

Program EVALPLOT
(Version 2018-1)

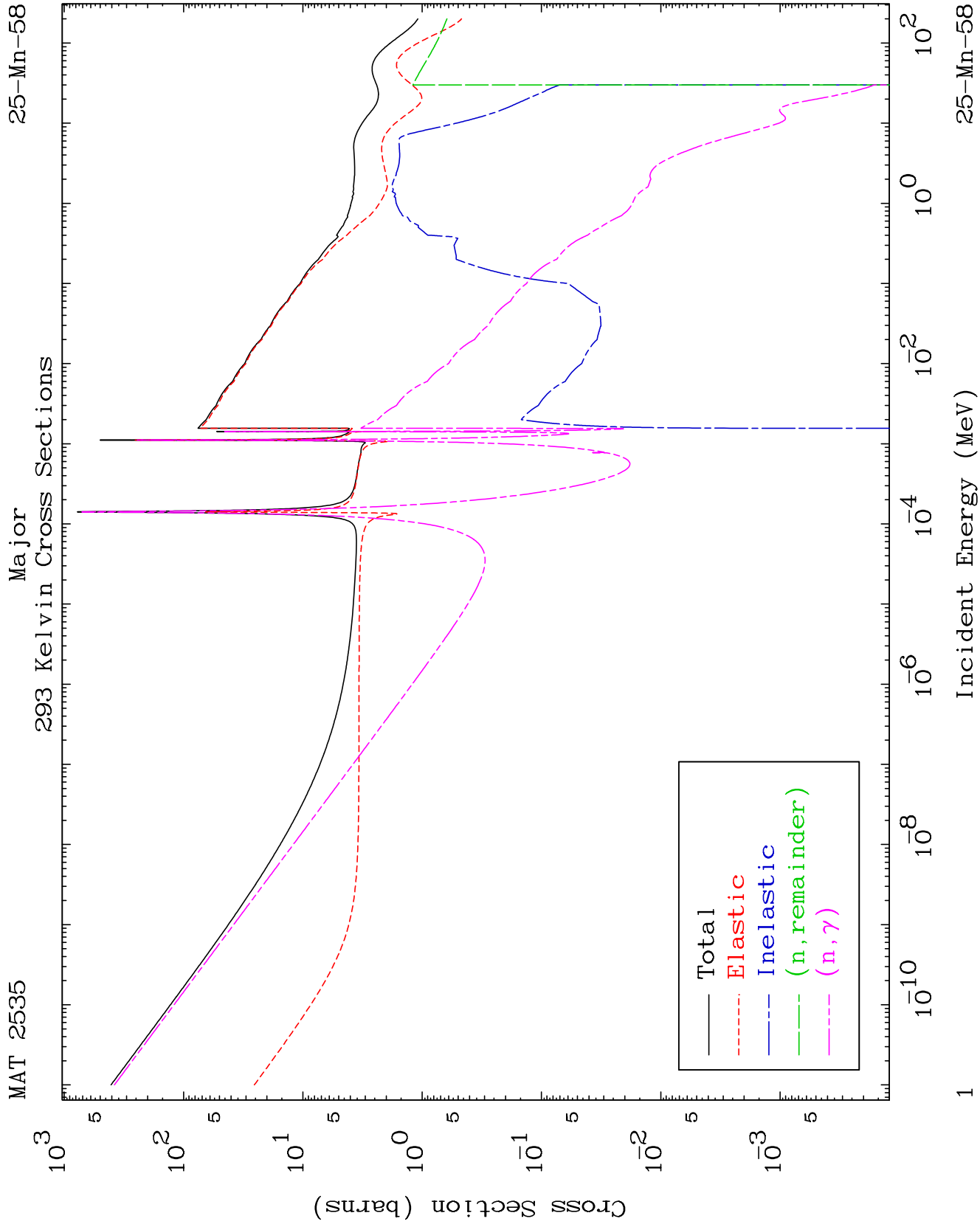
by

Dermott E. Cullen
(Present Contact Information)
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

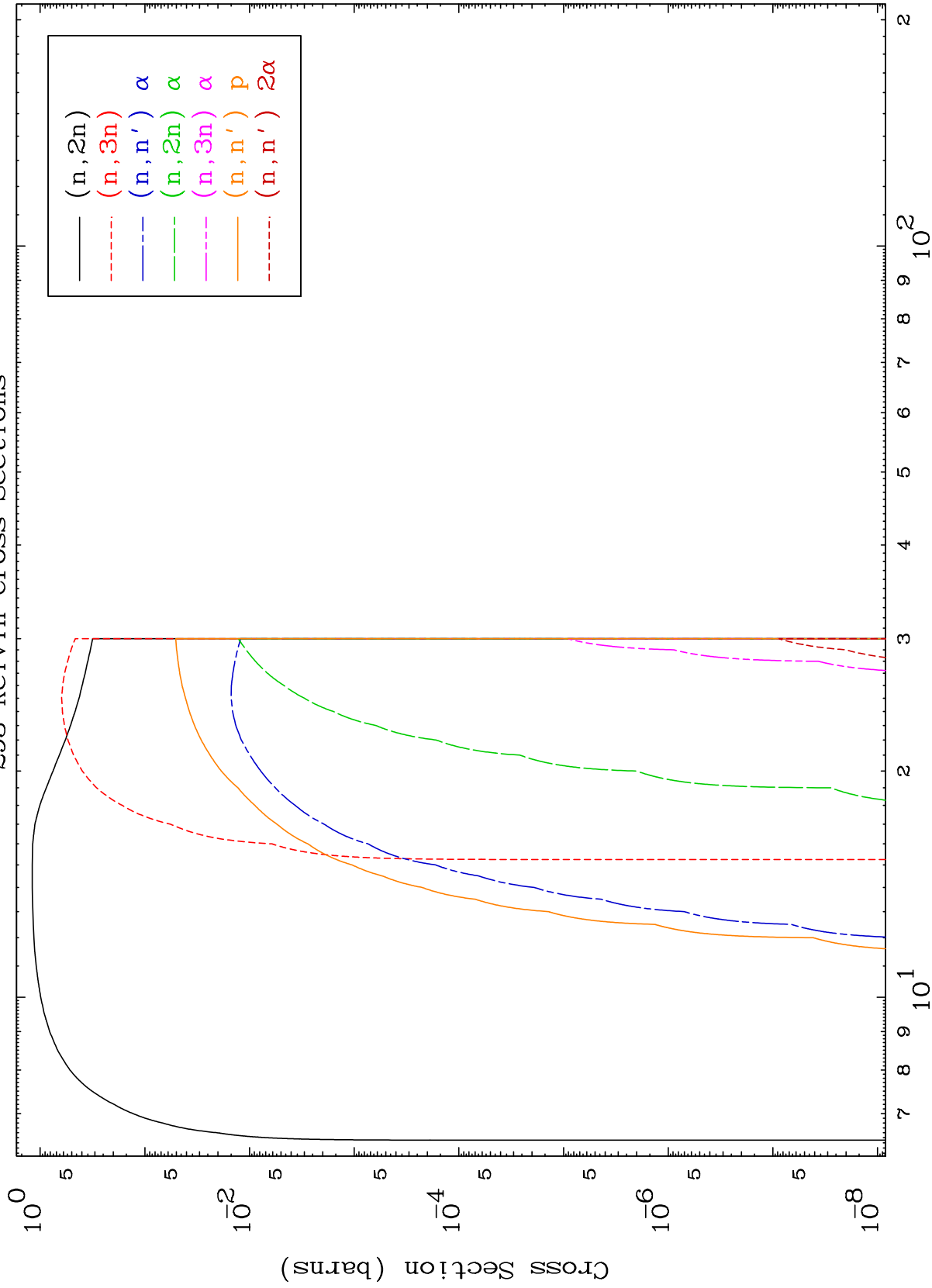
Press Mouse Button to Start



MAT 2535

Neutron Production
293 Kelvin Cross Sections

25-Mn-58



2

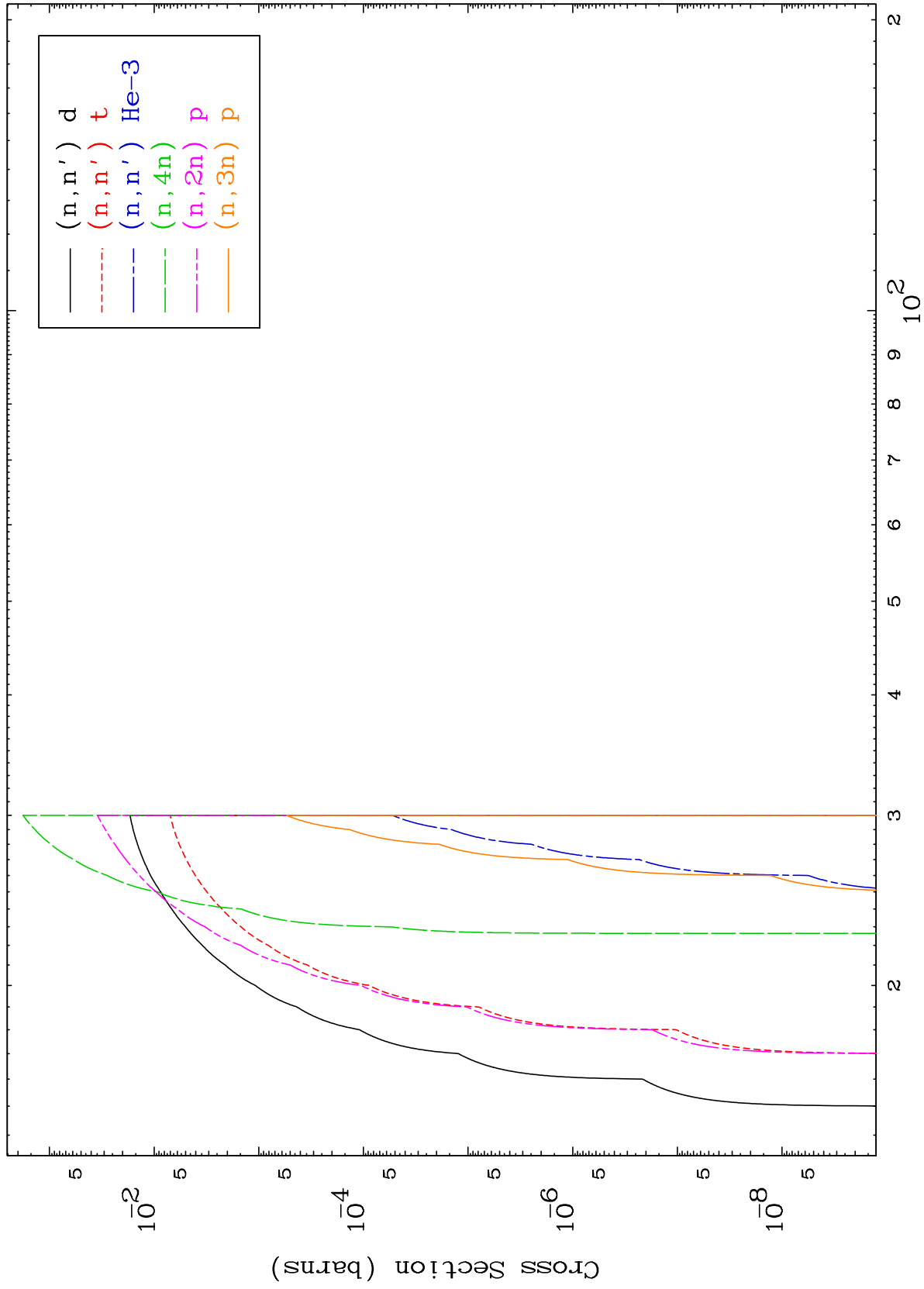
Incident Energy (MeV)

25-Mn-58

MAT 2535

Neutron Production
293 Kelvin Cross Sections

25-Mn-58



3

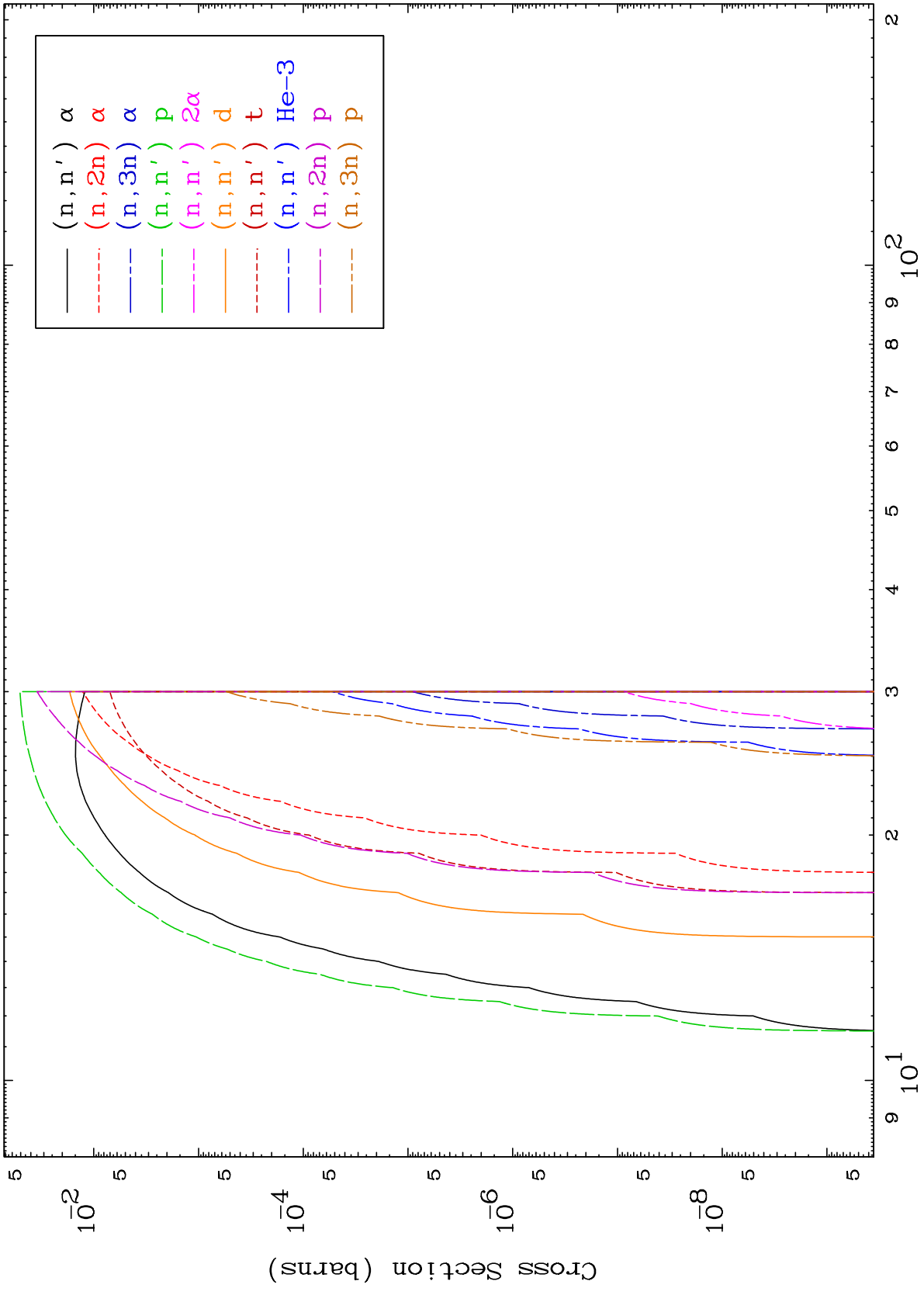
Incident Energy (MeV)

25-Mn-58

MAT 2535

Charged Particle
293 Kelvin Cross Sections

25-Mn-58



4

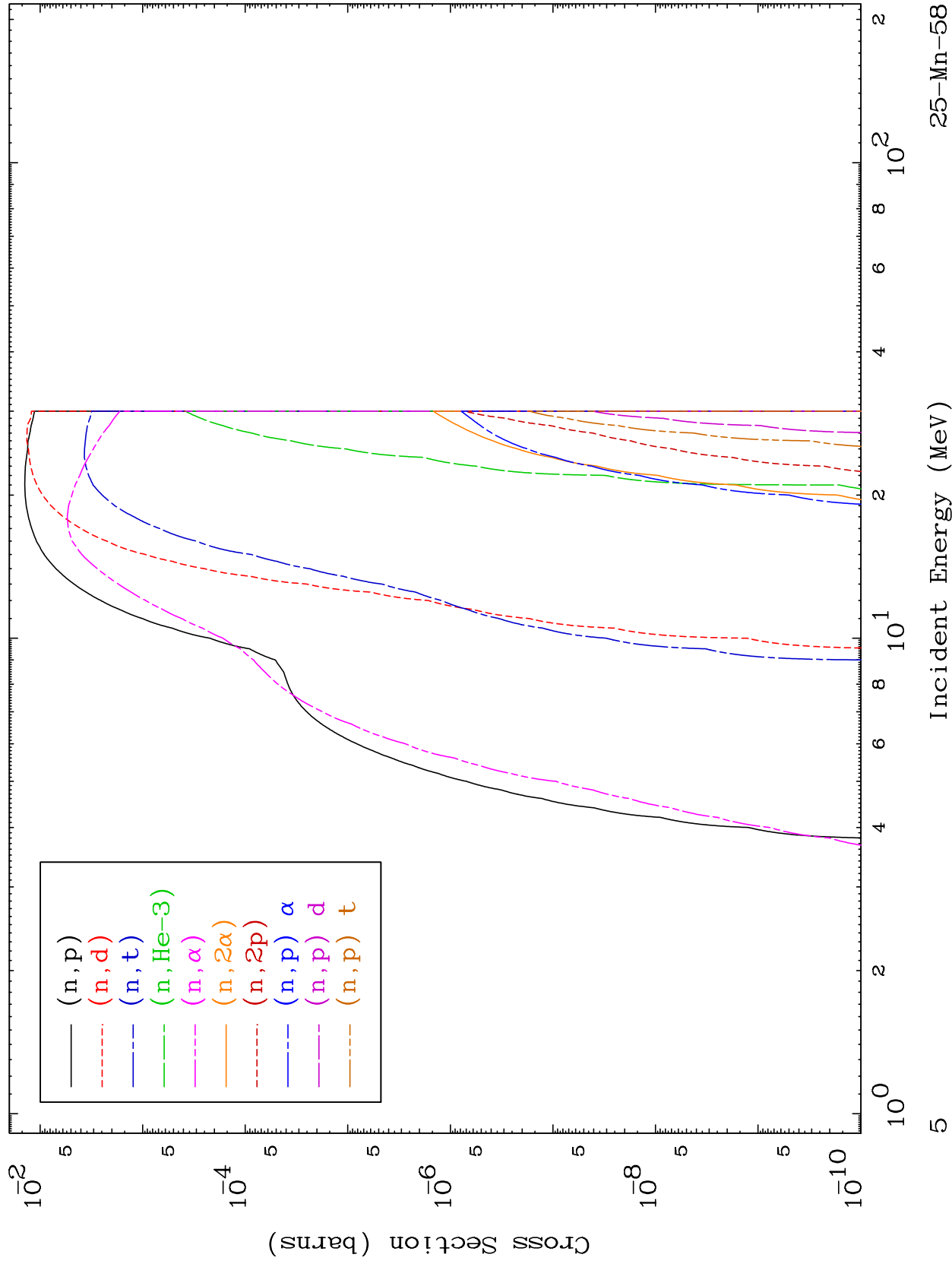
Incident Energy (MeV)

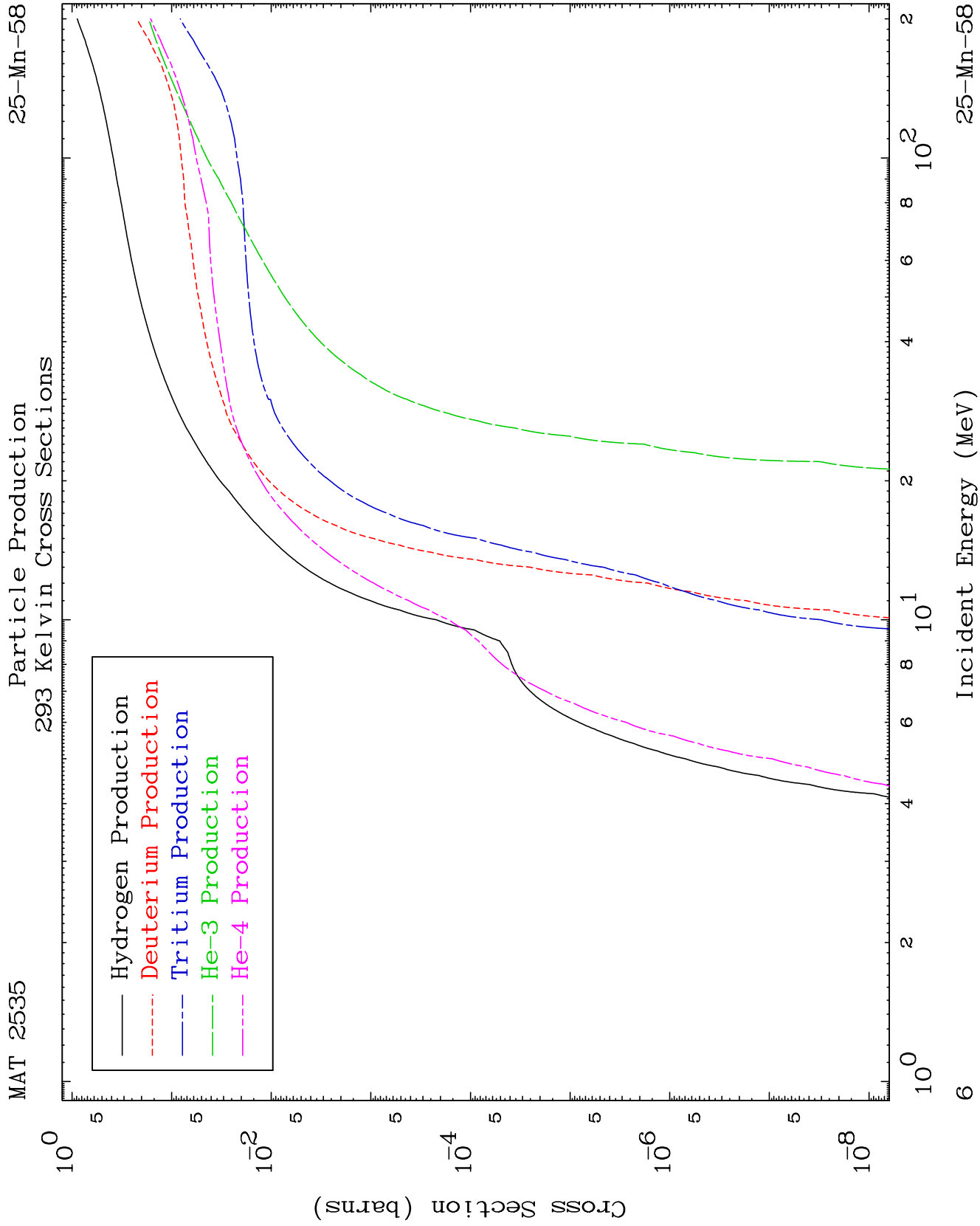
25-Mn-58

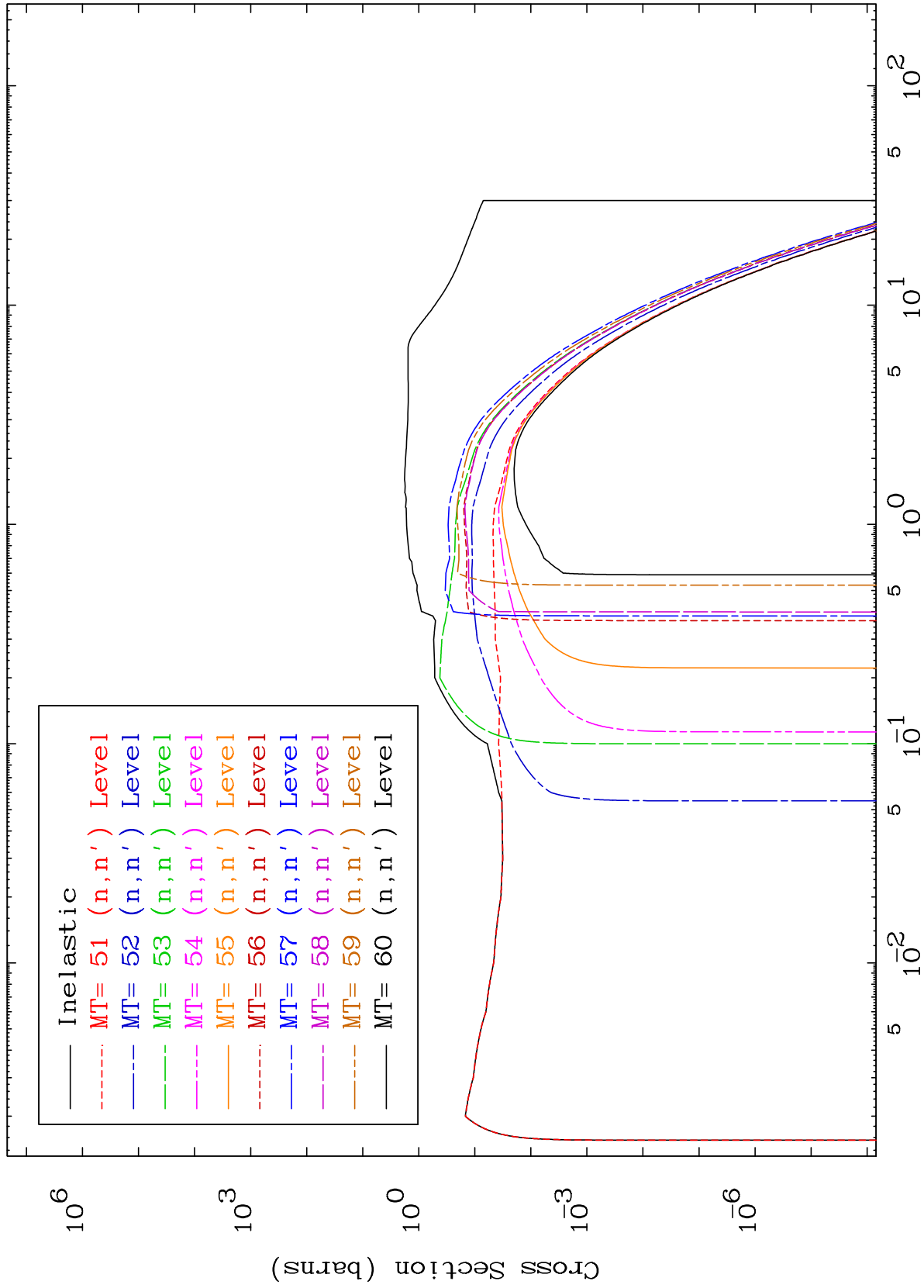
MAT 2535

Charged Particle
293 Kelvin Cross Sections

25-Mn-58





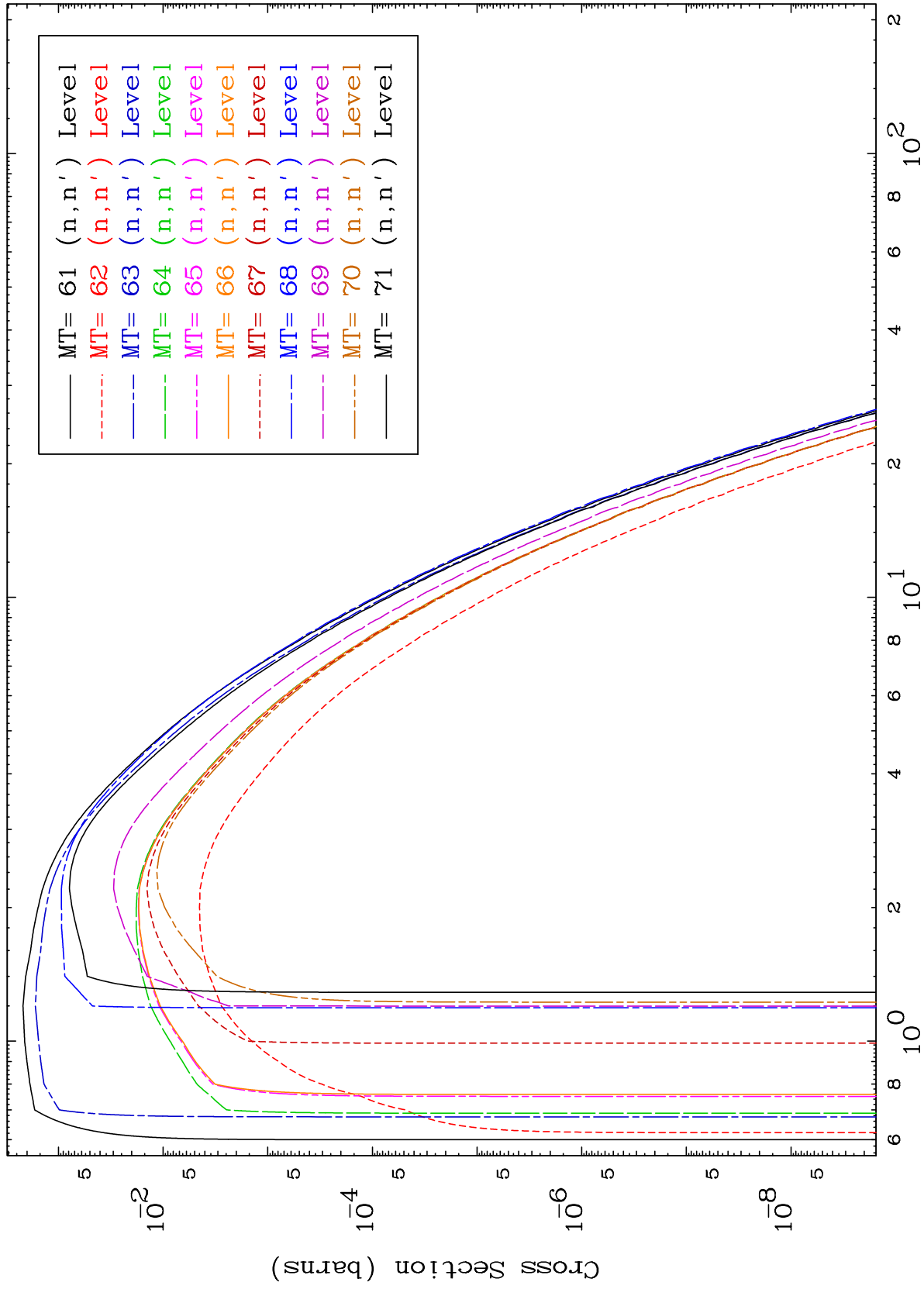


MAT 2535

(n,n') Level

25-Mn-58

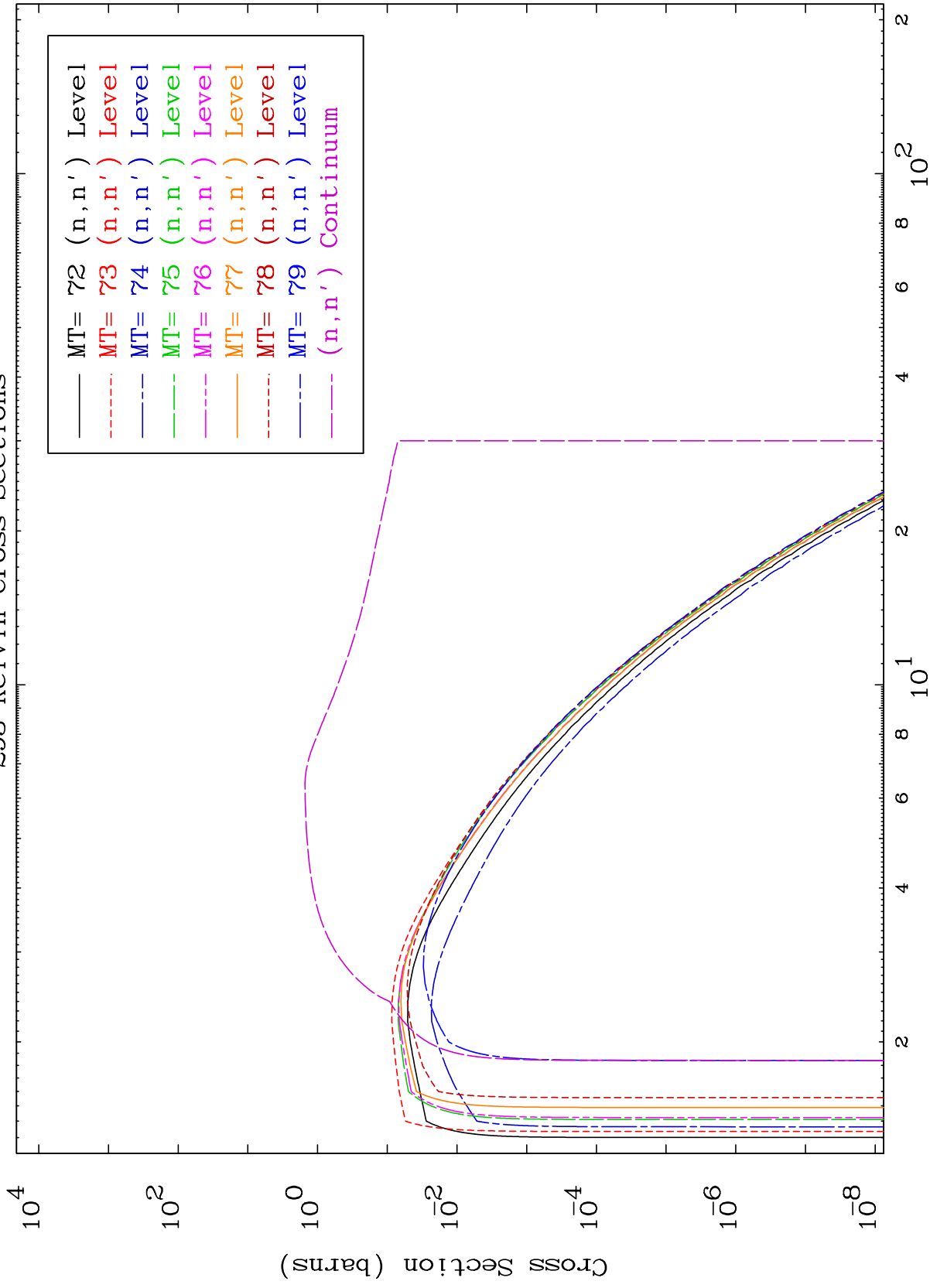
293 Kelvin Cross Sections



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Incident Energy (MeV)

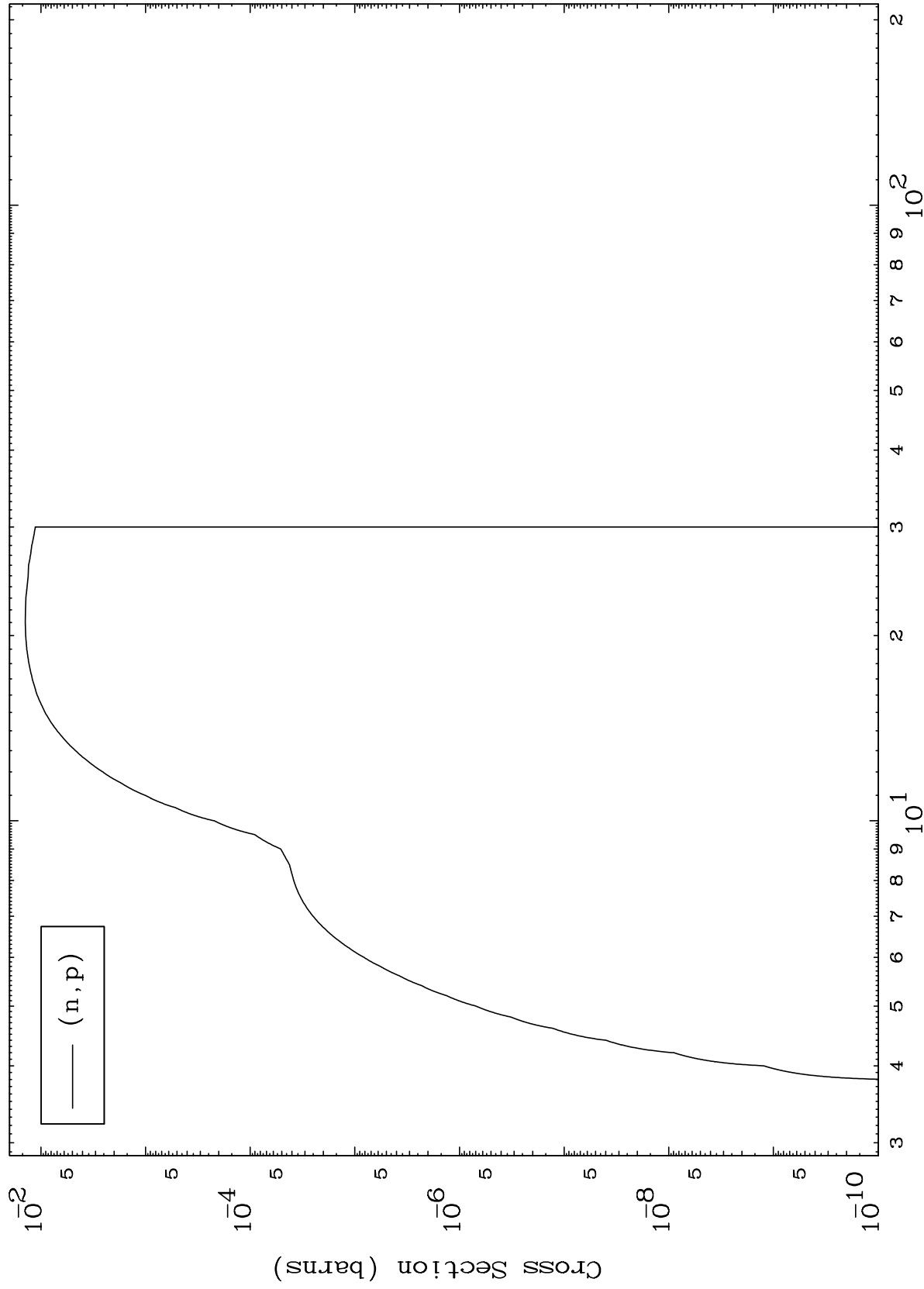
25-Mn-58



MAT 2535

(n,p) Levels
293 Kelvin Cross Sections

25-Mn-58



10

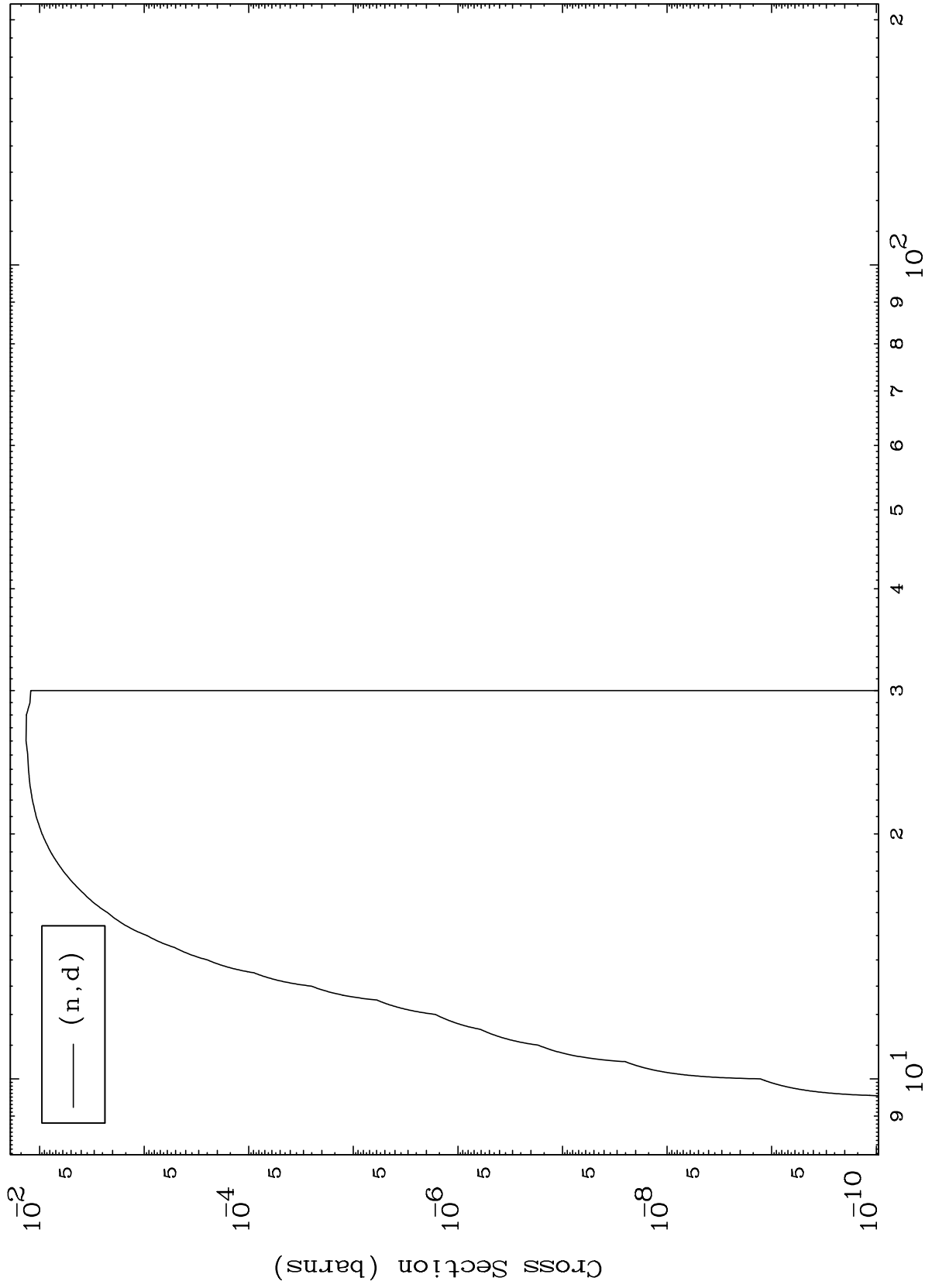
Incident Energy (MeV)

25-Mn-58

MAT 2535

25-Mn-58

(n,d) Levels
293 Kelvin Cross Sections



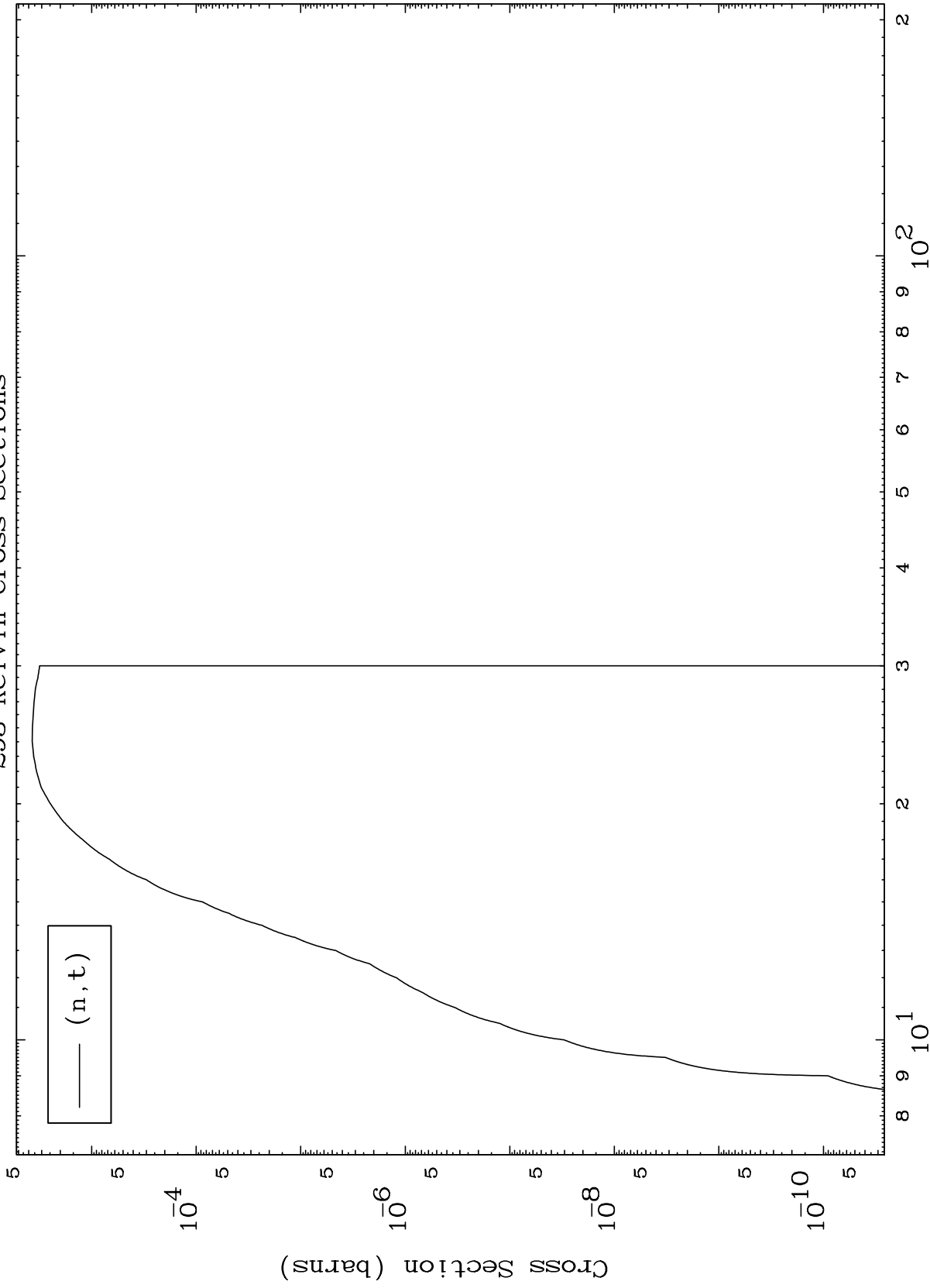
11

25-Mn-58

MAT 2535

25-Mn-58

(n,t) Levels
293 Kelvin Cross Sections



12

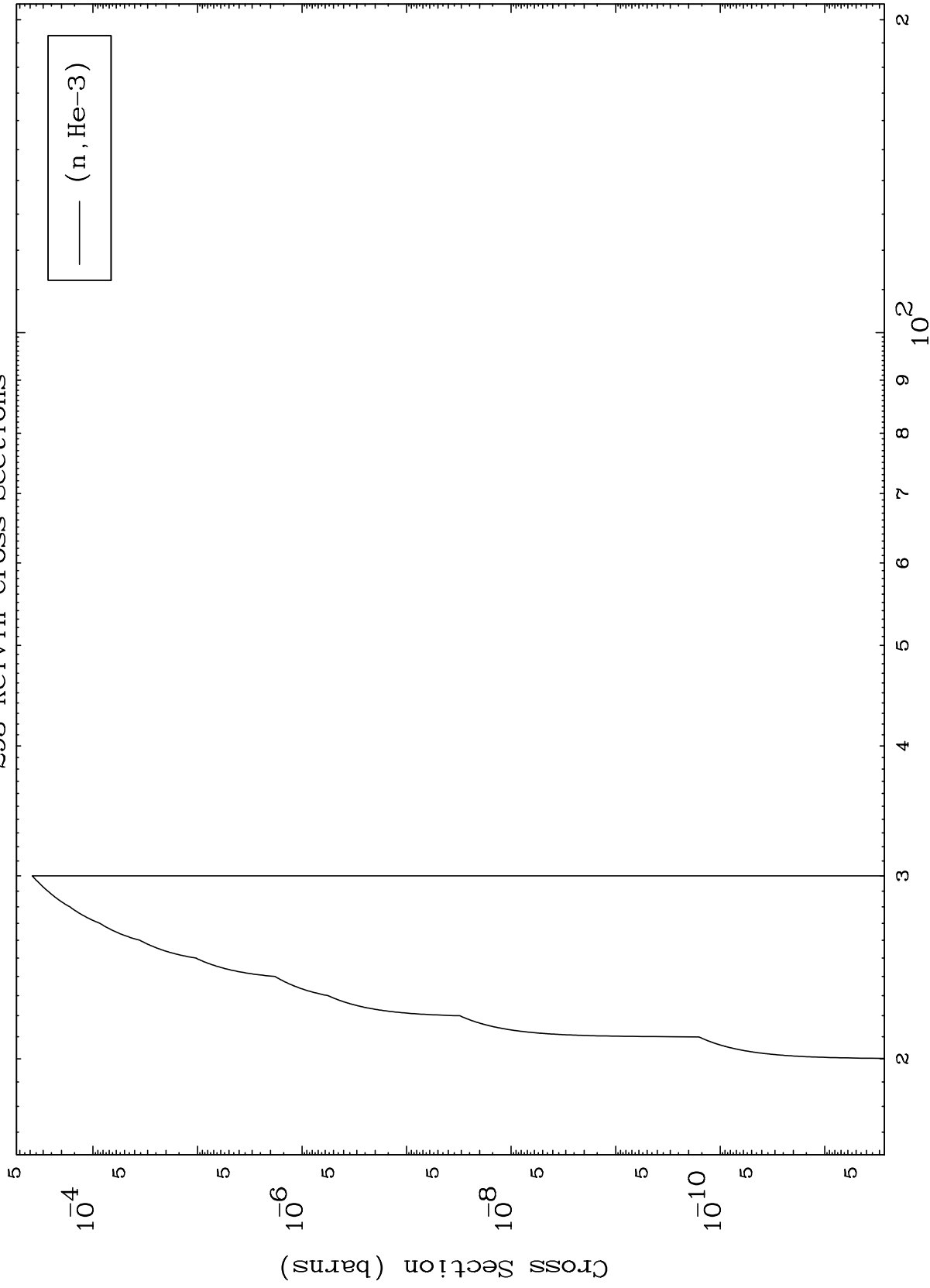
Incident Energy (MeV)

25-Mn-58

MAT 2535

(n,He3) Levels
293 Kelvin Cross Sections

25-Mn-58



13

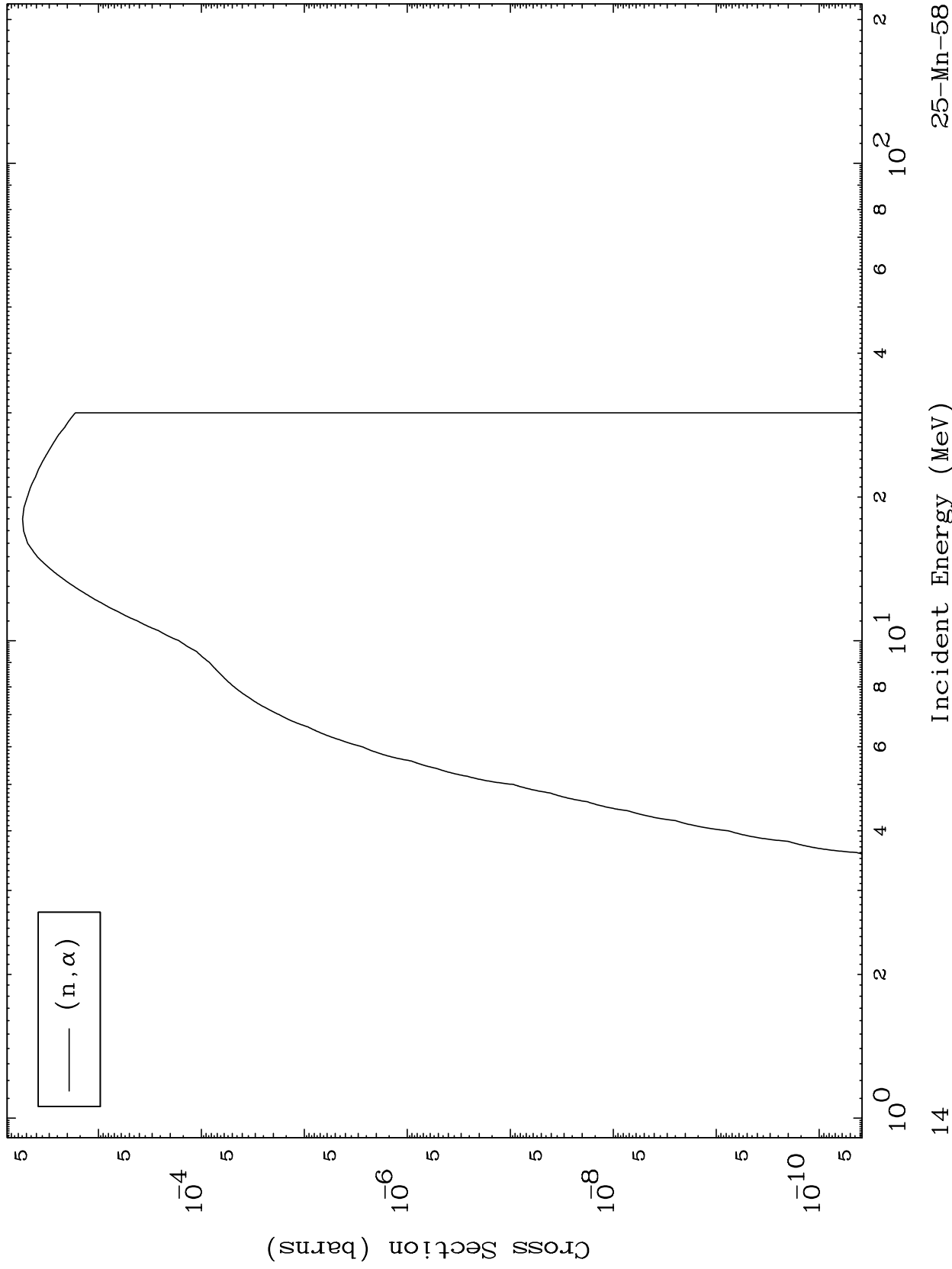
Incident Energy (MeV)

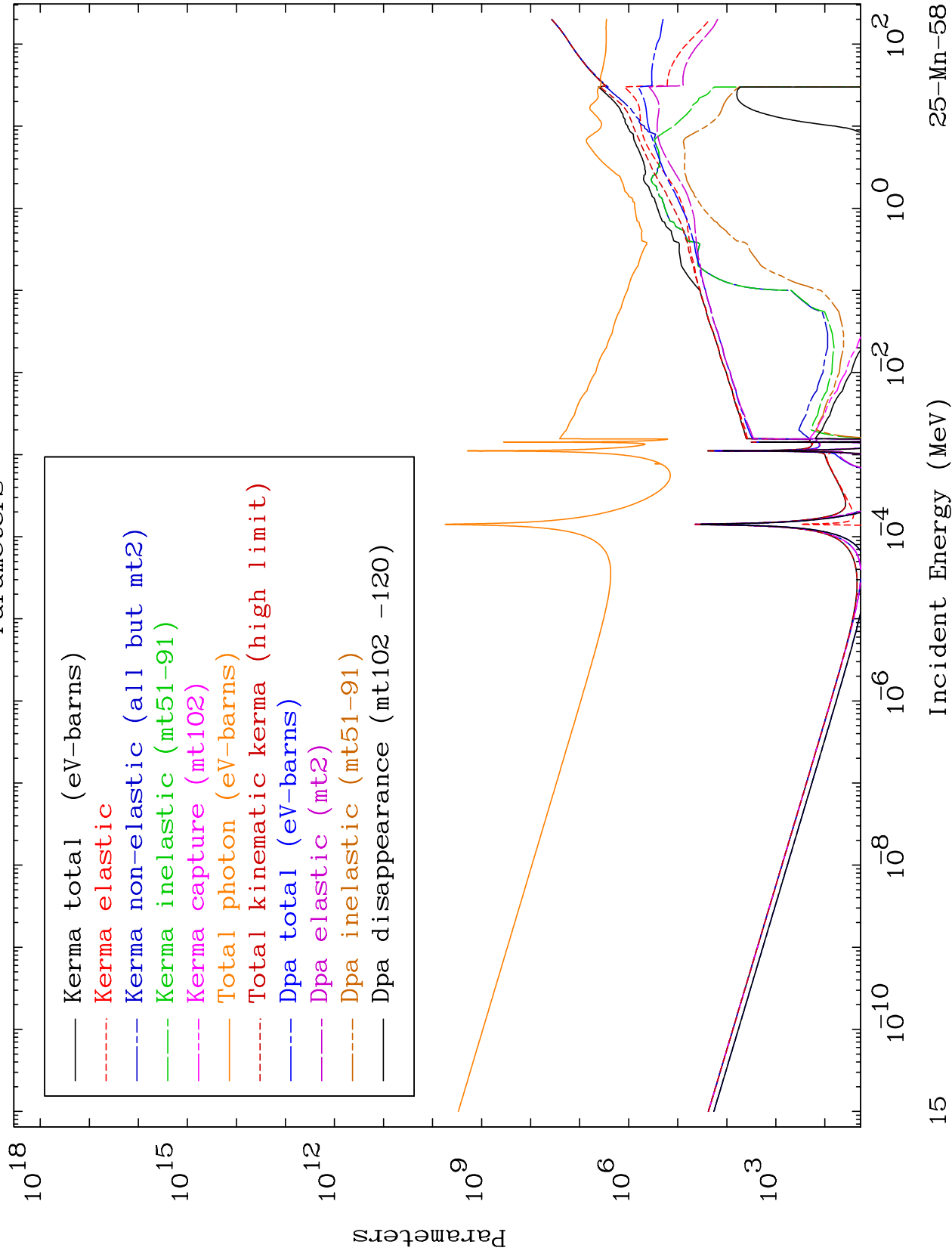
25-Mn-58

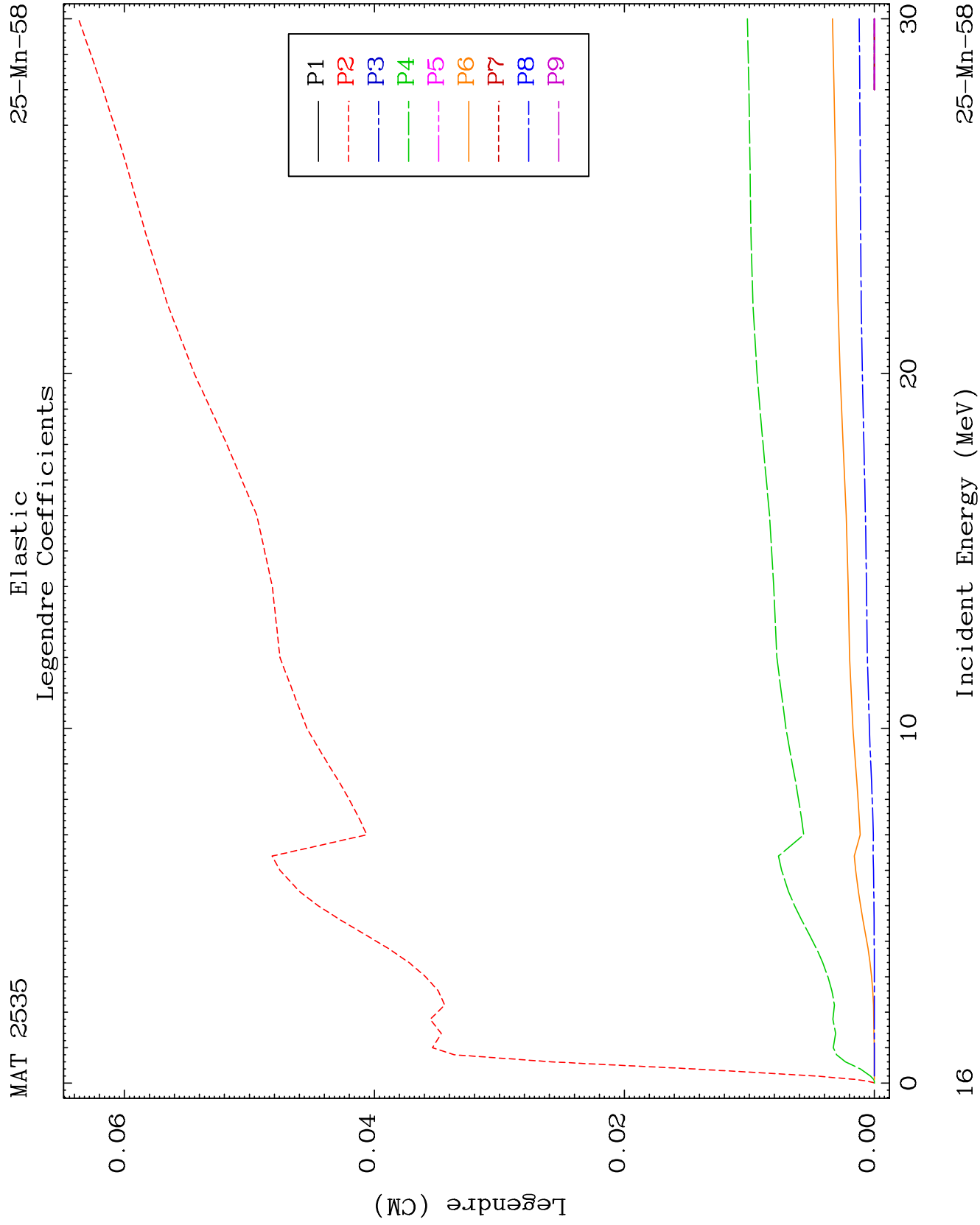
MAT 2535

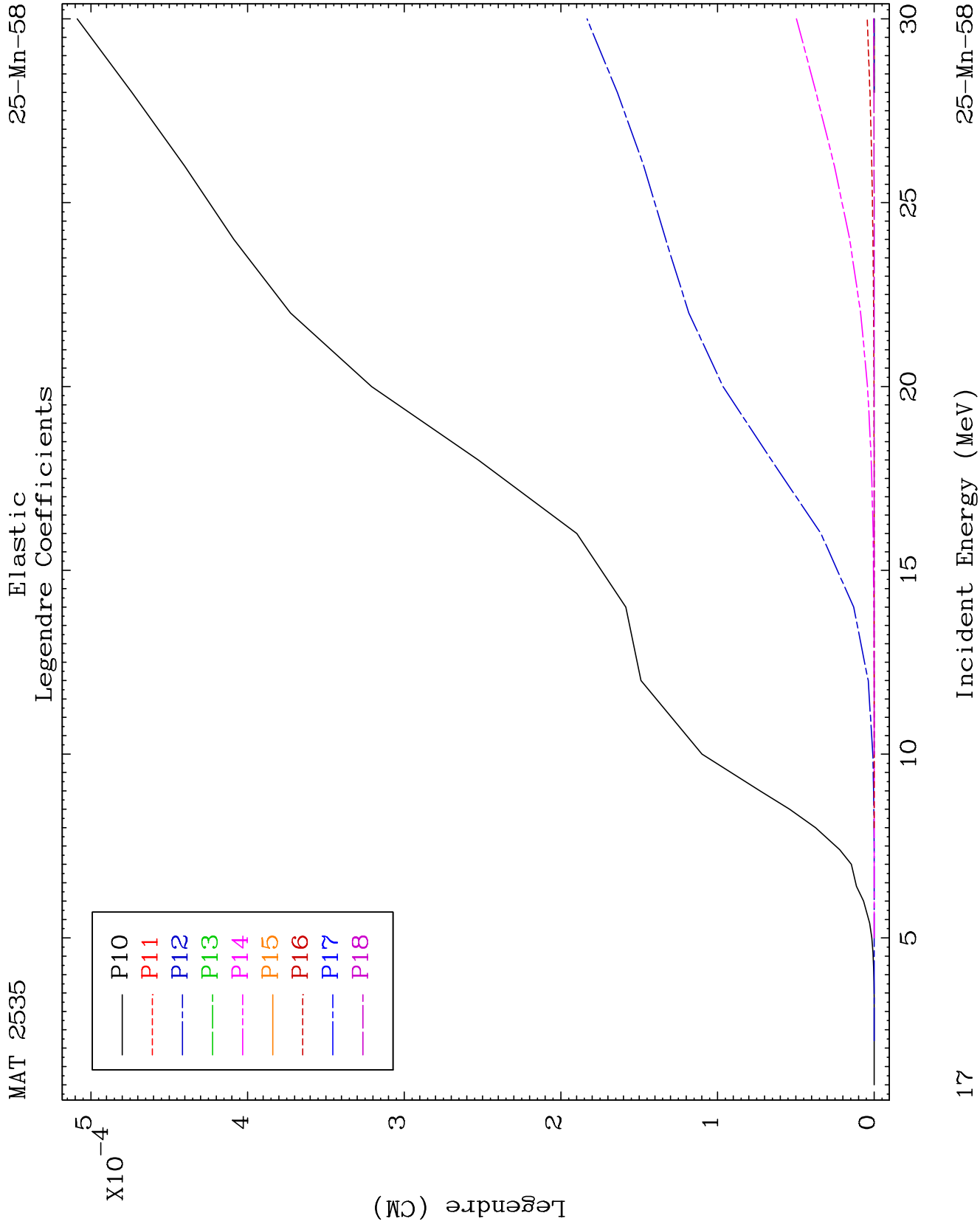
(n, α) Levels
293 Kelvin Cross Sections

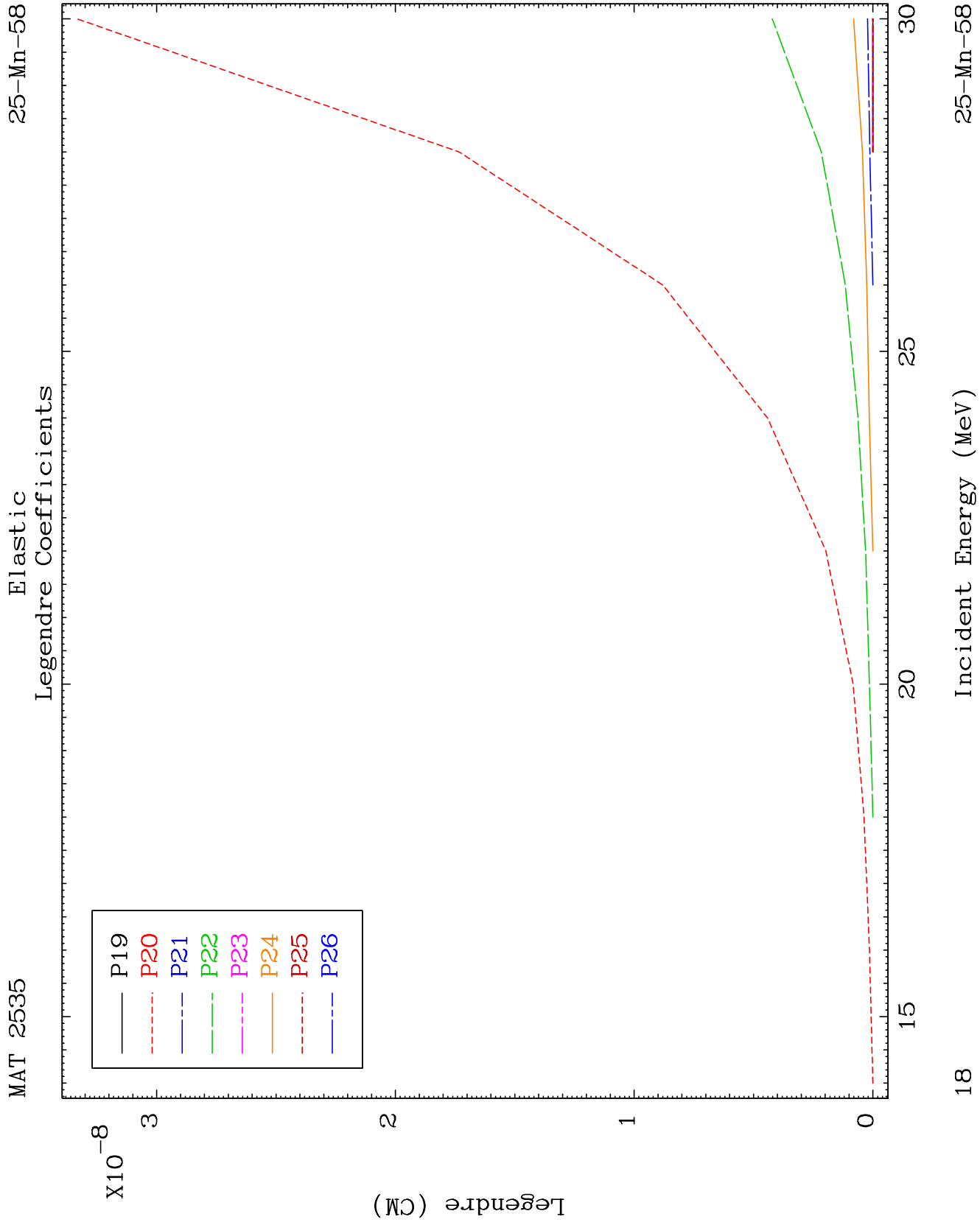
25-Mn-58





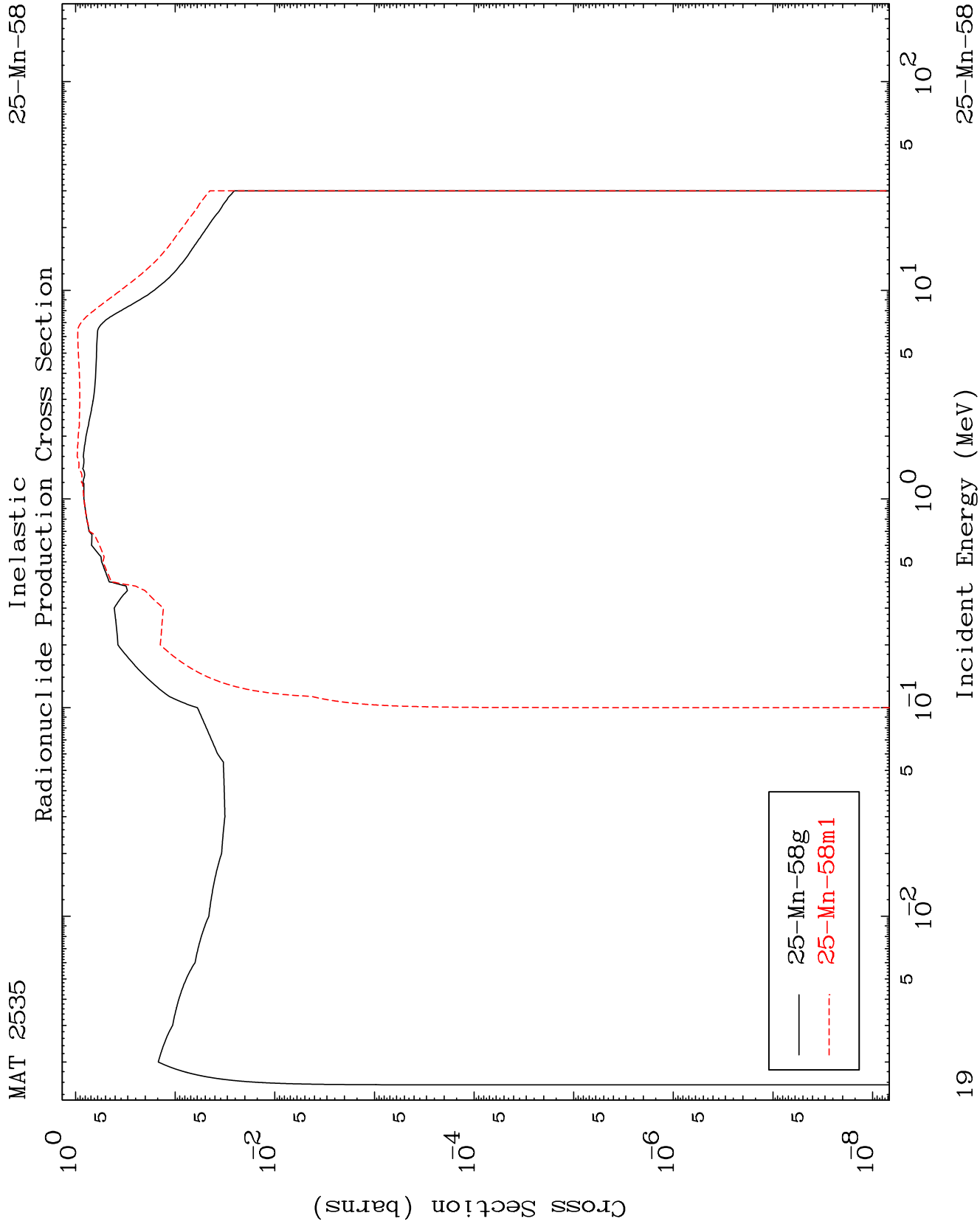






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25-Mn-58

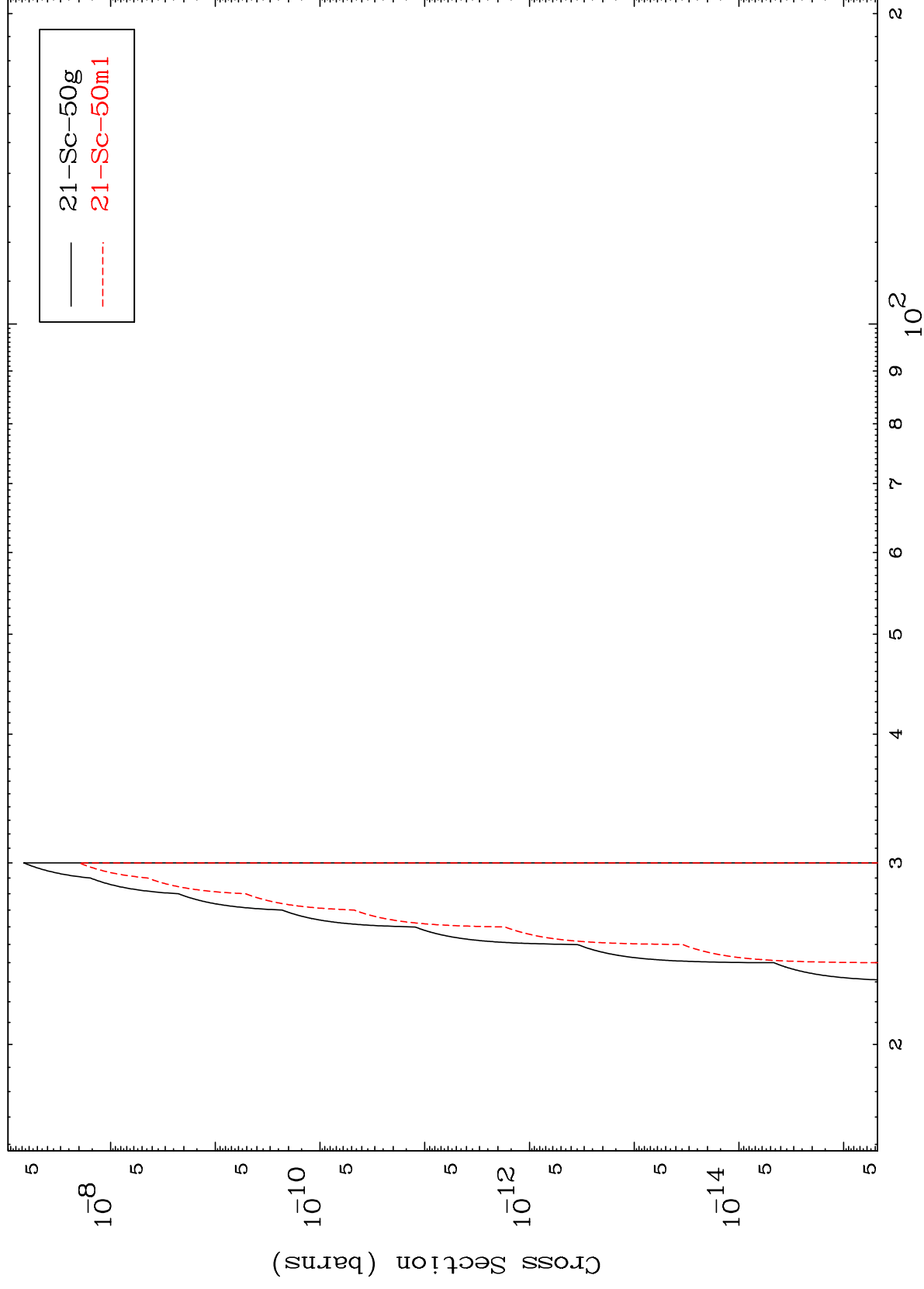


MAT 2535

 $(n, n') \, 2\alpha$

25-Mn-58

Radionuclide Production Cross Section



20

Incident Energy (MeV)

25-Mn-58