

Program EVALPLOT
(Version 2018-1)

by

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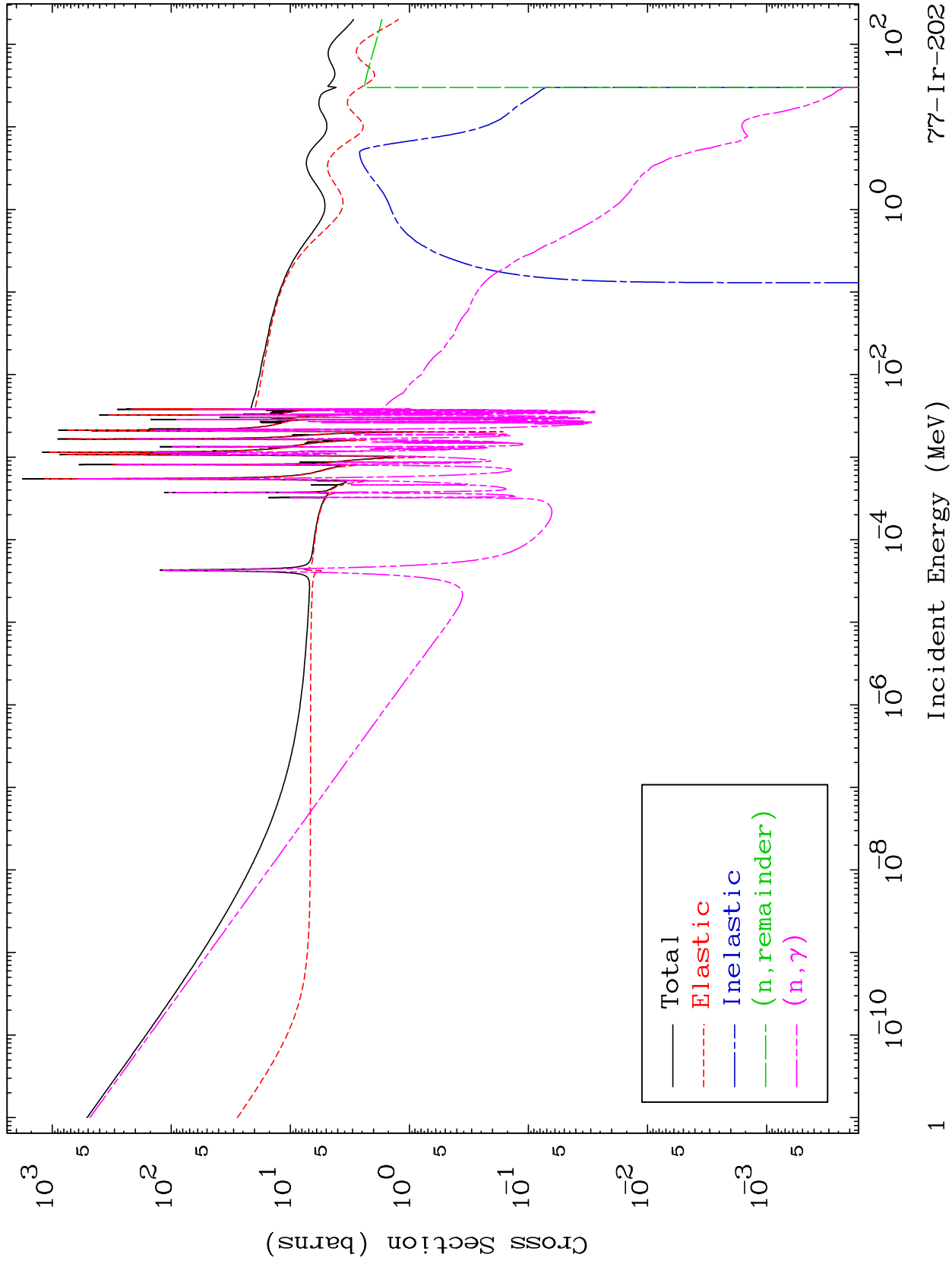
Press Mouse Button to Start

MAT 7758

Major

293 Kelvin Cross Sections

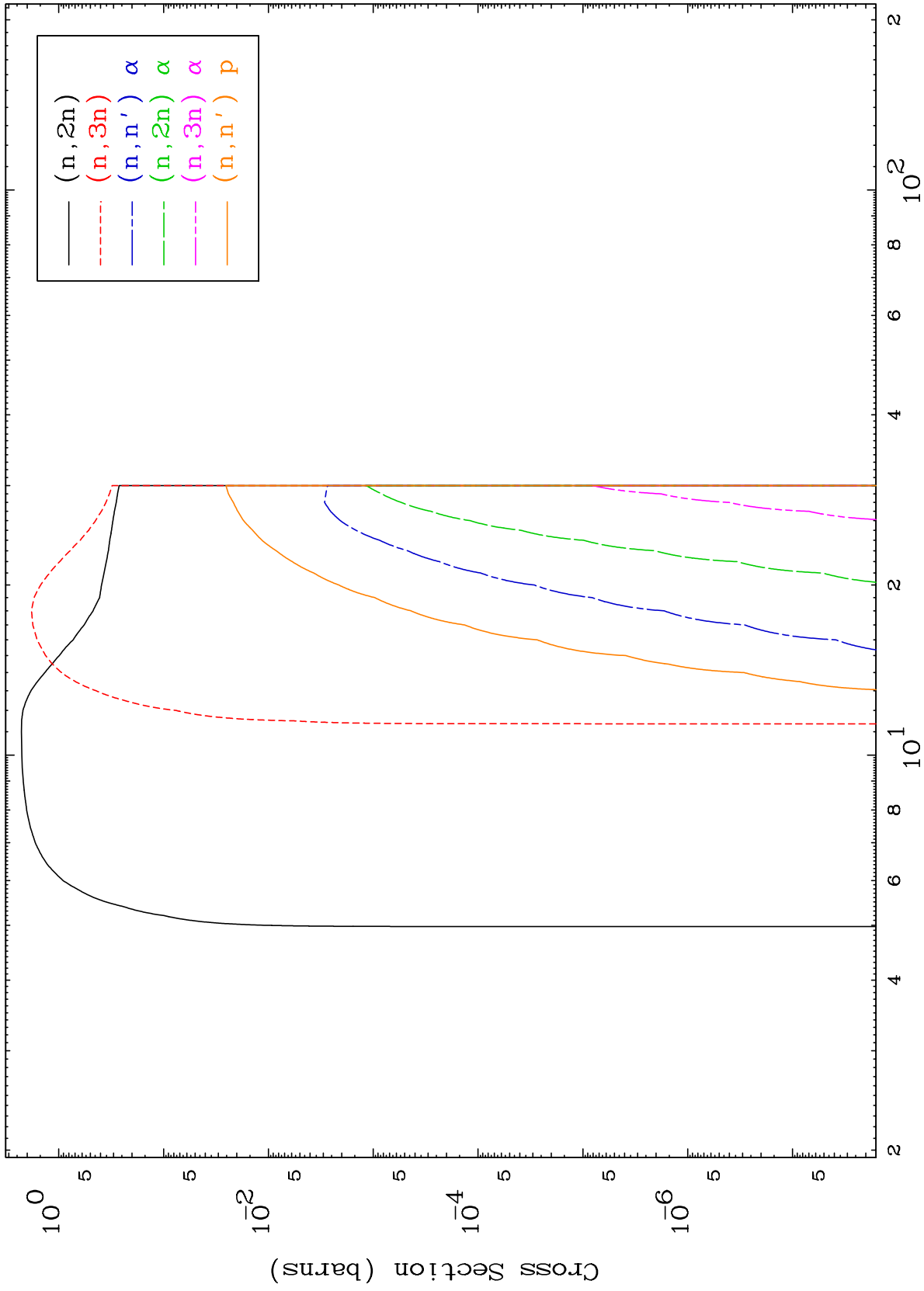
77-Ir-202



MAT 7758

Neutron Production
293 Kelvin Cross Sections

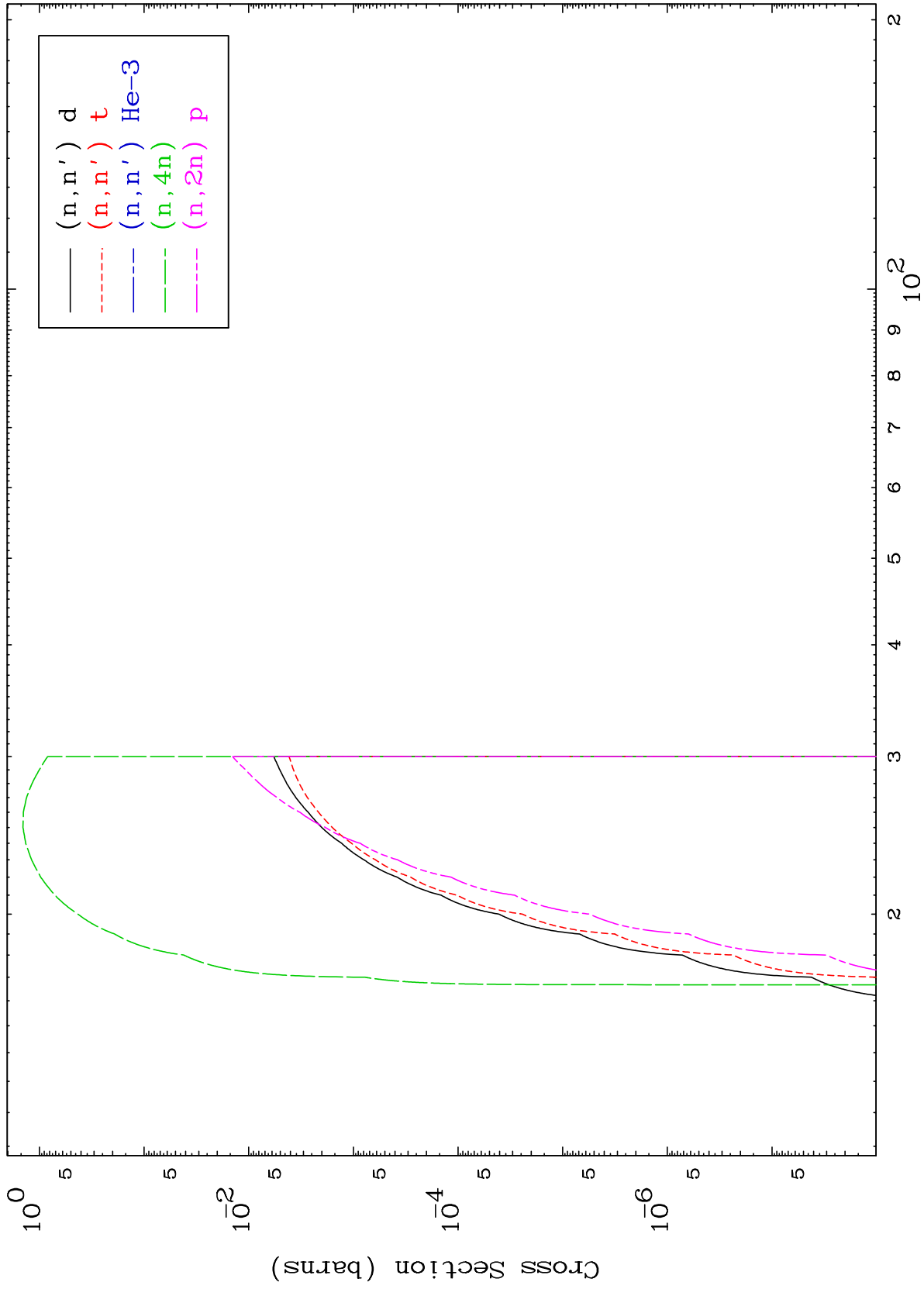
77-Ir-202

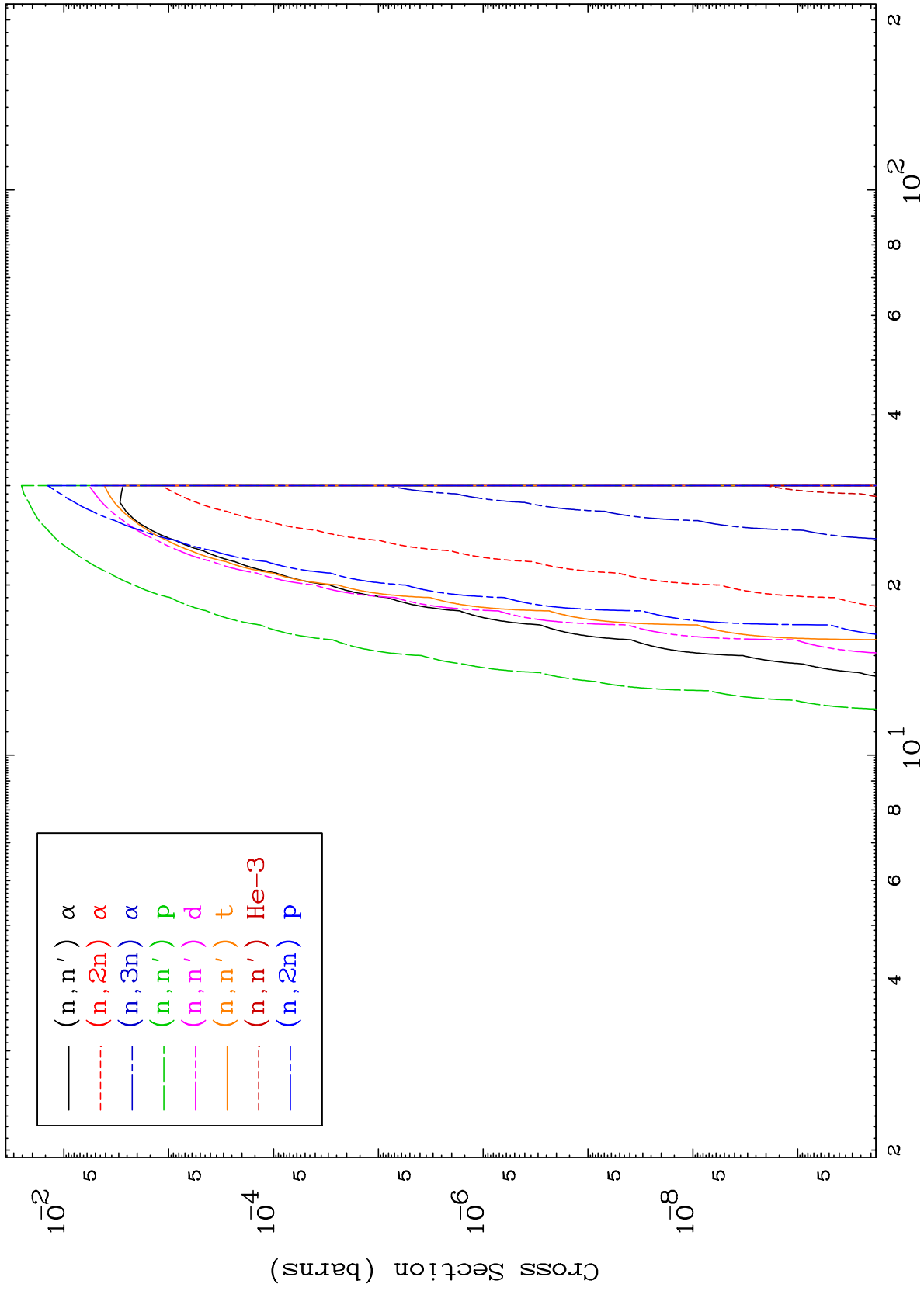


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Incident Energy (MeV)

77-Ir-202

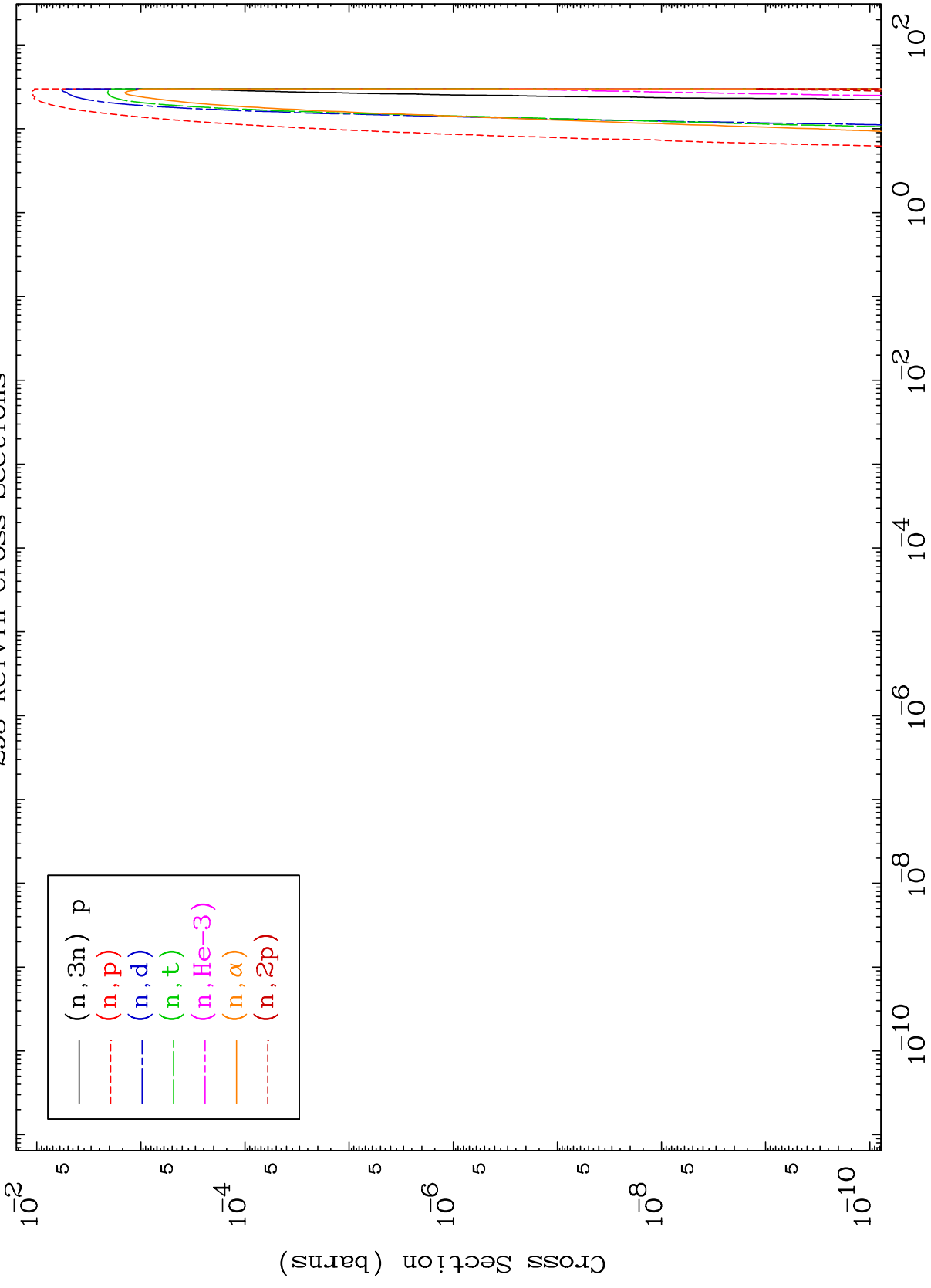




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293 Kelvin Cross Sections

77-Ir-202



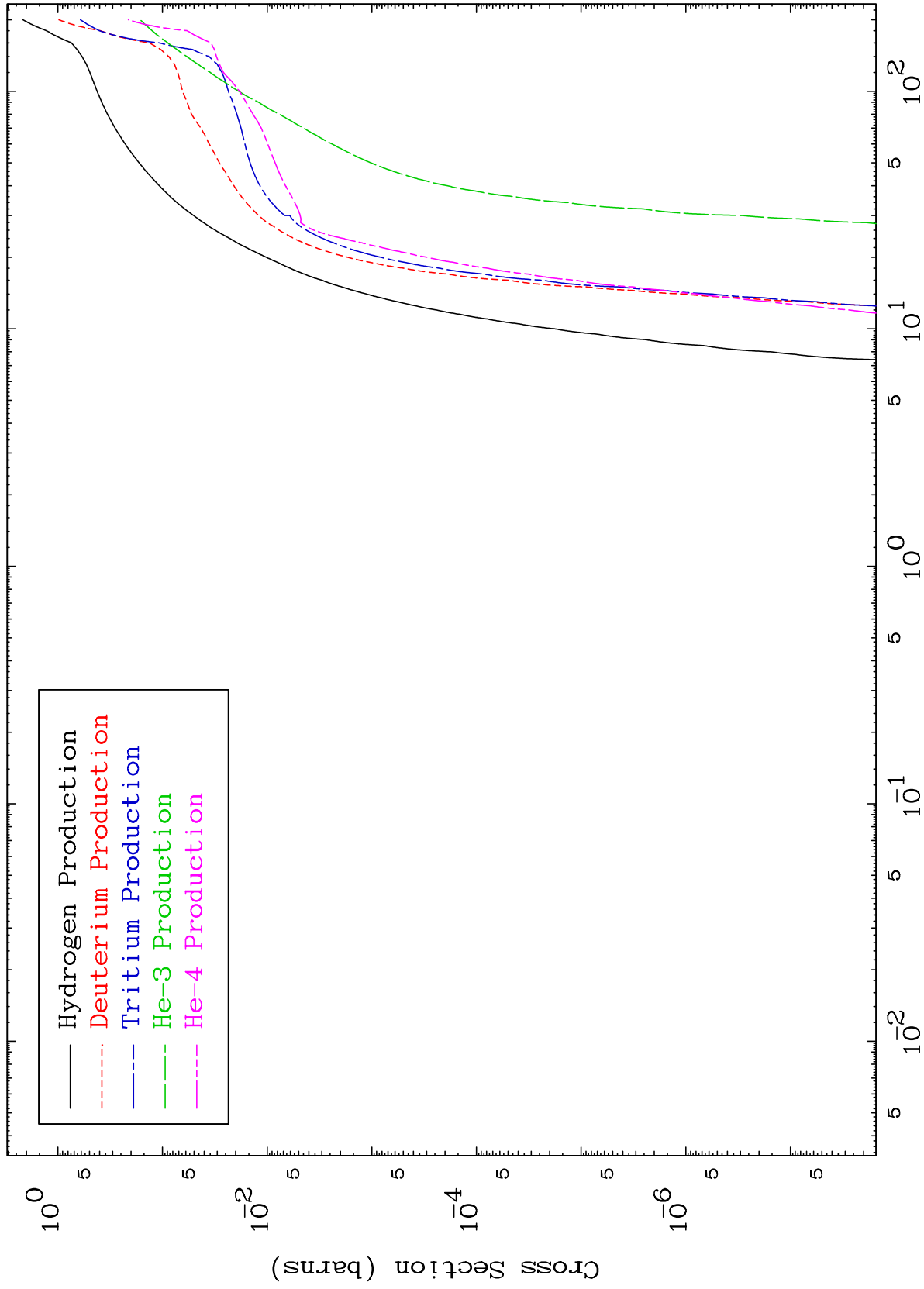
5

77-Ir-202

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Particle Production
293 Kelvin Cross Sections

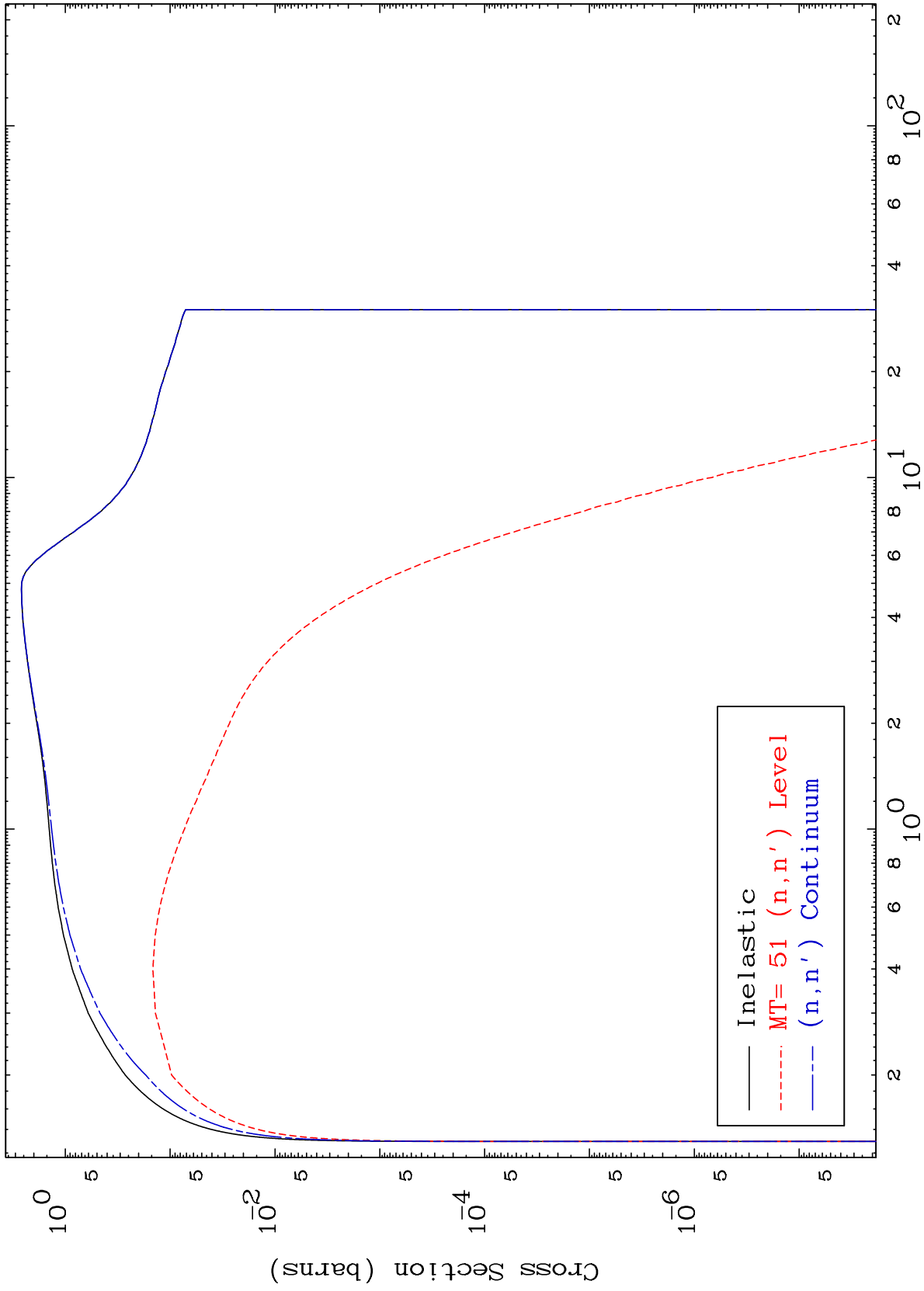
77-Ir-202



6

Incident Energy (MeV)

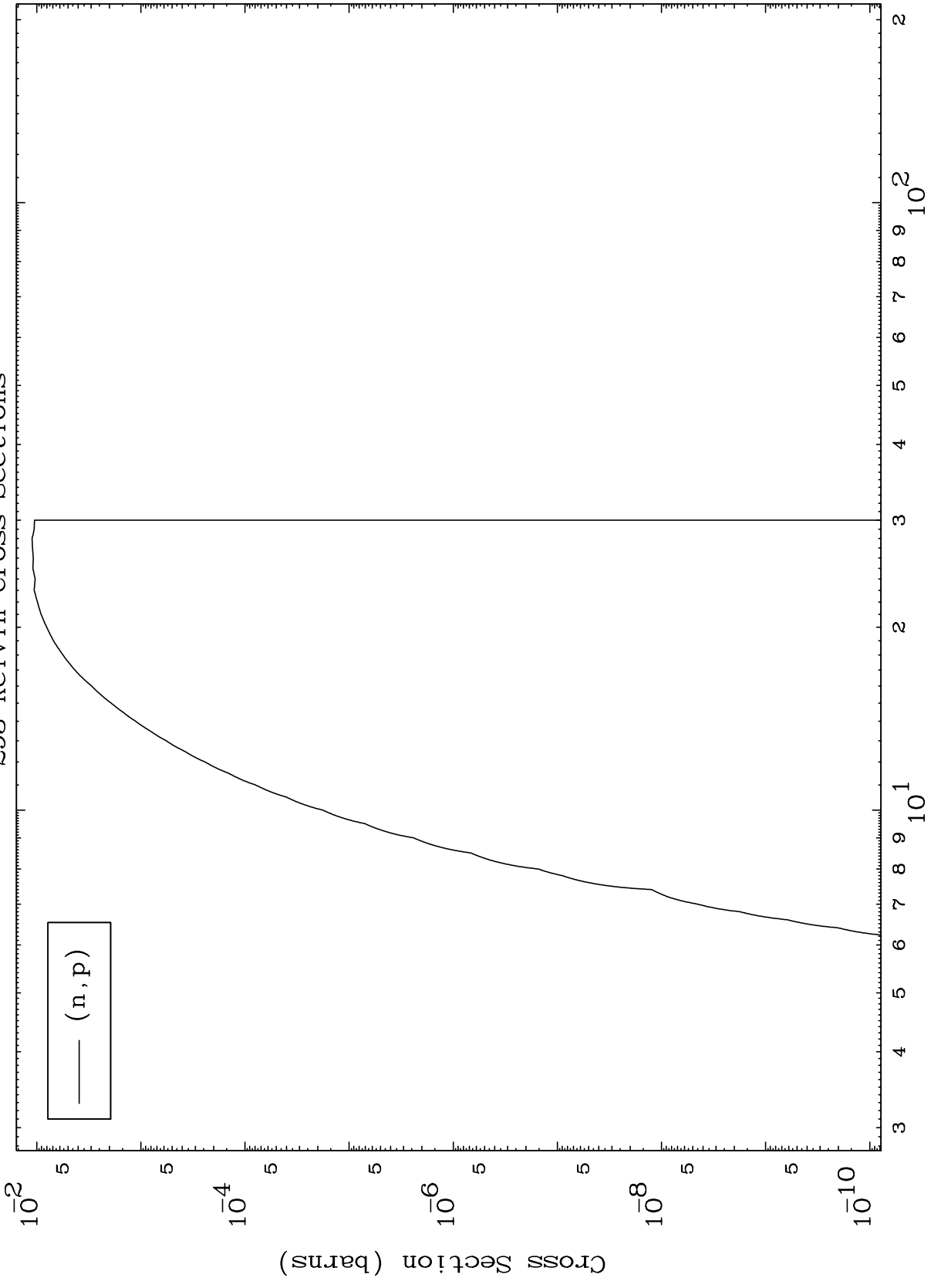
77-Ir-202



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(n,p) Levels
293 Kelvin Cross Sections

77-Ir-202



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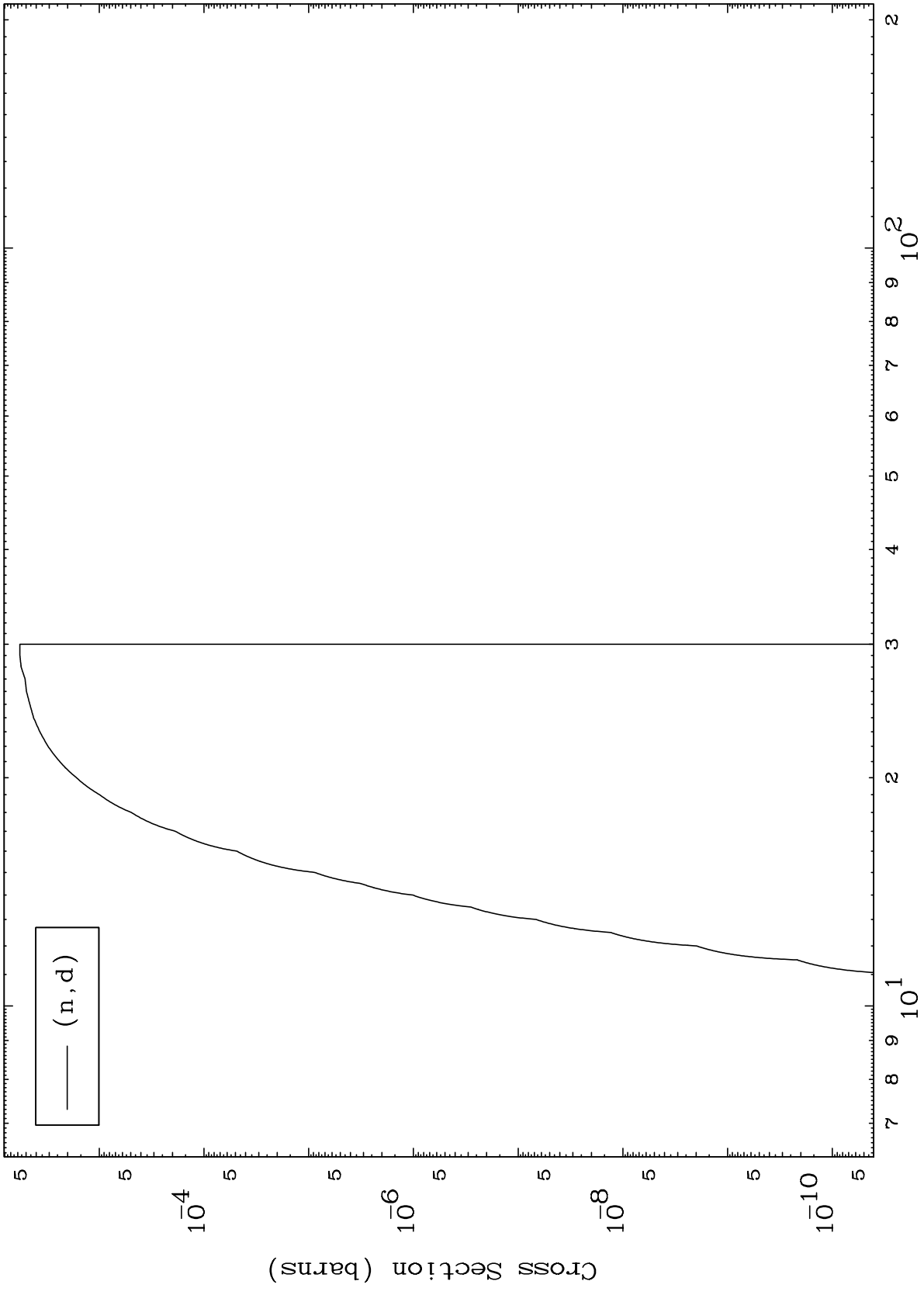
Incident Energy (MeV)

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(n,d) Levels
293 Kelvin Cross Sections

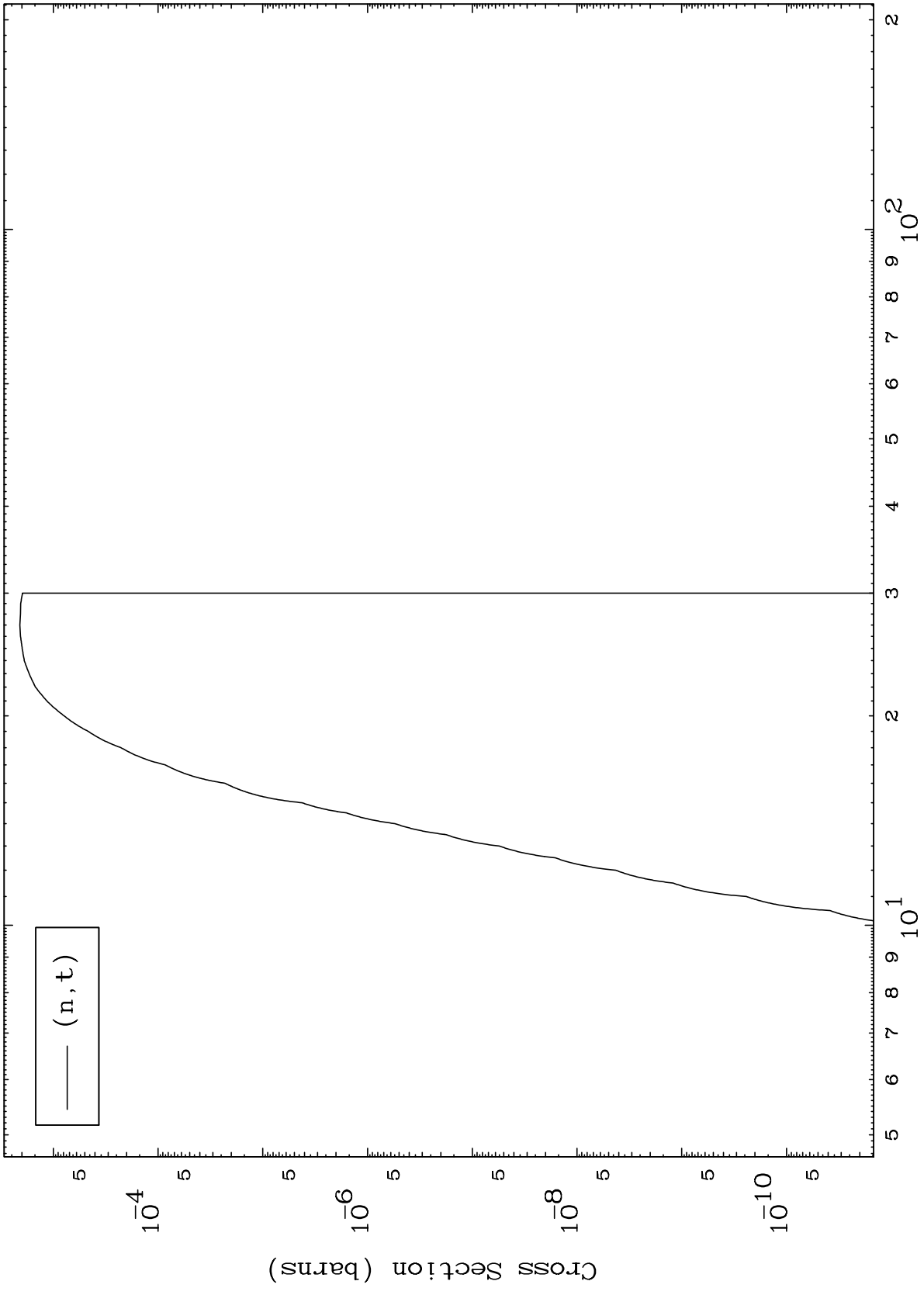
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(n,t) Levels
293 Kelvin Cross Sections

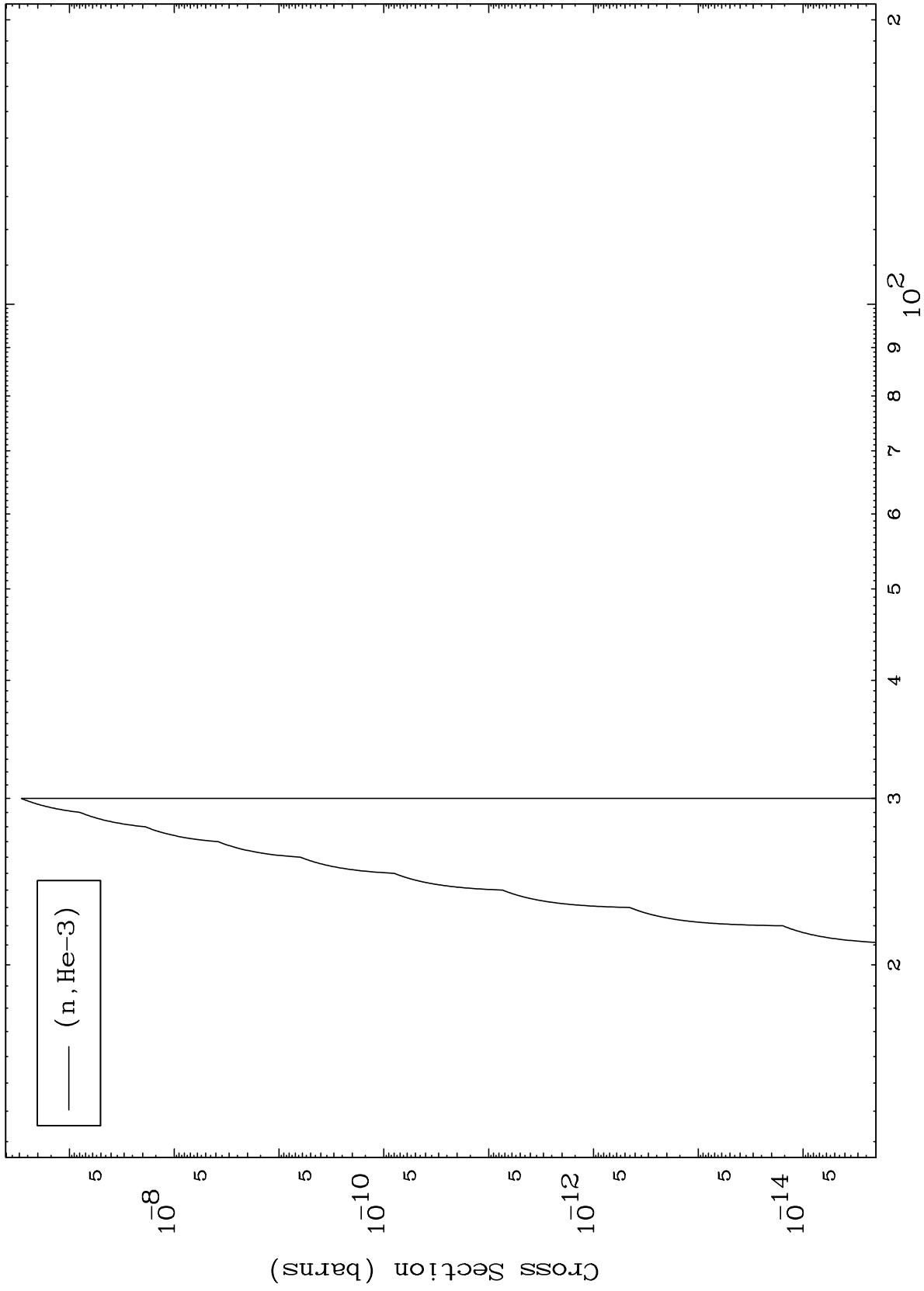
77-Ir-202



10

Incident Energy (MeV)

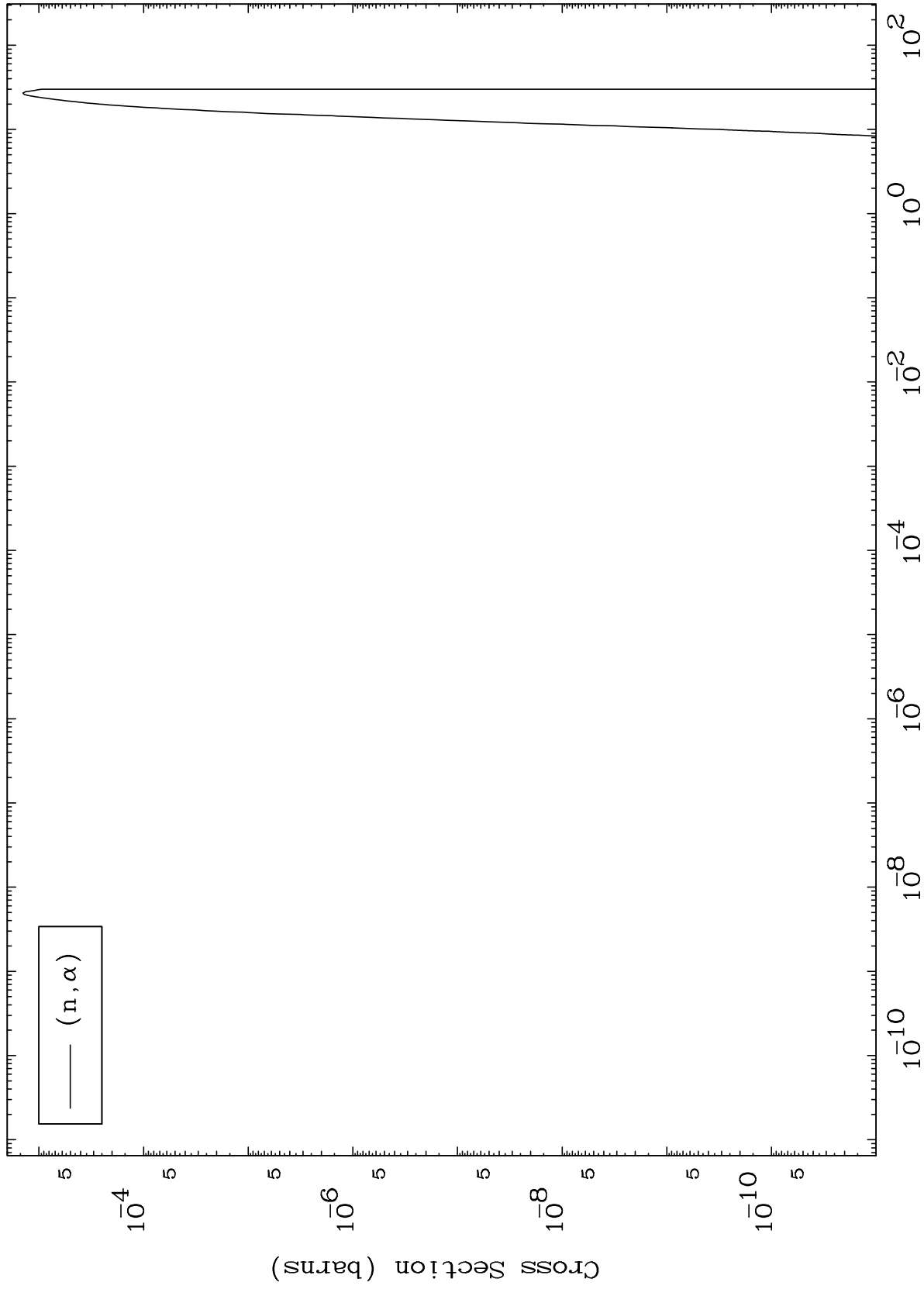
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(n,α) Levels
293 Kelvin Cross Sections

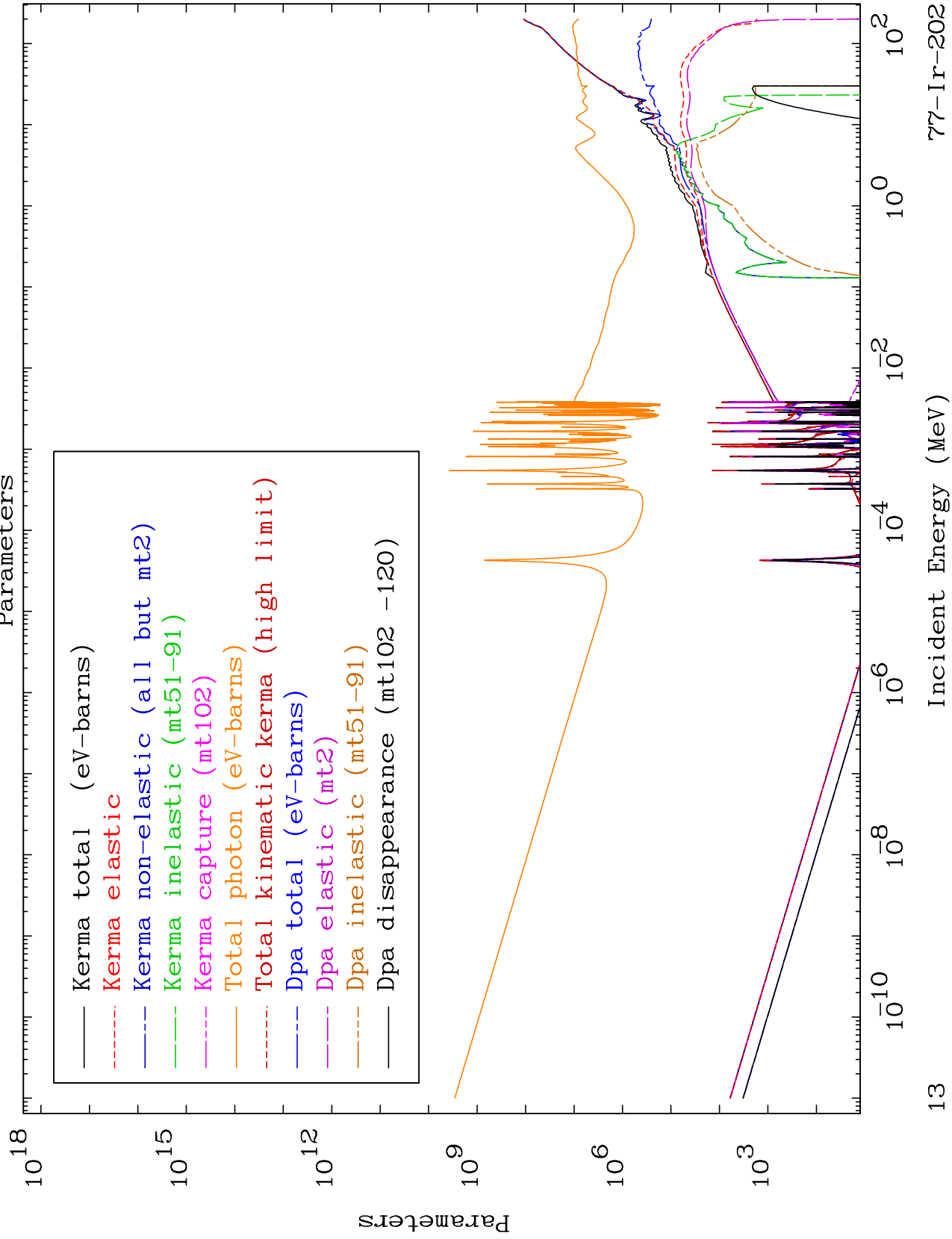
77-Ir-202



12

Incident Energy (MeV)

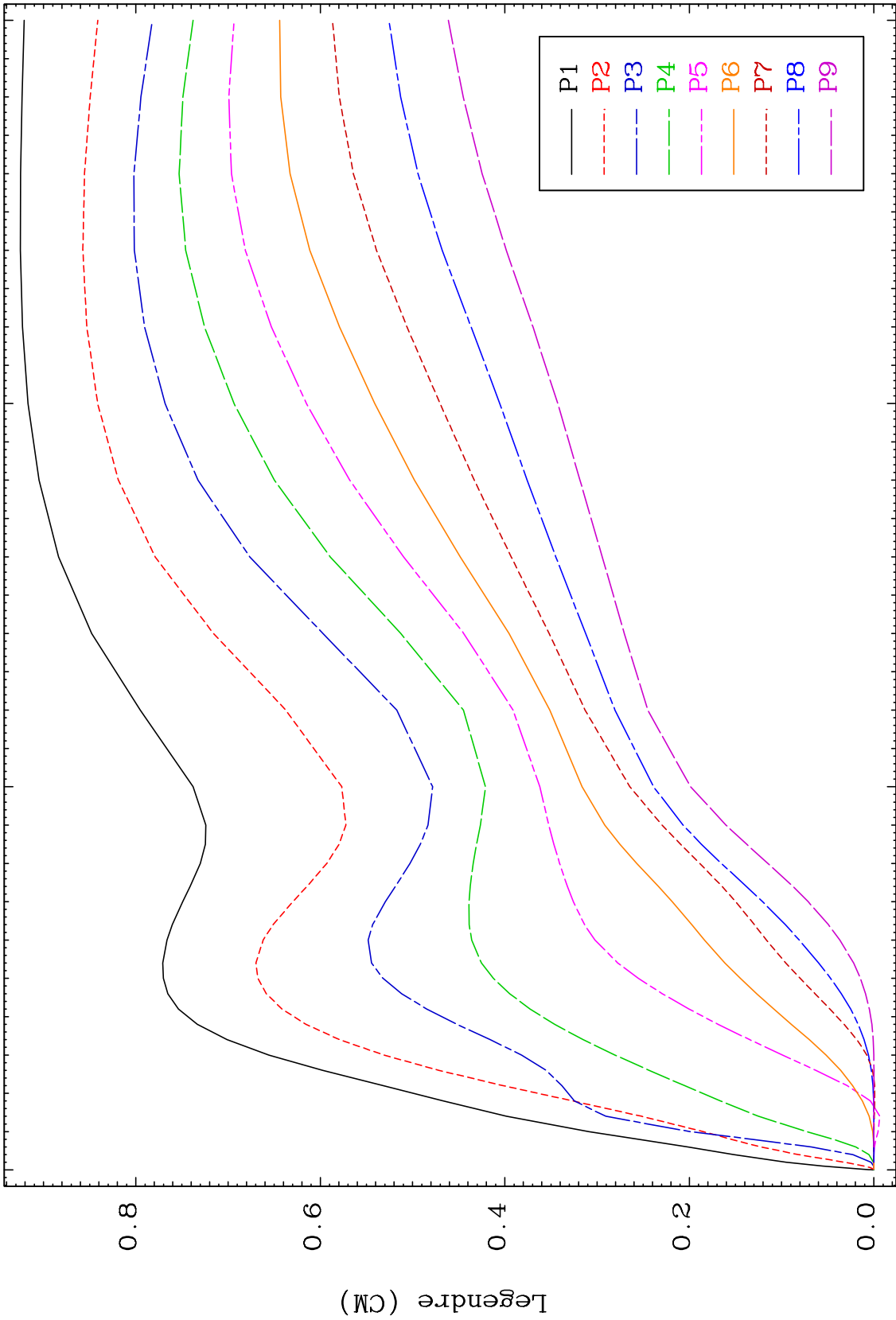
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77-Ir-202

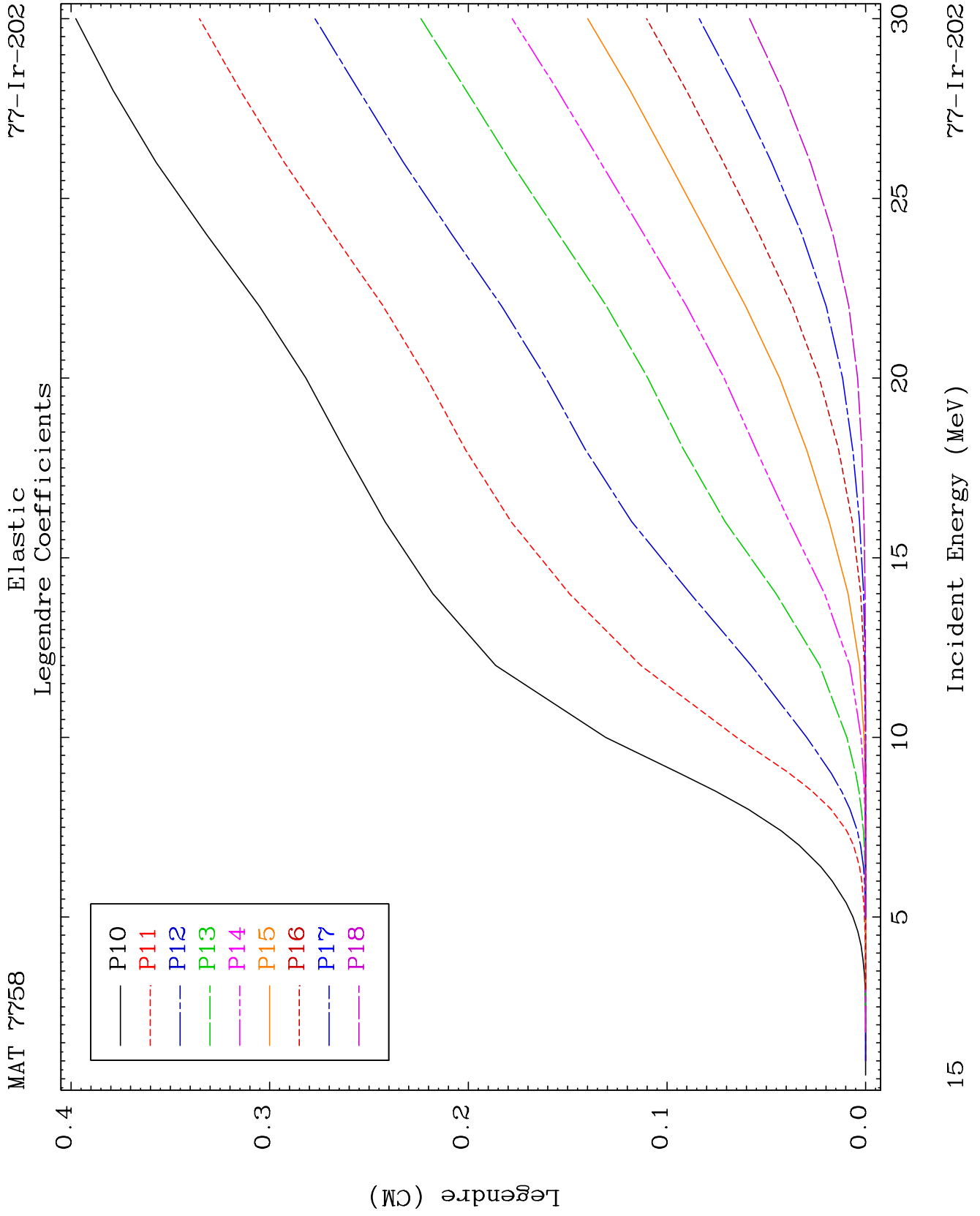
Elastic Legendre Coefficients



14

Incident Energy (MeV)

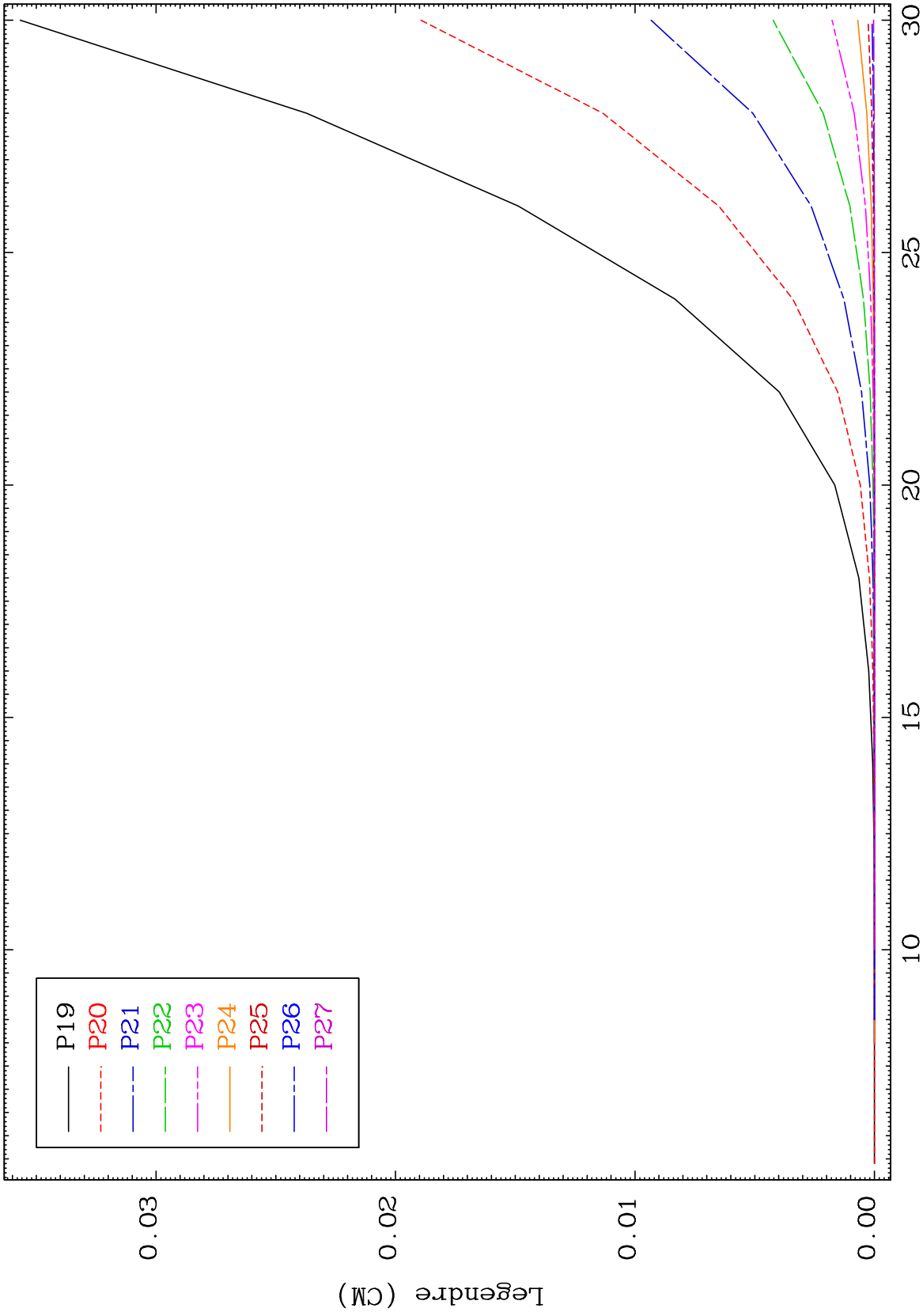
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MAT 7758

Elastic Legendre Coefficients

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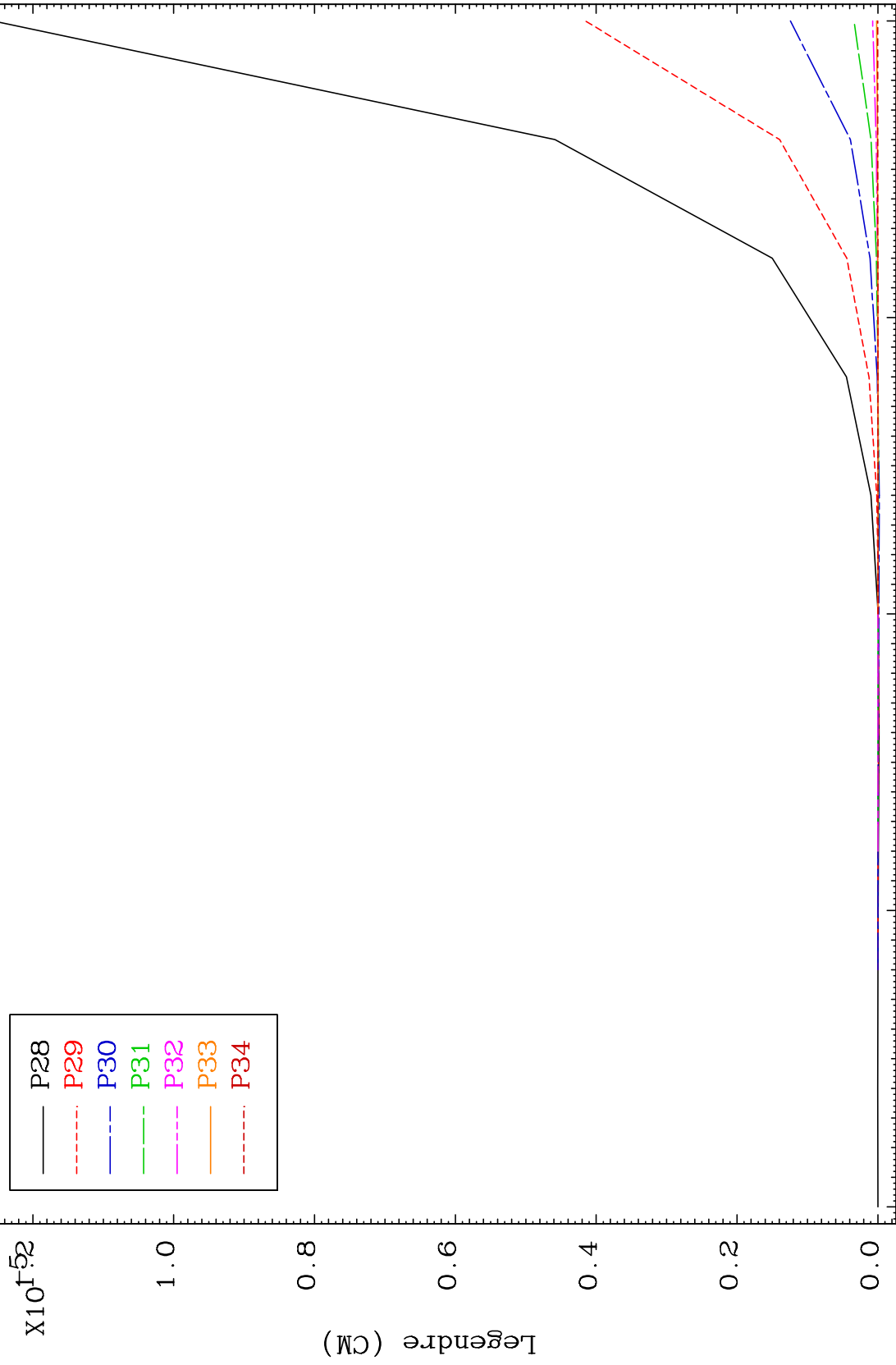
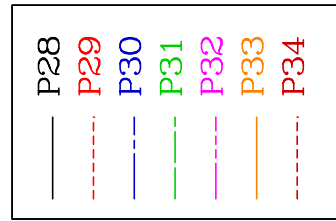
Incident Energy (MeV)

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Elastic Legendre Coefficients

77-Ir-202



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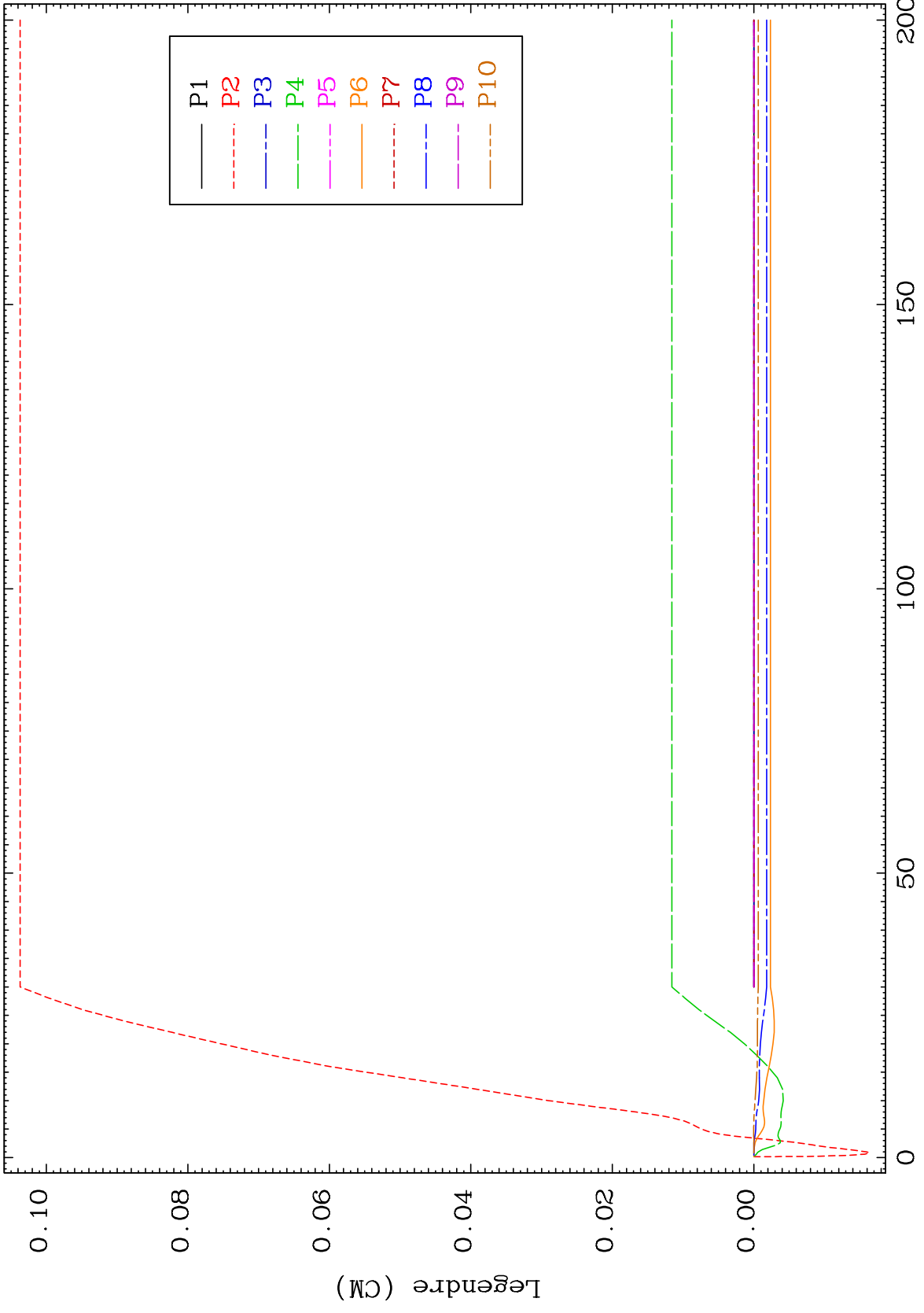
Incident Energy (MeV)

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MT= 51 (n,n') Level
Legendre Coefficients

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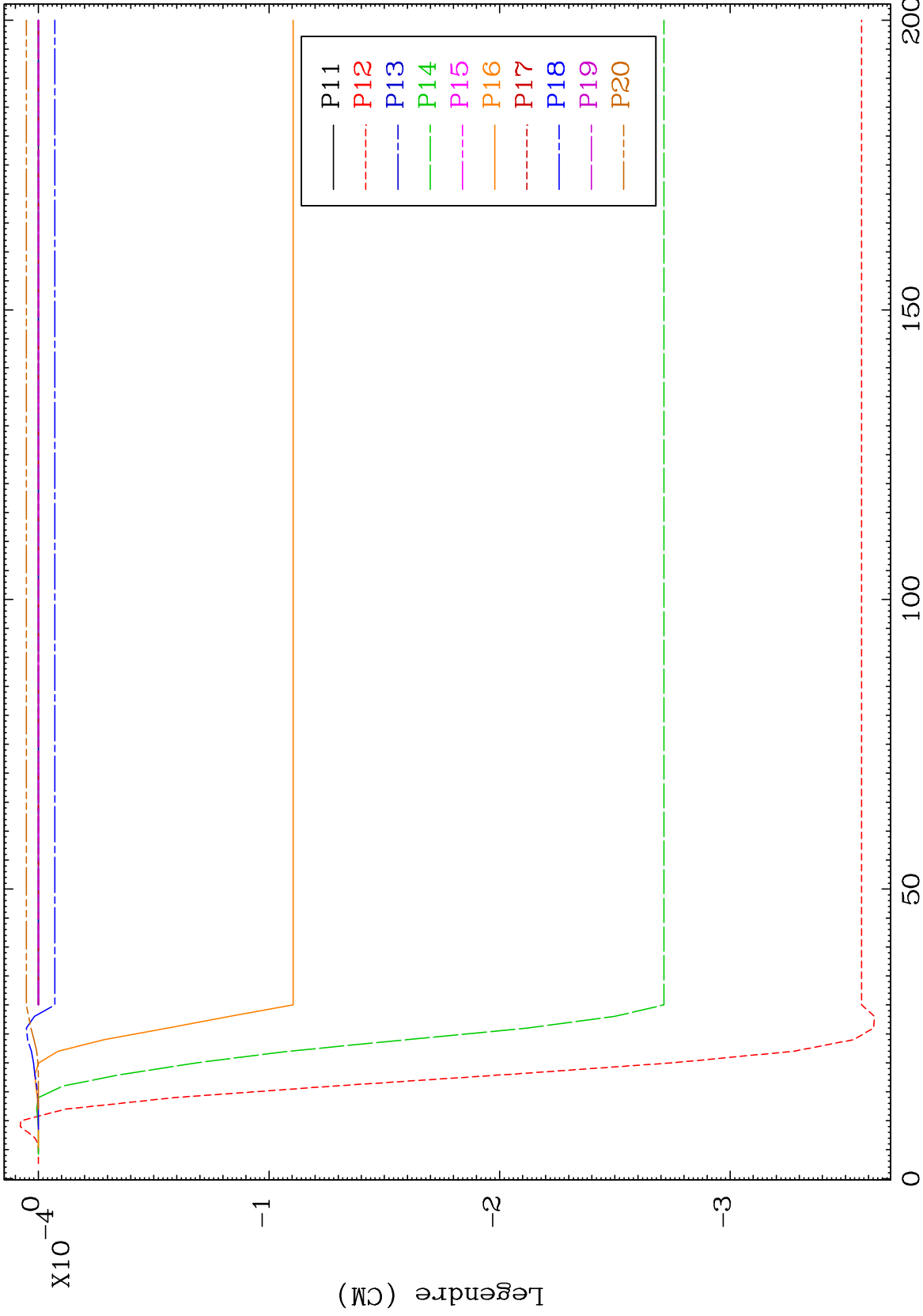
Incident Energy (MeV)

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MAT 7758

MT= 51 (n,n') Level
Legendre Coefficients

77-Ir-202



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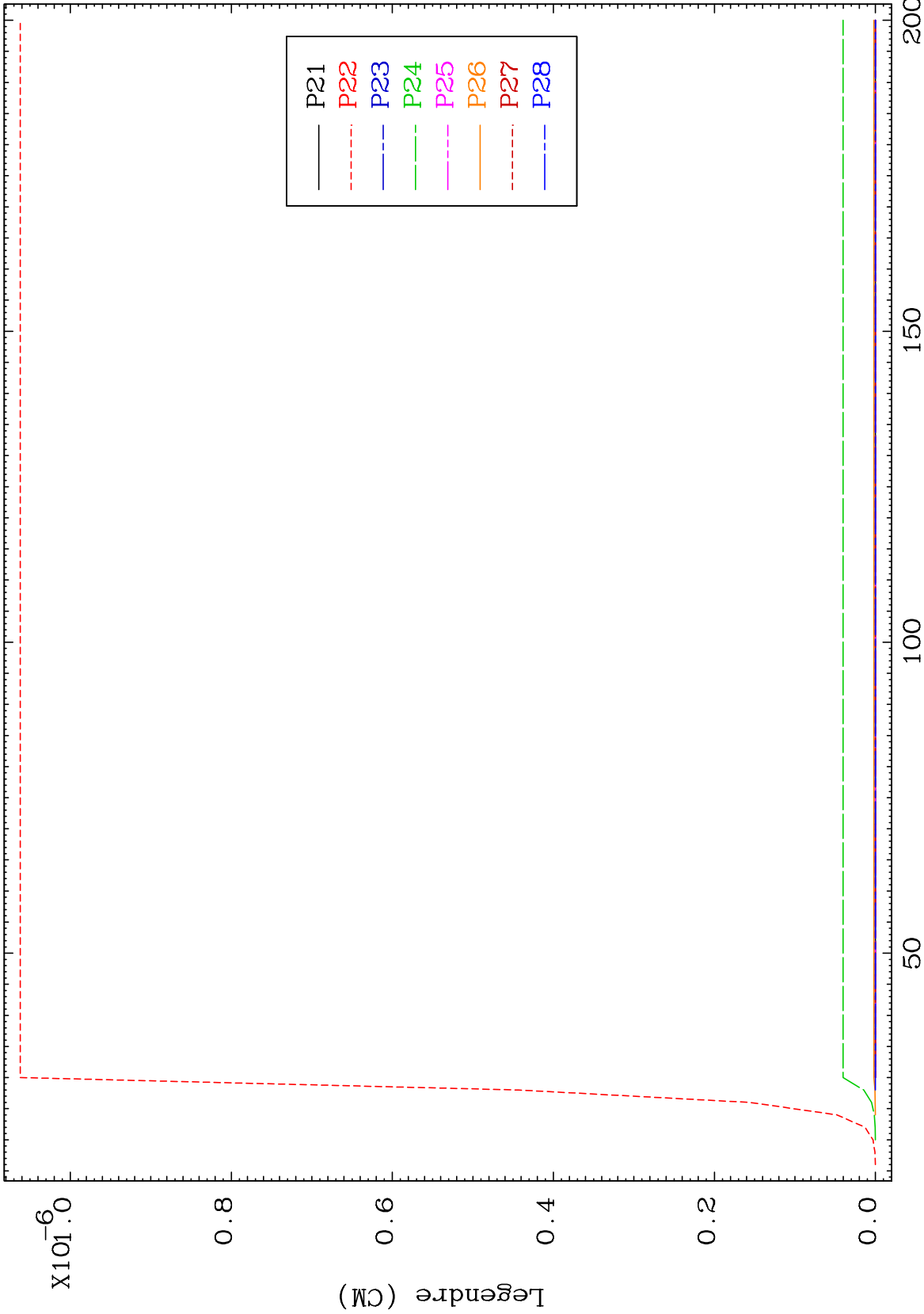
Incident Energy (MeV)

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MT= 51 (n,n') Level
Legendre Coefficients

77-Ir-202



77-Ir-202

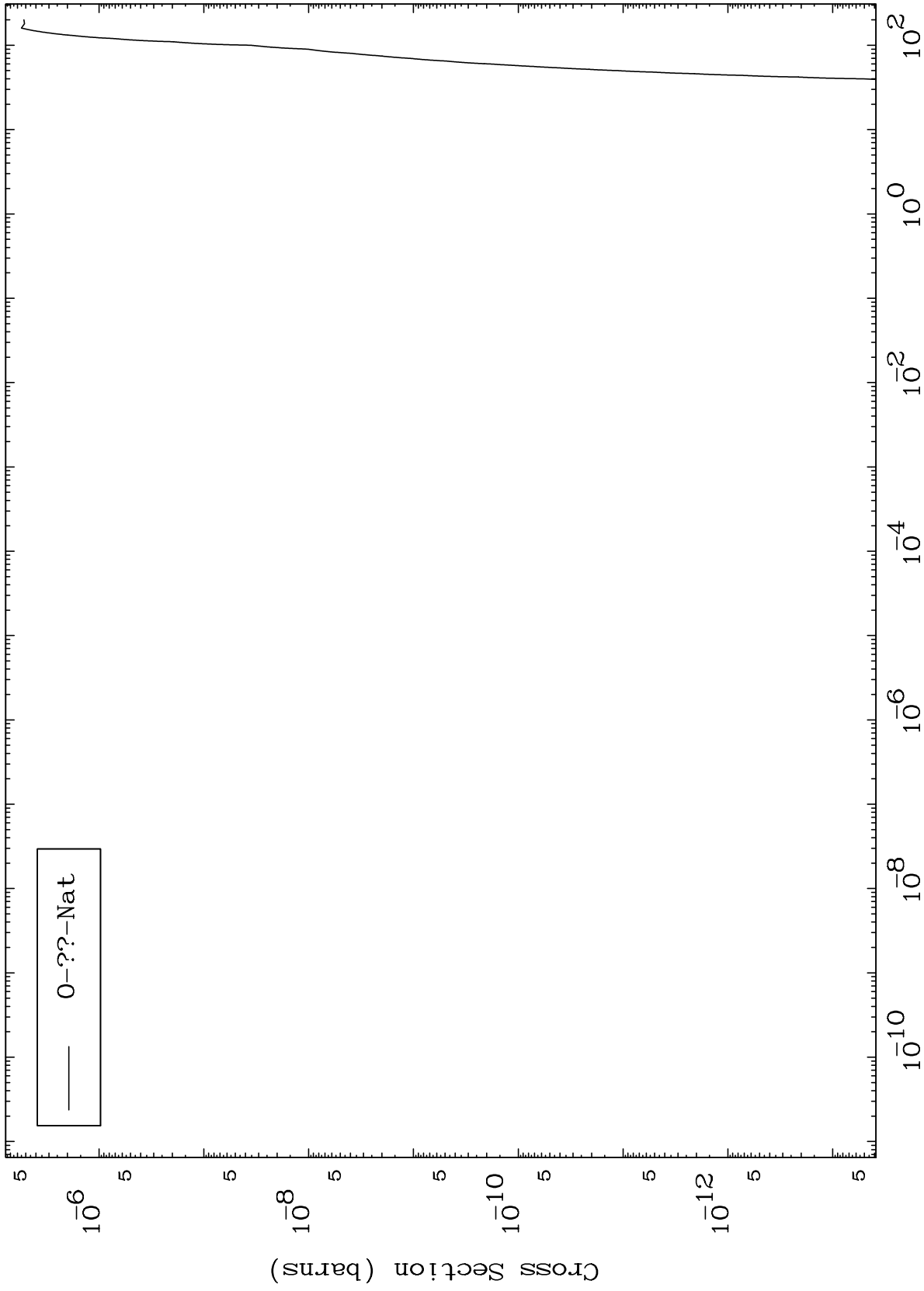
Incident Energy (MeV)

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Fission
Radionuclide Production Cross Section

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