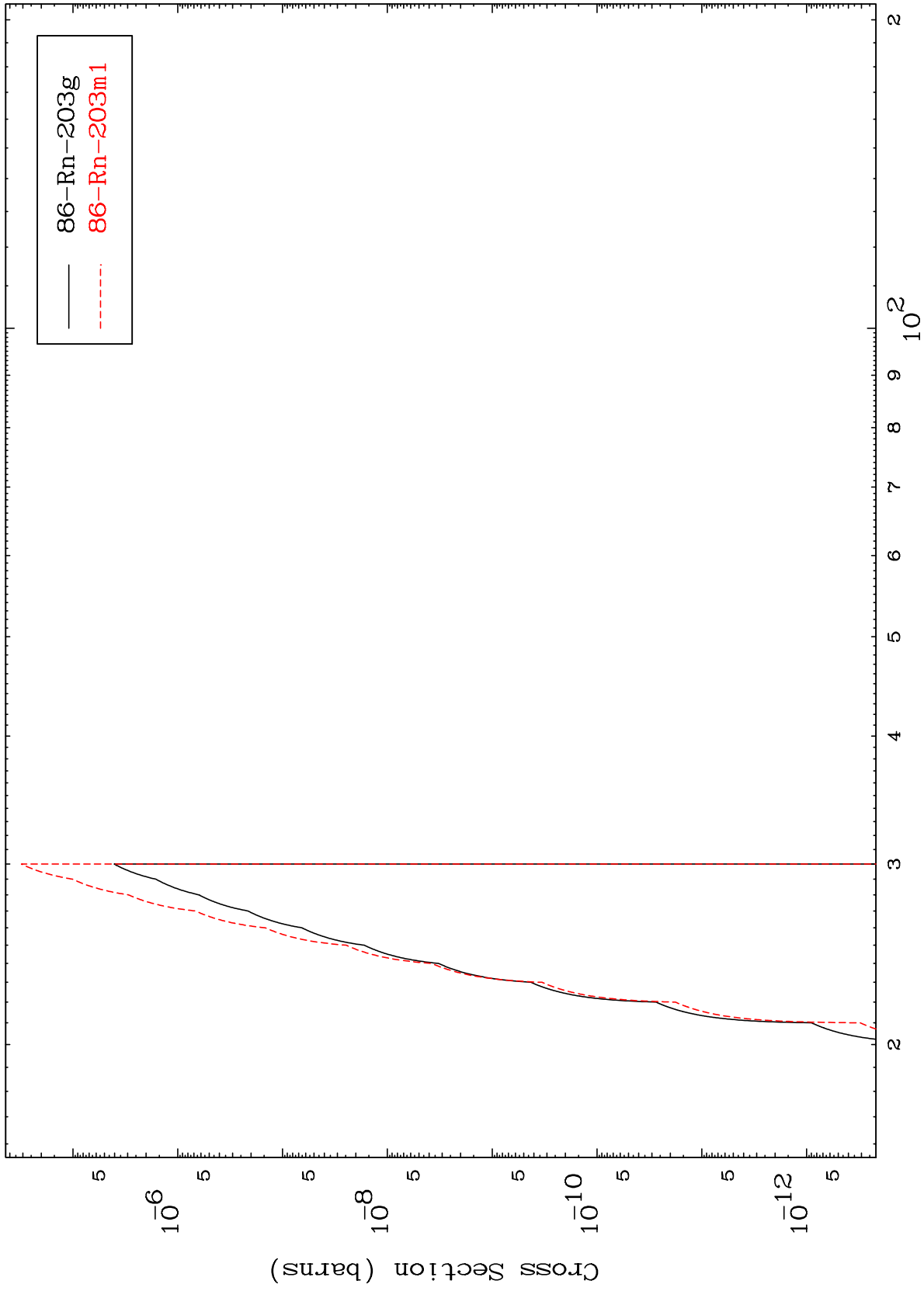
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(n,2n) d

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Radionuclide Production Cross Section

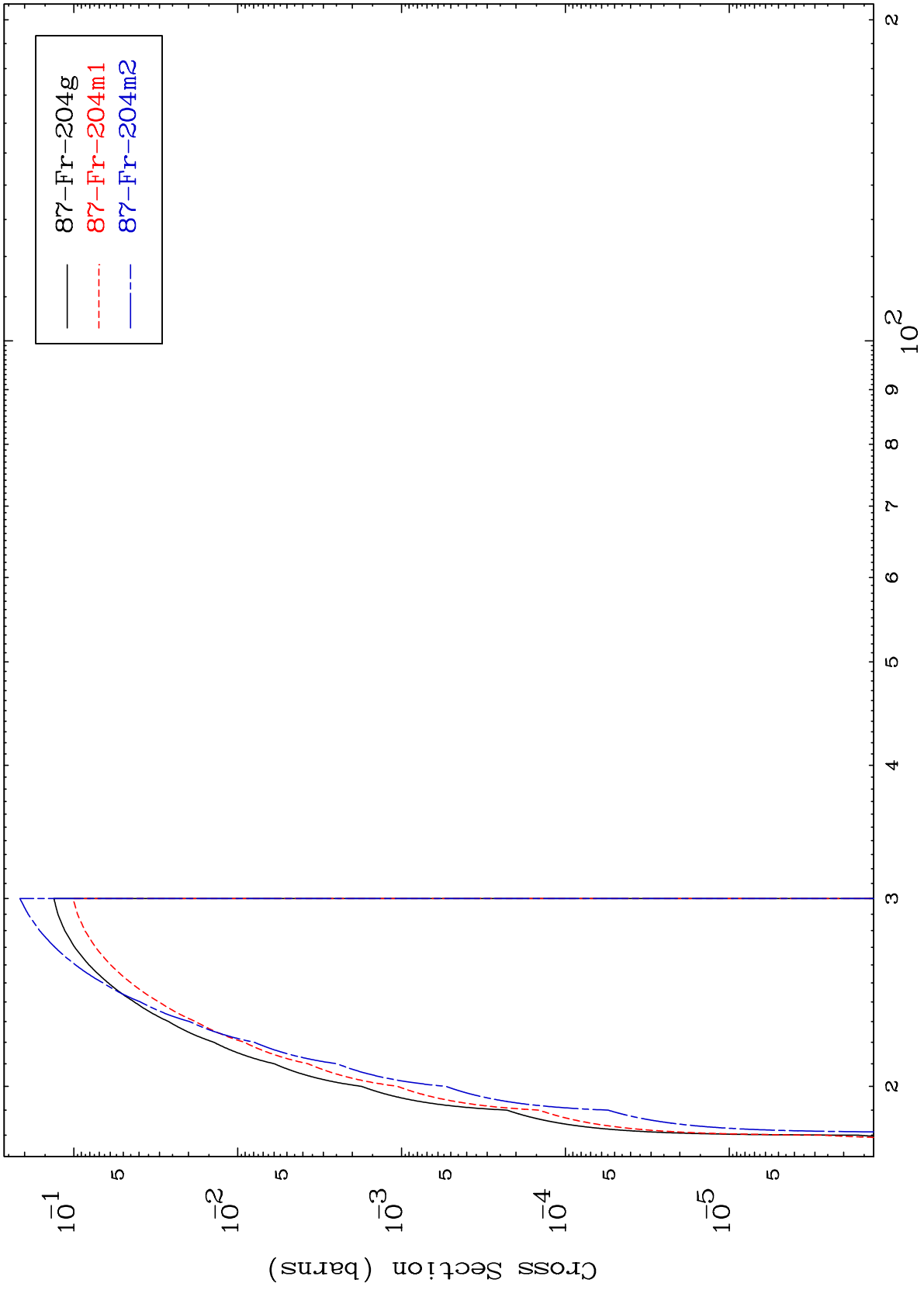


18

Incident Energy (MeV)

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(n,3n)
Radionuclide Production Cross Section

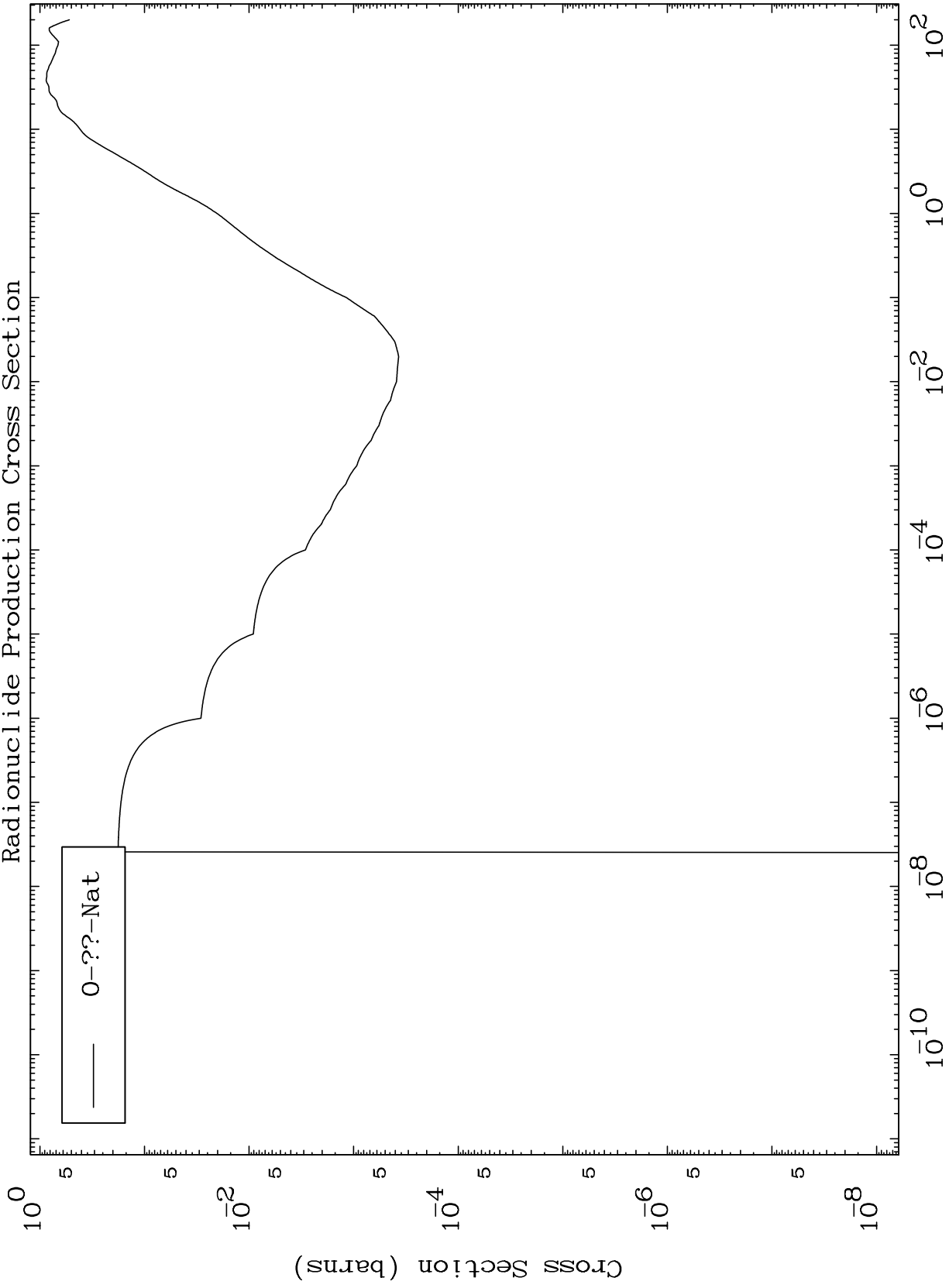


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Fission

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Radionuclide Production Cross Section



20

Incident Energy (MeV)

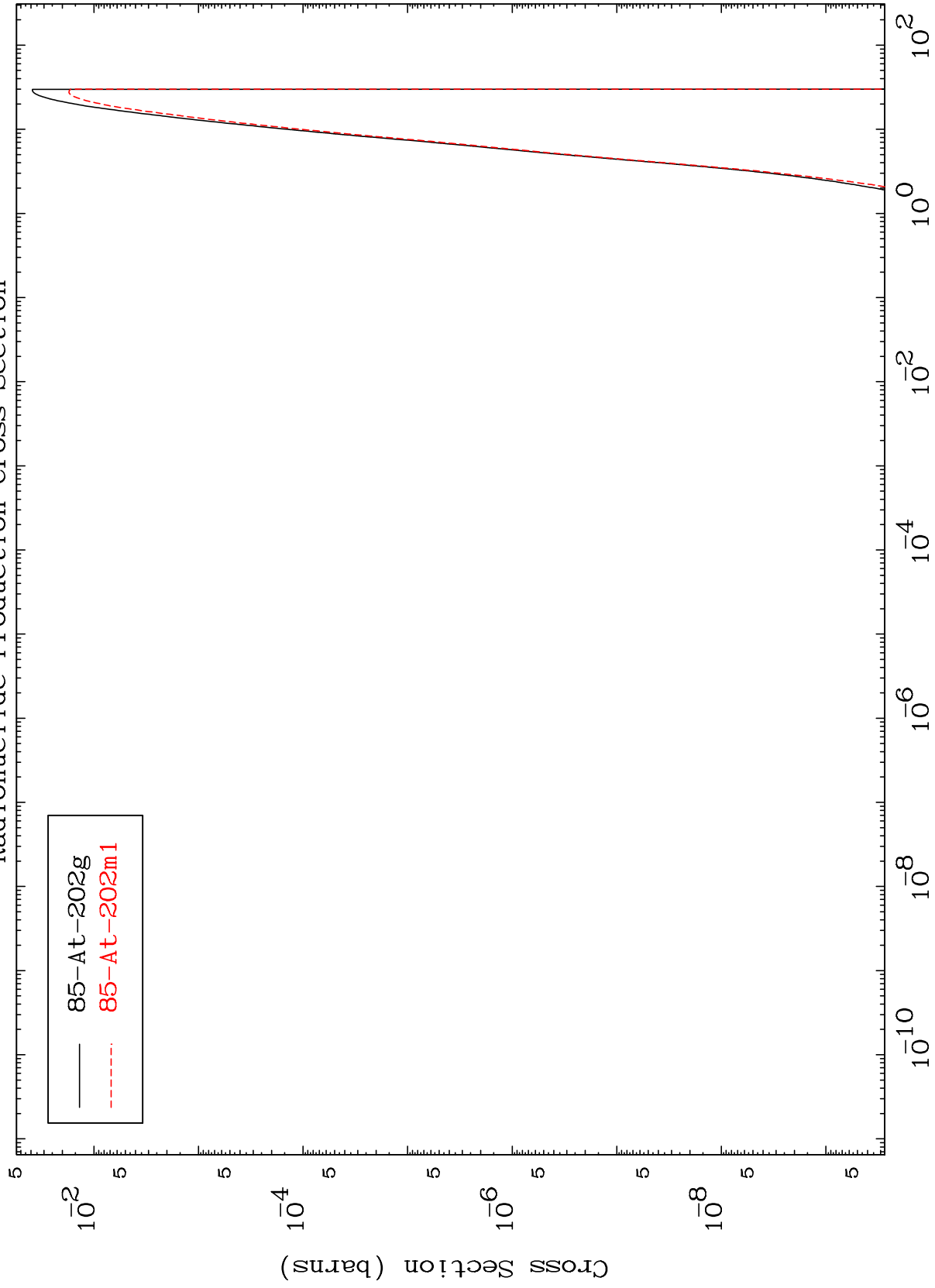
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(n,n') α

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Radionuclide Production Cross Section



21

Incident Energy (MeV)

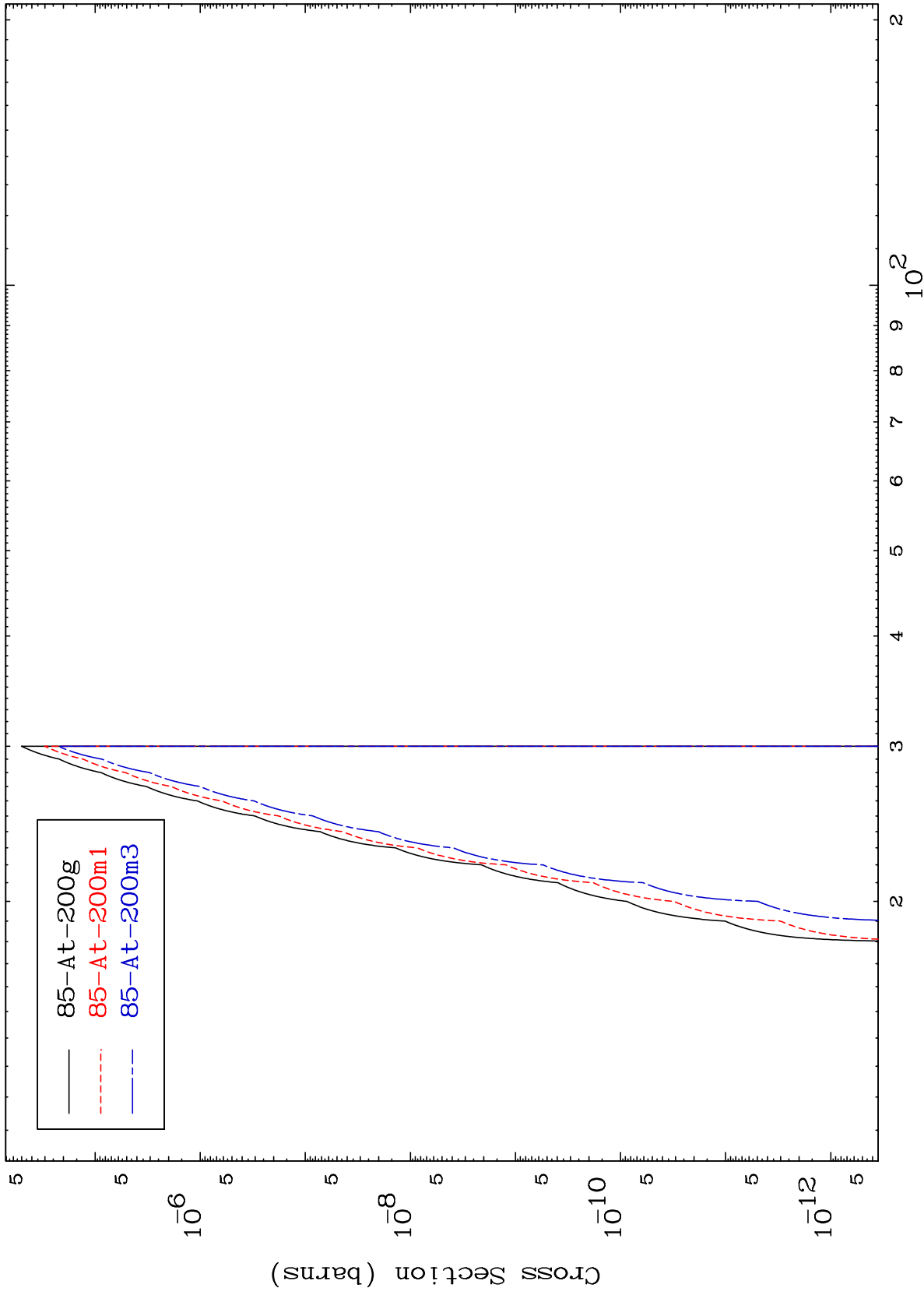
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MAT 8708

(n,3n) α

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Radionuclide Production Cross Section



22

Incident Energy (MeV)

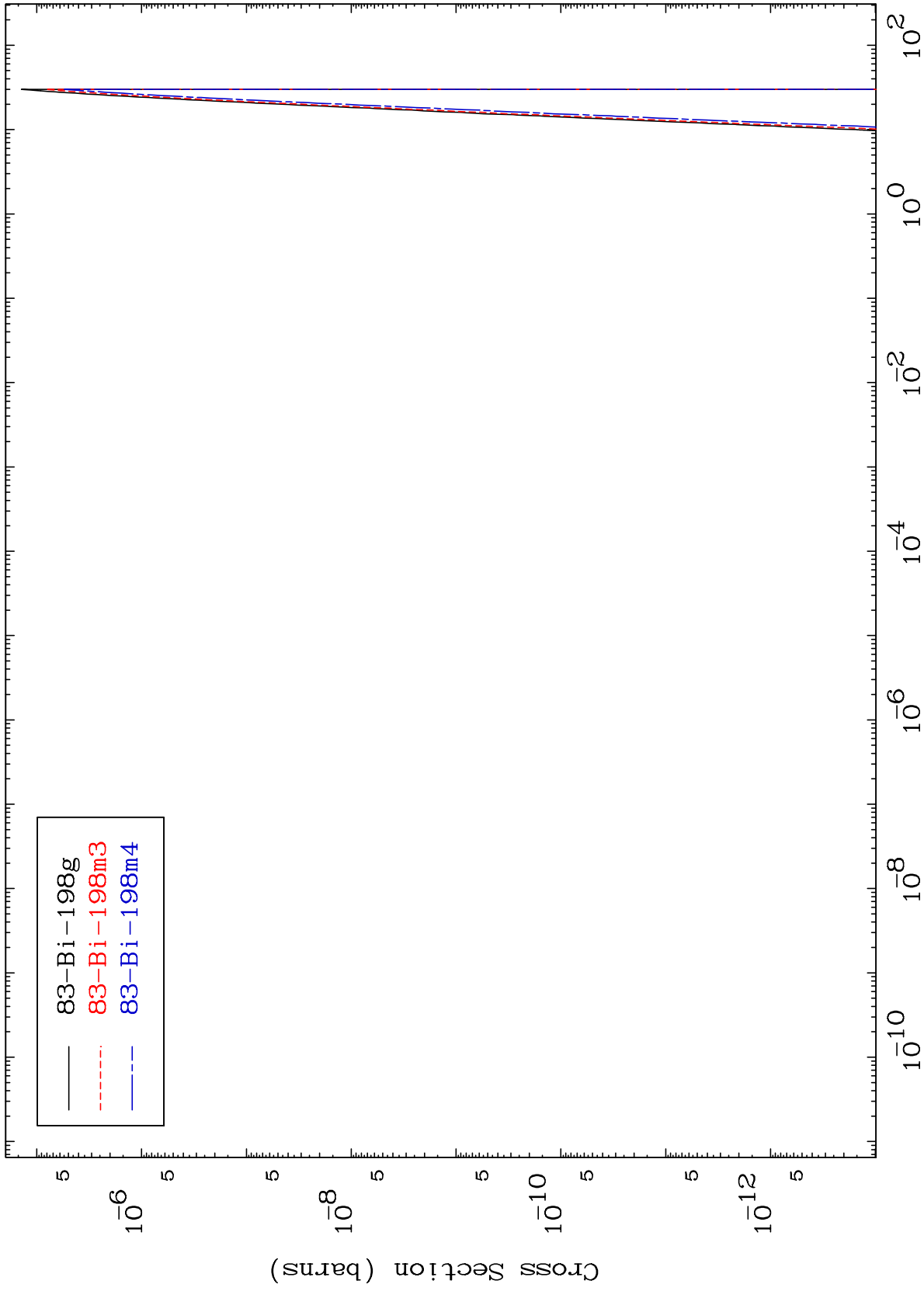
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MAT 8708

(n,n') 2 α

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Radionuclide Production Cross Section



23

Incident Energy (MeV)

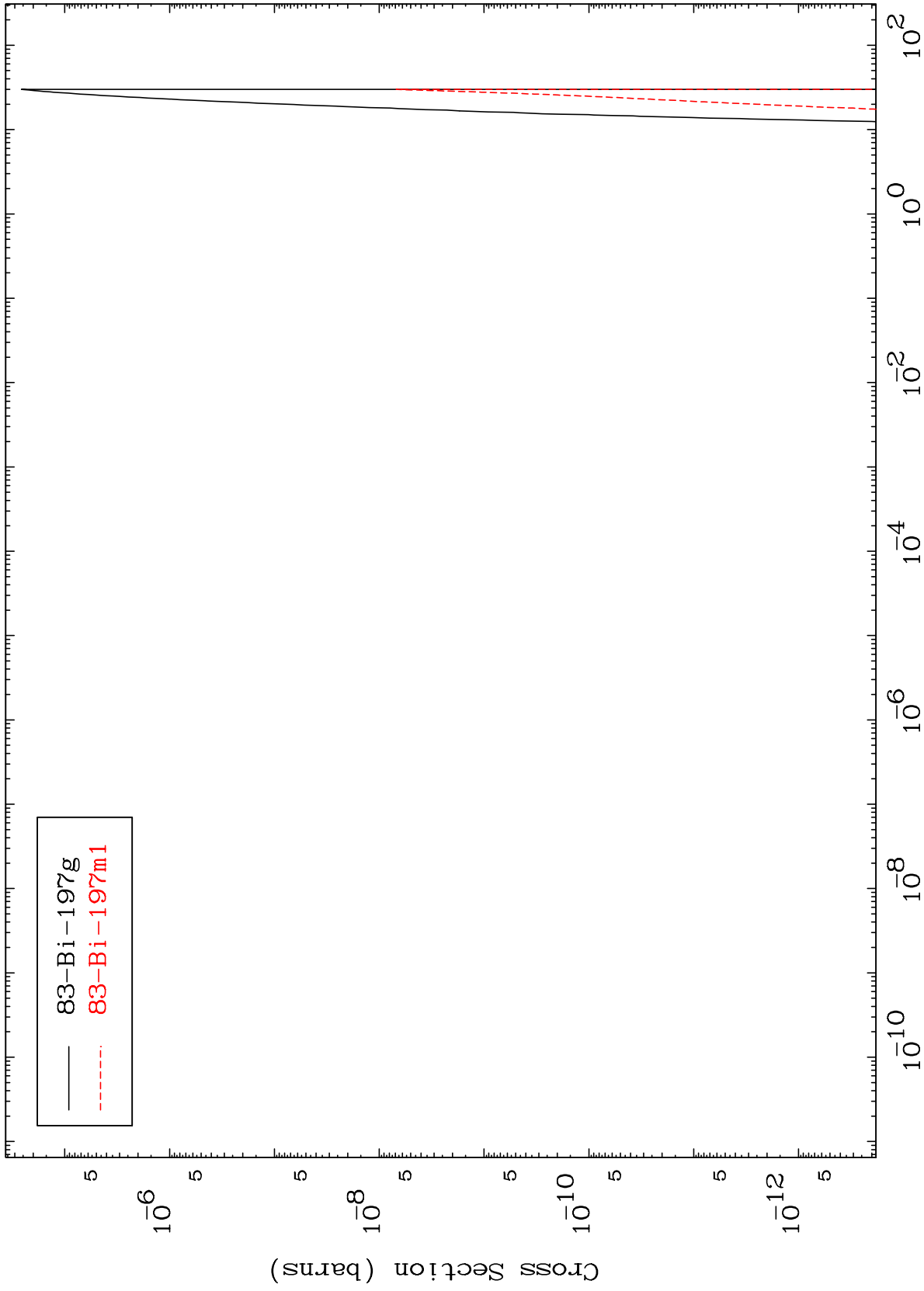
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MAT 8708

(n,2n) 2 α

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Radionuclide Production Cross Section



24

Incident Energy (MeV)

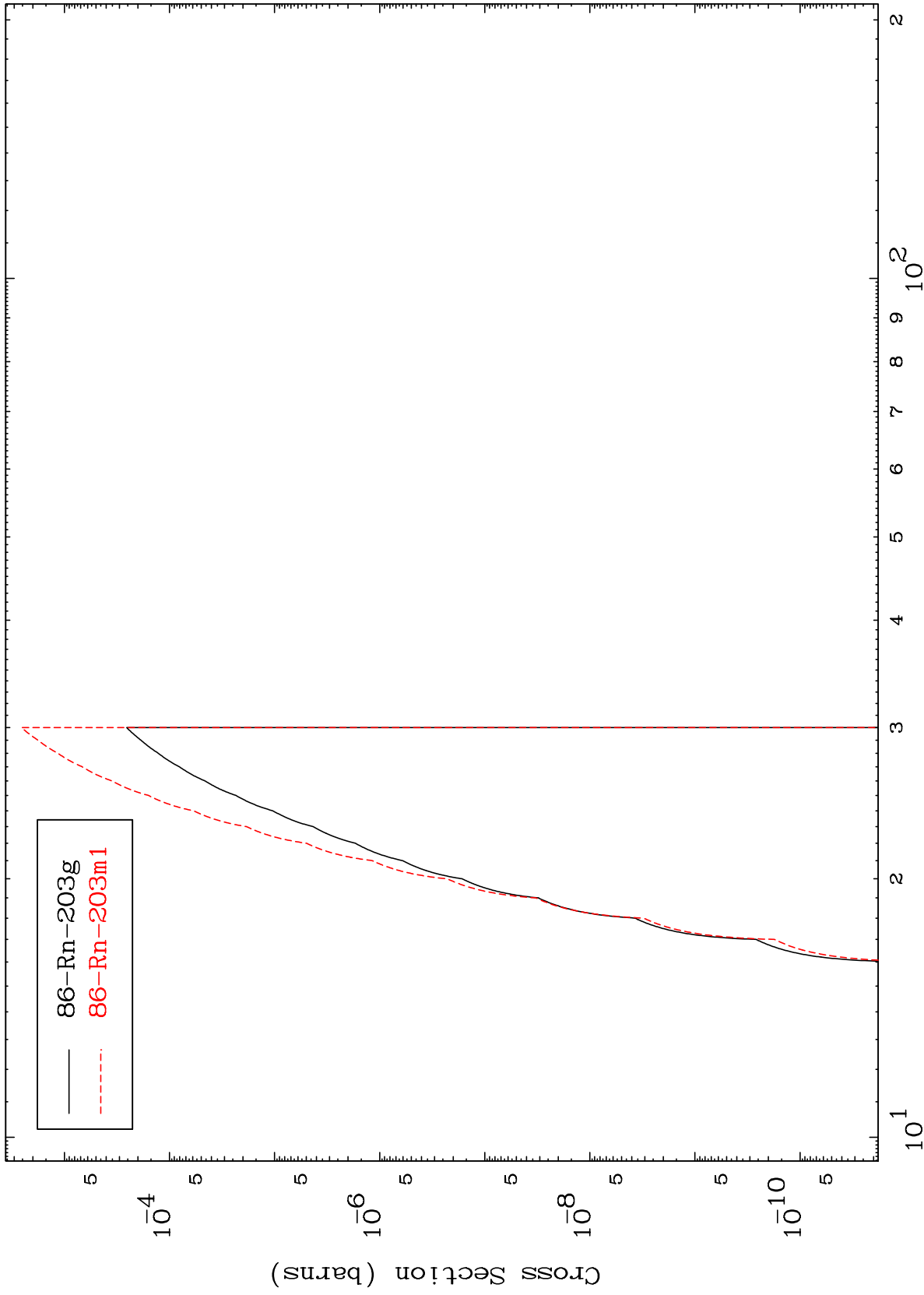
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MAT 8708

(n,n') t

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Radionuclide Production Cross Section



25

Incident Energy (MeV)

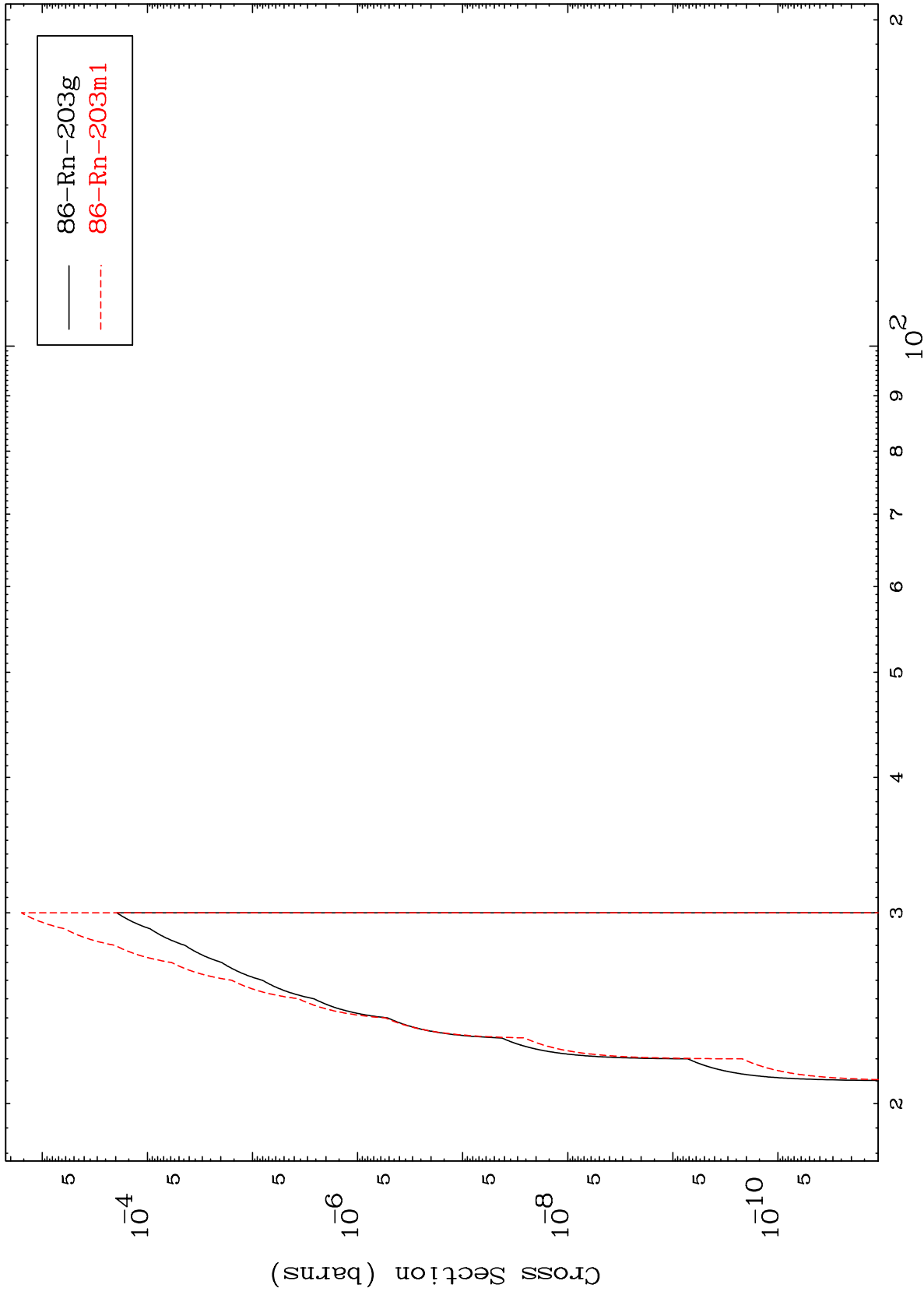
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MAT 8708

(n,3n) p

87-Fr-206

Radionuclide Production Cross Section



26

Incident Energy (MeV)

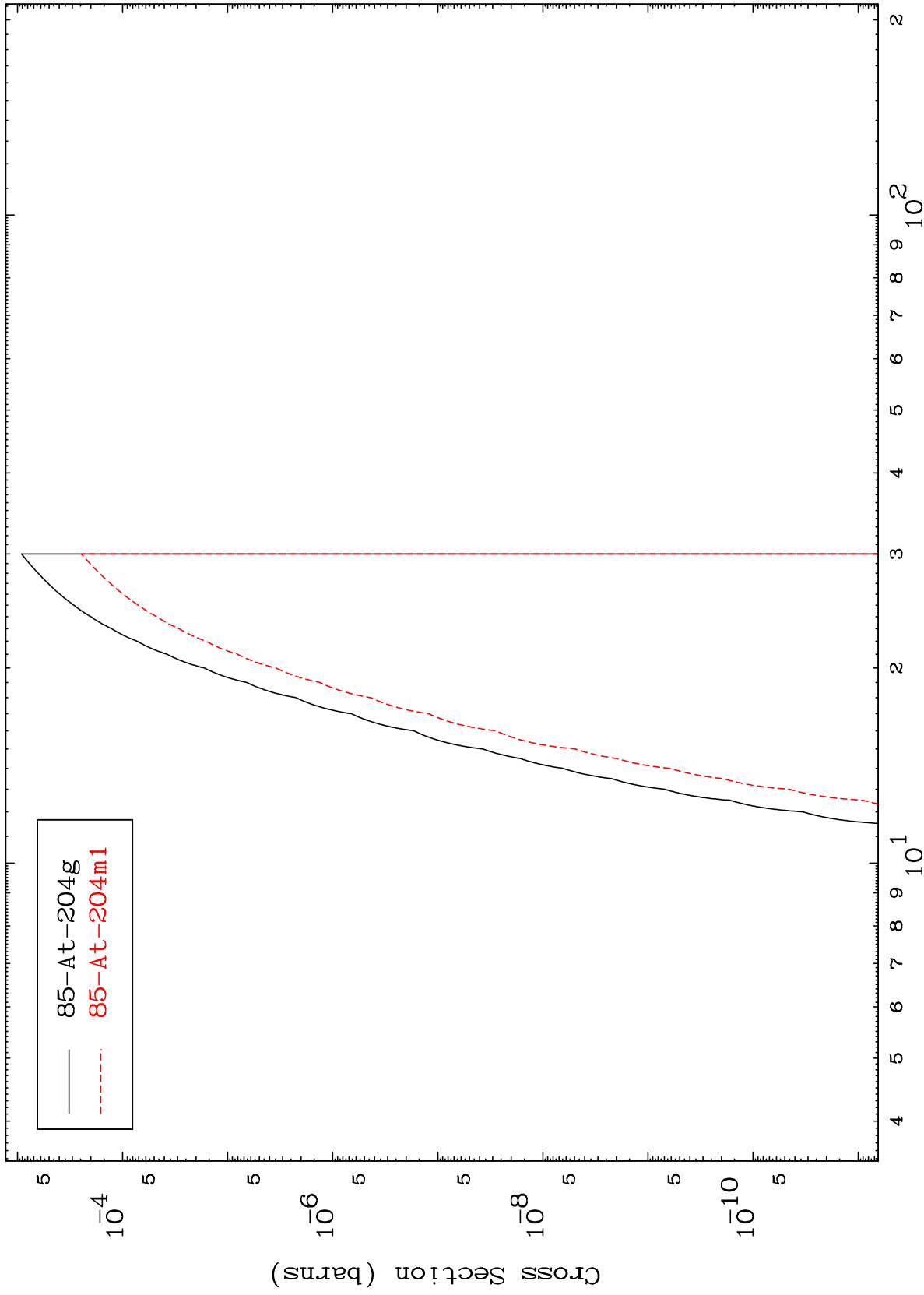
87-Fr-206

MAT 8708

(n,2n) p

87-Fr-206

Radionuclide Production Cross Section



27

Incident Energy (MeV)

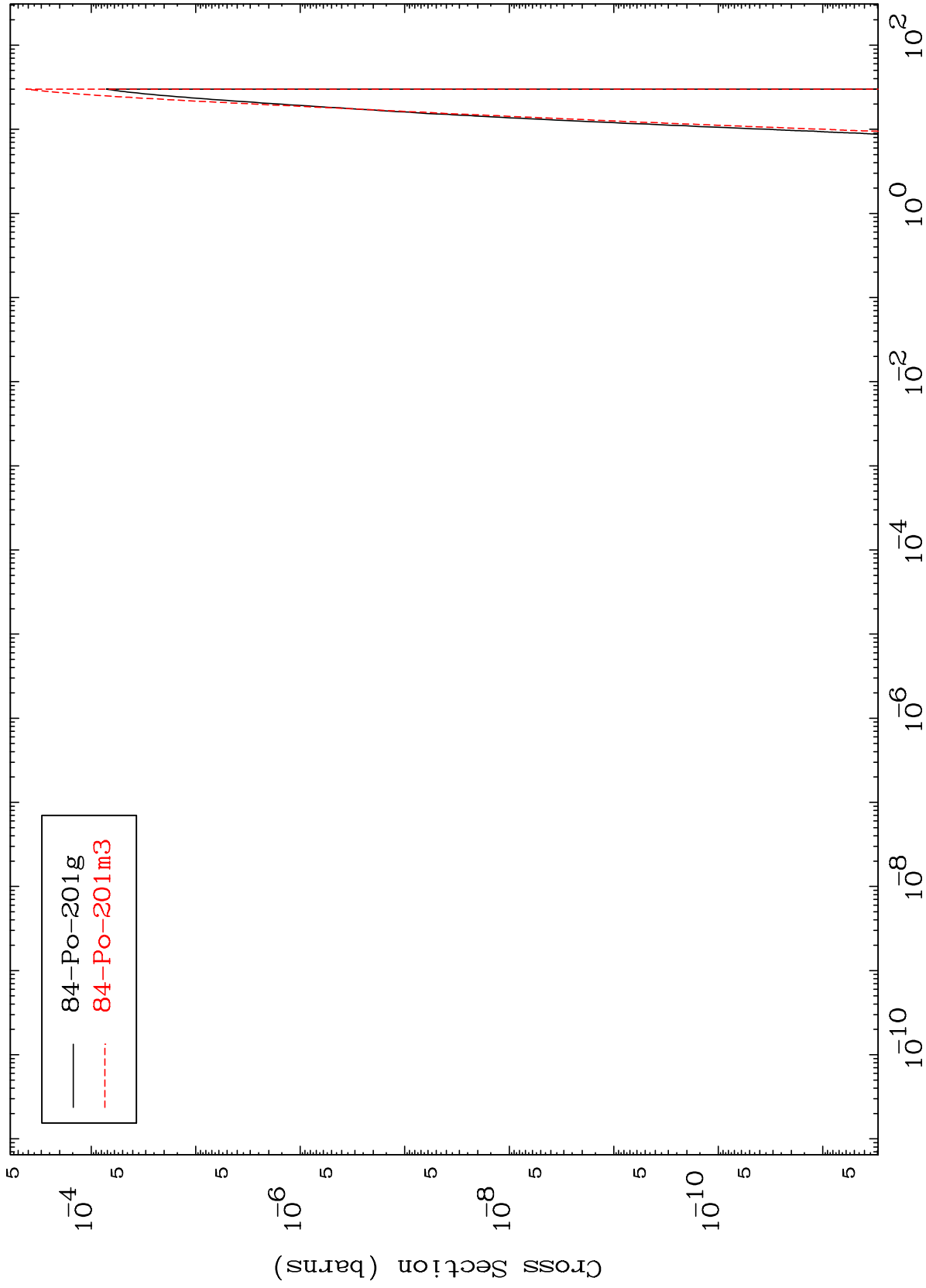
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MAT 8708

(n,n') p α

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Radionuclide Production Cross Section



28

Incident Energy (MeV)

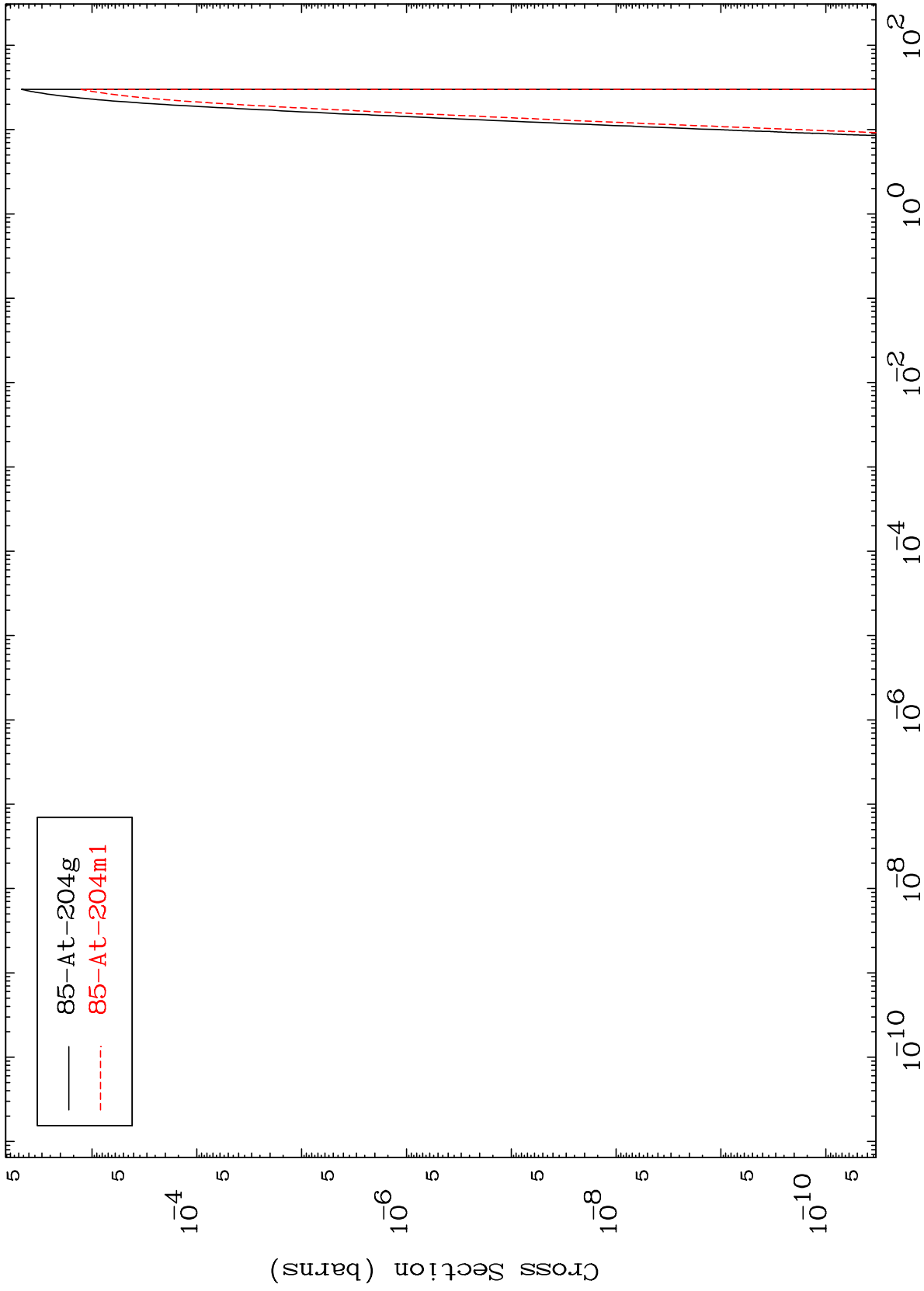
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(n, He-3)

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Radionuclide Production Cross Section



29

Incident Energy (MeV)

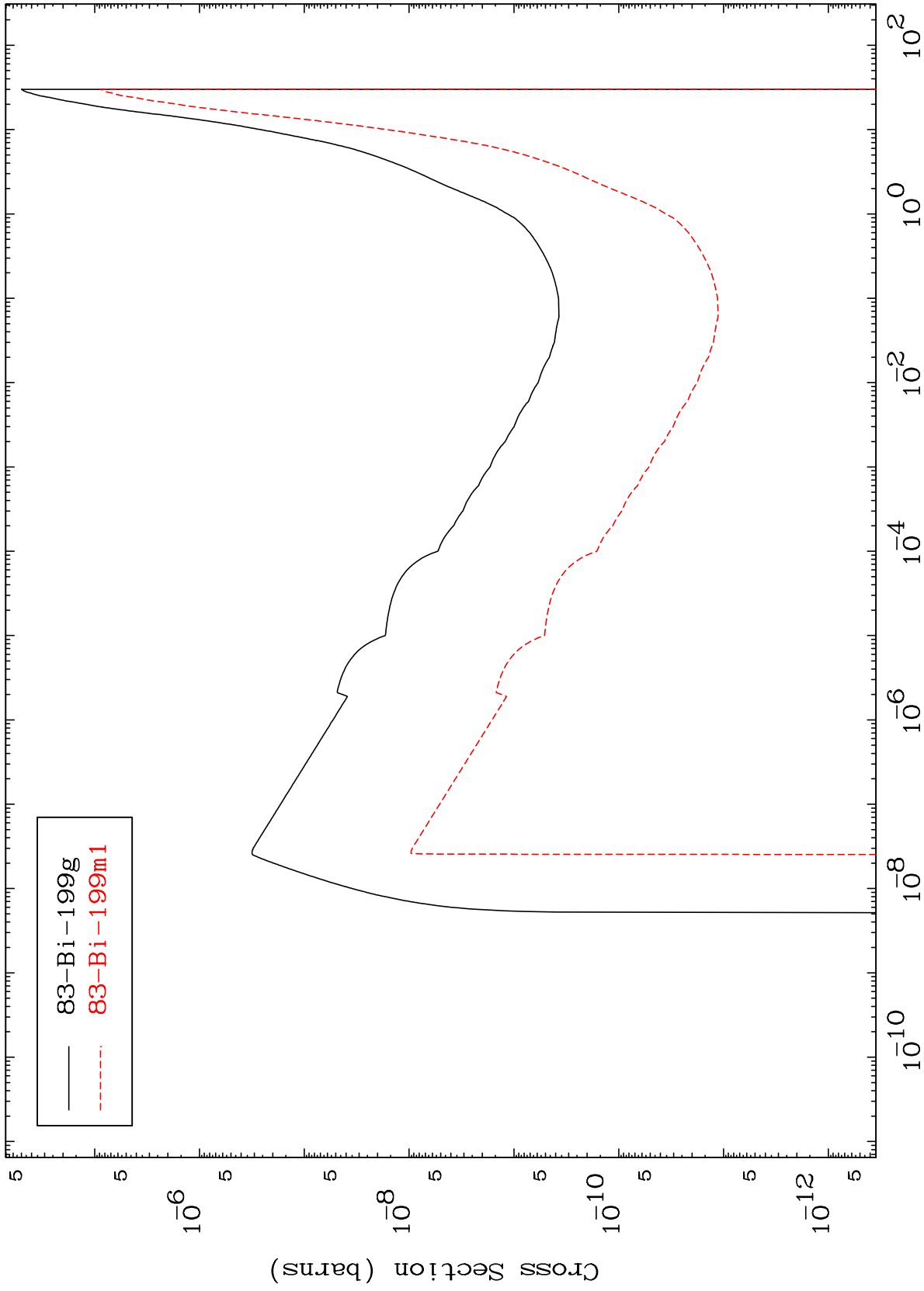
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(n,2α)

87-Fr-206

Radionuclide Production Cross Section



30

Incident Energy (MeV)

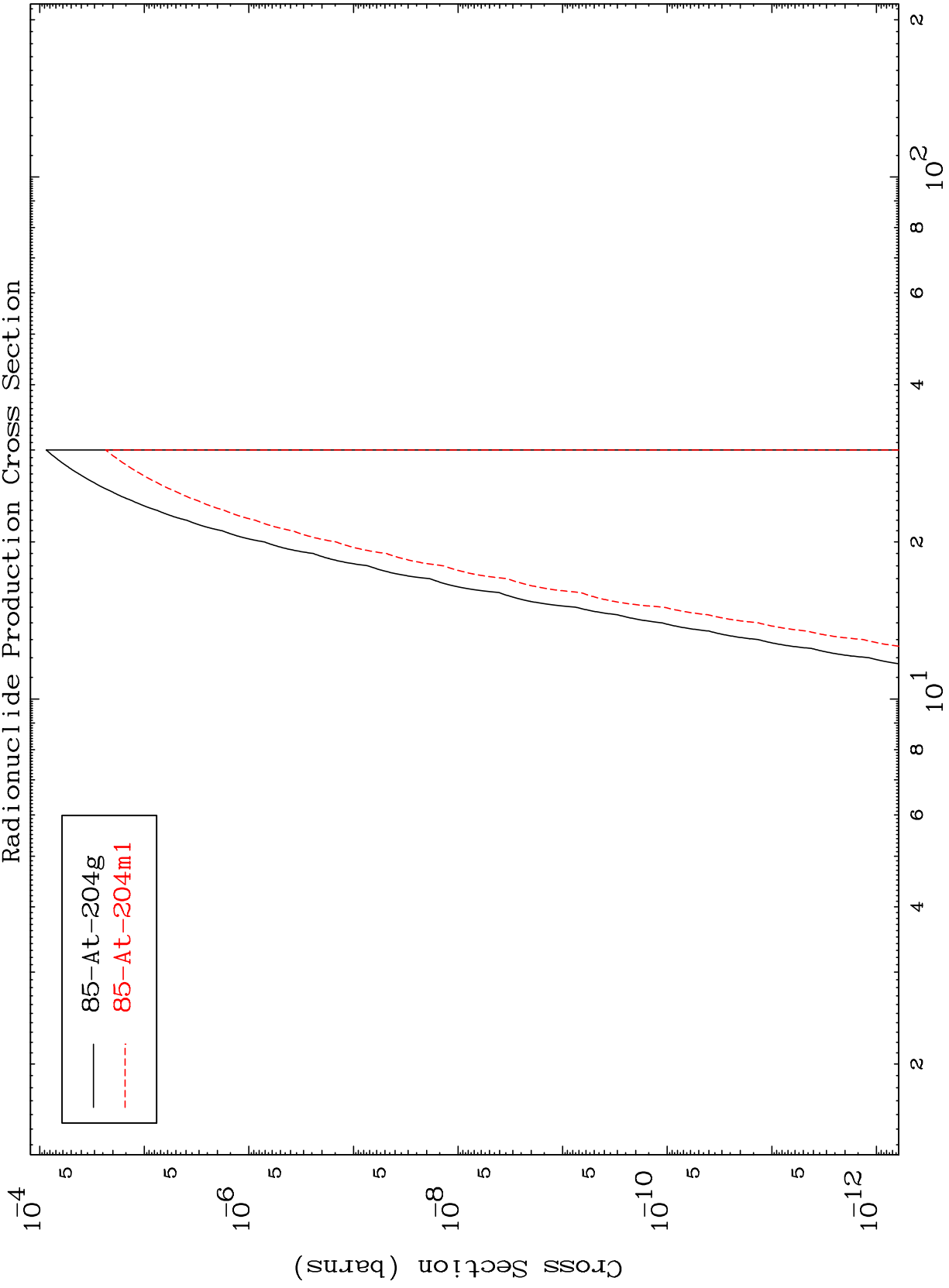
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MAT 8708

(n,p) d

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Radionuclide Production Cross Section



31

Incident Energy (MeV)

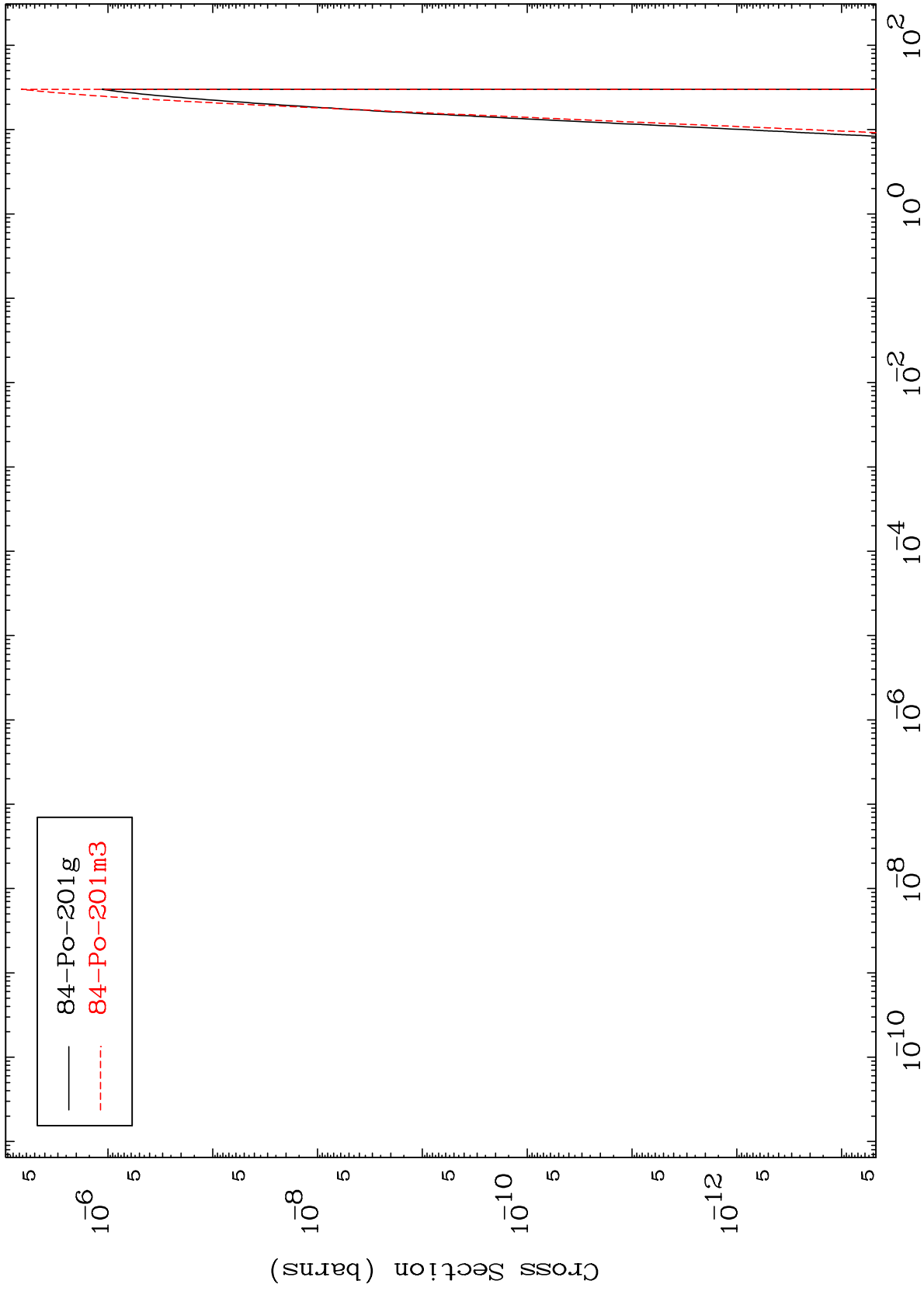
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MAT 8708

(n,d) α

87-Fr-206

Radionuclide Production Cross Section



32

Incident Energy (MeV)

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