

Program EVALPLOT
(Version 2018-1)

by

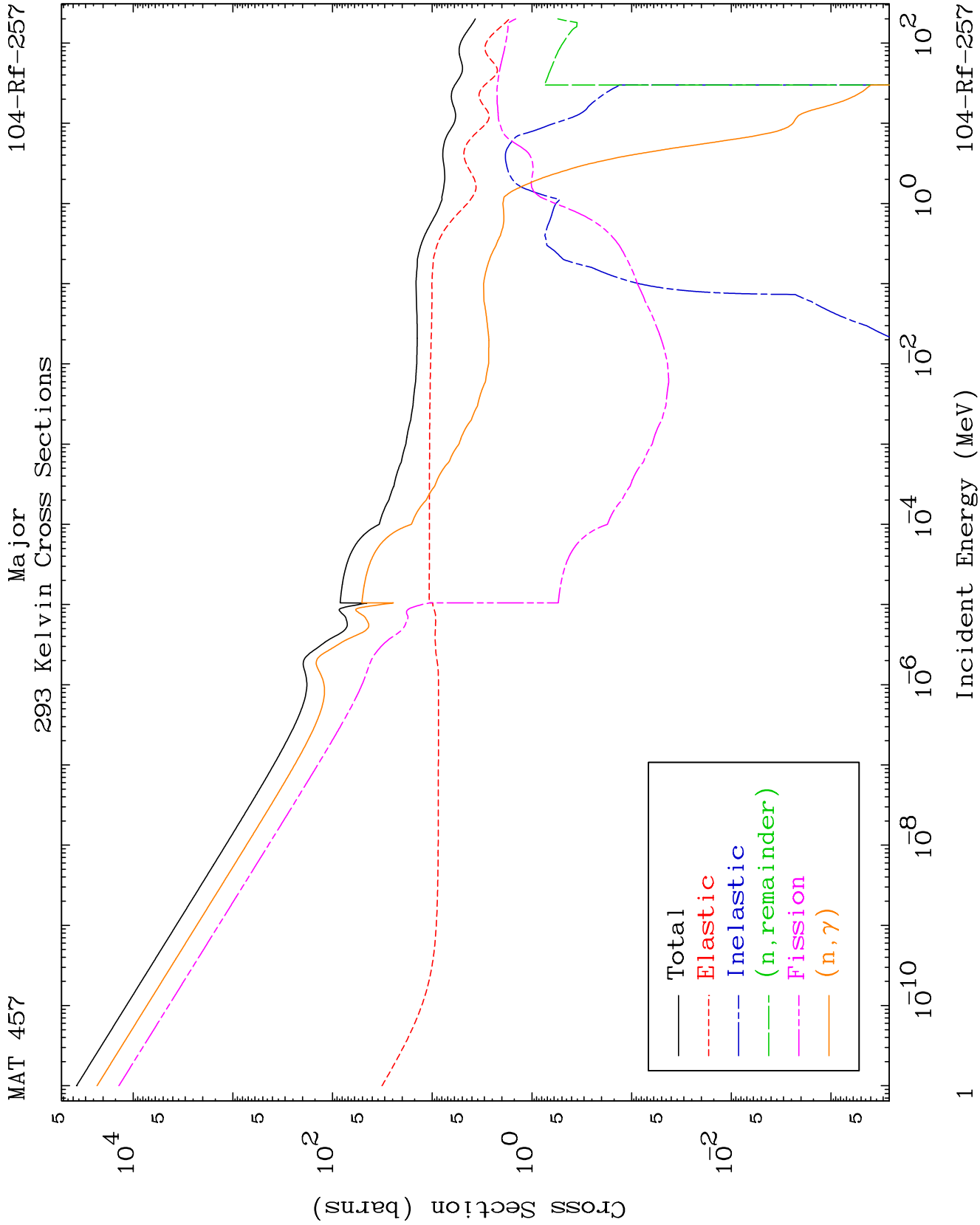
Dermott E. Cullen
(Present Contact Information)

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Livermore, CA 94550
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Tele: 925-443-1911

E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

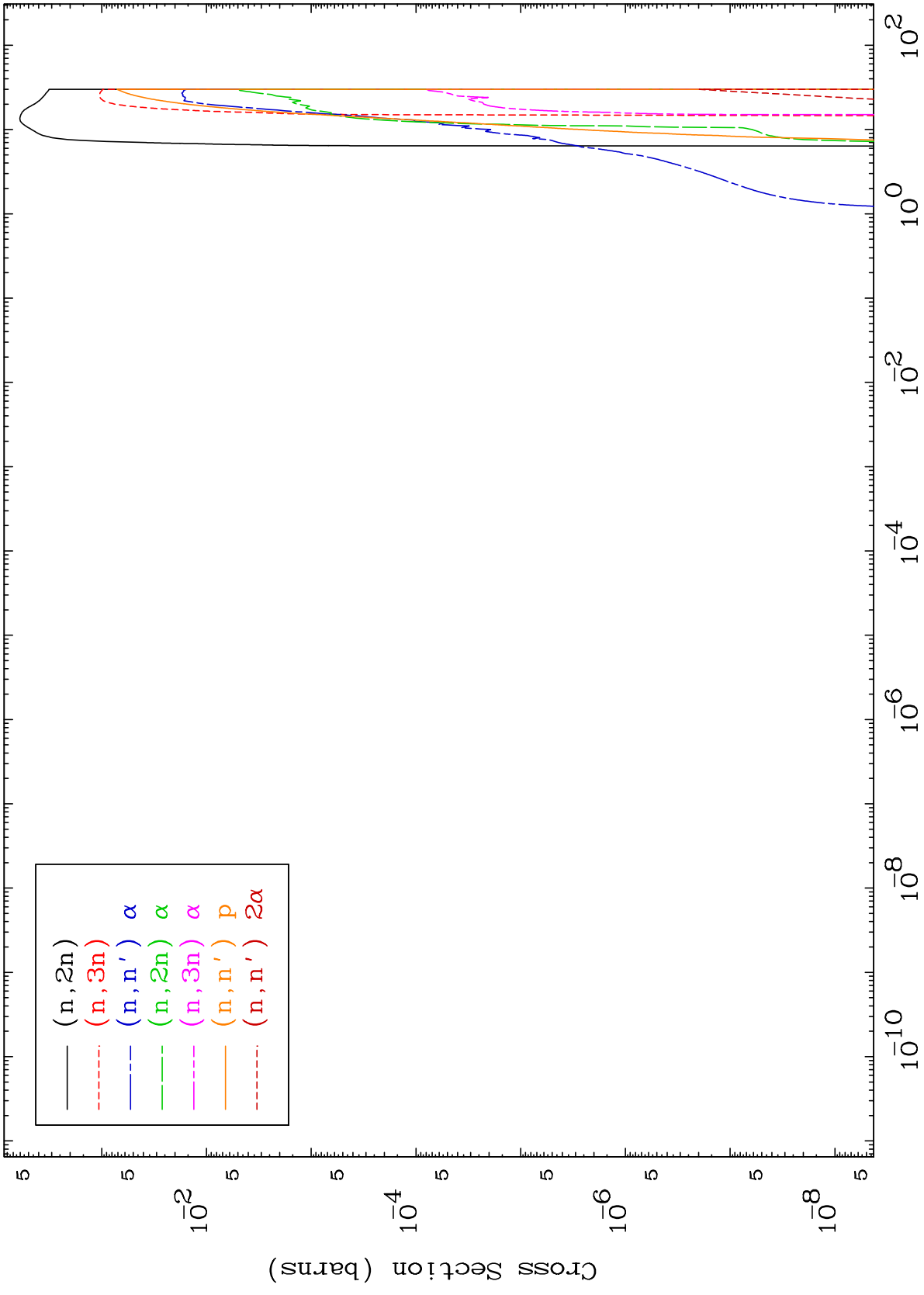
Press Mouse Button to Start



MAT 457

Neutron Production
293 Kelvin Cross Sections

104-Rf-257

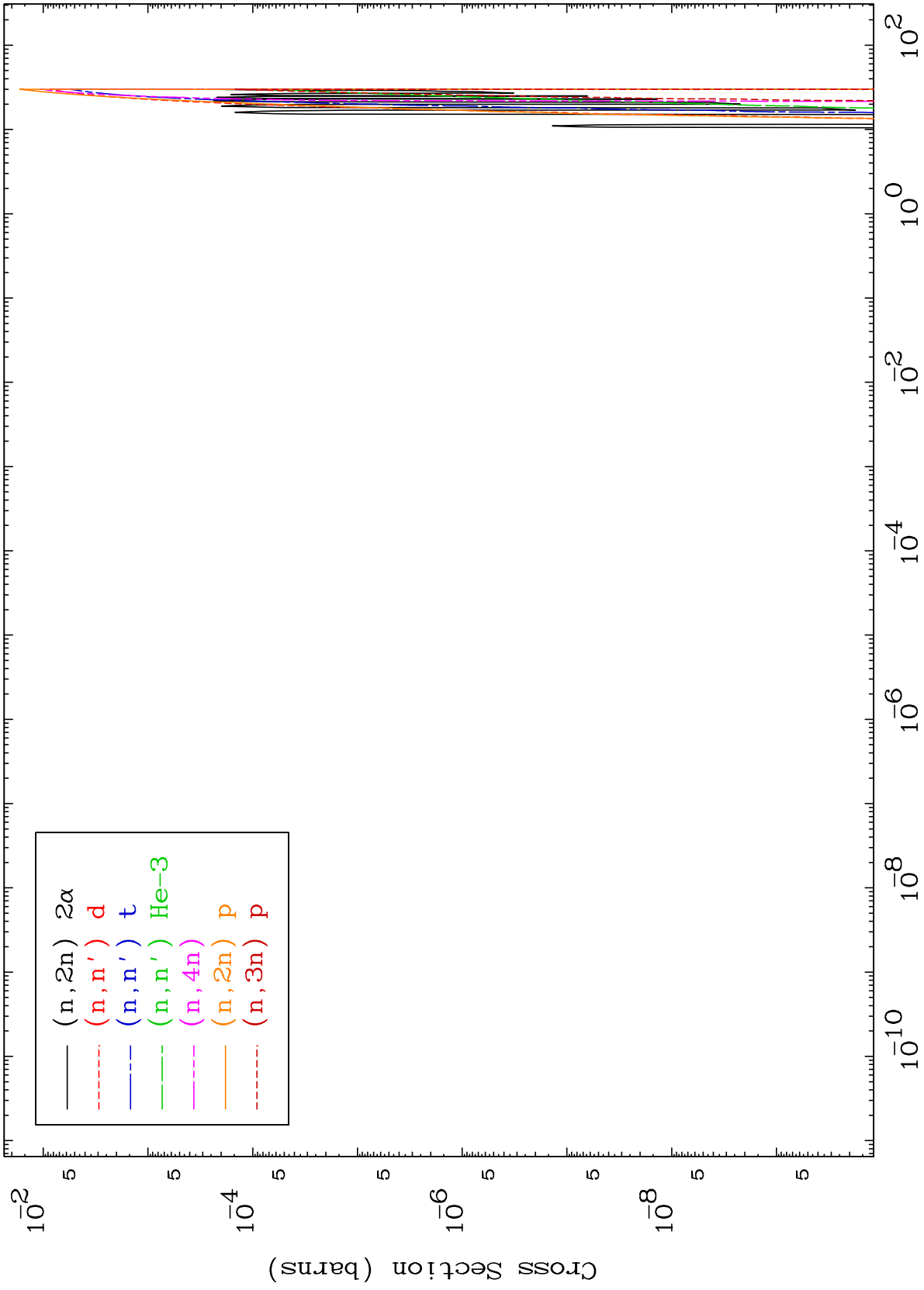


104-Rf-257

MAT 457

Neutron Production
293 Kelvin Cross Sections

104-Rf-257



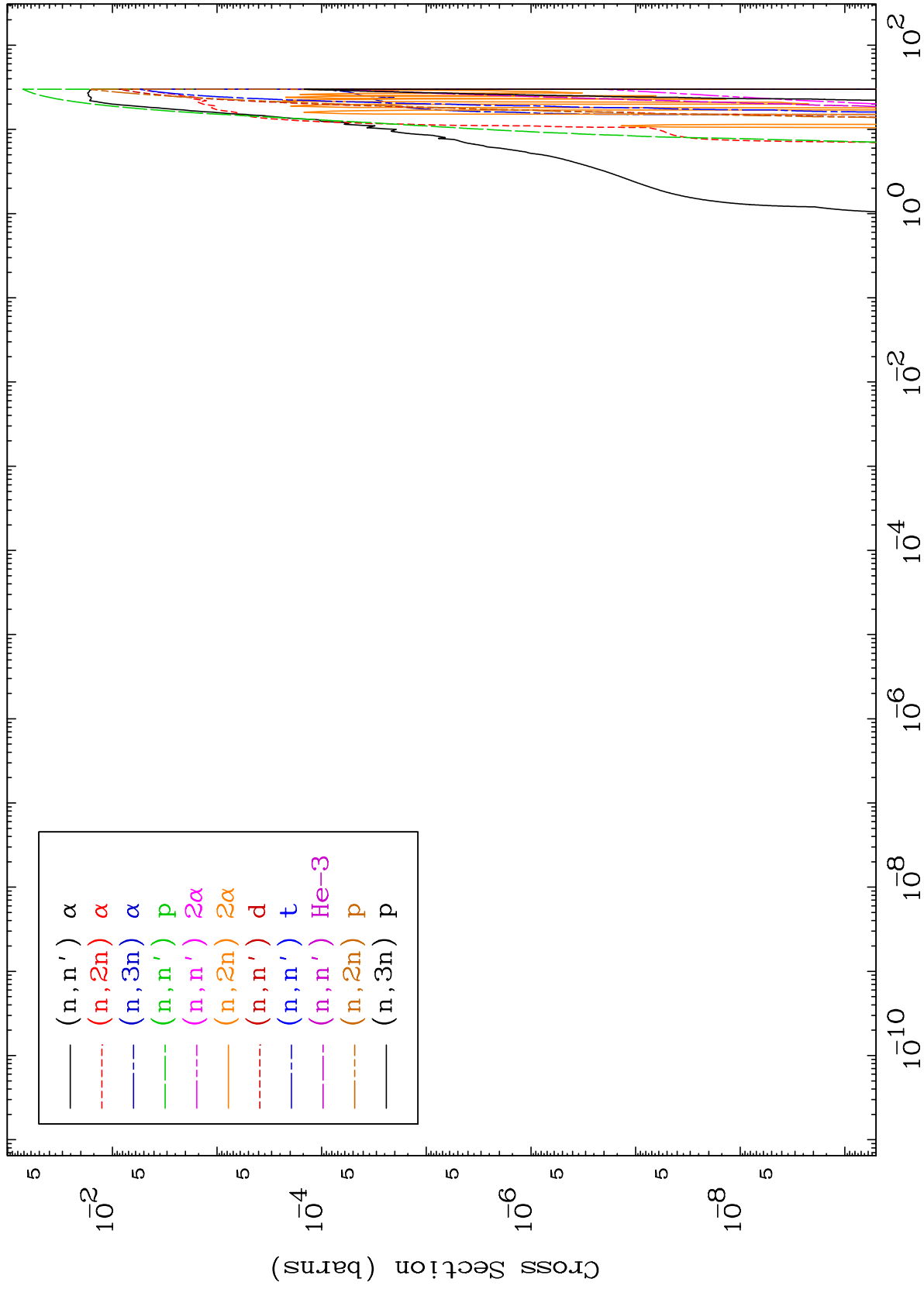
3

104-Rf-257

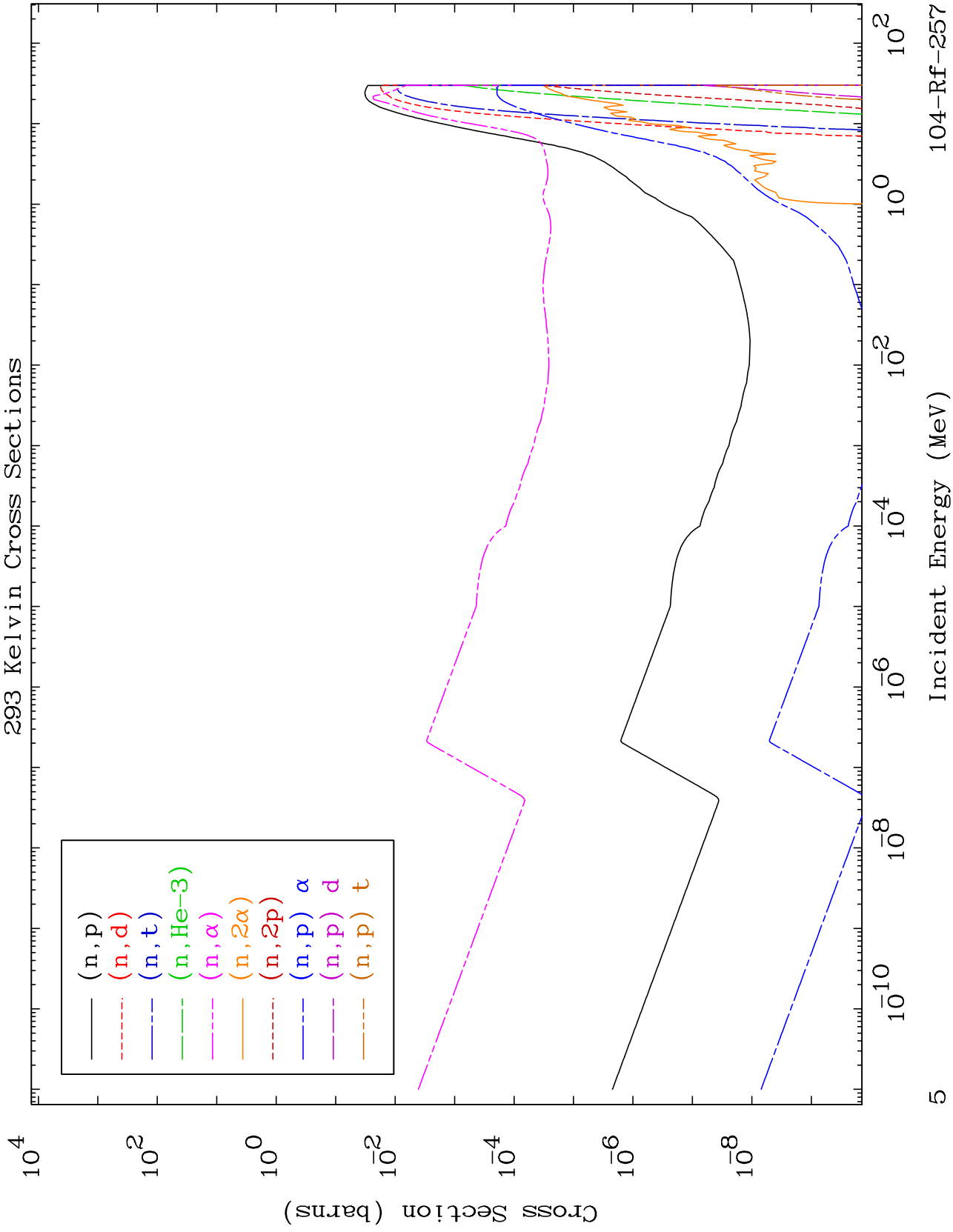
MAT 457

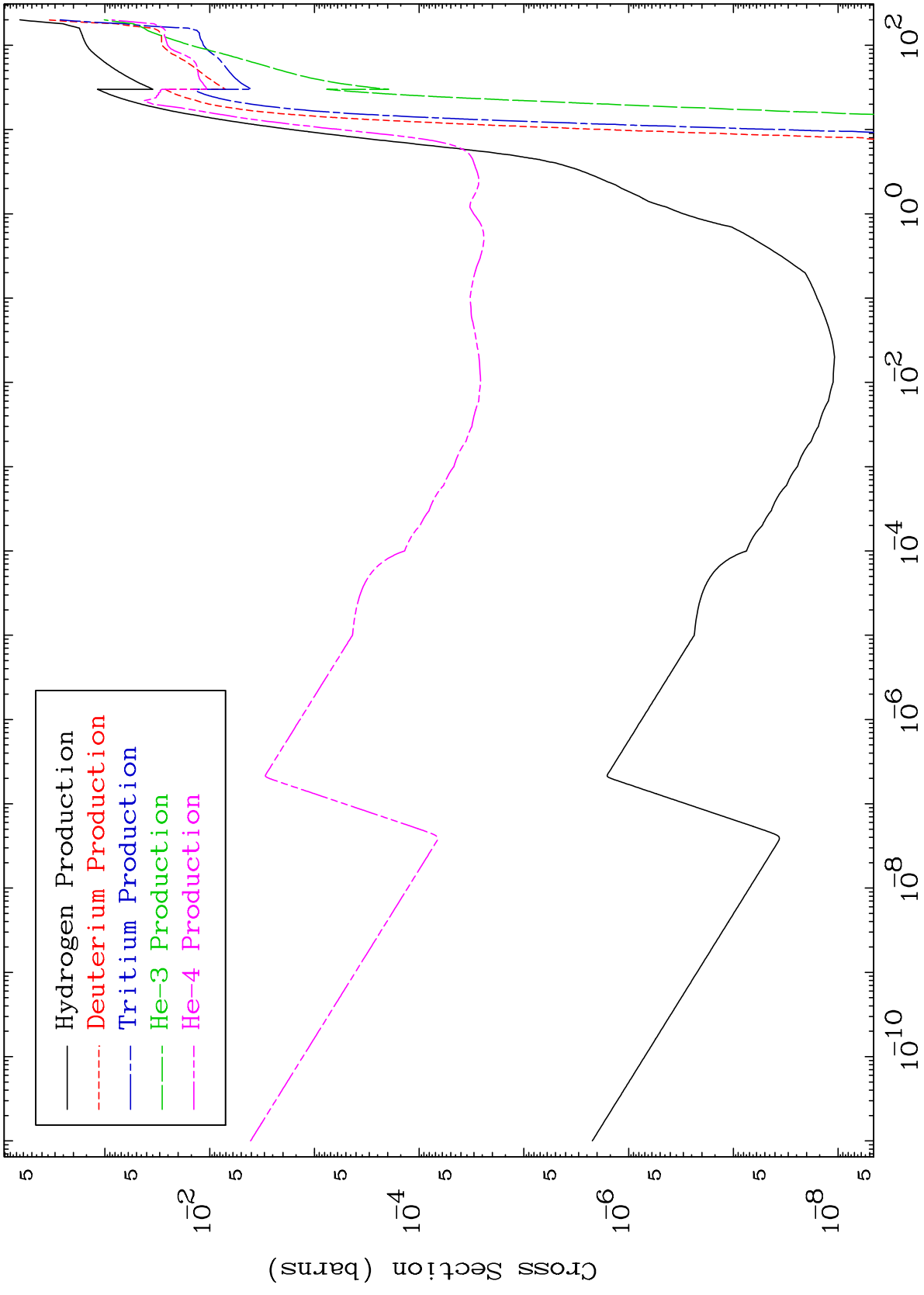
Charged Particle
293 Kelvin Cross Sections

104-Rf-257



104-Rf-257

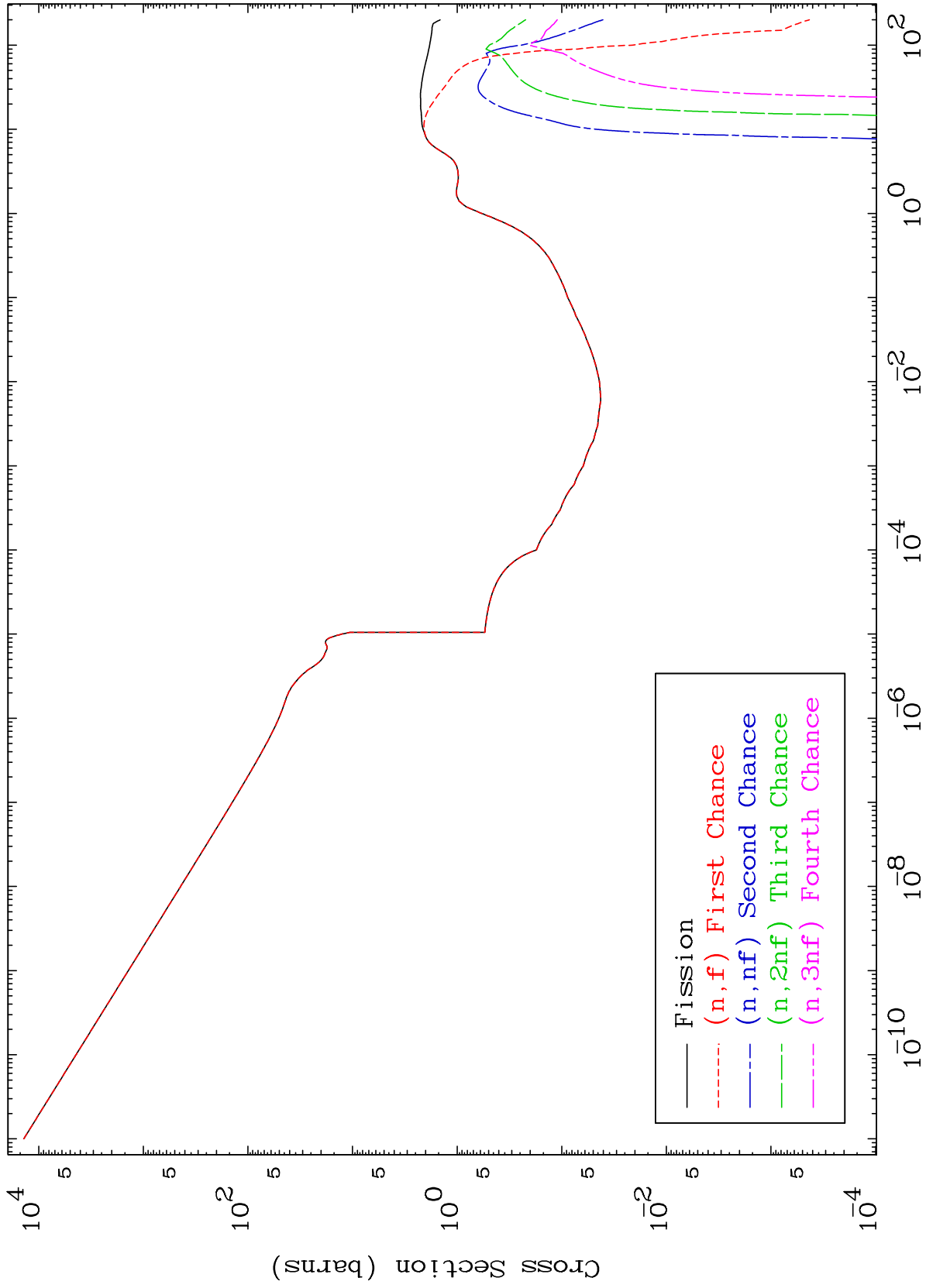




MAT 457

Fission
293 Kelvin Cross Sections

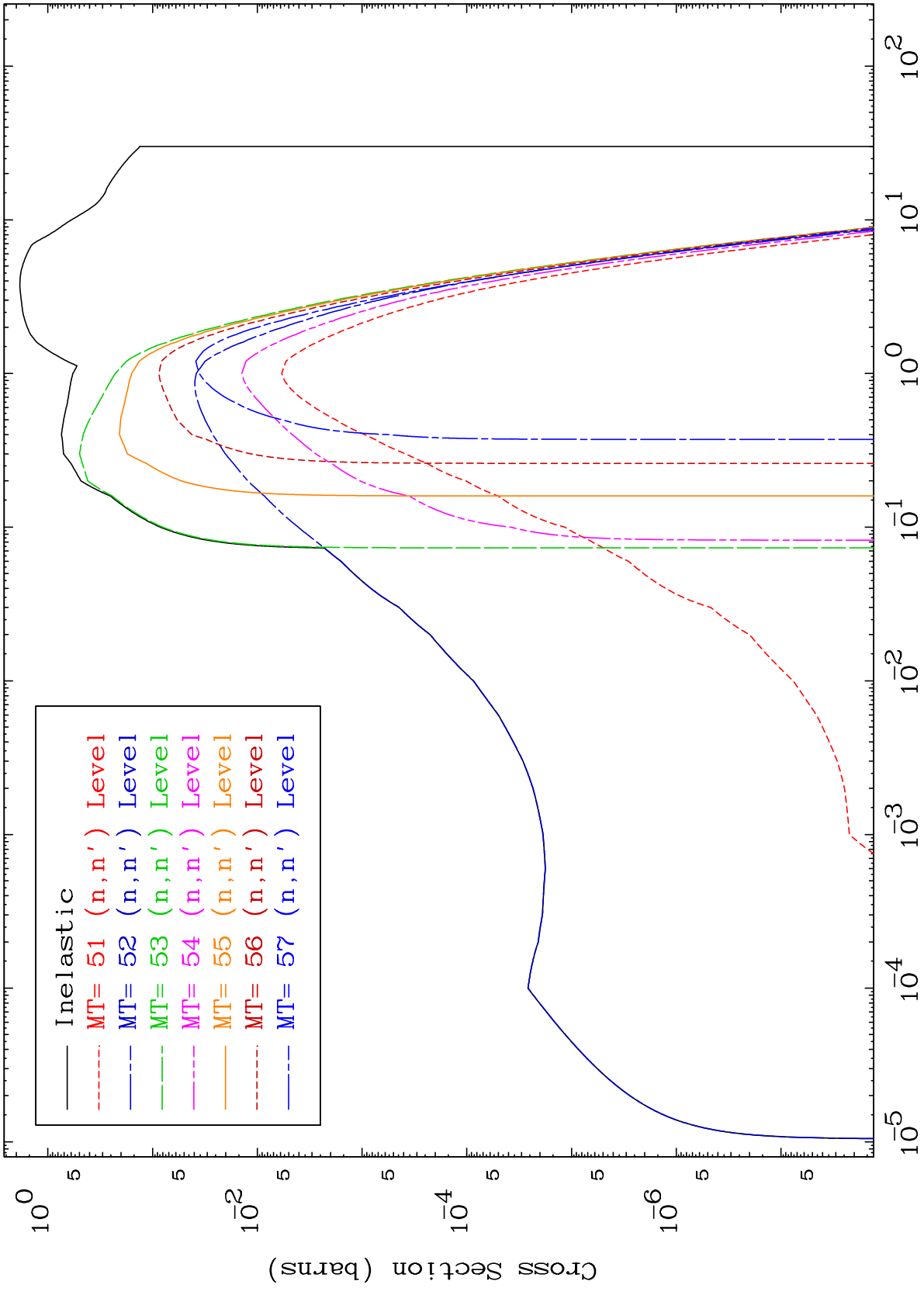
104-Rf-257



MAT 457

293 (n,n') Level

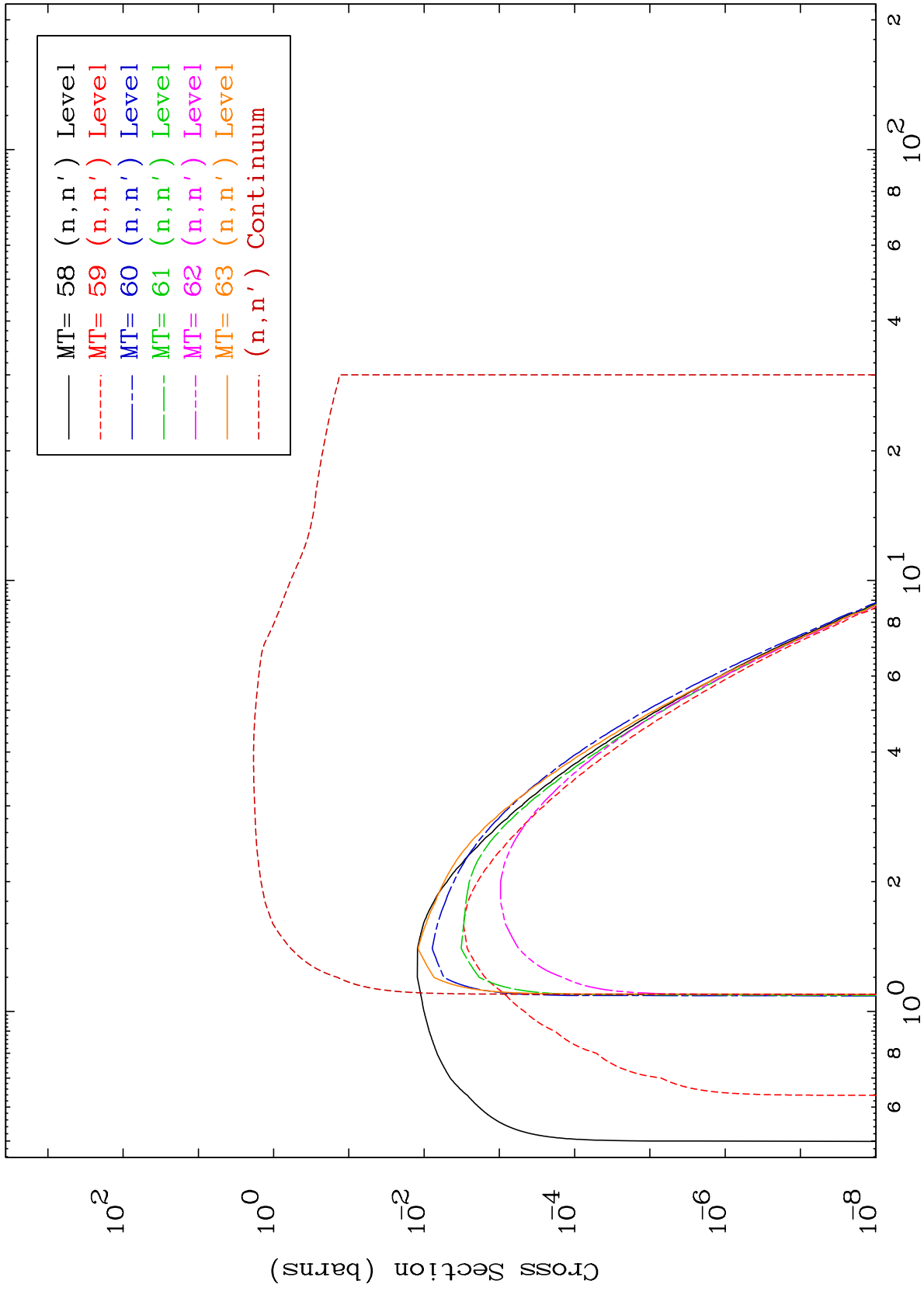
104-Rf-257



MAT 457

(n,n') Level
293 Kelvin Cross Sections

104-Rf-257



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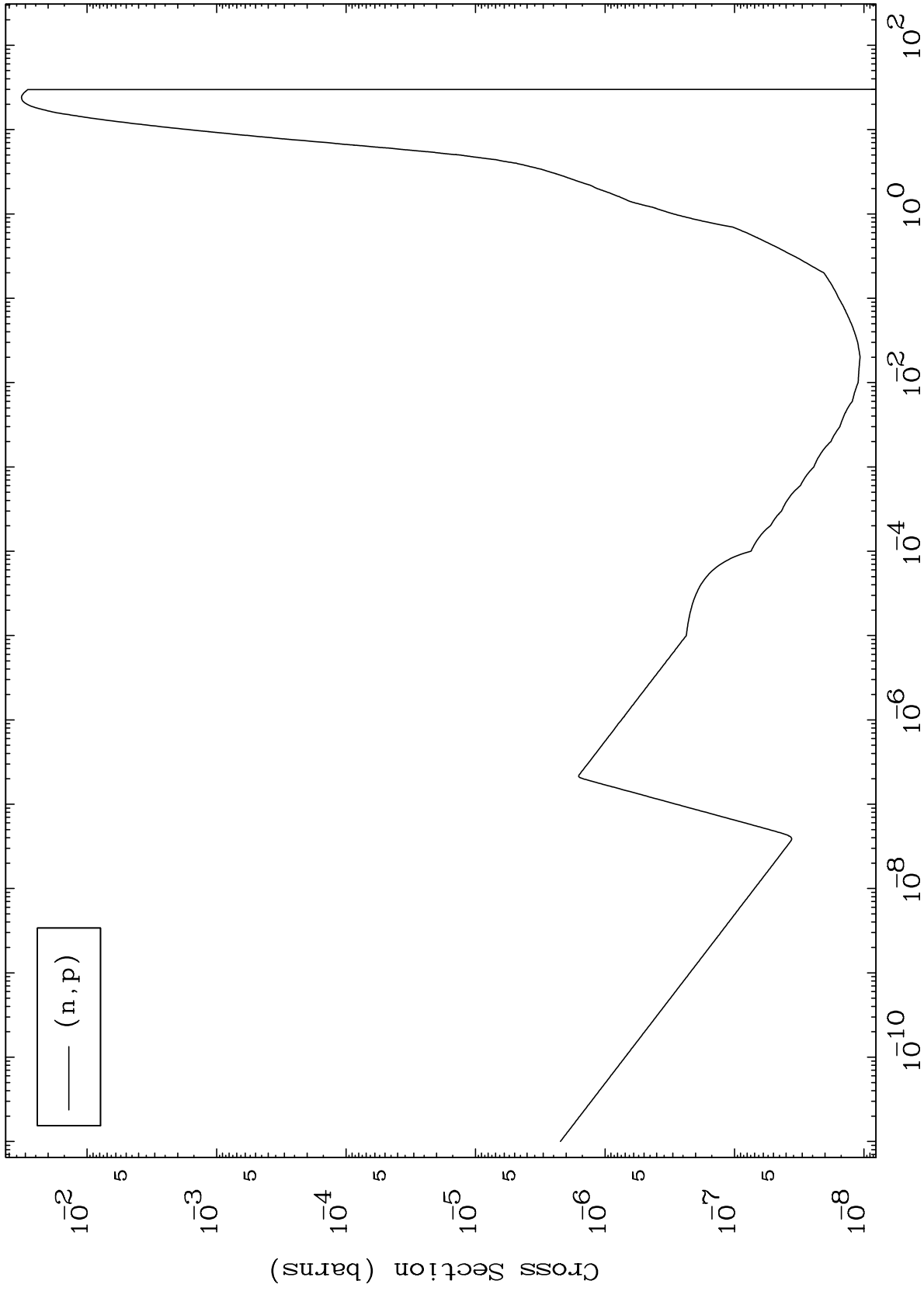
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,p) Levels
293 Kelvin Cross Sections

104-Rf-257



10

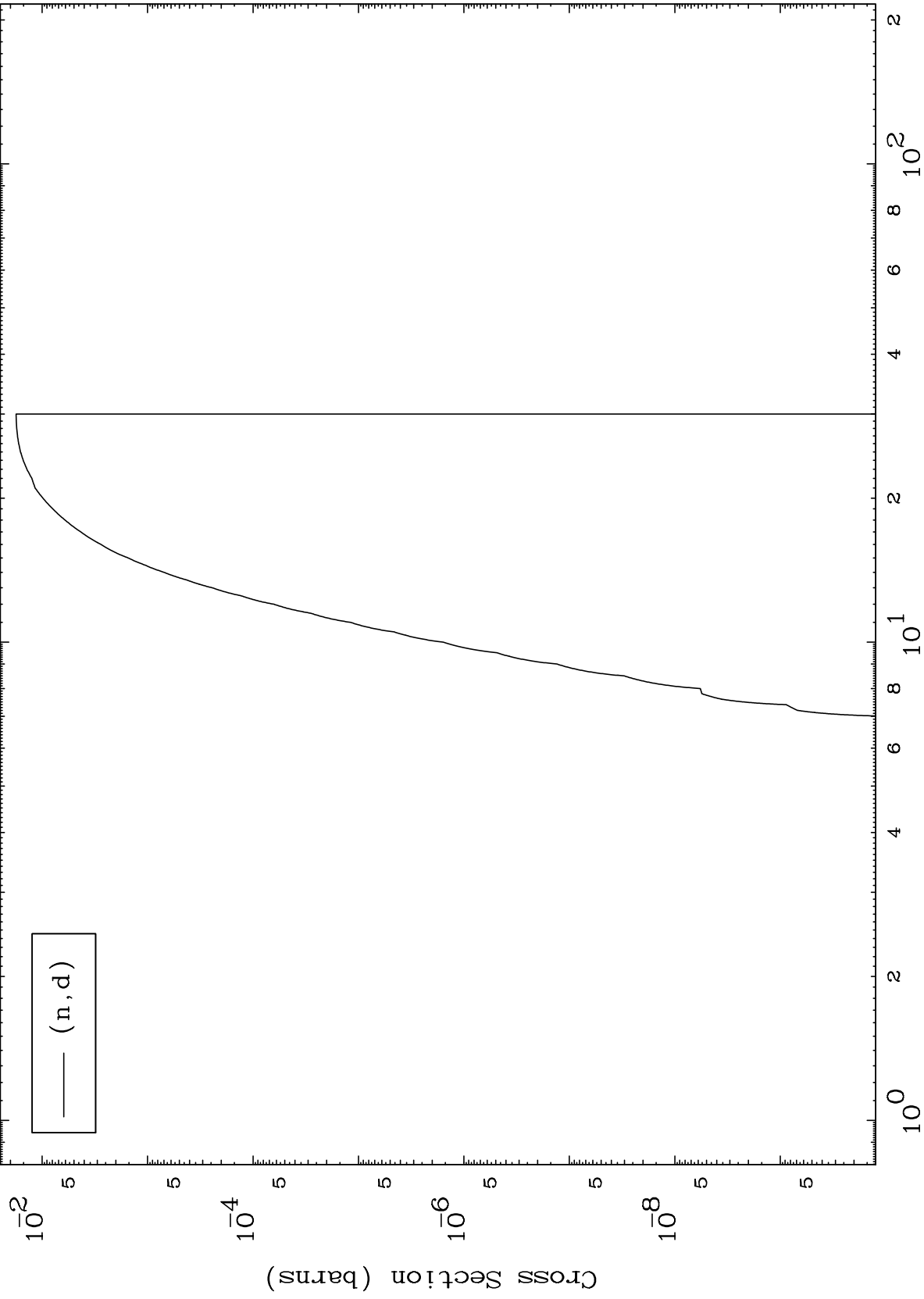
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,d) Levels
293 Kelvin Cross Sections

104-Rf-257



11

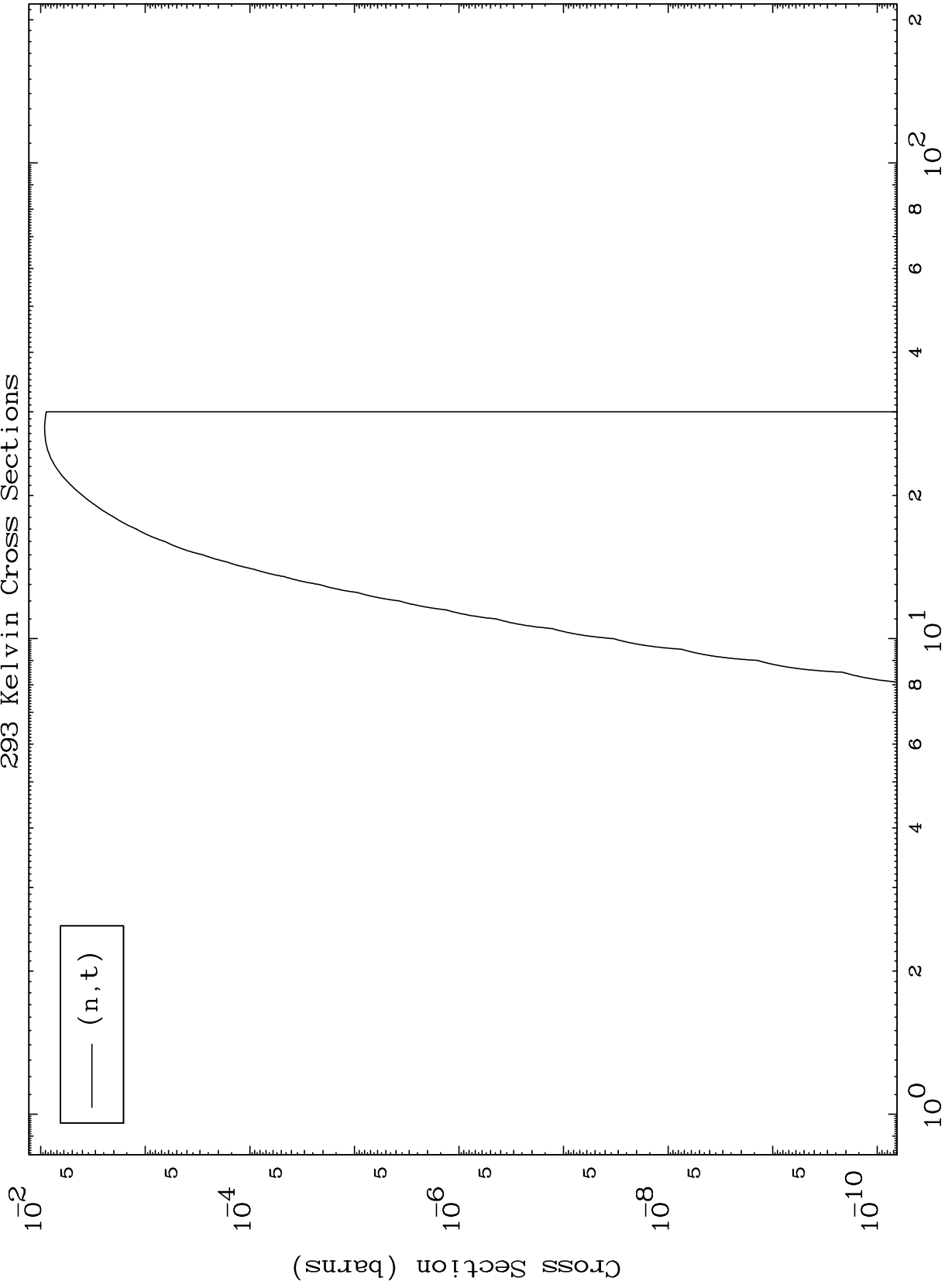
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,t) Levels
293 Kelvin Cross Sections

104-Rf-257



12

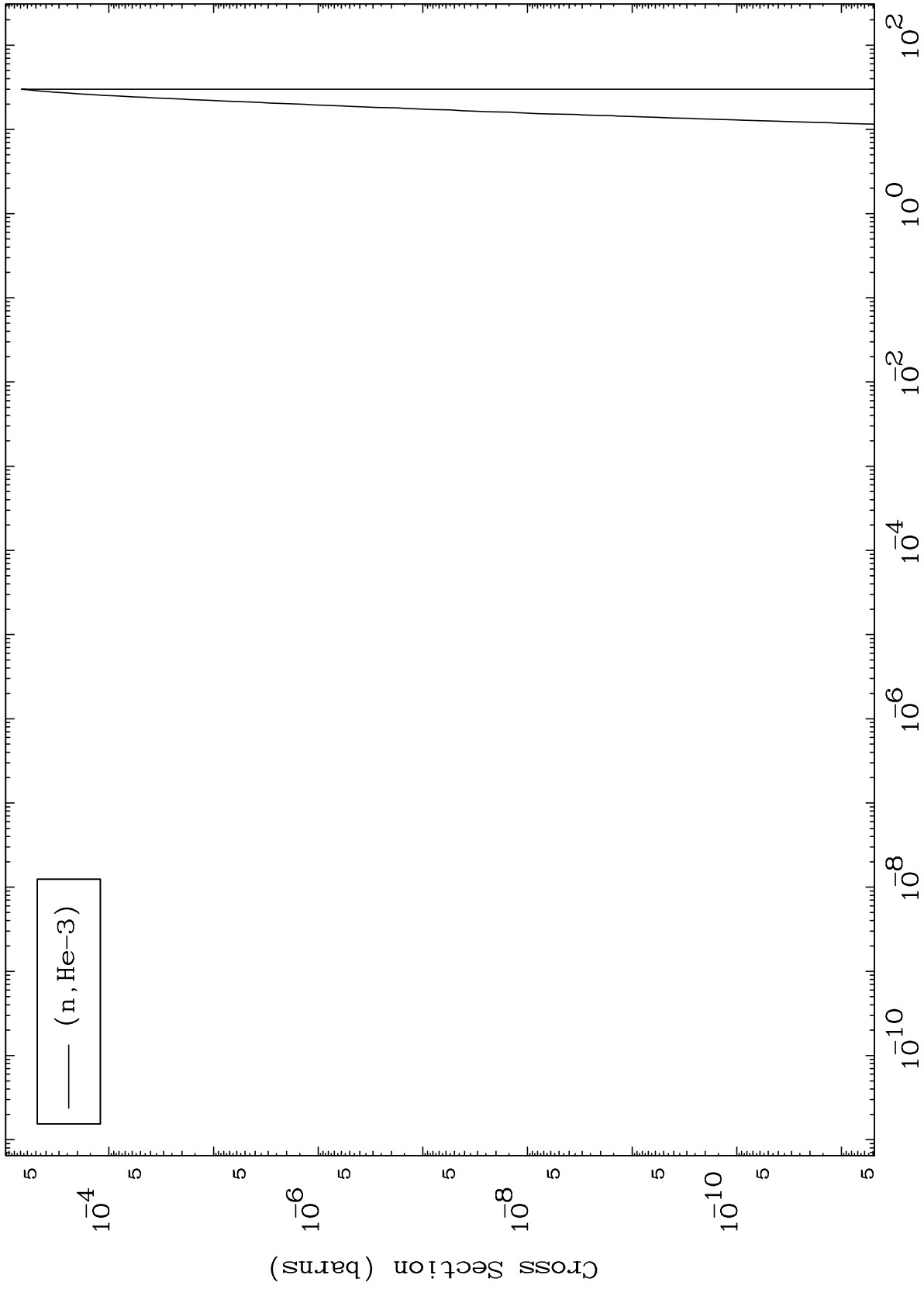
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,He3) Levels
293 Kelvin Cross Sections

104-Rf-257



13

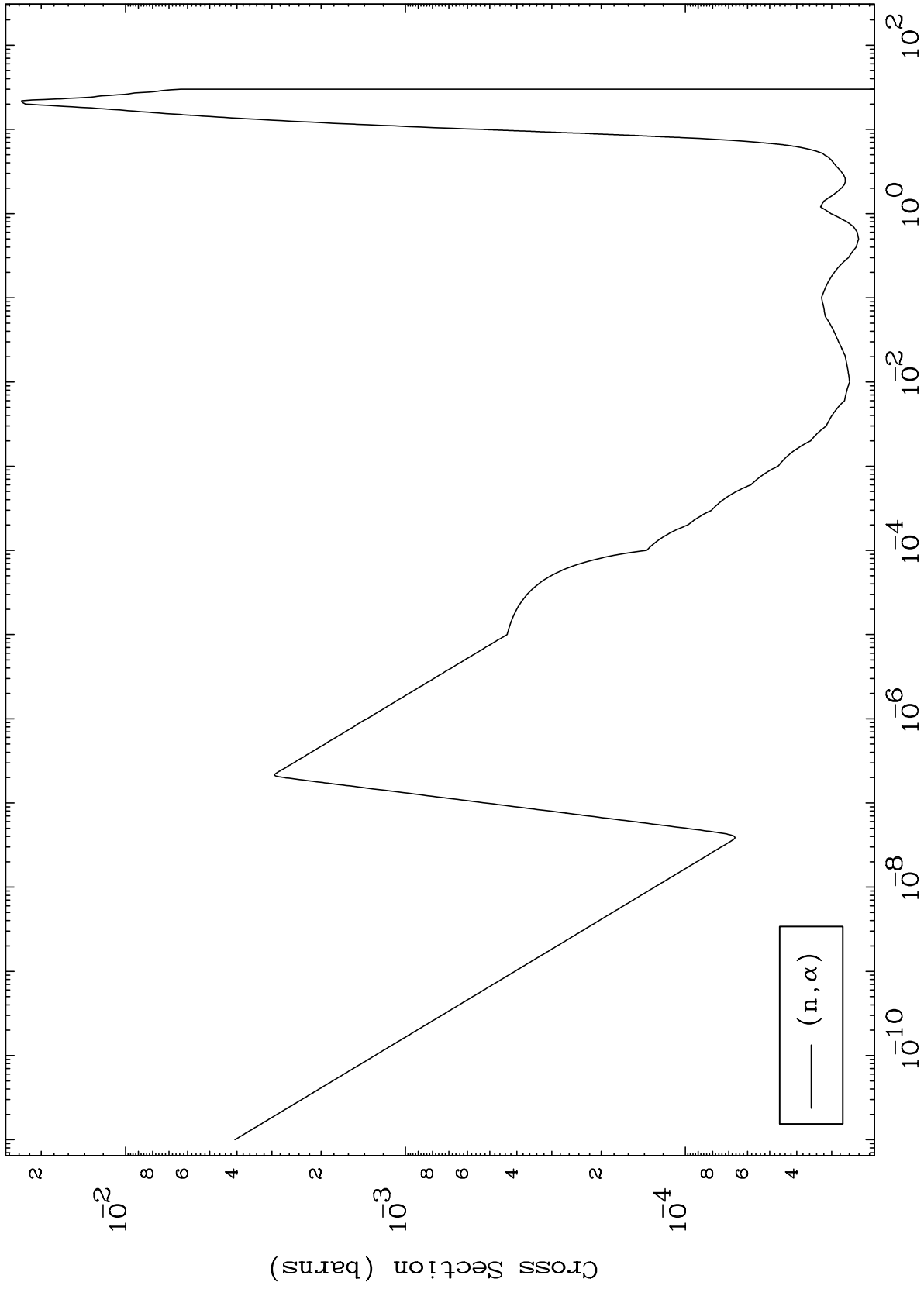
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,α) Levels
293 Kelvin Cross Sections

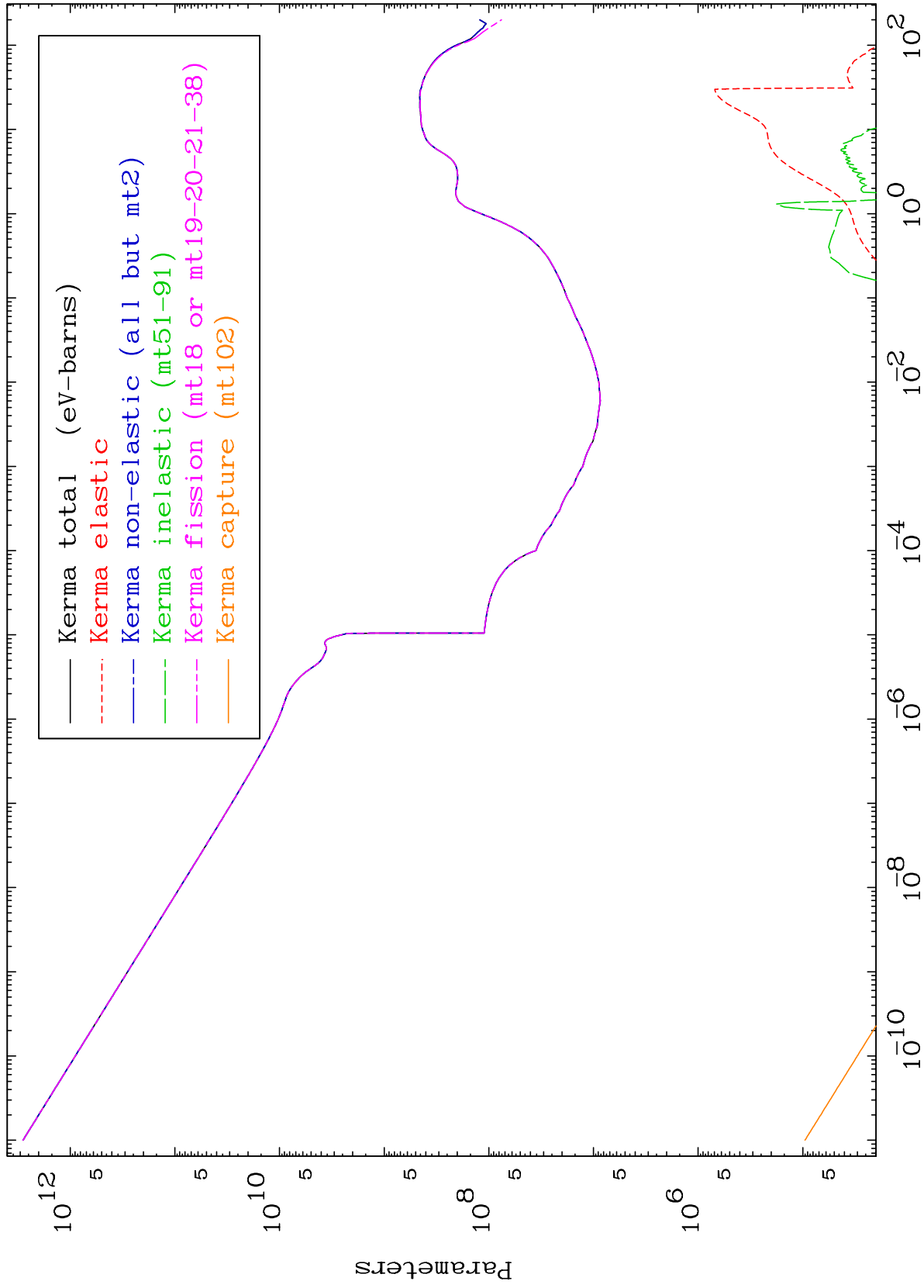
104-Rf-257

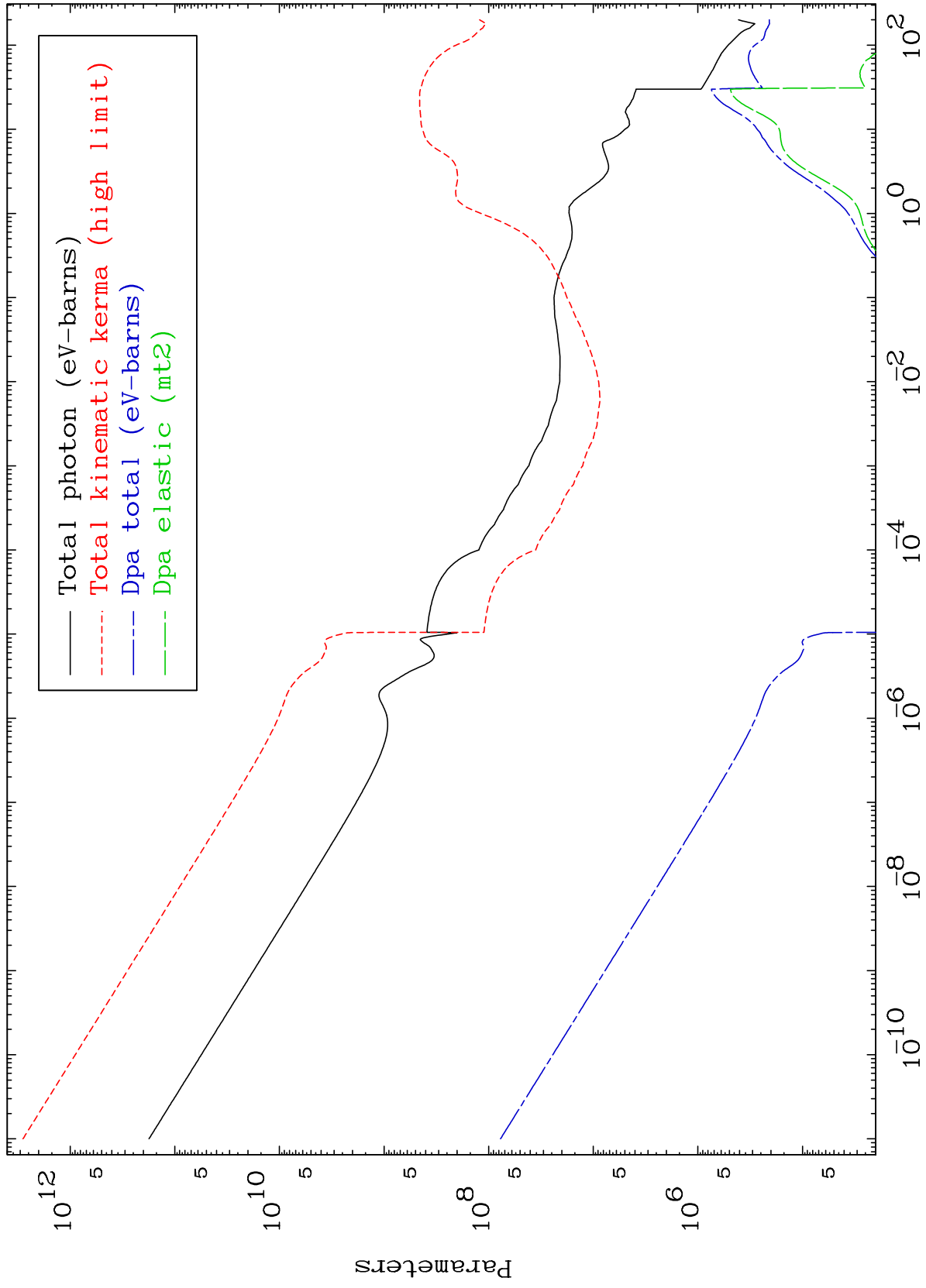


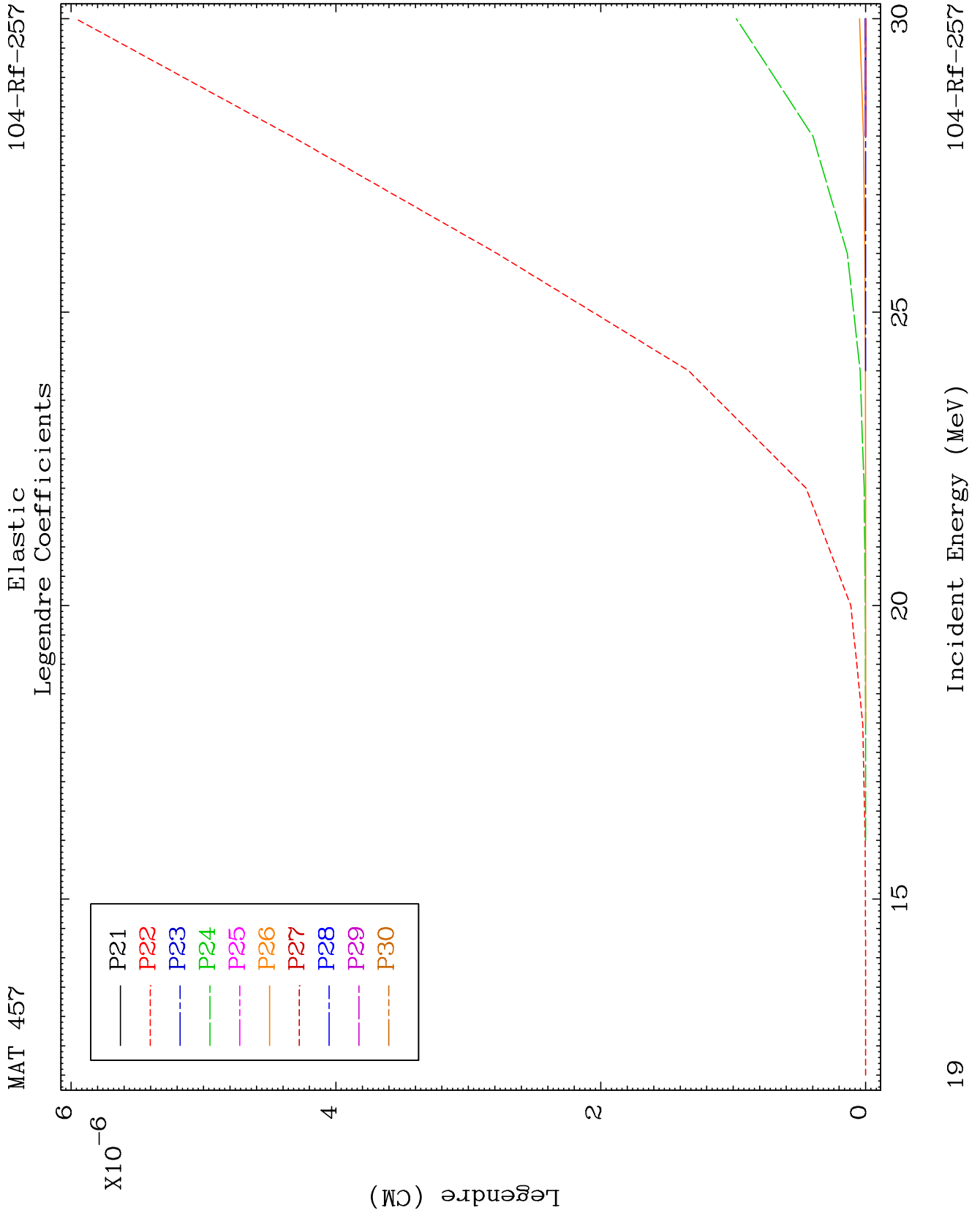
14

Incident Energy (MeV)

104-Rf-257



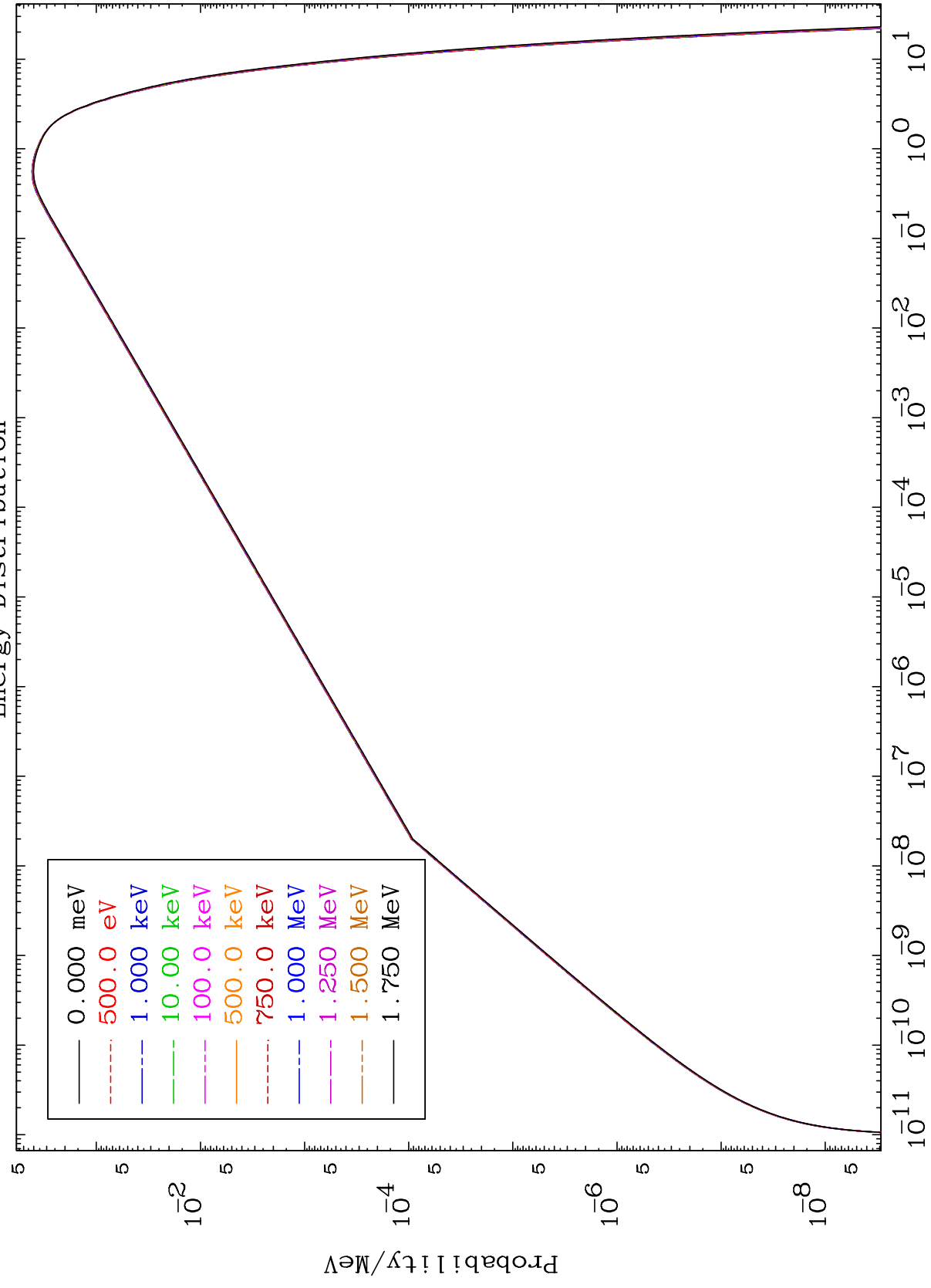




MAT 457

Fission Energy Distribution

104-Rf-257

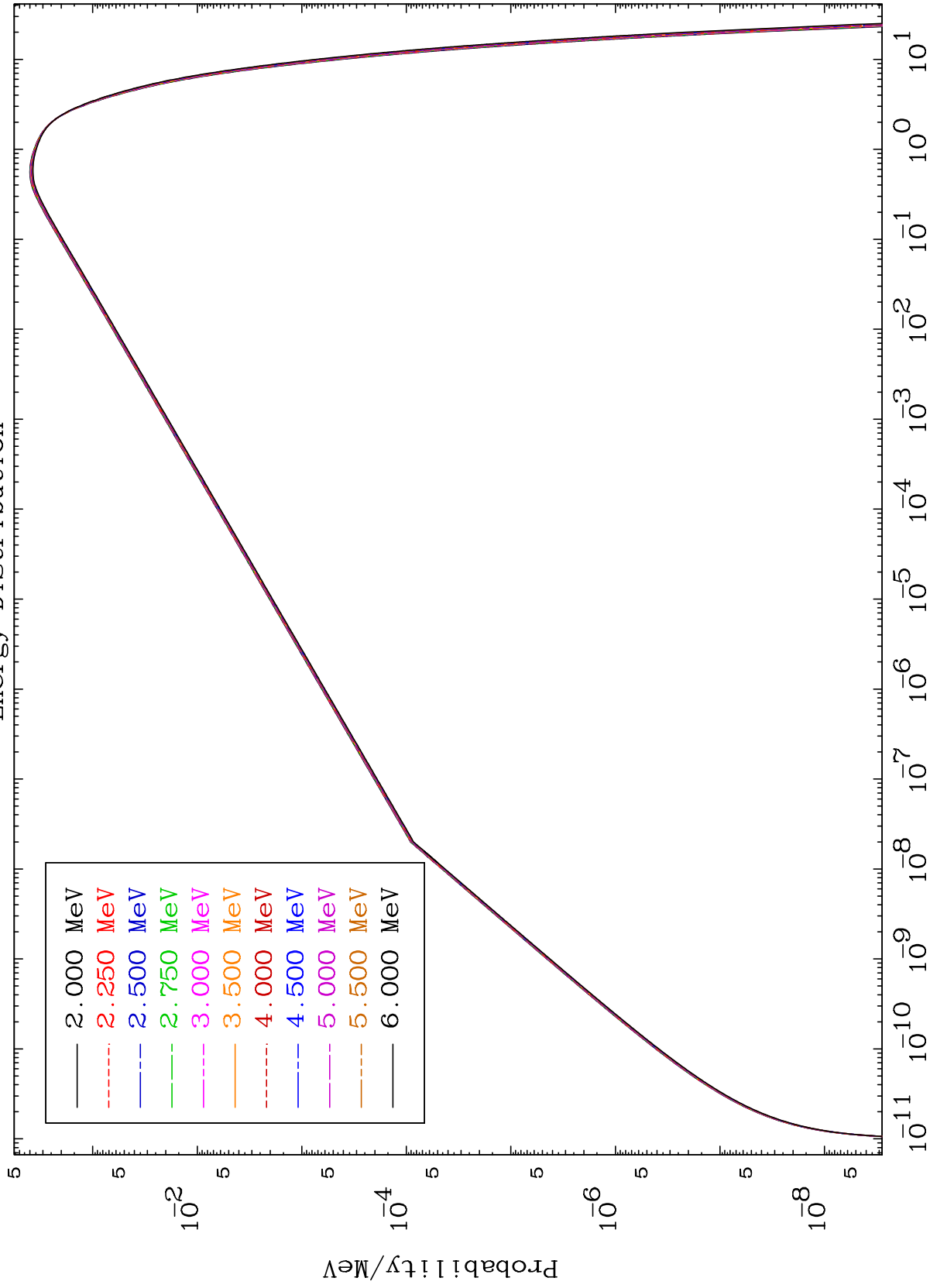


104-Rf-257

MAT 457

Fission Energy Distribution

104-Rf-257

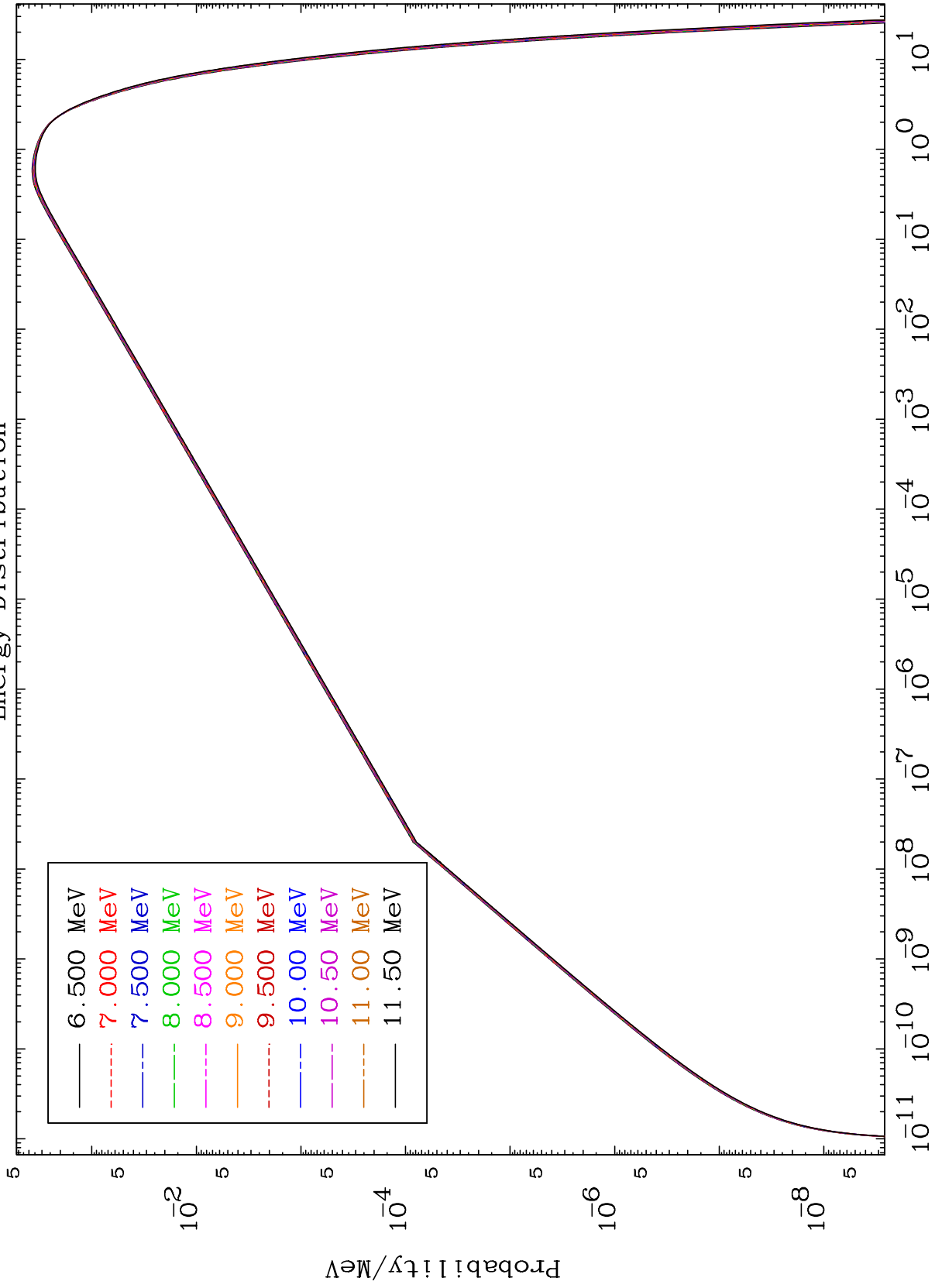


104-Rf-257

MAT 457

Fission
Energy Distribution

104-Rf-257



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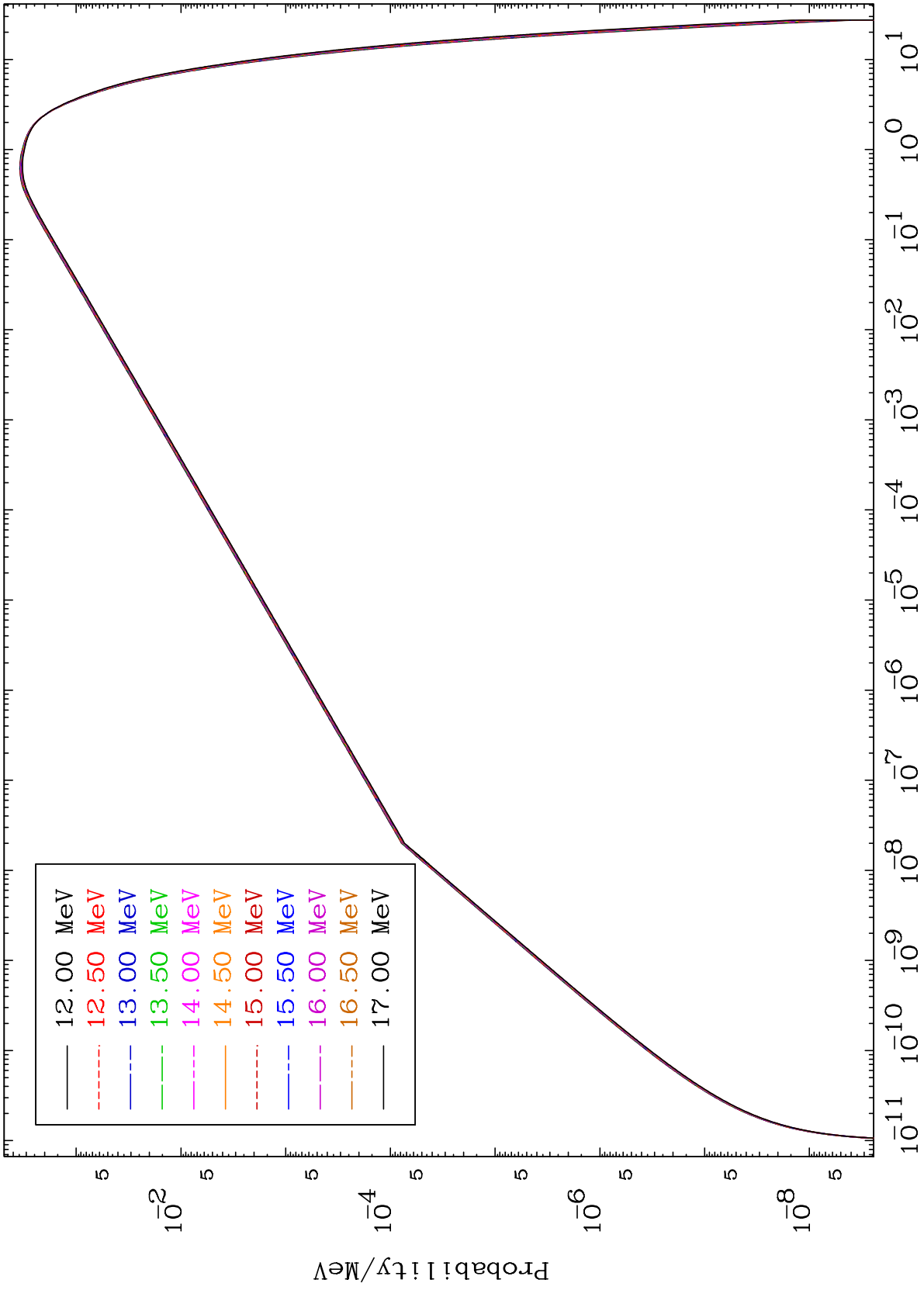
Secondary Energy (MeV)

104-Rf-257

MAT 457

Fission Energy Distribution

104-Rf-257

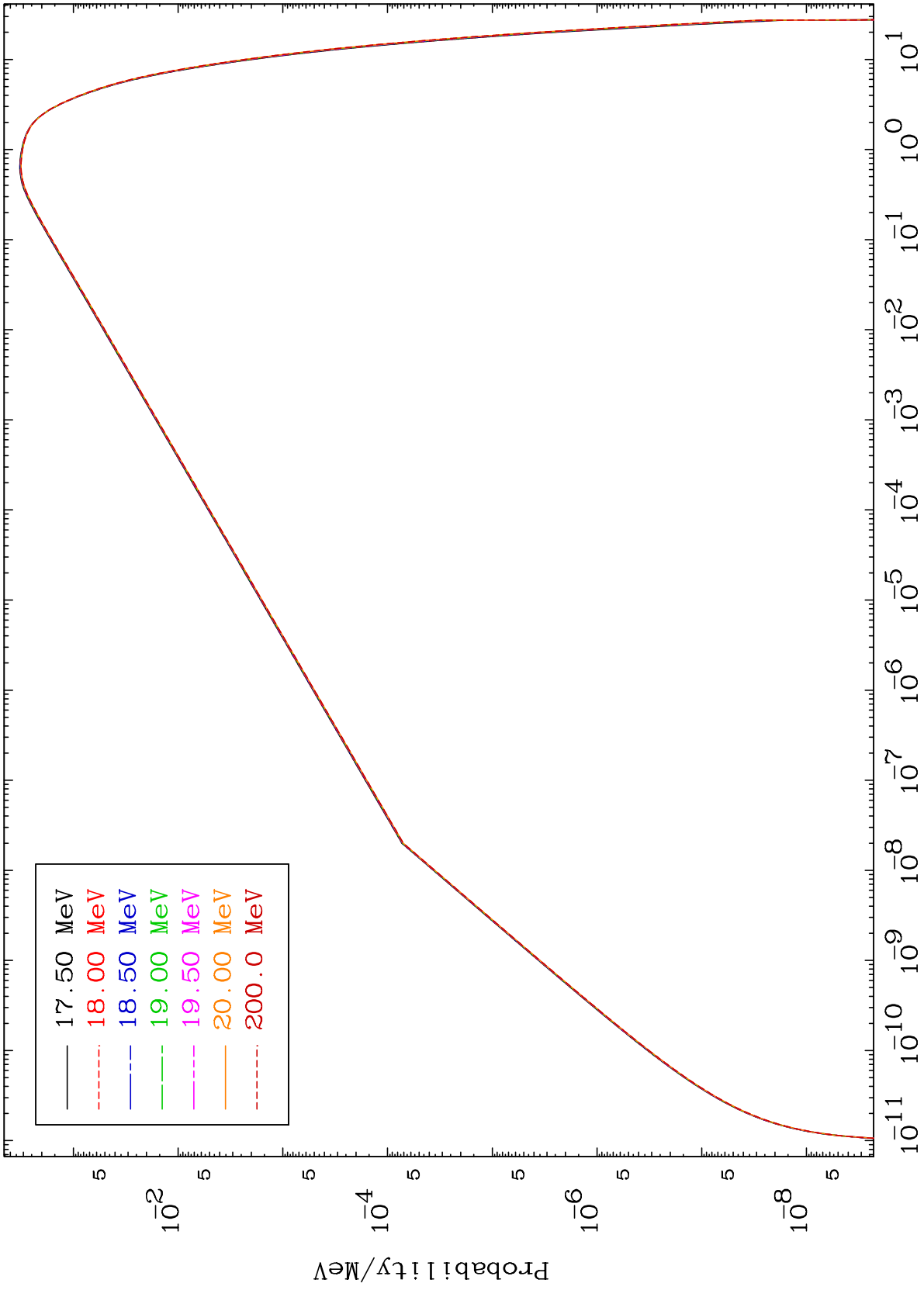


104-Rf-257

MAT 457

Fission Energy Distribution

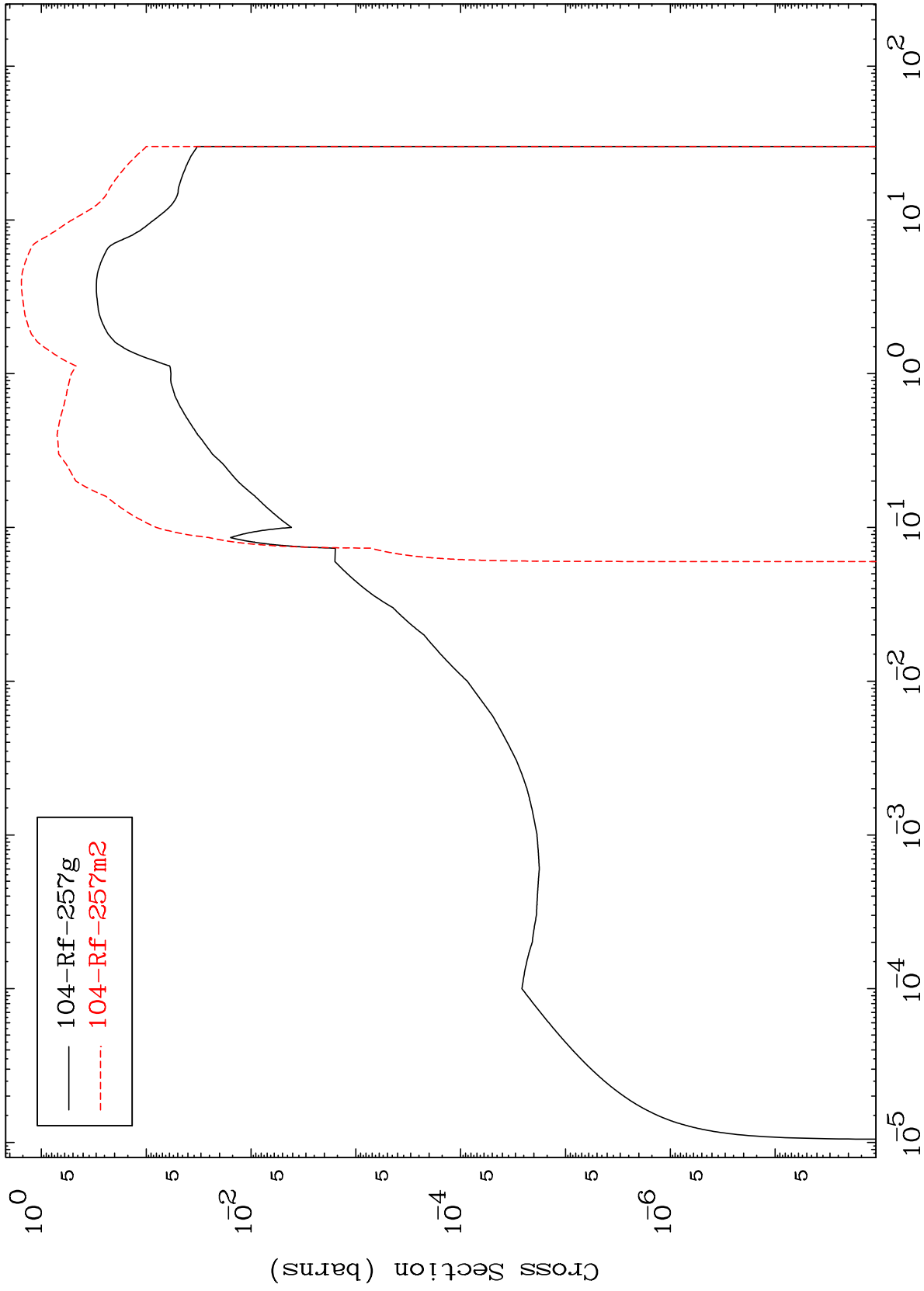
104-Rf-257



MAT 457

Inelastic
Radionuclide Production Cross Section

104-Rf-257



— 104-Rf-257g
- - - 104-Rf-257m2

Incident Energy (MeV)

104-Rf-257

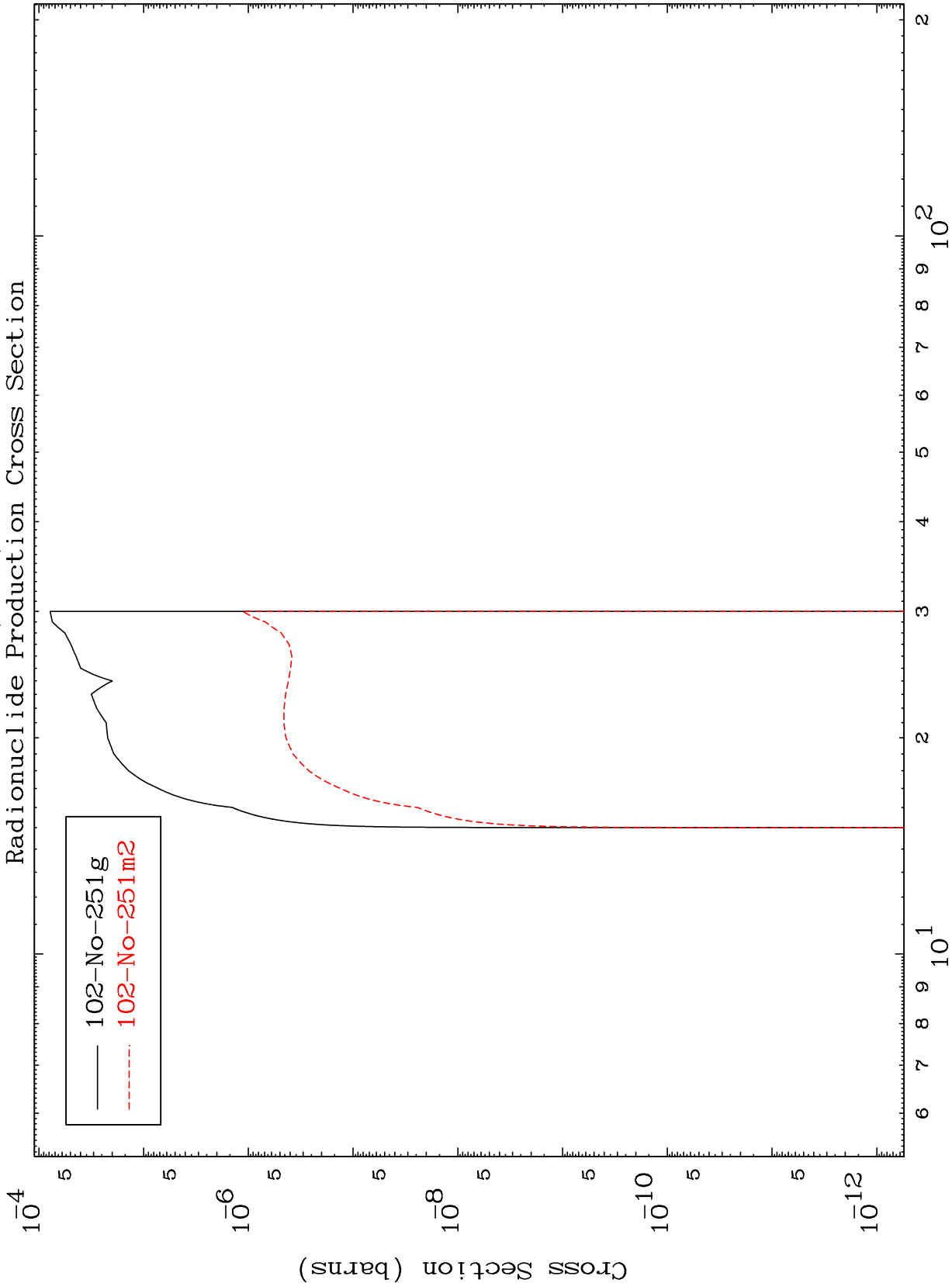
25

MAT 457

104-Rf-257

(n,3n) α

Radionuclide Production Cross Section



104-Rf-257

Incident Energy (MeV)

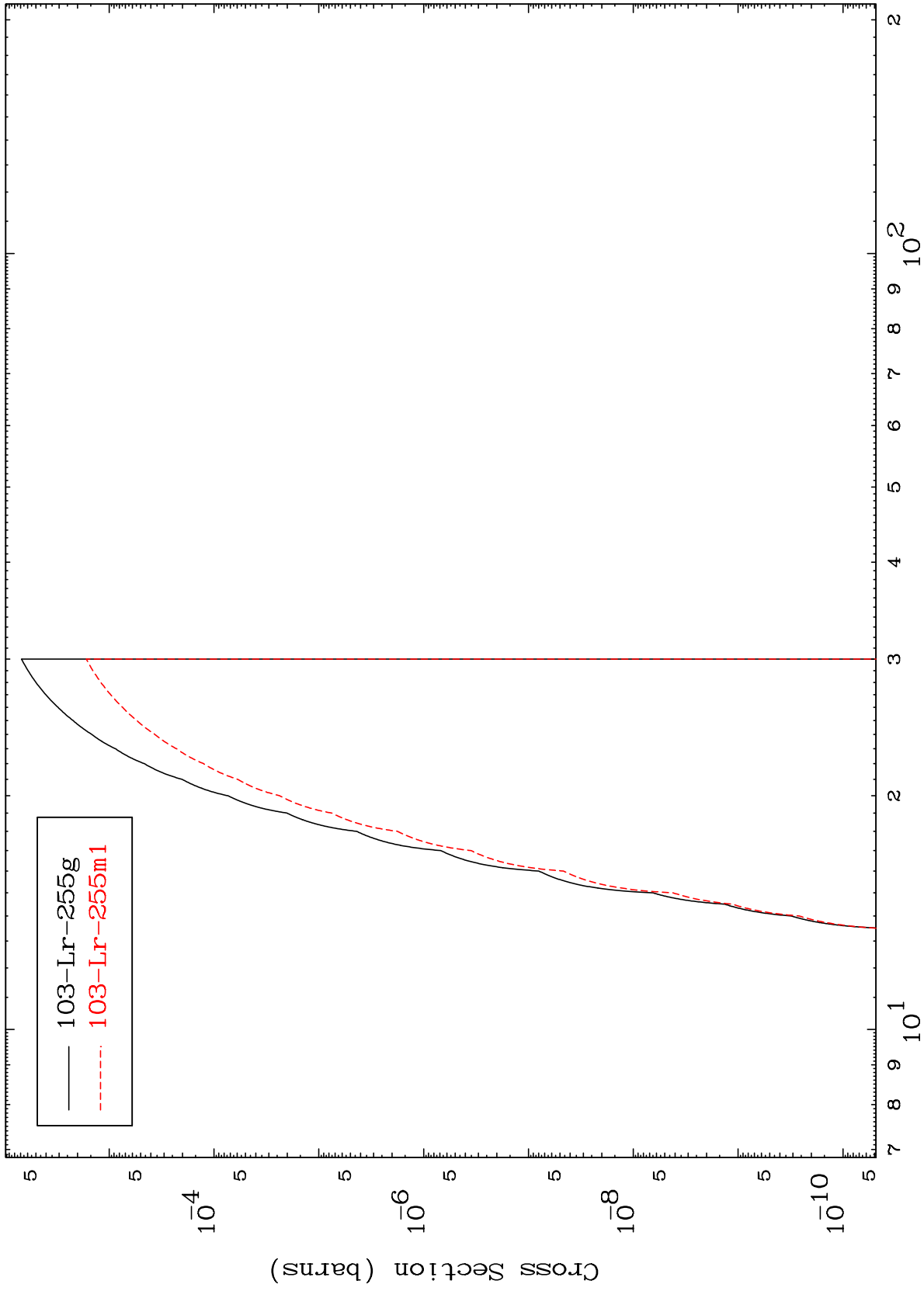
26

MAT 457

(n,n') d

104-Rf-257

Radionuclide Production Cross Section



27

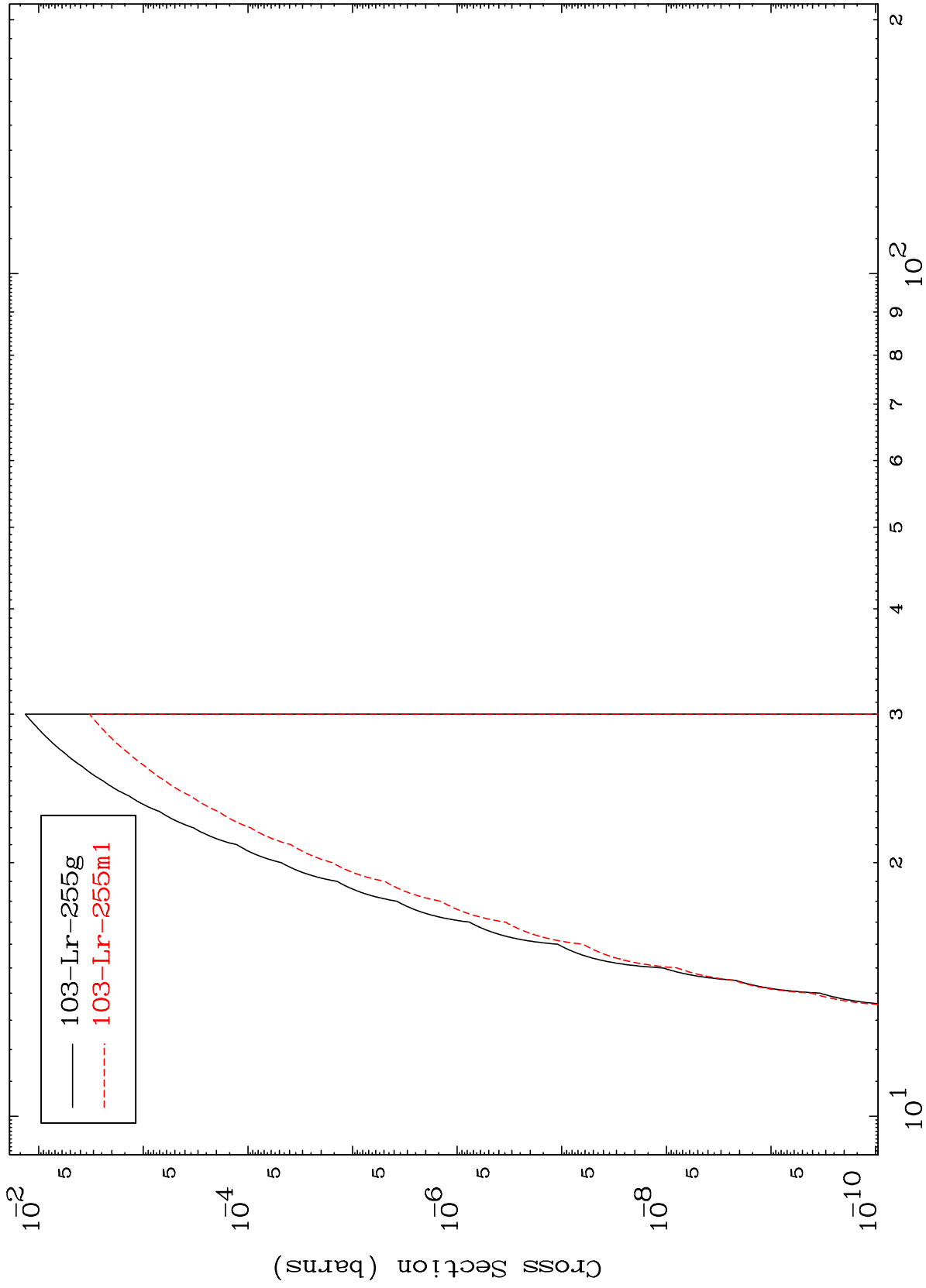
104-Rf-257

MAT 457

(n,2n) p

104-Rf-257

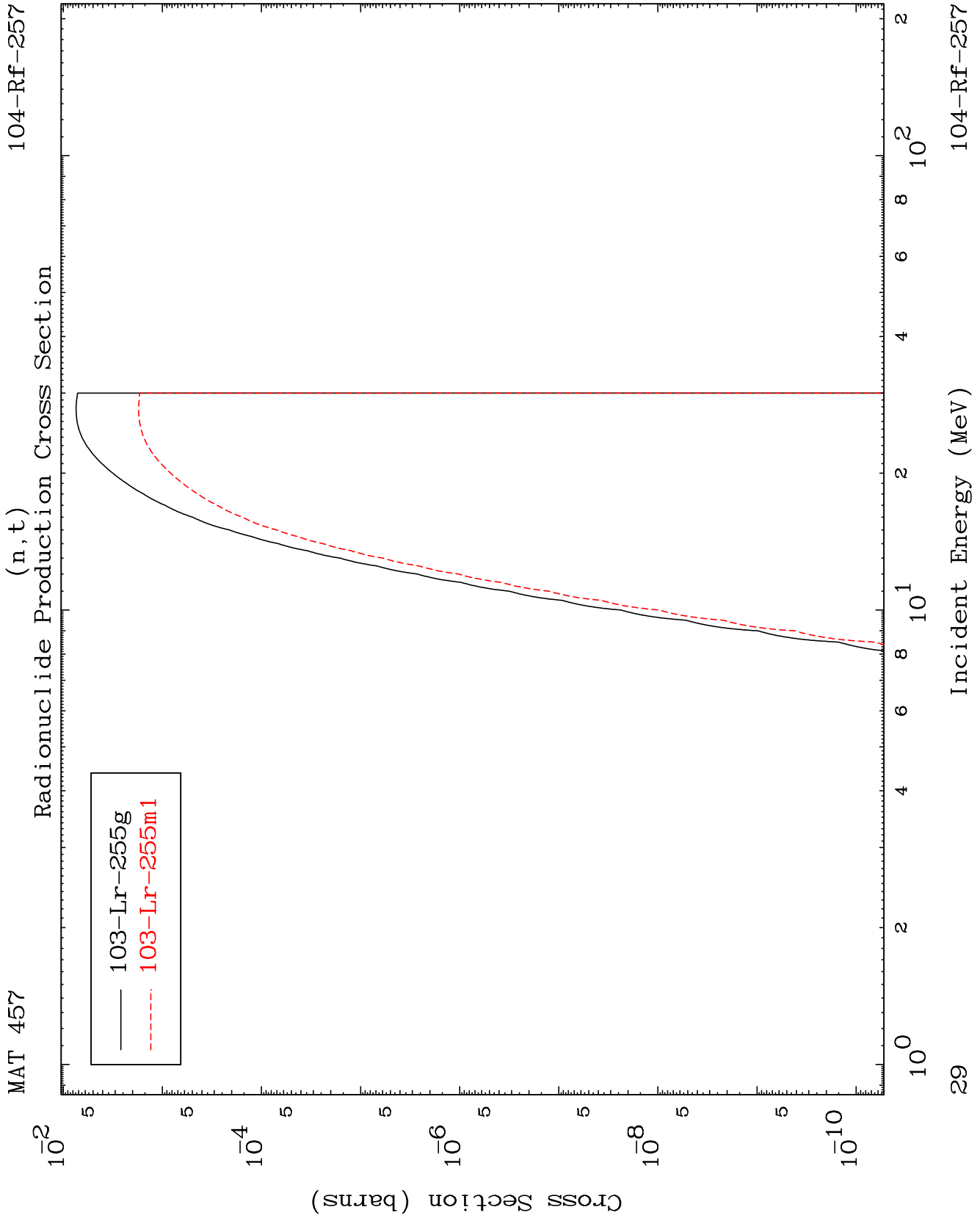
Radionuclide Production Cross Section



28

Incident Energy (MeV)

104-Rf-257



MAT 457

(n,2α)

104-Rf-257

Radionuclide Production Cross Section

