

Program EVALPLOT  
(Version 2018-1)

by

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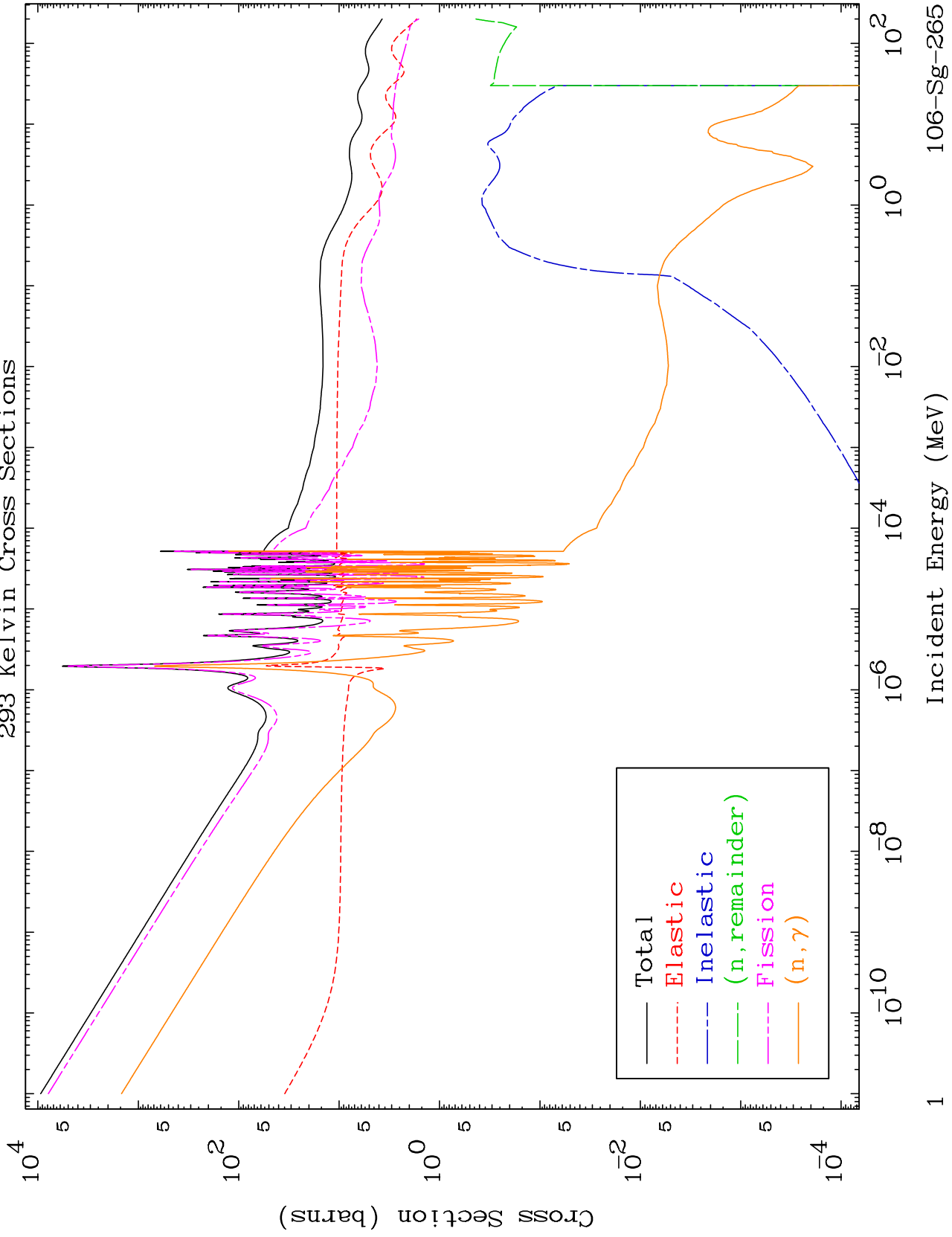
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 665

Major  
293 Kelvin Cross Sections

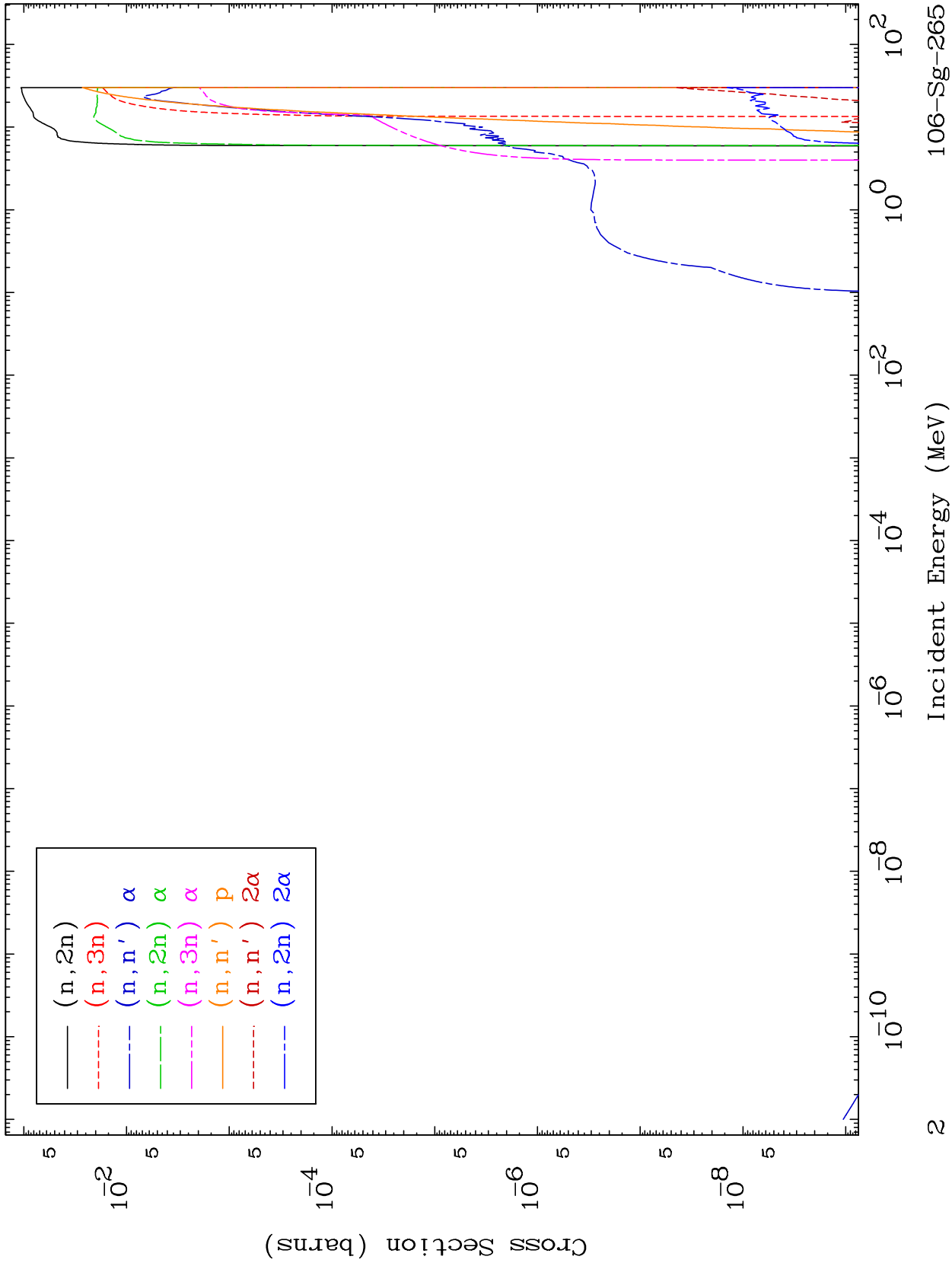
106-Sg-265



MAT 665

Neutron Production  
293 Kelvin Cross Sections

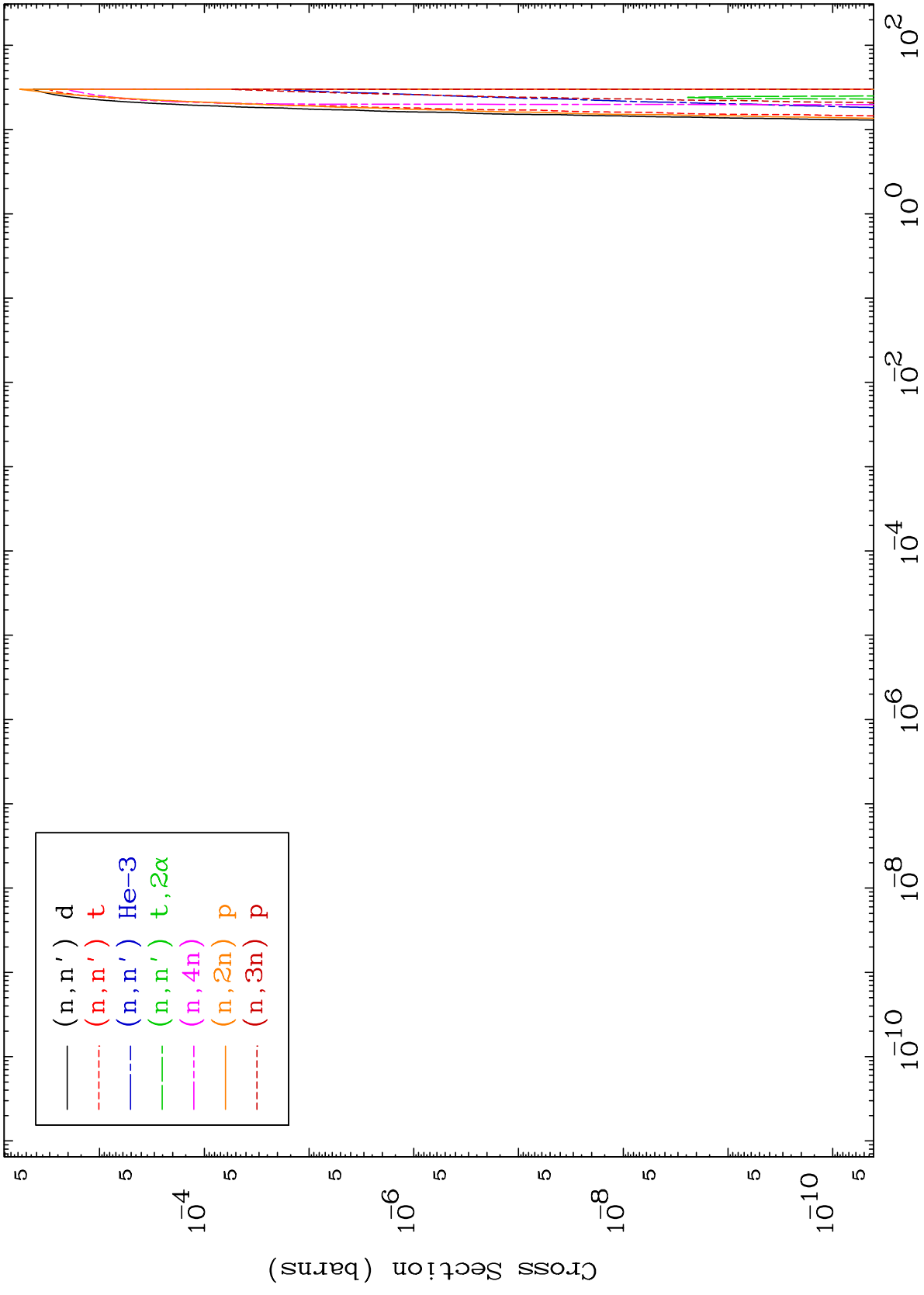
106-Sg-265



MAT 665

Neutron Production  
293 Kelvin Cross Sections

106-Sg-265

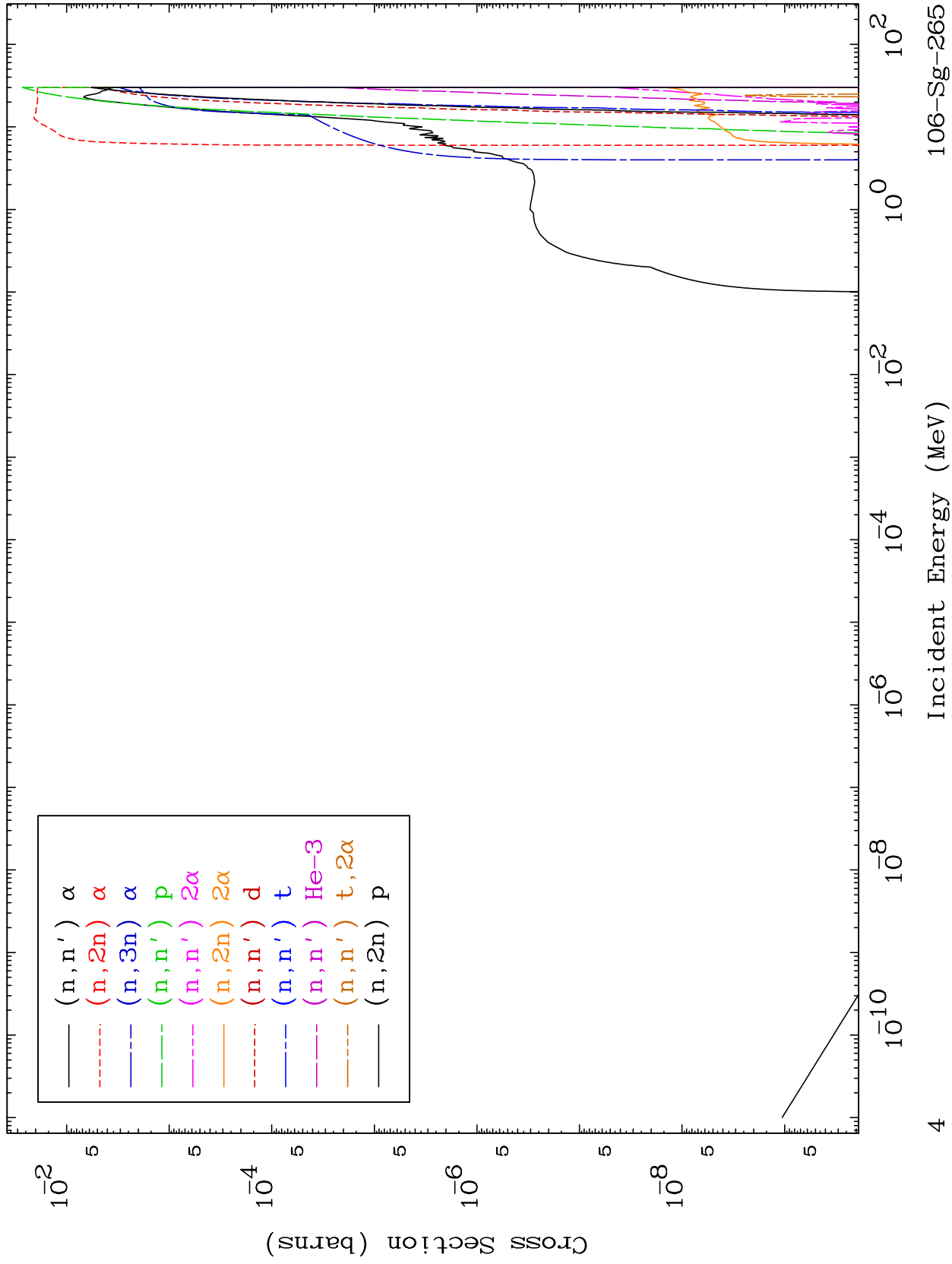


106-Sg-265

MAT 665

Charged Particle  
293 Kelvin Cross Sections

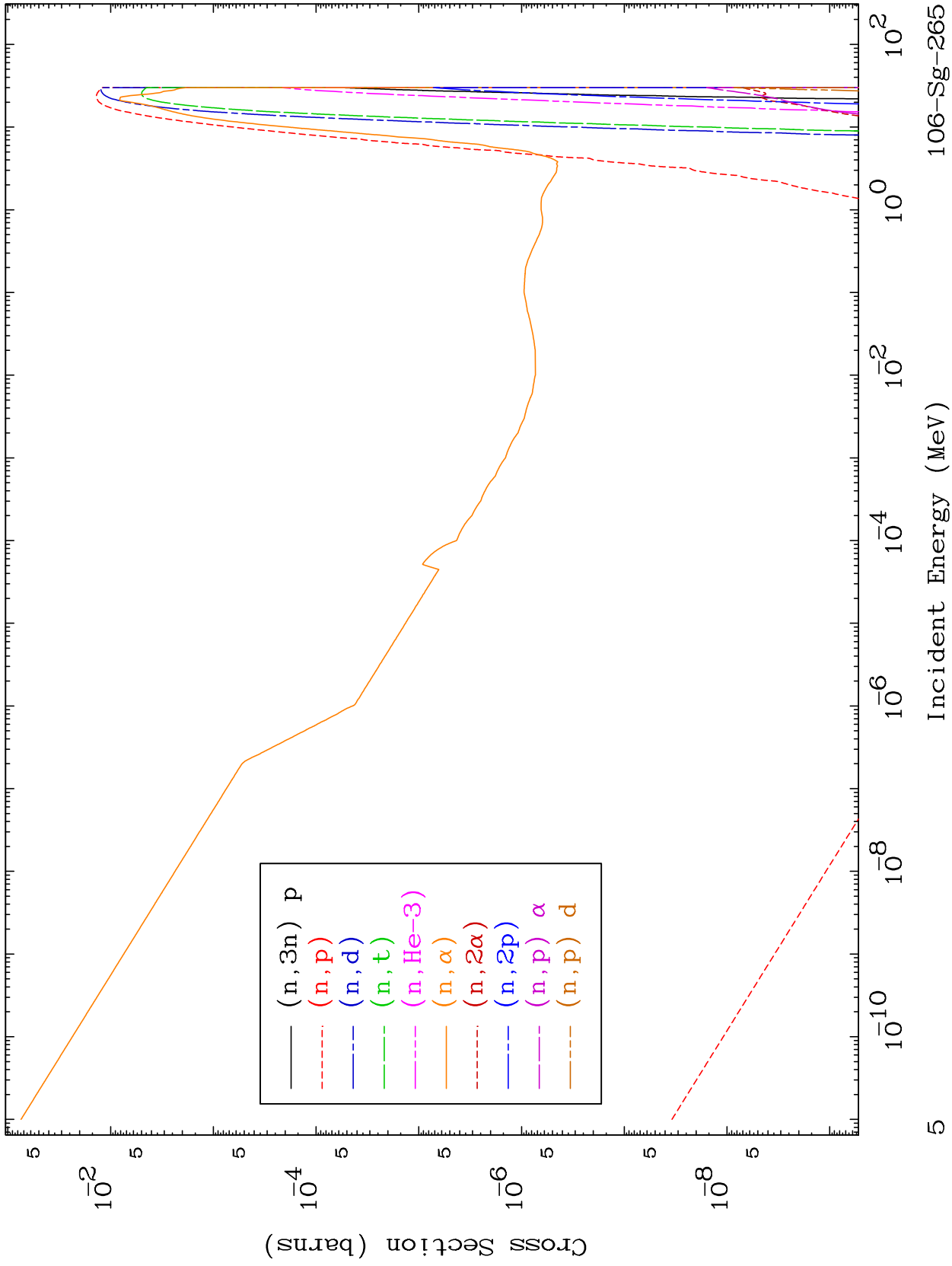
106-Sg-265



MAT 665

293 Kelvin Cross Sections

106-Sg-265

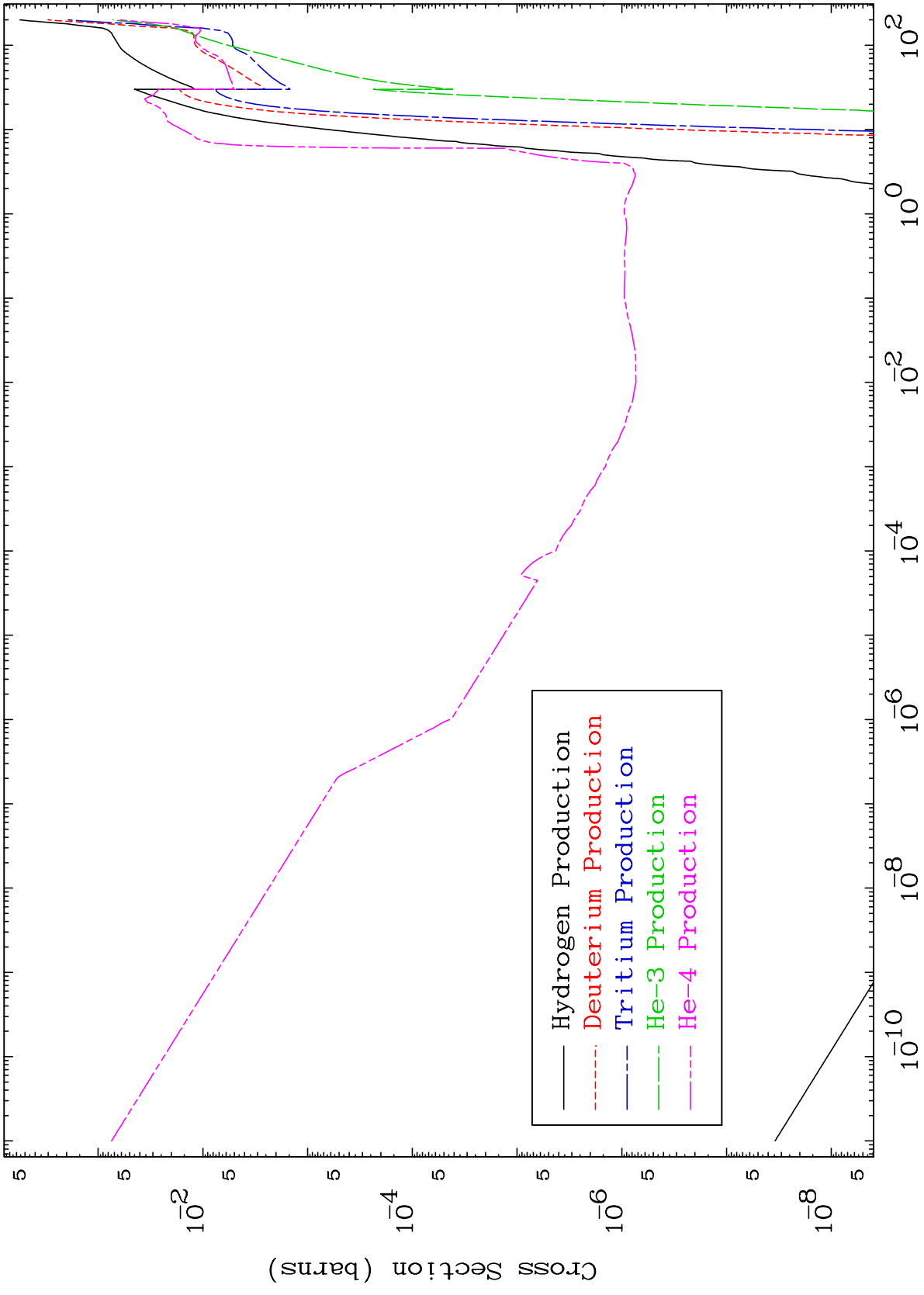


5

MAT 665

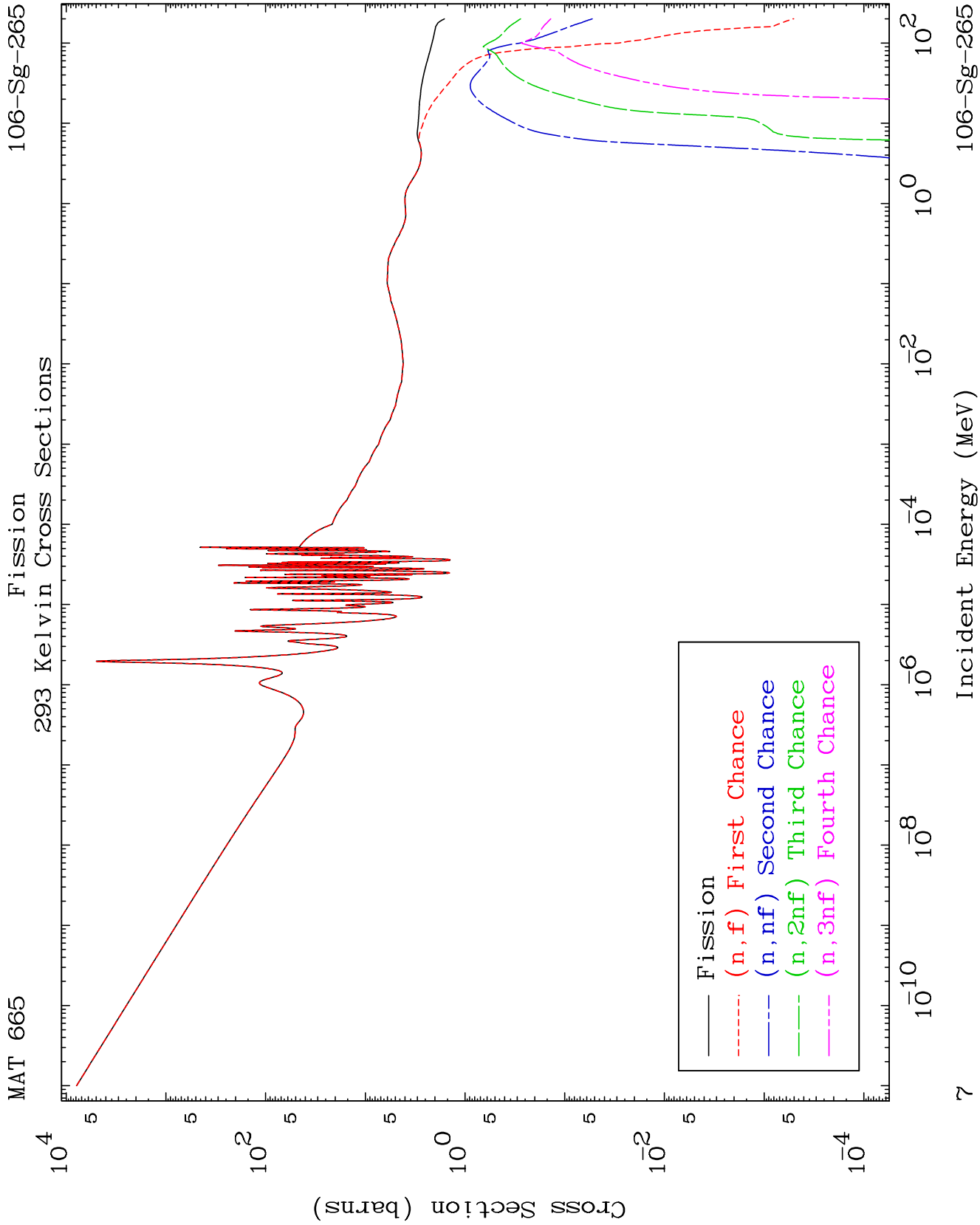
Particle Production  
293 Kelvin Cross Sections

106-Sg-265

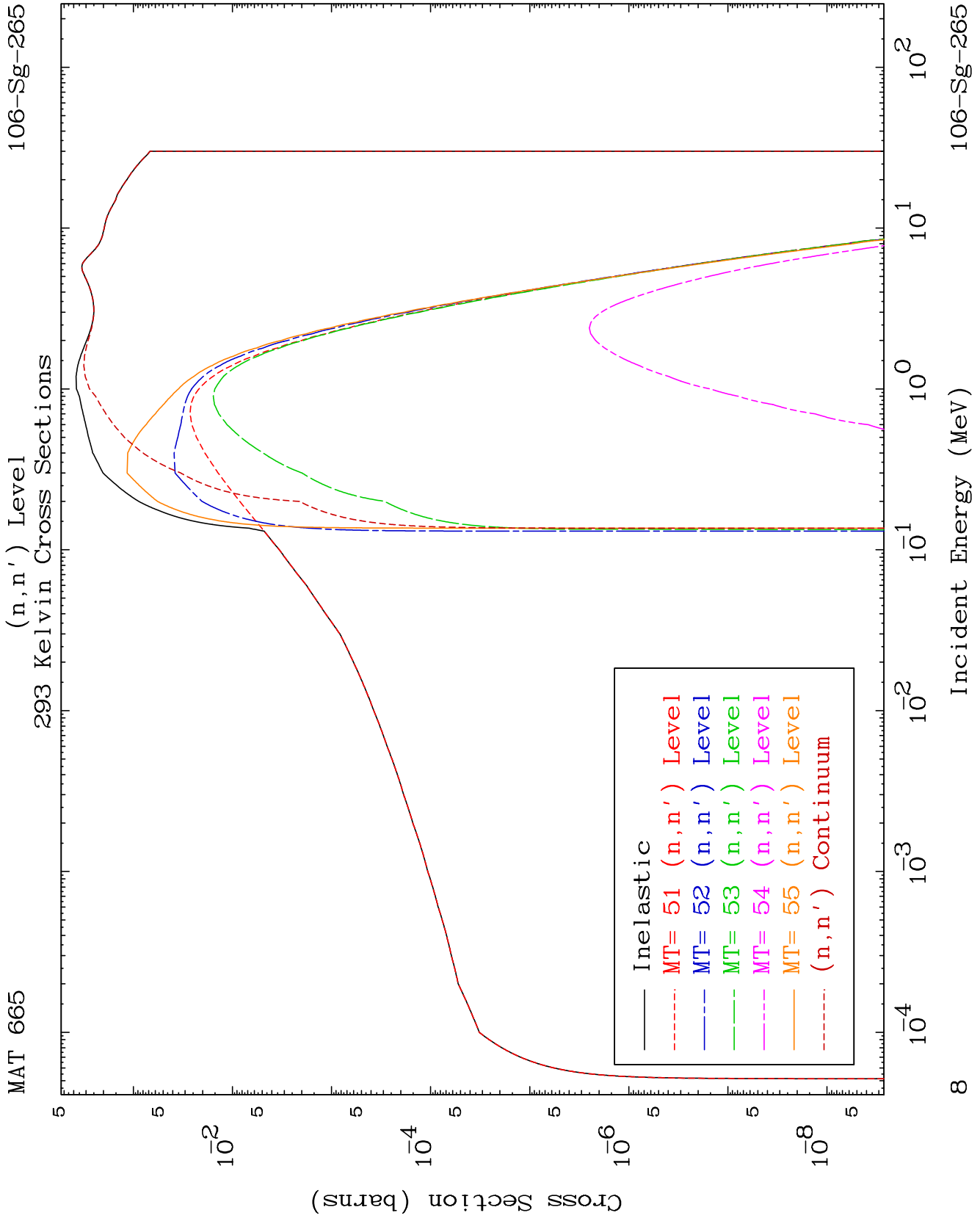


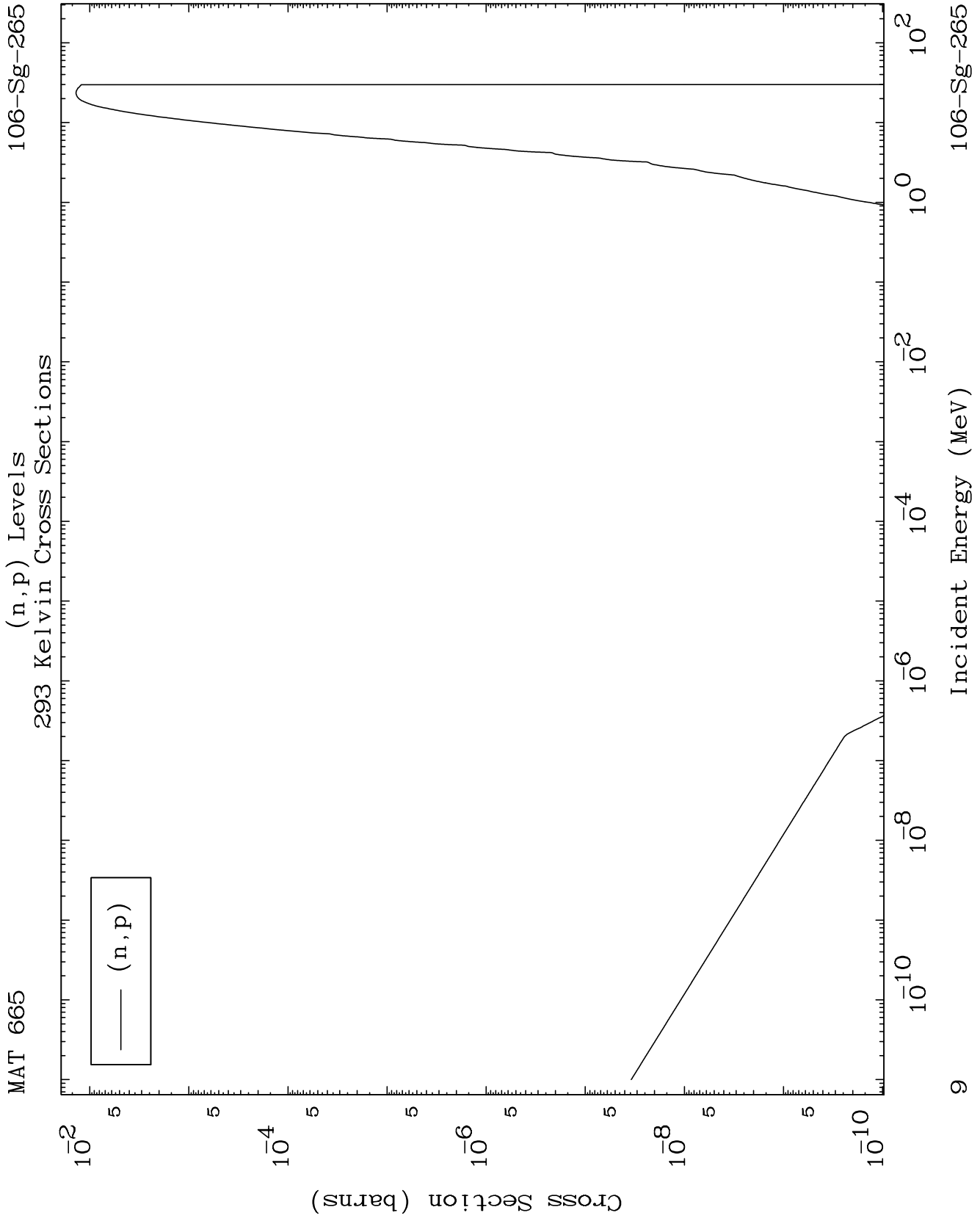
6

106-Sg-265





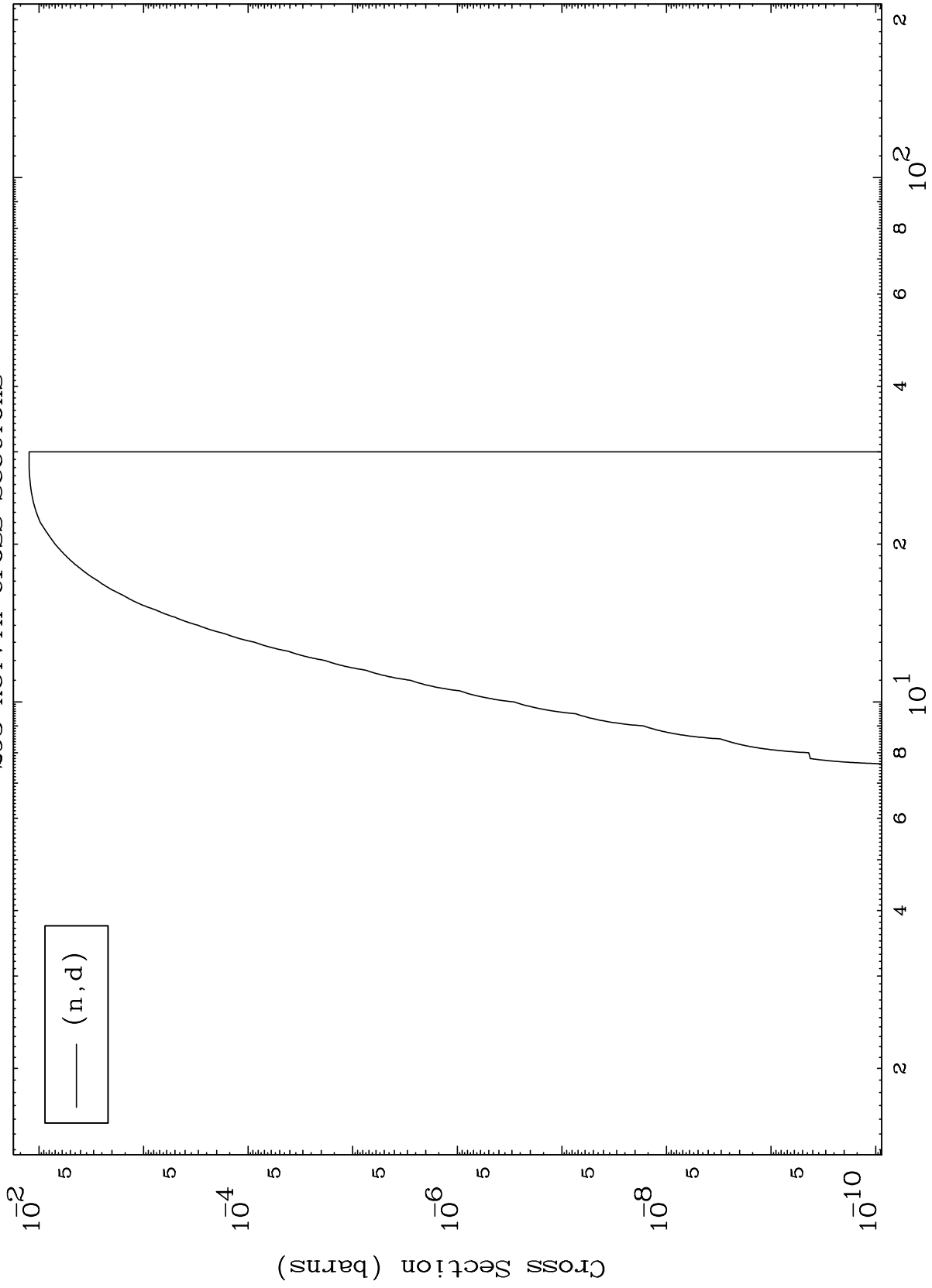




MAT 665

(n,d) Levels  
293 Kelvin Cross Sections

106-Sg-265



10

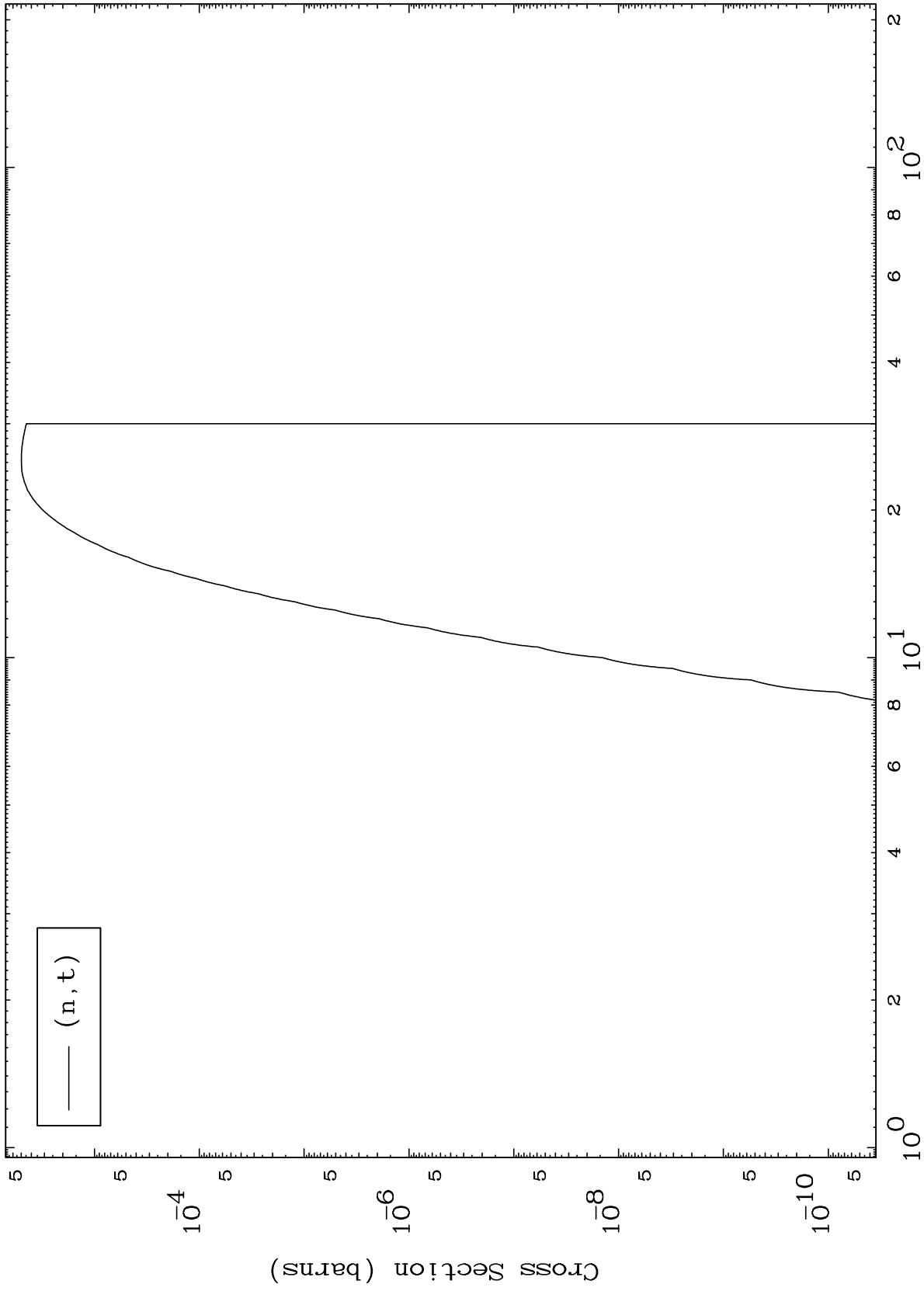
Incident Energy (MeV)

106-Sg-265

MAT 665

(n,t) Levels  
293 Kelvin Cross Sections

106-Sg-265



11

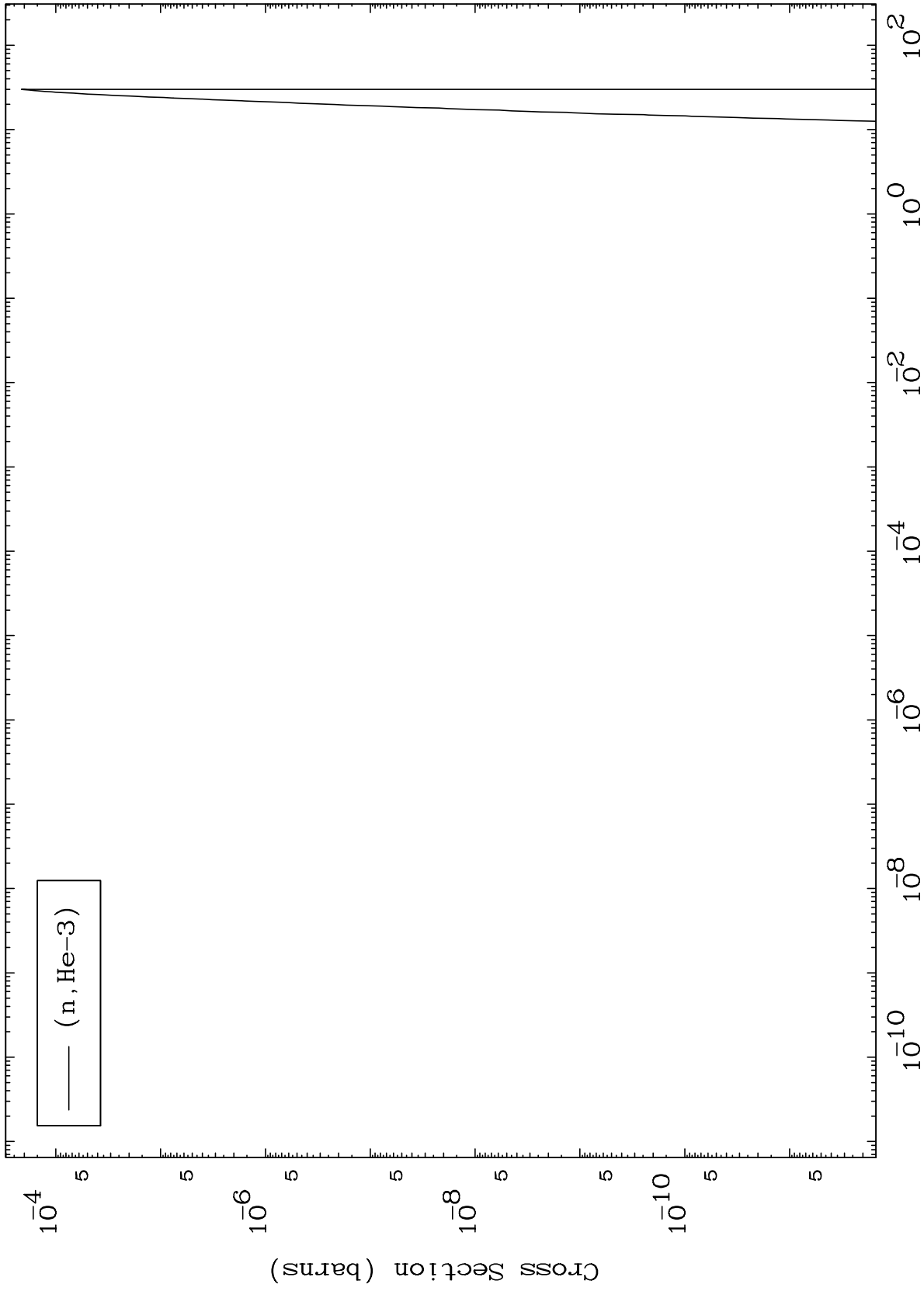
Incident Energy (MeV)

106-Sg-265

MAT 665

(n,He3) Levels  
293 Kelvin Cross Sections

106-Sg-265



12

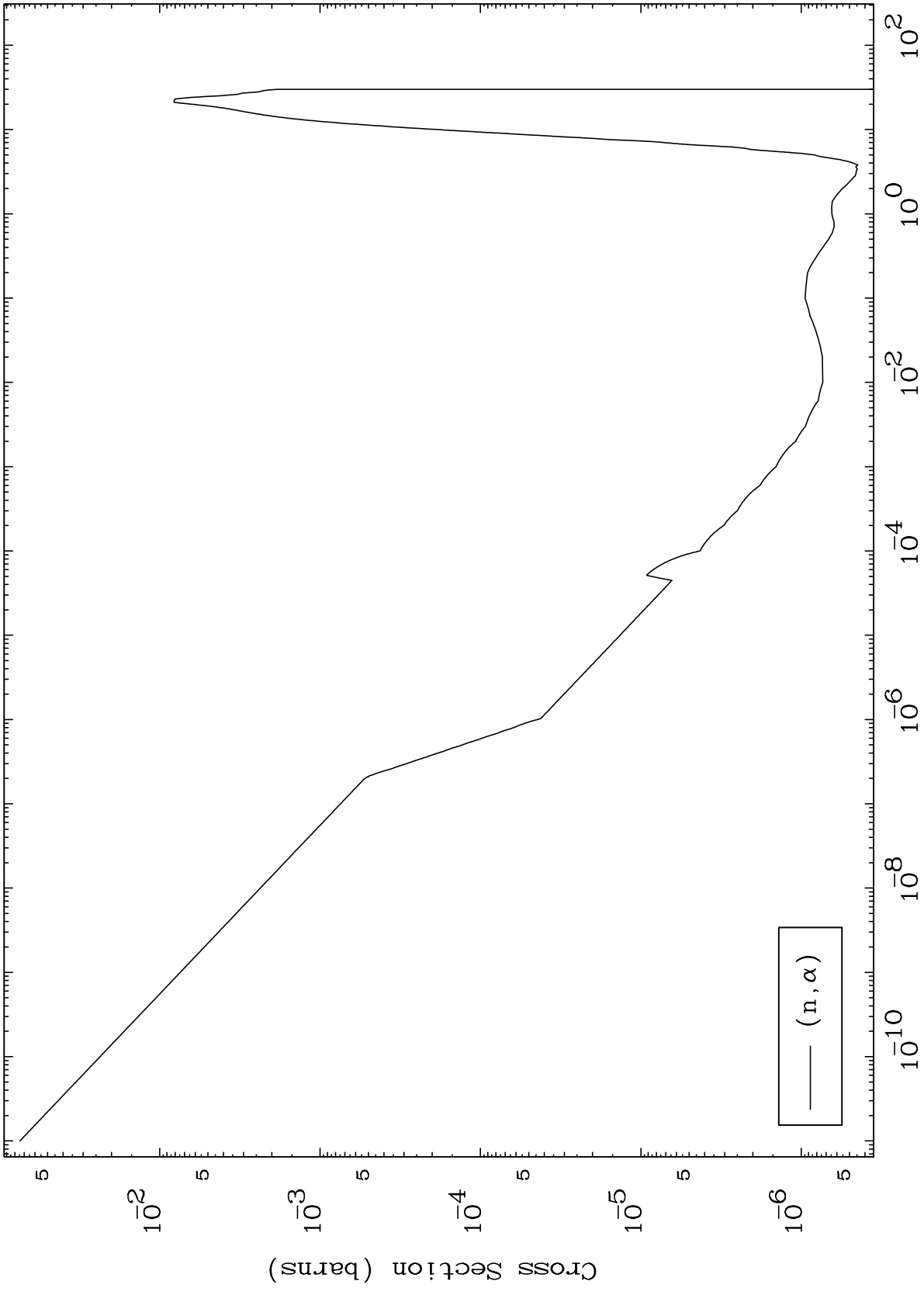
Incident Energy (MeV)

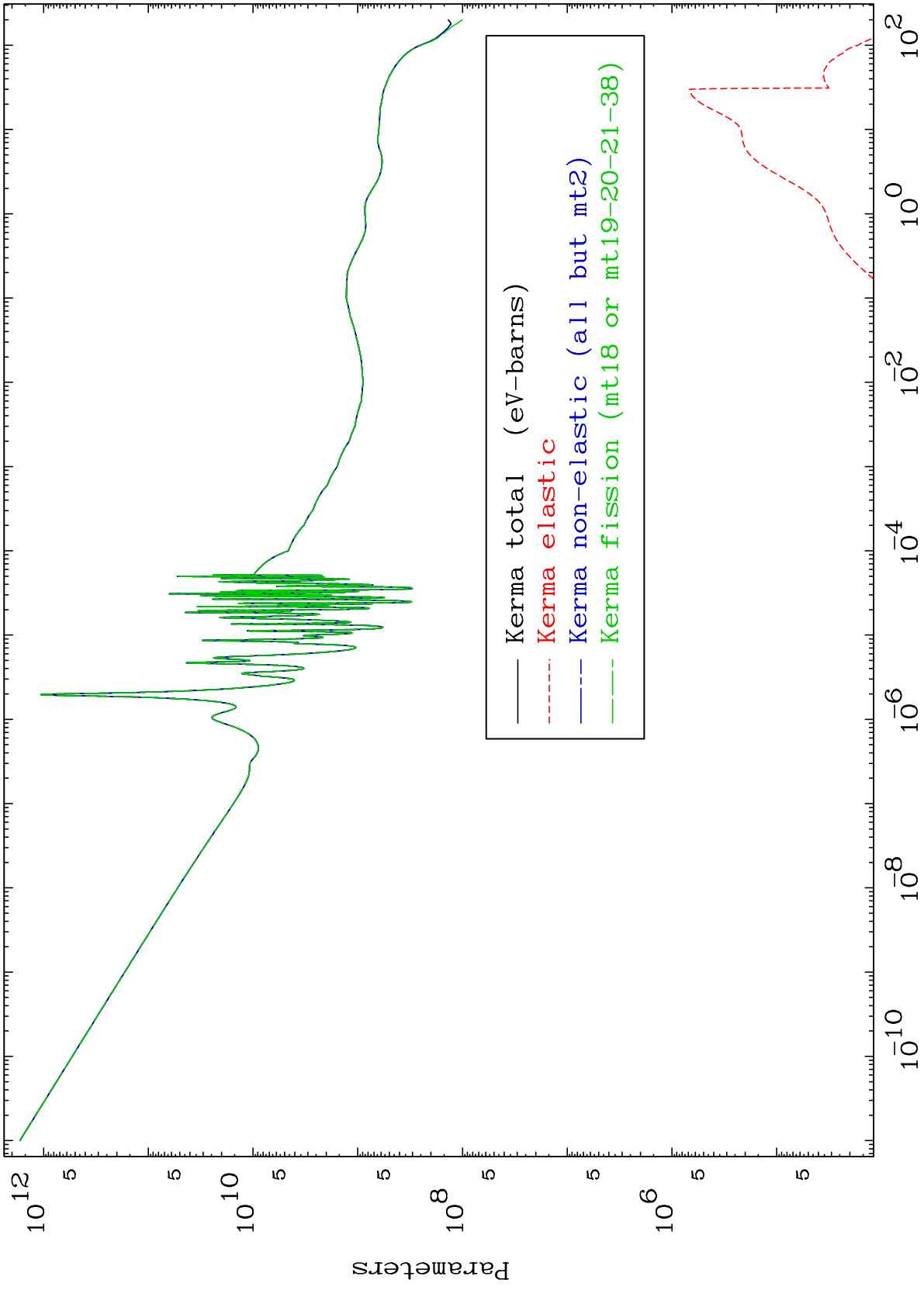
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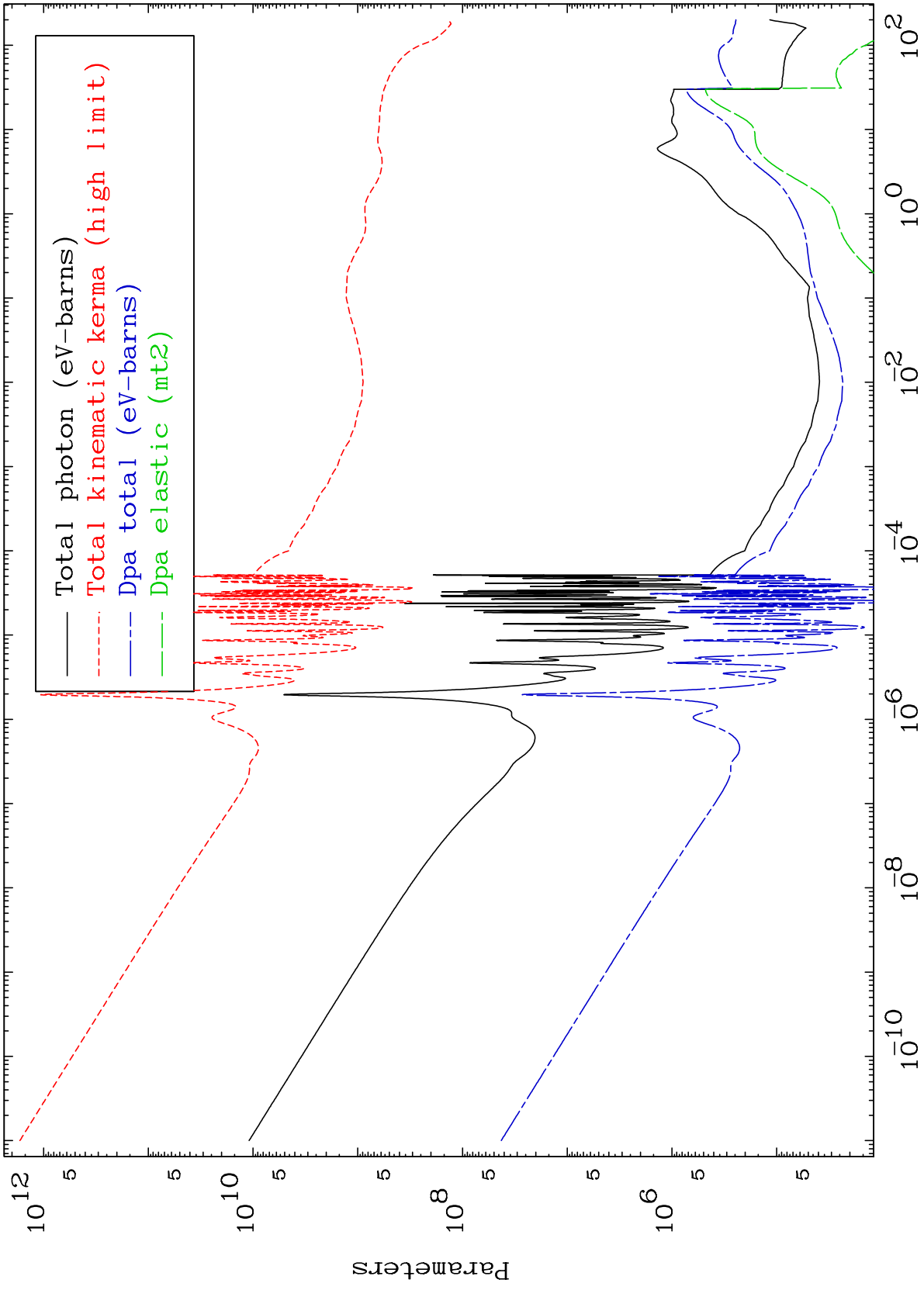
MAT 665

(n, $\alpha$ ) Levels  
293 Kelvin Cross Sections

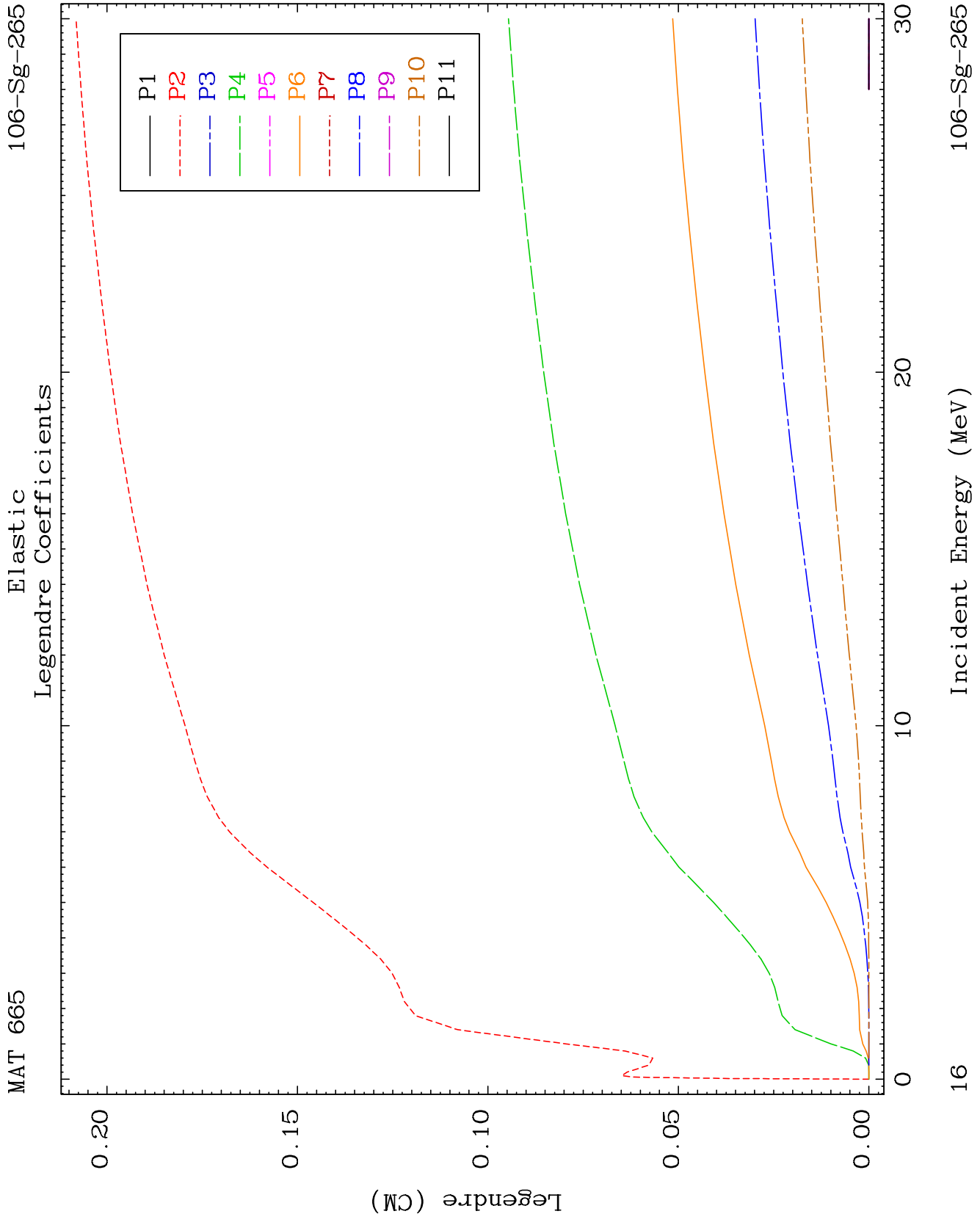
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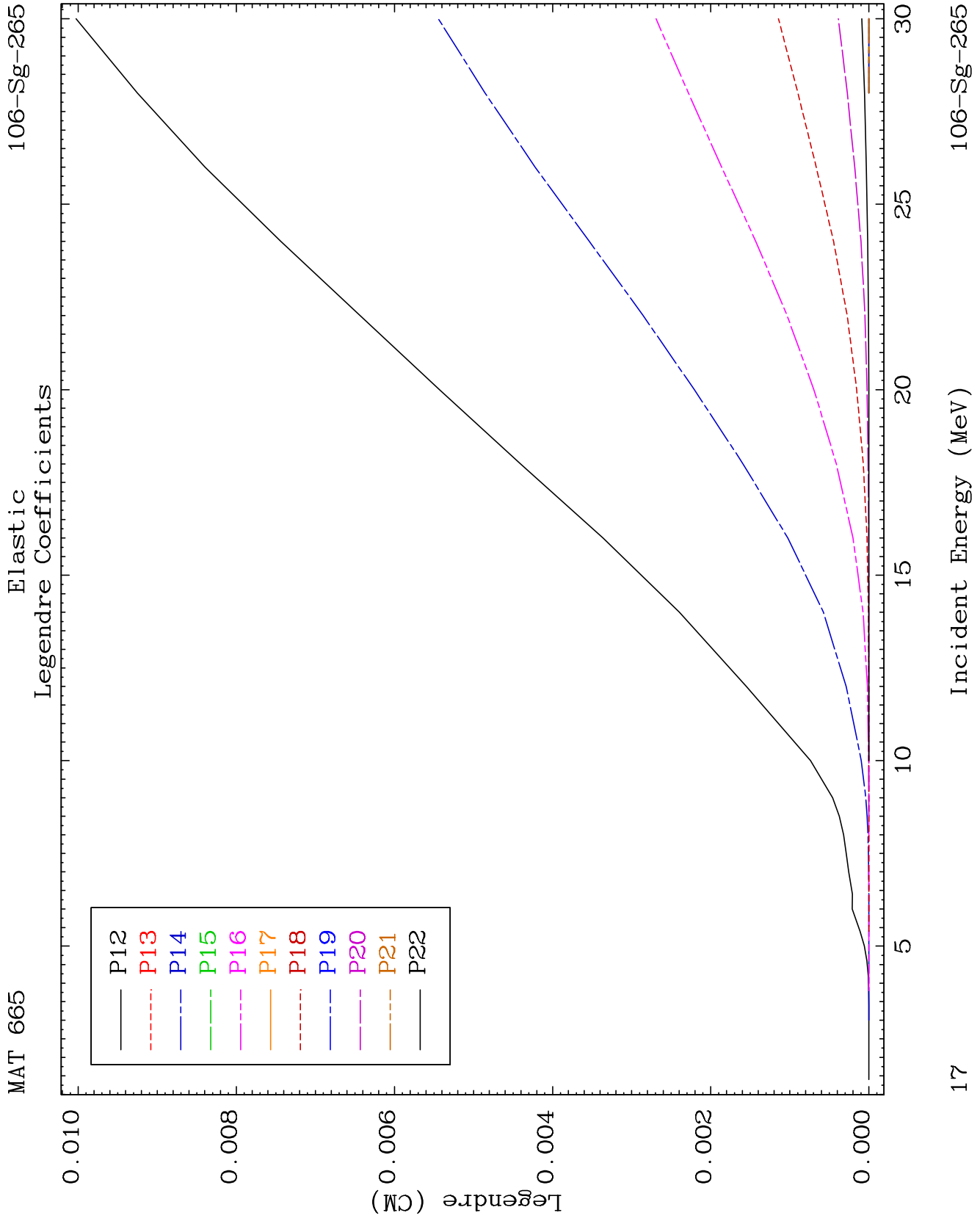


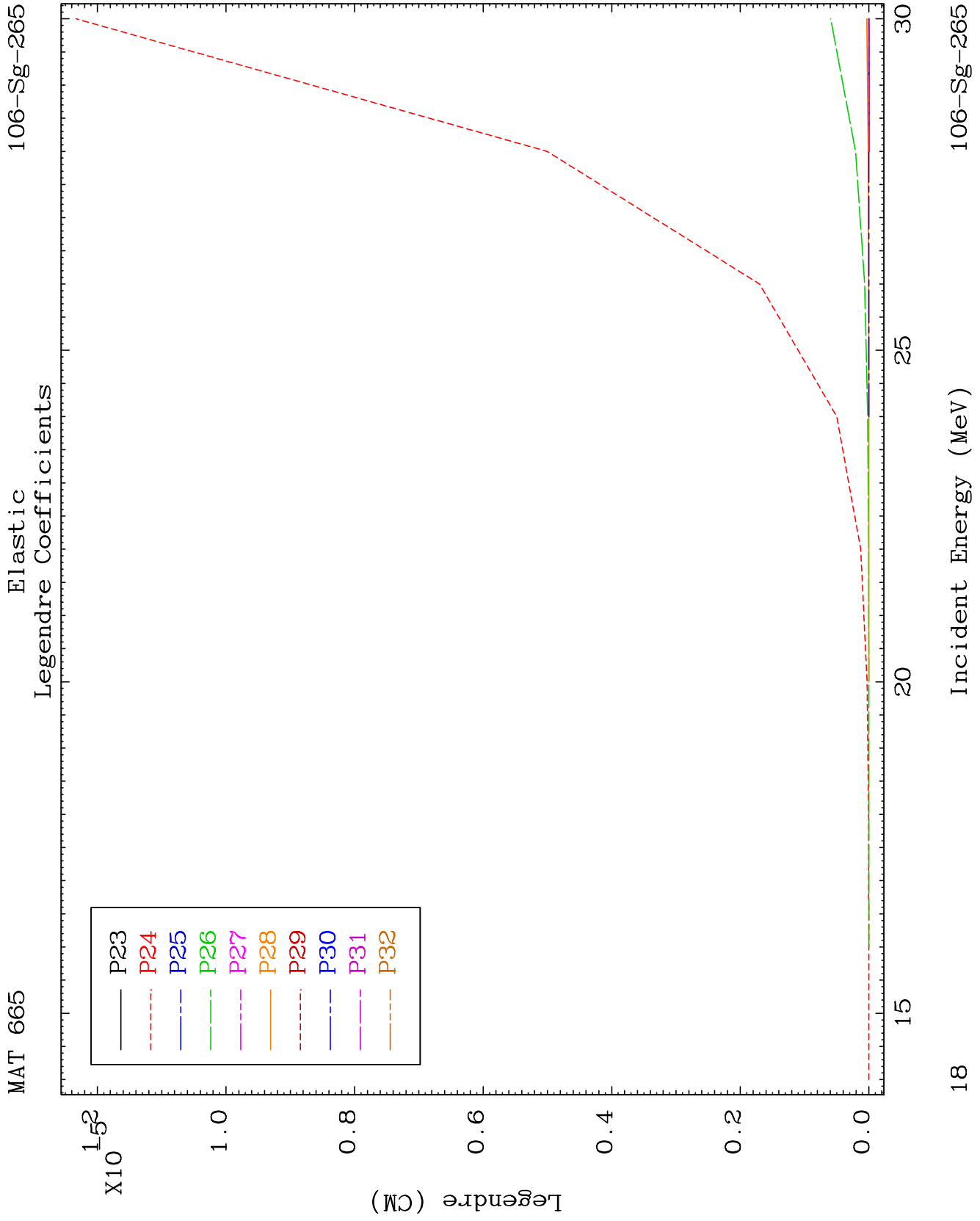








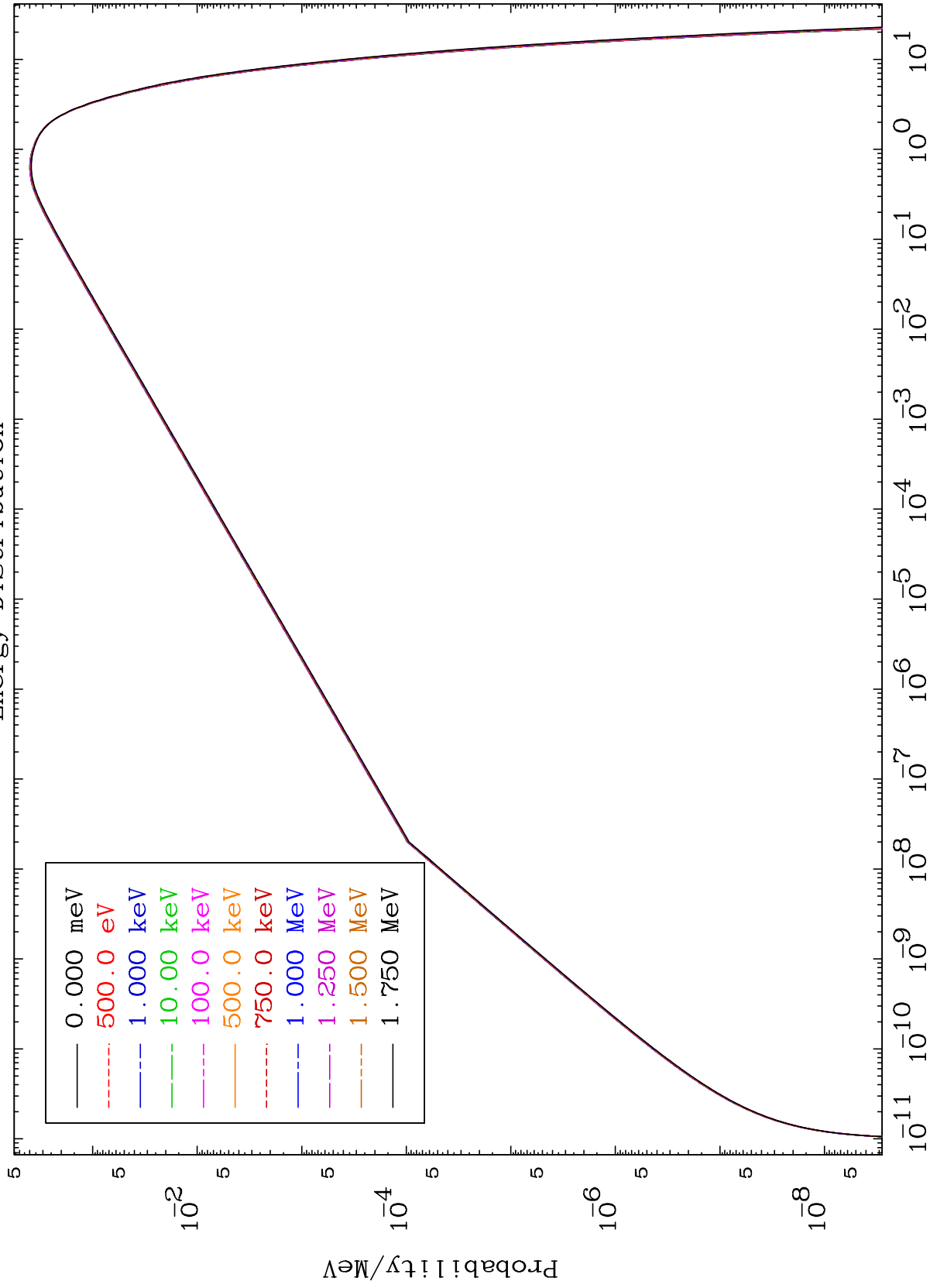




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### Fission Energy Distribution

106-Sg-265

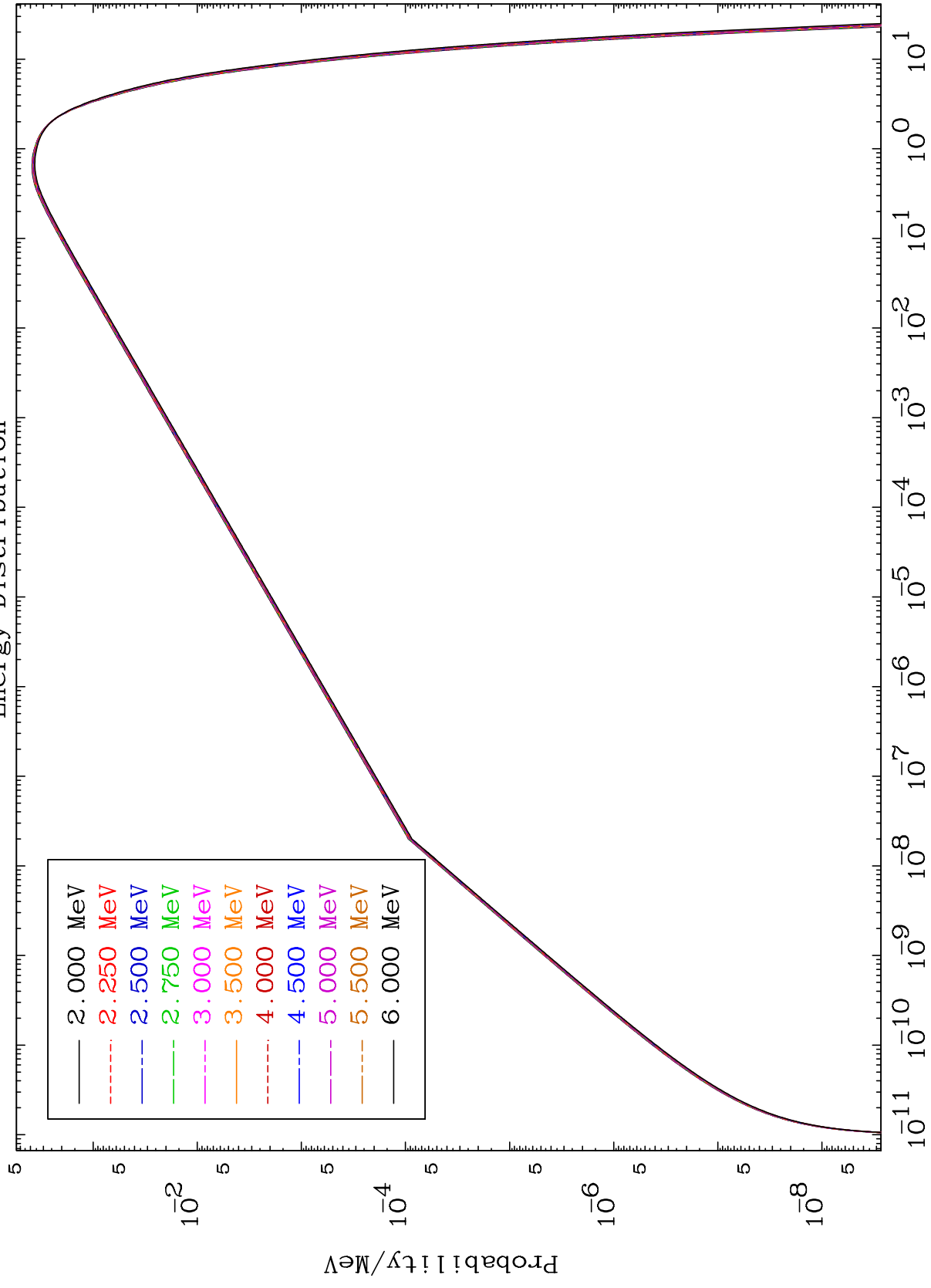


106-Sg-265

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# Fission Energy Distribution

106-Sg-265

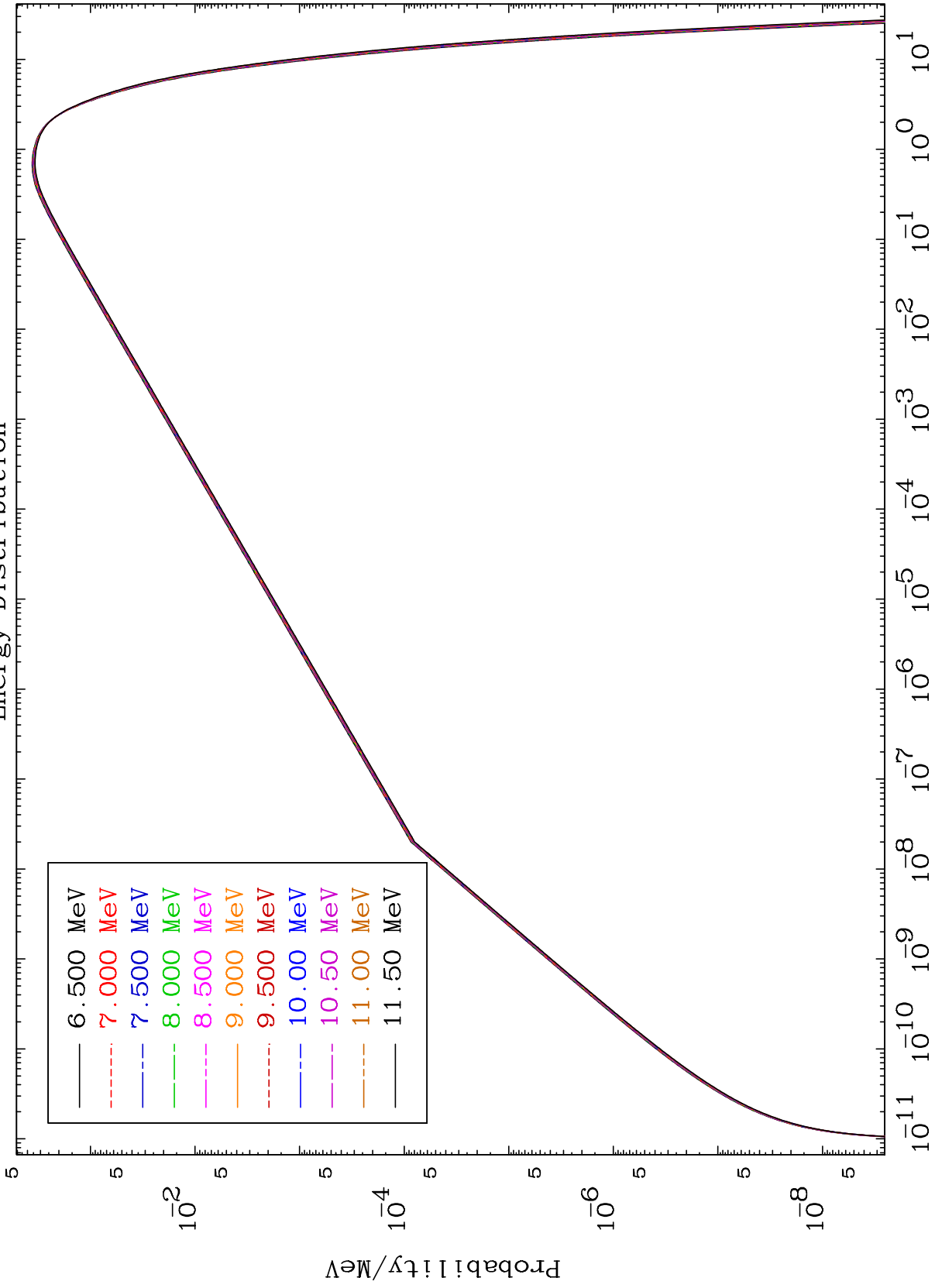


106-Sg-265

MAT 665

### Fission Energy Distribution

106-Sg-265

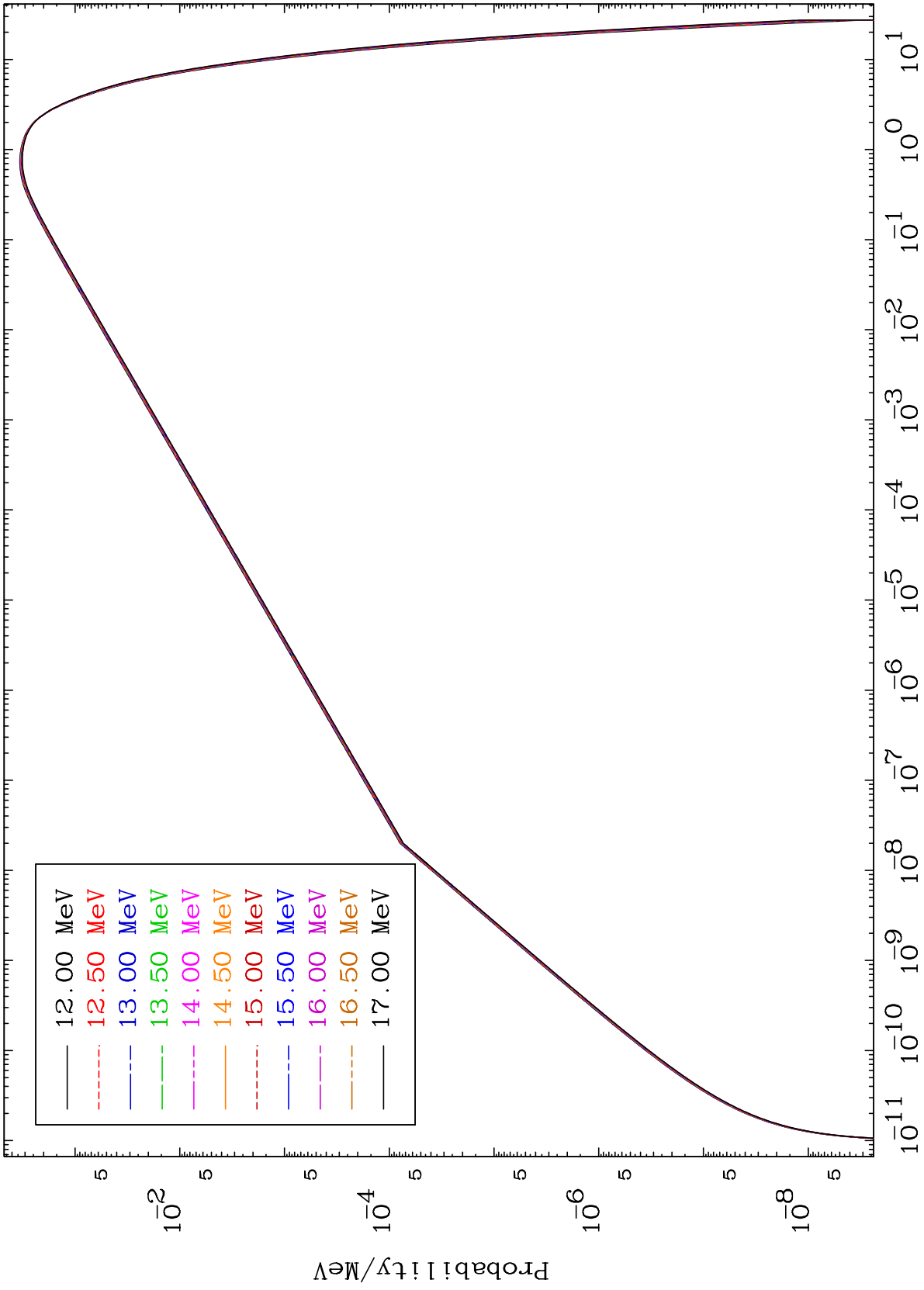


106-Sg-265

MAT 665

# Fission Energy Distribution

<sup>106</sup>Sg-265

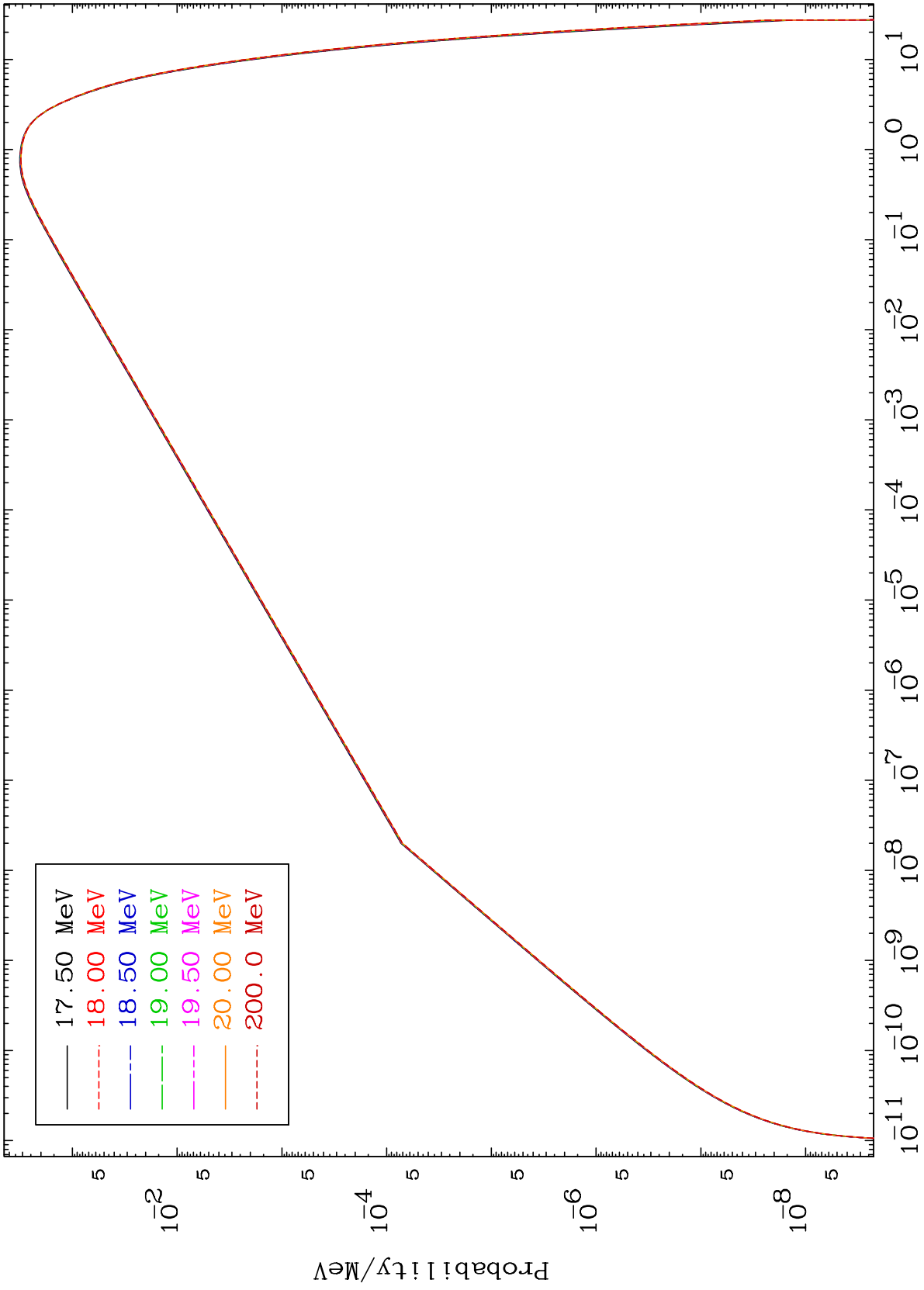


<sup>106</sup>Sg-265

MAT 665

# Fission Energy Distribution

106-Sg-265



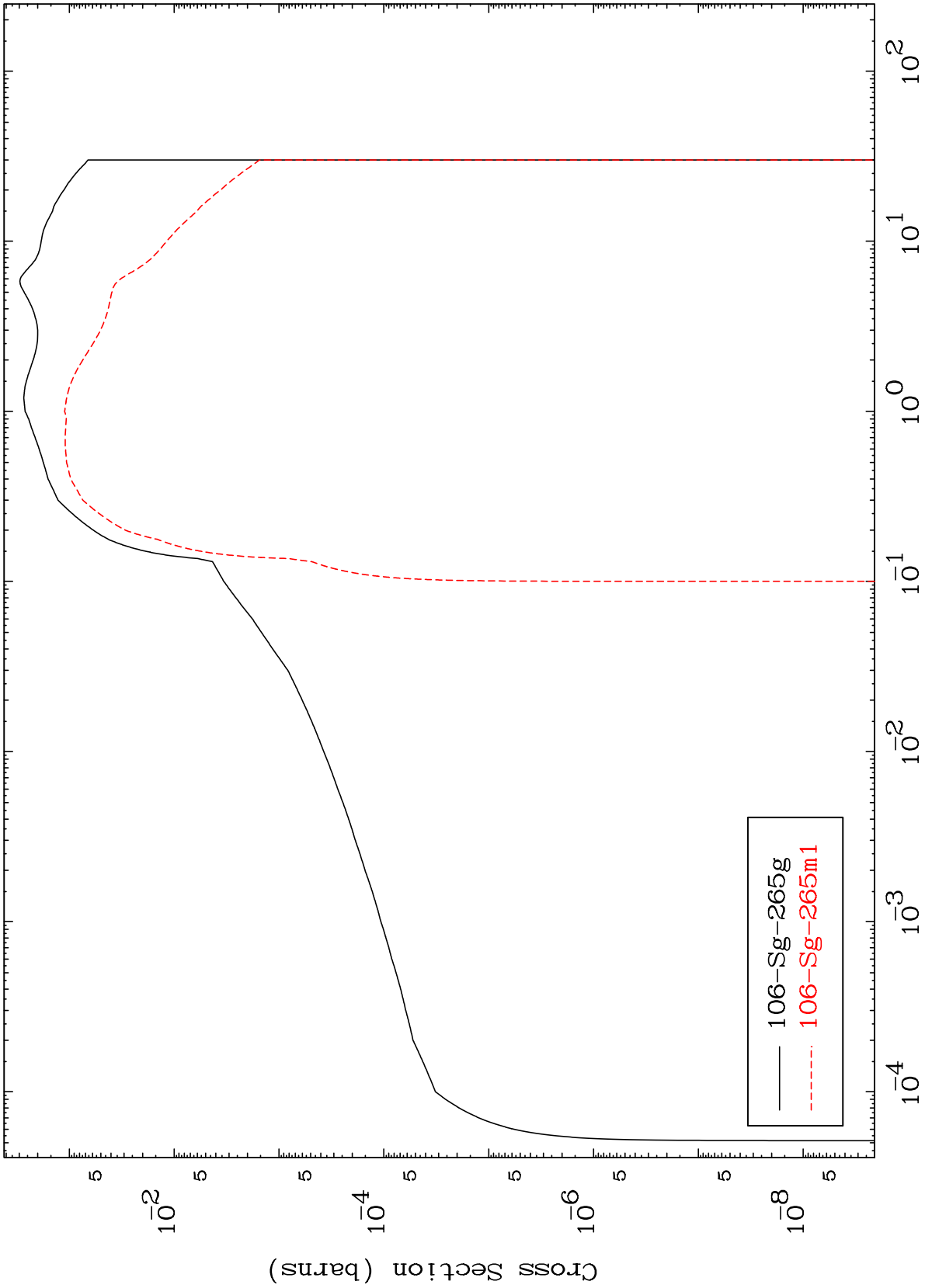
106-Sg-265



MAT 665

106-Sg-265

Inelastic  
Radionuclide Production Cross Section



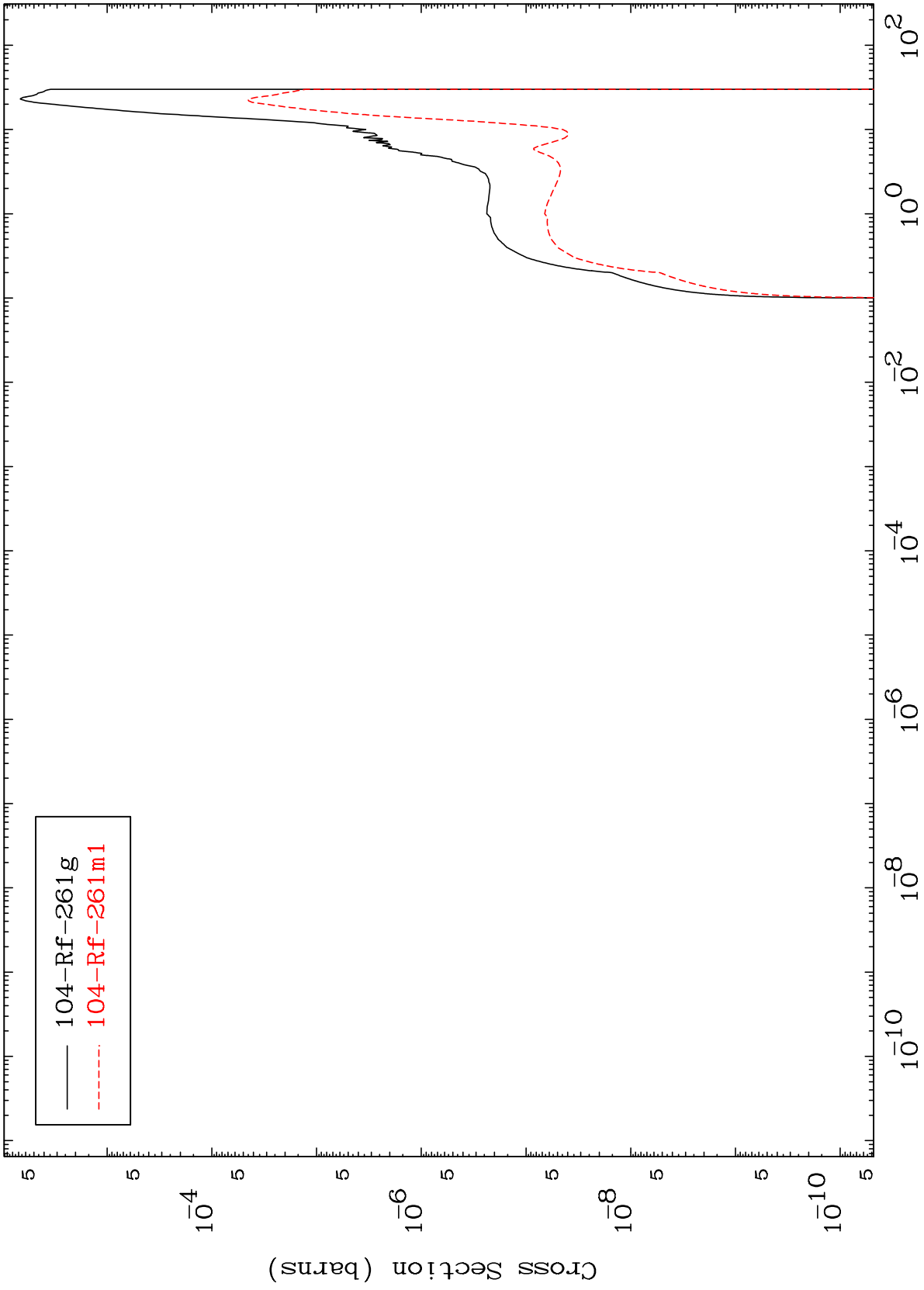
— 106-Sg-265g  
- - - 106-Sg-265m1

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(n,n')  $\alpha$

106-Sg-265

Radionuclide Production Cross Section



25

Incident Energy (MeV)

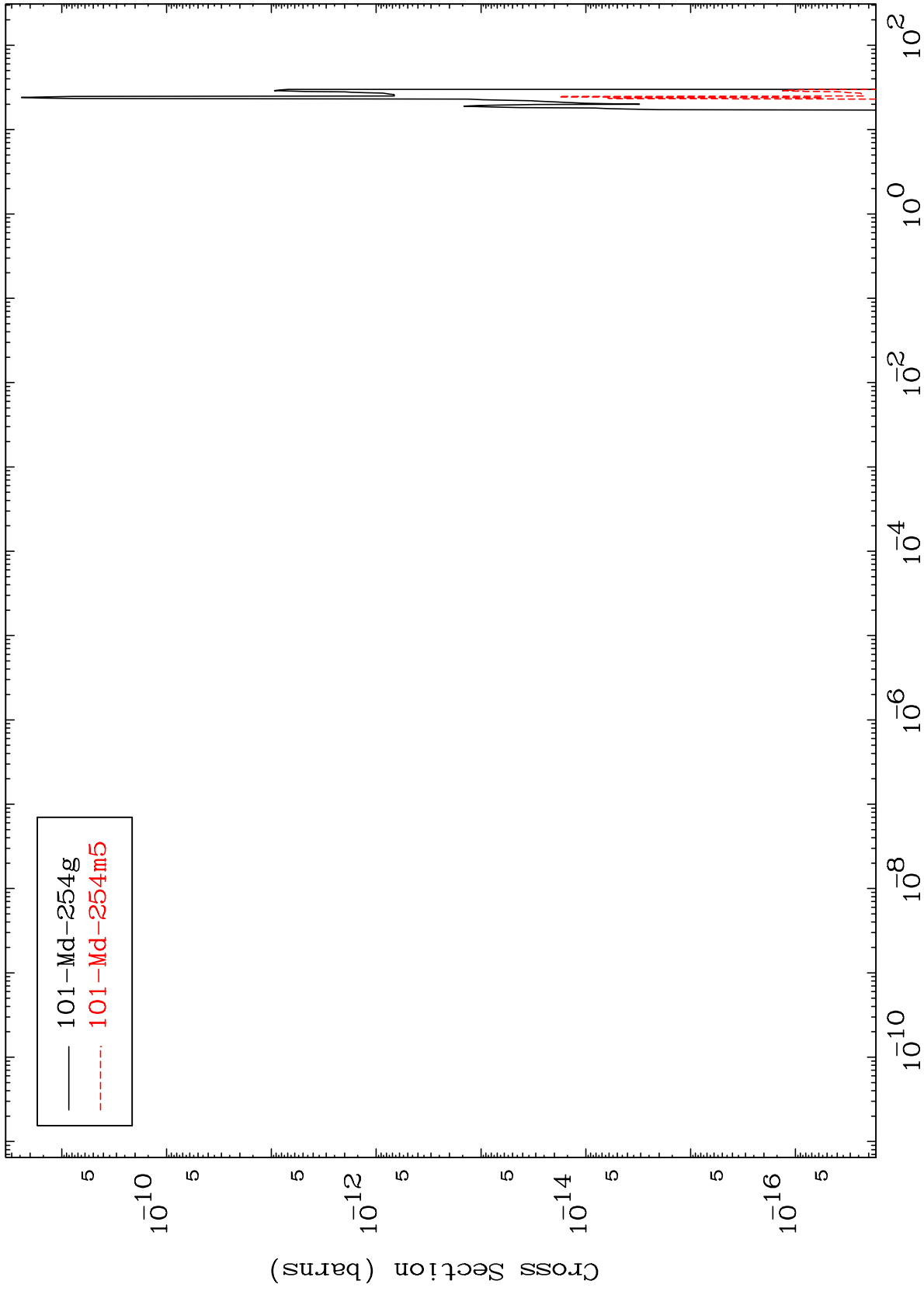
106-Sg-265

MAT 665

(n, n') t, 2 $\alpha$

106-Sg-265

Radionuclide Production Cross Section



26

Incident Energy (MeV)

106-Sg-265