

Program EVALPLOT
(Version 2018-1)

by

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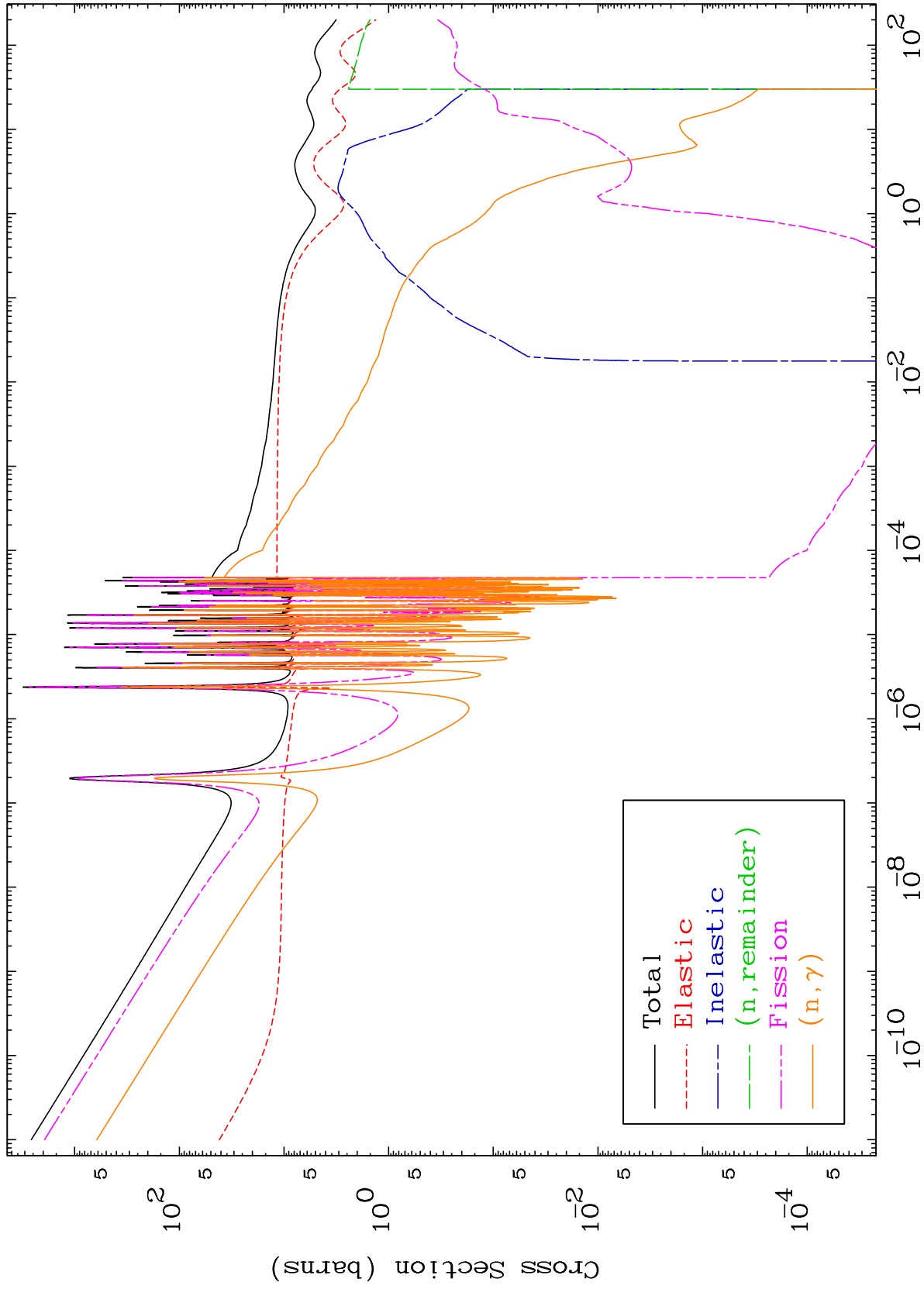
Press Mouse Button to Start

MAT 8922

Major

293 Kelvin Cross Sections

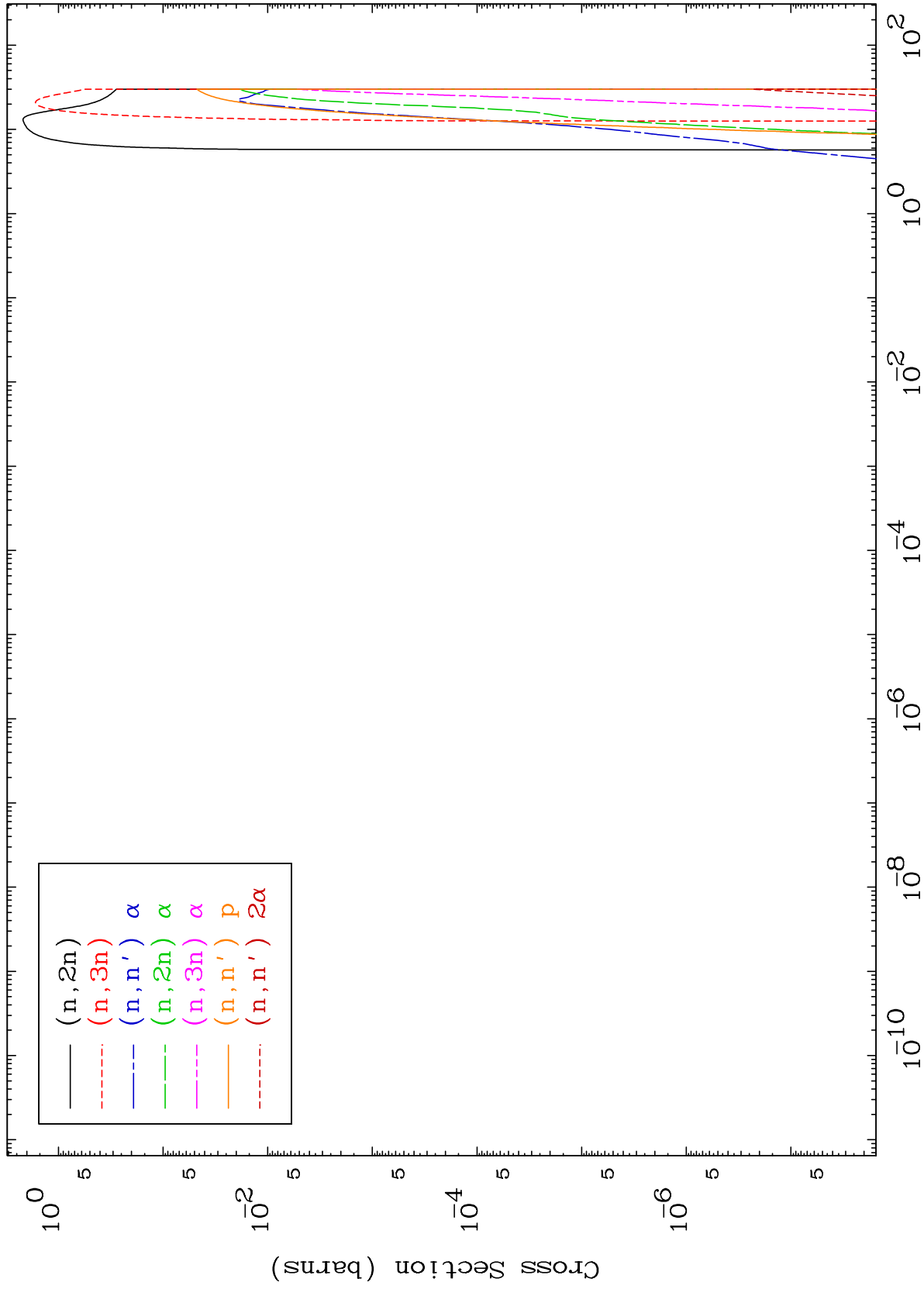
89-Ac-224



MAT 8922

Neutron Production
293 Kelvin Cross Sections

89-Ac-224



2

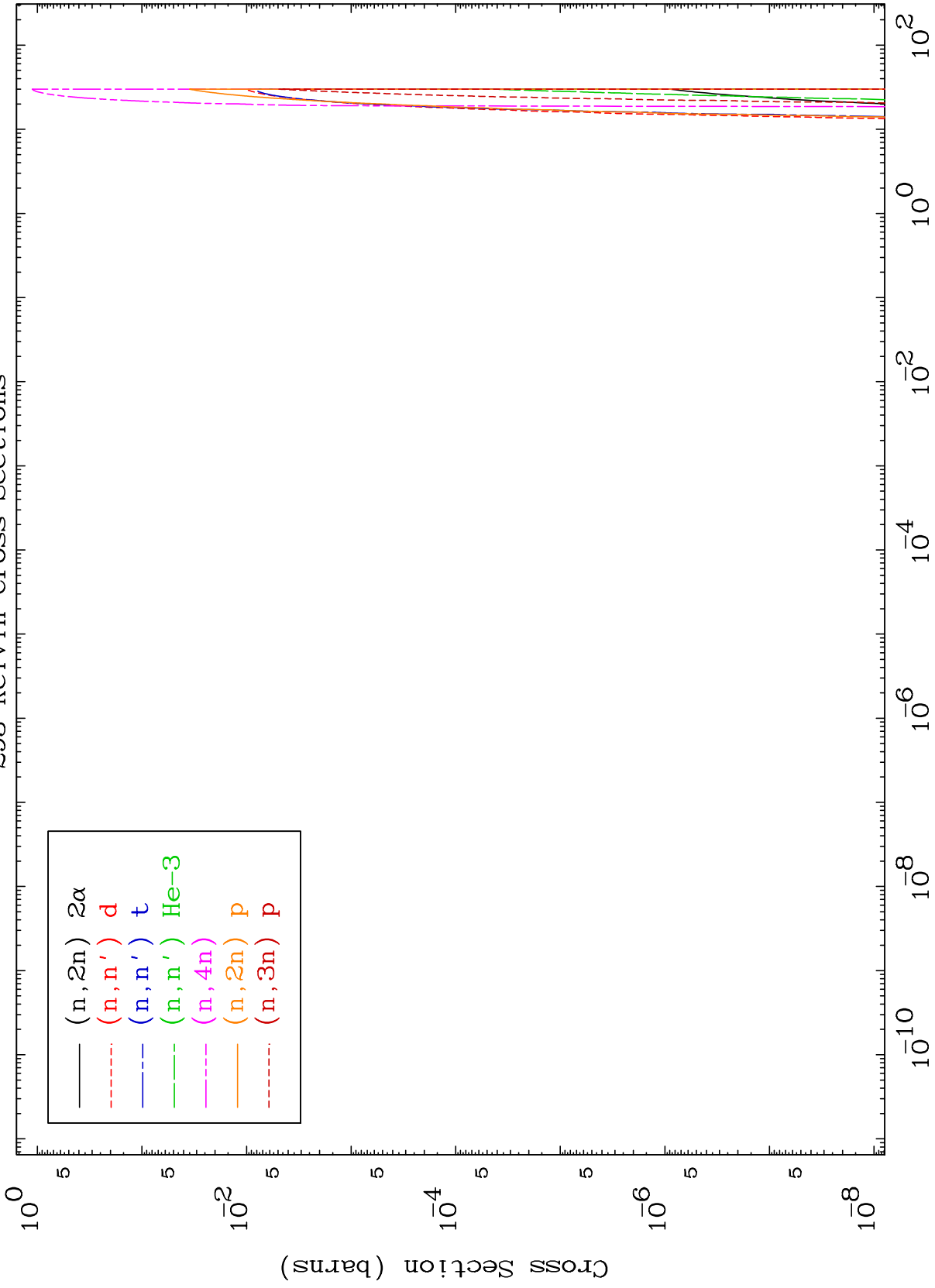
Incident Energy (MeV)

89-Ac-224

MAT 8922

Neutron Production
293 Kelvin Cross Sections

89-Ac-224



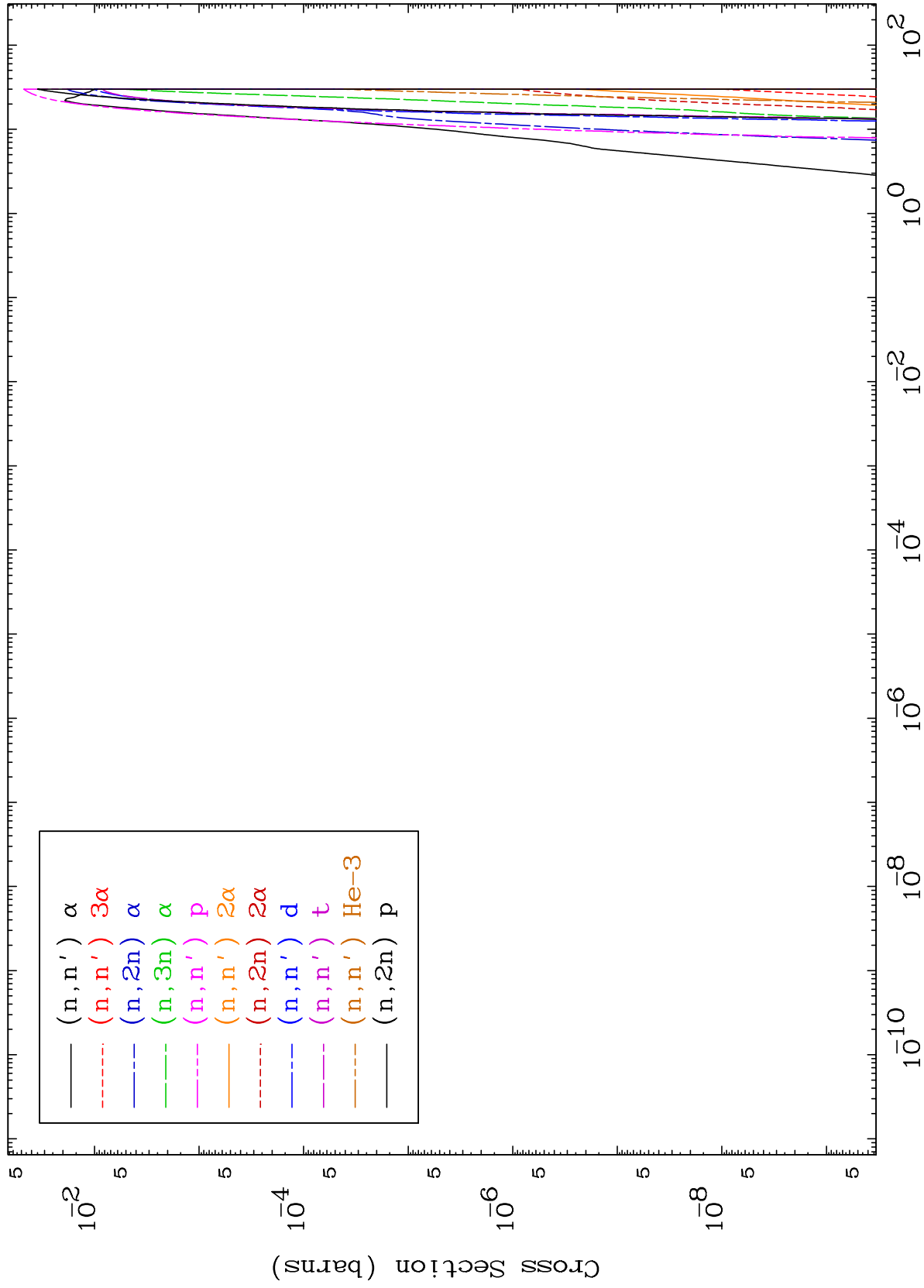
3

89-Ac-224

MAT 8922

Charged Particle
293 Kelvin Cross Sections

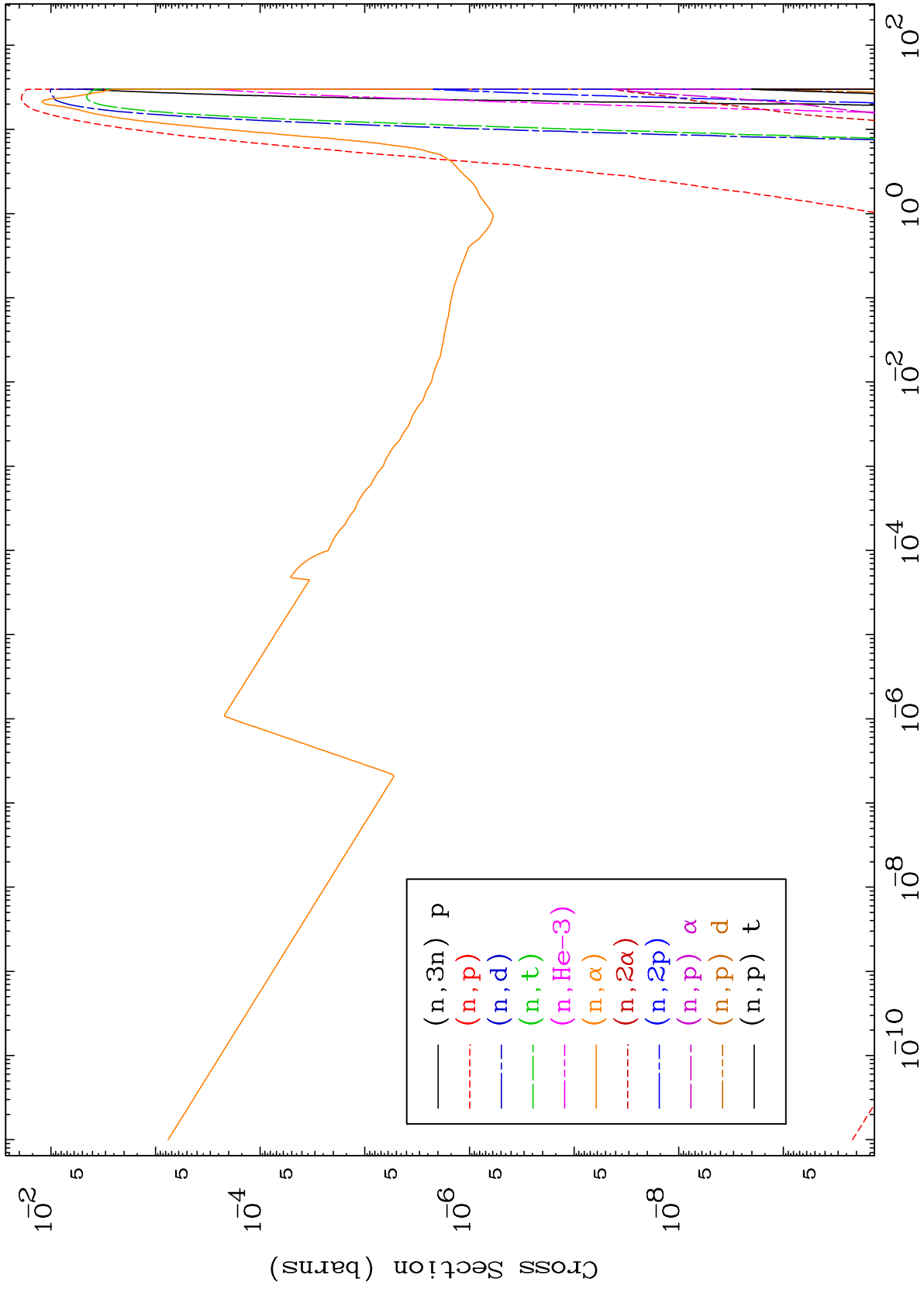
89-Ac-224



MAT 8922

Charged Particle
293 Kelvin Cross Sections

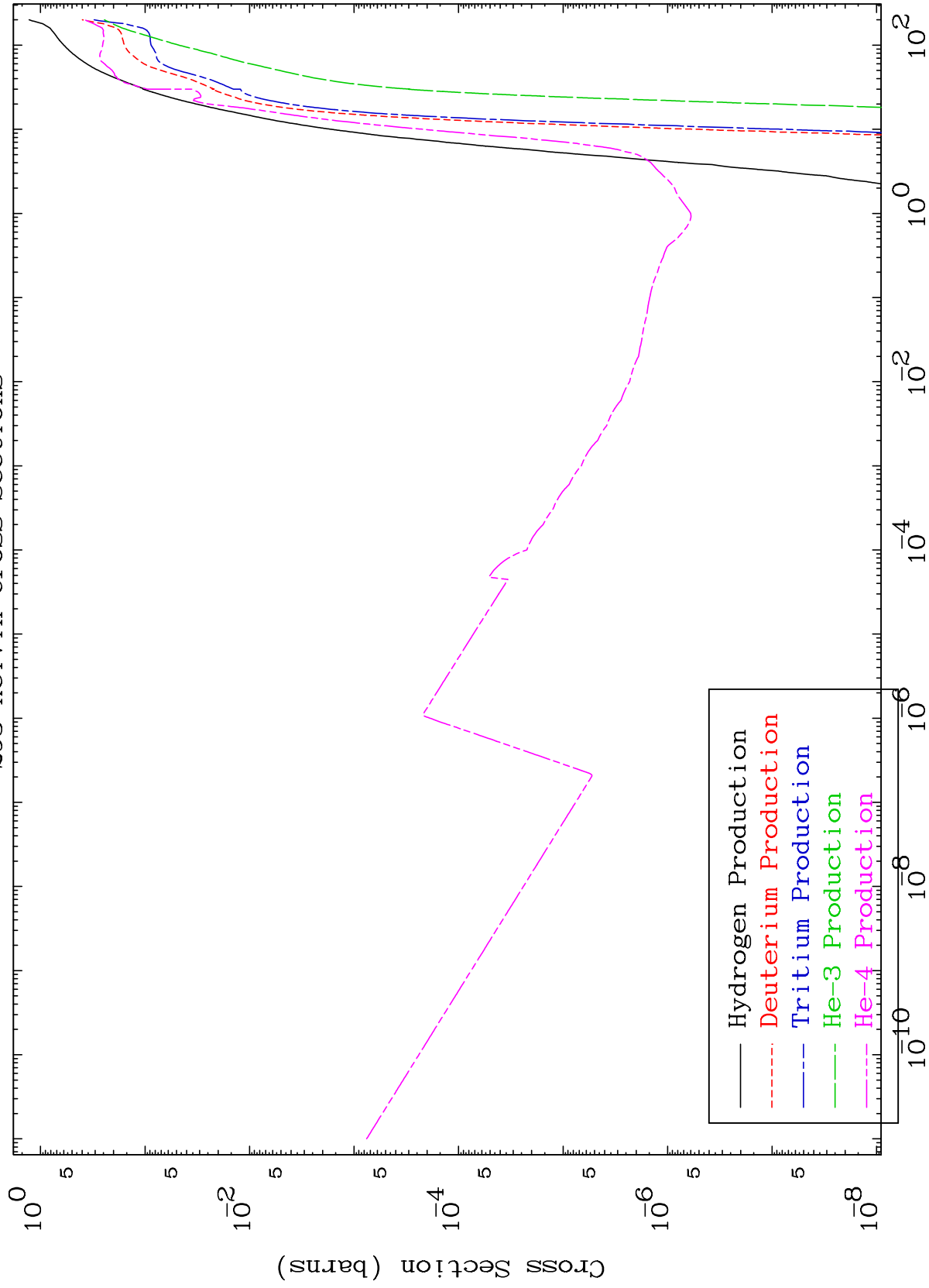
89-Ac-224



MAT 8922

Particle Production
293 Kelvin Cross Sections

89-Ac-224



6

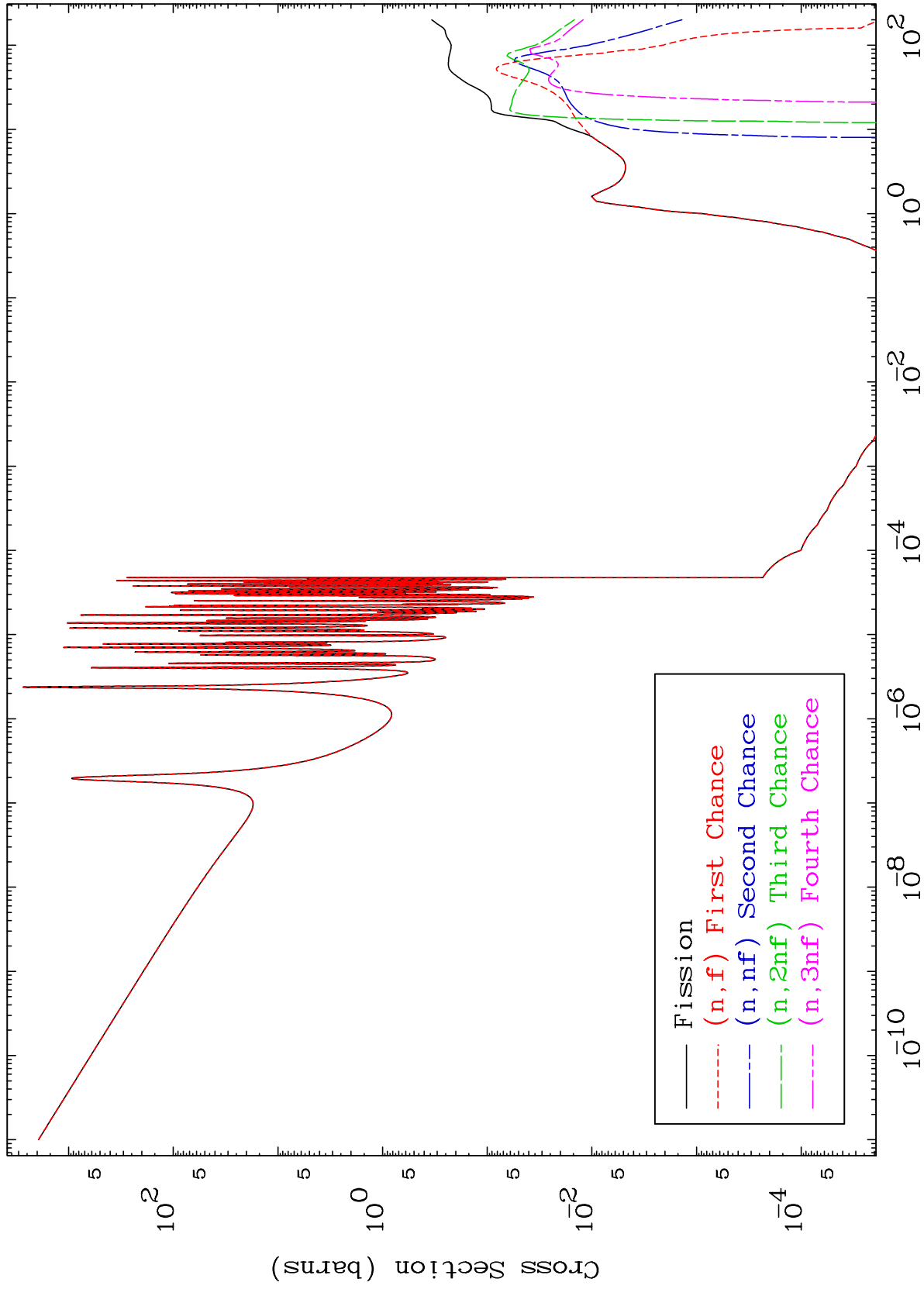
Incident Energy (MeV)

89-Ac-224

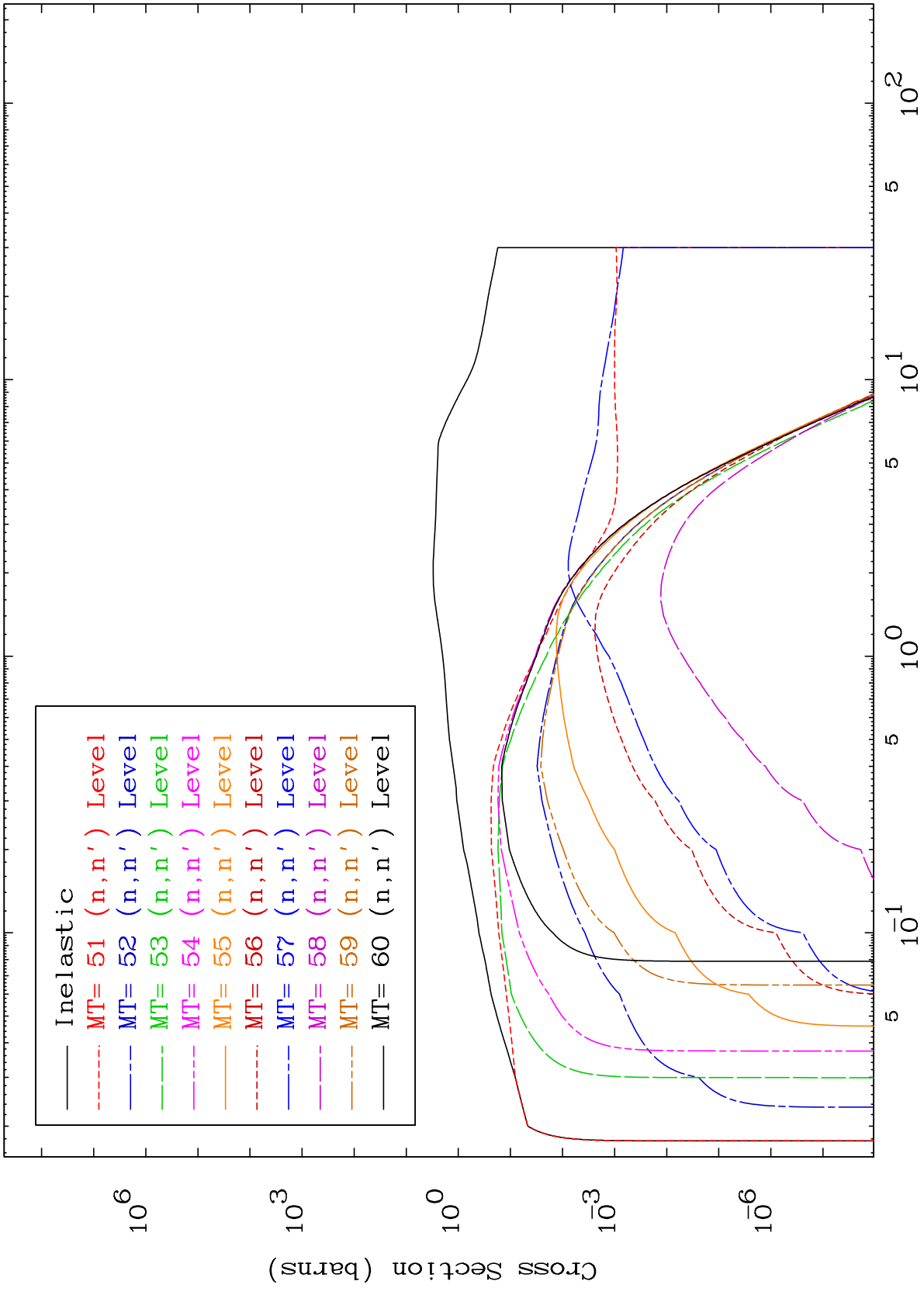
MAT 8922

Fission
293 Kelvin Cross Sections

89-Ac-224



89-Ac-224

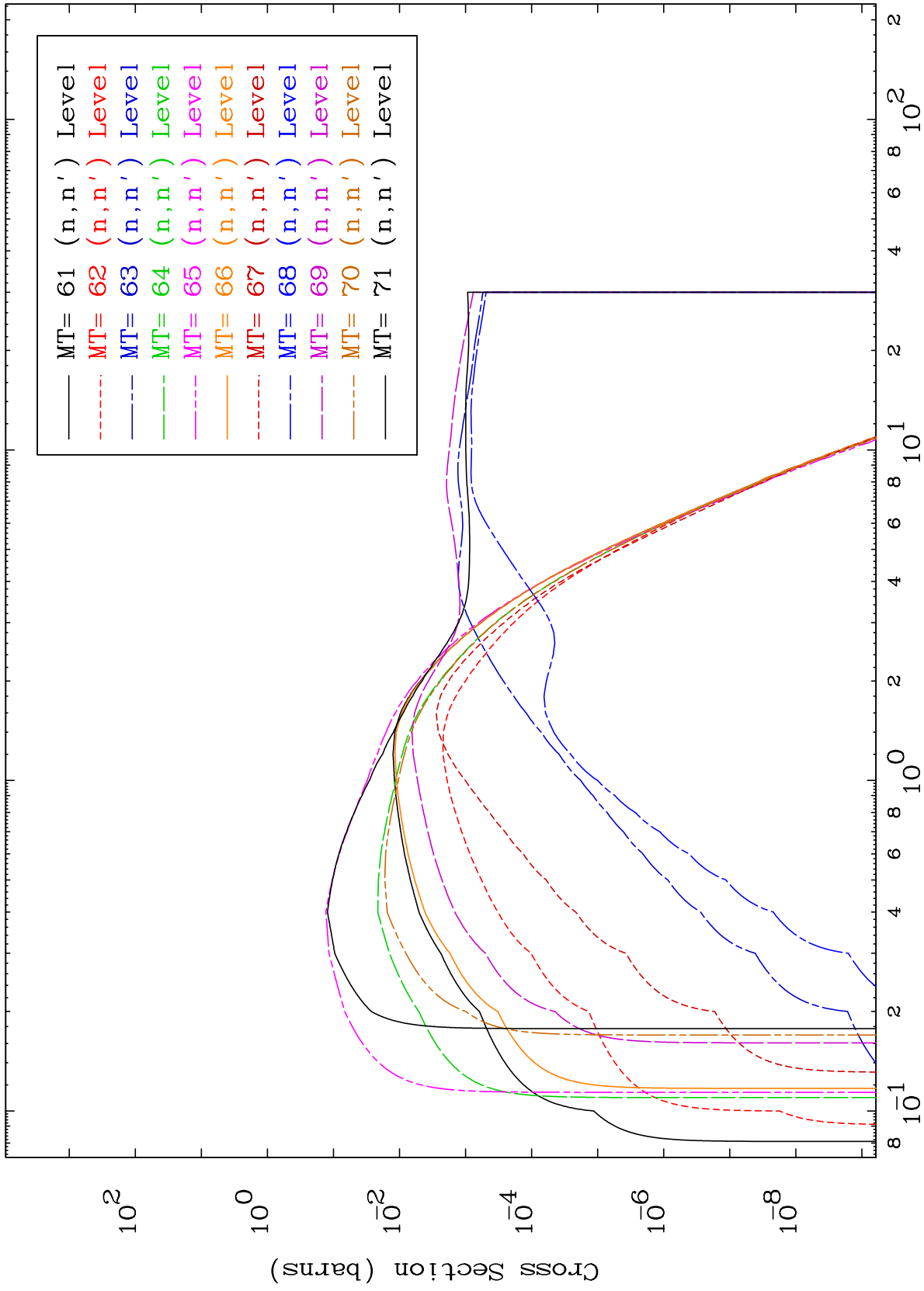


MAT 8922

(n,n') Level

89-Ac-224

293 Kelvin Cross Sections



9

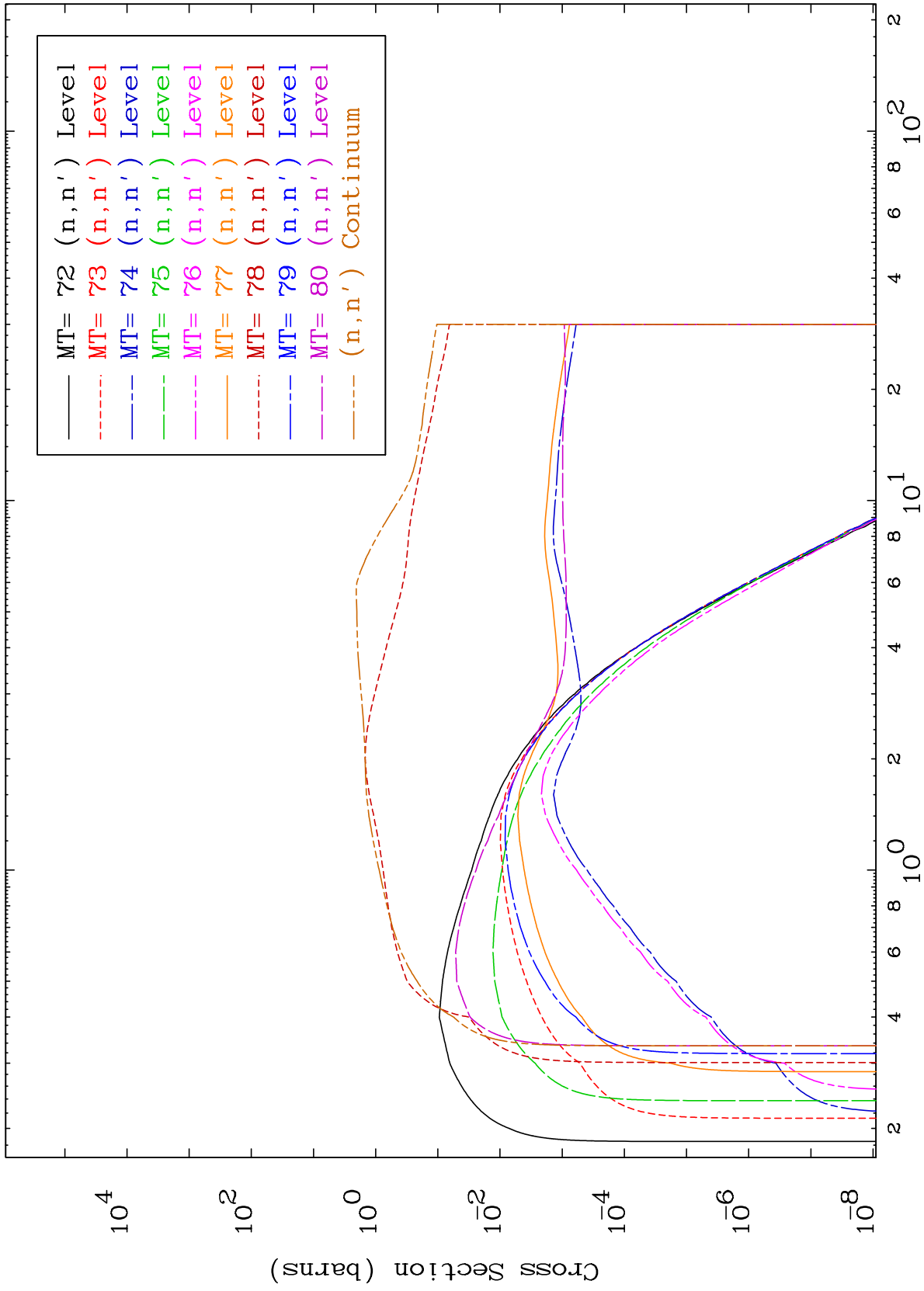
Incident Energy (MeV)

89-Ac-224

MAT 8922

(n,n') Level
293 Kelvin Cross Sections

89-Ac-224



10

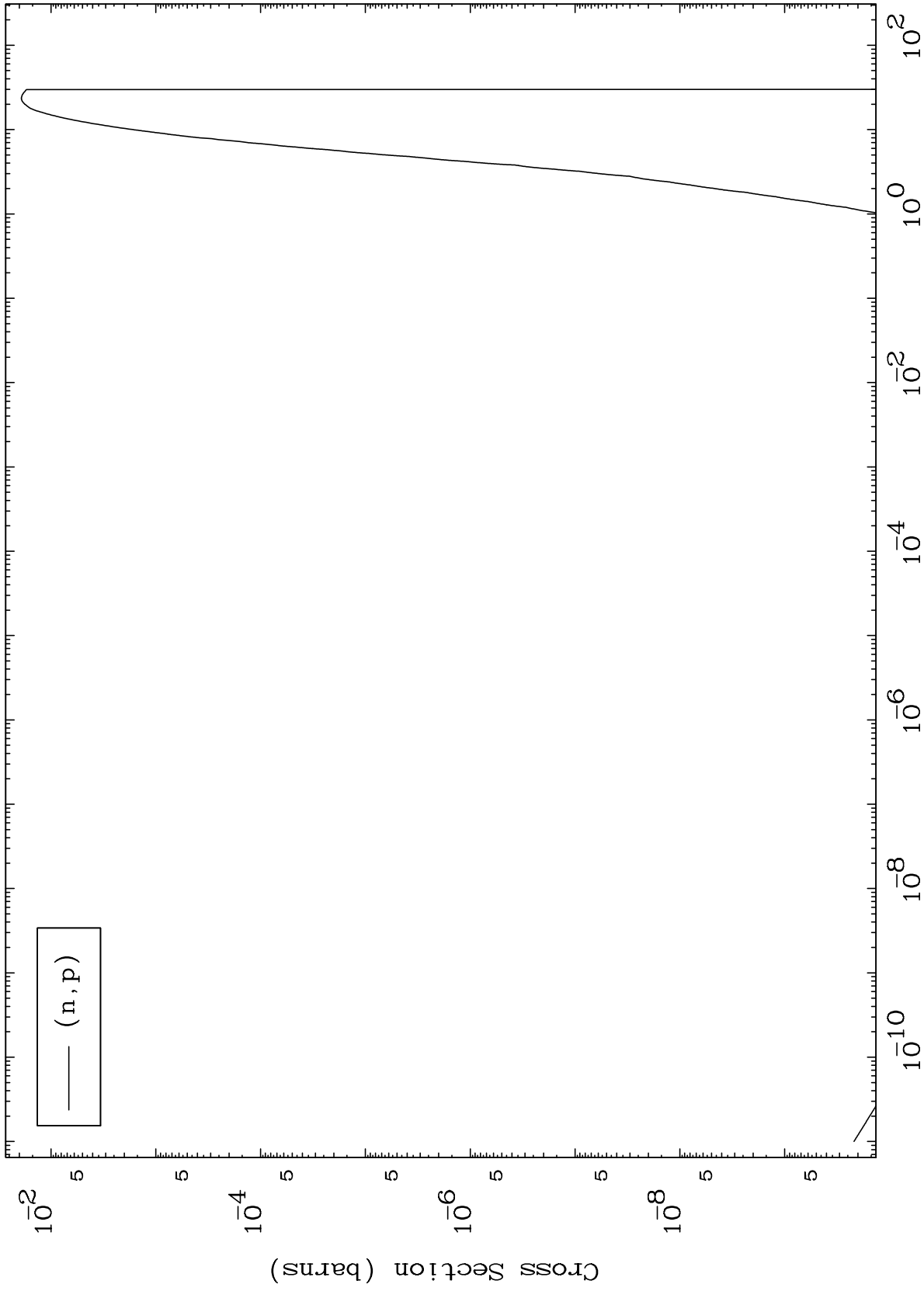
Incident Energy (MeV)

89-Ac-224

MAT 8922

(n,p) Levels
293 Kelvin Cross Sections

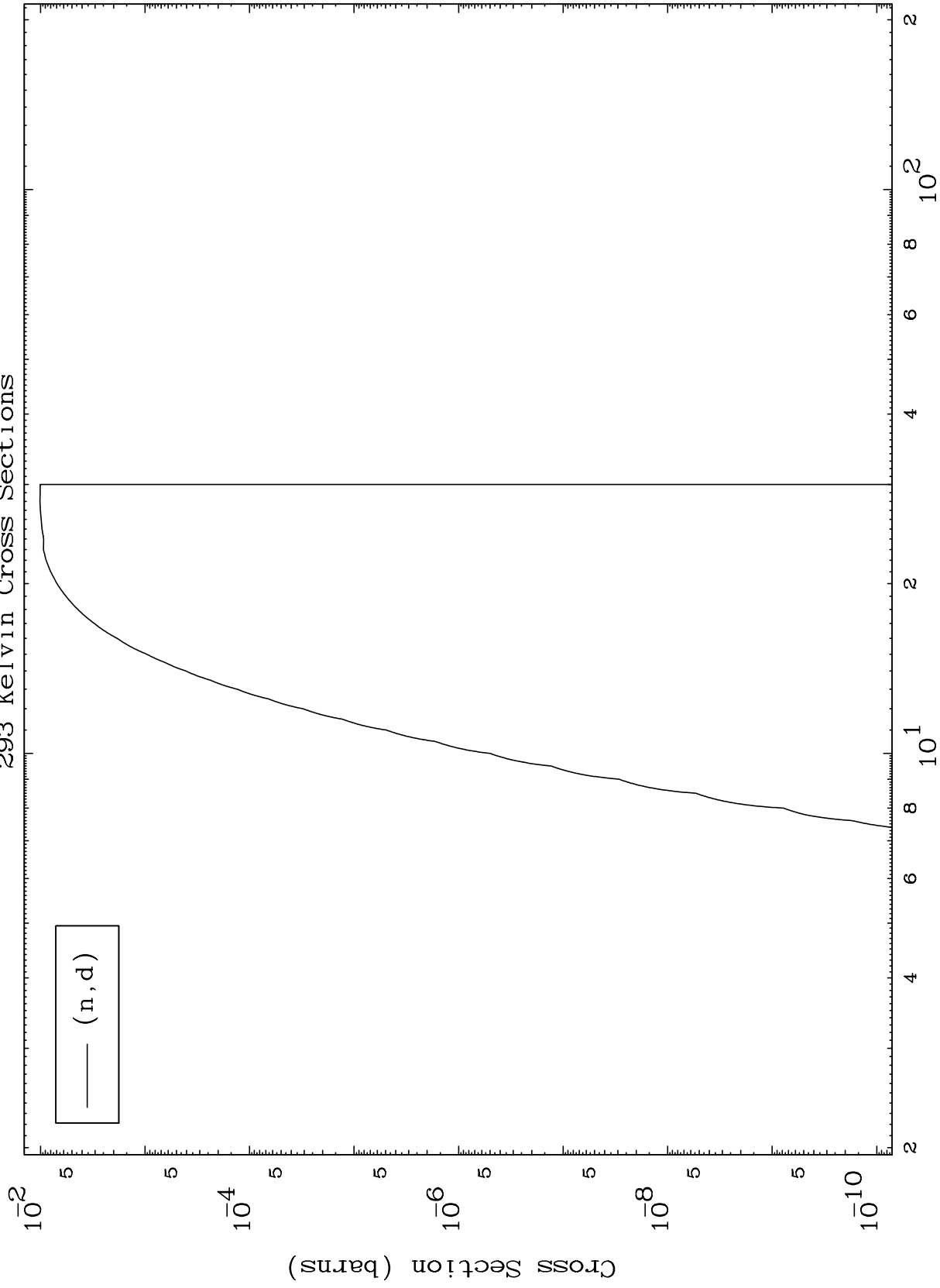
89-Ac-224



MAT 8922

(n,d) Levels
293 Kelvin Cross Sections

89-Ac-224



12

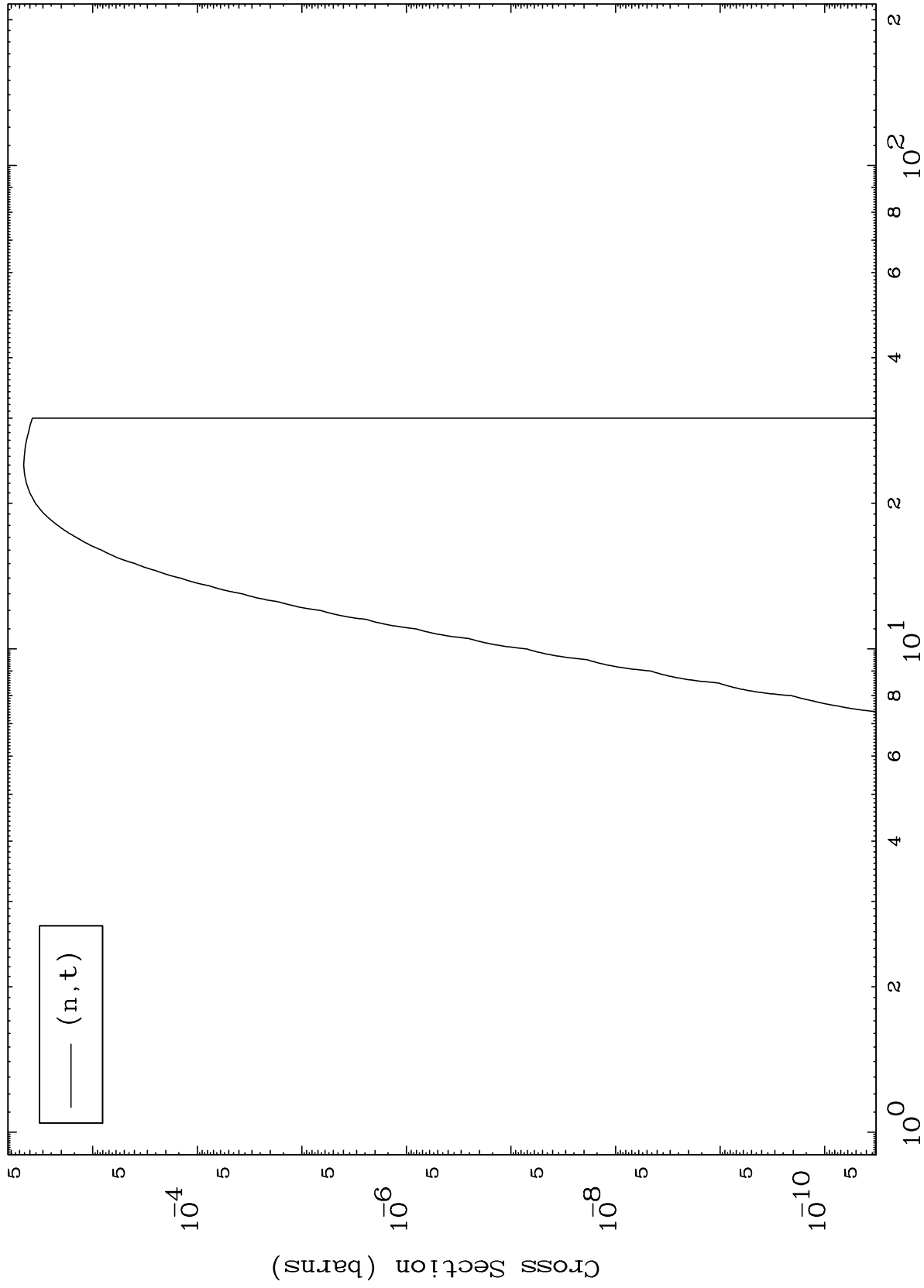
Incident Energy (MeV)

89-Ac-224

MAT 8922

(n,t) Levels
293 Kelvin Cross Sections

89-Ac-224



13

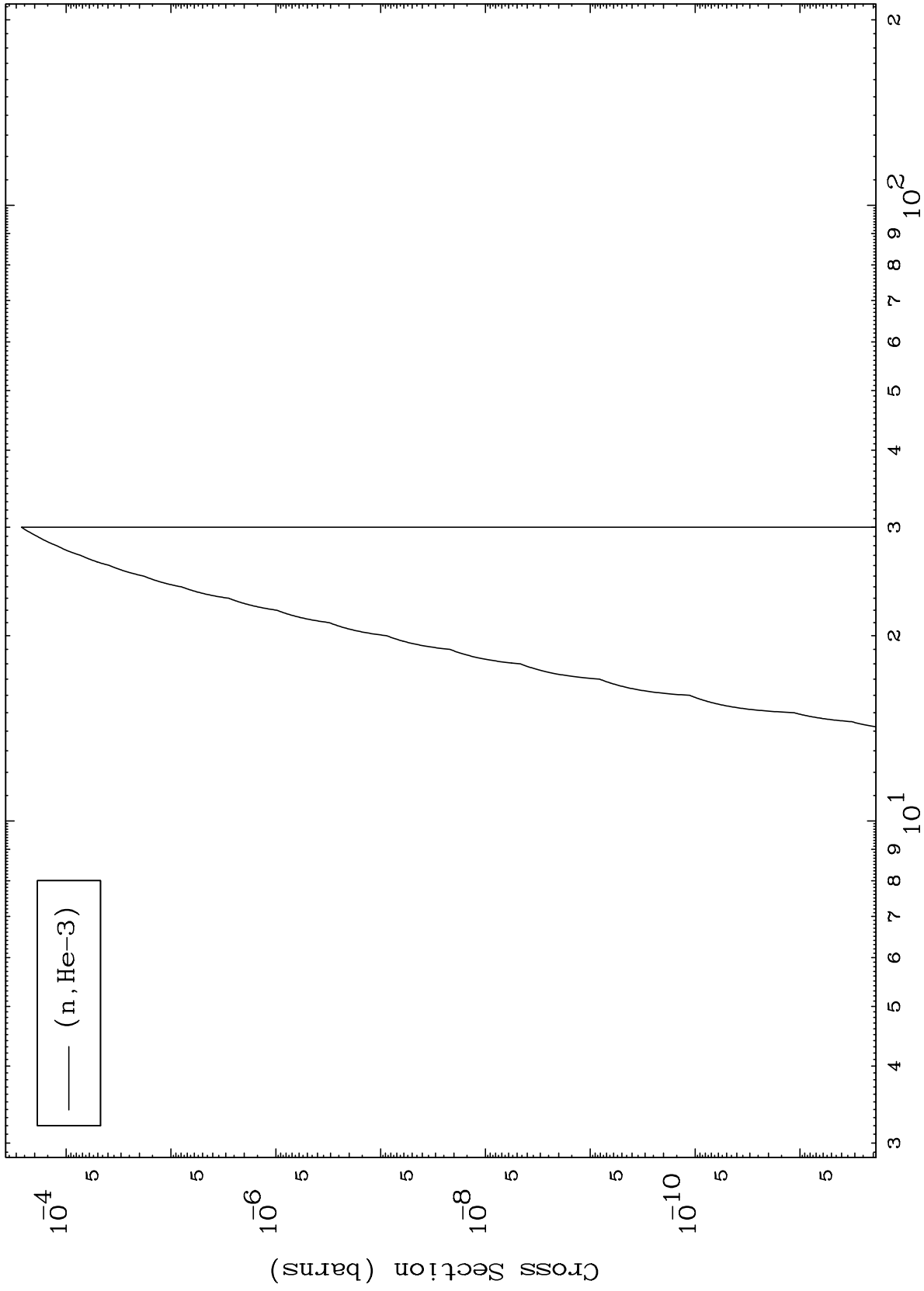
Incident Energy (MeV)

89-Ac-224

MAT 8922

(n,He3) Levels
293 Kelvin Cross Sections

89-Ac-224



14

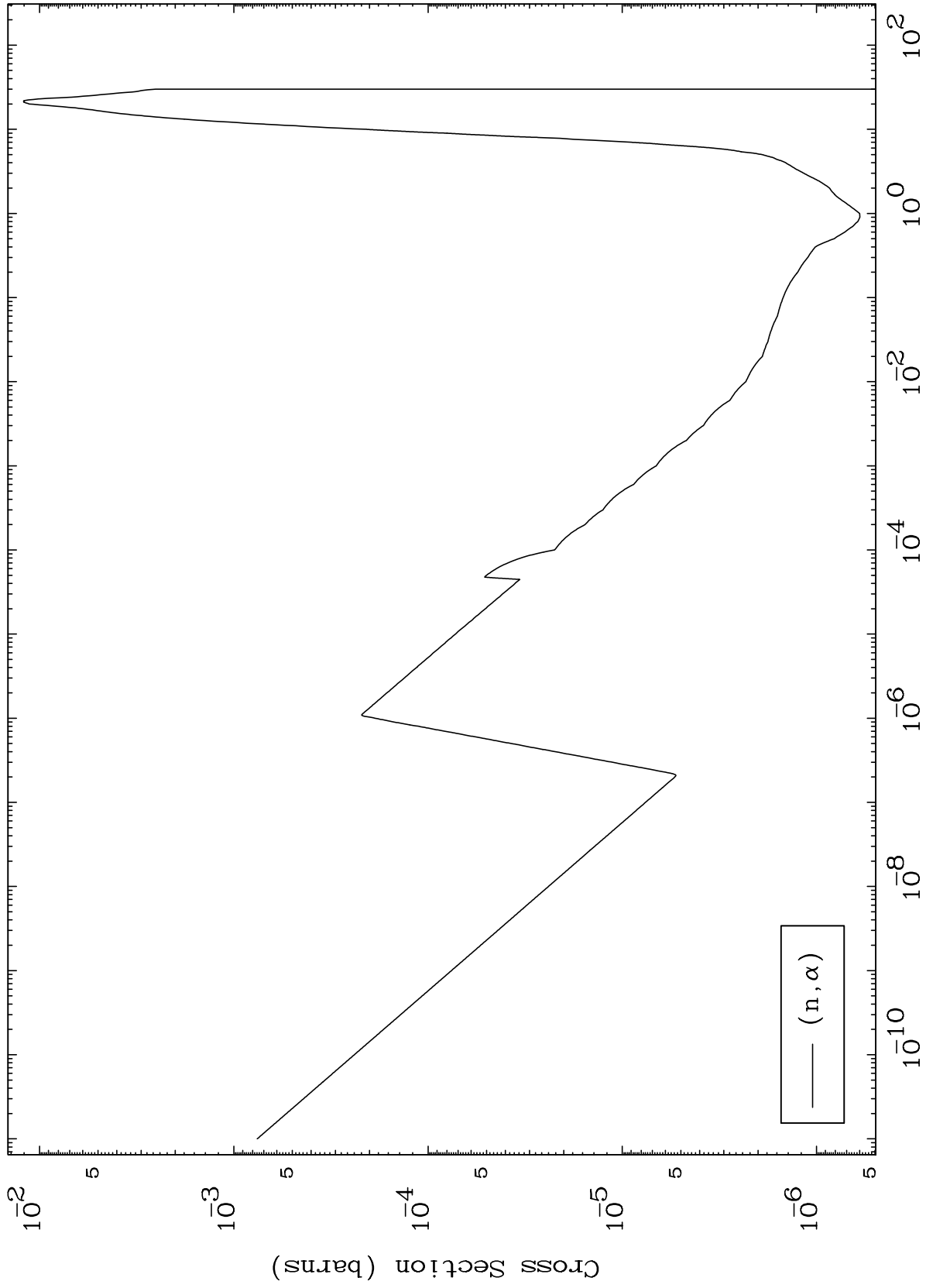
Incident Energy (MeV)

89-Ac-224

MAT 8922

(n,α) Levels
293 Kelvin Cross Sections

89-Ac-224

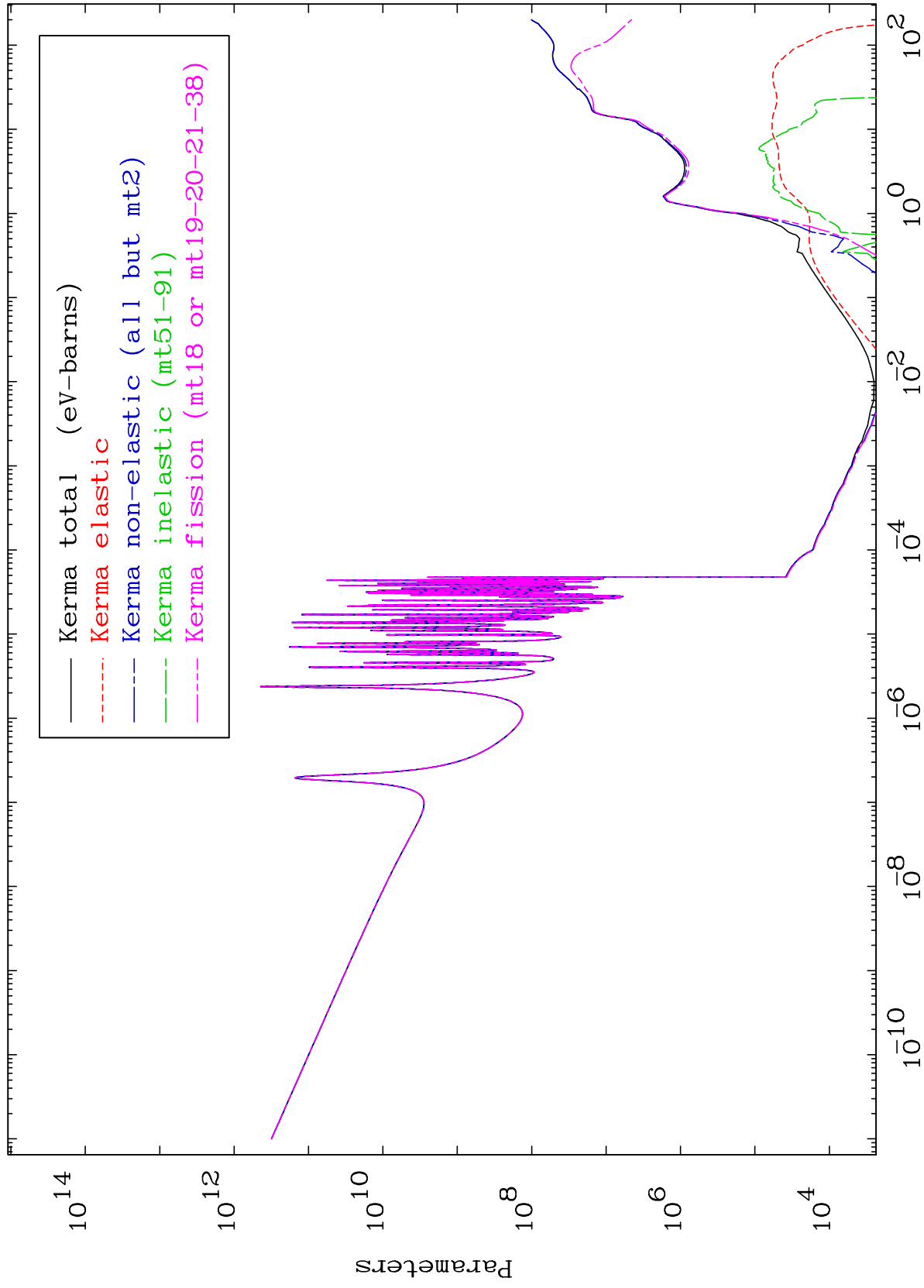


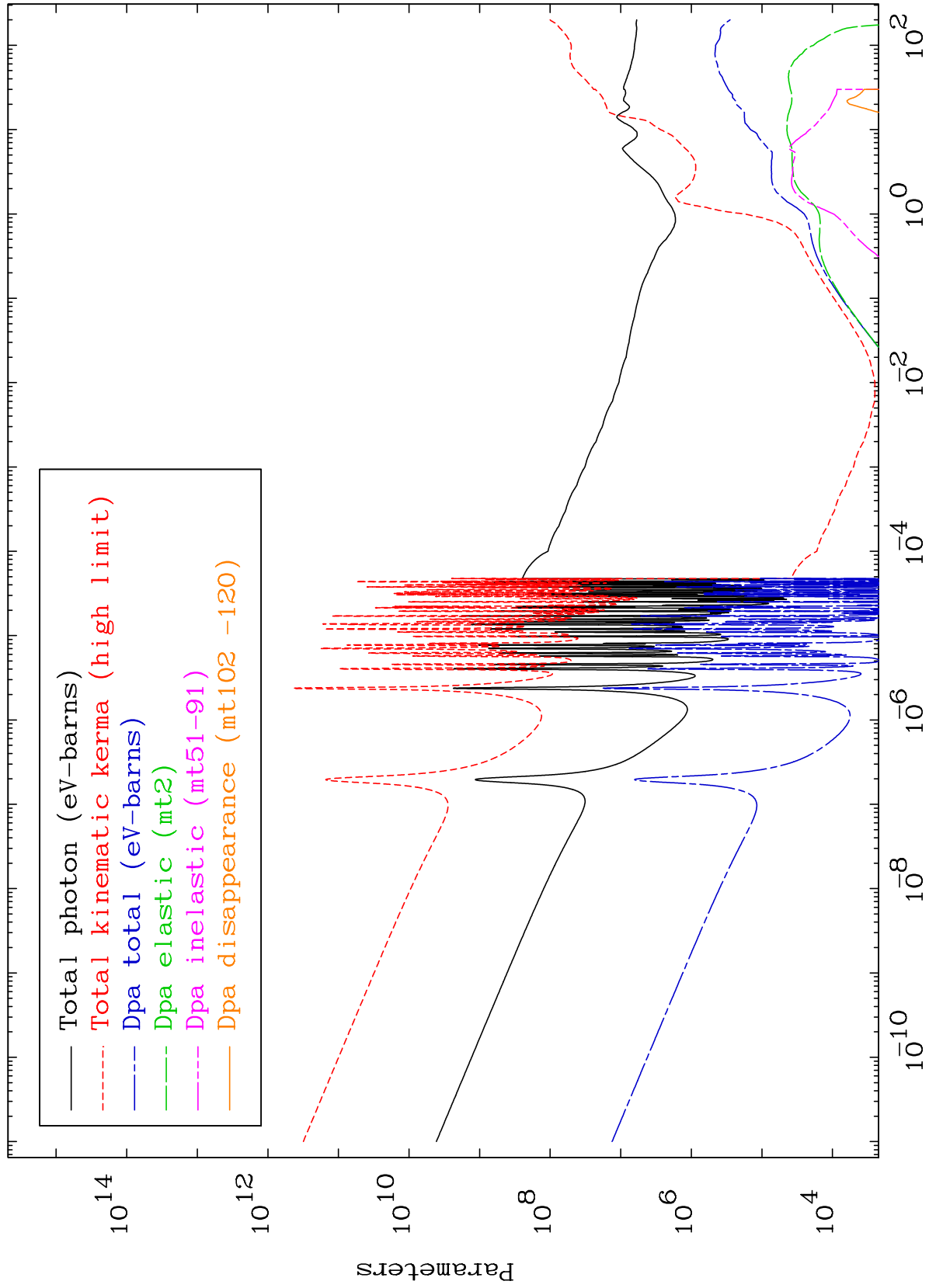
15

Incident Energy (MeV)

89-Ac-224

— (n,α)

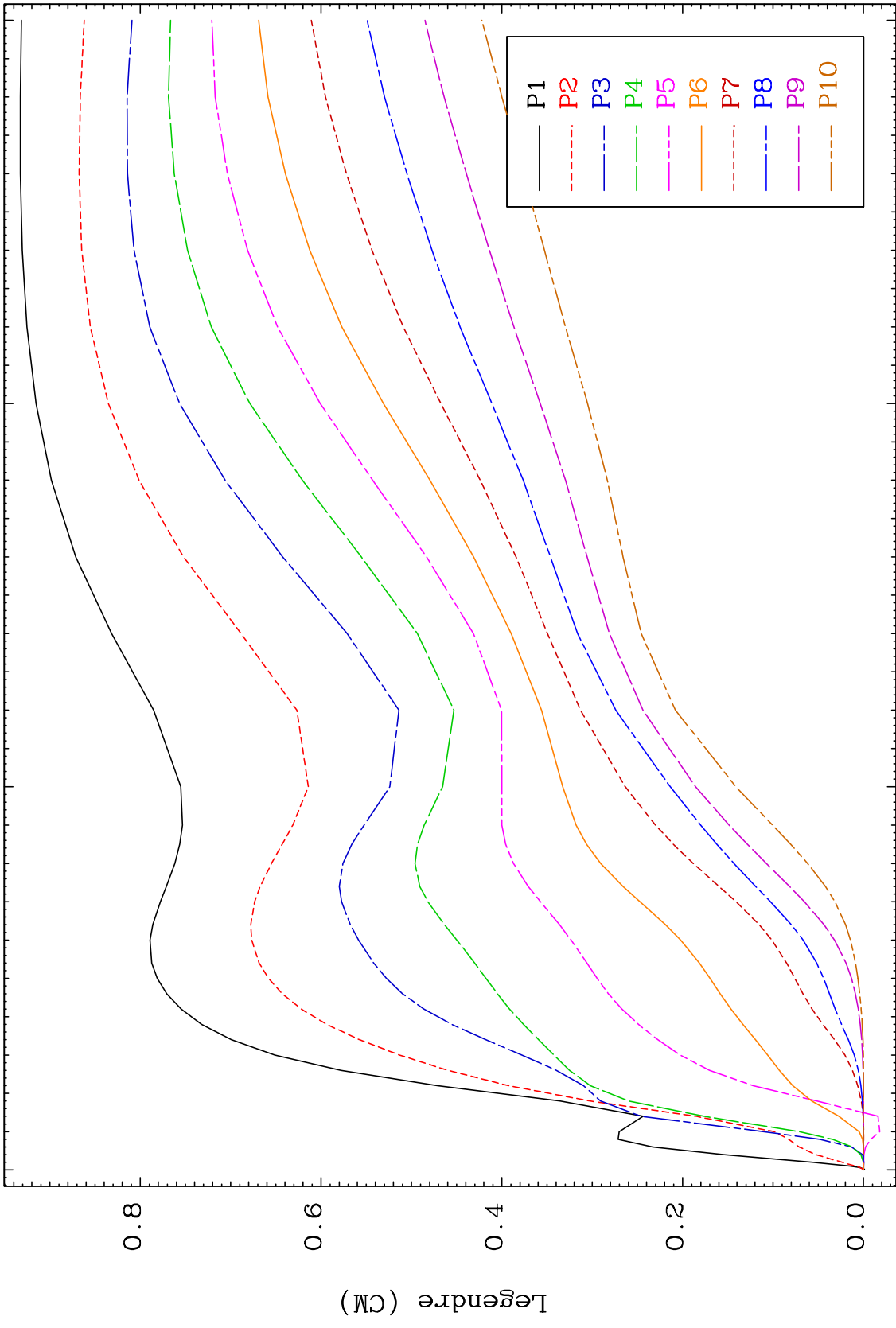




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Elastic Legendre Coefficients

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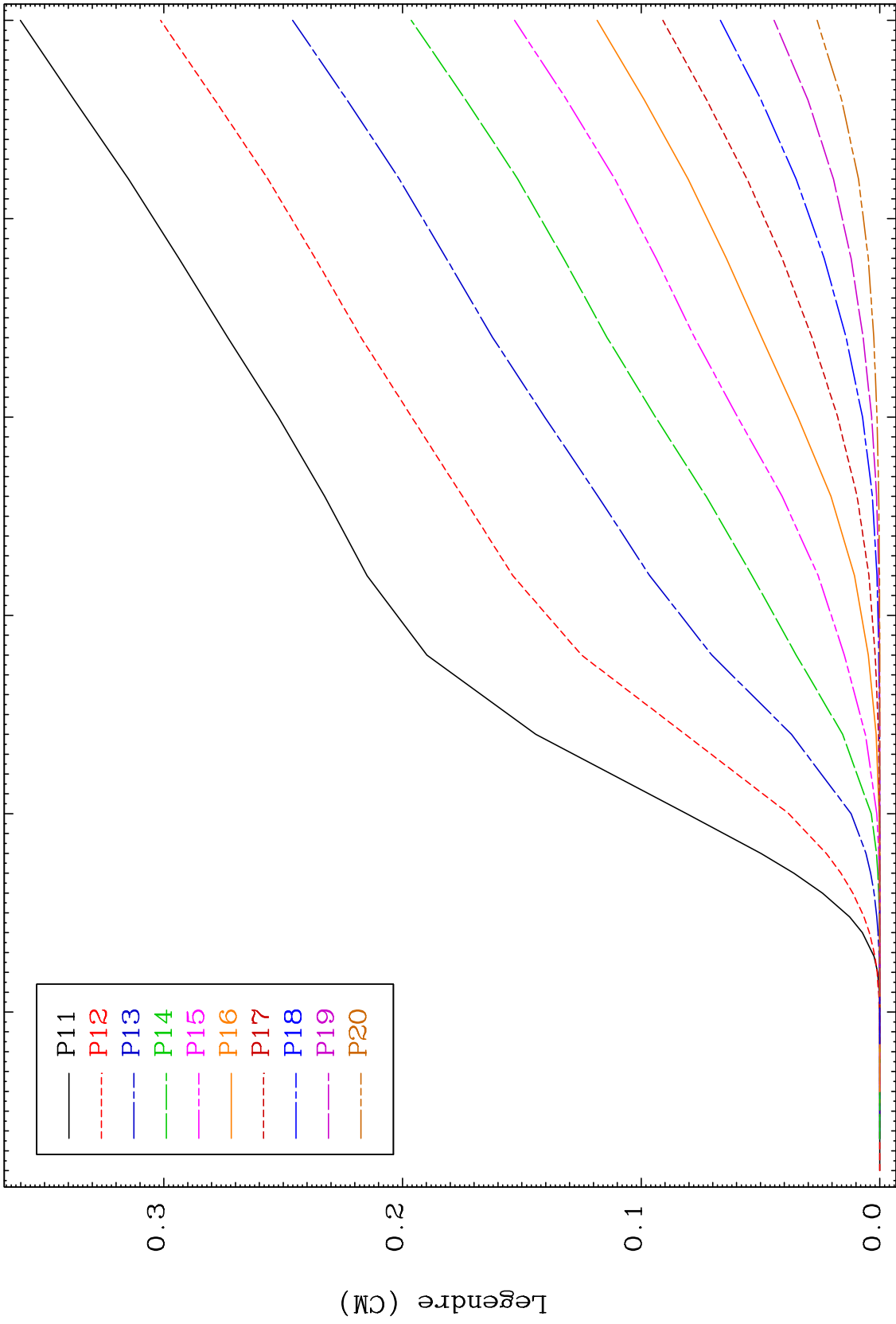
Incident Energy (MeV)

89-Ac-224

MAT 8922

Elastic Legendre Coefficients

89-Ac-224



19

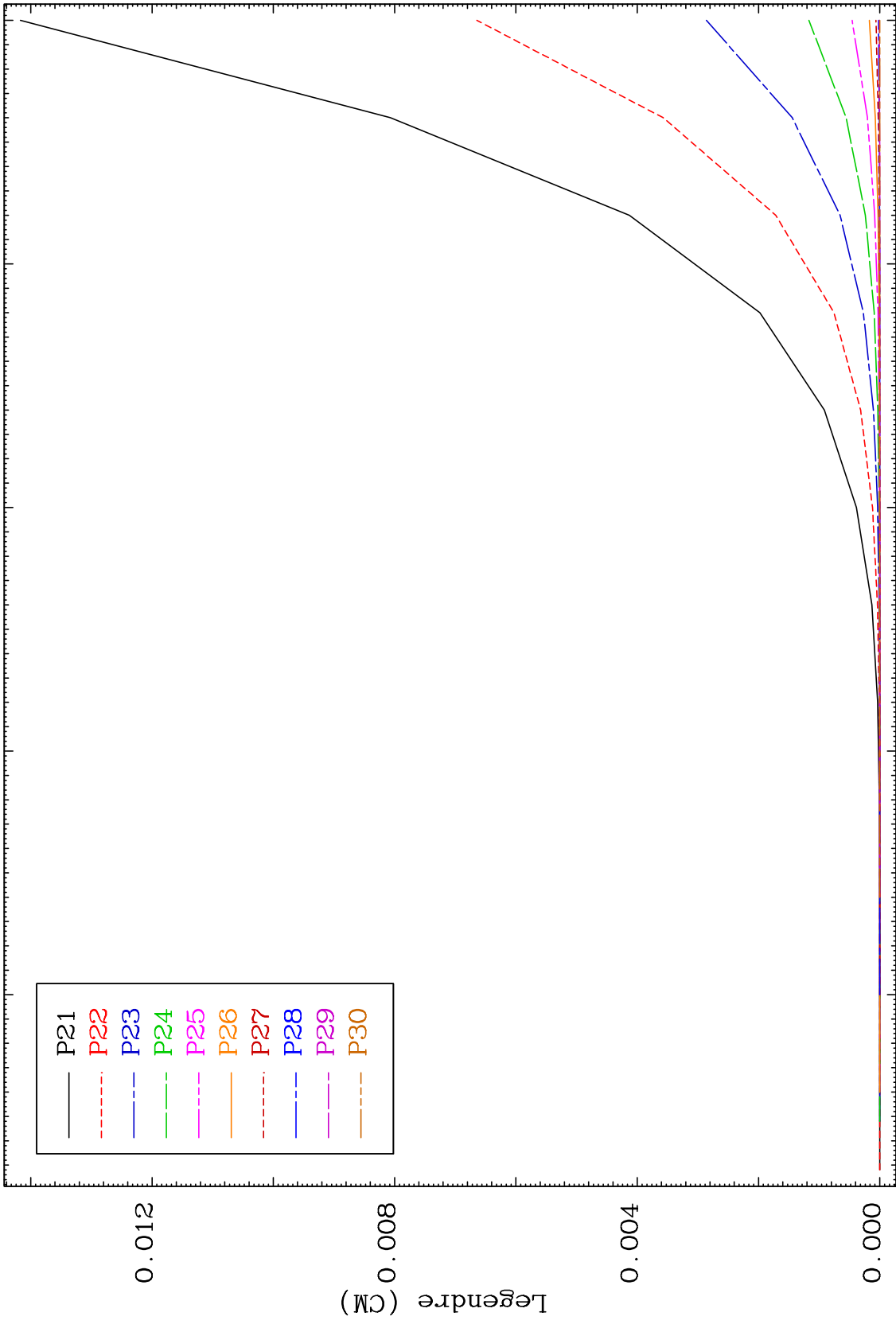
Incident Energy (MeV)

89-Ac-224

MAT 8922

Elastic Legendre Coefficients

89-Ac-224



20

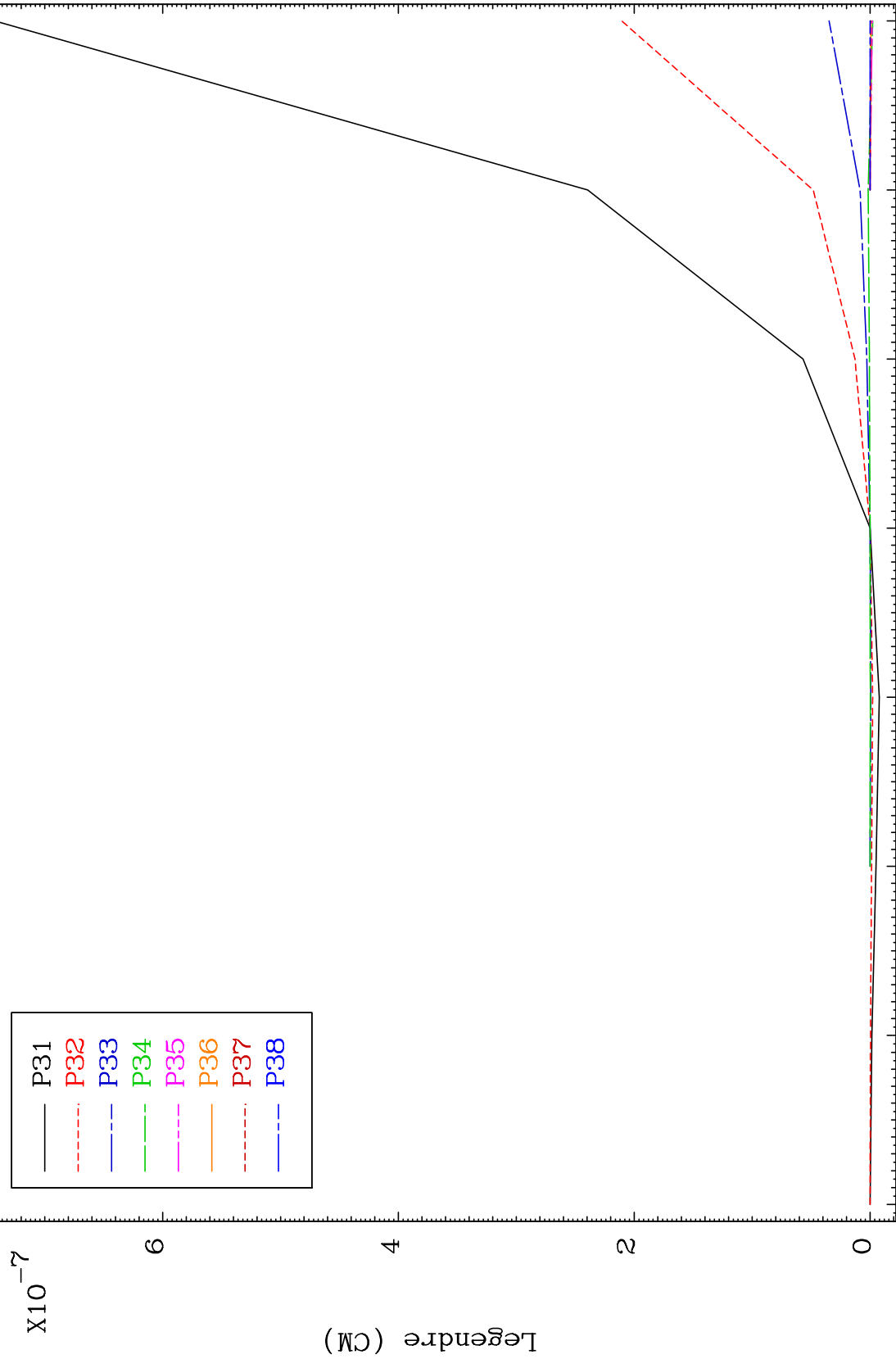
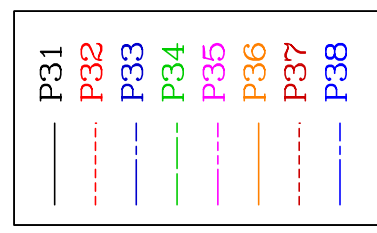
Incident Energy (MeV)

89-Ac-224

MAT 8922

Elastic Legendre Coefficients

89-Ac-224



21

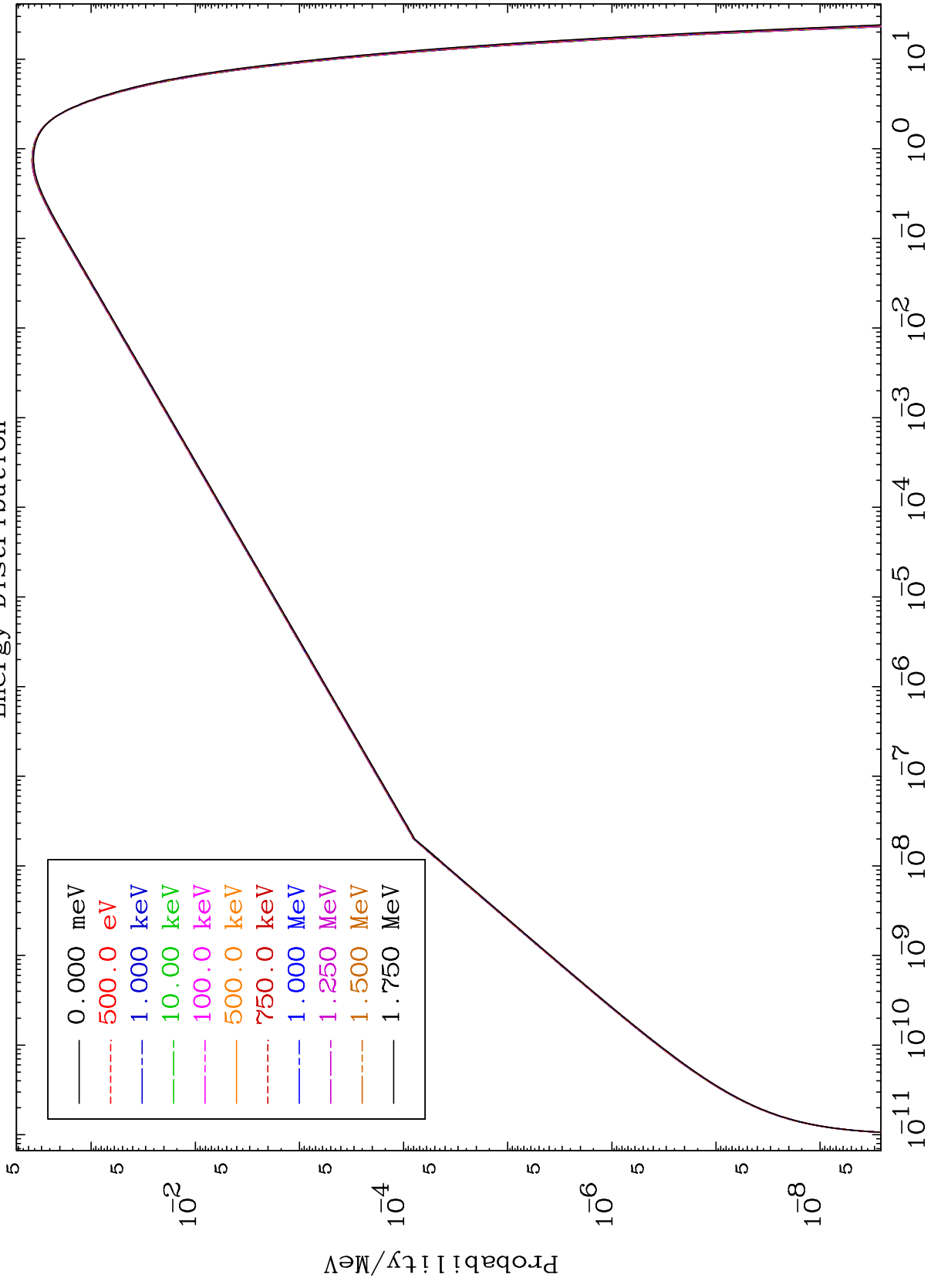
Incident Energy (MeV)

89-Ac-224

MAT 8922

Fission Energy Distribution

89-Ac-224



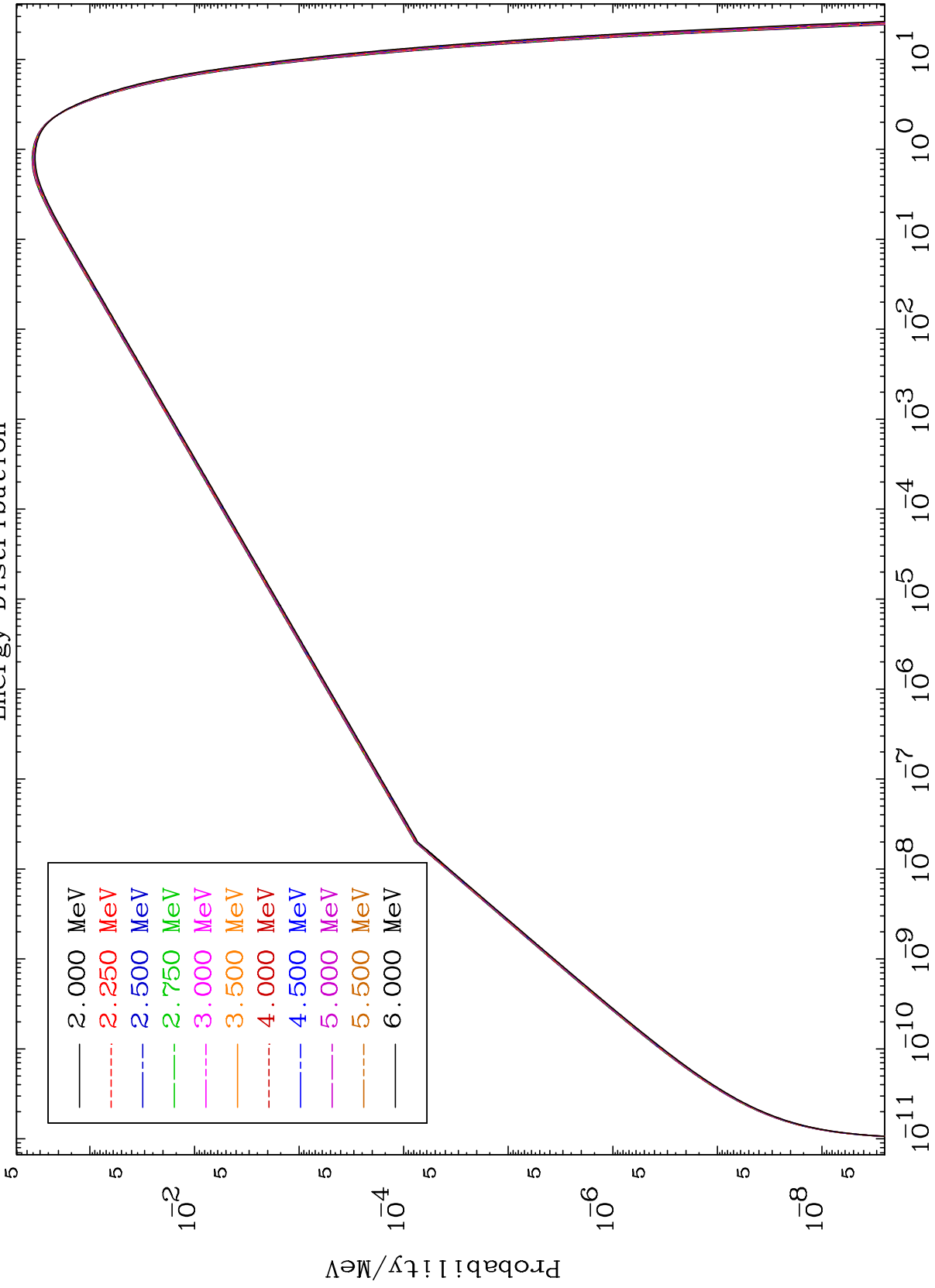
- 0.000 meV
- 500.0 eV
- 1.000 keV
- 10.00 keV
- 100.0 keV
- 500.0 keV
- 750.0 keV
- 1.000 MeV
- 1.250 MeV
- 1.500 MeV
- 1.750 MeV

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Fission Energy Distribution

89-Ac-224



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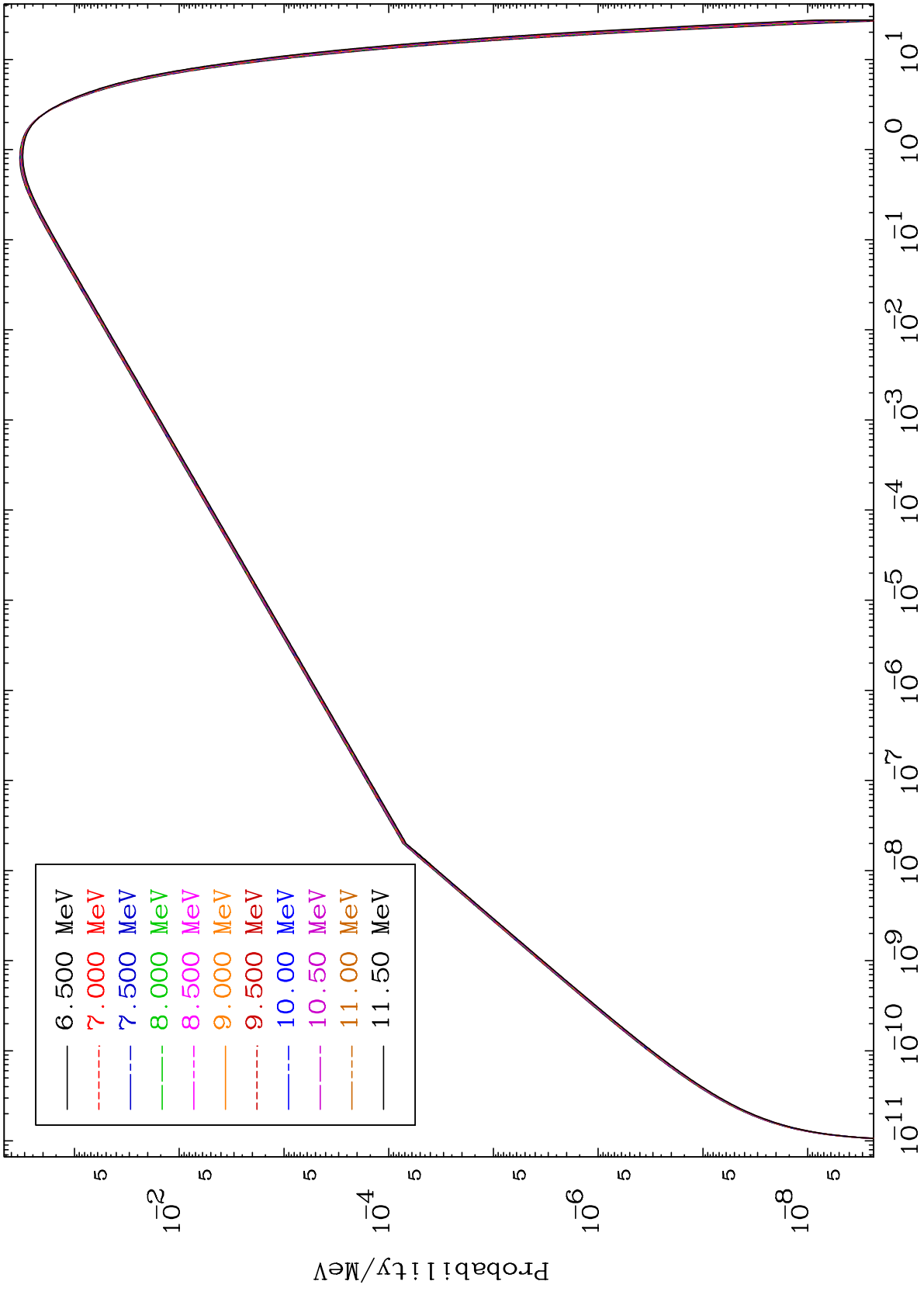
Secondary Energy (MeV)

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MAT 8922

Fission Energy Distribution

89-Ac-224

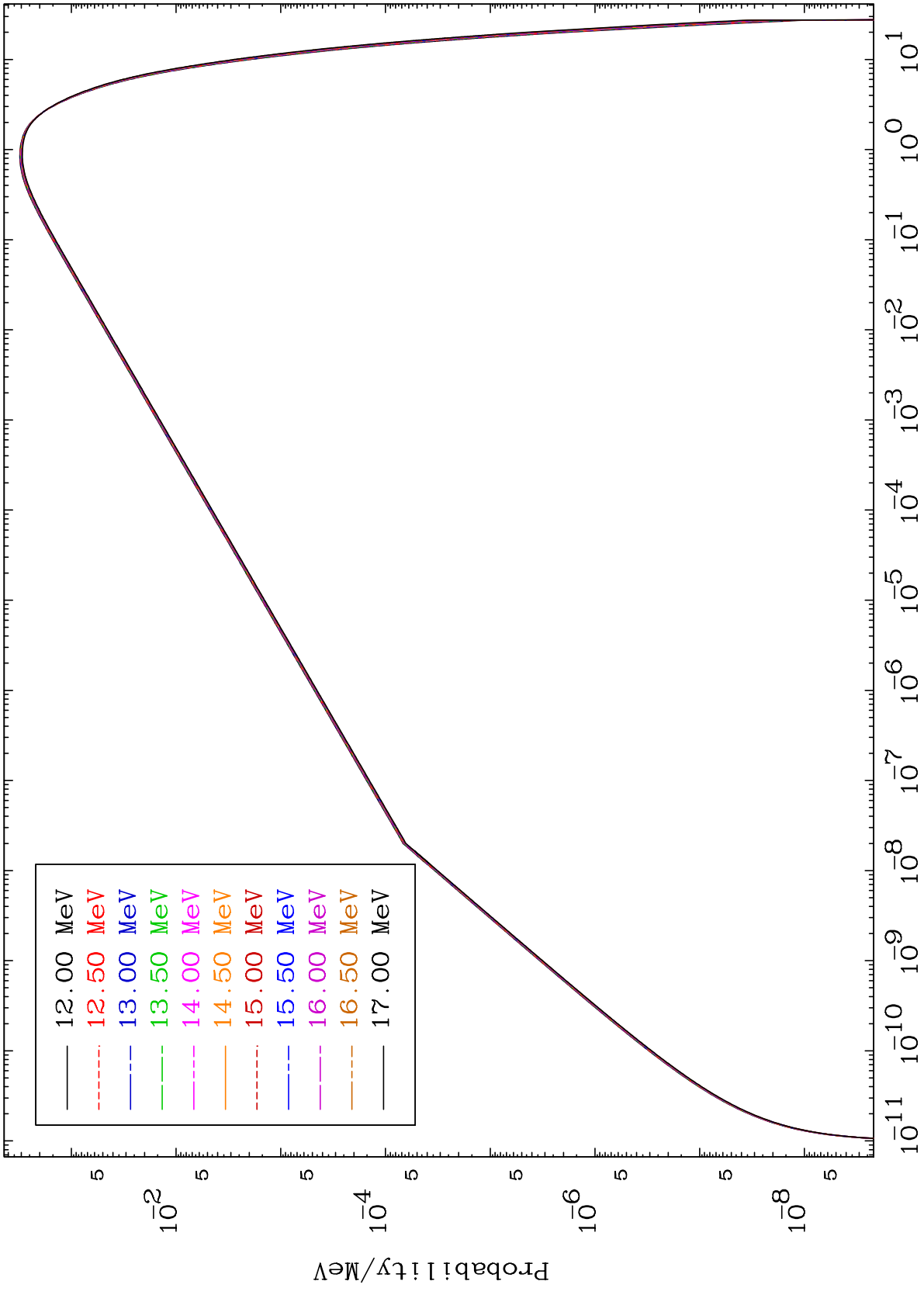


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Fission Energy Distribution

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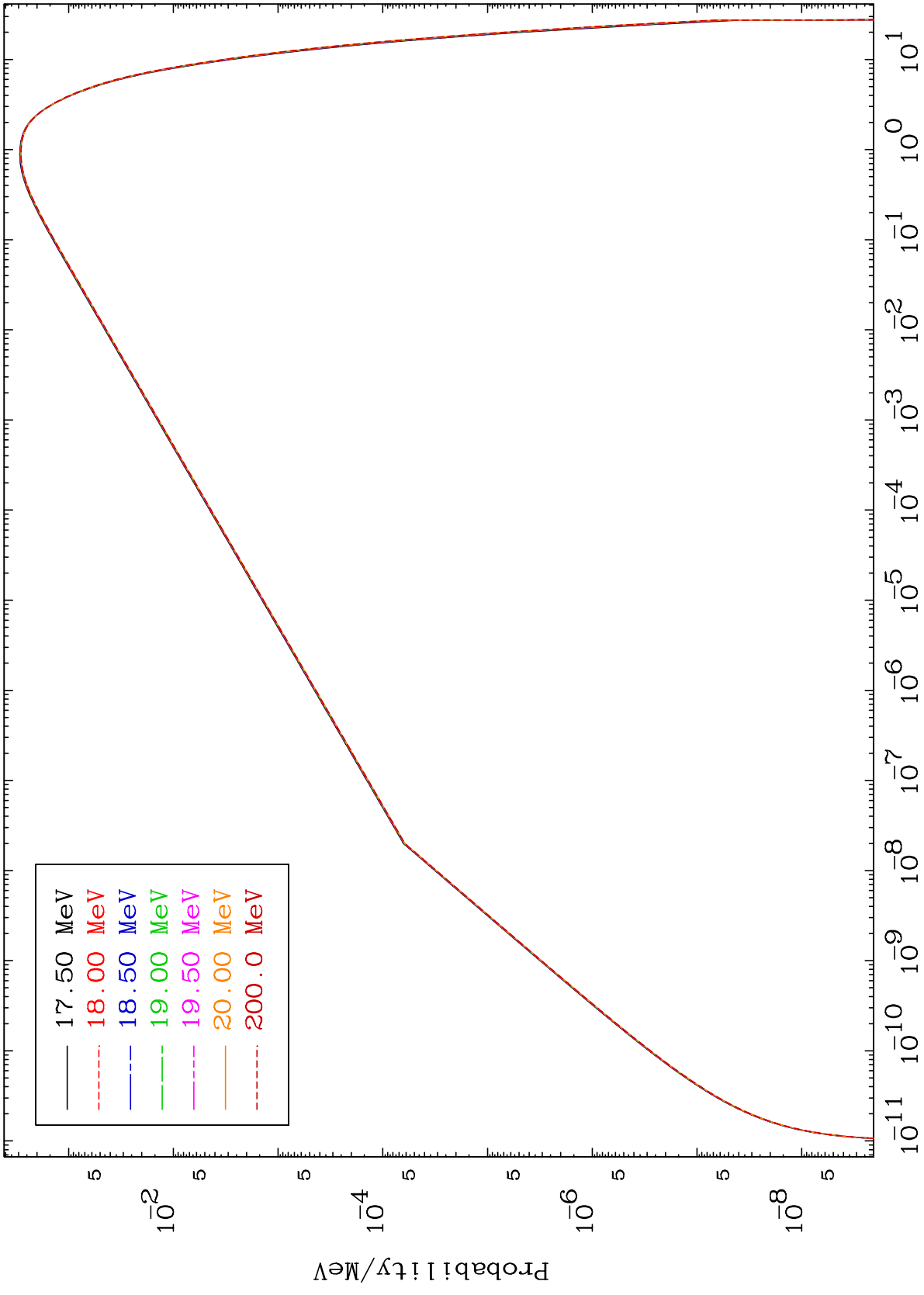
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Secondary Energy (MeV)

MAT 8922

Fission Energy Distribution

89-Ac-224

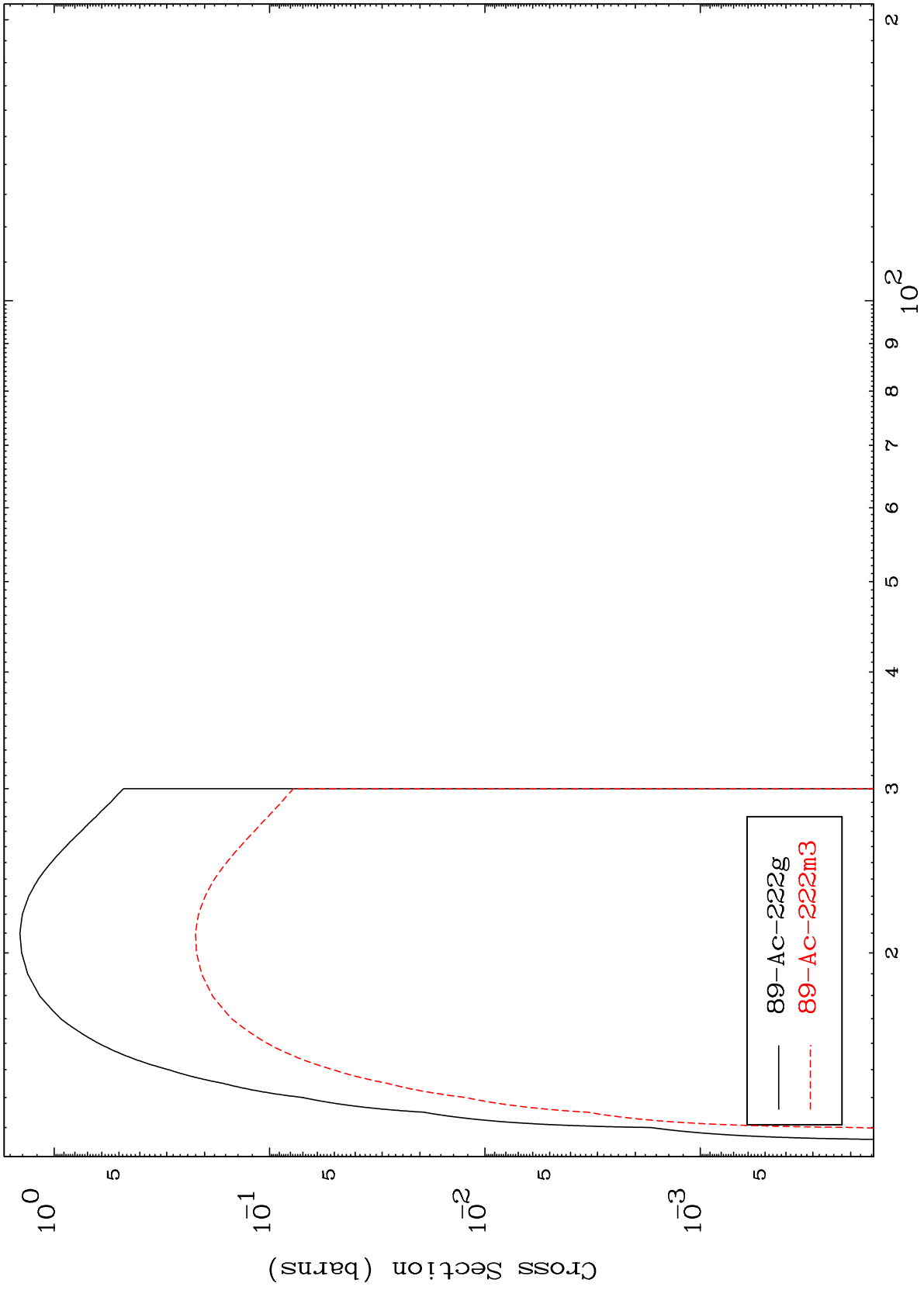


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(n,3n)
Radionuclide Production Cross Section



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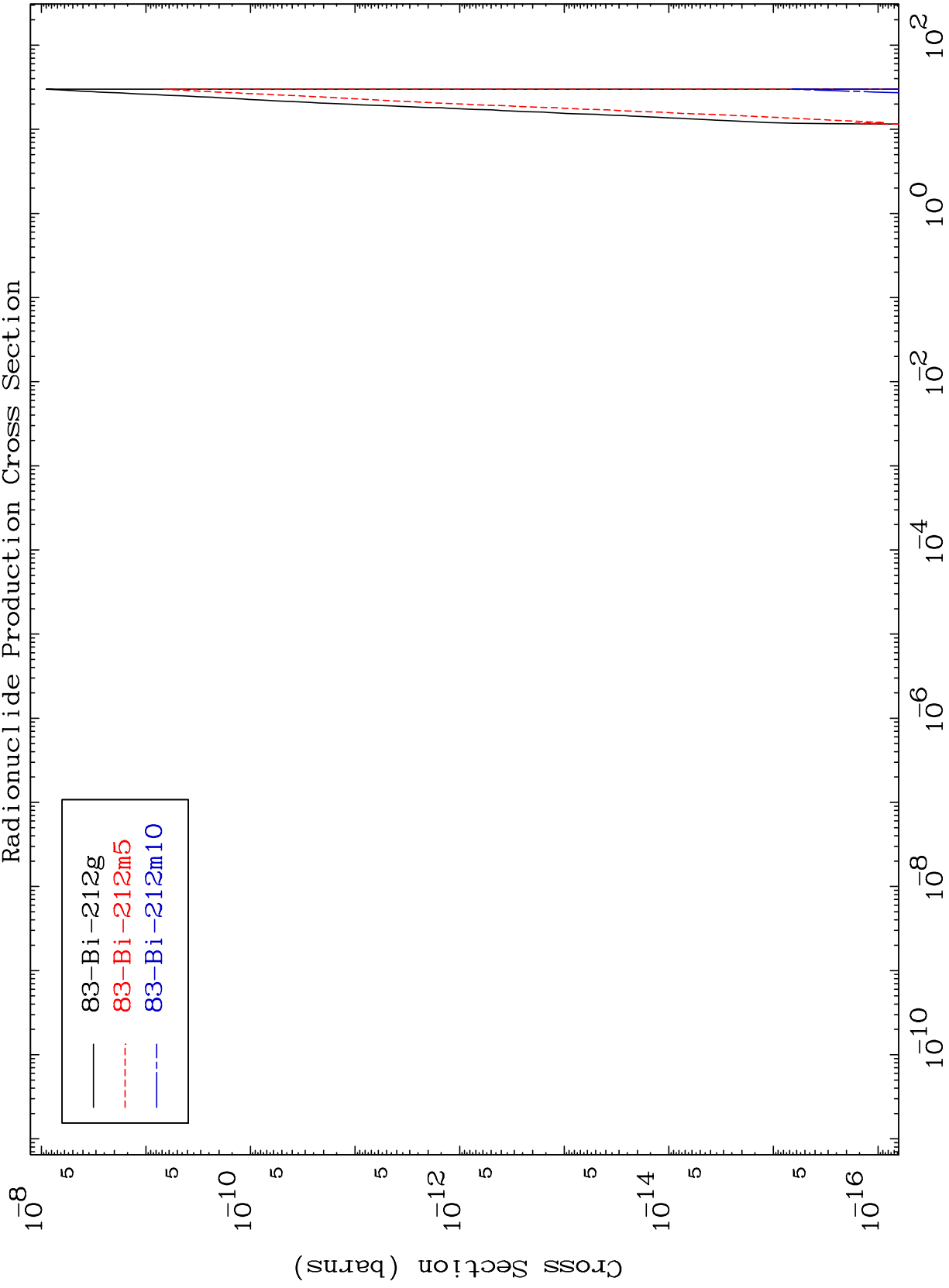
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MAT 8922

(n,n') 3 α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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