

Program EVALPLOT  
(Version 2018-1)

by

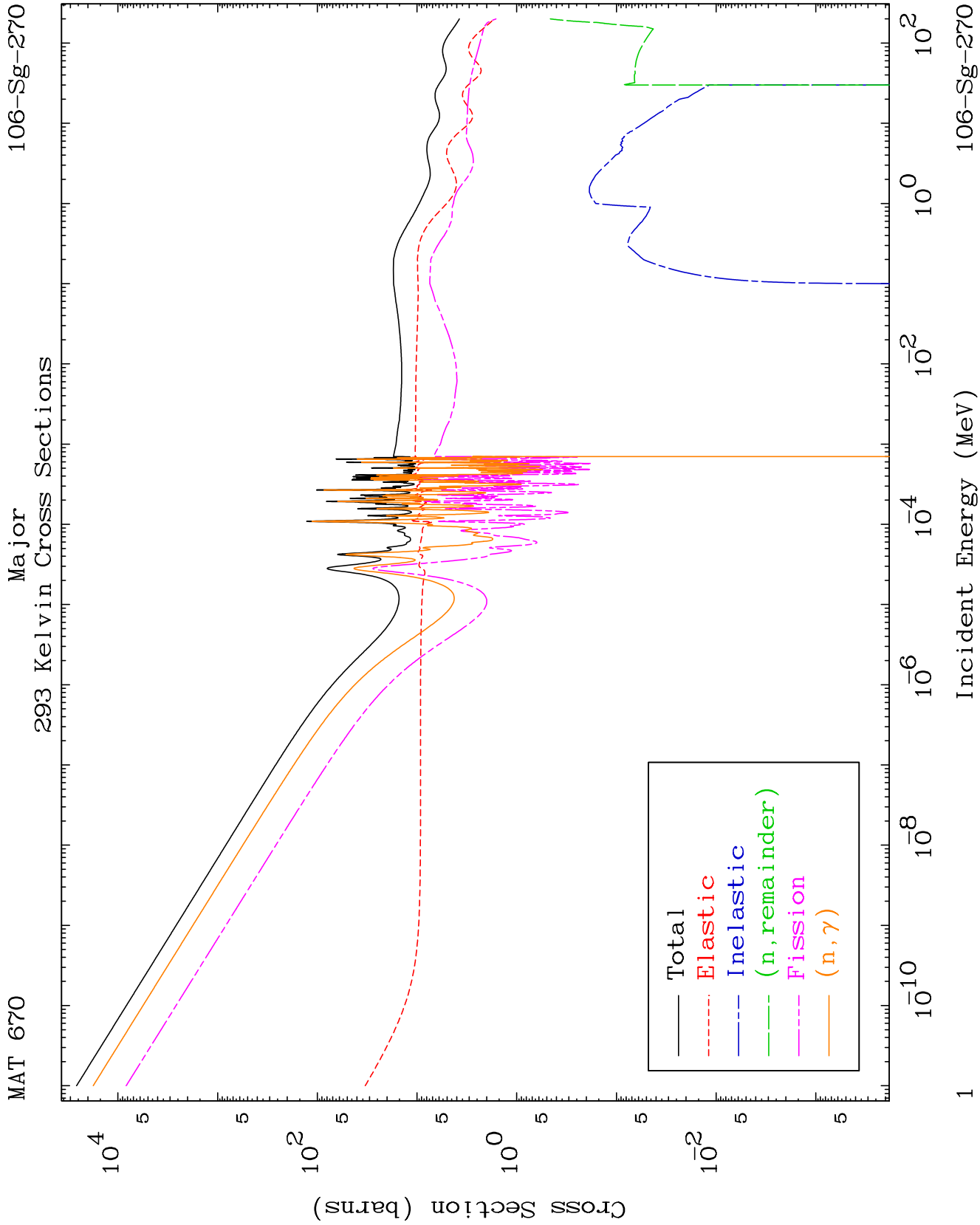
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Livermore, CA 94550  
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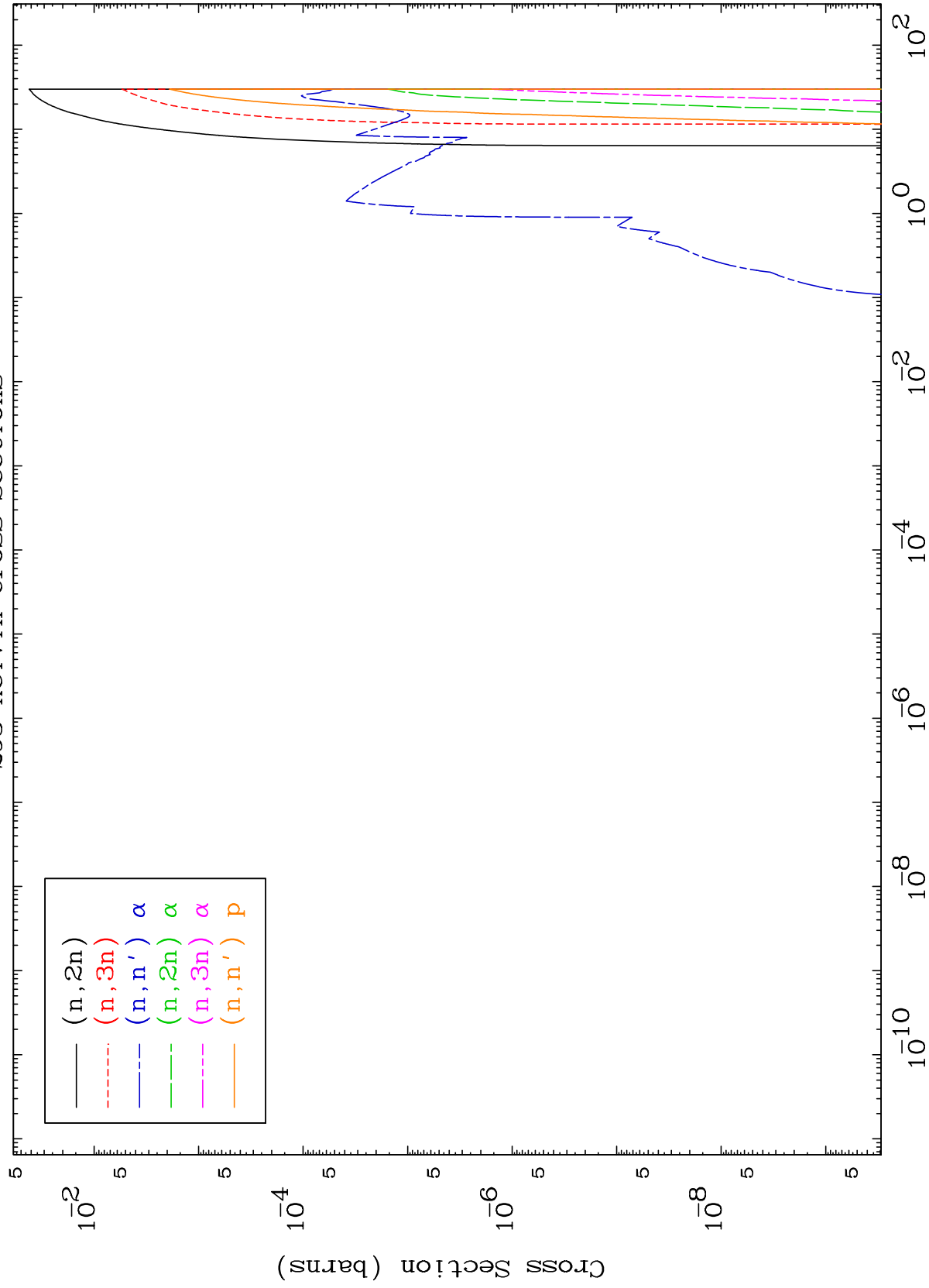
Press Mouse Button to Start



MAT 670

Neutron Production  
293 Kelvin Cross Sections

106-Sg-270



2

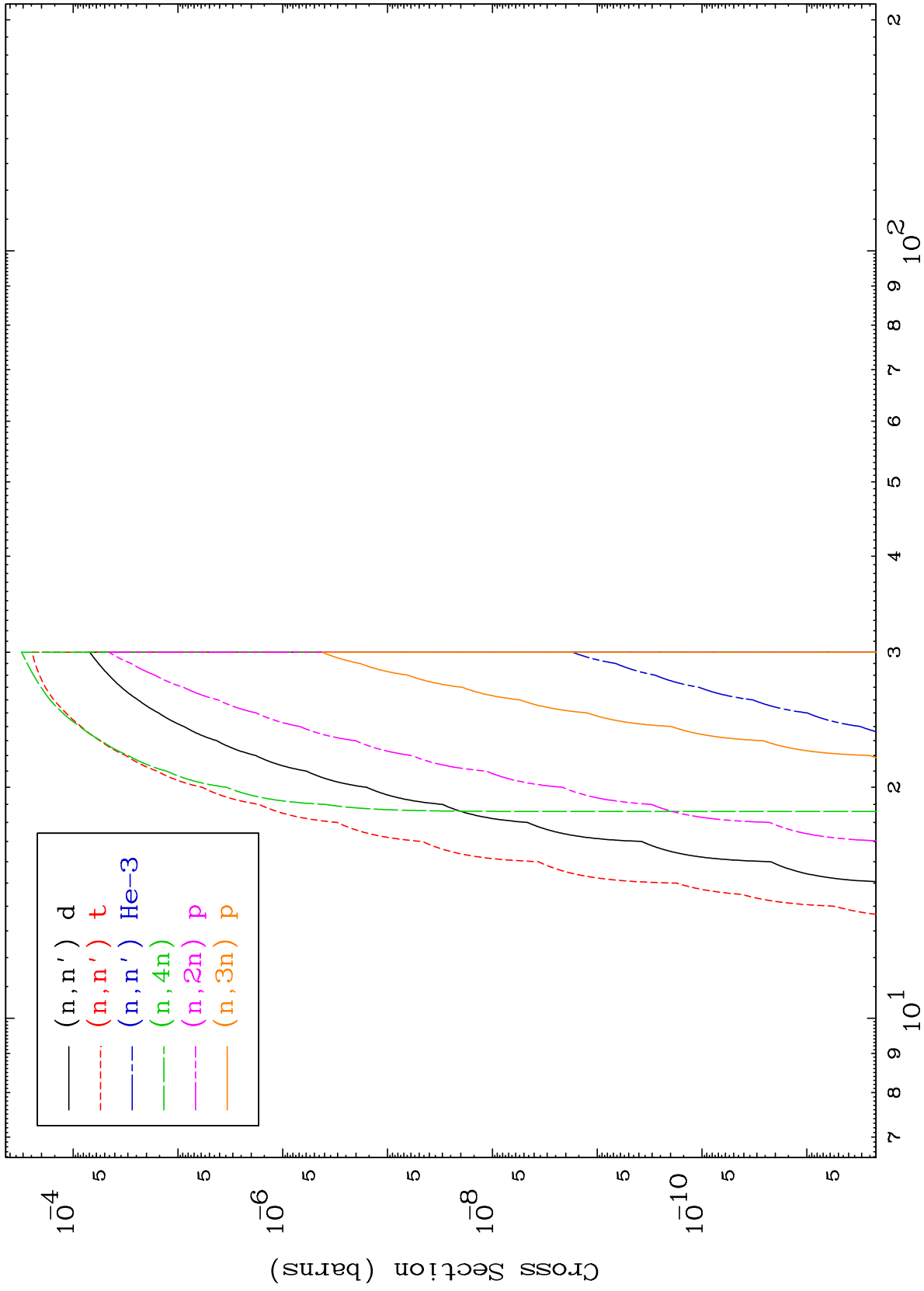
Incident Energy (MeV)

106-Sg-270

MAT 670

Neutron Production  
293 Kelvin Cross Sections

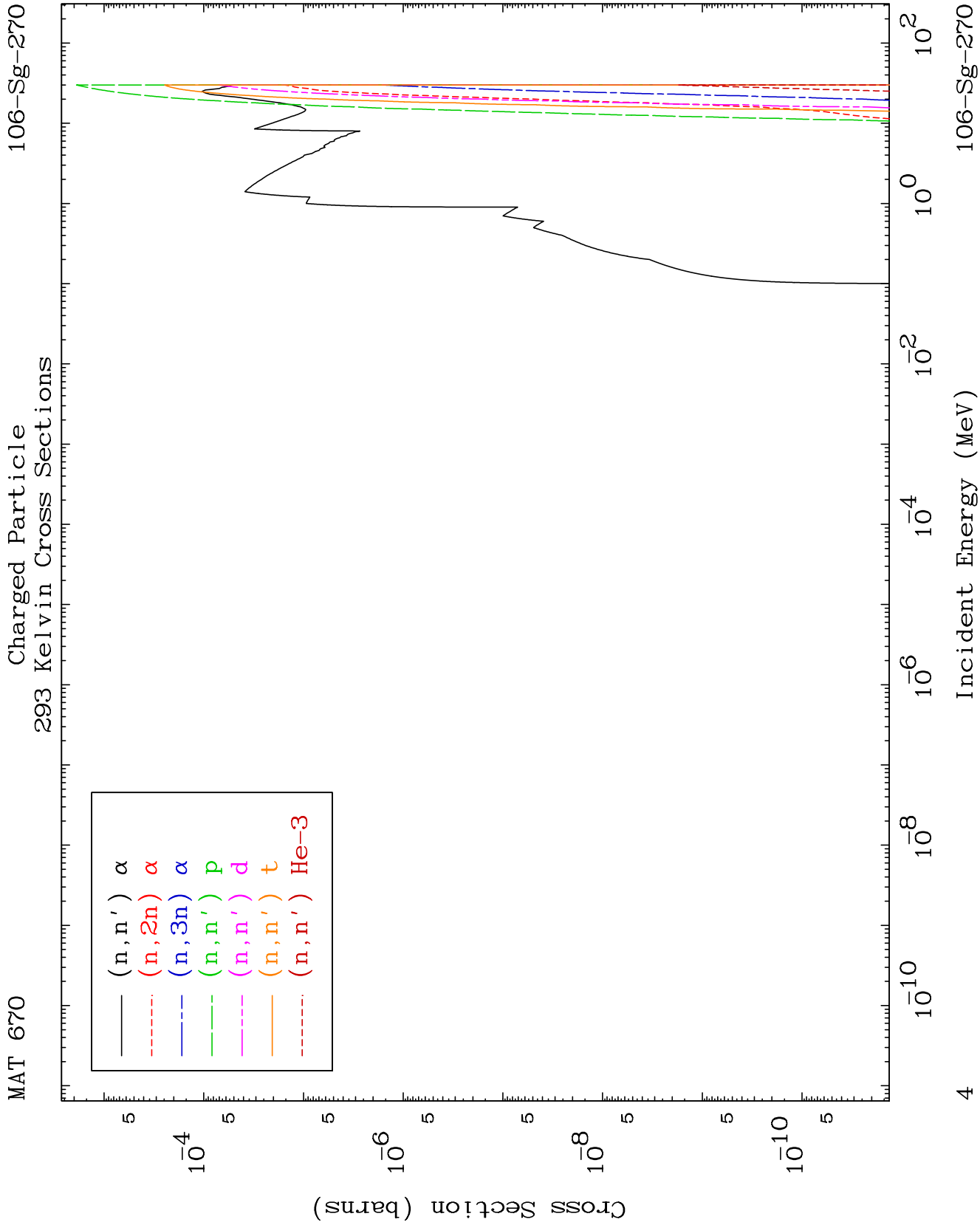
106-Sg-270



3

Incident Energy (MeV)

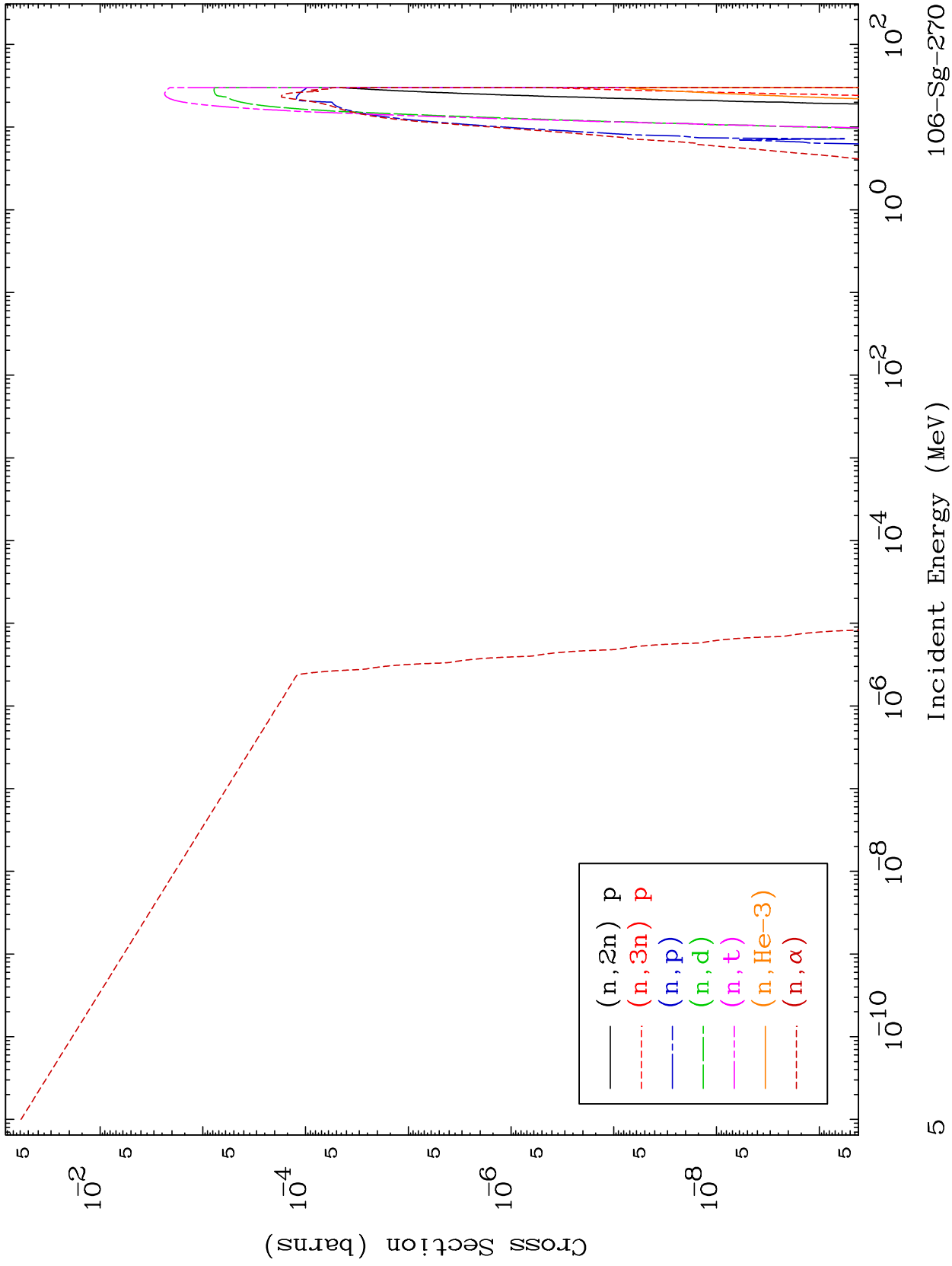
106-Sg-270



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Charged Particle  
293 Kelvin Cross Sections

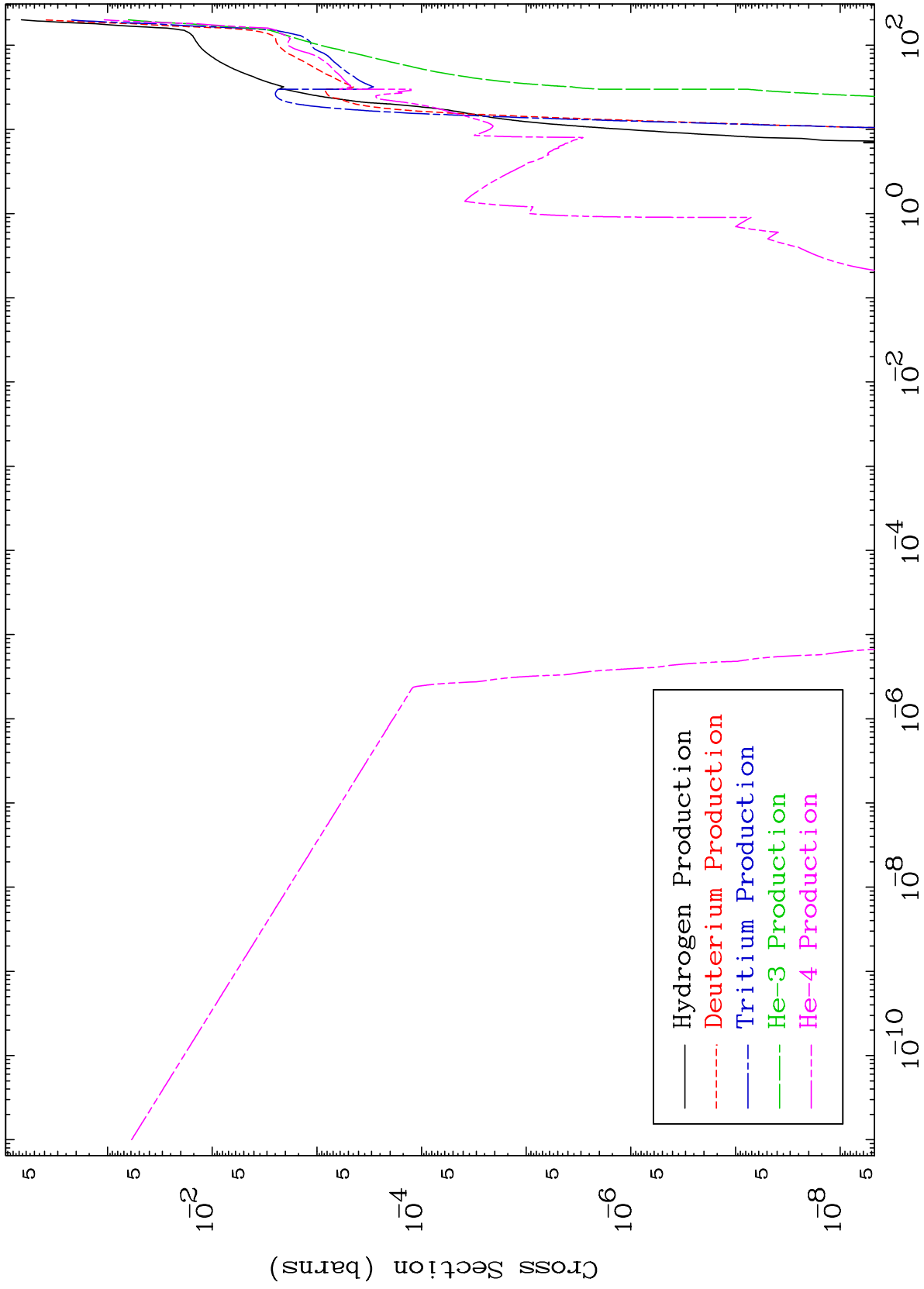
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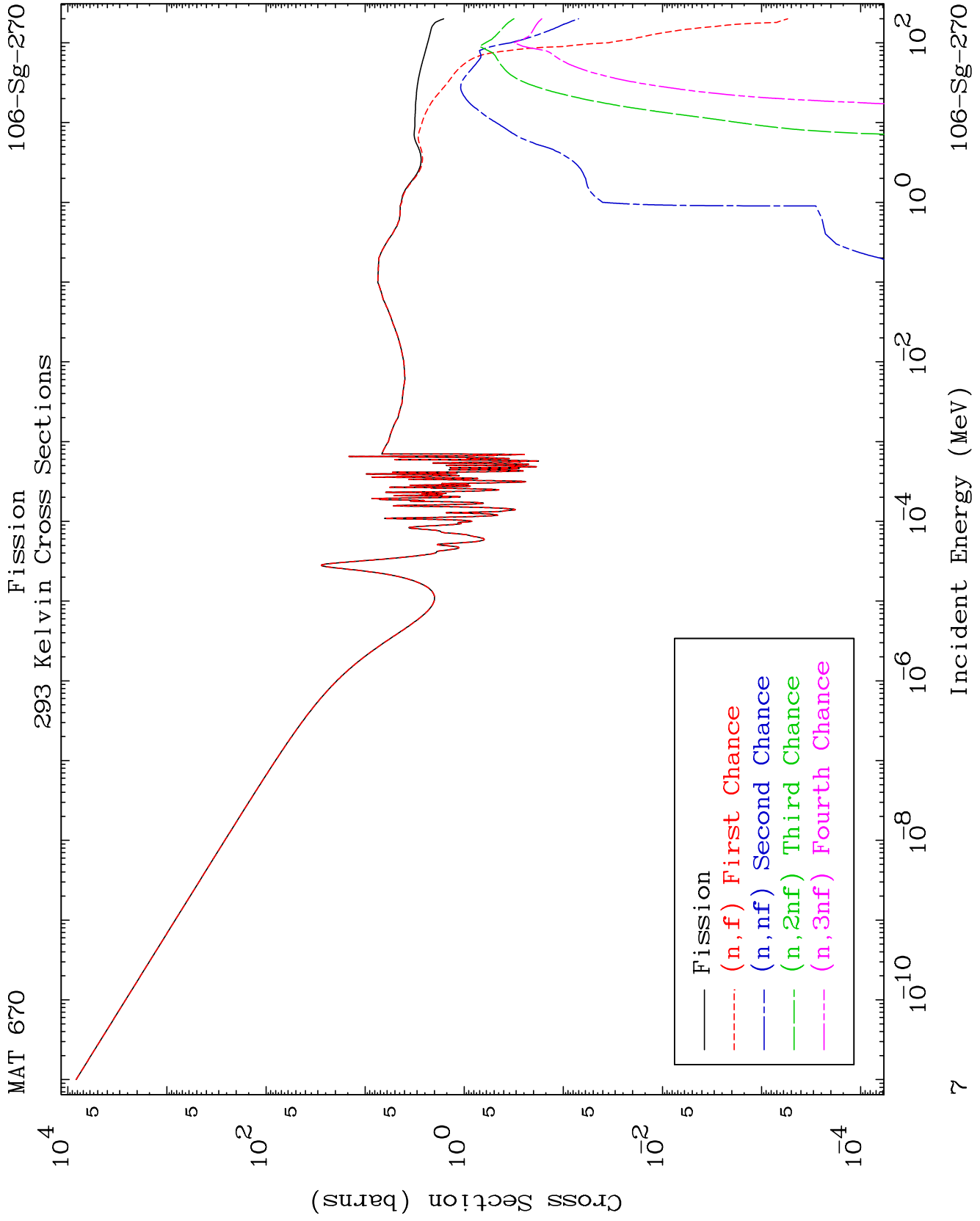


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Particle Production  
293 Kelvin Cross Sections

106-Sg-270



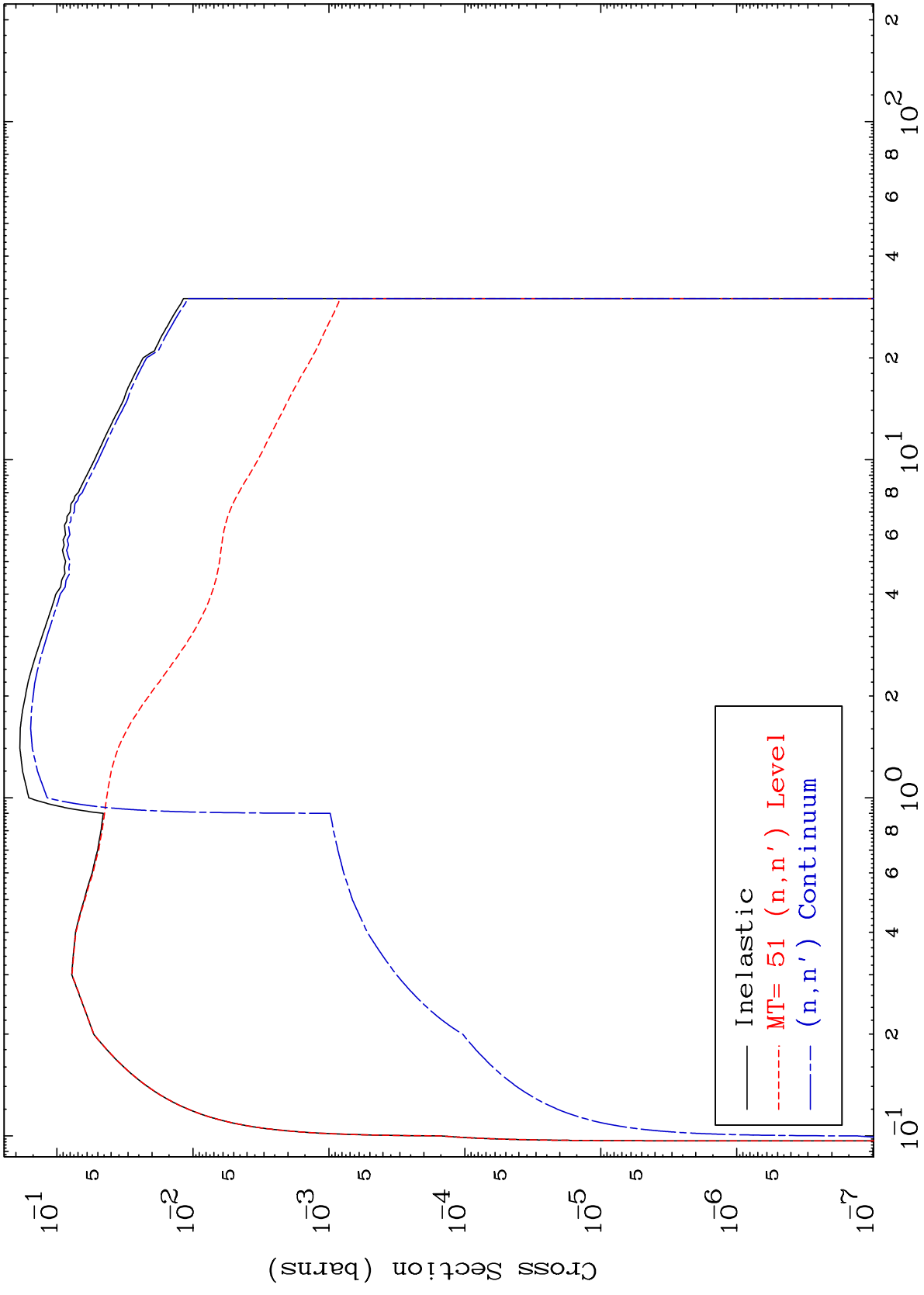




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106-Sg-270

(n,n') Level  
293 Kelvin Cross Sections



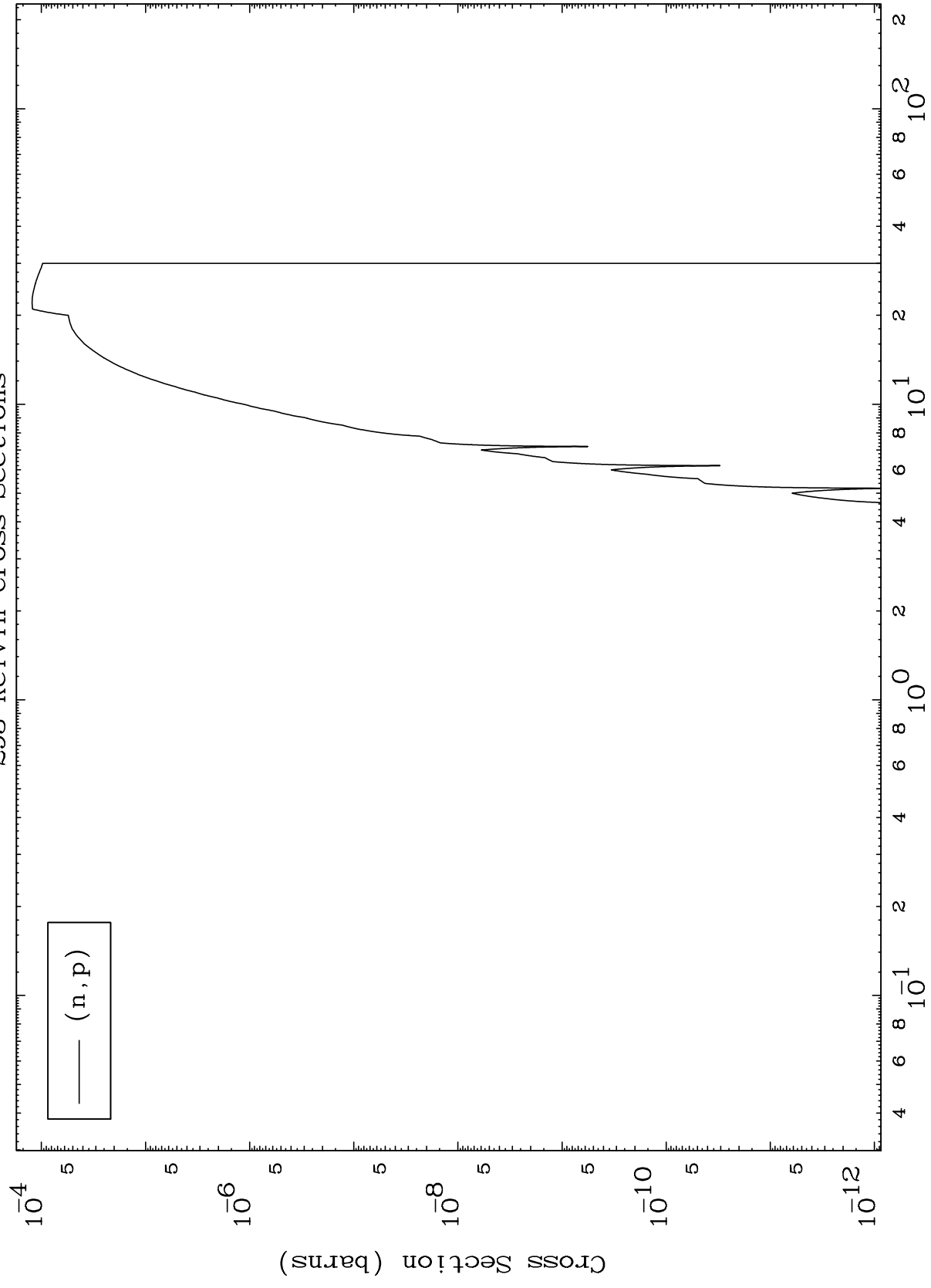
8

106-Sg-270

MAT 670

(n,p) Levels  
293 Kelvin Cross Sections

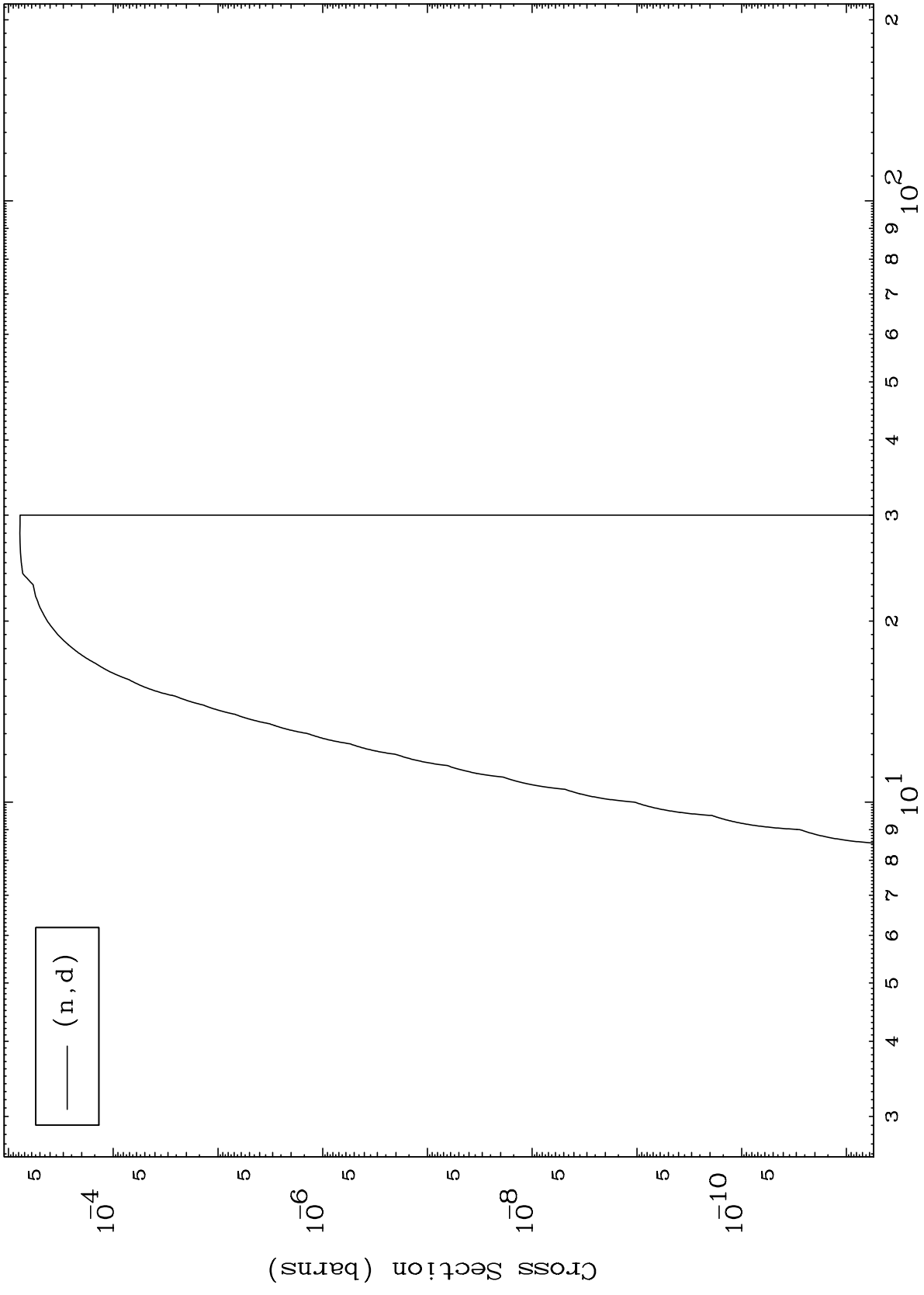
106-Sg-270



MAT 670

(n,d) Levels  
293 Kelvin Cross Sections

106-Sg-270



10

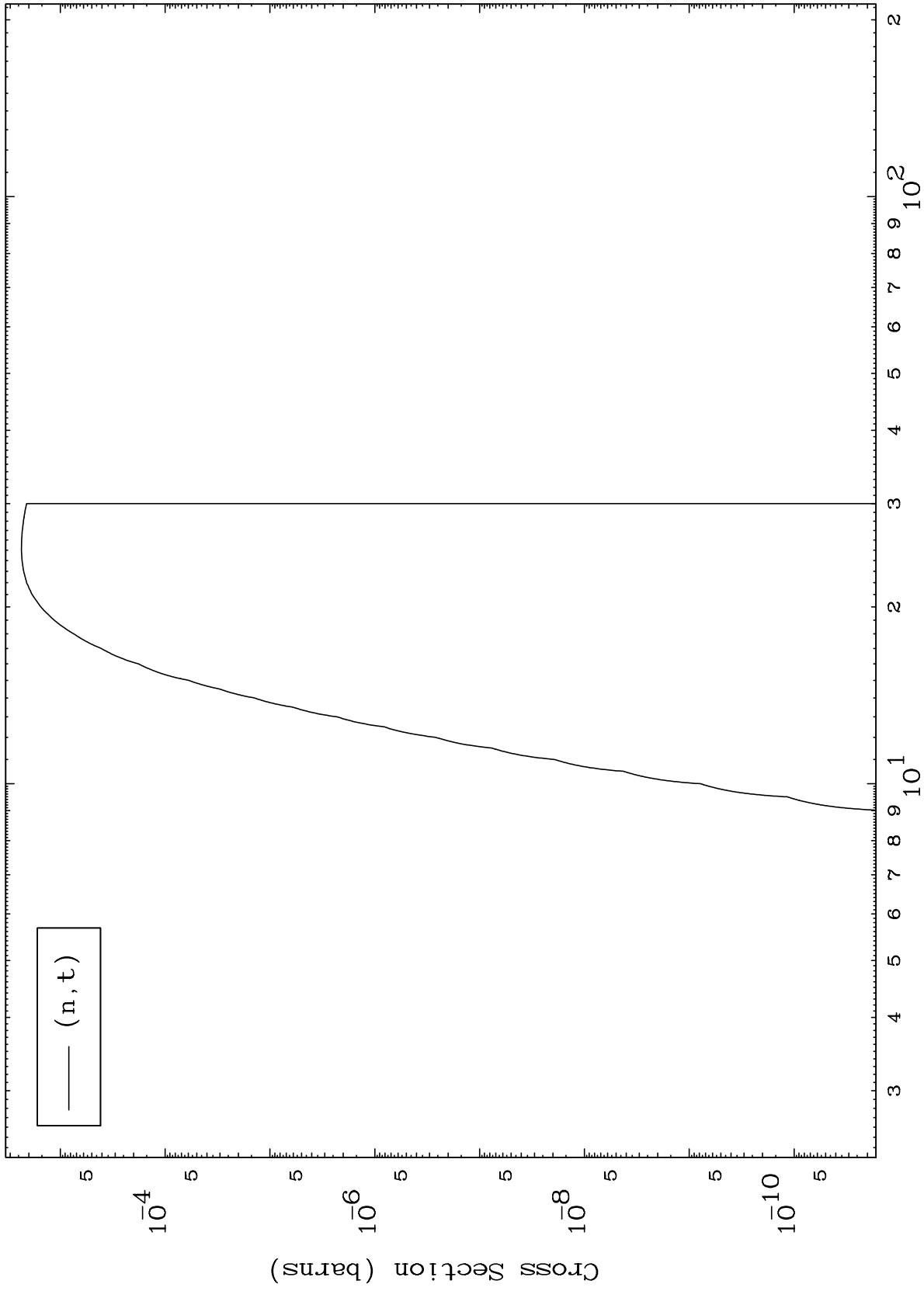
Incident Energy (MeV)

106-Sg-270

MAT 670

(n,t) Levels  
293 Kelvin Cross Sections

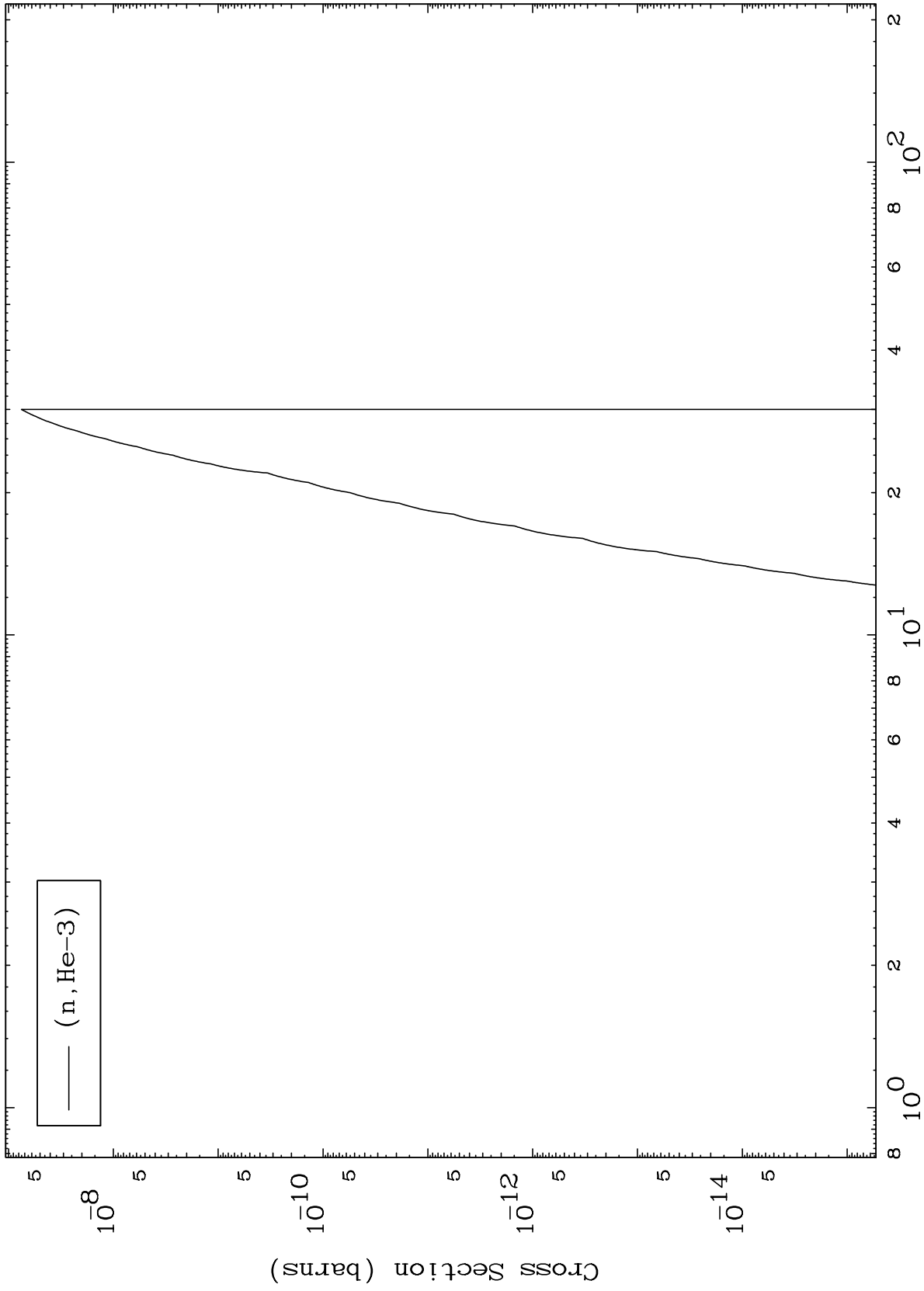
106-Sg-270



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(n,He3) Levels  
293 Kelvin Cross Sections

106-Sg-270



12

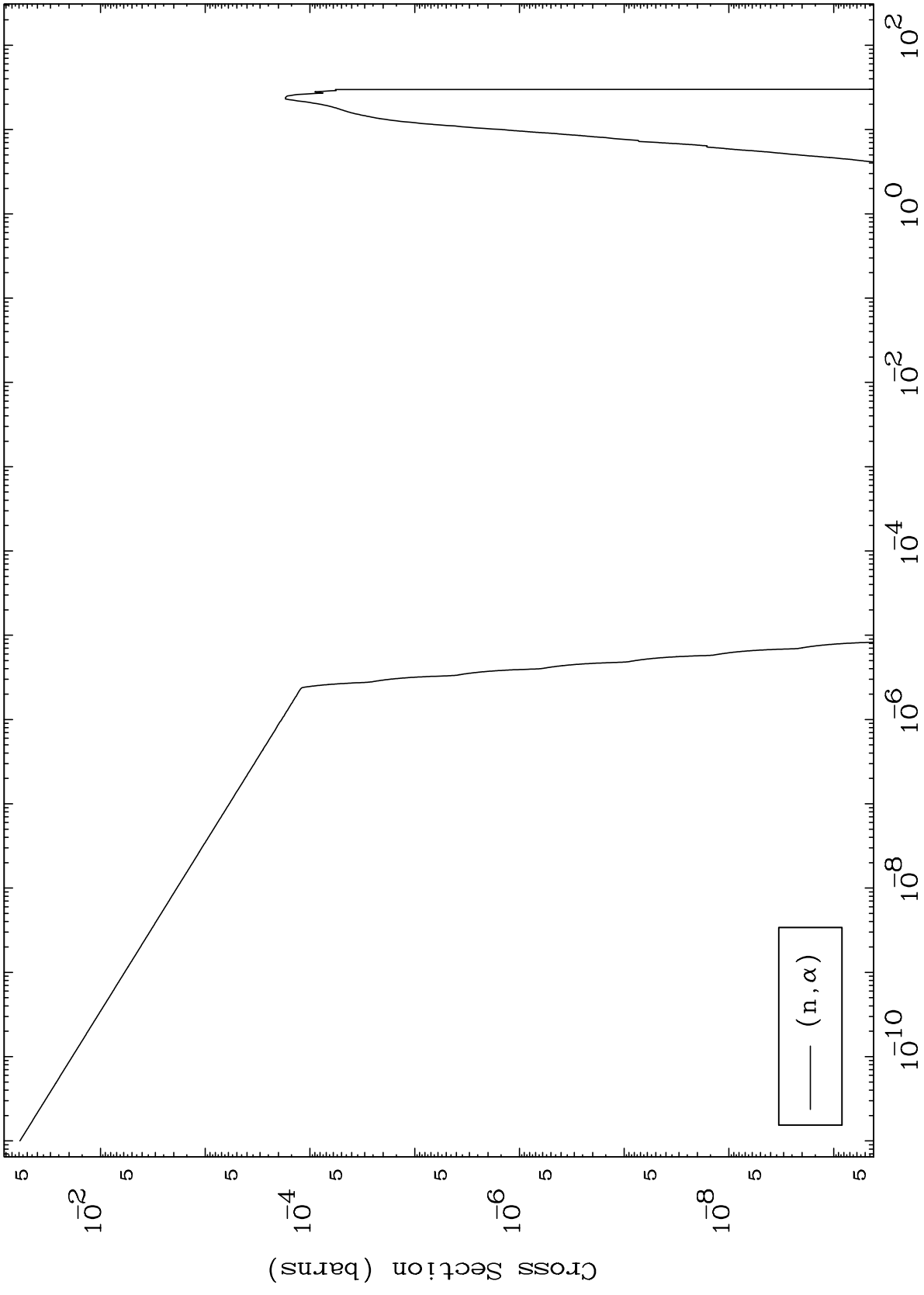
Incident Energy (MeV)

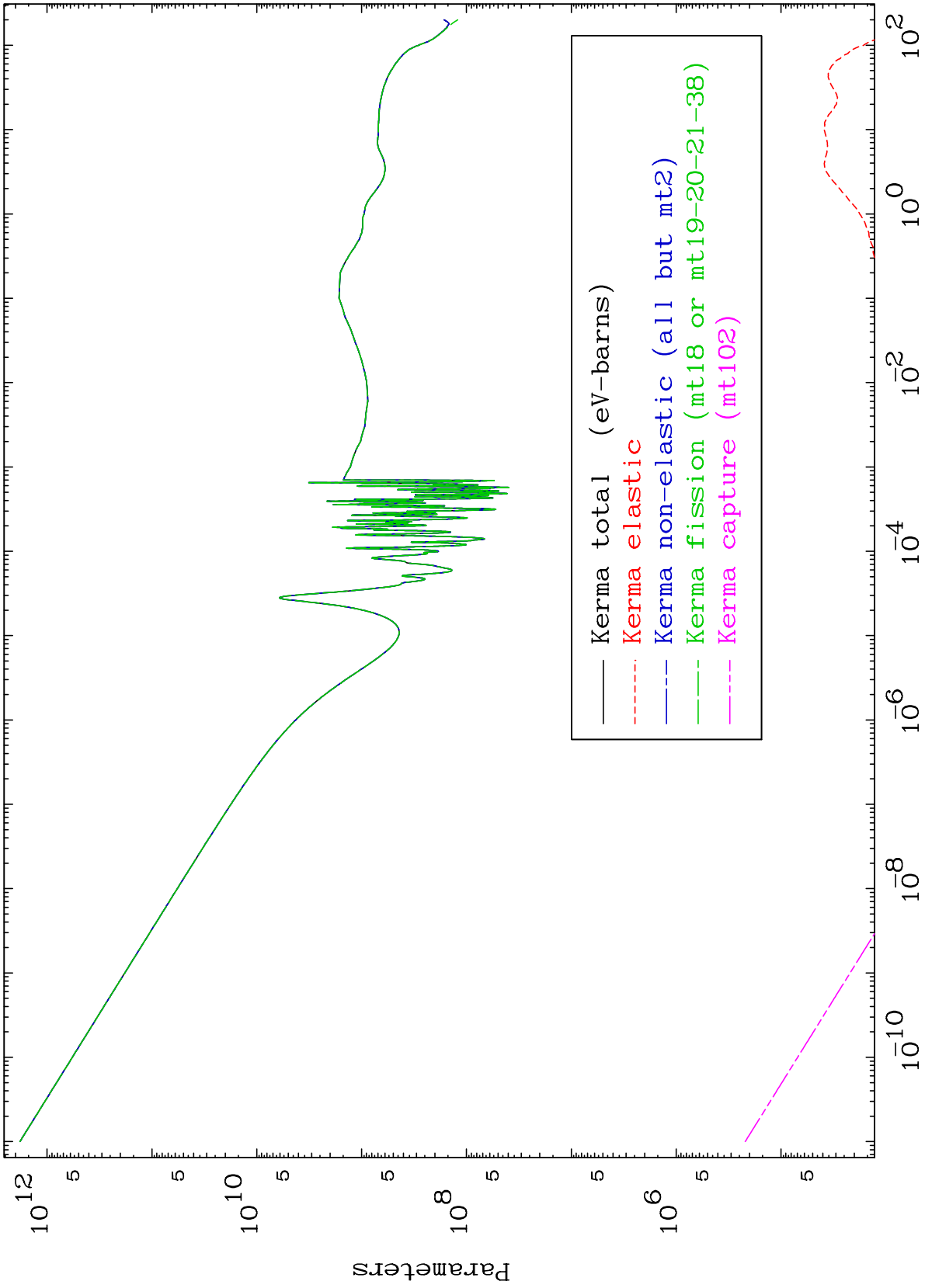
106-Sg-270

MAT 670

(n,α) Levels  
293 Kelvin Cross Sections

106-Sg-270

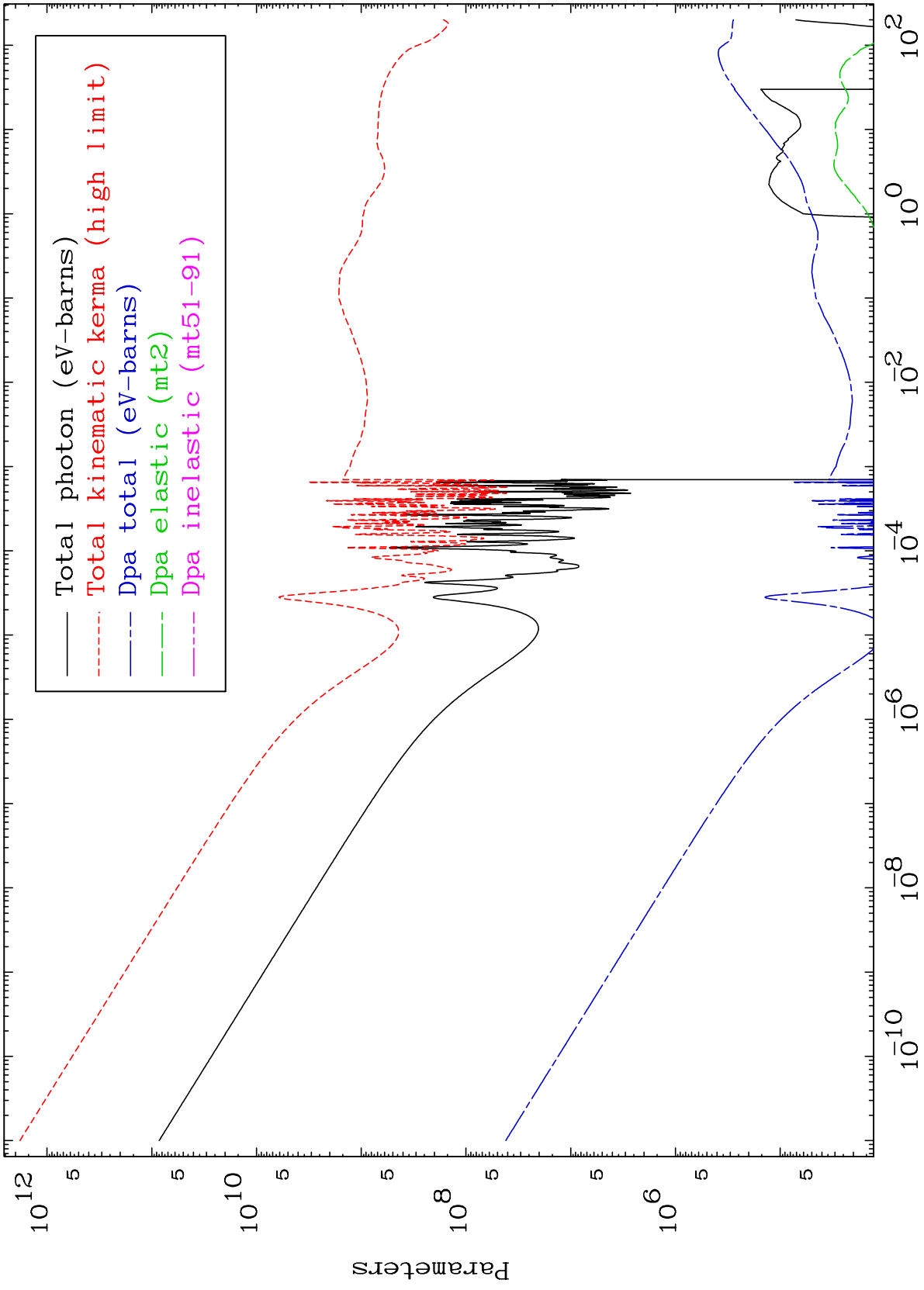




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Energy Release  
Parameters

106-Sg-270

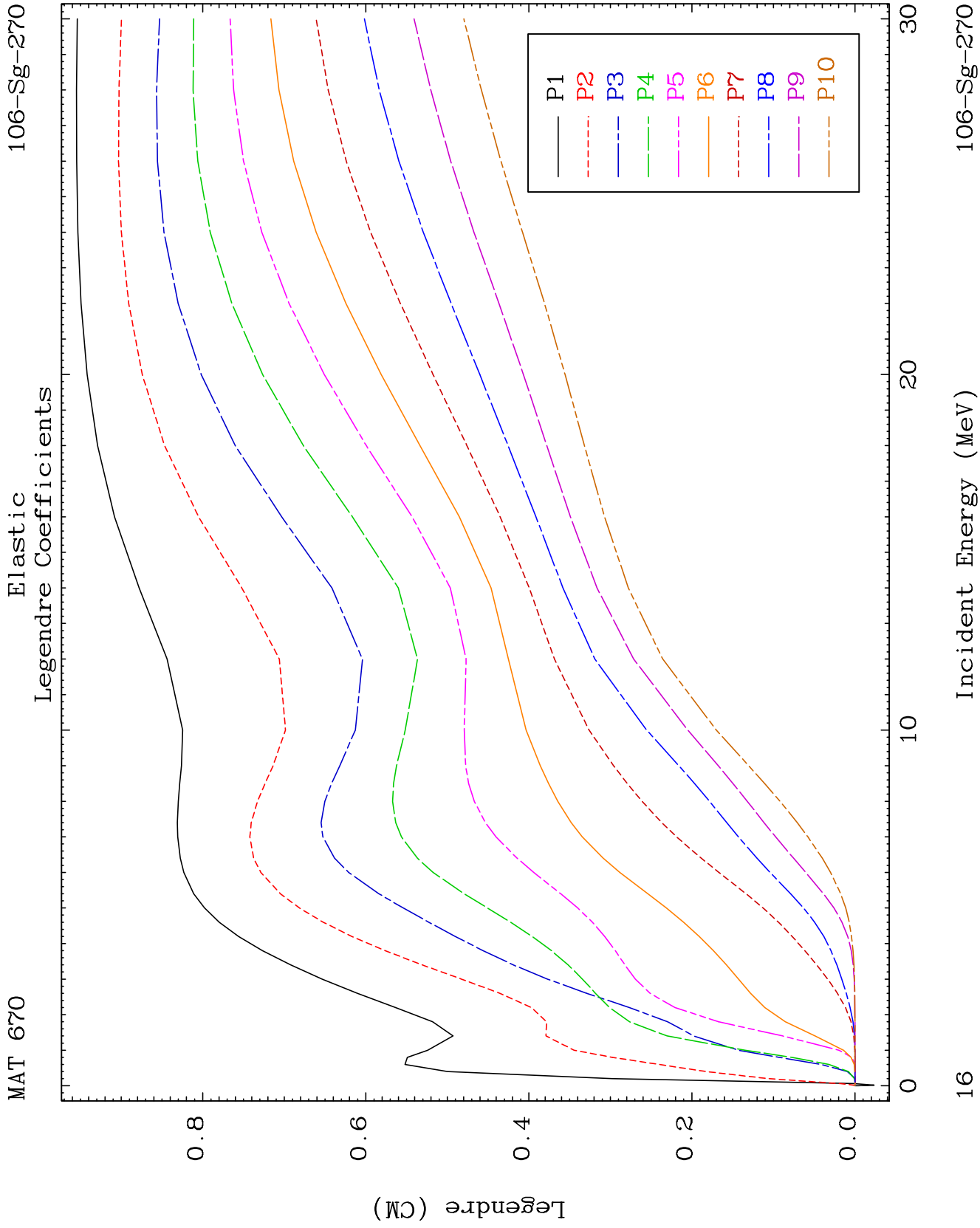


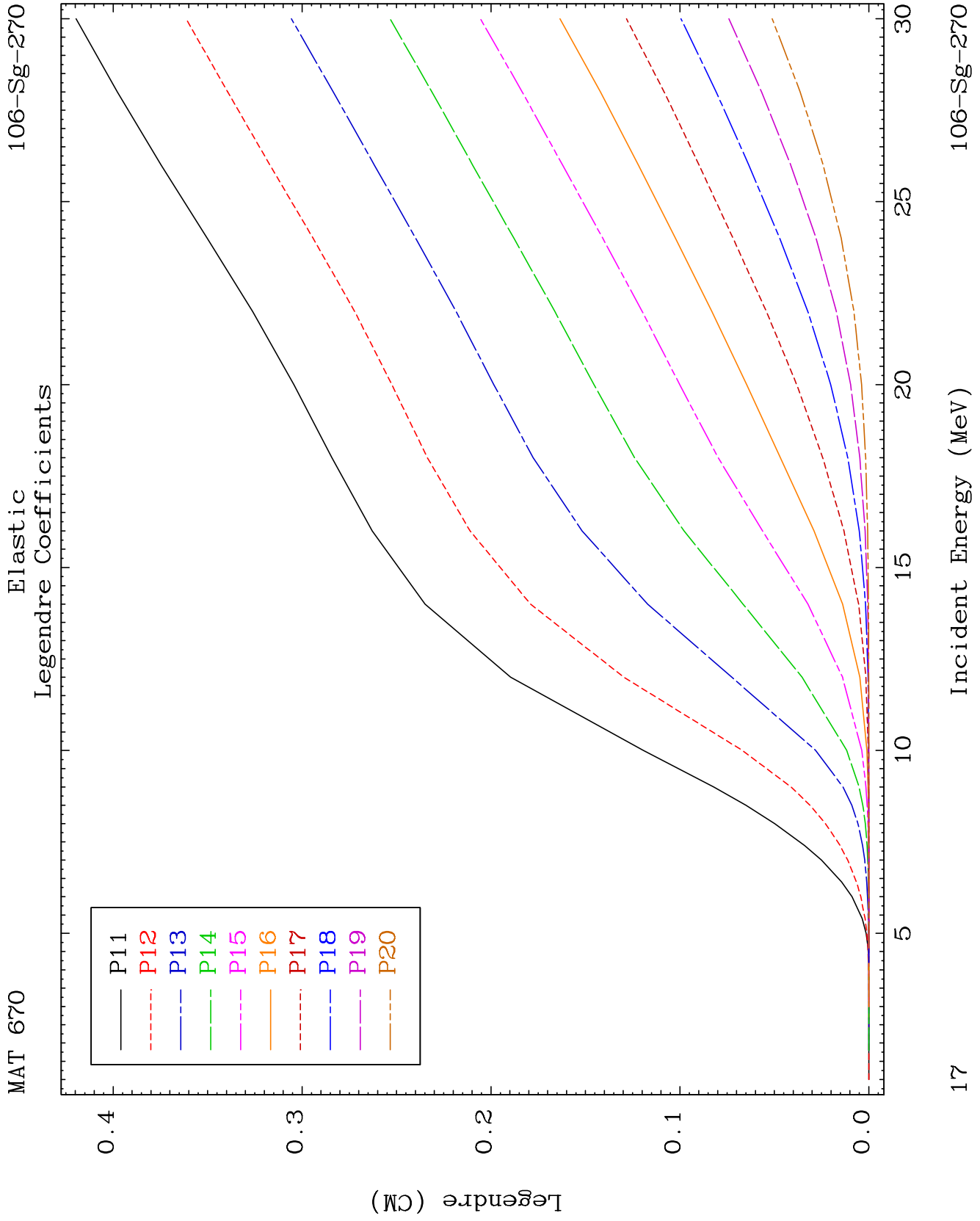
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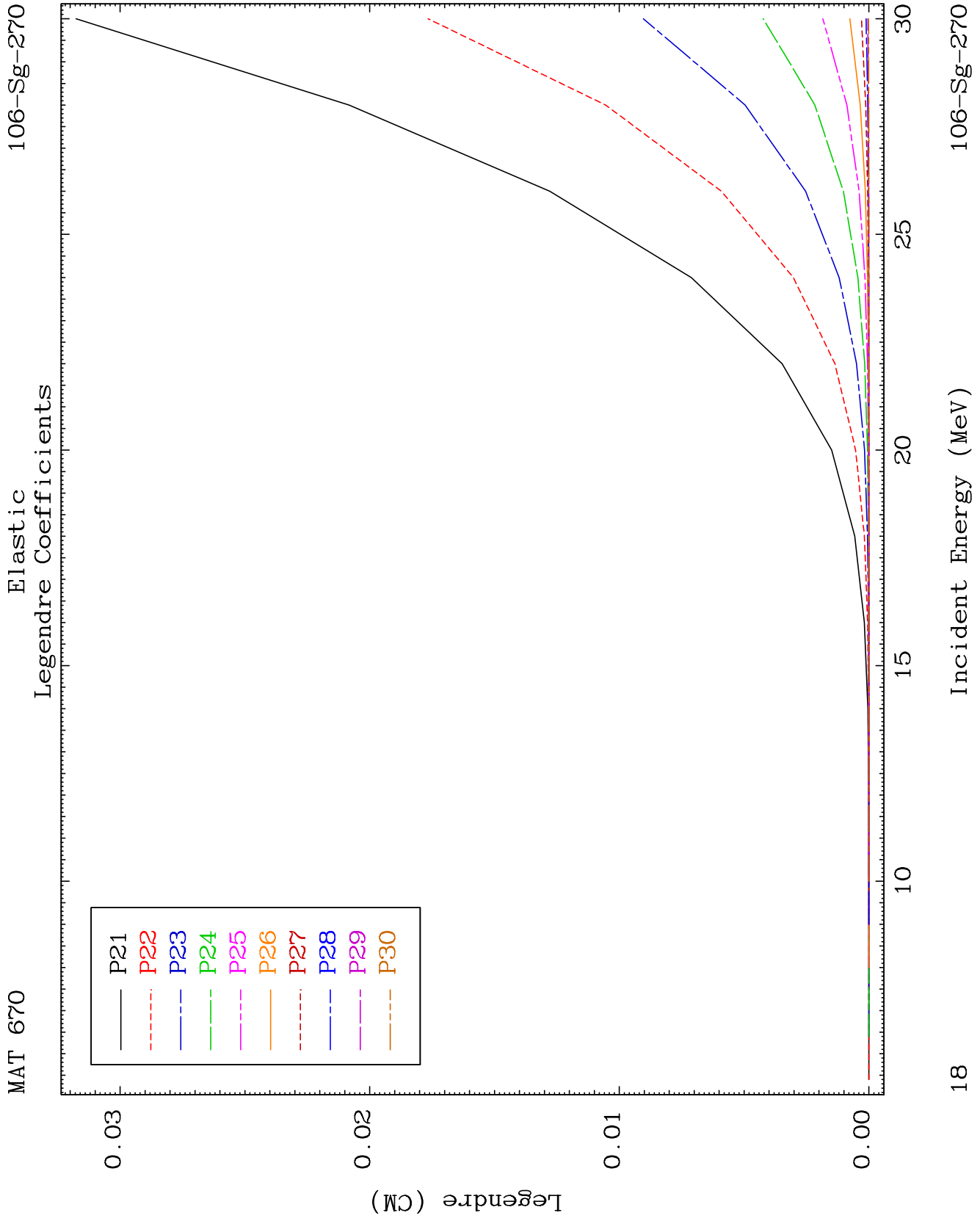
Incident Energy (MeV)

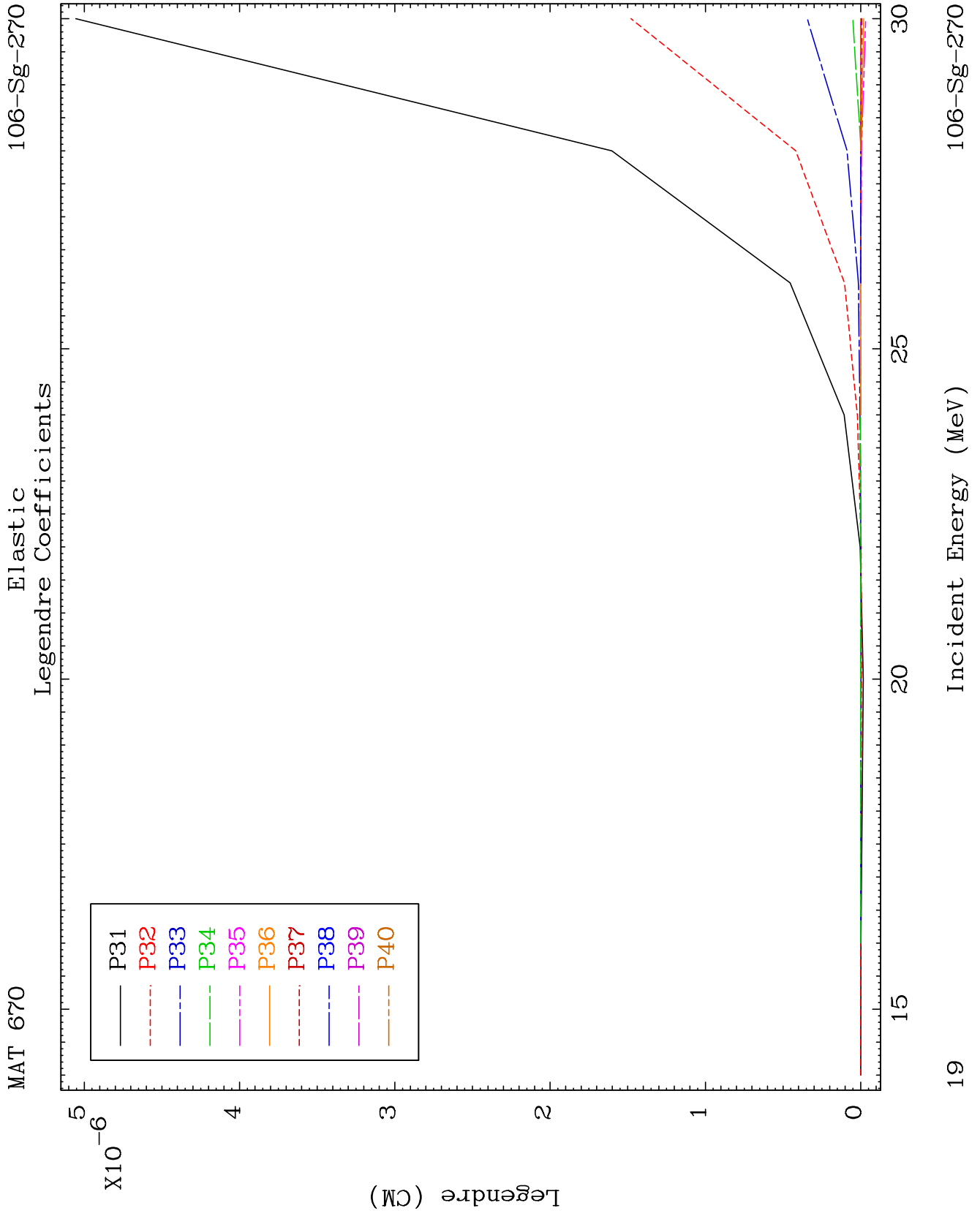
106-Sg-270







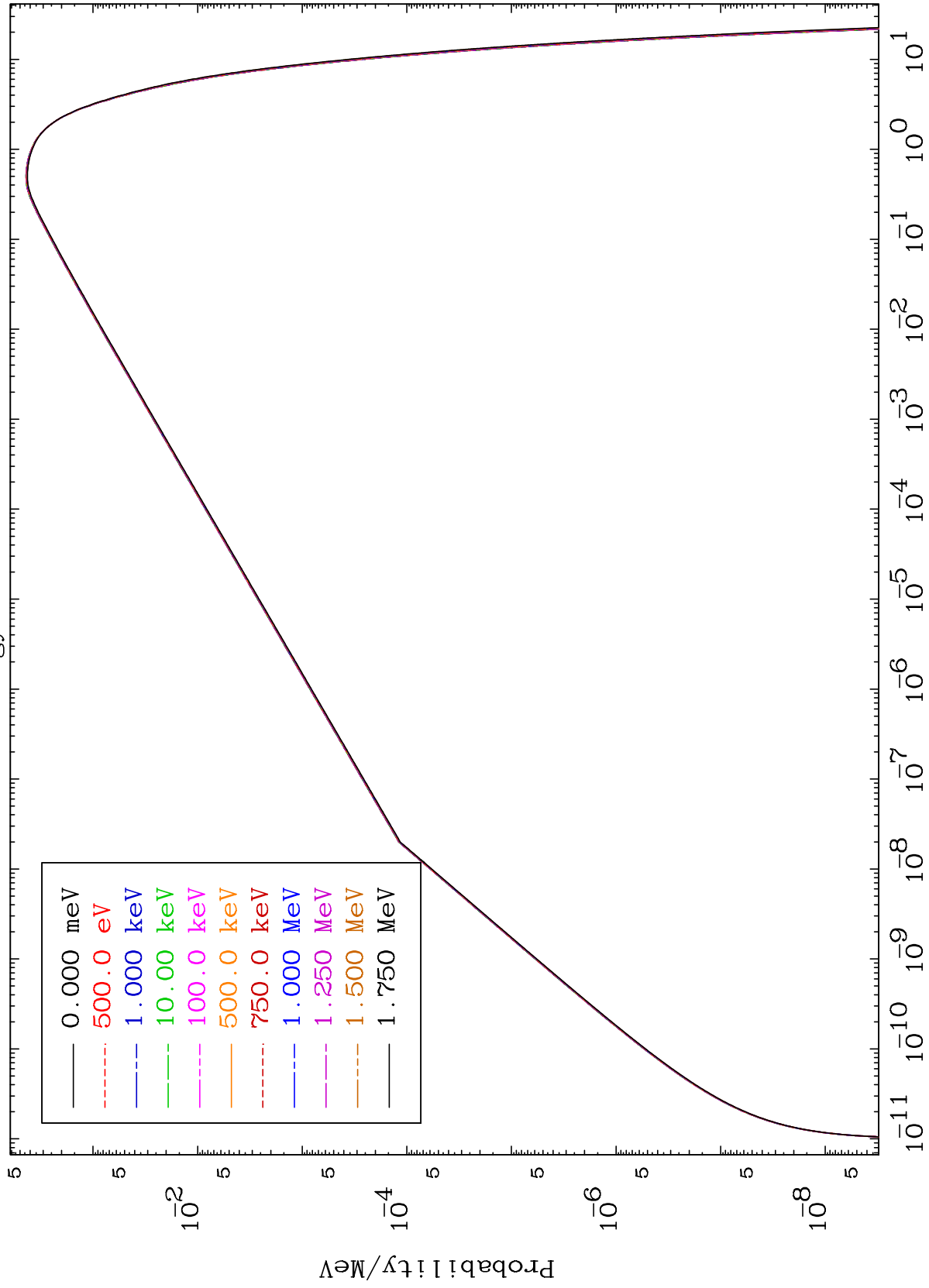




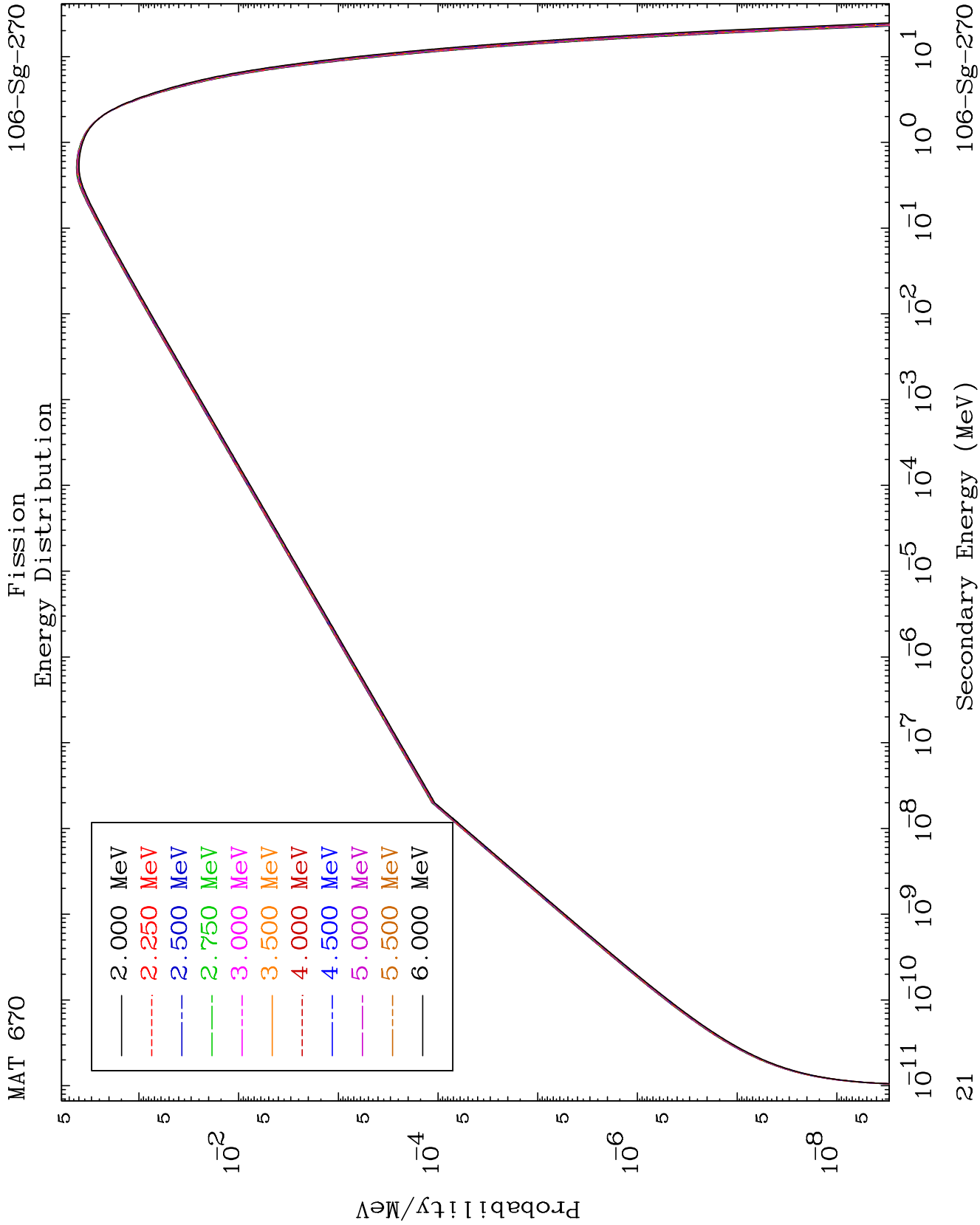
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### Fission Energy Distribution

106-Sg-270



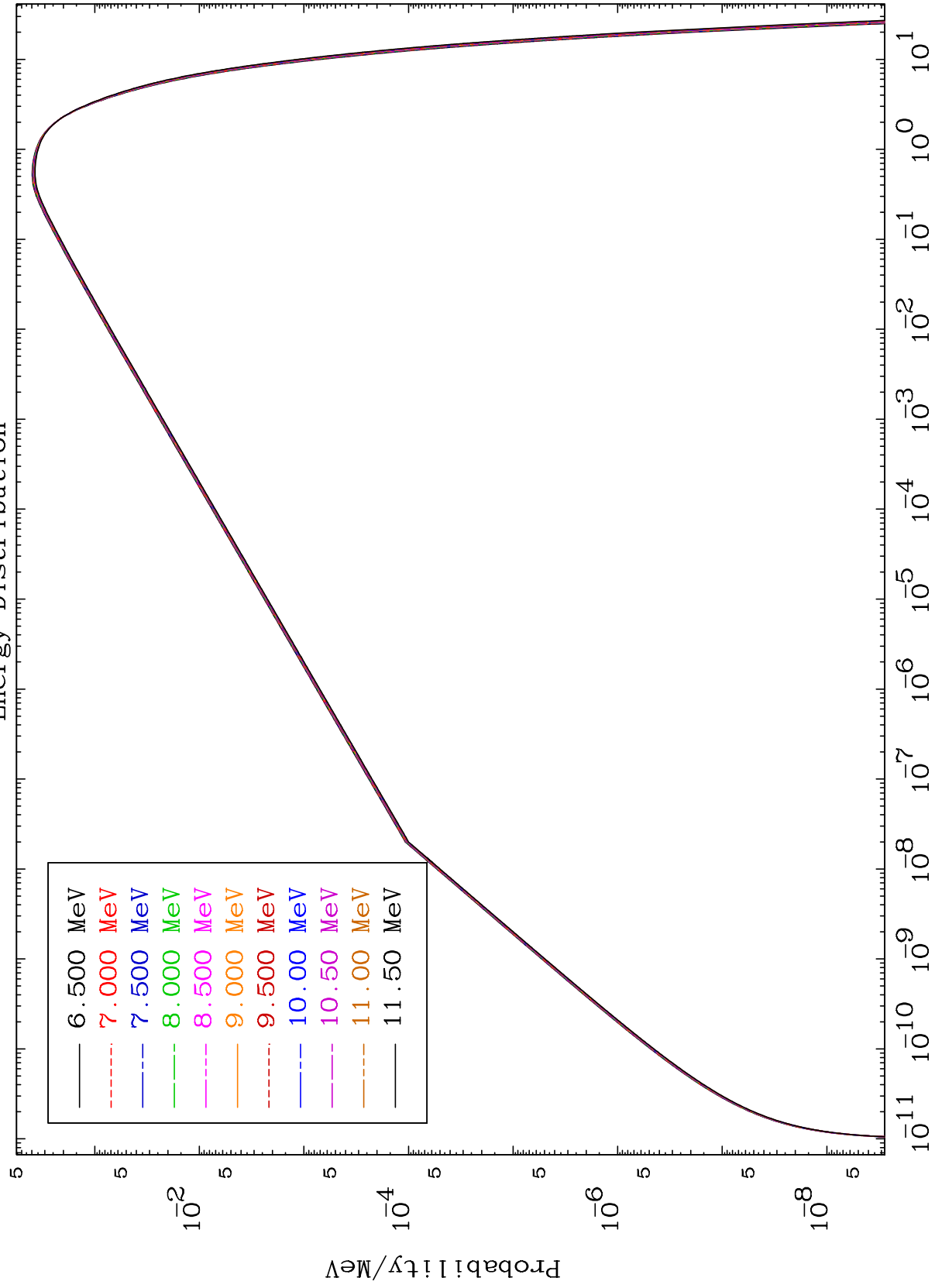
106-Sg-270



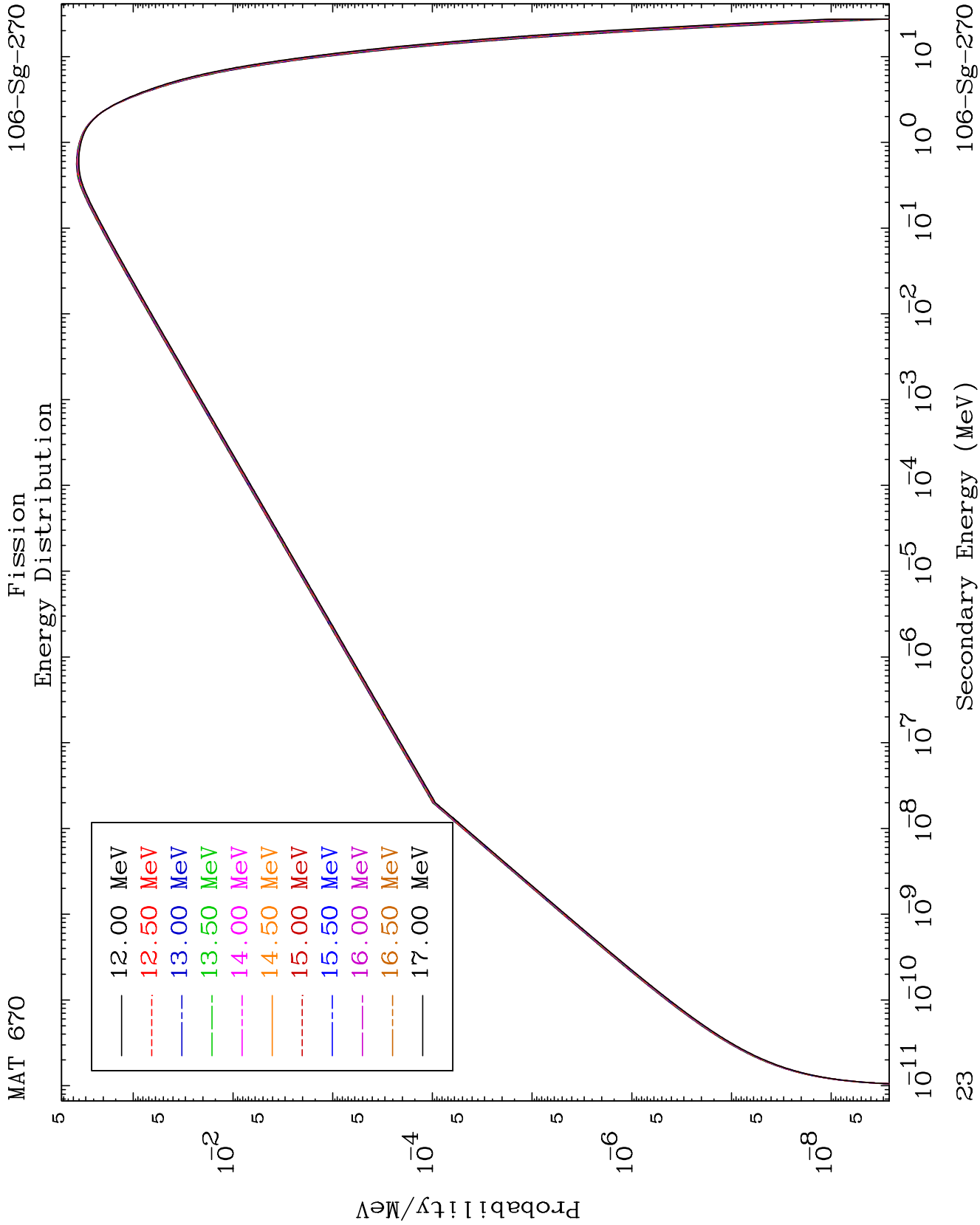
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# Fission Energy Distribution

106-Sg-270



106-Sg-270





MAT 670

# Fission Energy Distribution

106-Sg-270

