

Program EVALPLOT  
(Version 2018-1)

by

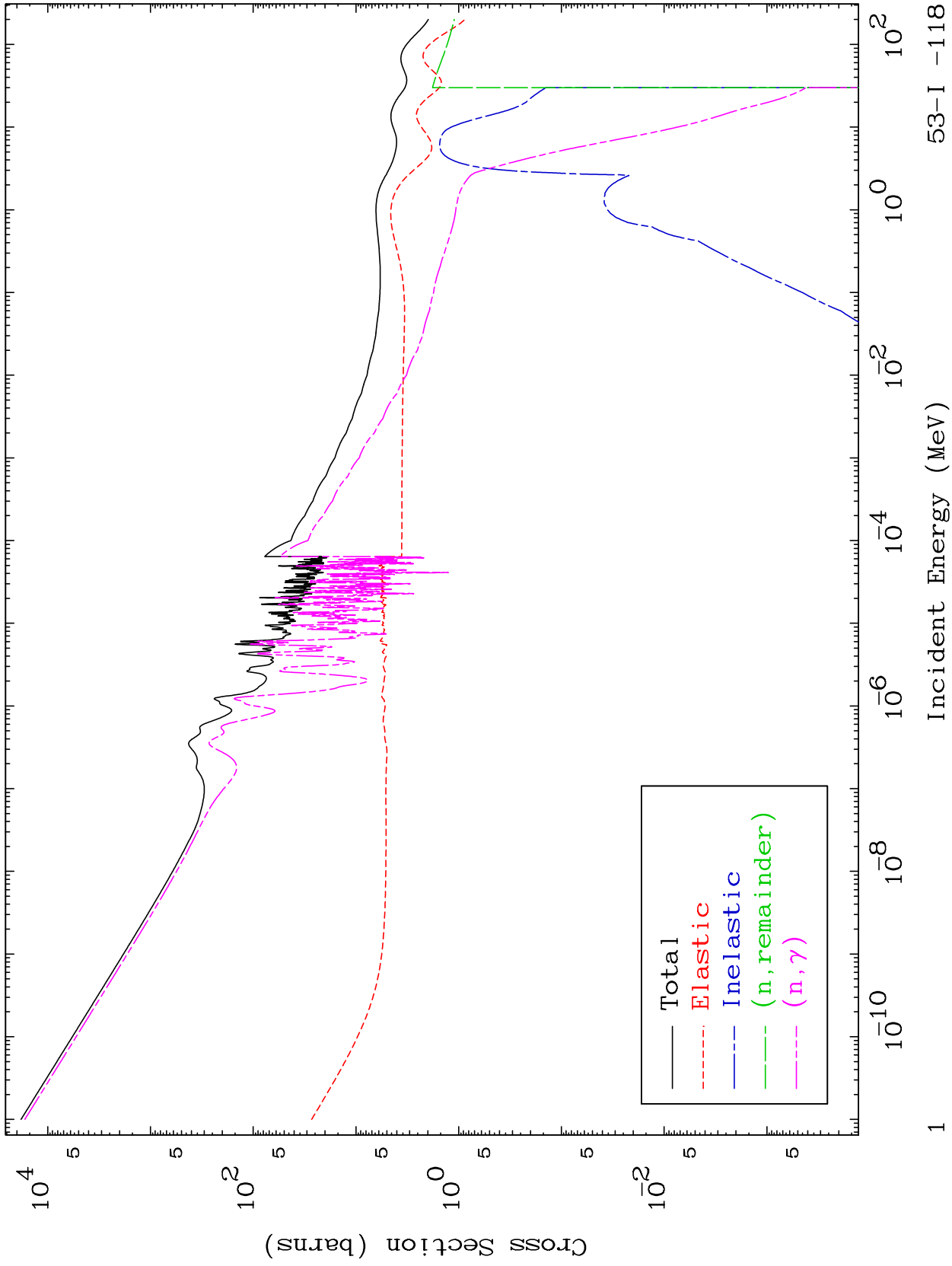
Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

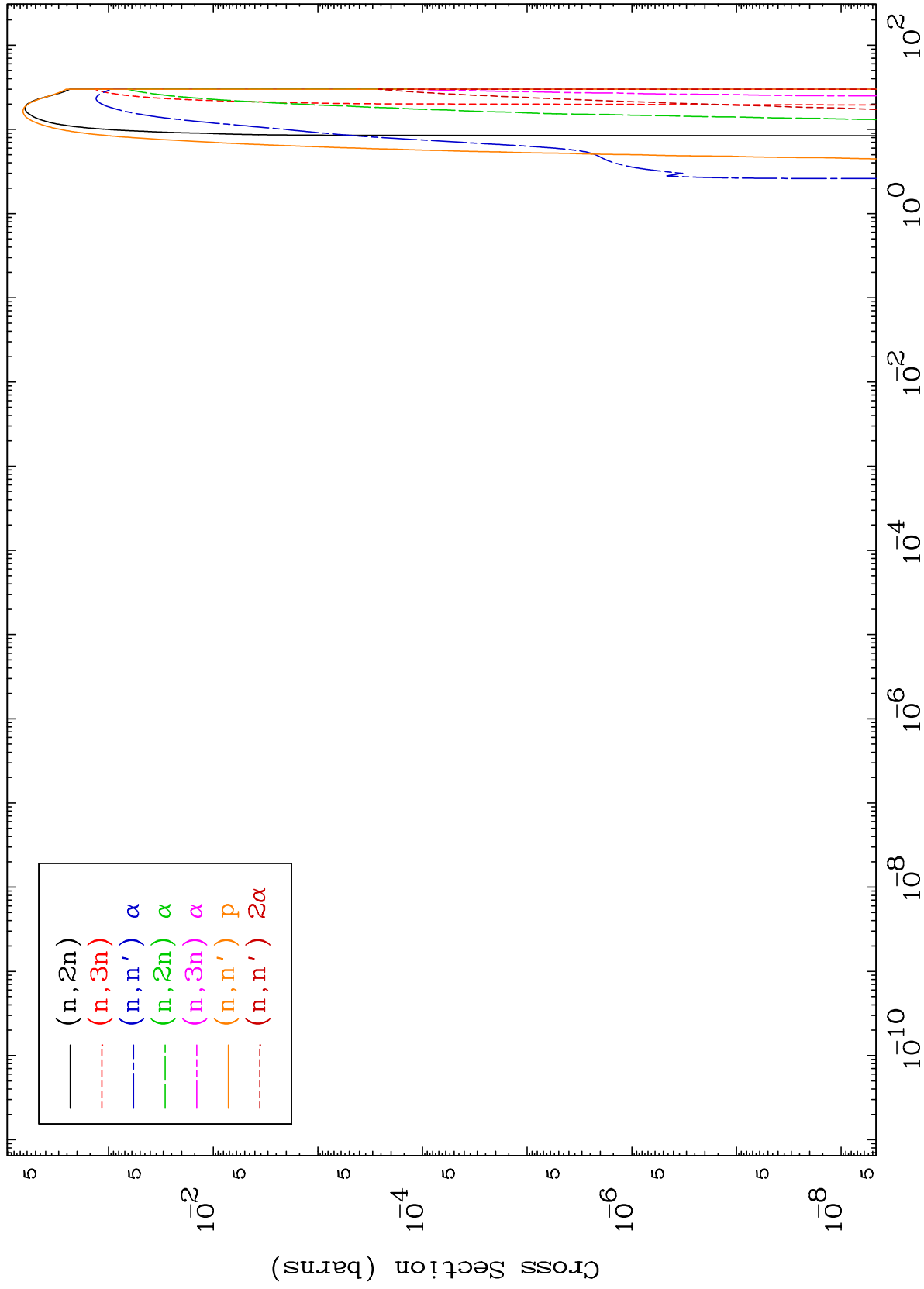
Press Mouse Button to Start



MAT 5299

Neutron Production  
293 Kelvin Cross Sections

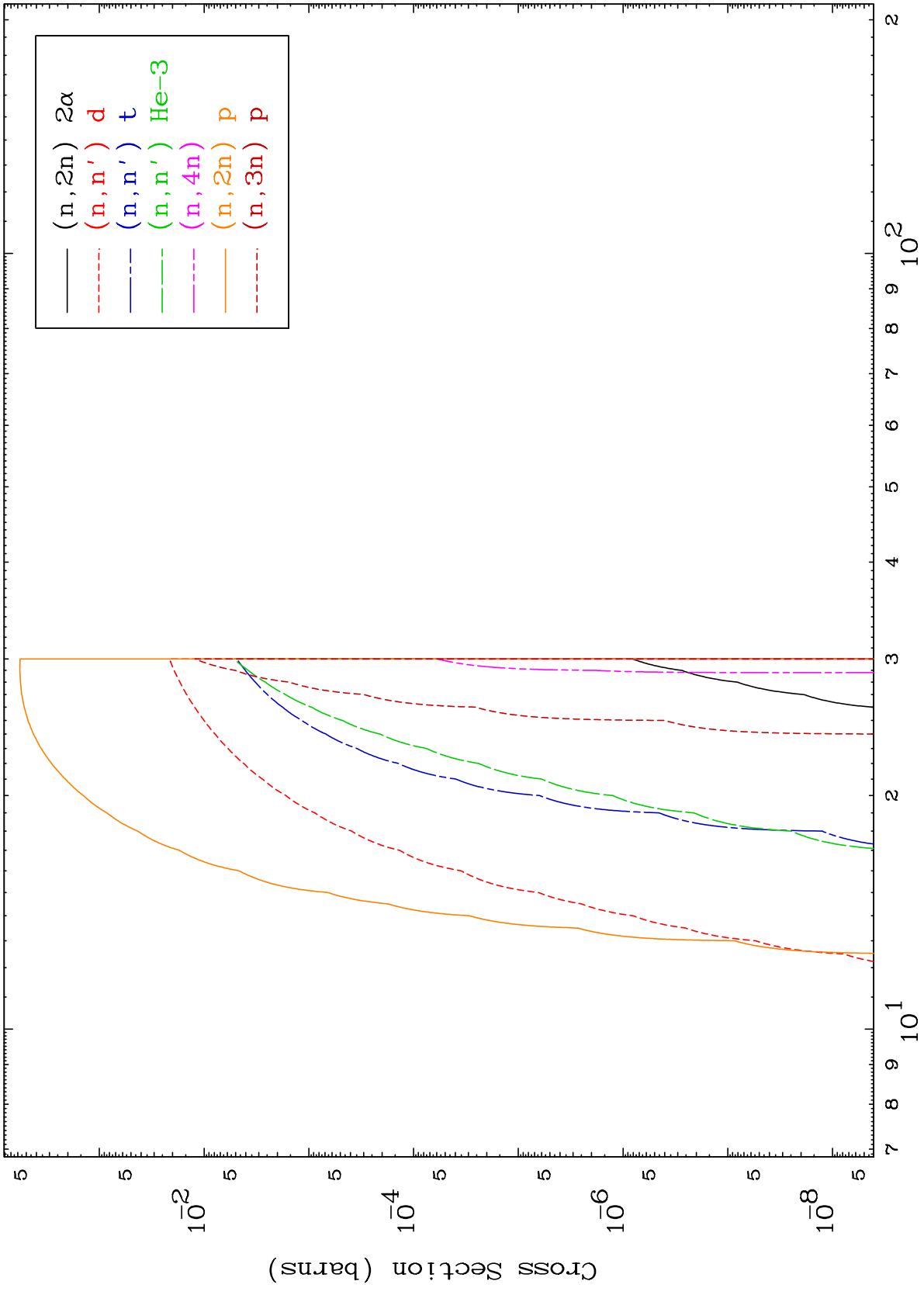
53-I -118

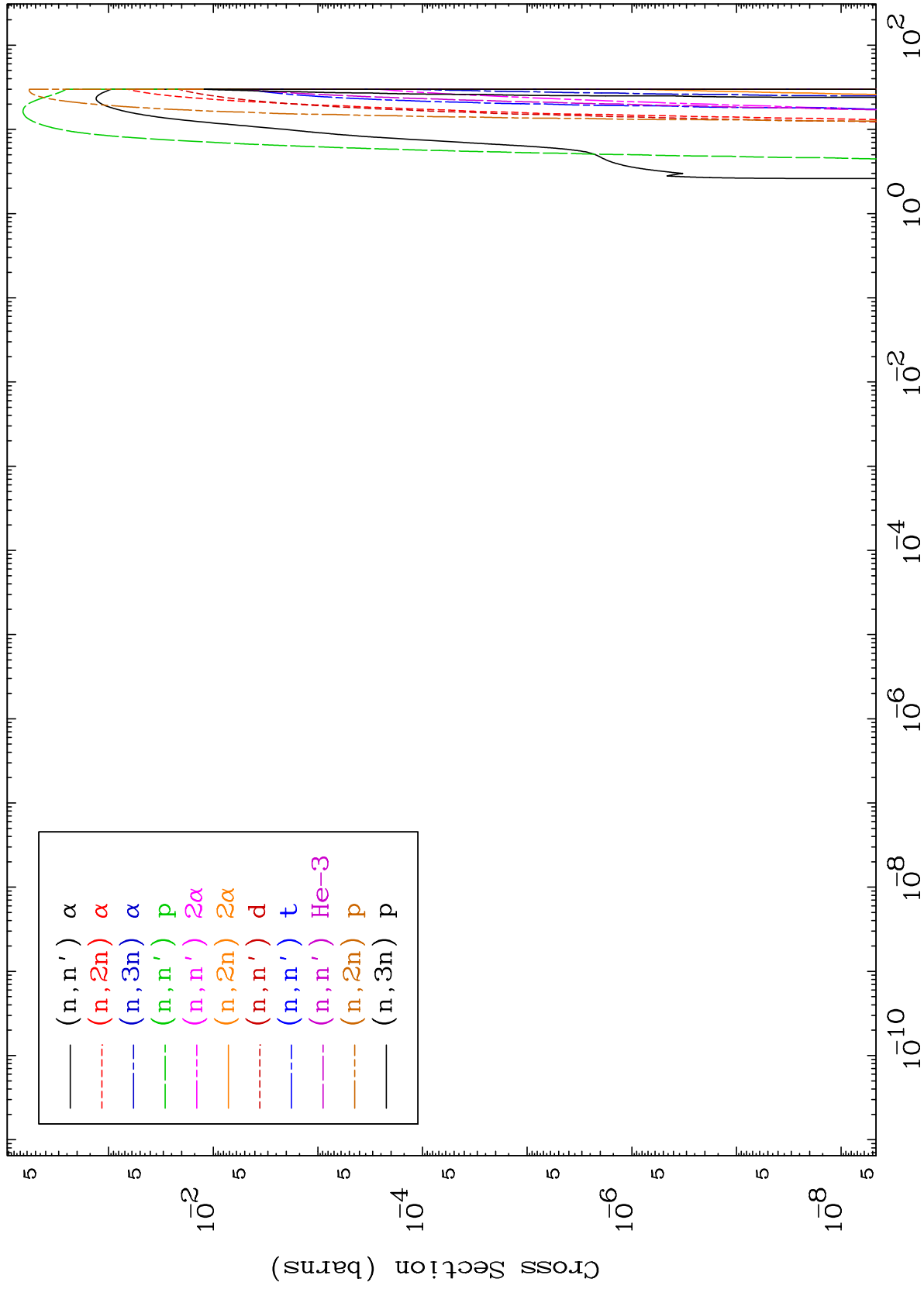


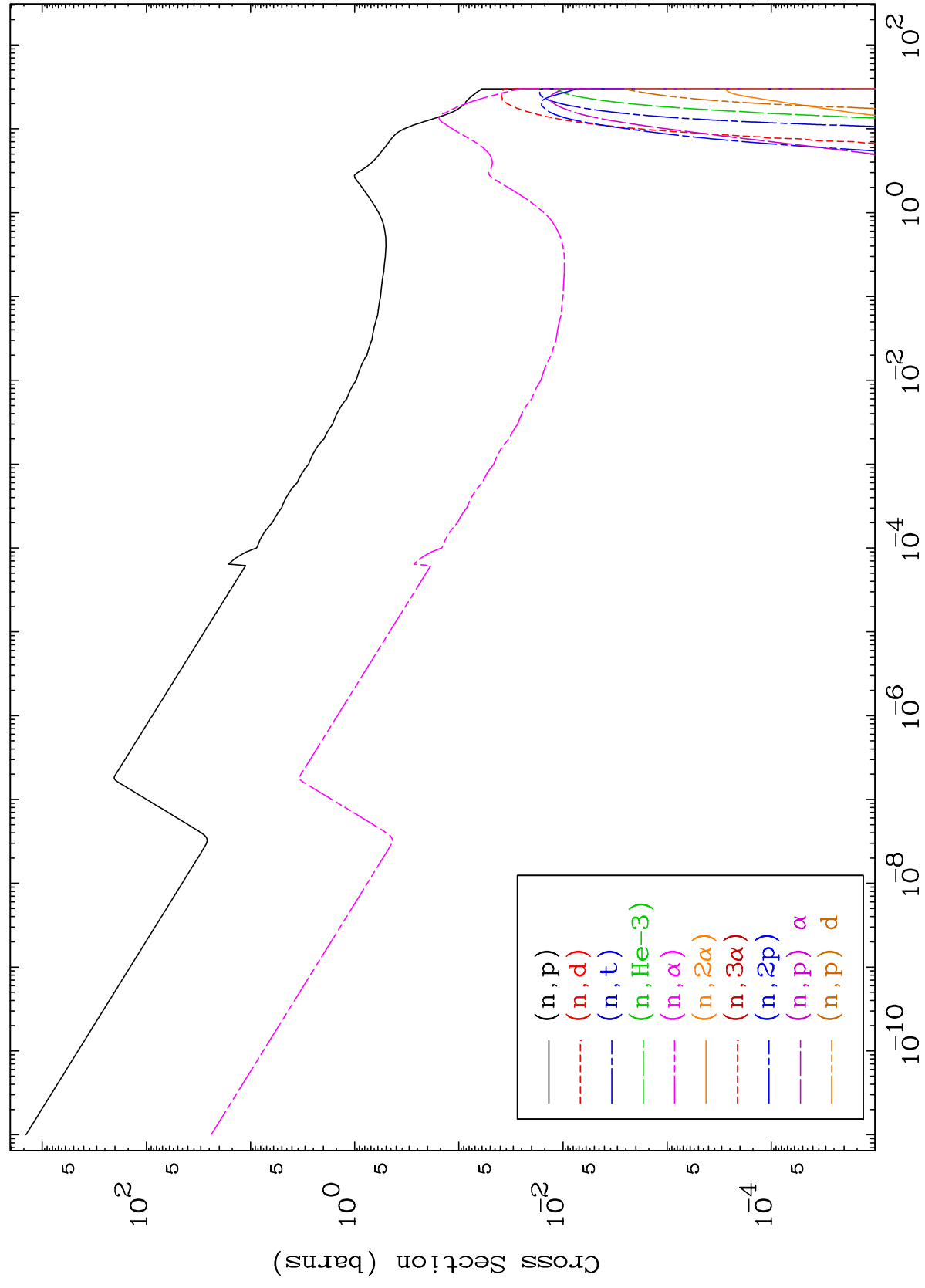
2

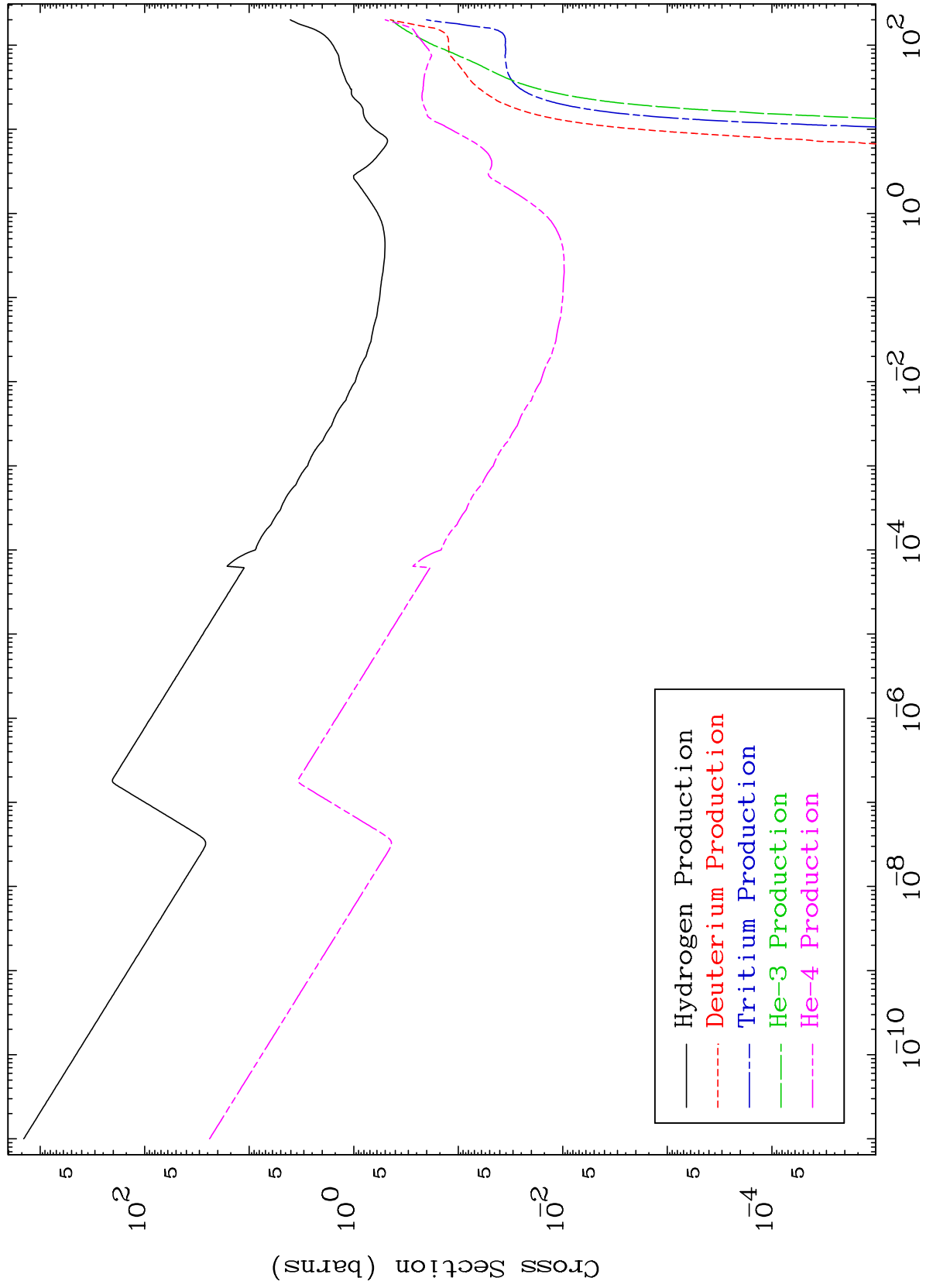
Incident Energy (MeV)

53-I -118





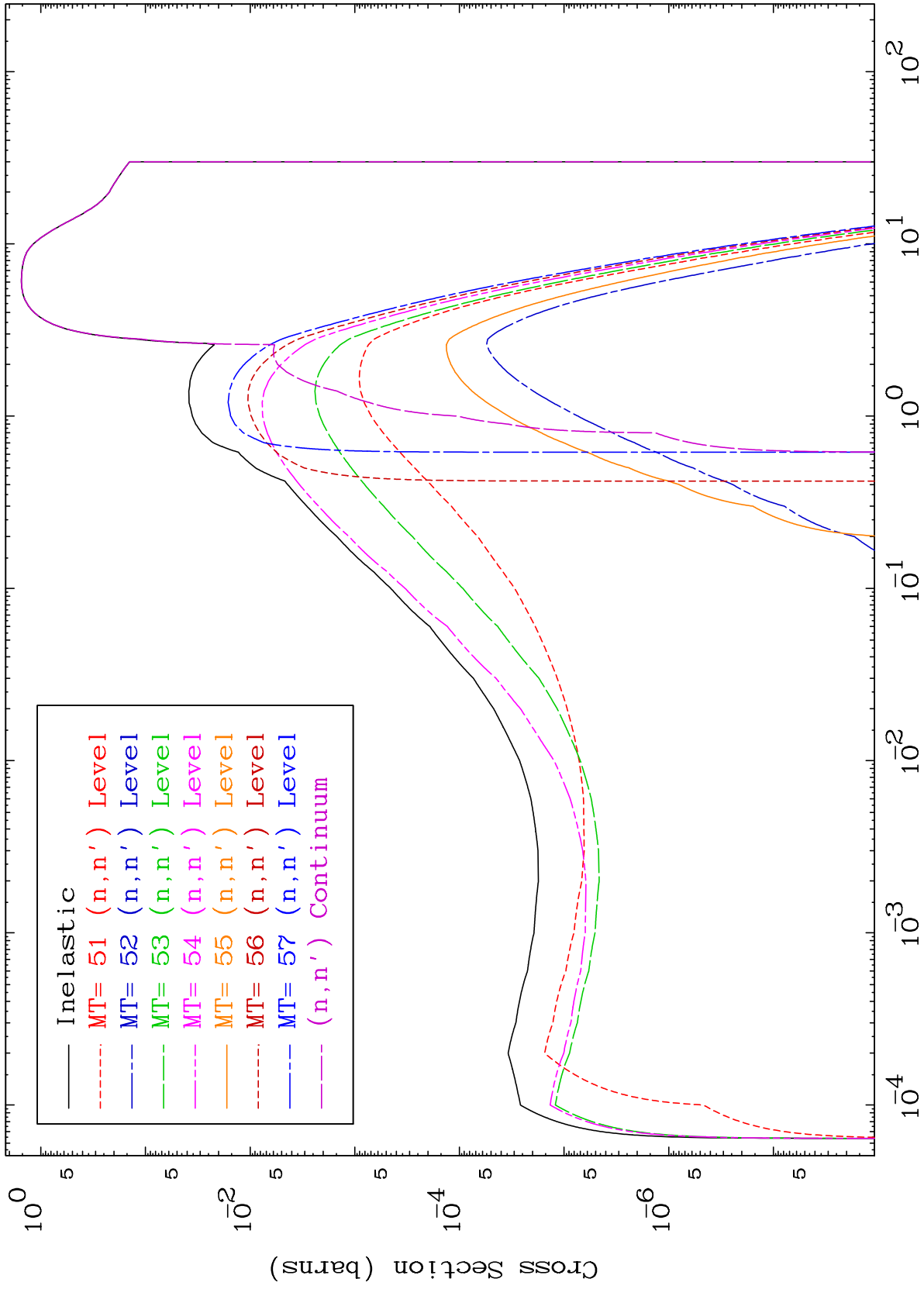




MAT 5299

(n,n') Level  
293 Kelvin Cross Sections

53-I -118



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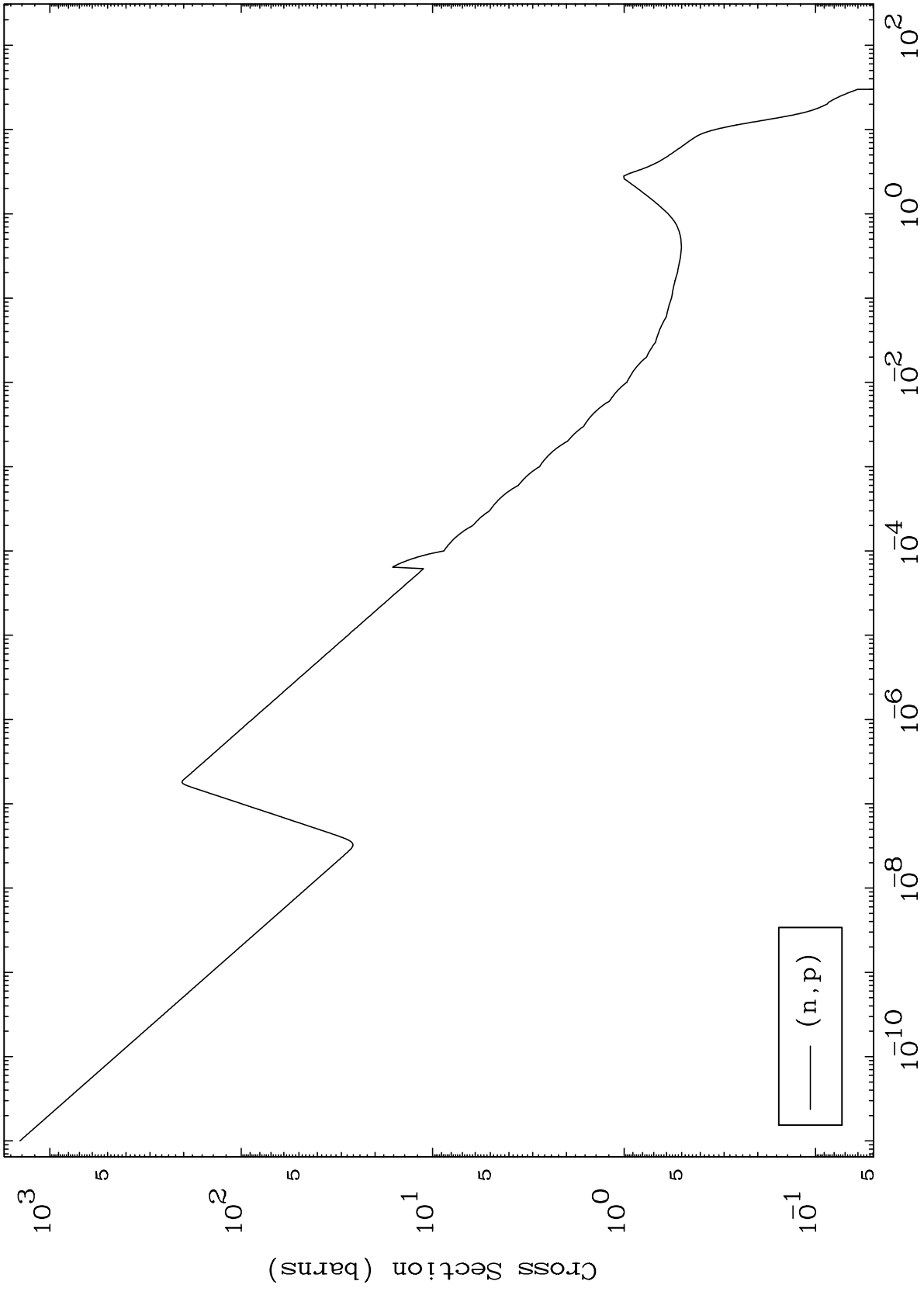
53-I -118



MAT 5299

(n,p) Levels  
293 Kelvin Cross Sections

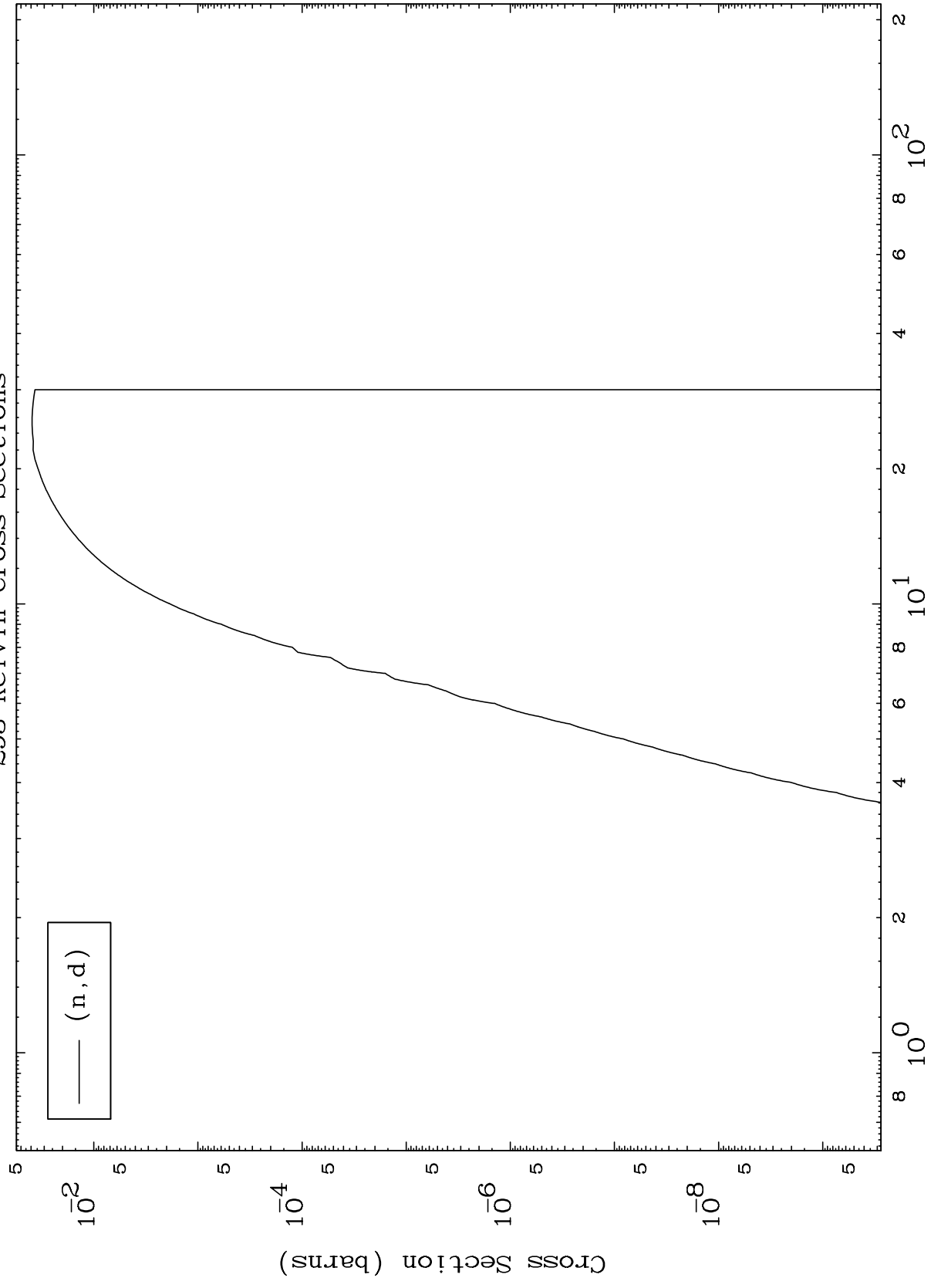
53-I -118



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(n,d) Levels  
293 Kelvin Cross Sections

53-I -118



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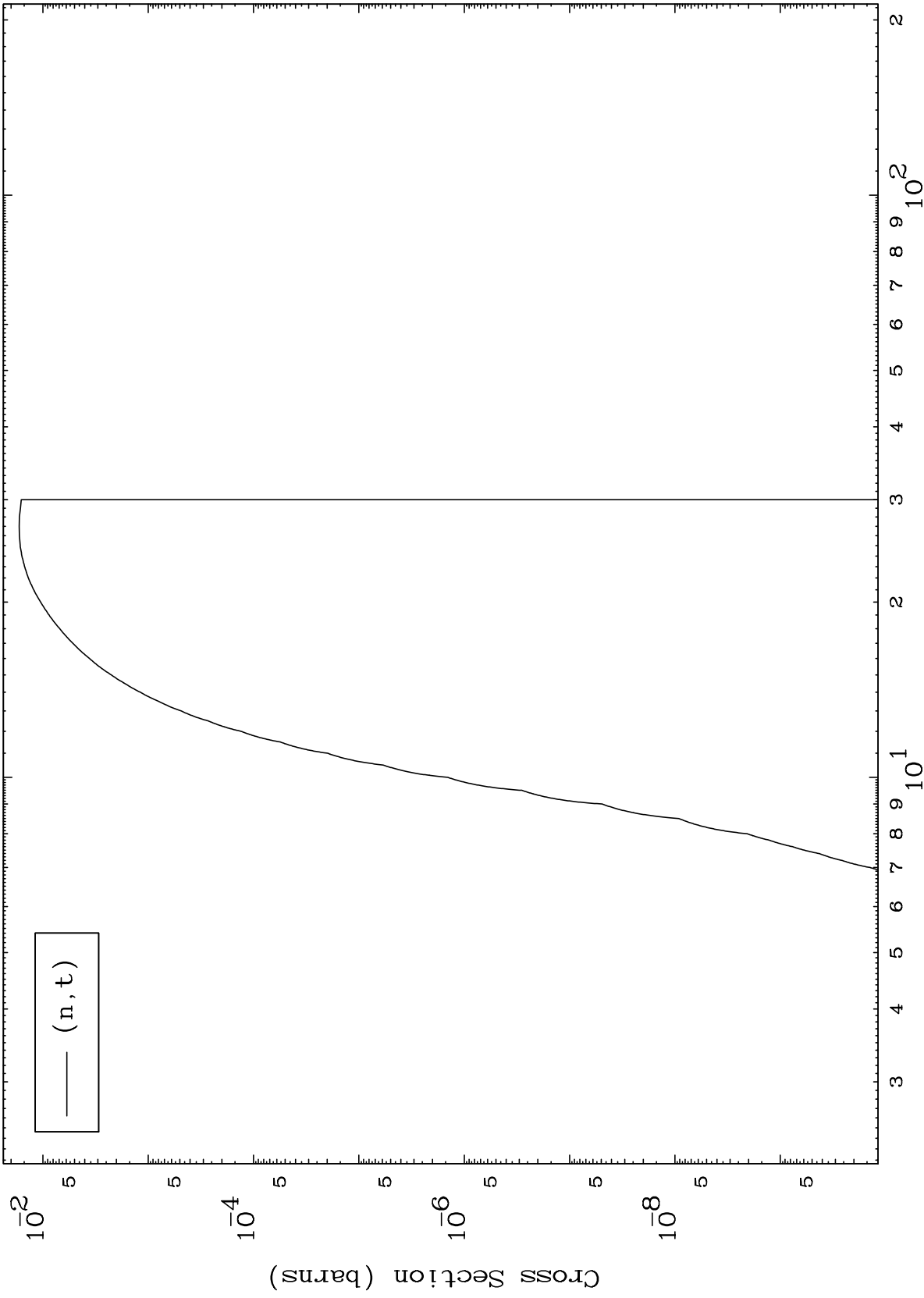
Incident Energy (MeV)

53-I -118

MAT 5299

(n,t) Levels  
293 Kelvin Cross Sections

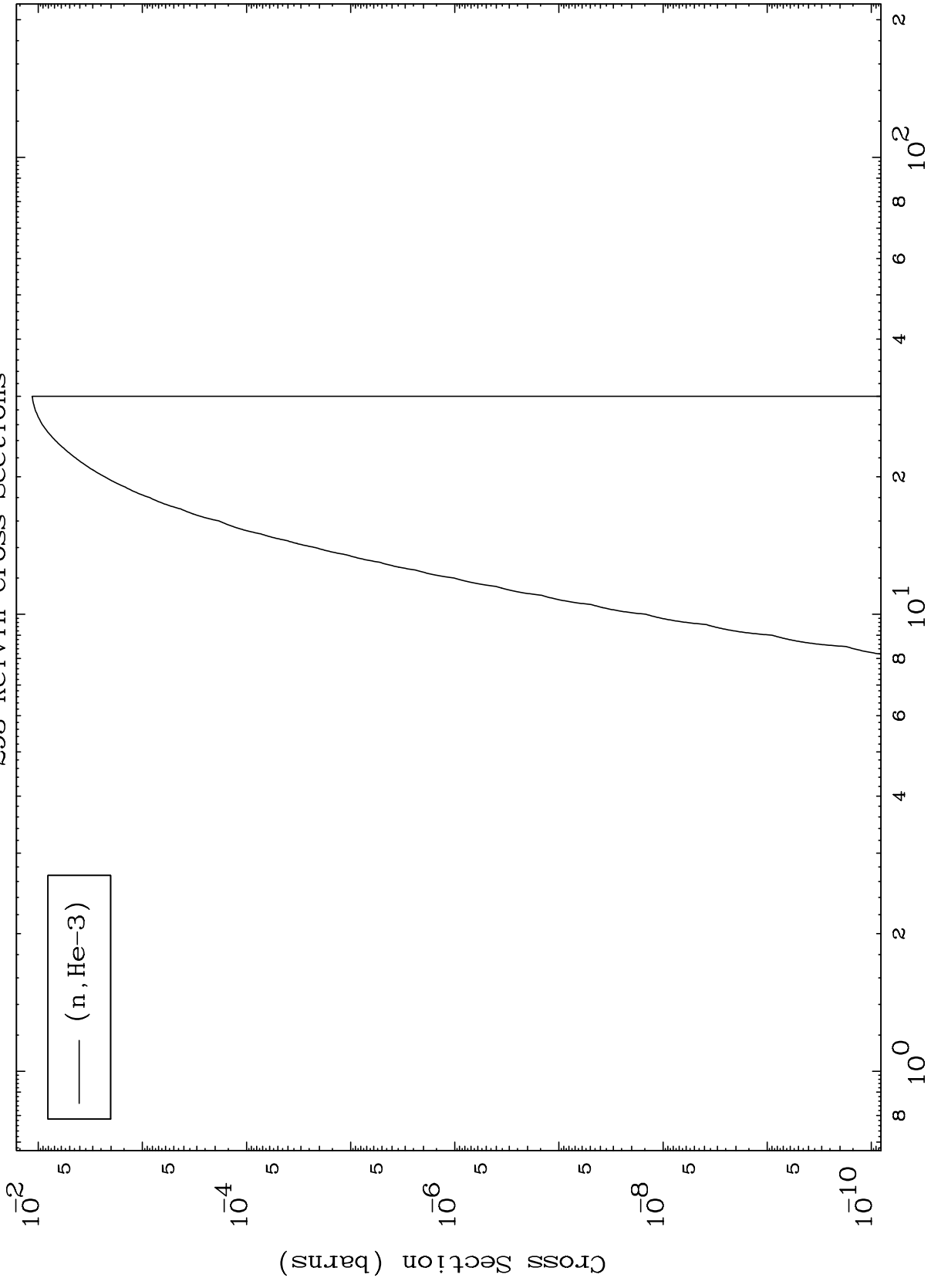
53-I -118



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Incident Energy (MeV)

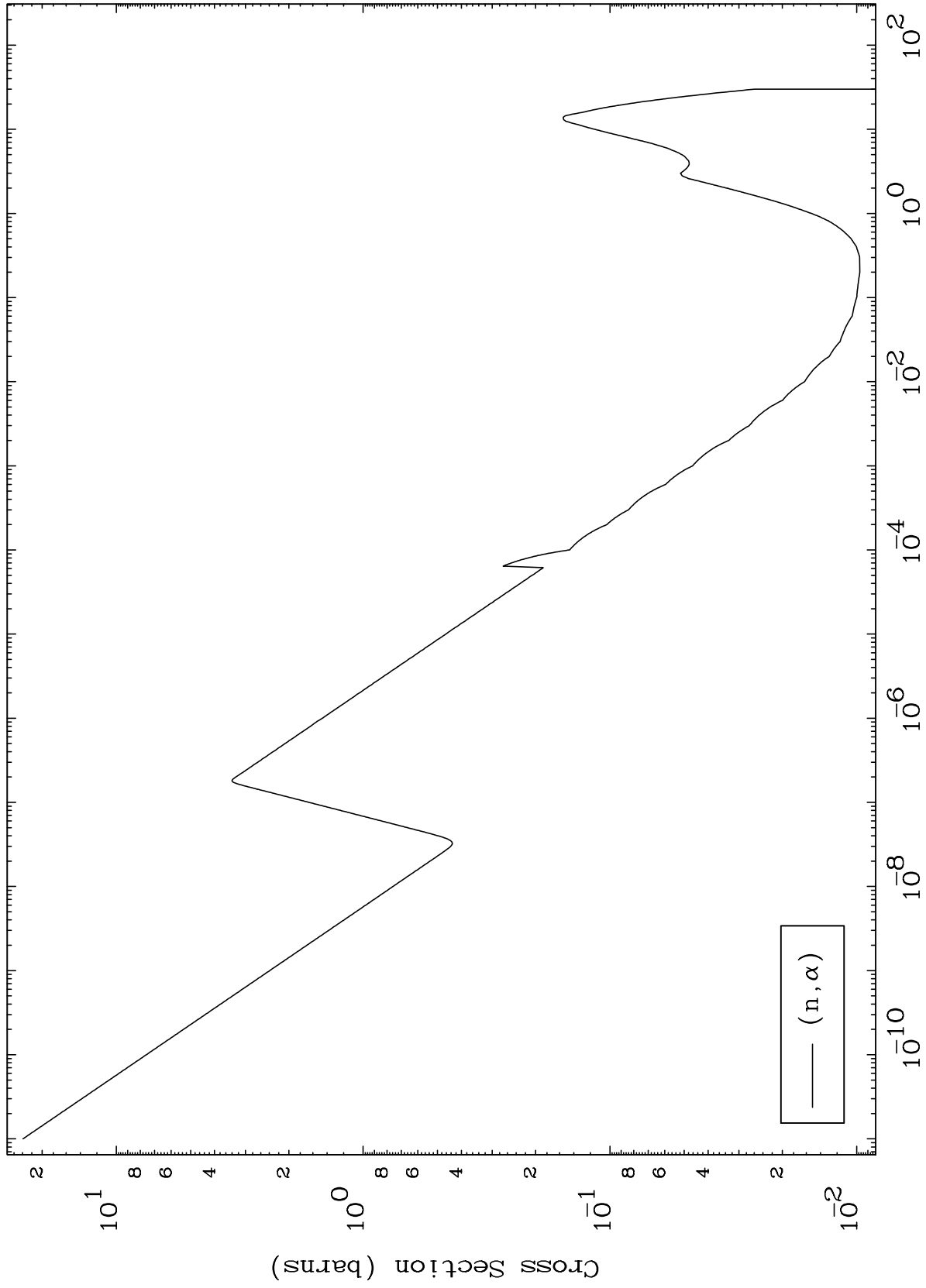
53-I -118



MAT 5299

(n, $\alpha$ ) Levels  
293 Kelvin Cross Sections

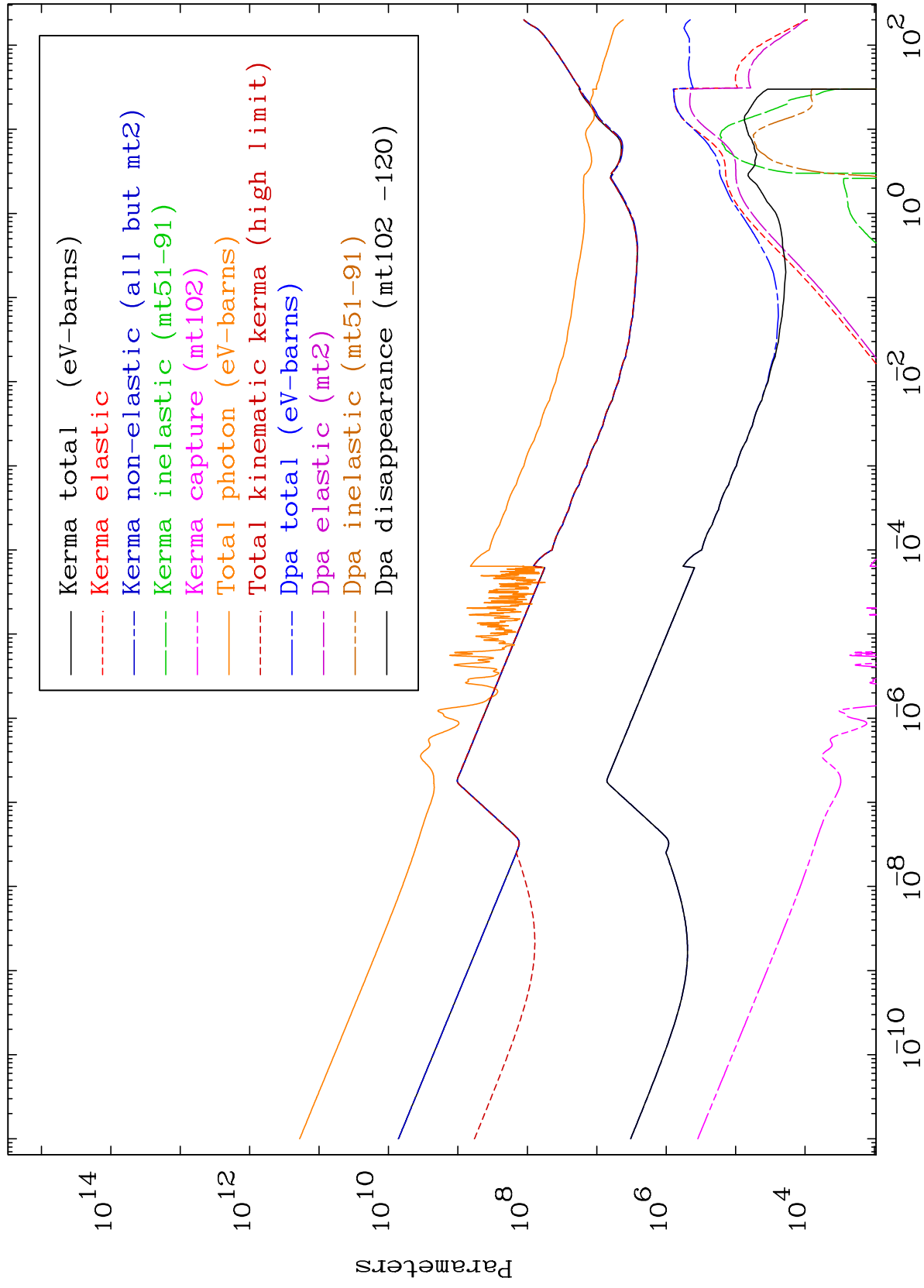
53-I -118



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Incident Energy (MeV)

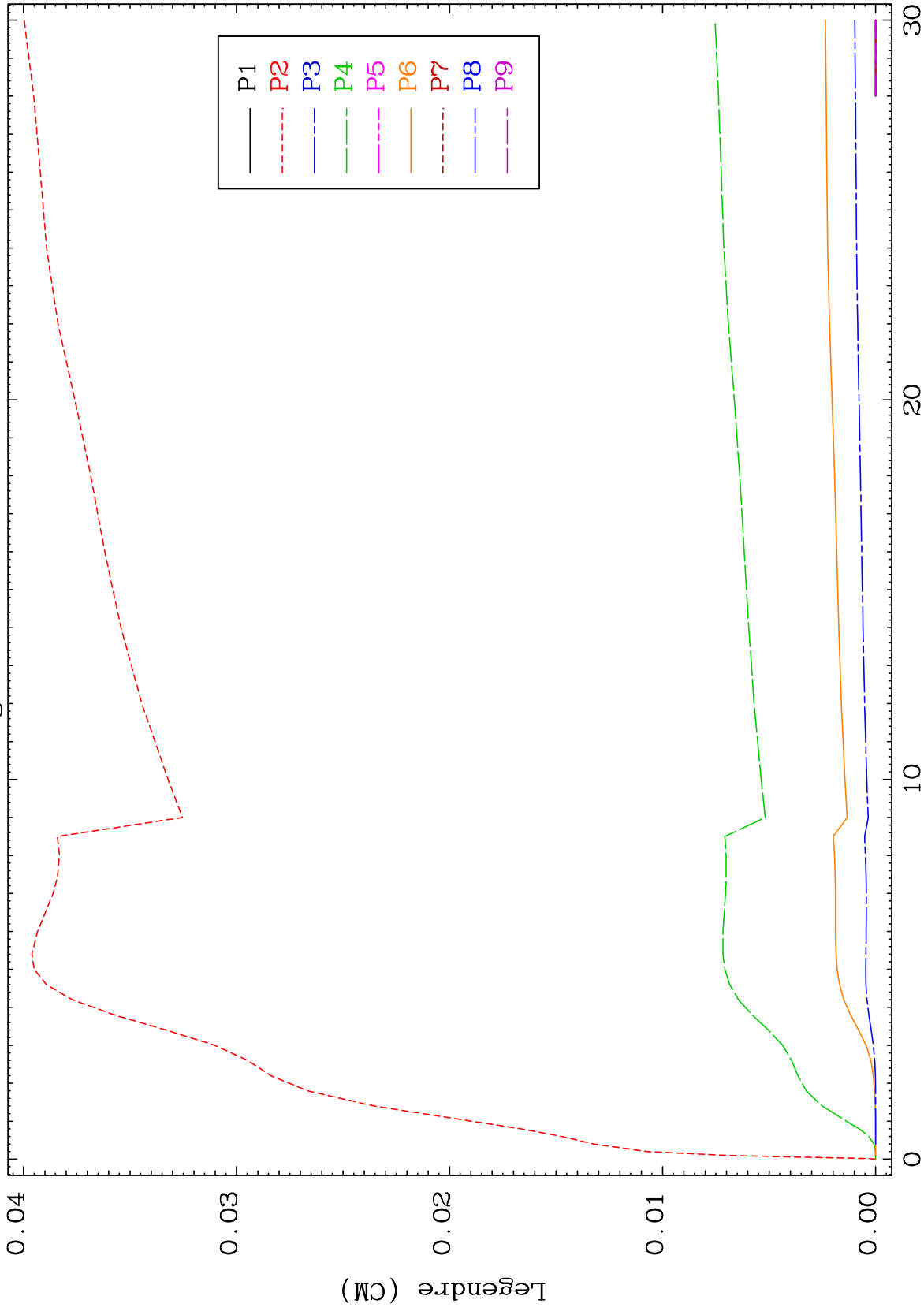
53-I -118



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Elastic Legendre Coefficients

53-I -118



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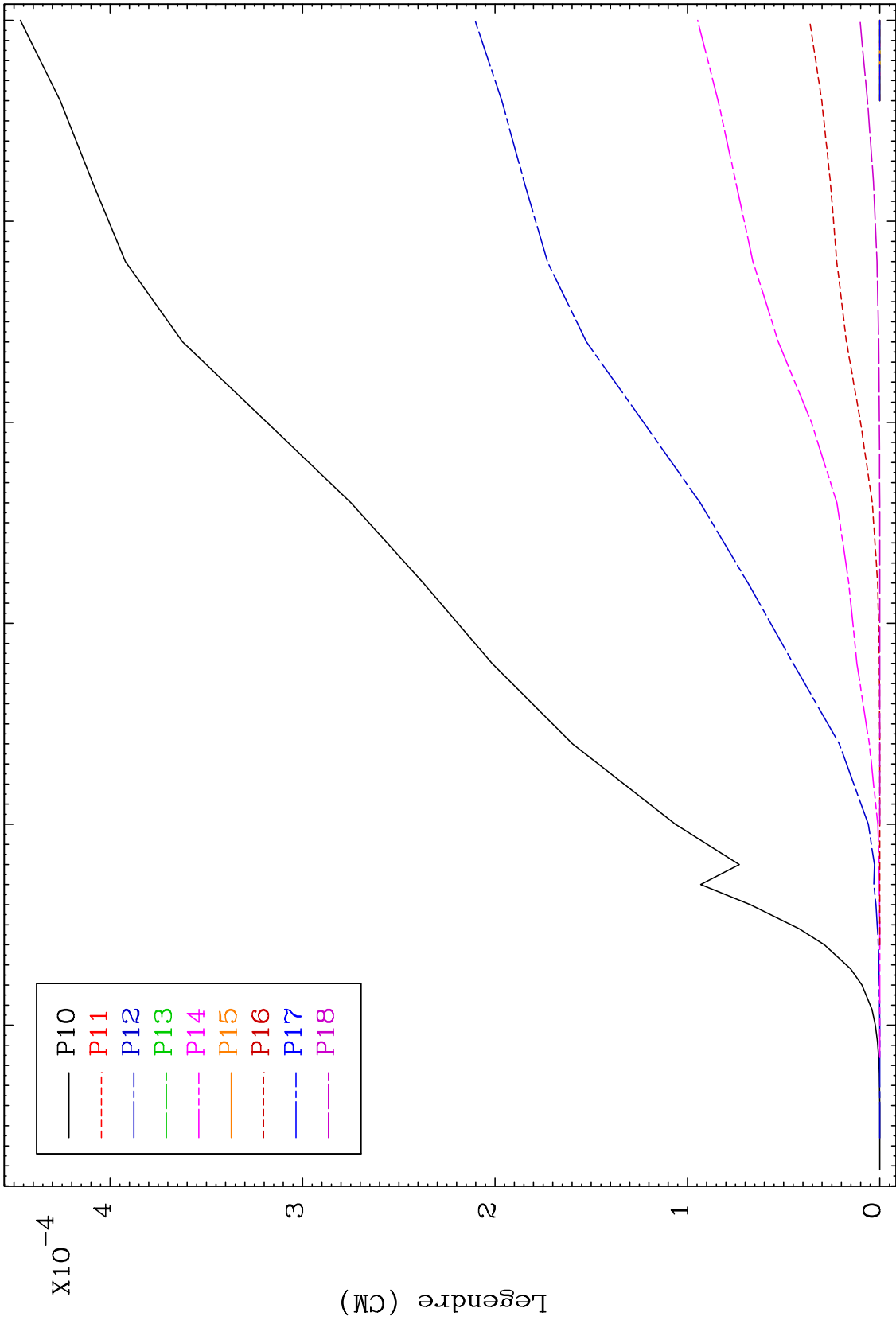
Incident Energy (MeV)

53-I -118

MAT 5299

Elastic Legendre Coefficients

53-I -118



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Incident Energy (MeV)

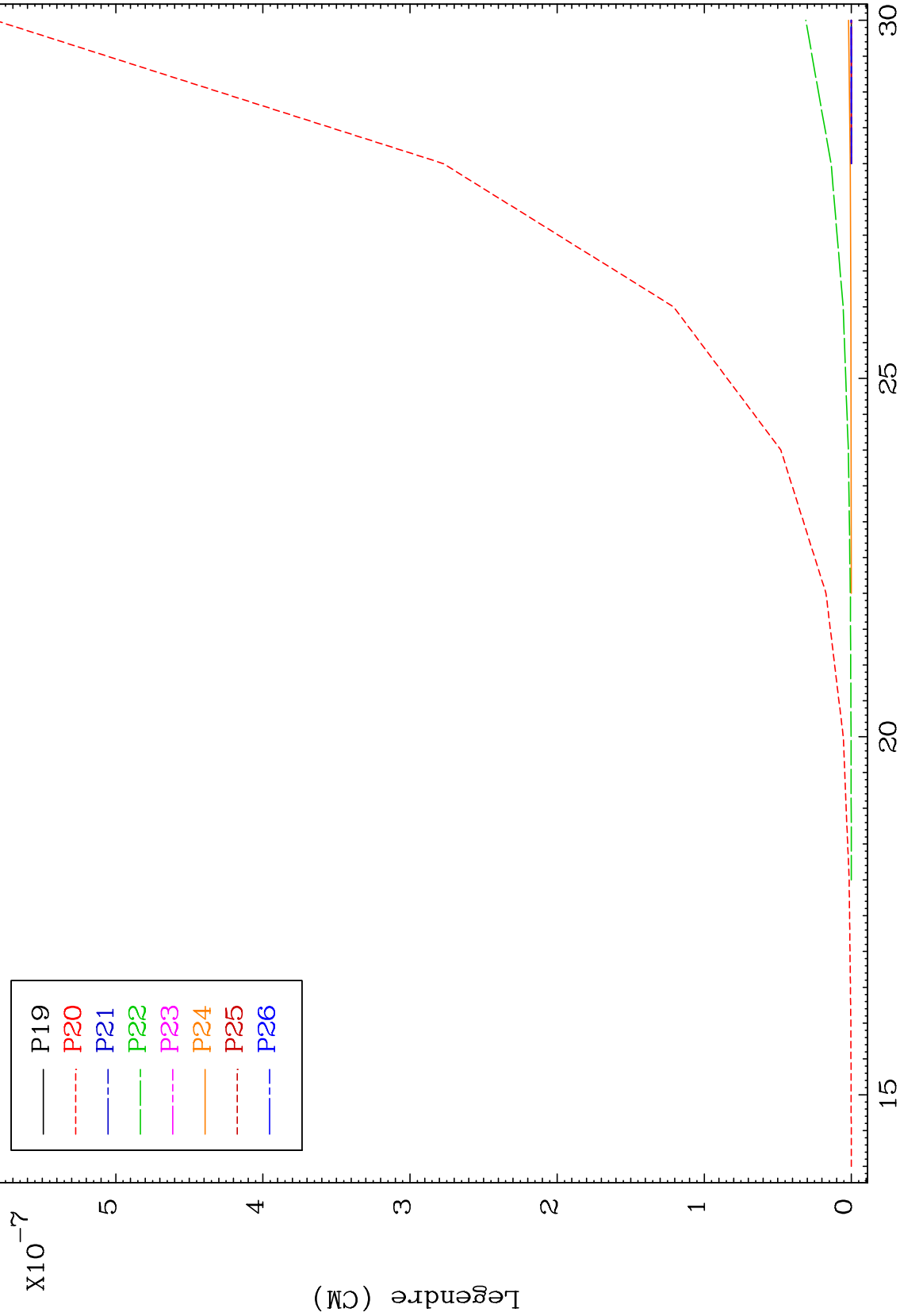
53-I -118



MAT 5299

Elastic  
Legendre Coefficients

53-I -118



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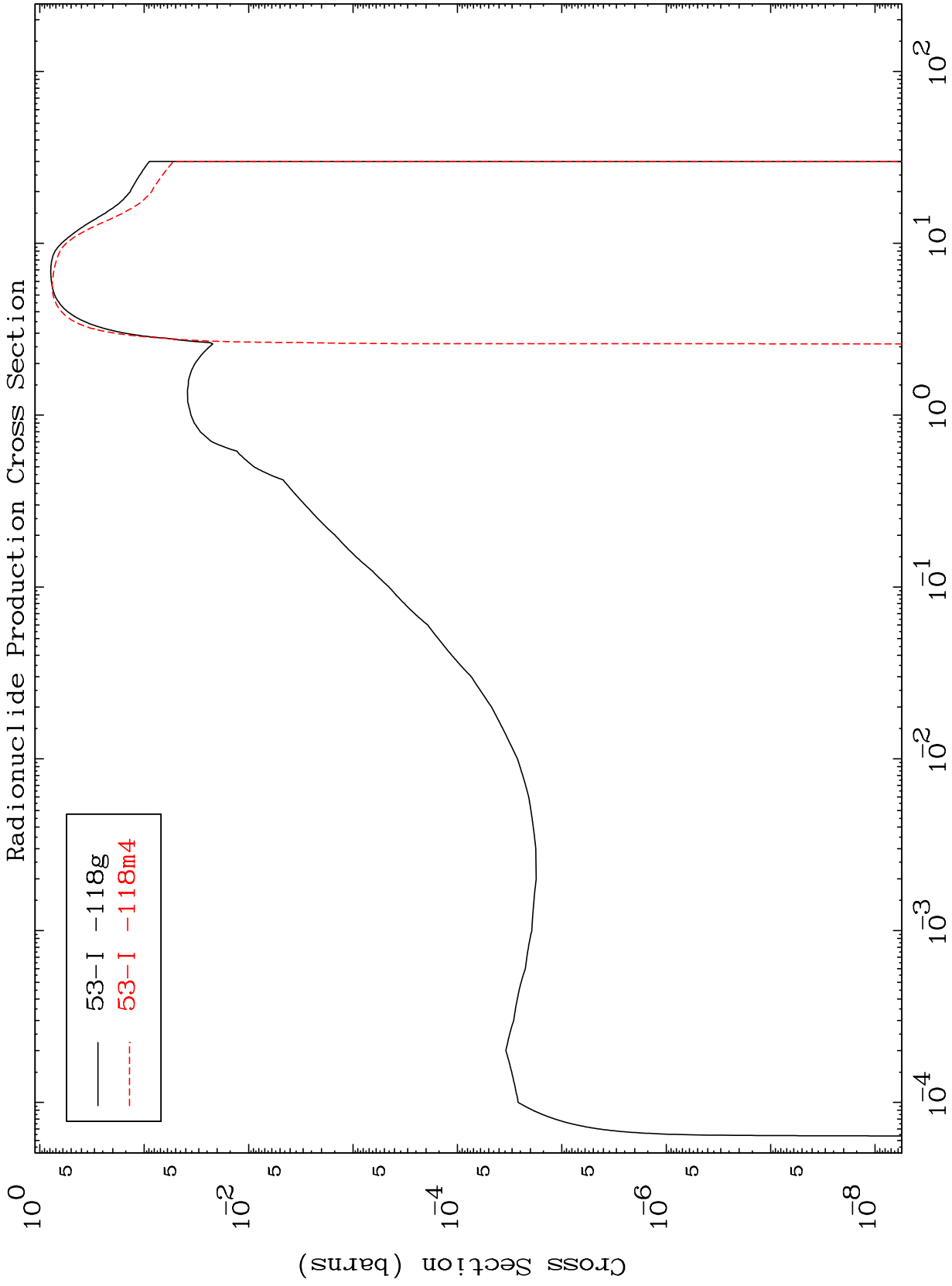
Incident Energy (MeV)

53-I -118

MAT 5299

Inelastic  
Radionuclide Production Cross Section

53-I -118



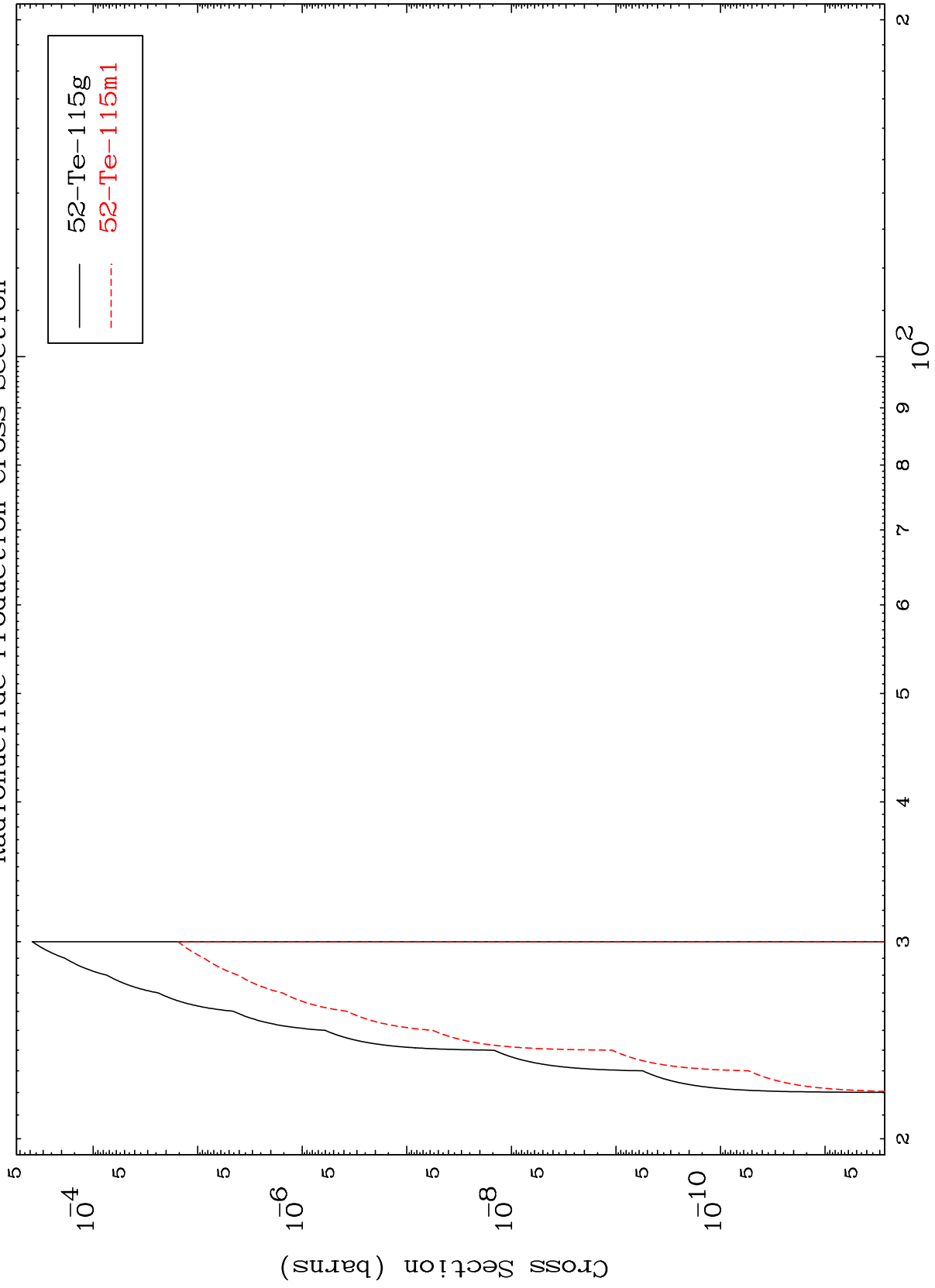
53-I -118g  
53-I -118m4

MAT 5299

(n,2n) d

53-I -118

Radionuclide Production Cross Section



18

Incident Energy (MeV)

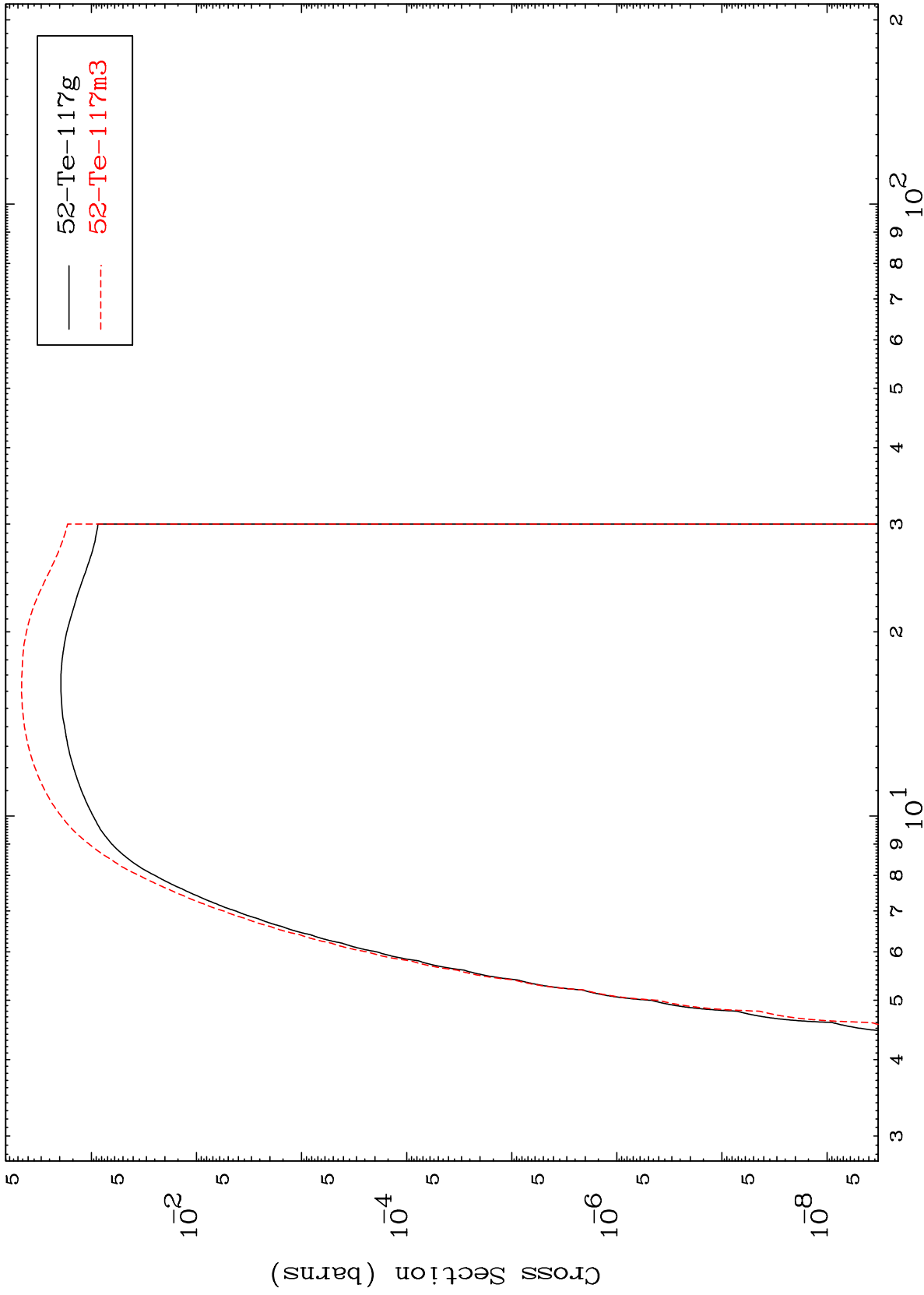
53-I -118

MAT 5299

(n,n') p

53-I -118

Radionuclide Production Cross Section



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Incident Energy (MeV)

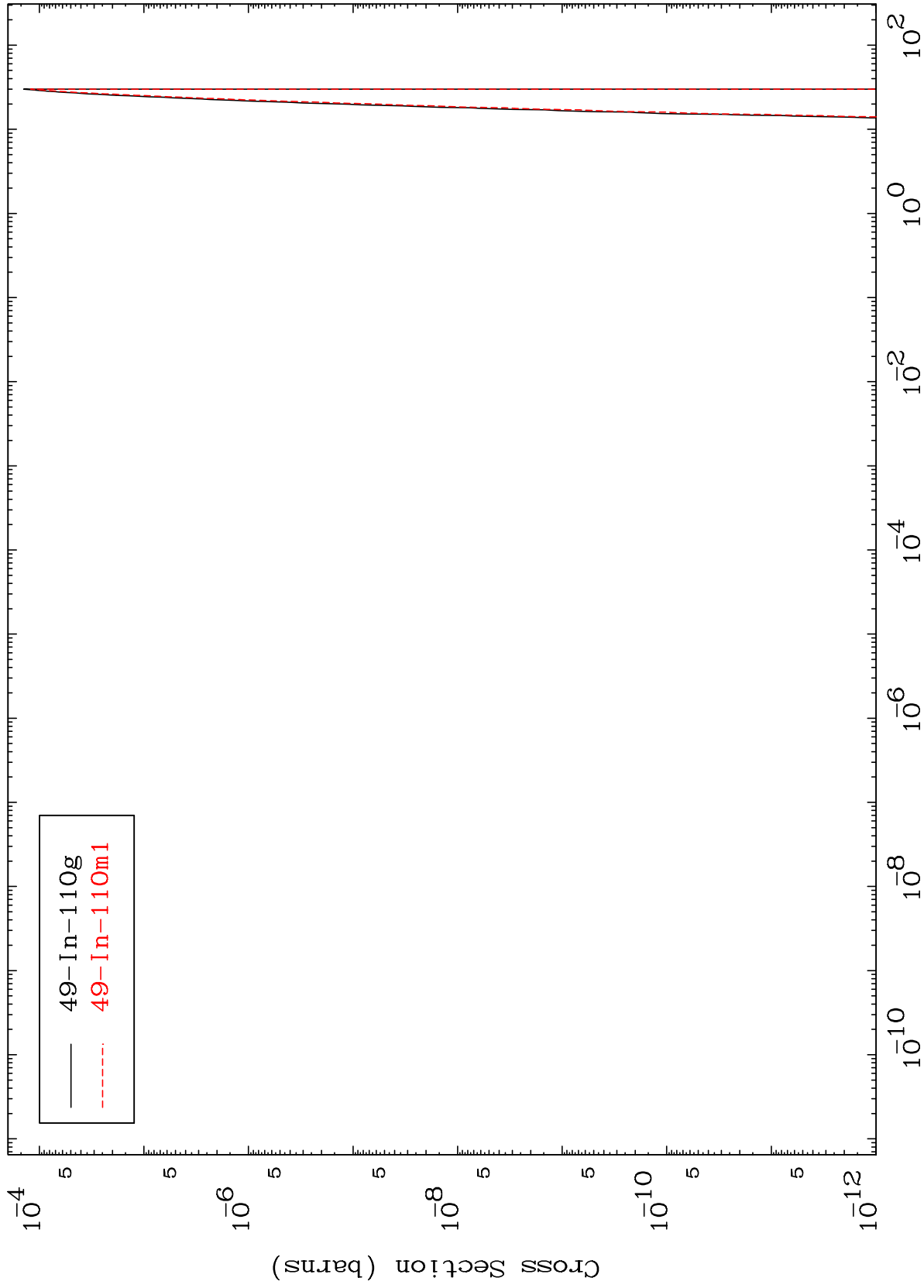
53-I -118

MAT 5299

(n,n') 2 $\alpha$

53-I -118

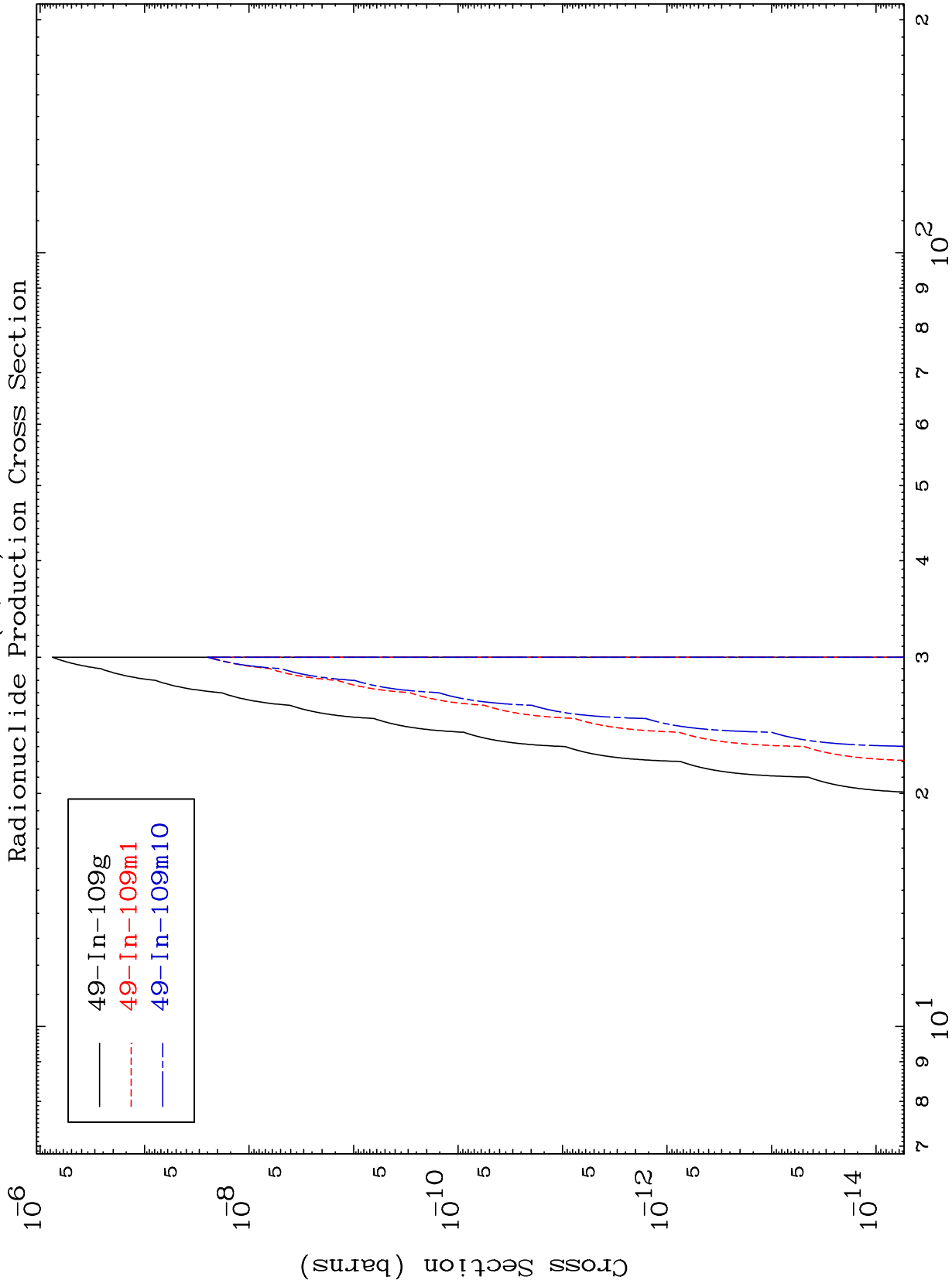
Radionuclide Production Cross Section



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Incident Energy (MeV)

53-I -118

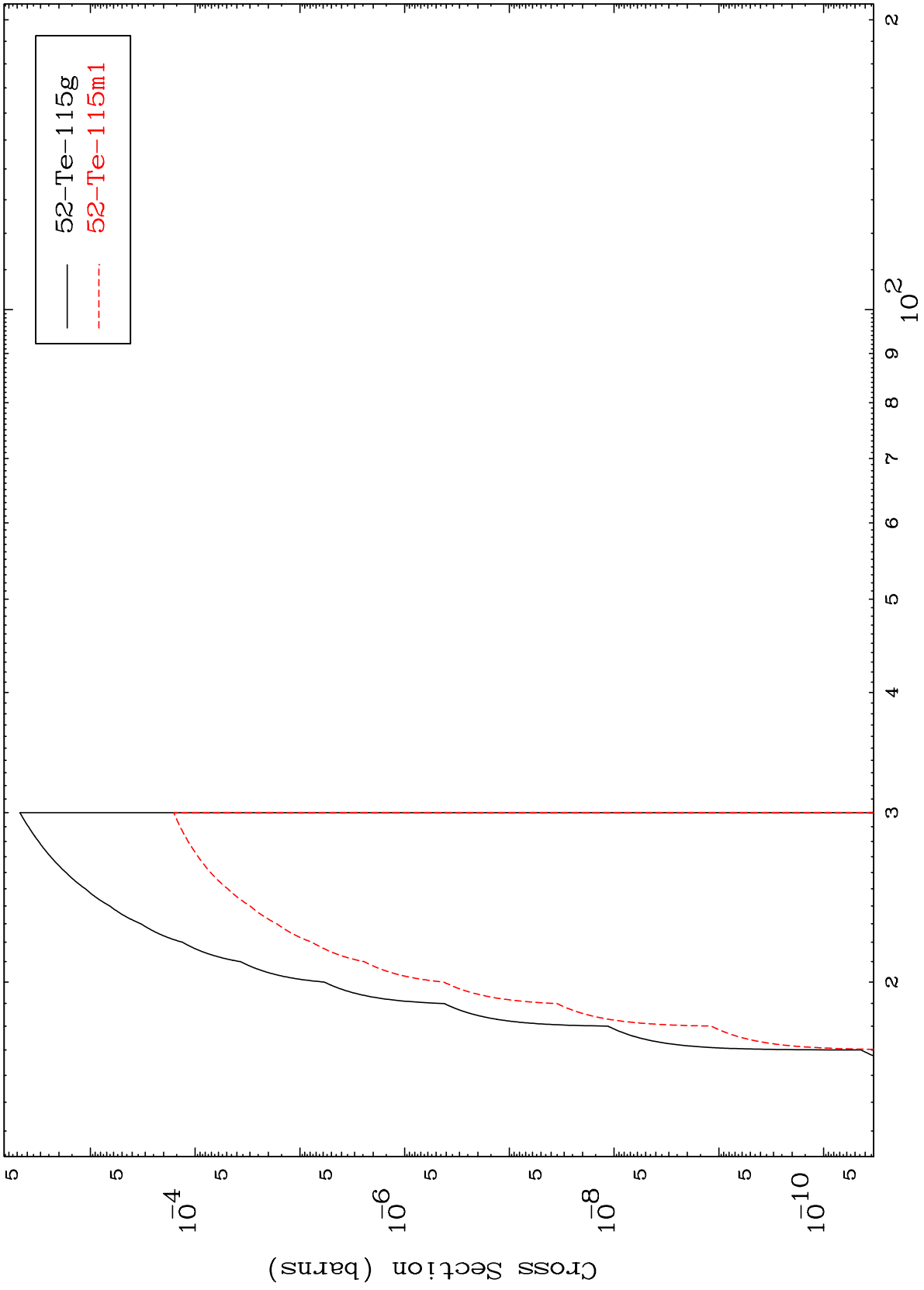


MAT 5299

(n,n') t

53-I -118

Radionuclide Production Cross Section

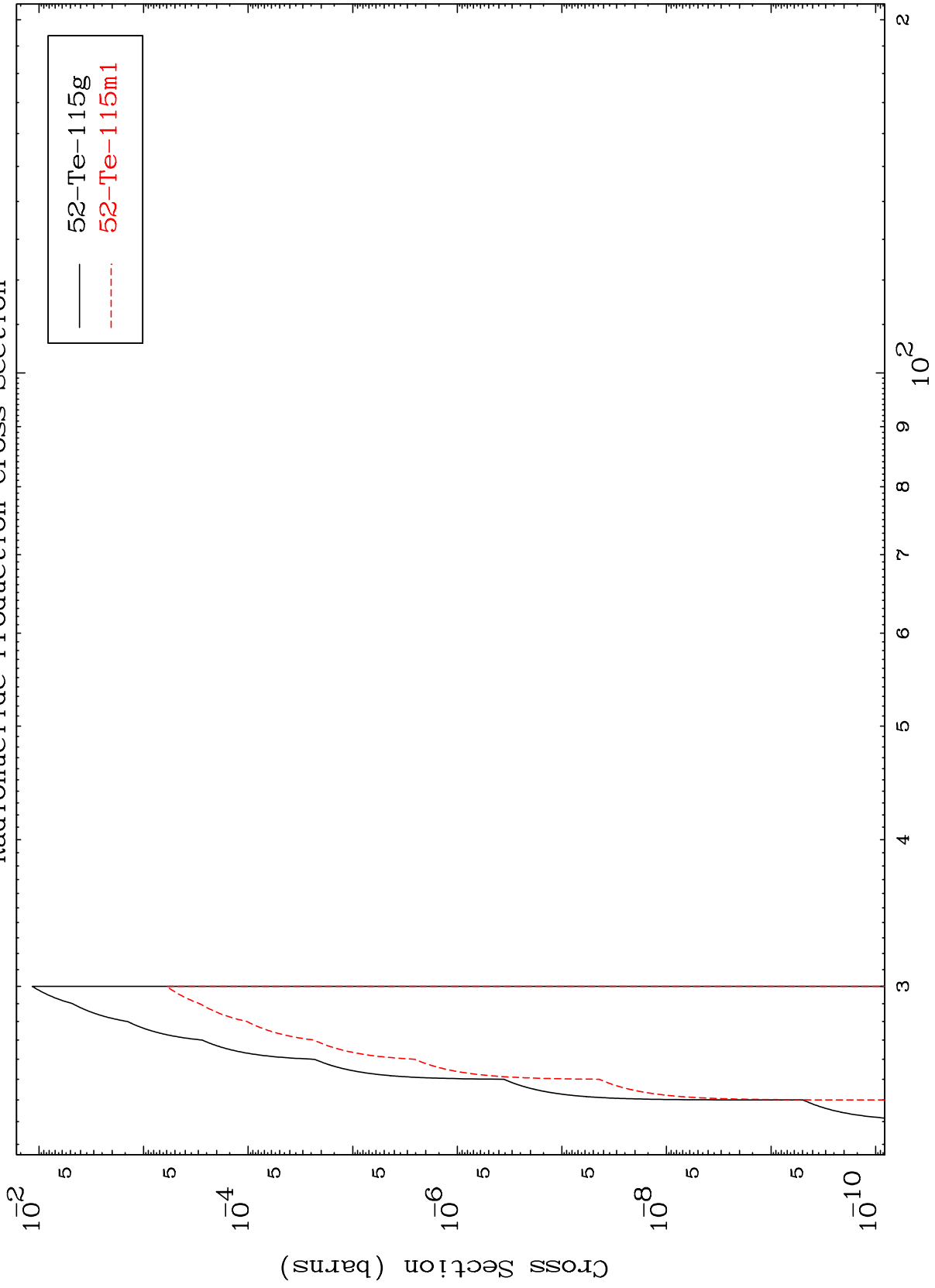


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Incident Energy (MeV)

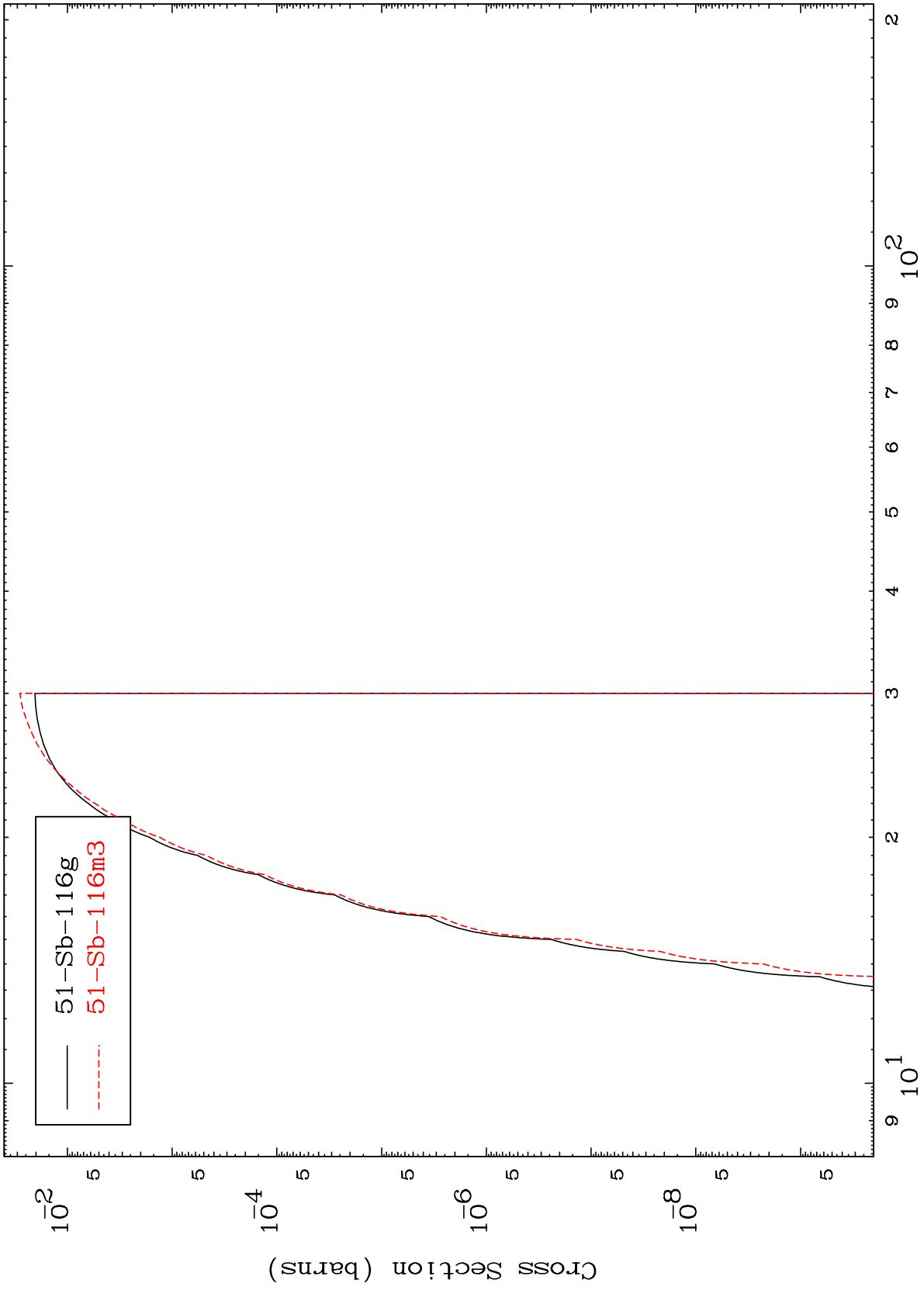
53-I -118

Radionuclide Production Cross Section

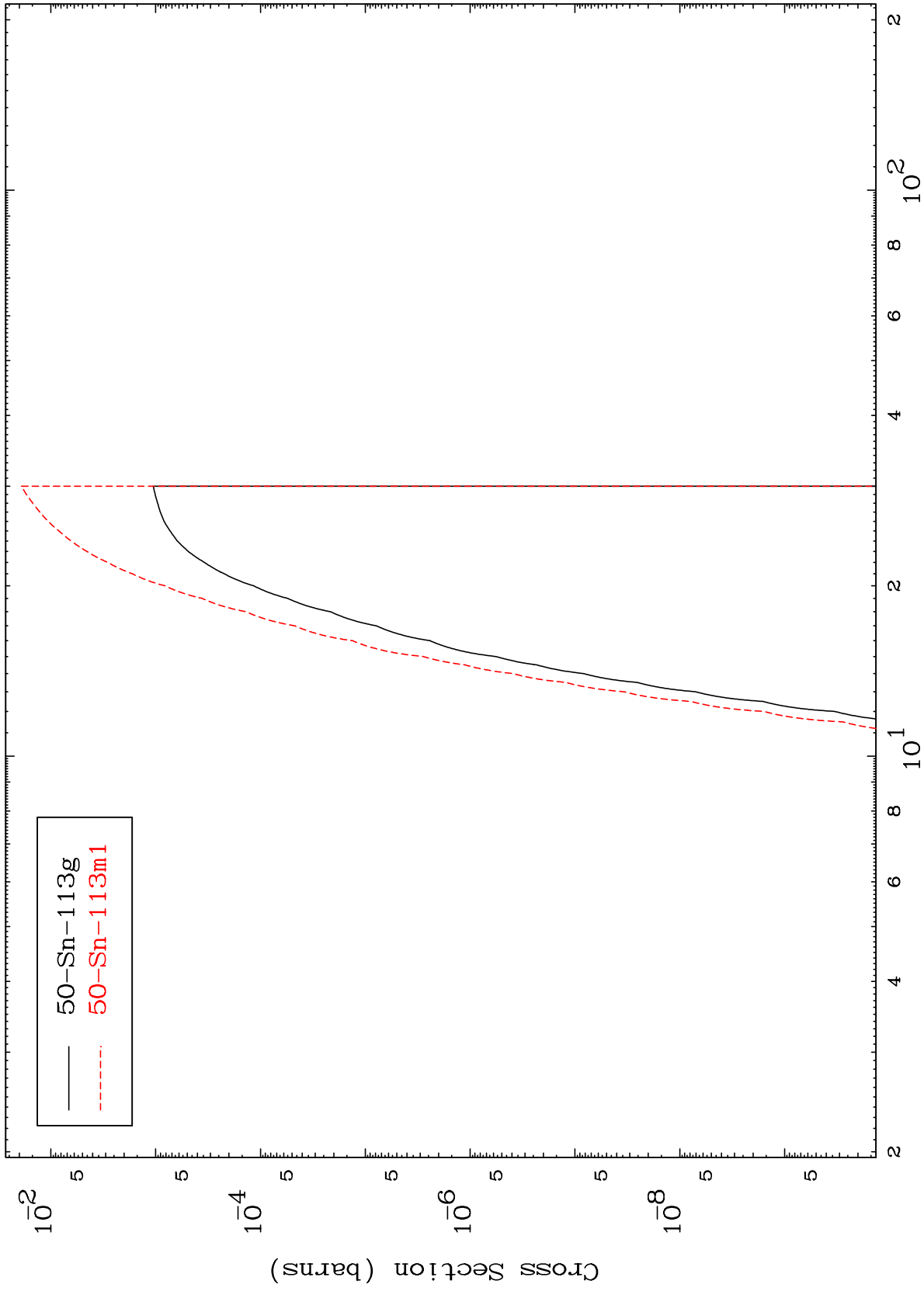




Radionuclide Production Cross Section



(n,n') p  $\alpha$   
Radionuclide Production Cross Section

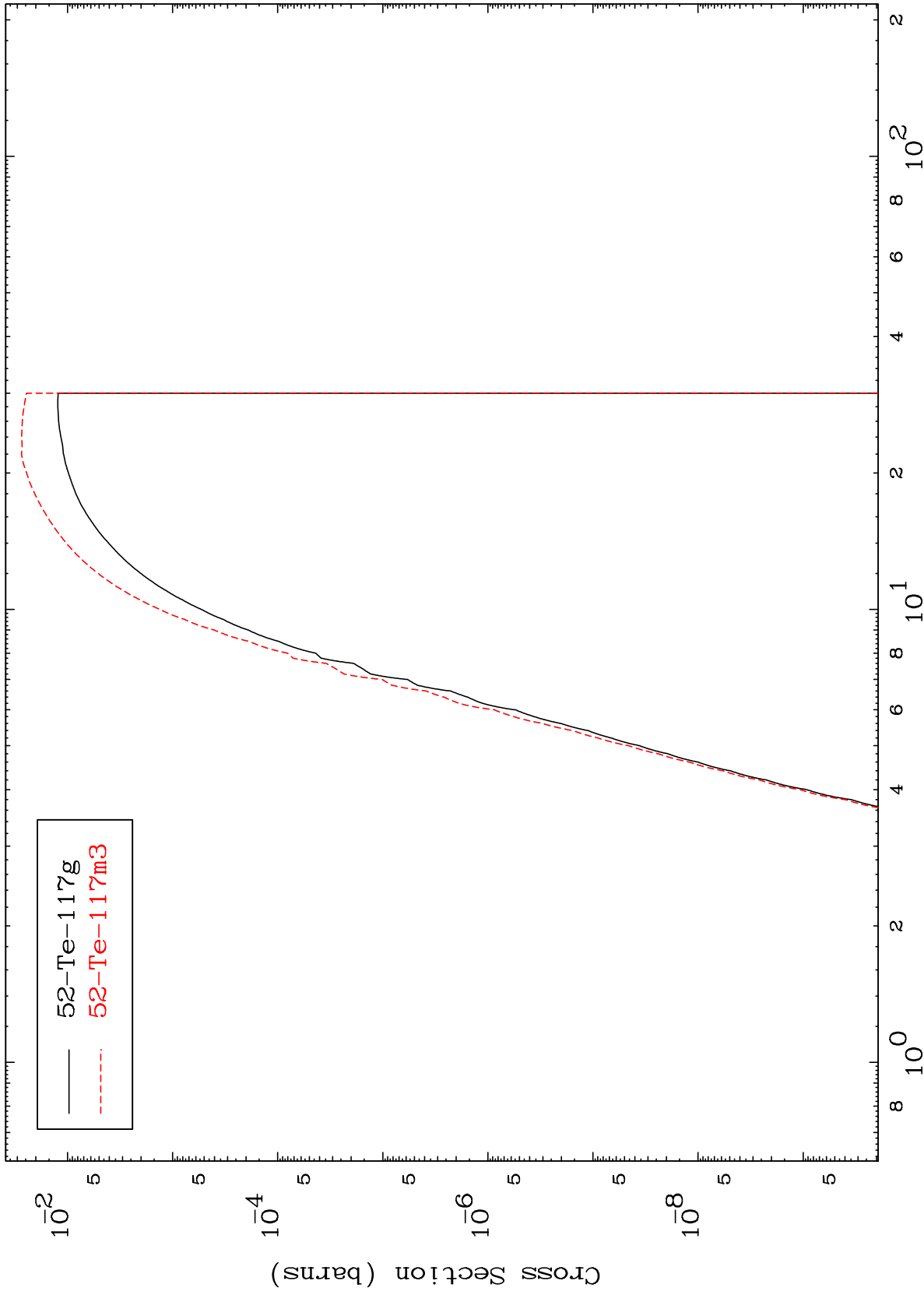


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(n,d)

53-I -118

Radionuclide Production Cross Section

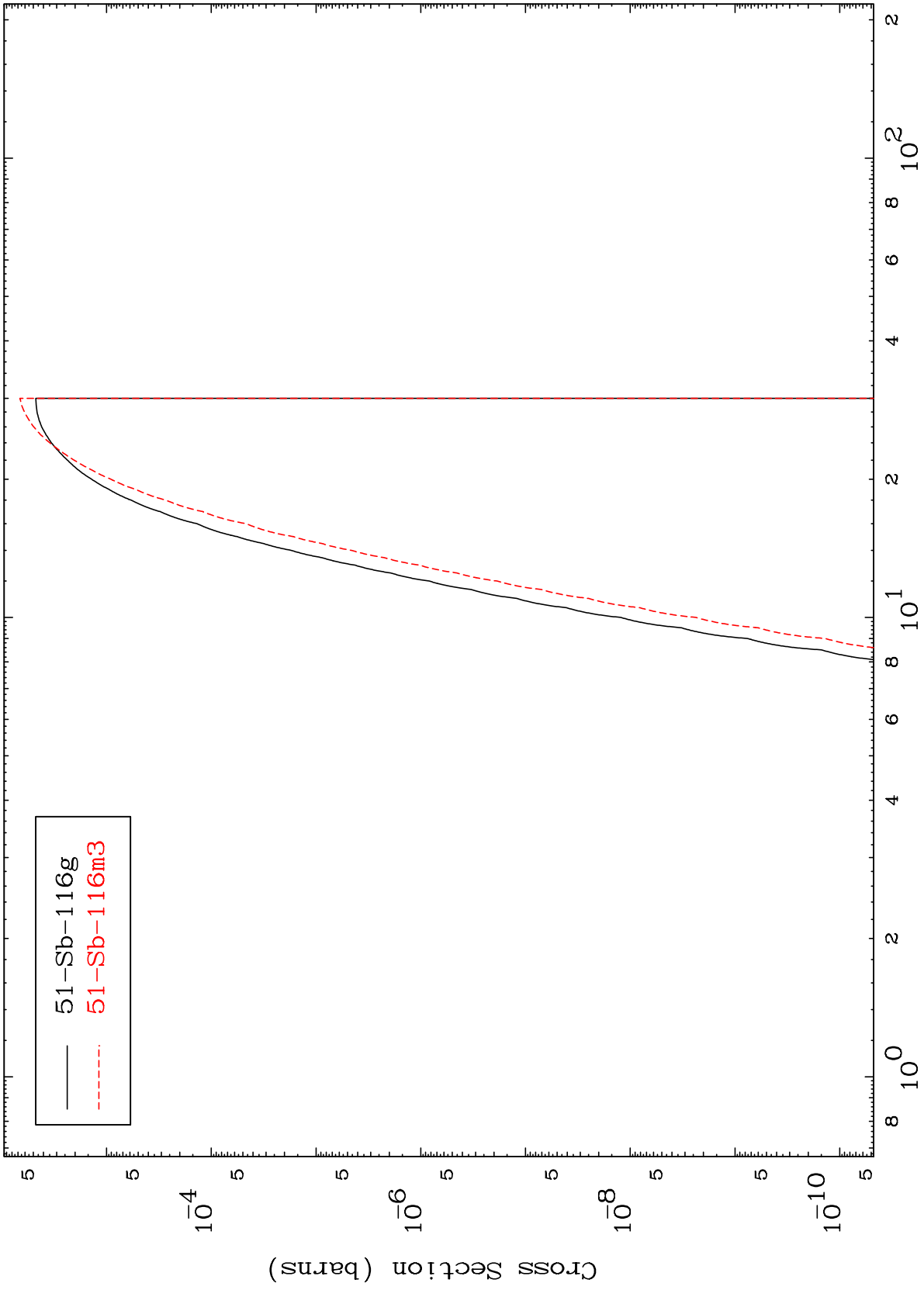


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Incident Energy (MeV)

53-I -118

Radionuclide Production Cross Section

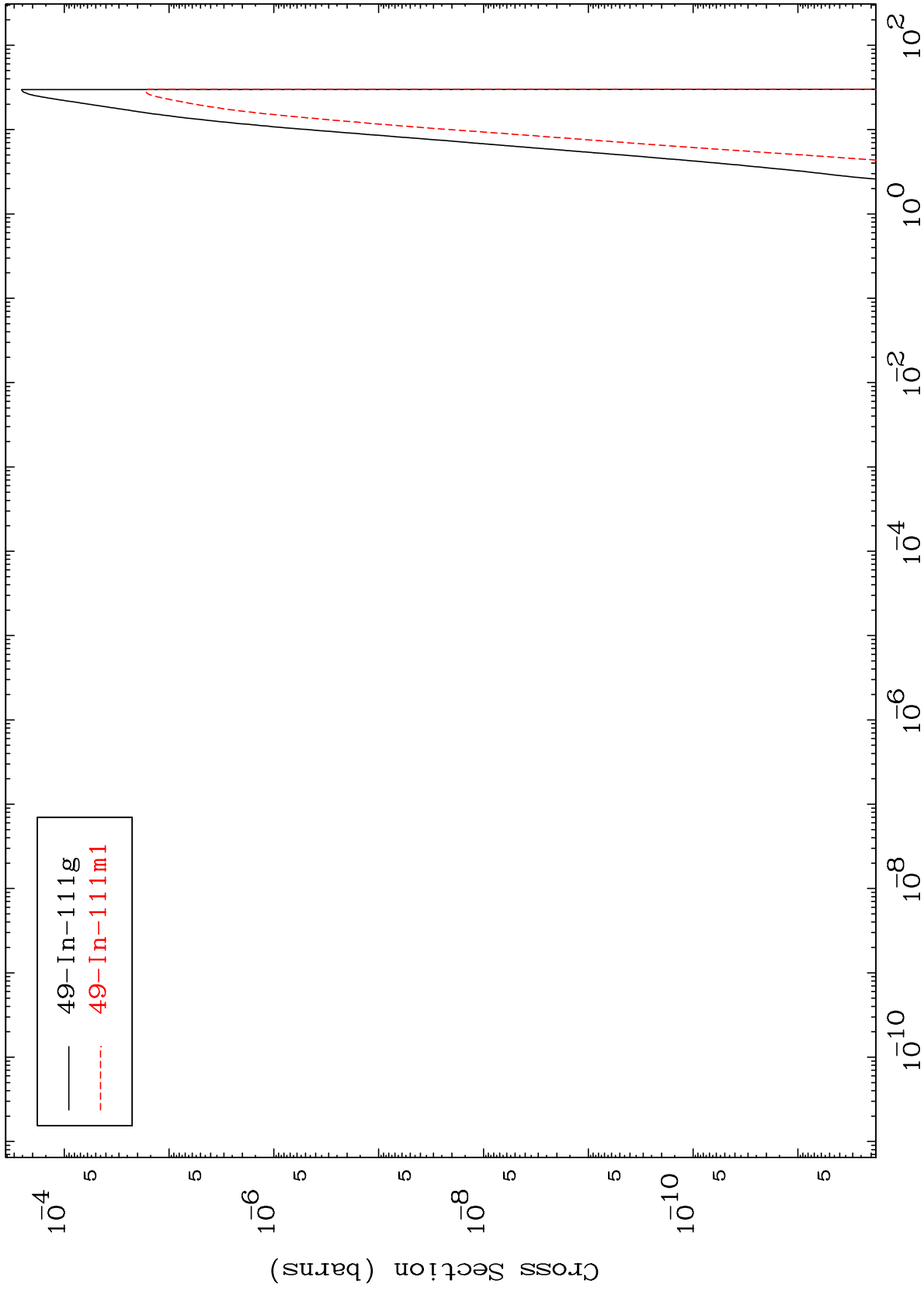


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(n,2α)

53-I -118

Radionuclide Production Cross Section



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Incident Energy (MeV)

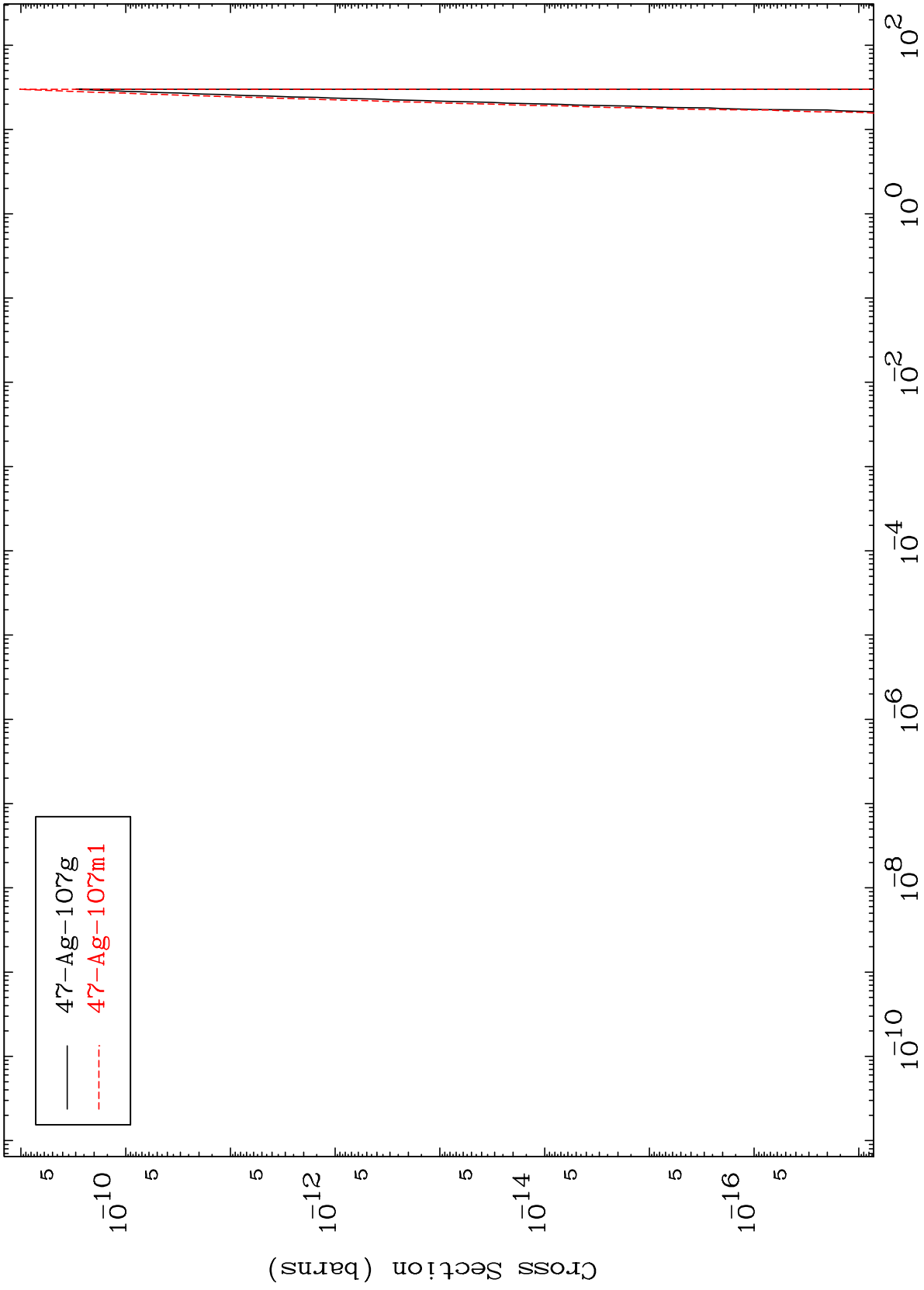
53-I -118

MAT 5299

(n,3α)

53-I -118

### Radionuclide Production Cross Section



29

Incident Energy (MeV)

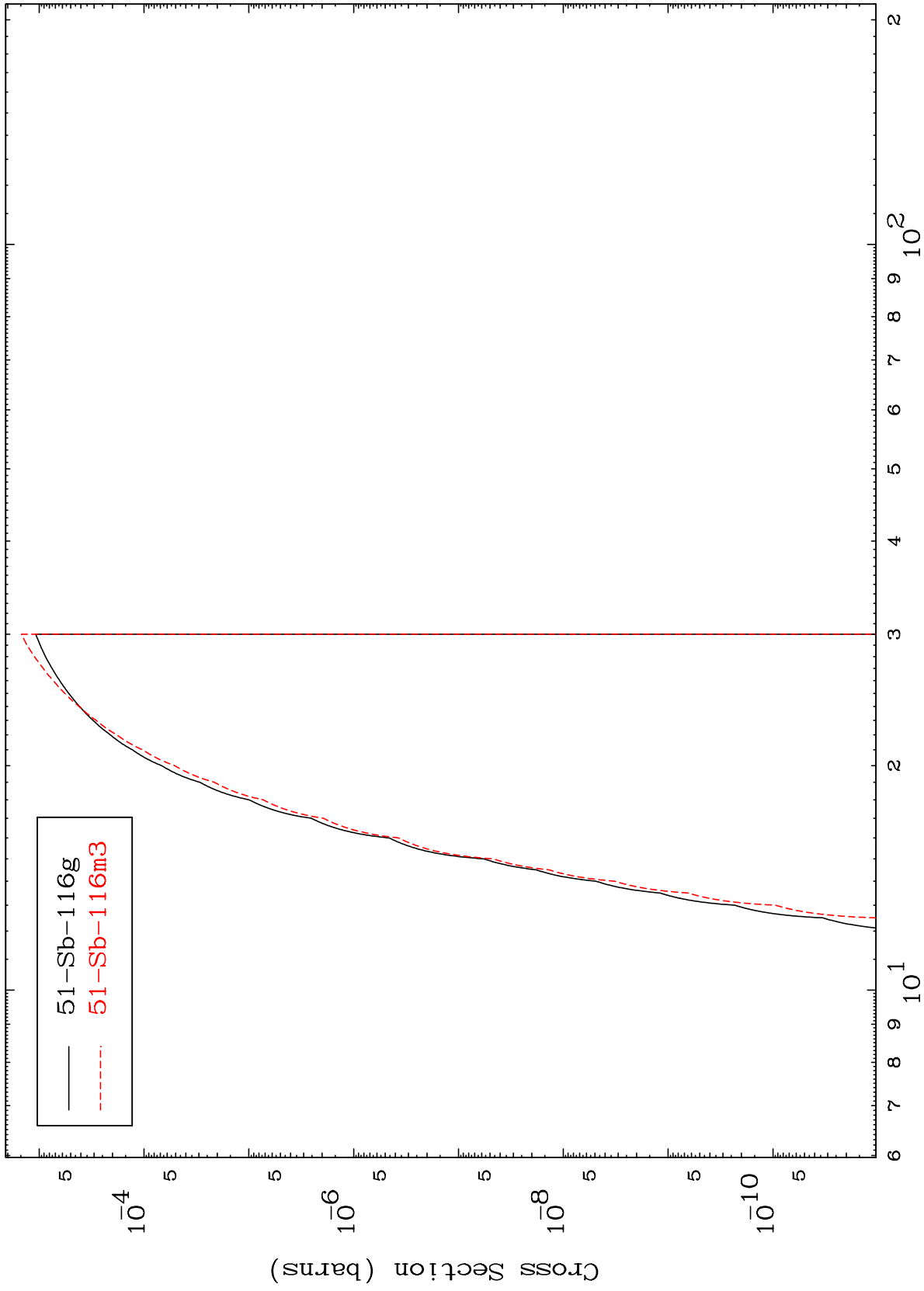
53-I -118

MAT 5299

(n,p) d

53-I -118

Radionuclide Production Cross Section



30

Incident Energy (MeV)

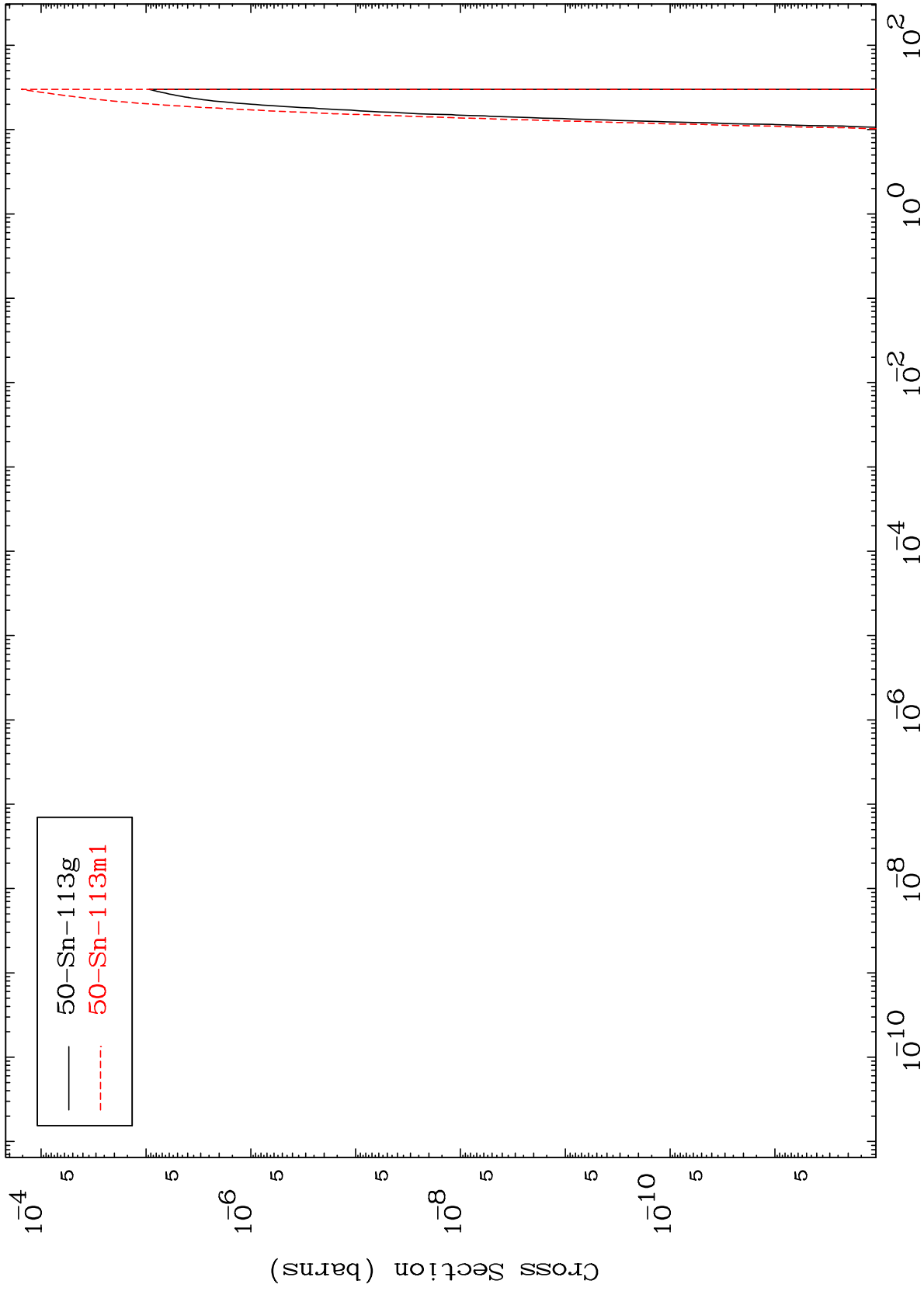
53-I -118

MAT 5299

(n,d)  $\alpha$

53-I -118

Radionuclide Production Cross Section



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Incident Energy (MeV)

53-I -118