

Program EVALPLOT  
(Version 2018-1)

by

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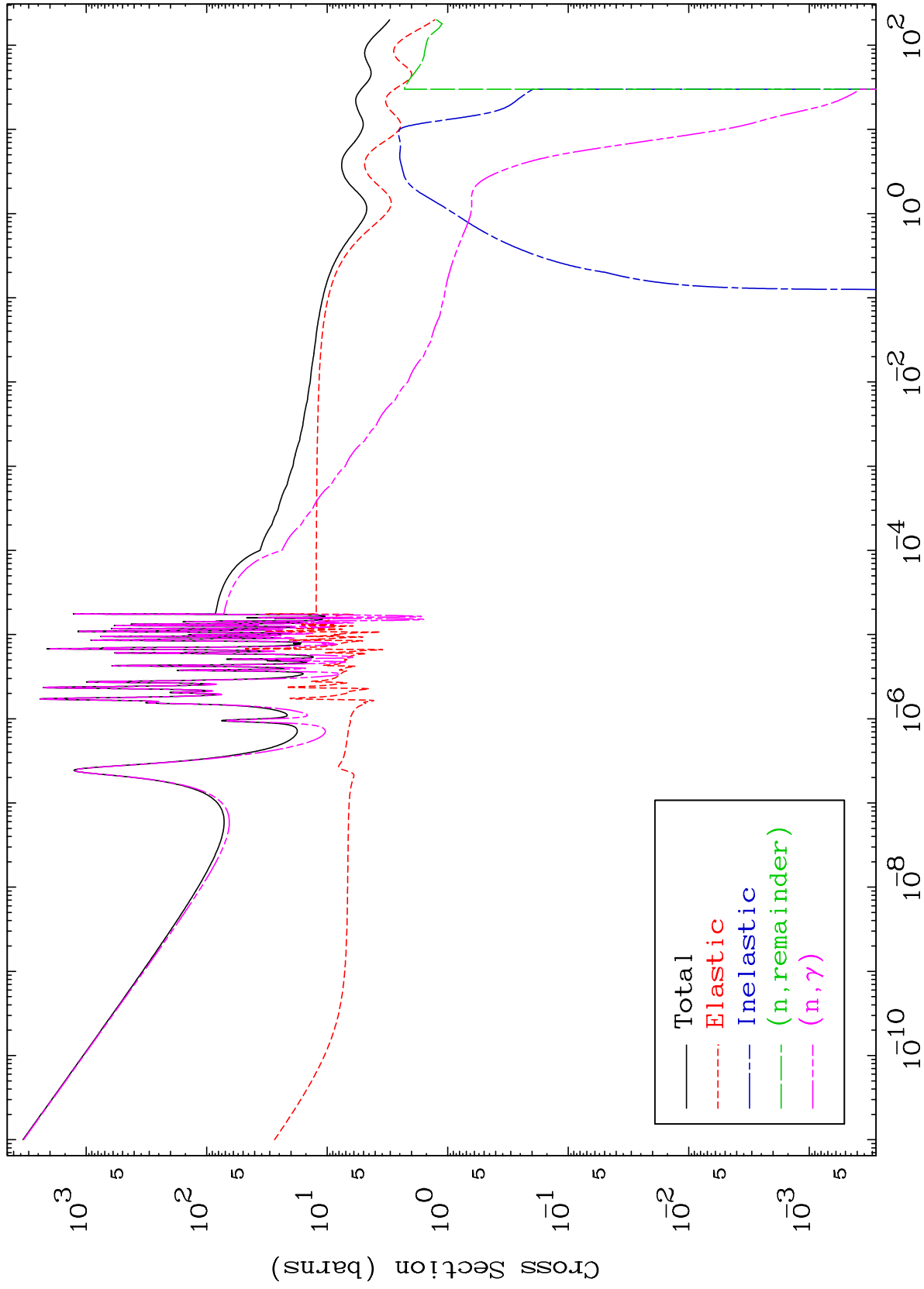
Press Mouse Button to Start

MAT 8716

Major

293 Kelvin Cross Sections

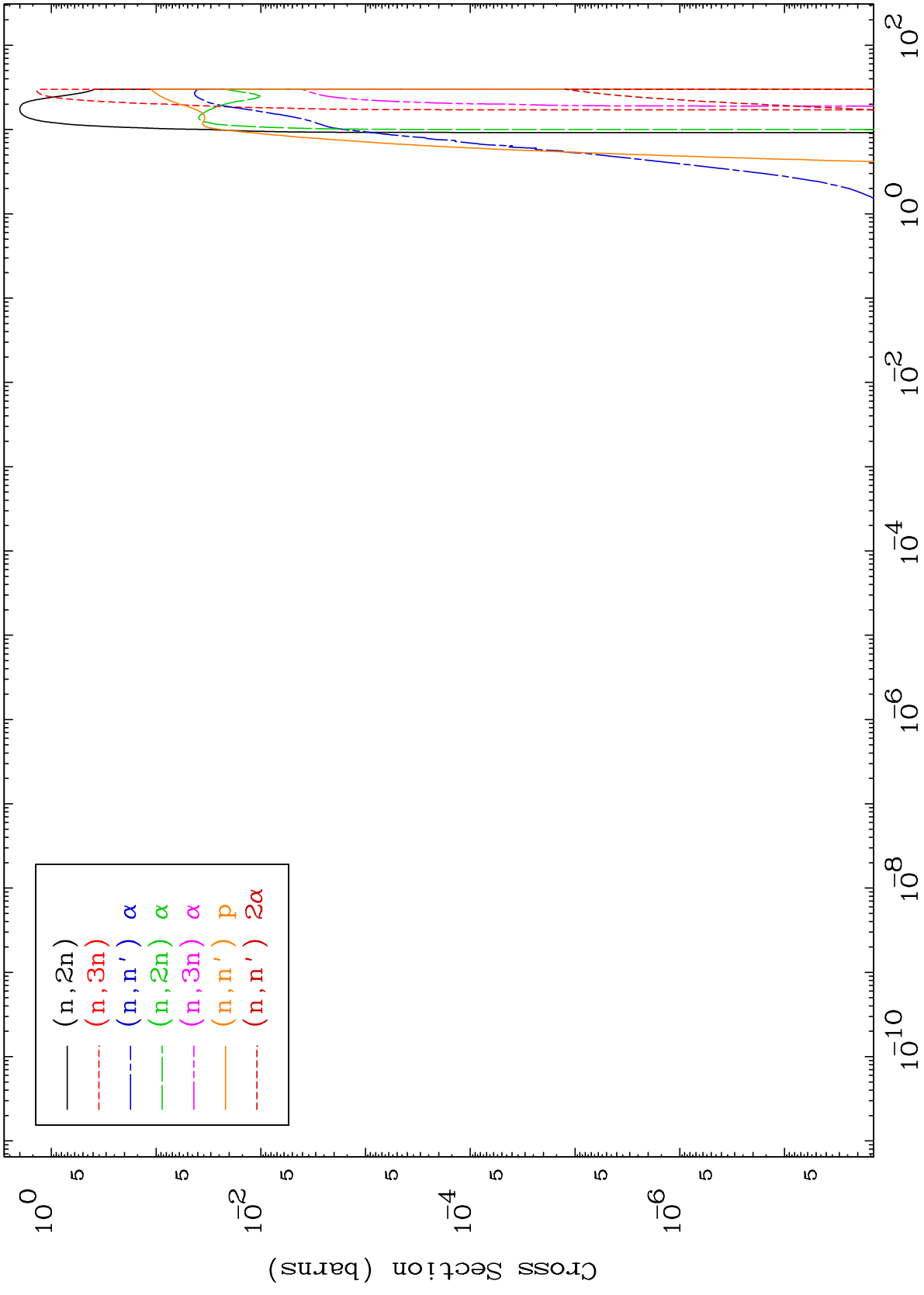
87-Fr-209



MAT 8716

Neutron Production  
293 Kelvin Cross Sections

87-Fr-209

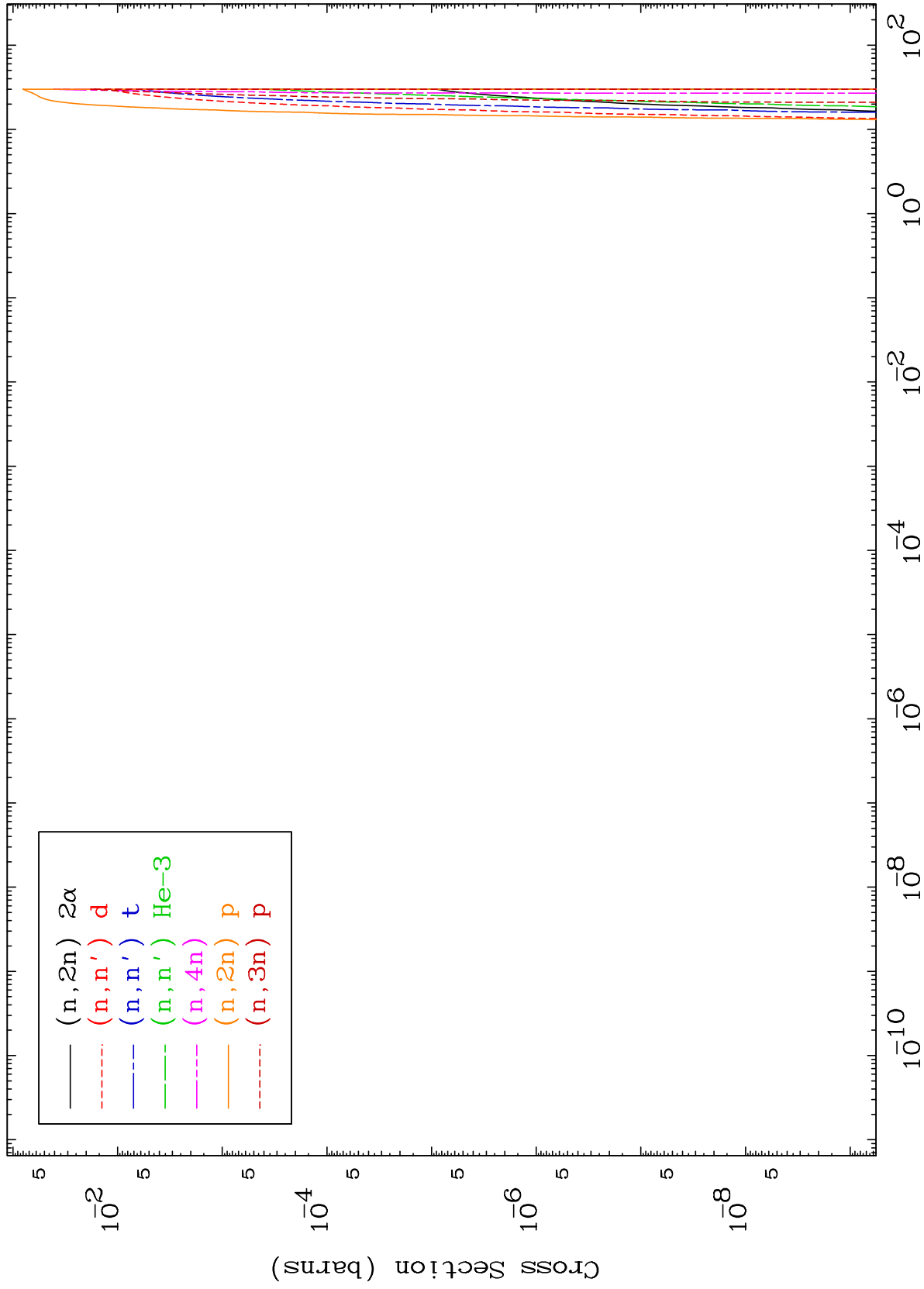


87-Fr-209

MAT 8716

Neutron Production  
293 Kelvin Cross Sections

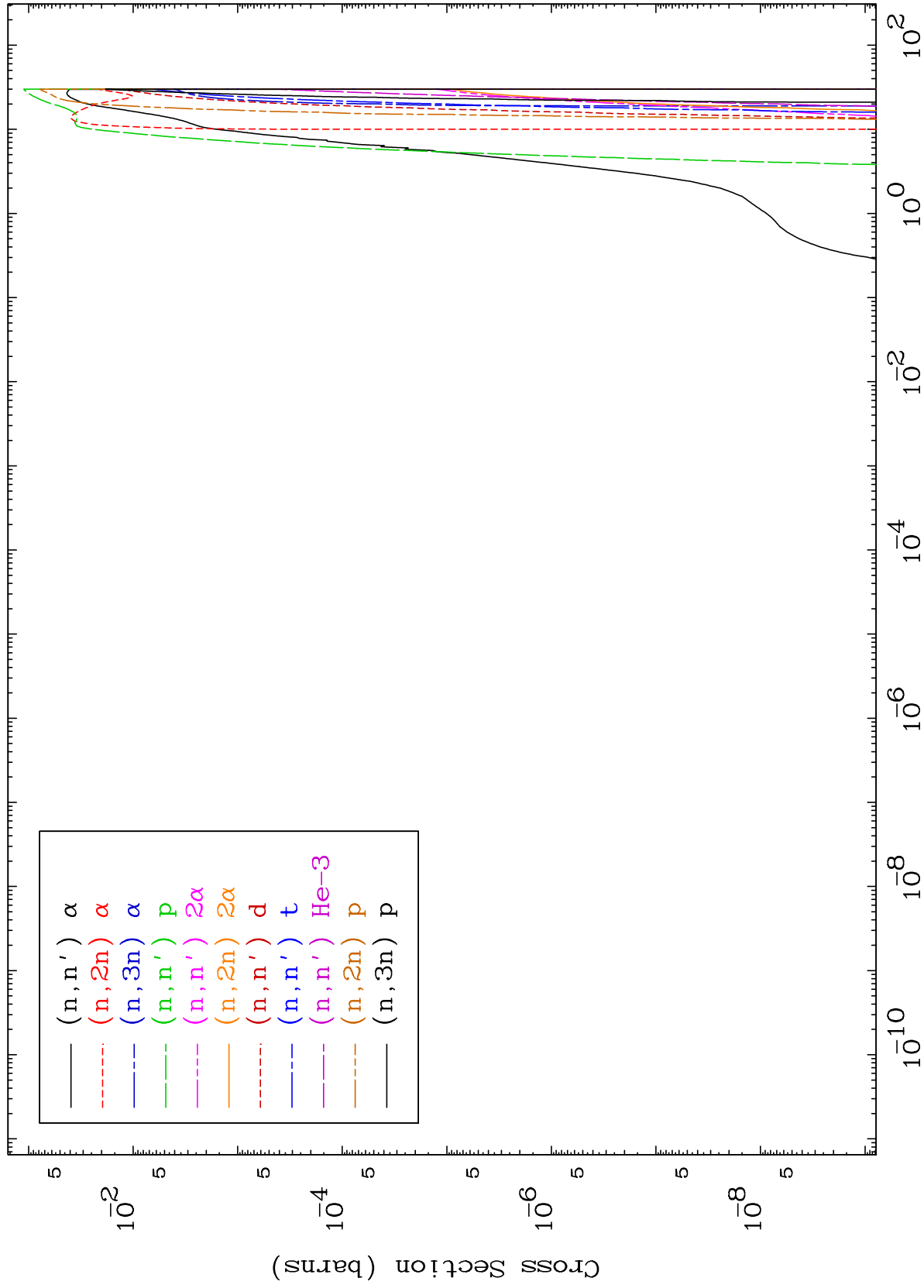
87-Fr-209



MAT 8716

293 Kelvin Cross Sections  
Charged Particle

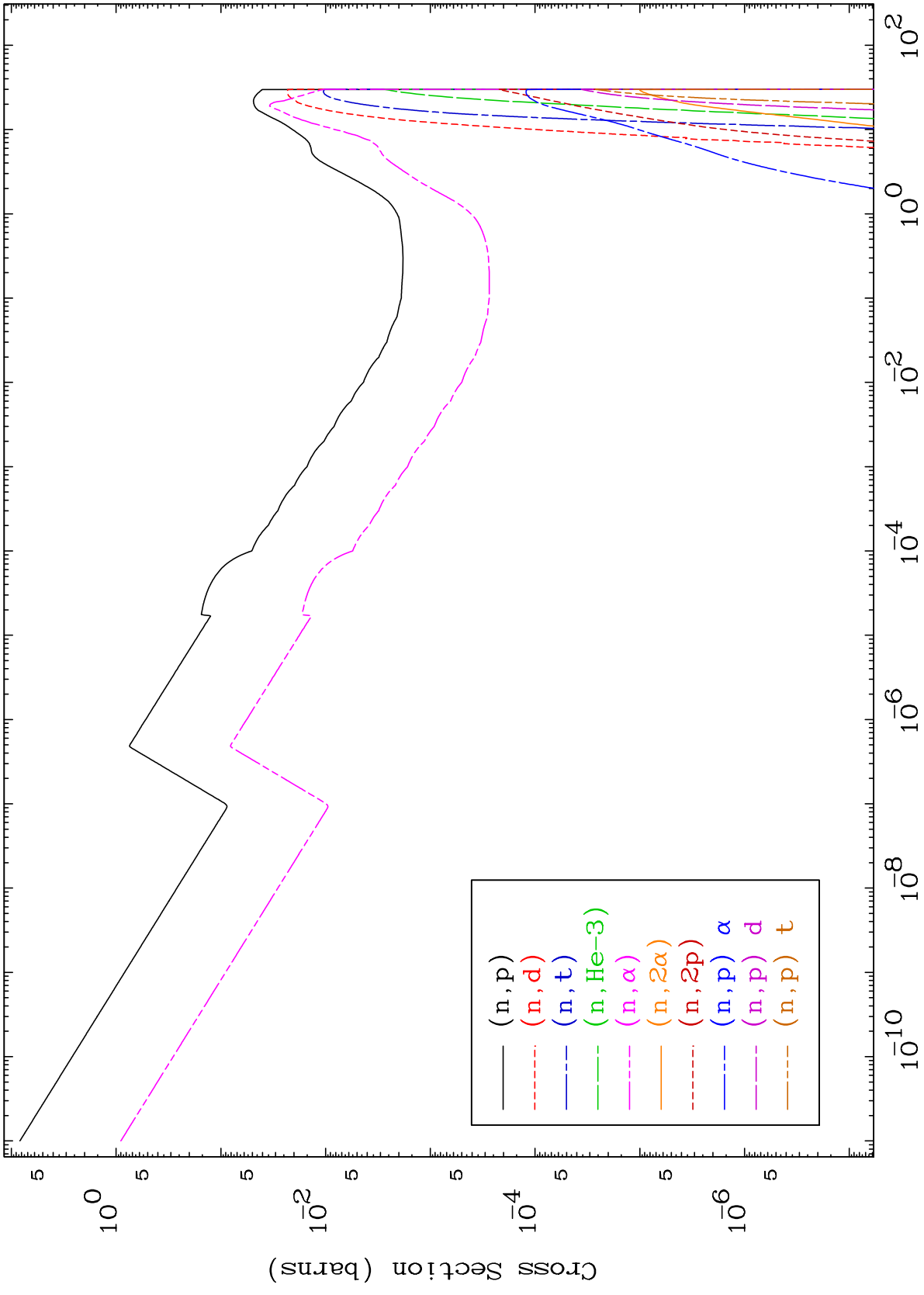
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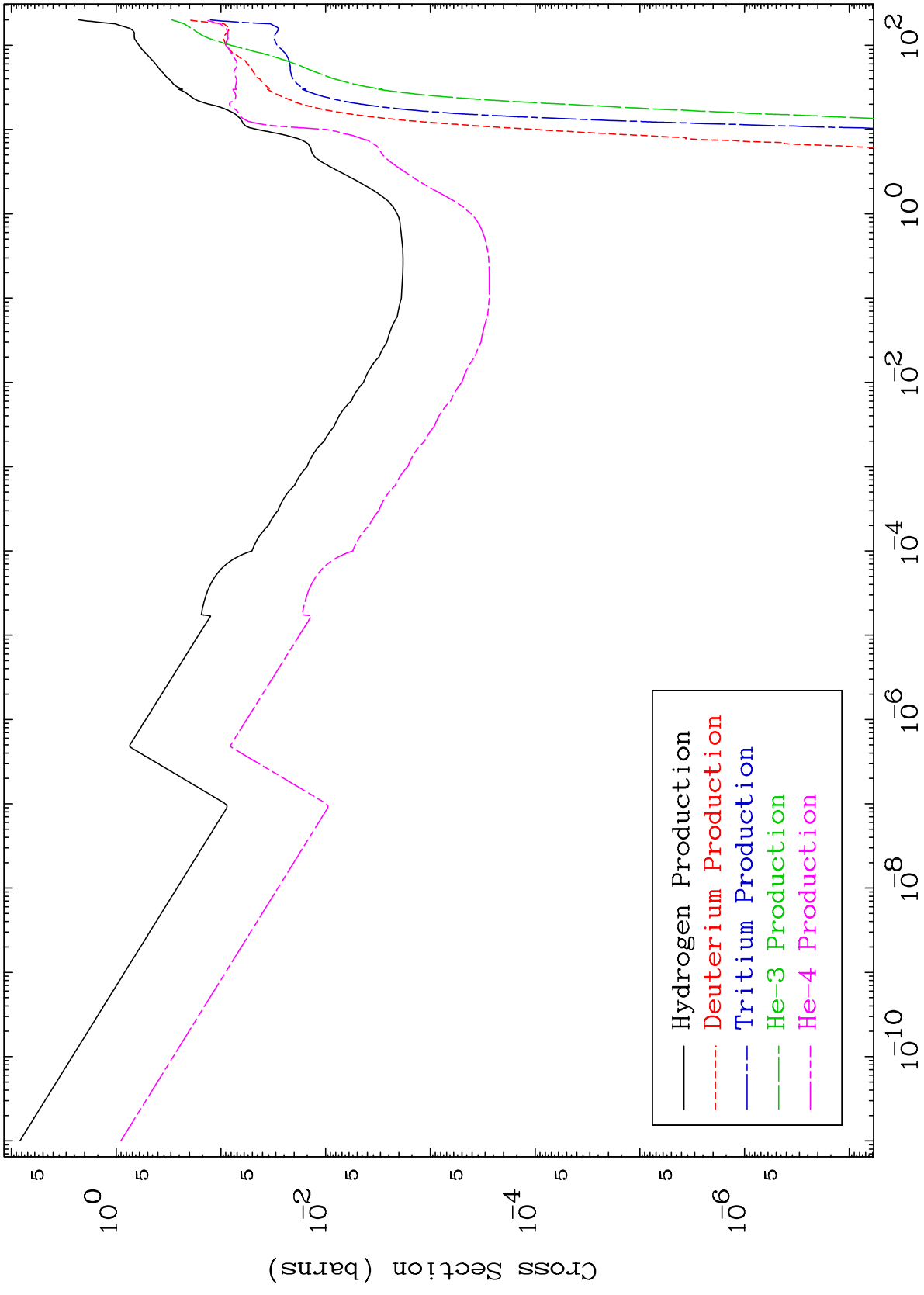


MAT 8716

Charged Particle  
293 Kelvin Cross Sections

87-Fr-209

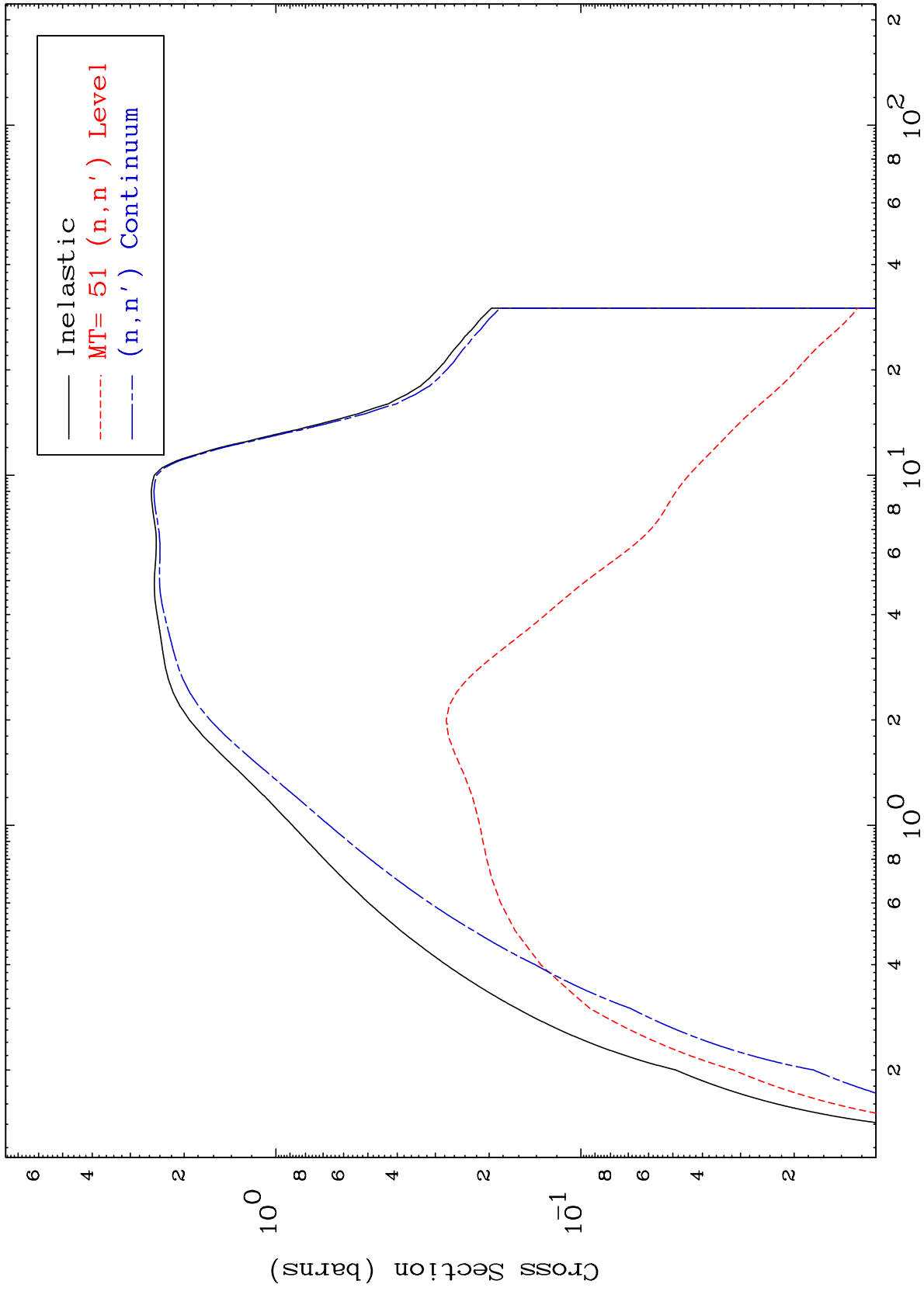




MAT 8716

(n,n') Level  
293 Kelvin Cross Sections

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Incident Energy (MeV)

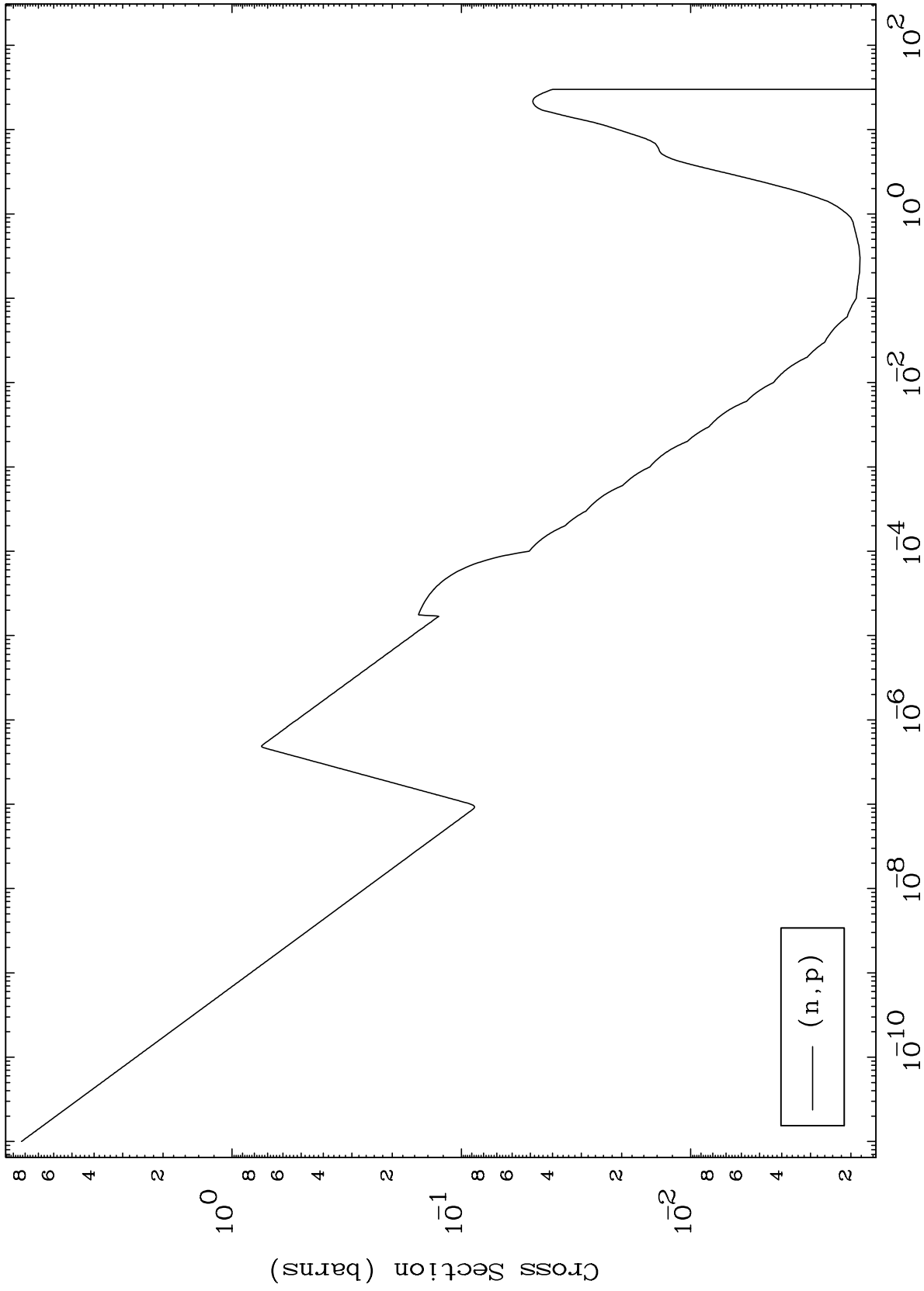
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MAT 8716

(n,p) Levels  
293 Kelvin Cross Sections

87-Fr-209



8

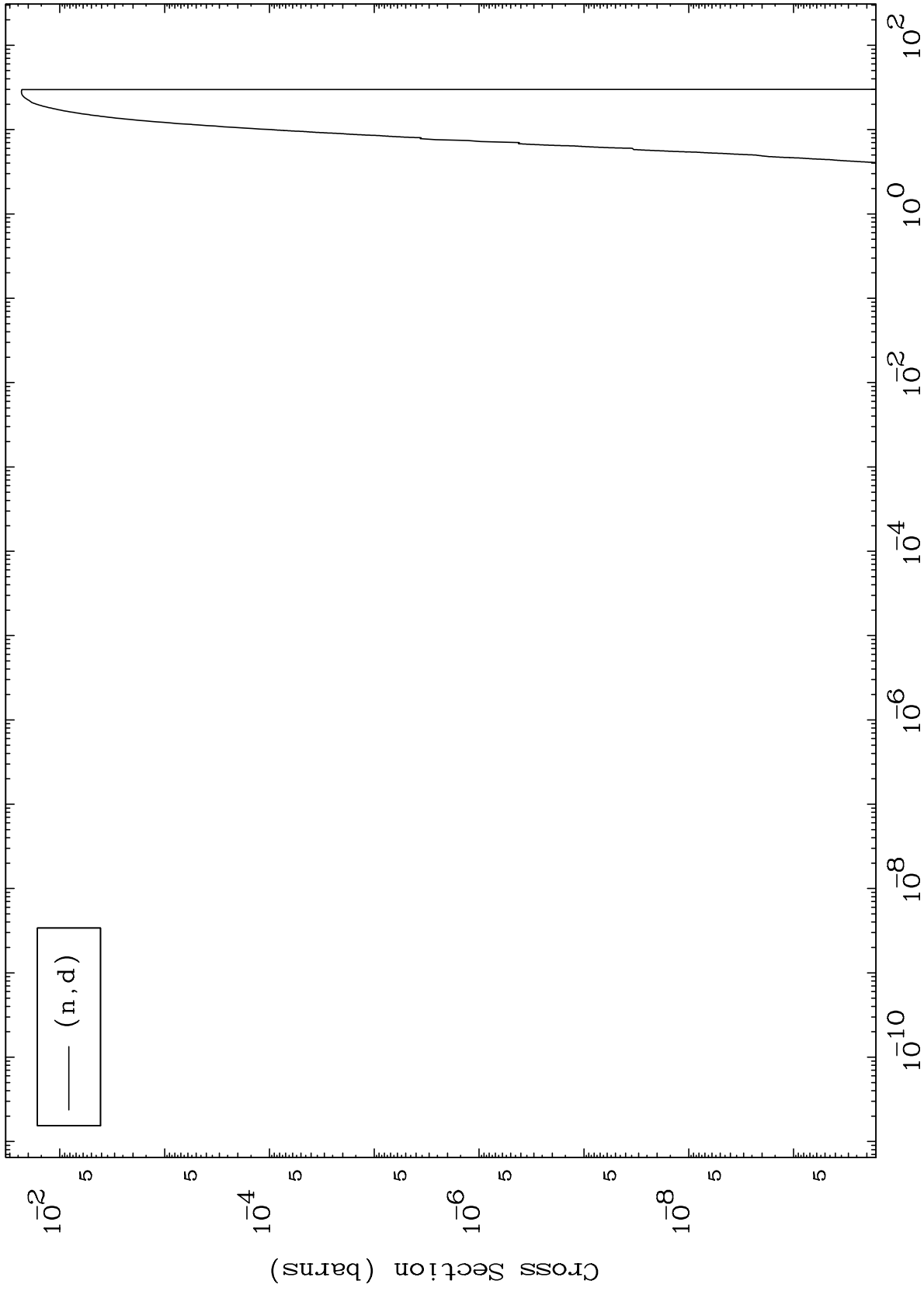
Incident Energy (MeV)

87-Fr-209

MAT 8716

(n,d) Levels  
293 Kelvin Cross Sections

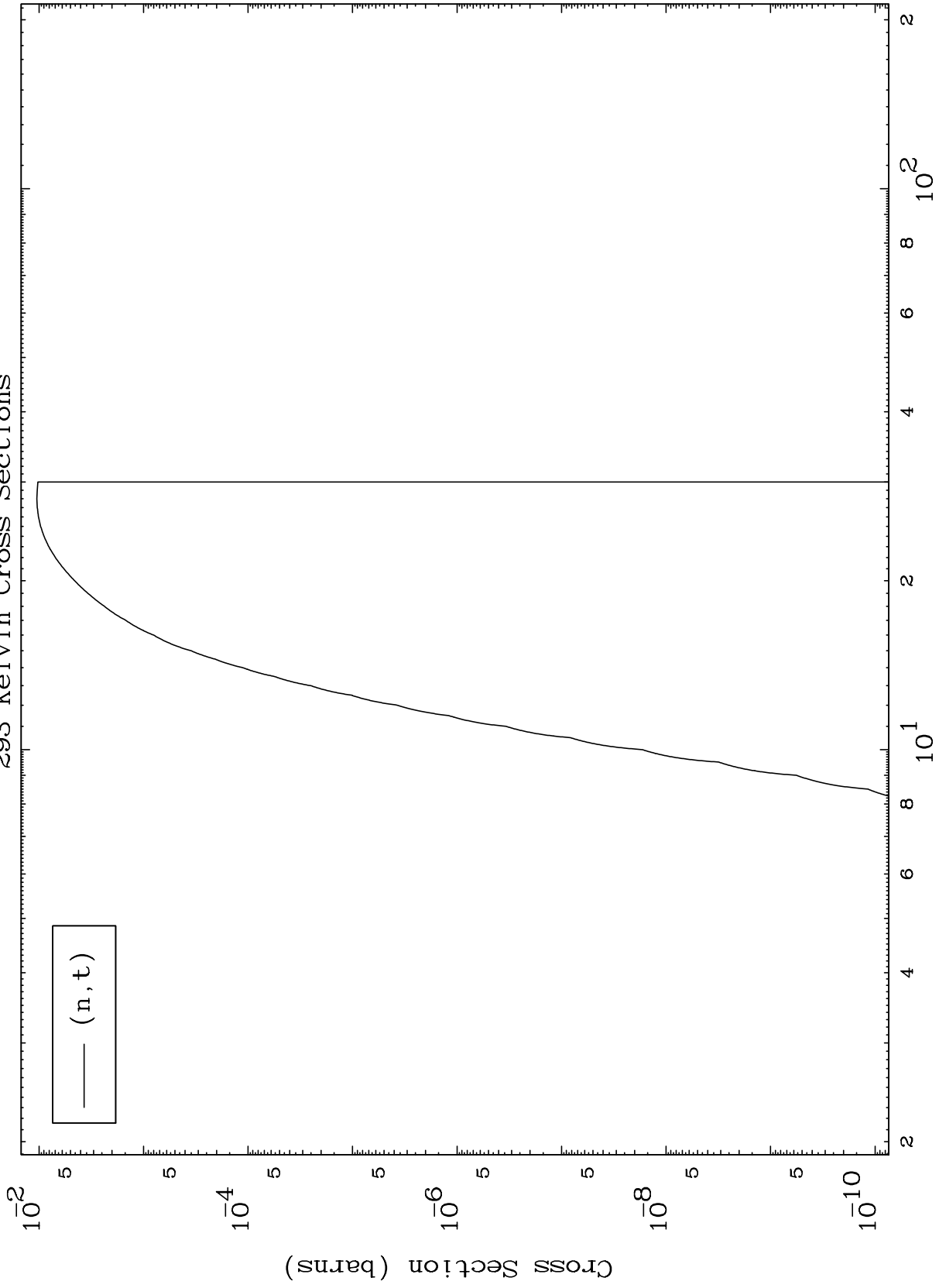
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MAT 8716

(n,t) Levels  
293 Kelvin Cross Sections

87-Fr-209



10

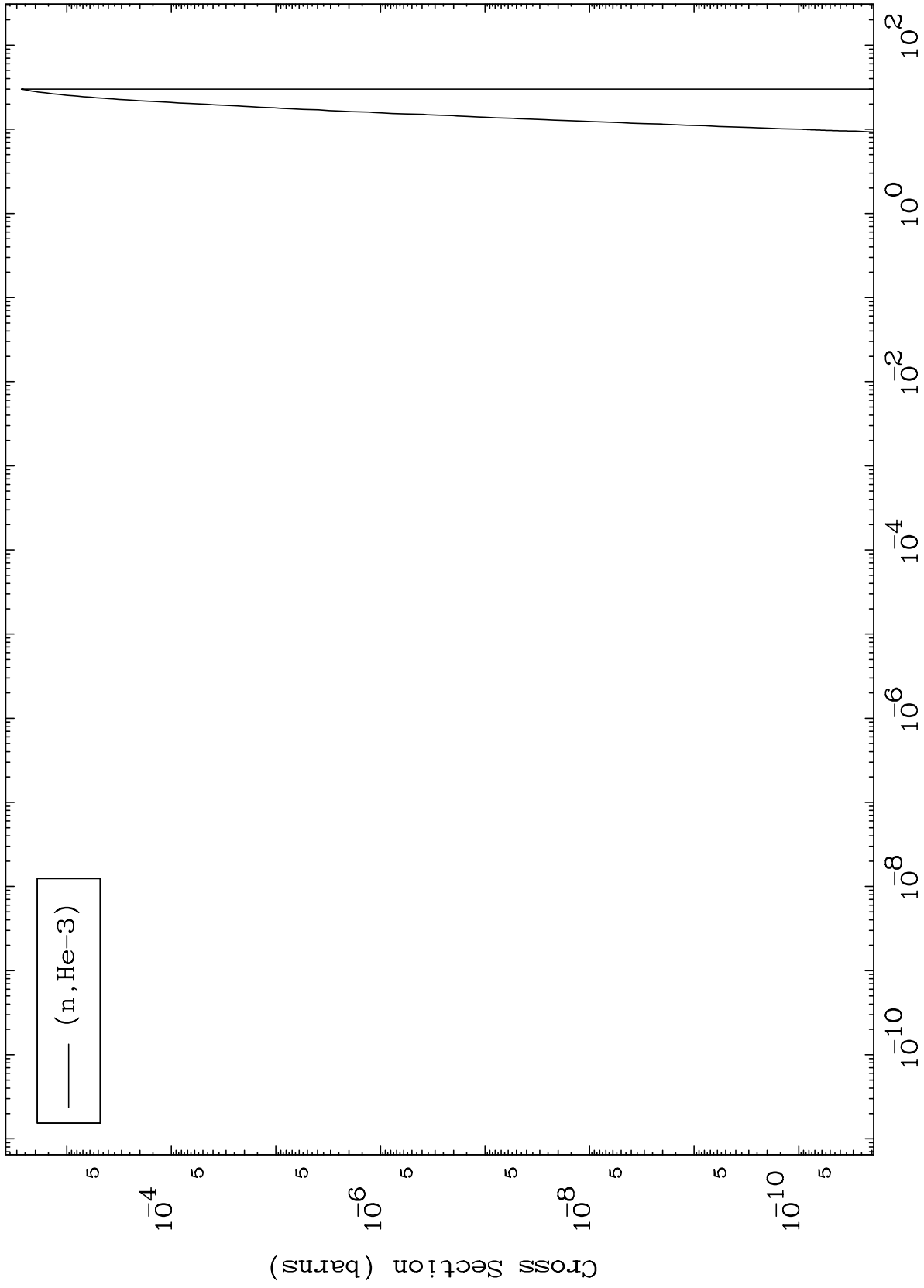
Incident Energy (MeV)

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MAT 8716

(n,He3) Levels  
293 Kelvin Cross Sections

87-Fr-209



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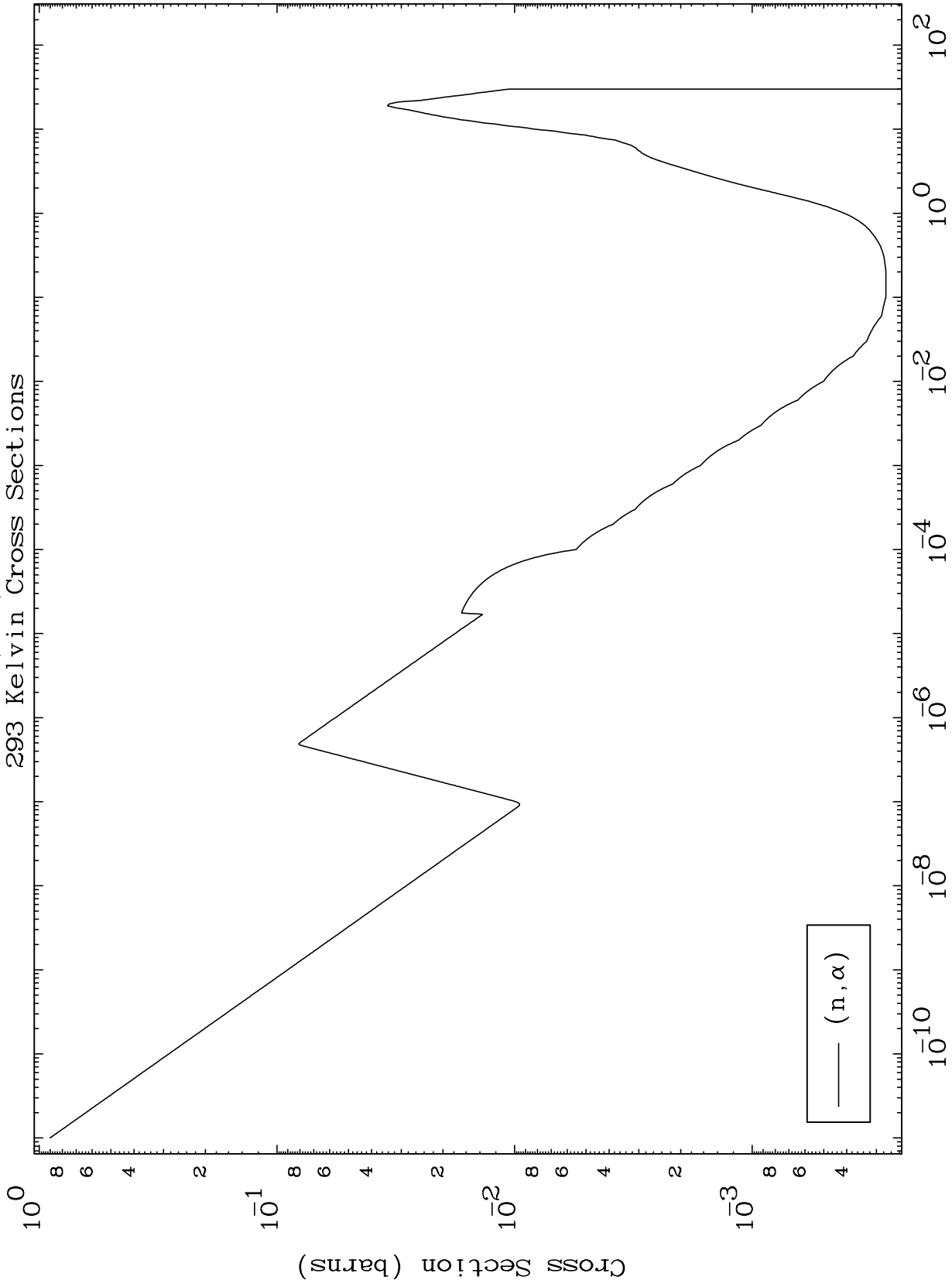
Incident Energy (MeV)

87-Fr-209

MAT 8716

(n, $\alpha$ ) Levels  
293 Kelvin Cross Sections

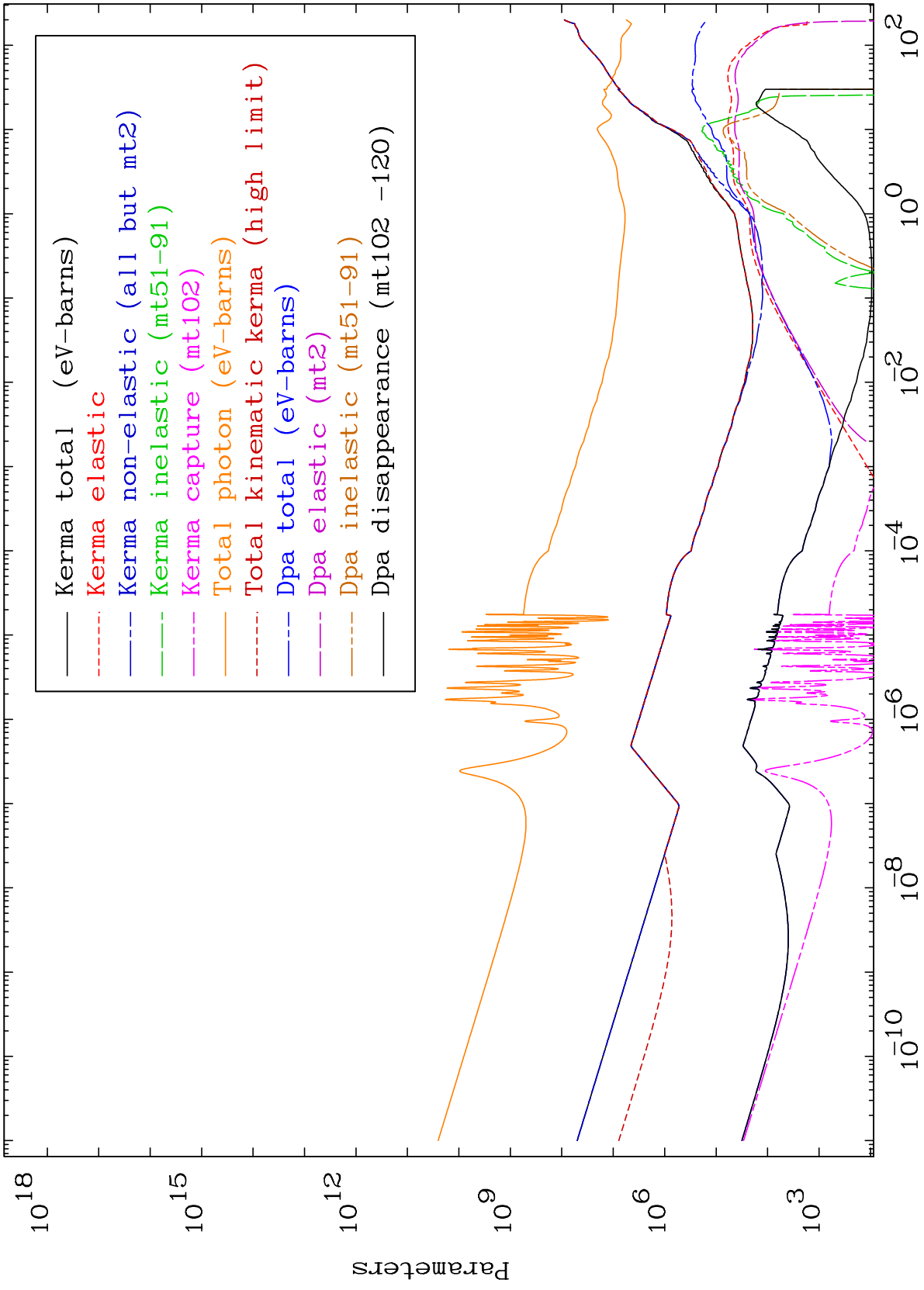
87-Fr-209



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Incident Energy (MeV)

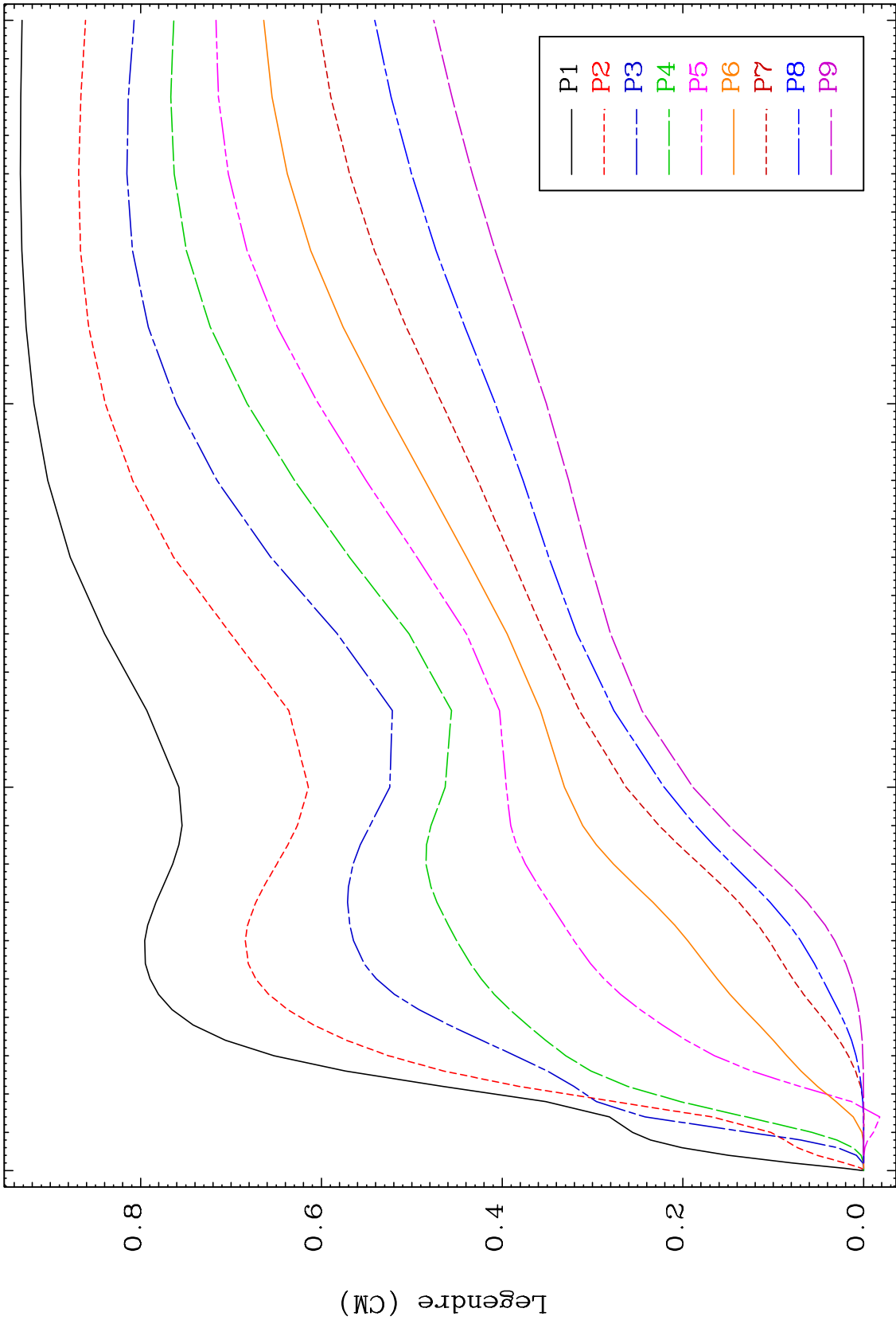
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Elastic Legendre Coefficients



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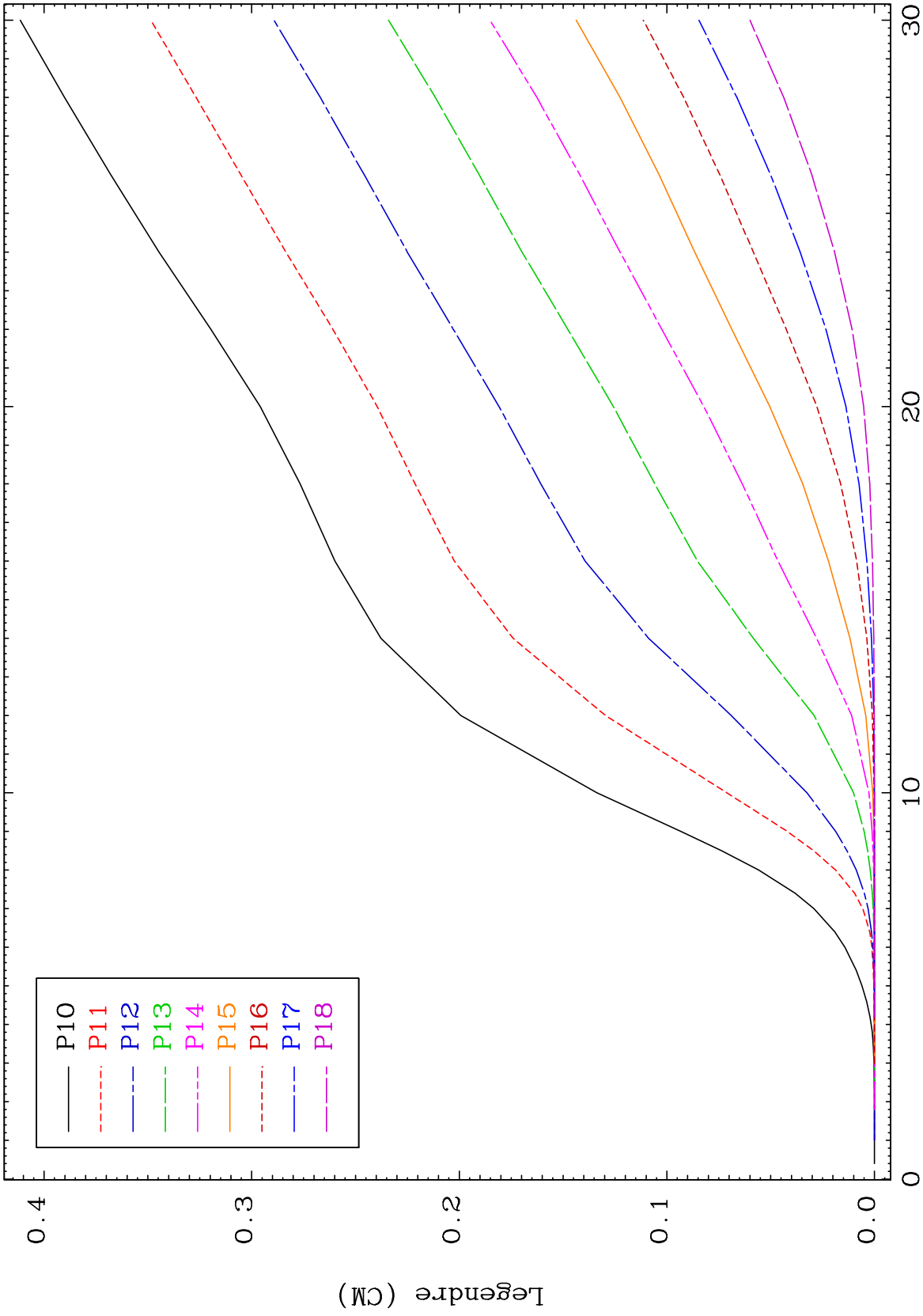
Incident Energy (MeV)

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MAT 8716

Elastic Legendre Coefficients

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15

10

20

30

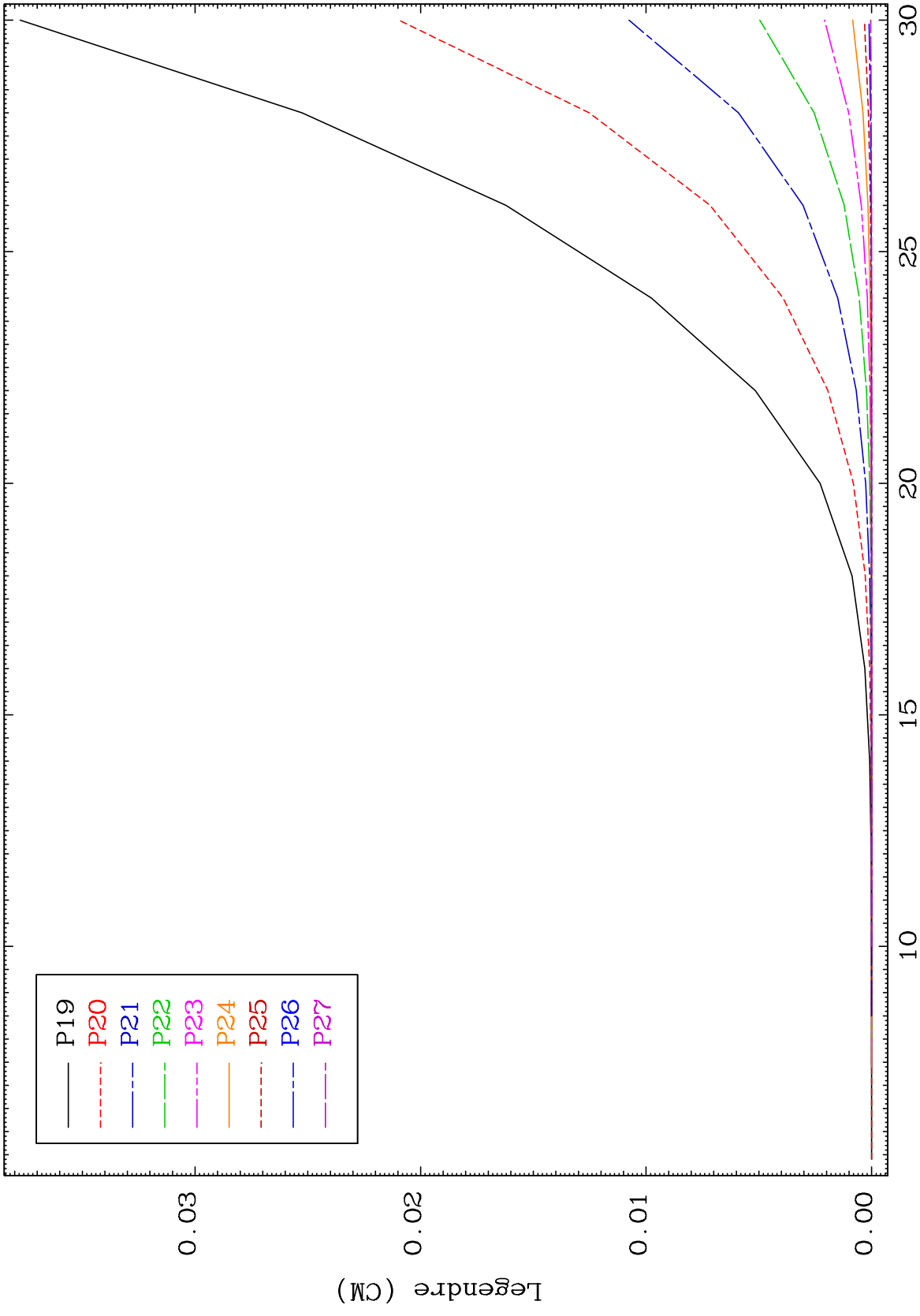
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Elastic Legendre Coefficients

87-Fr-209



16

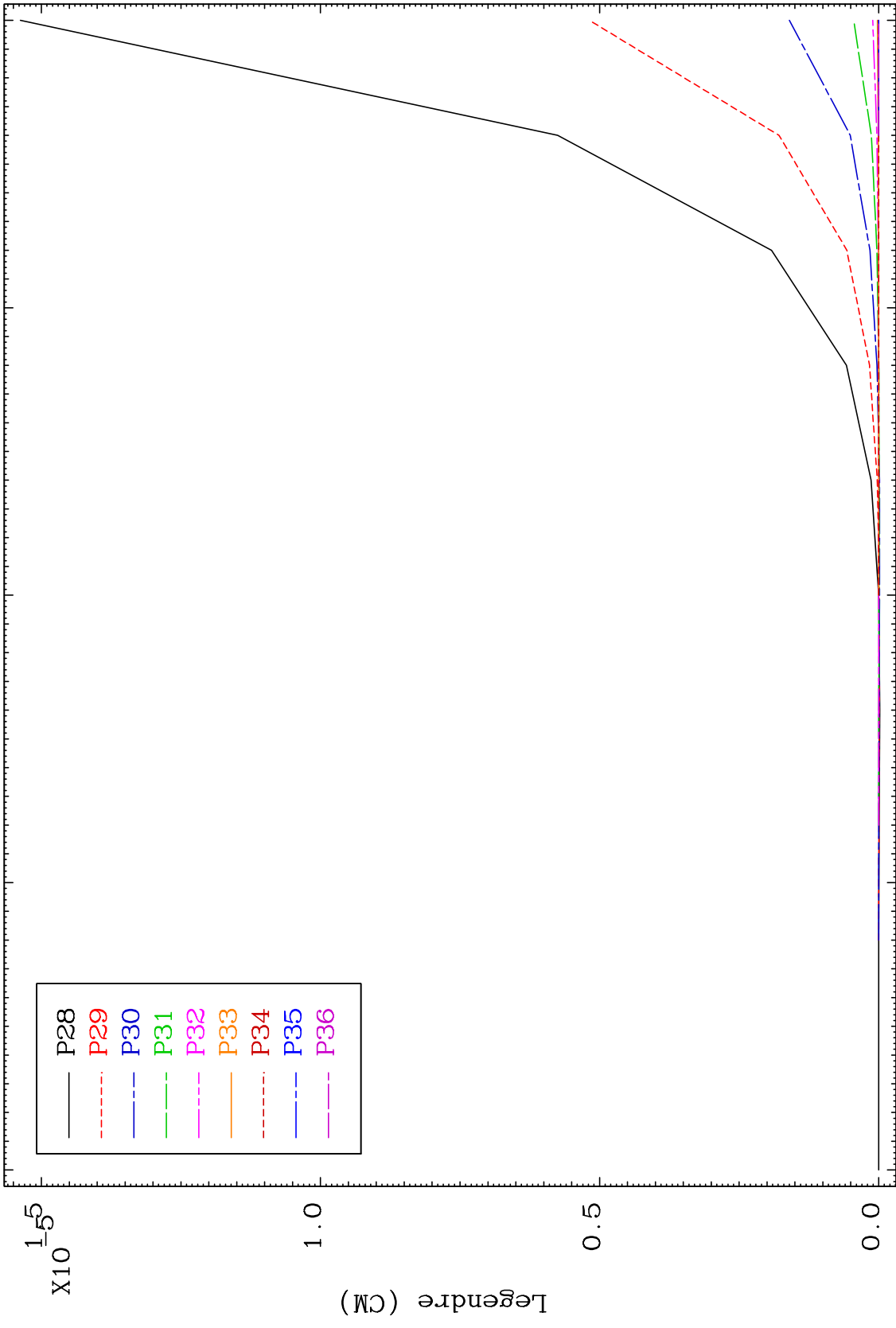
Incident Energy (MeV)

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MAT 8716

Elastic Legendre Coefficients

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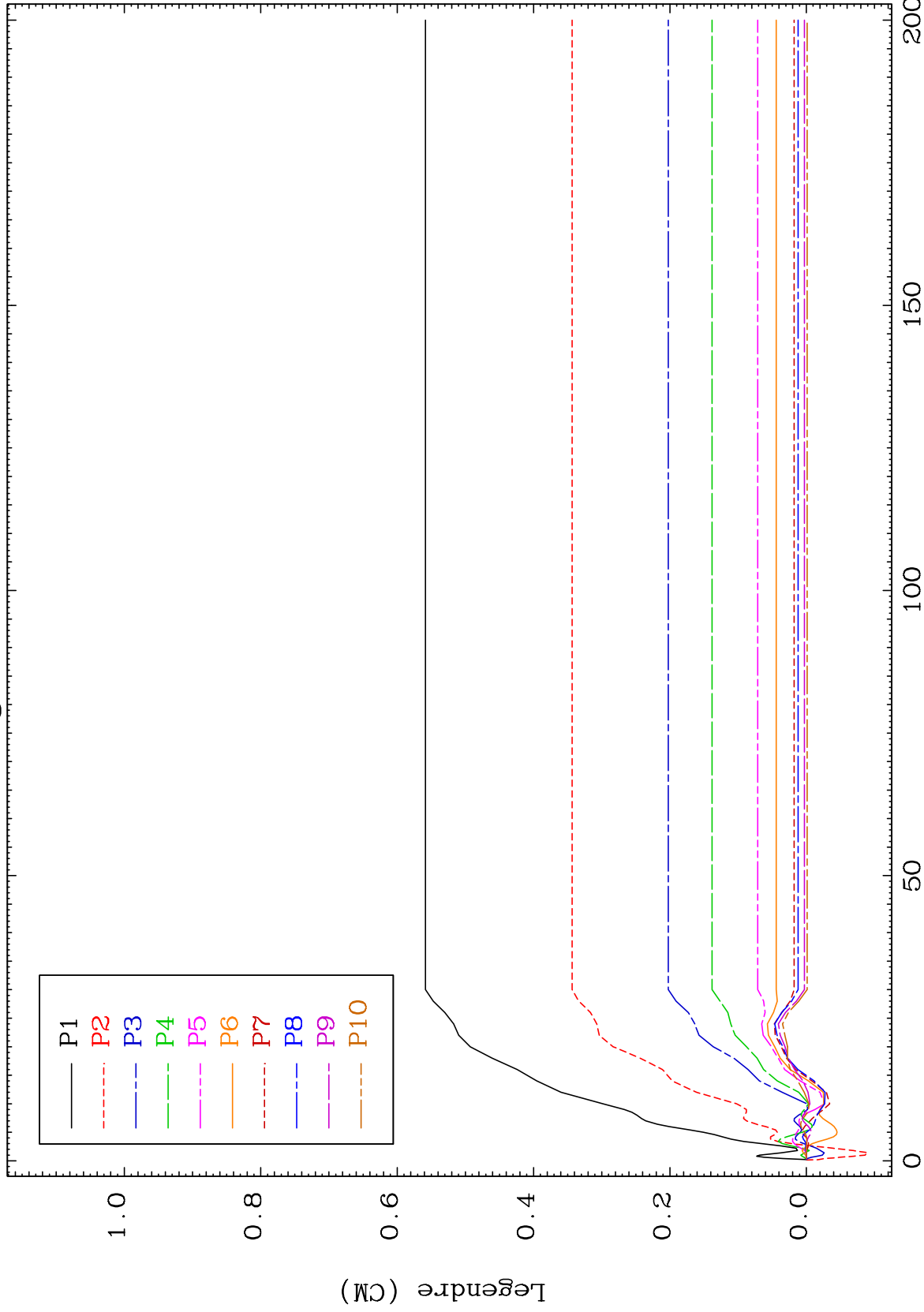
Incident Energy (MeV)

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MT= 51 (n,n') Level  
Legendre Coefficients

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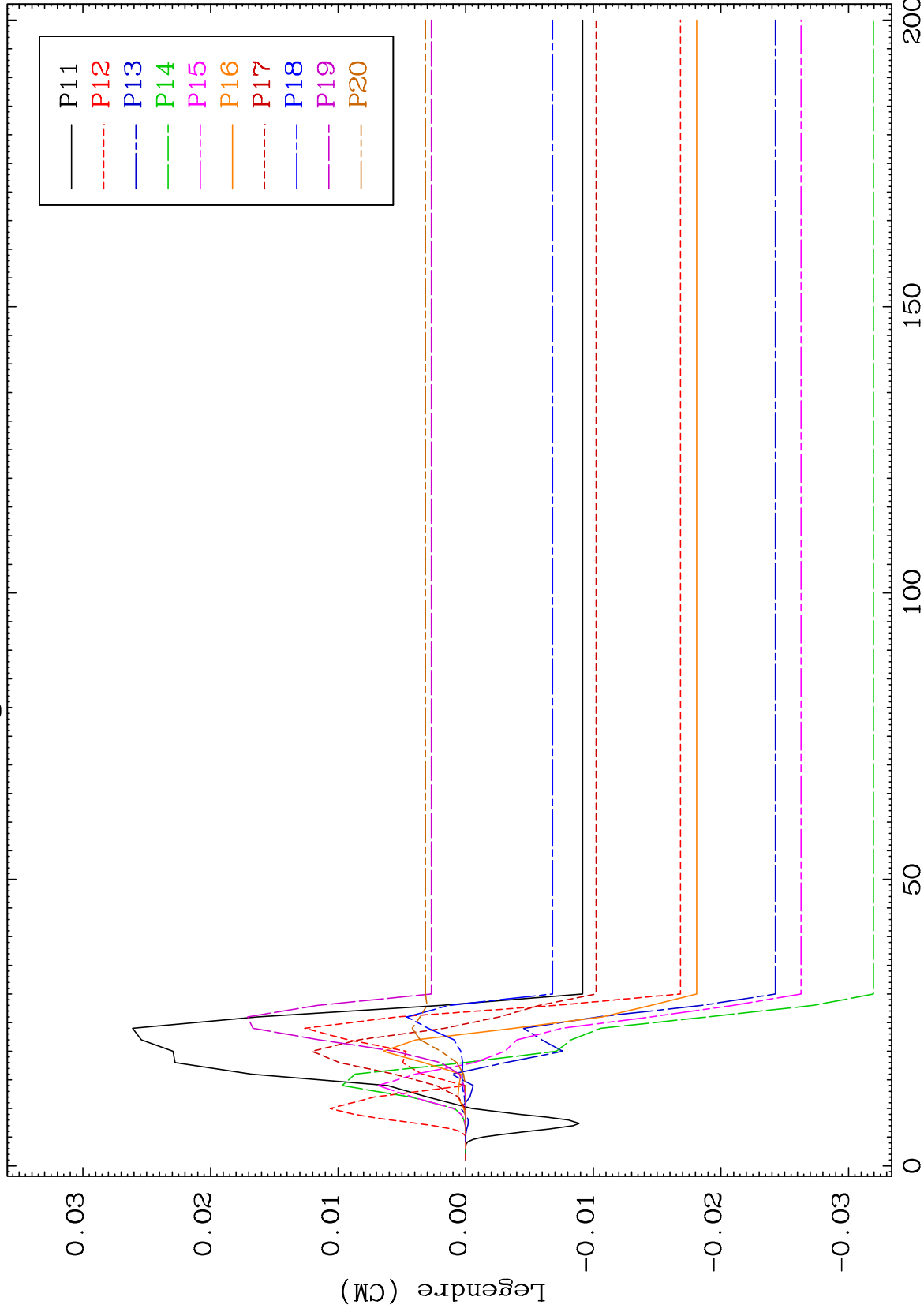
Incident Energy (MeV)

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MAT 8716

MT= 51 (n,n') Level  
Legendre Coefficients

87-Fr-209



19

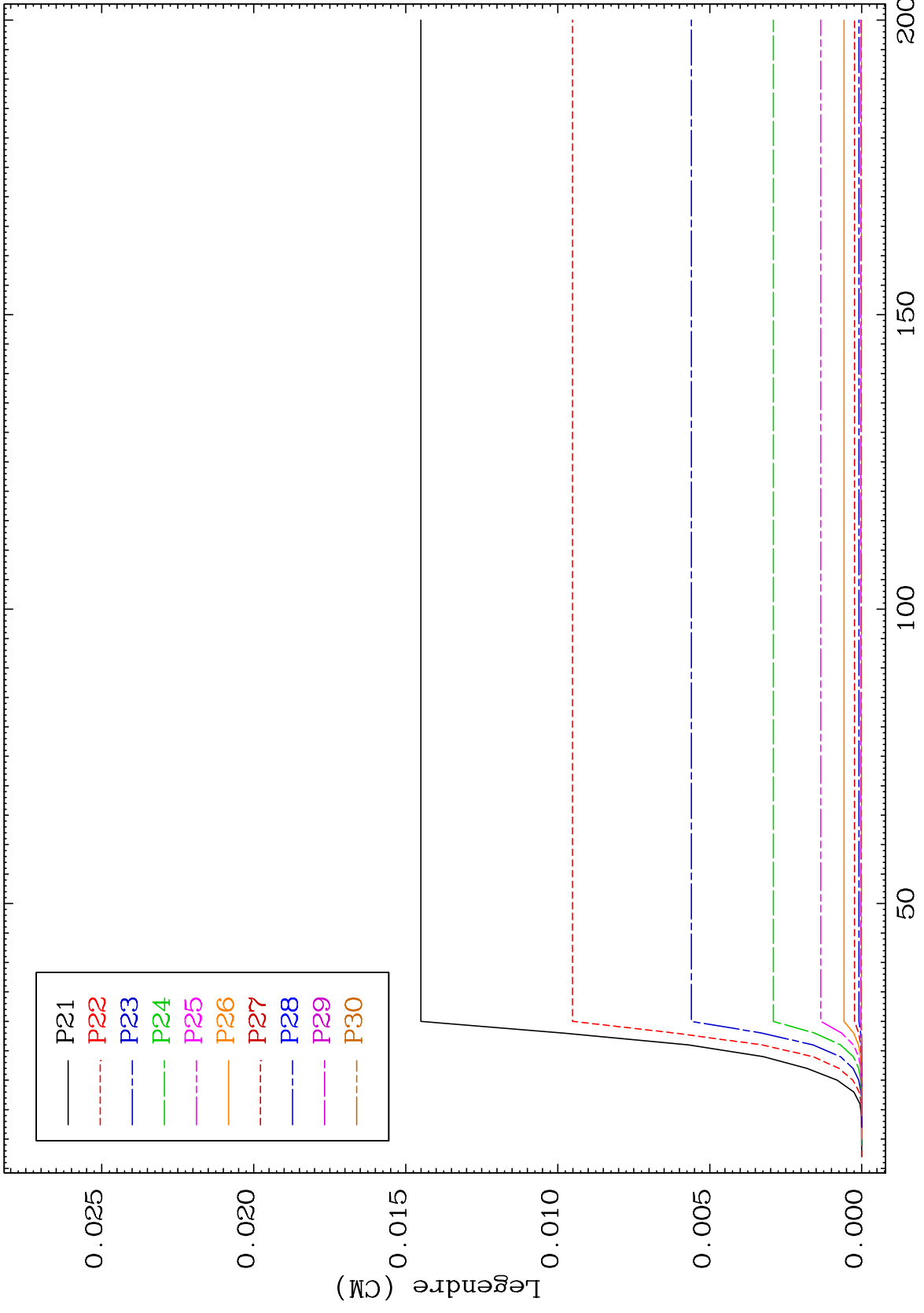
Incident Energy (MeV)

87-Fr-209

MAT 8716

MT= 51 (n,n') Level  
Legendre Coefficients

87-Fr-209



20

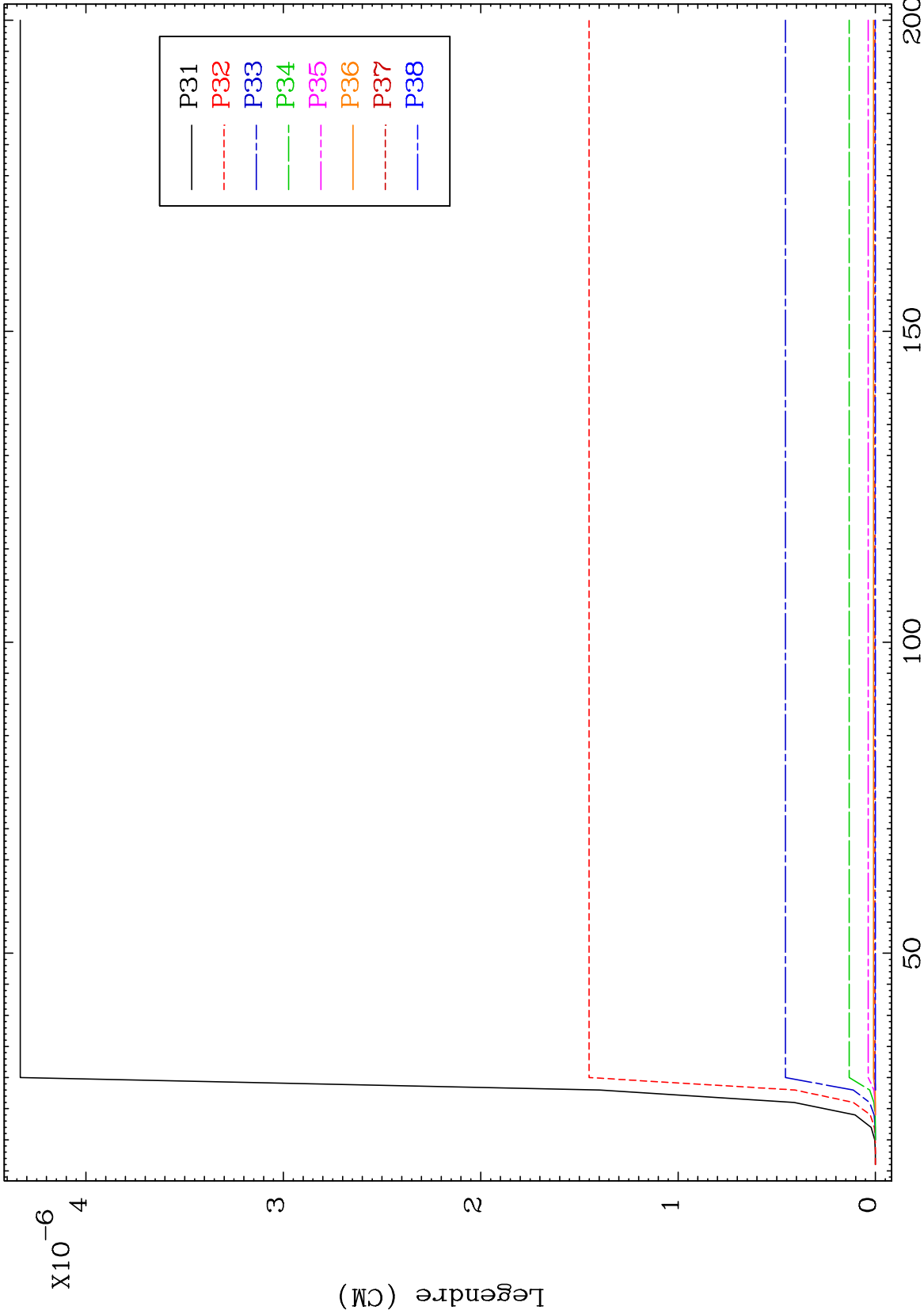
Incident Energy (MeV)

87-Fr-209

MAT 8716

MT= 51 (n,n') Level  
Legendre Coefficients

87-Fr-209



21

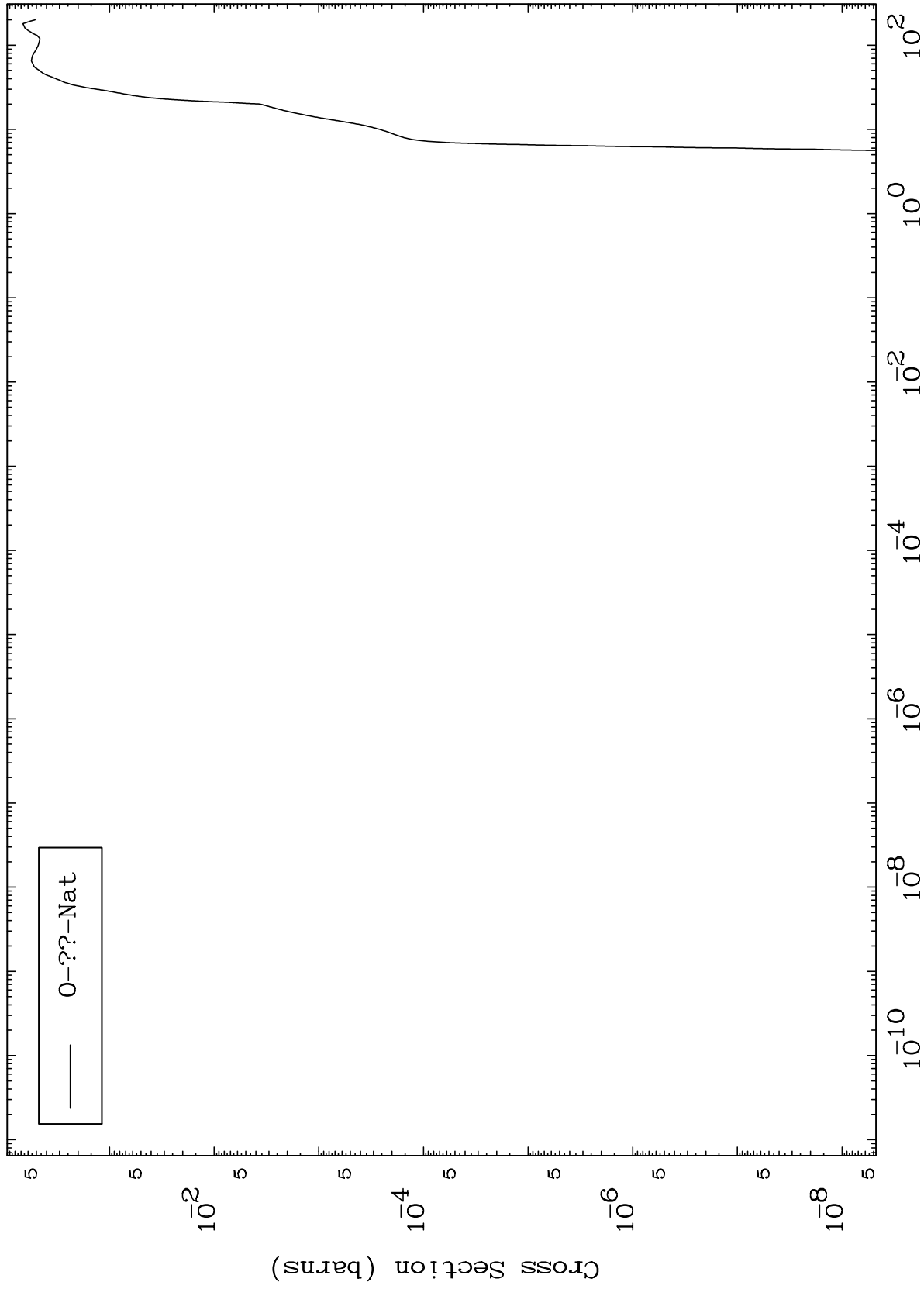
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Fission

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Radionuclide Production Cross Section



22

Incident Energy (MeV)

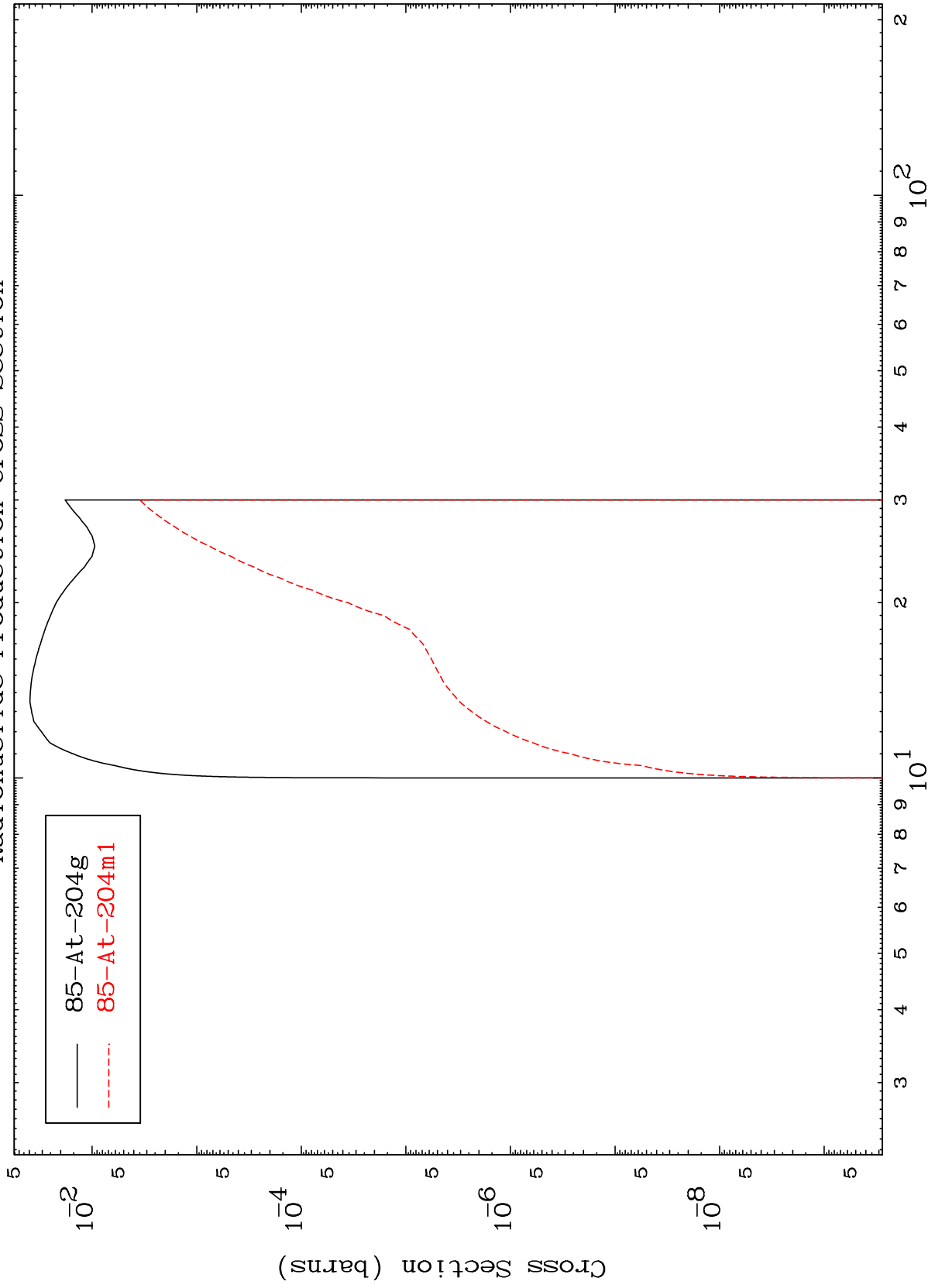
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(n,2n)  $\alpha$

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Radionuclide Production Cross Section



23

Incident Energy (MeV)

87-Fr-209

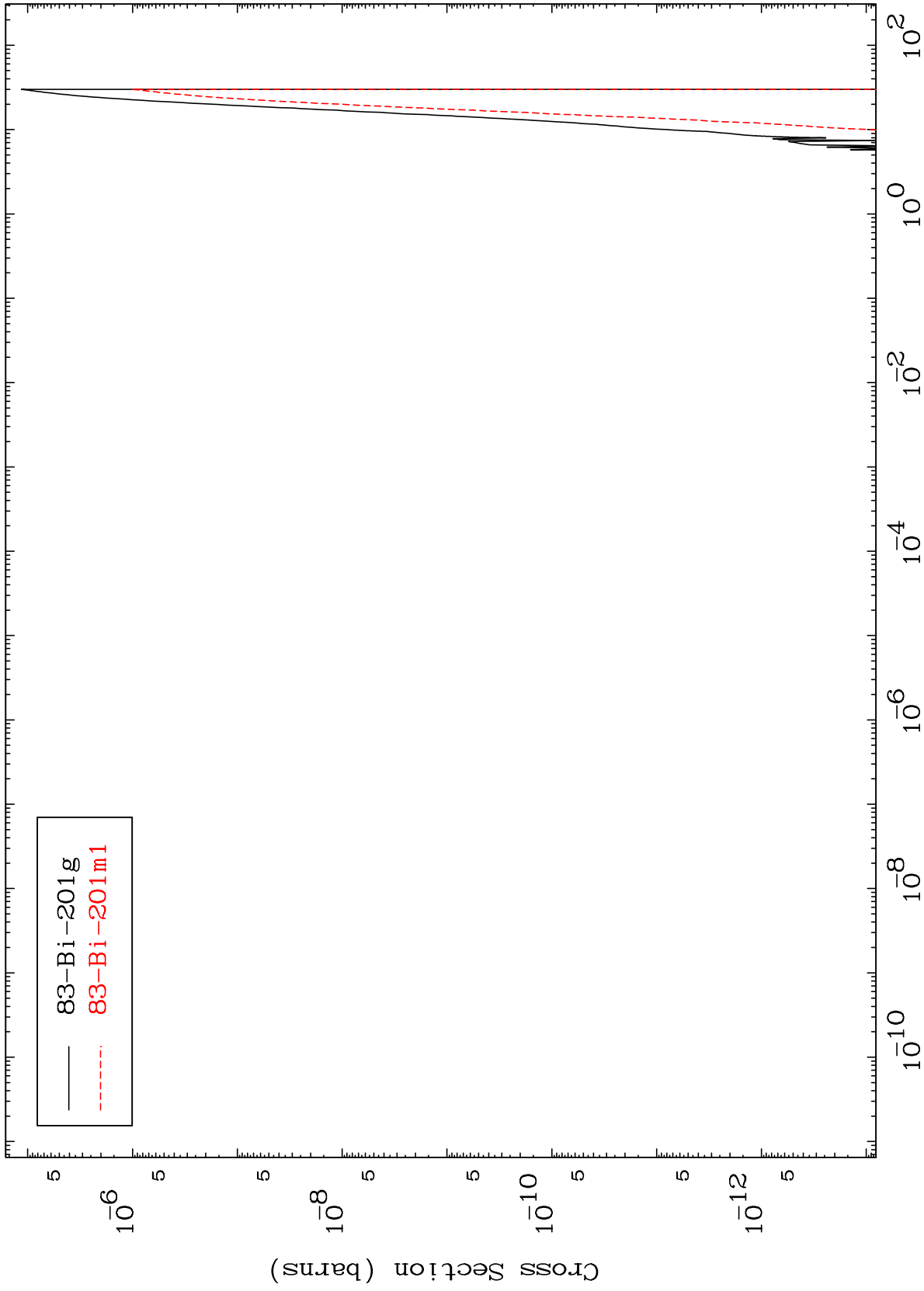


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(n,n') 2 $\alpha$

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Radionuclide Production Cross Section



24

Incident Energy (MeV)

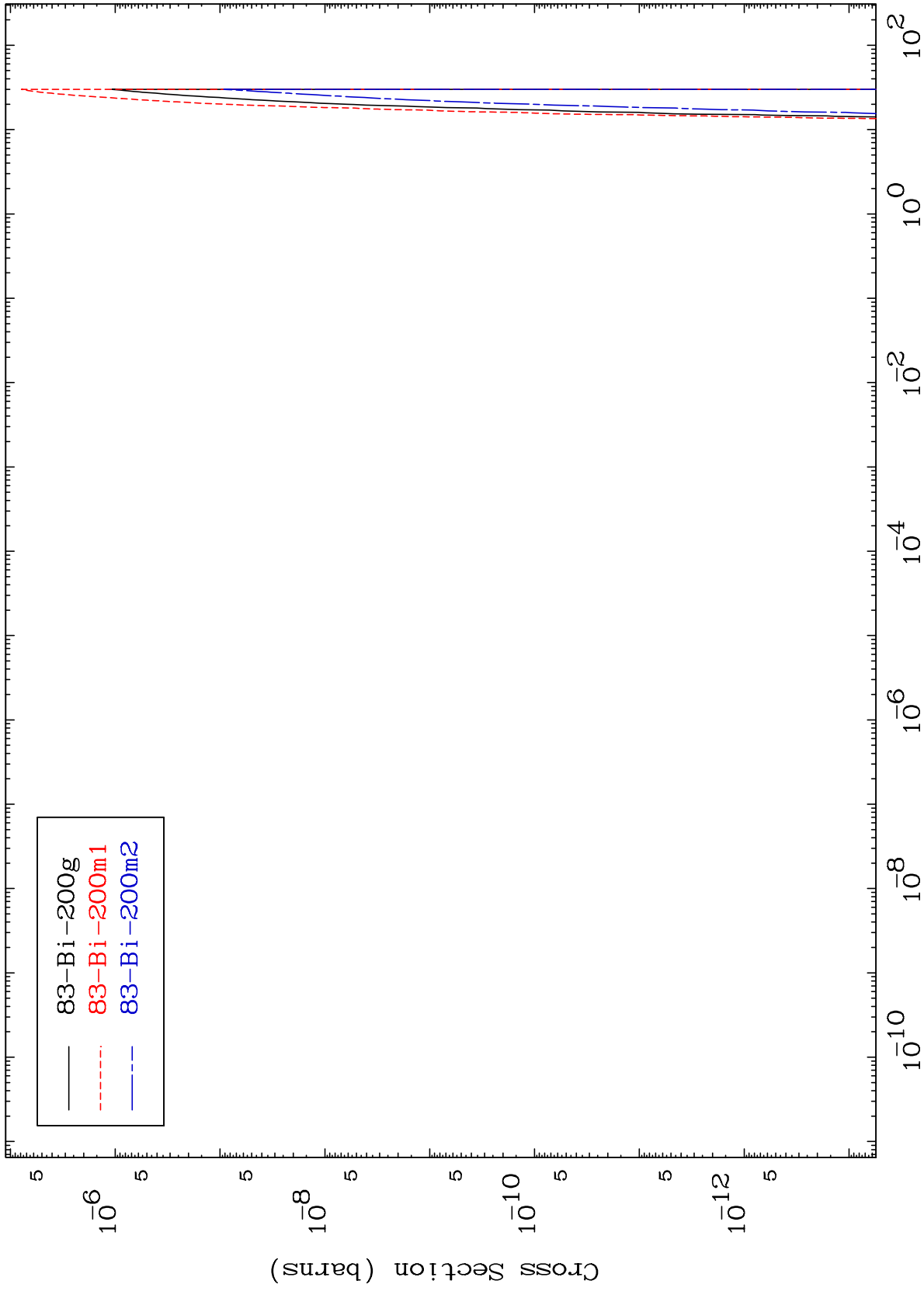
87-Fr-209

MAT 8716

(n,2n) 2 $\alpha$

87-Fr-209

Radionuclide Production Cross Section



25

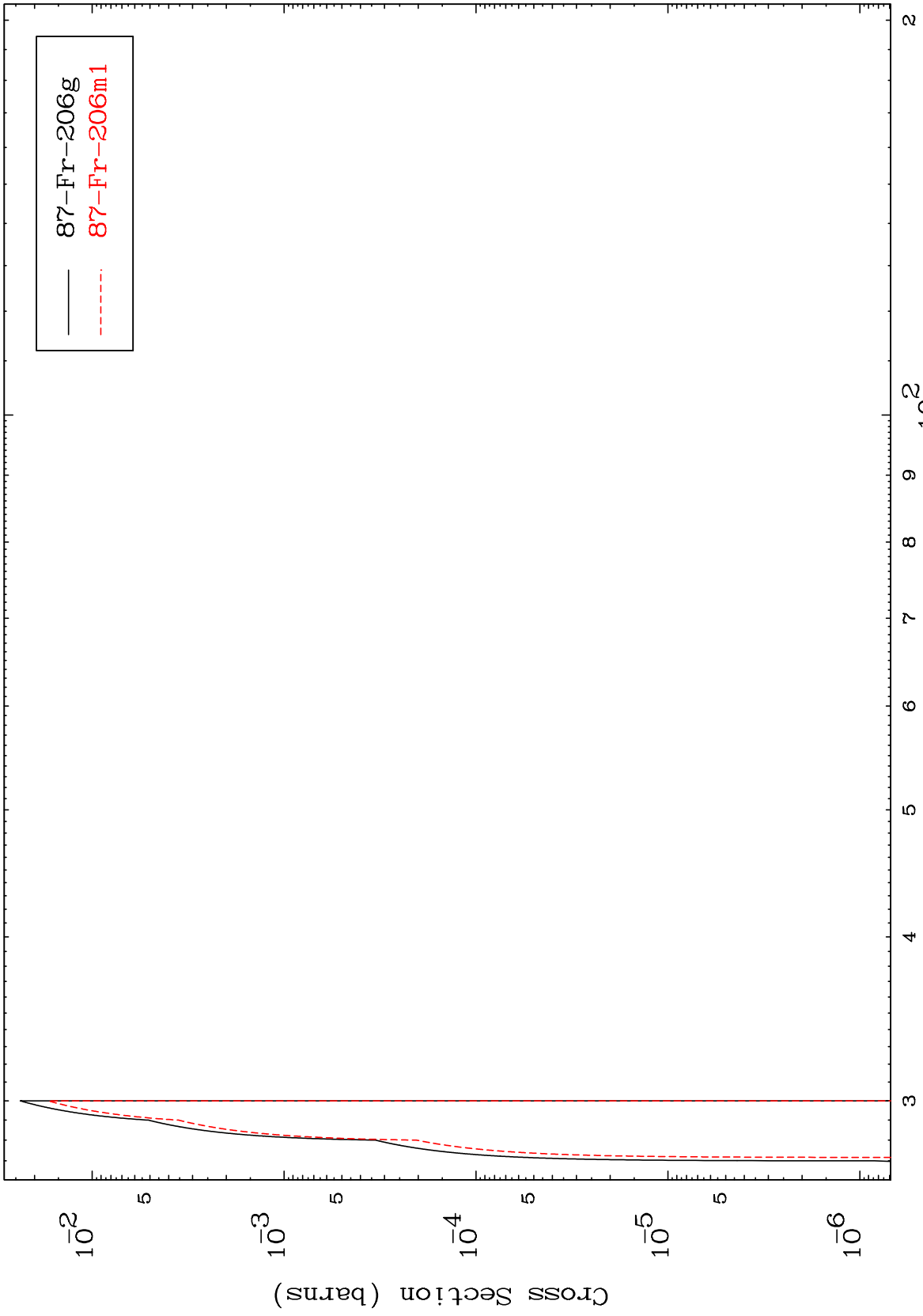
Incident Energy (MeV)

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(n,4n)  
Radionuclide Production Cross Section



87-Fr-206g  
87-Fr-206m1

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87-Fr-209

Incident Energy (MeV)