

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

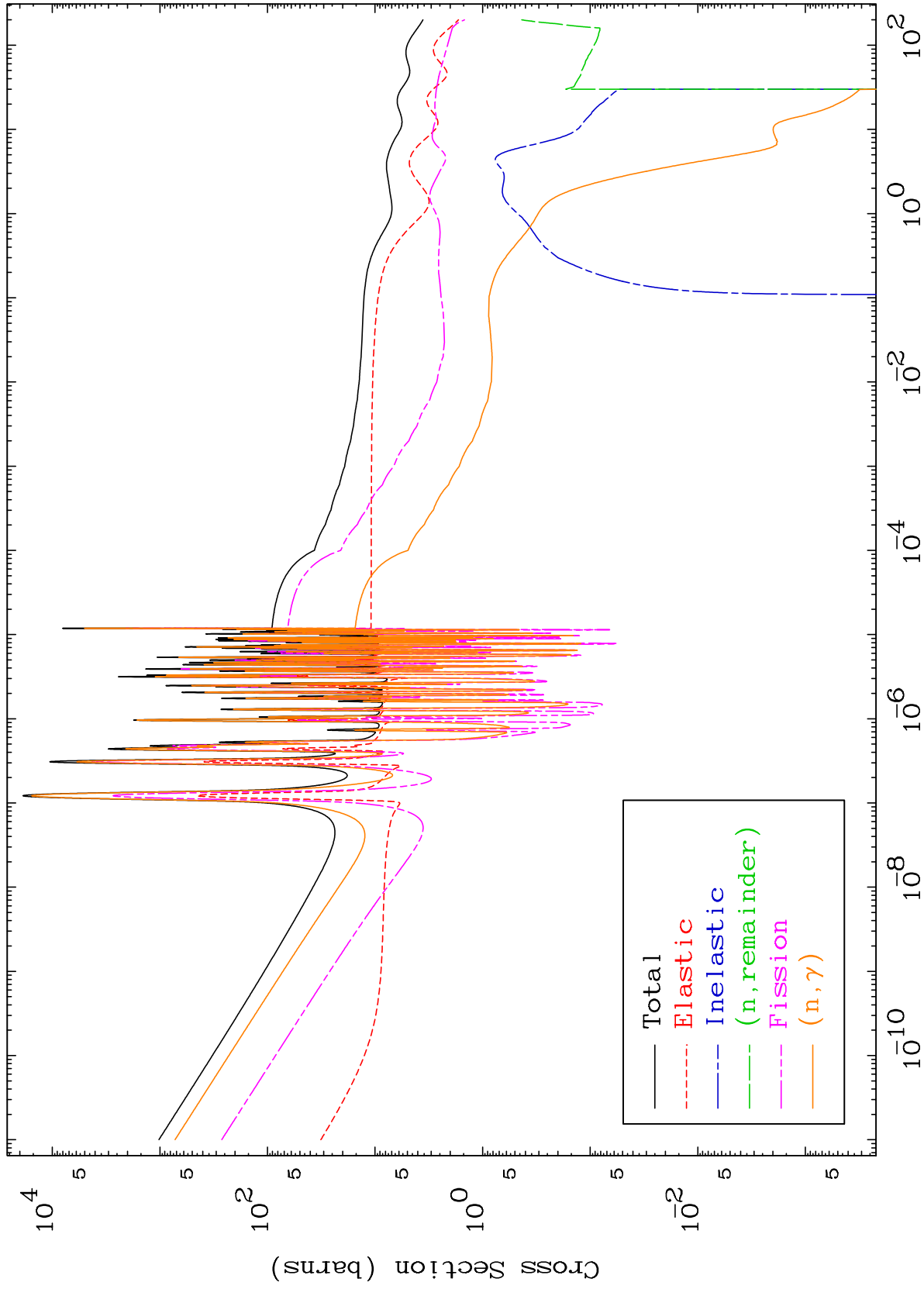
Press Mouse Button to Start

MAT 9722

Major

293 Kelvin Cross Sections

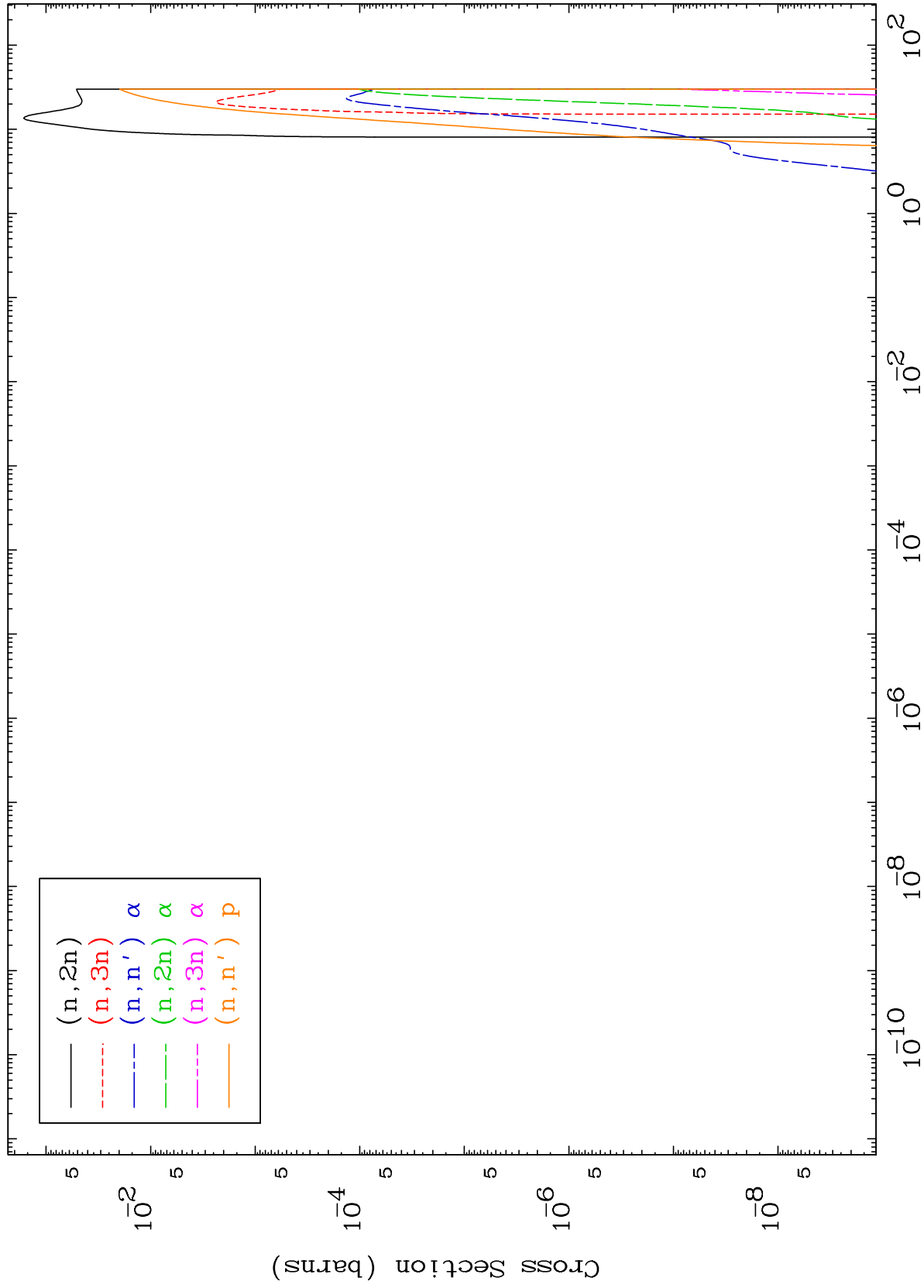
97-Bk-239



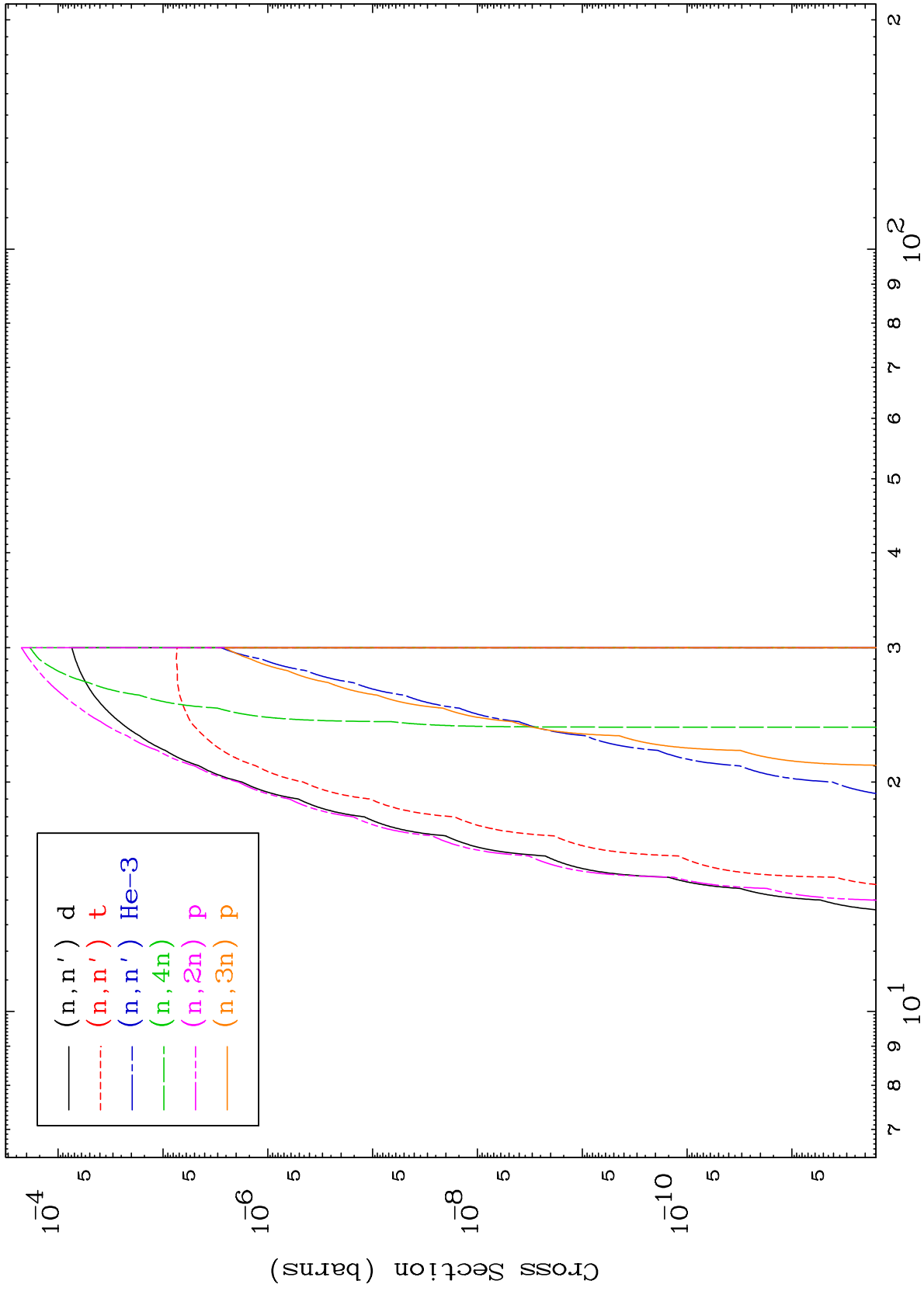
MAT 9722

Neutron Production  
293 Kelvin Cross Sections

97-Bk-239



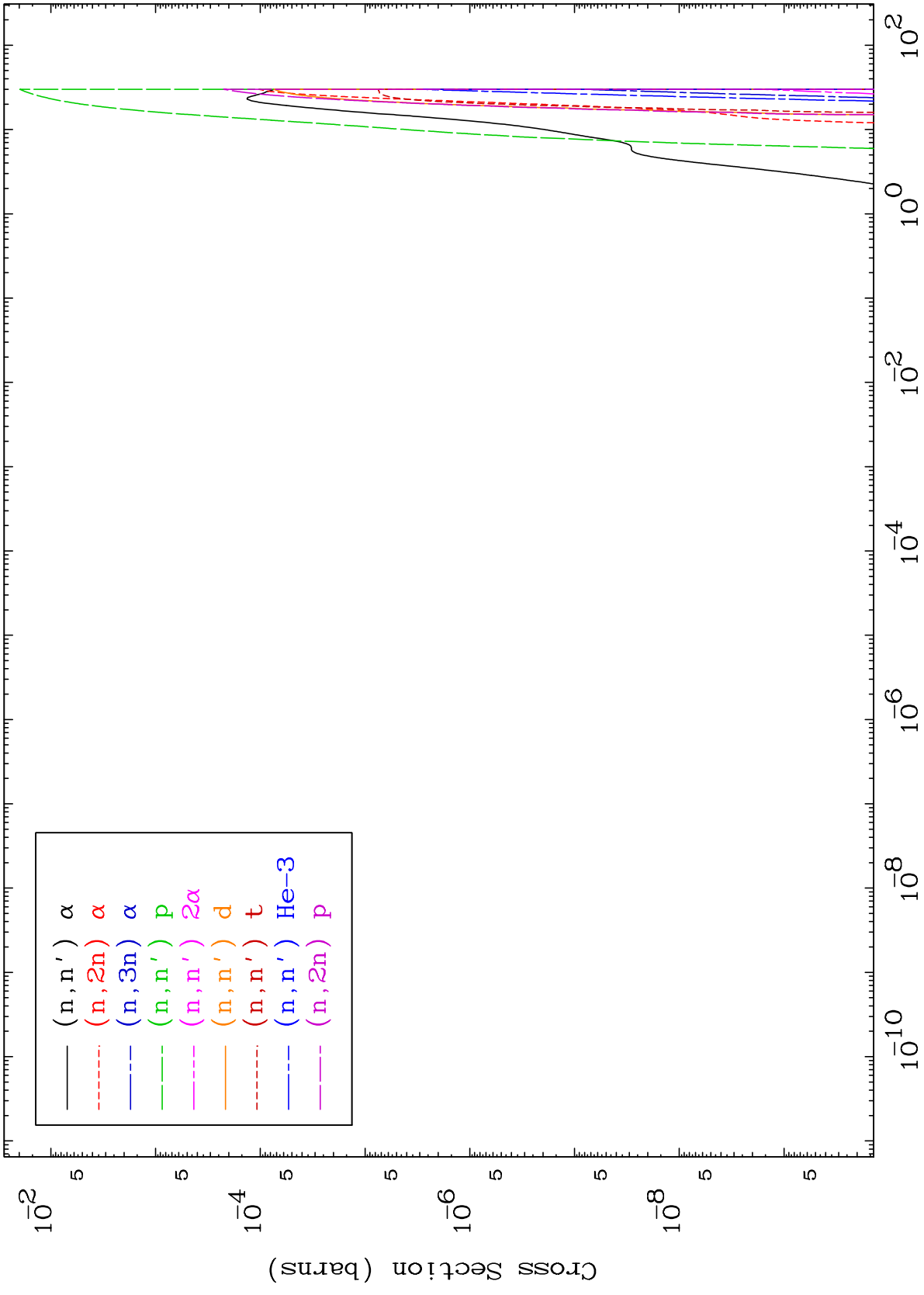
97-Bk-239



MAT 9722

Charged Particle  
293 Kelvin Cross Sections

97-Bk-239

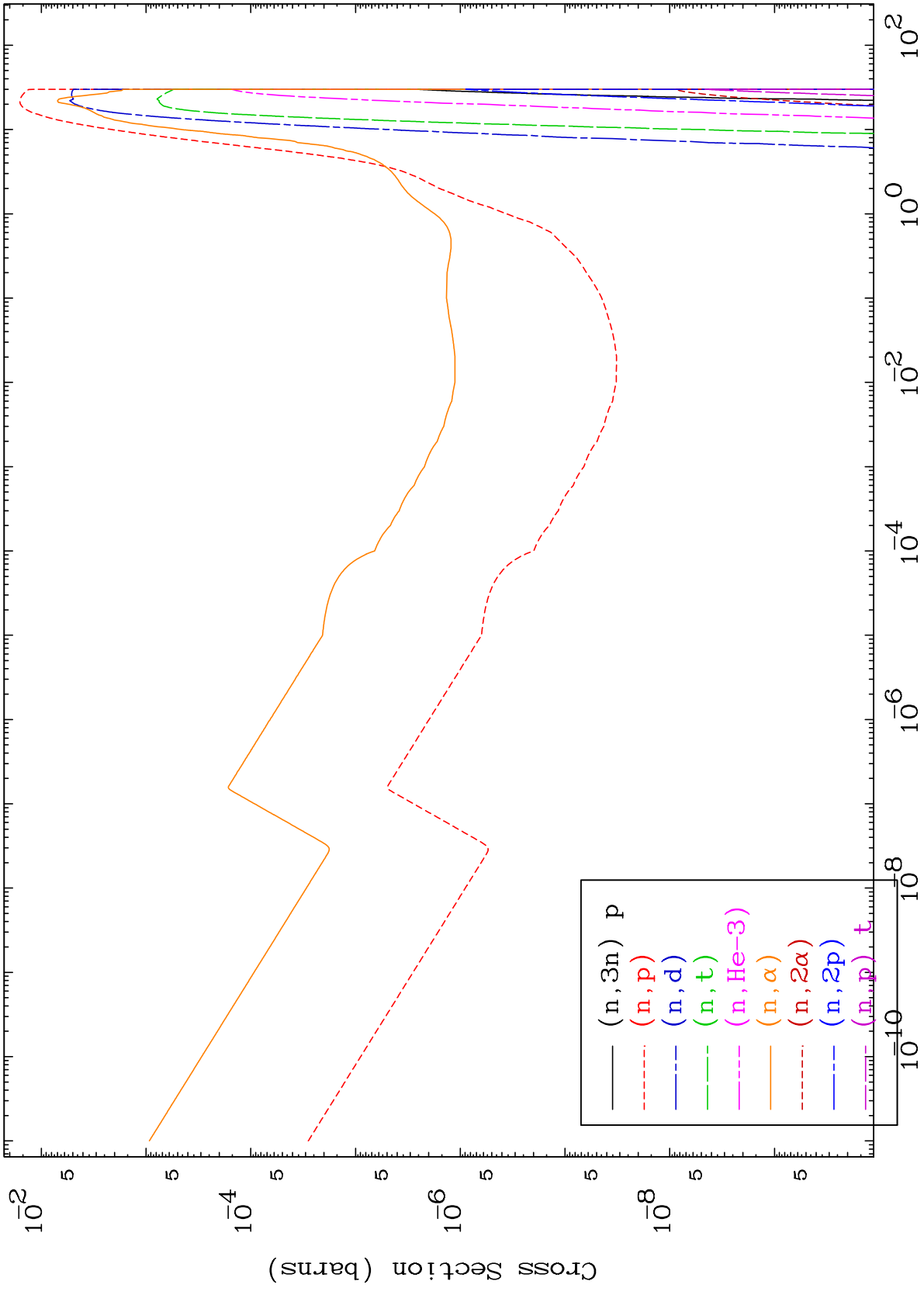


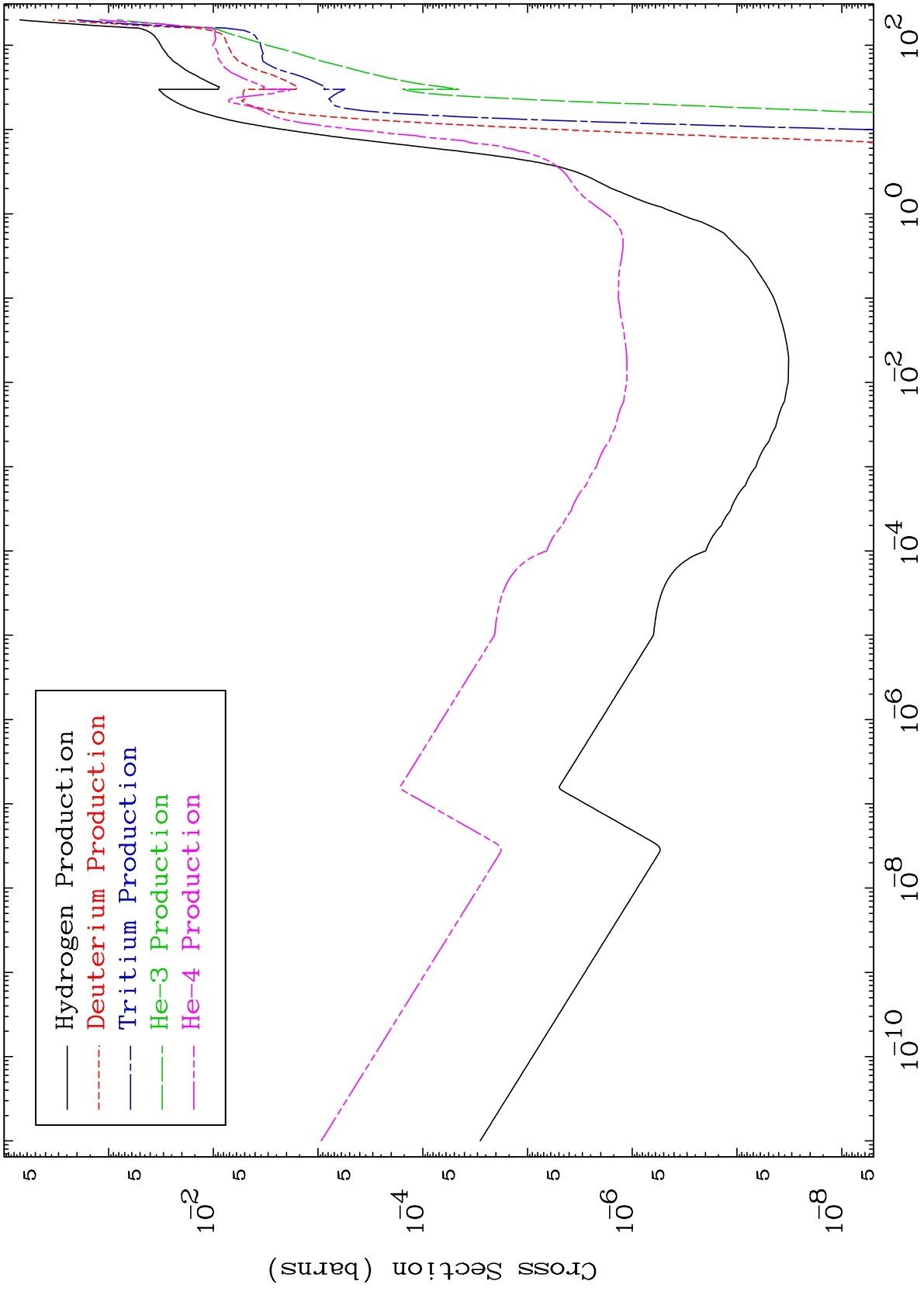
97-Bk-239

MAT 9722

Charged Particle  
293 Kelvin Cross Sections

97-Bk-239

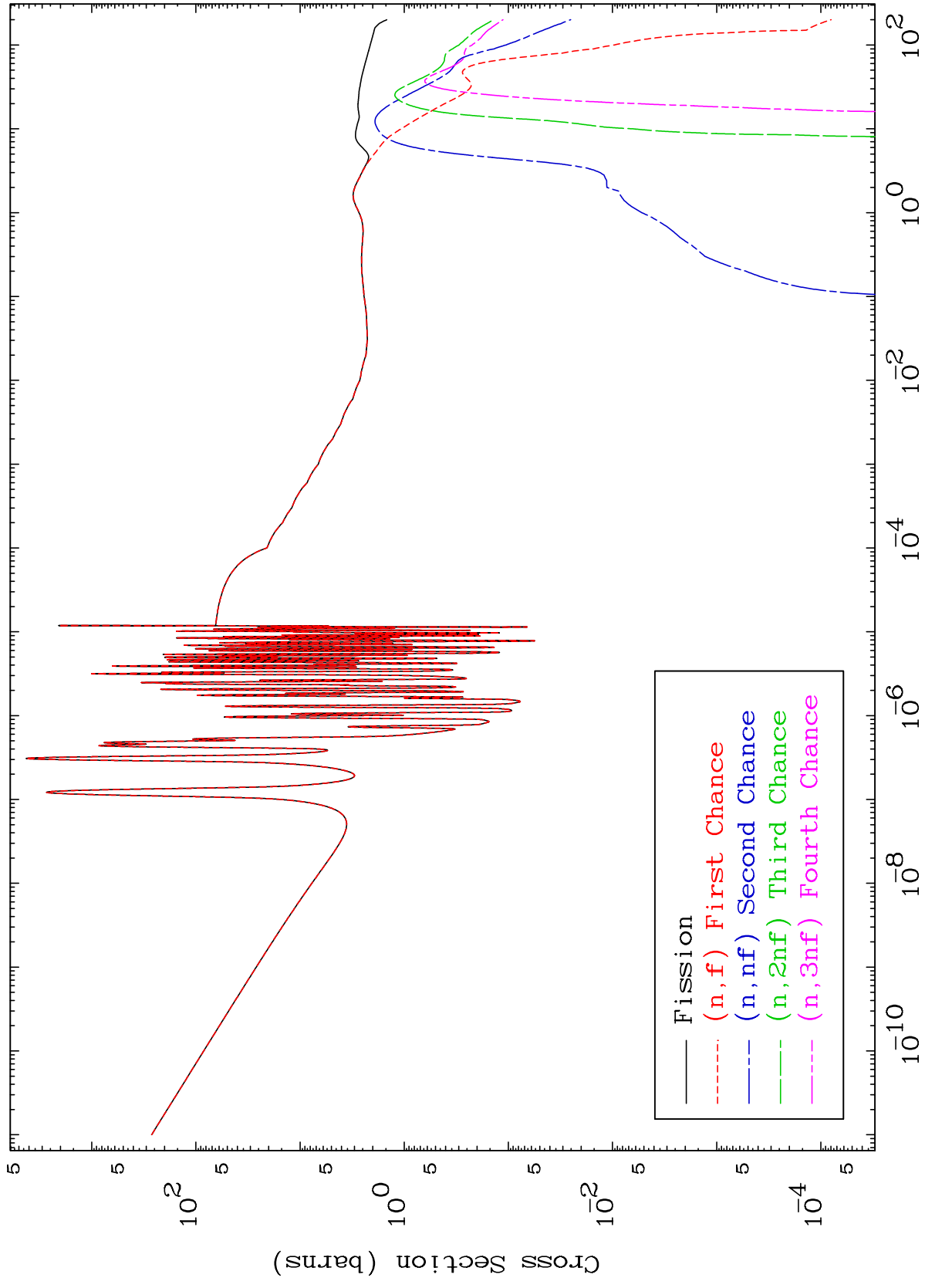




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Fission  
293 Kelvin Cross Sections

97-Bk-239



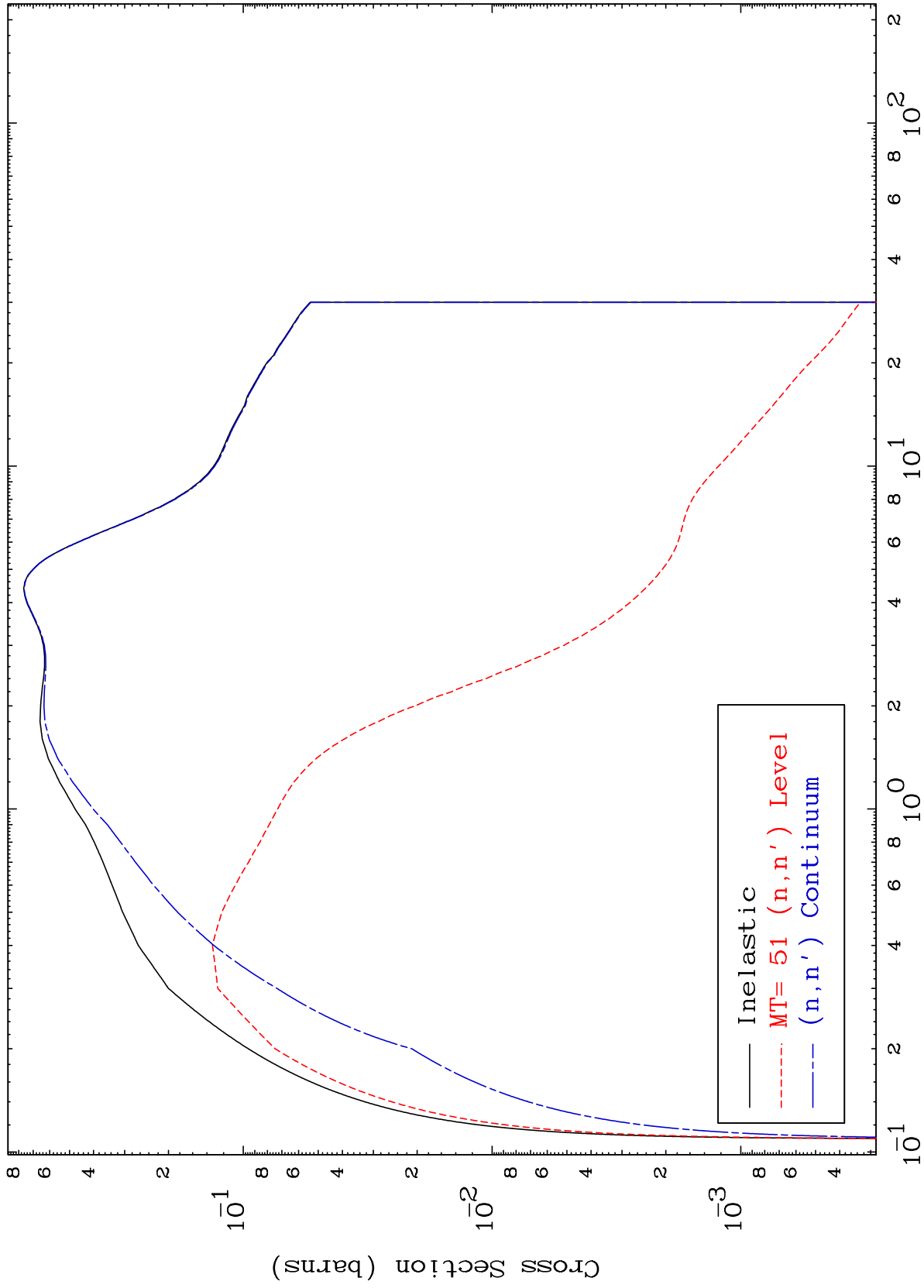
97-Bk-239



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(n,n') Level  
293 Kelvin Cross Sections

97-Bk-239



97-Bk-239

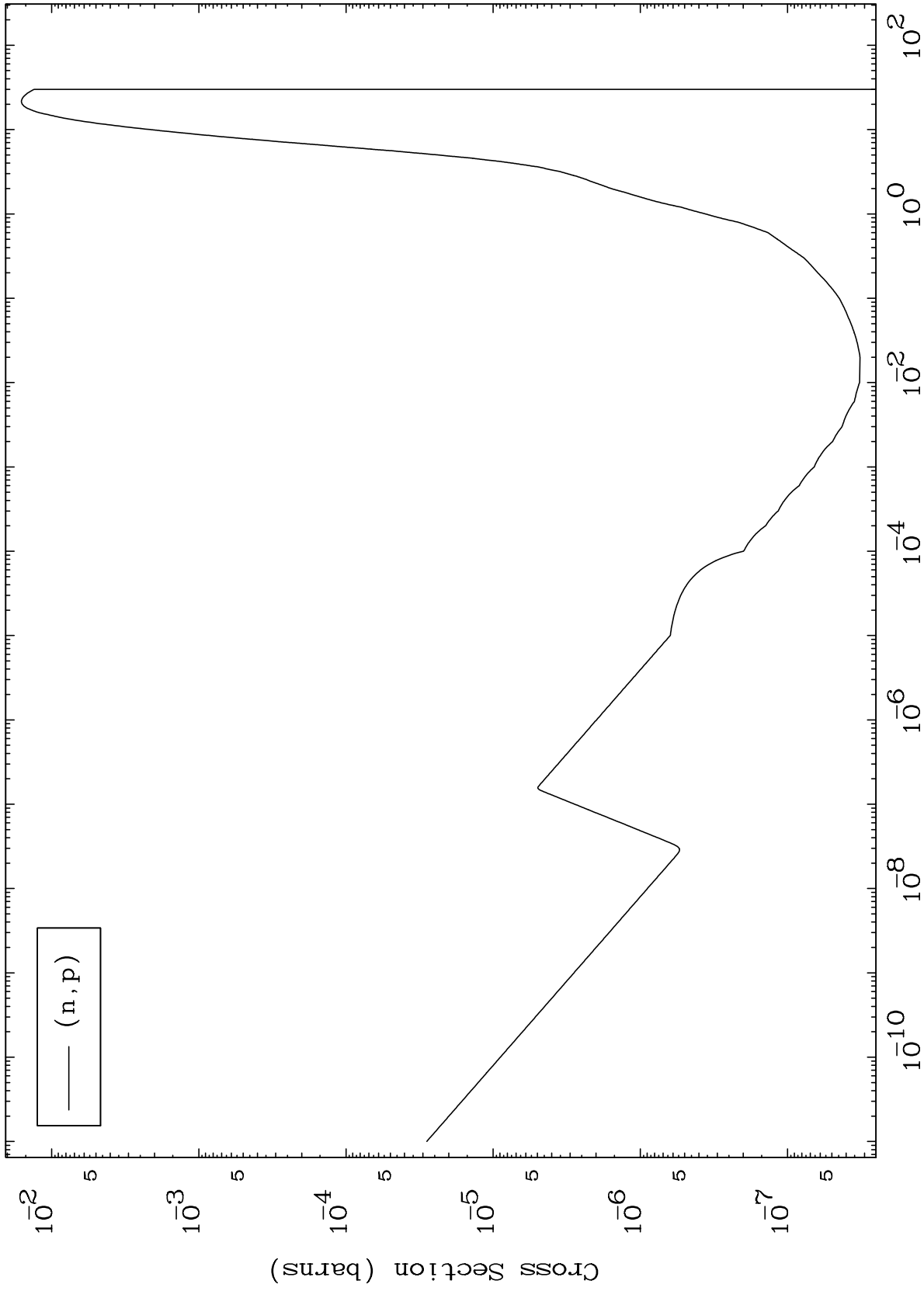
Incident Energy (MeV)

8

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(n,p) Levels  
293 Kelvin Cross Sections

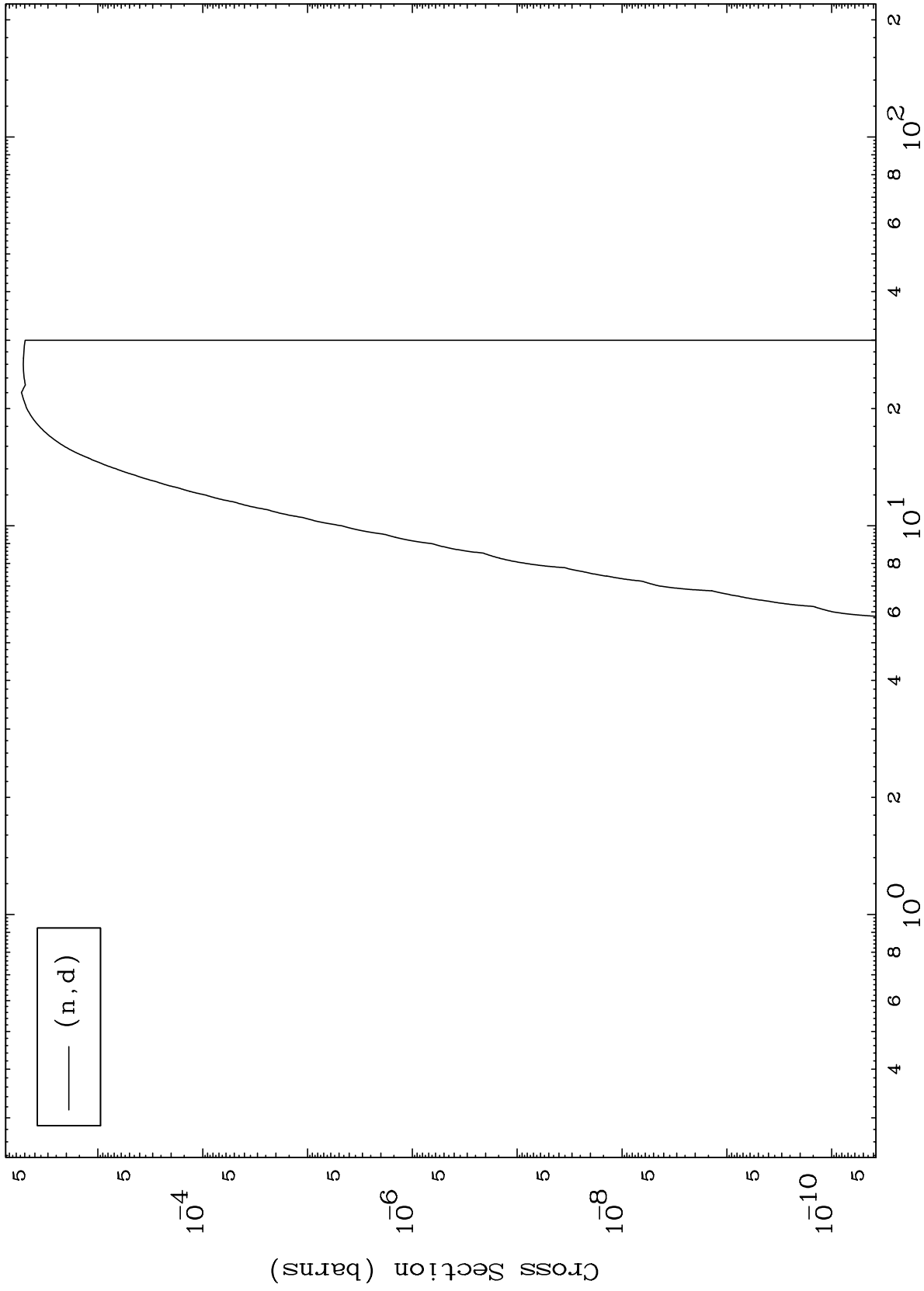
97-Bk-239



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(n,d) Levels  
293 Kelvin Cross Sections

97-Bk-239



10

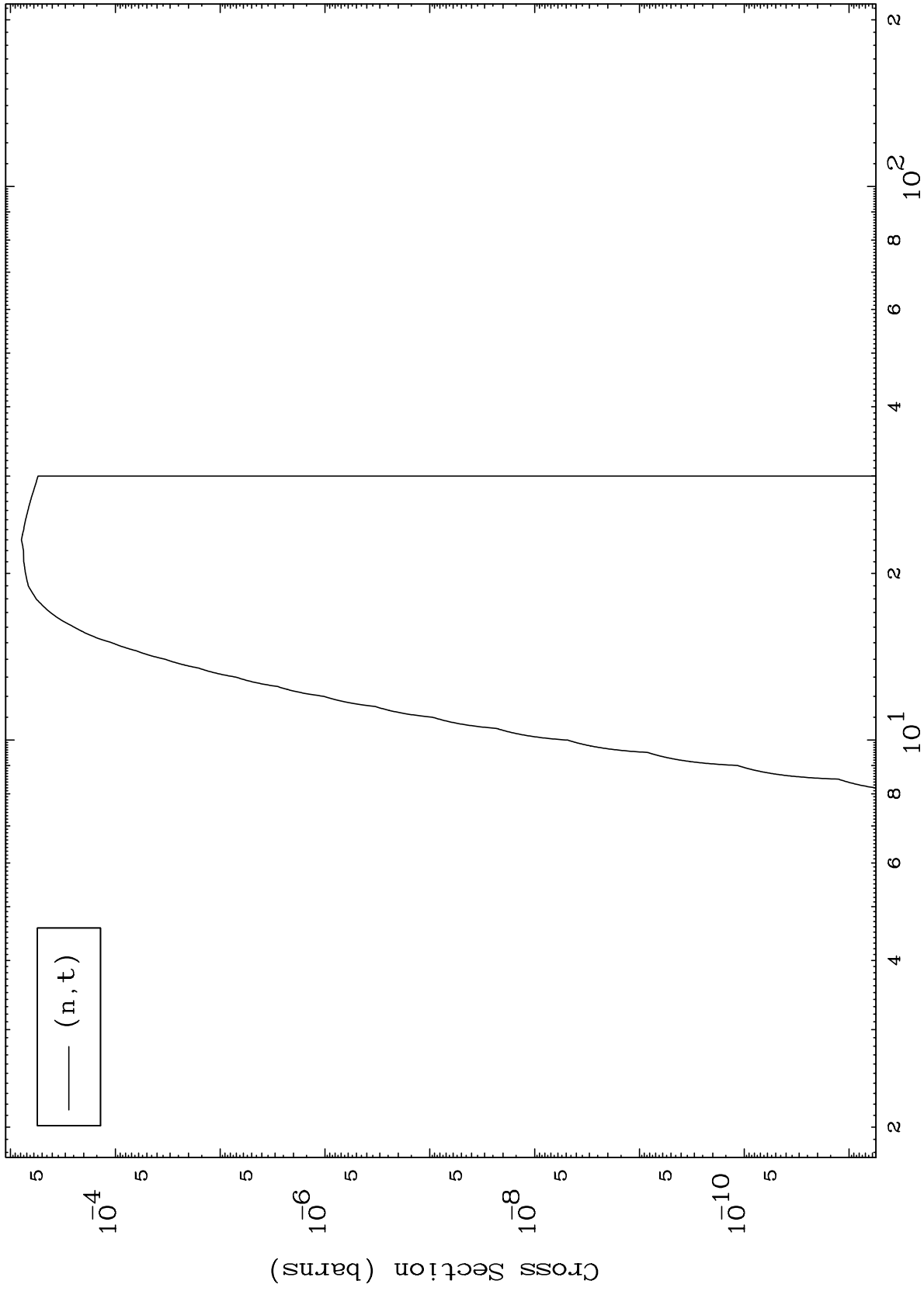
Incident Energy (MeV)

97-Bk-239

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(n,t) Levels  
293 Kelvin Cross Sections

97-Bk-239



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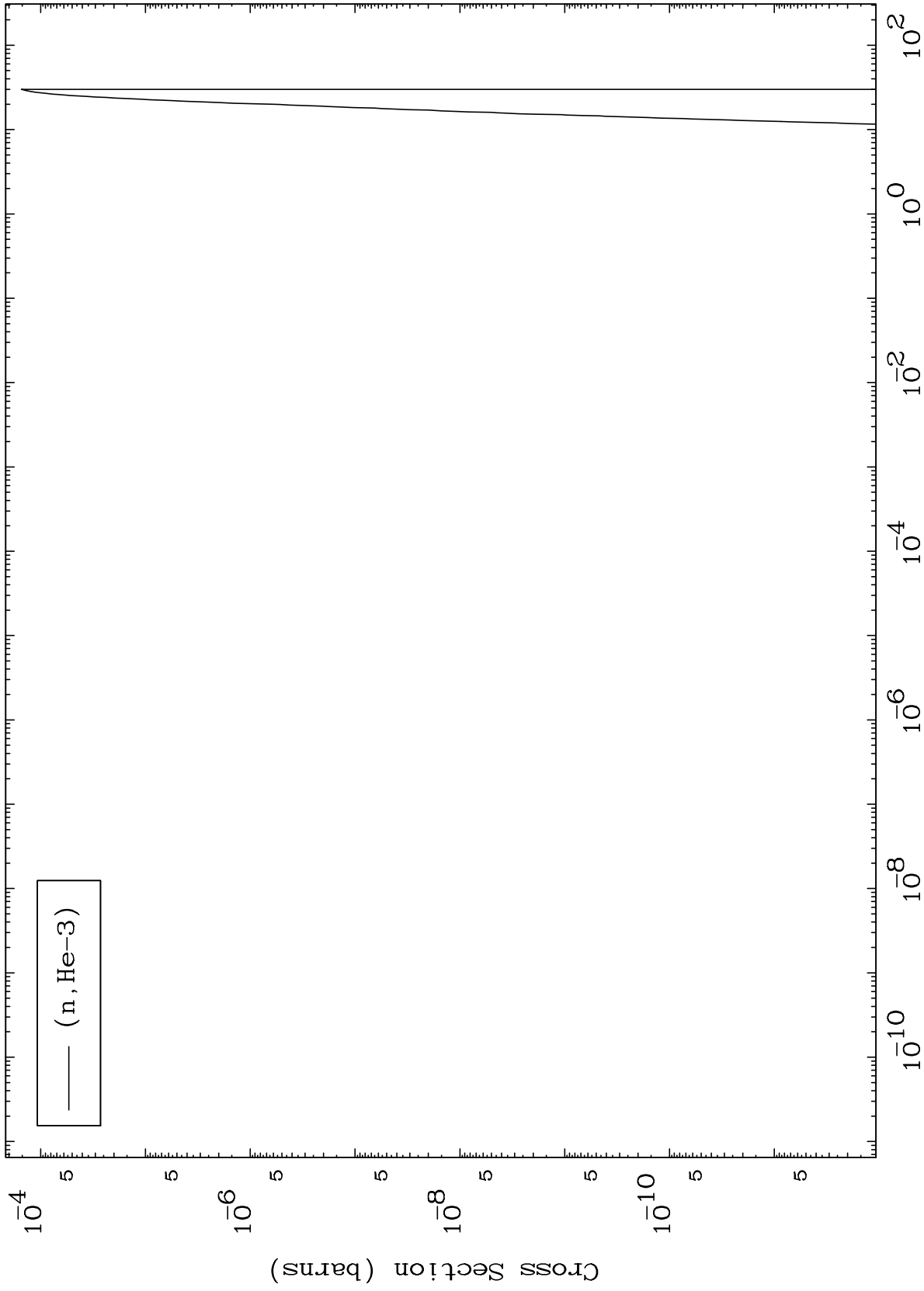
Incident Energy (MeV)

97-Bk-239

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(n,He3) Levels  
293 Kelvin Cross Sections

97-Bk-239



(n, He-3)

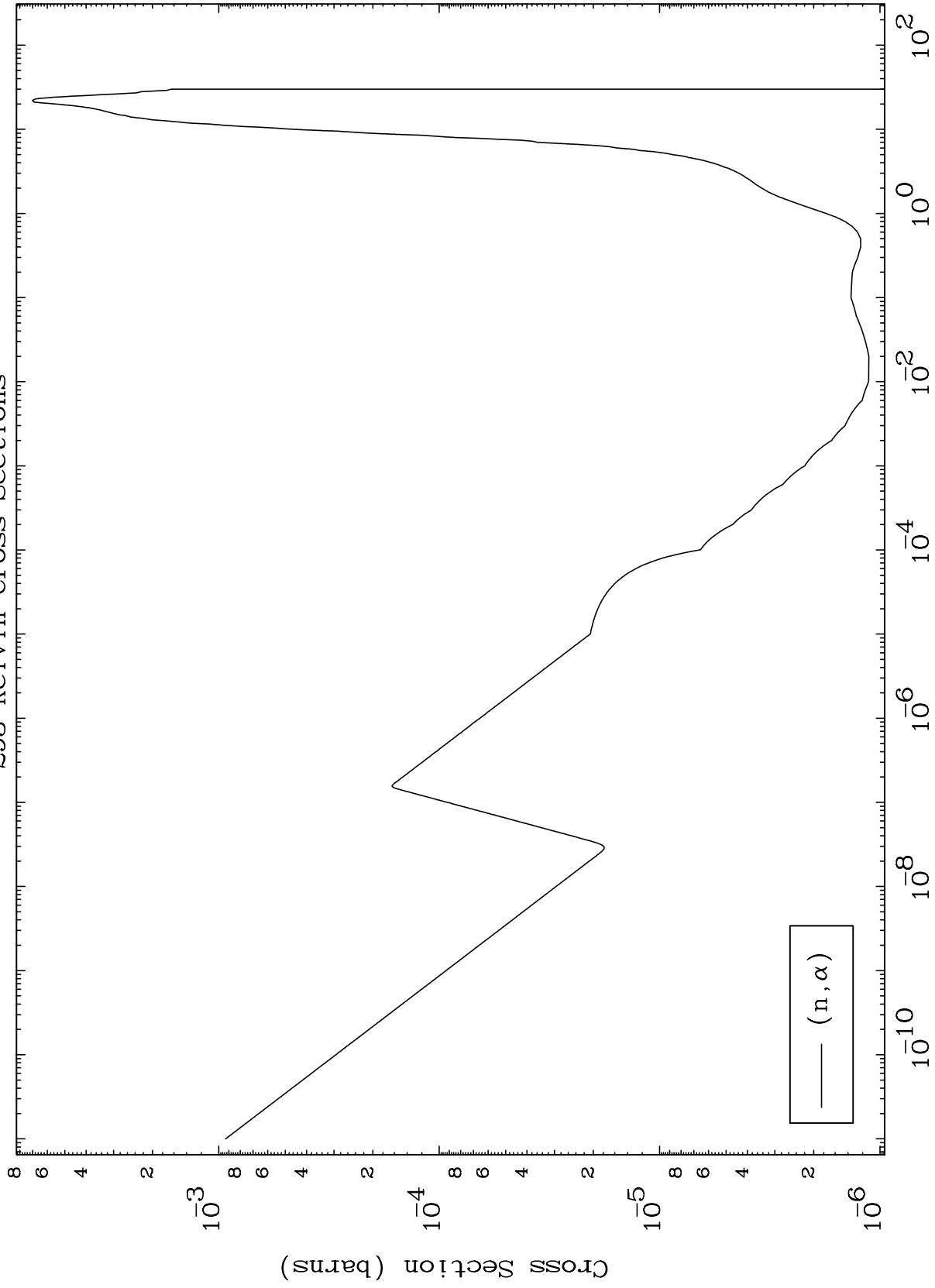
97-Bk-239

Incident Energy (MeV)

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(n,α) Levels  
293 Kelvin Cross Sections

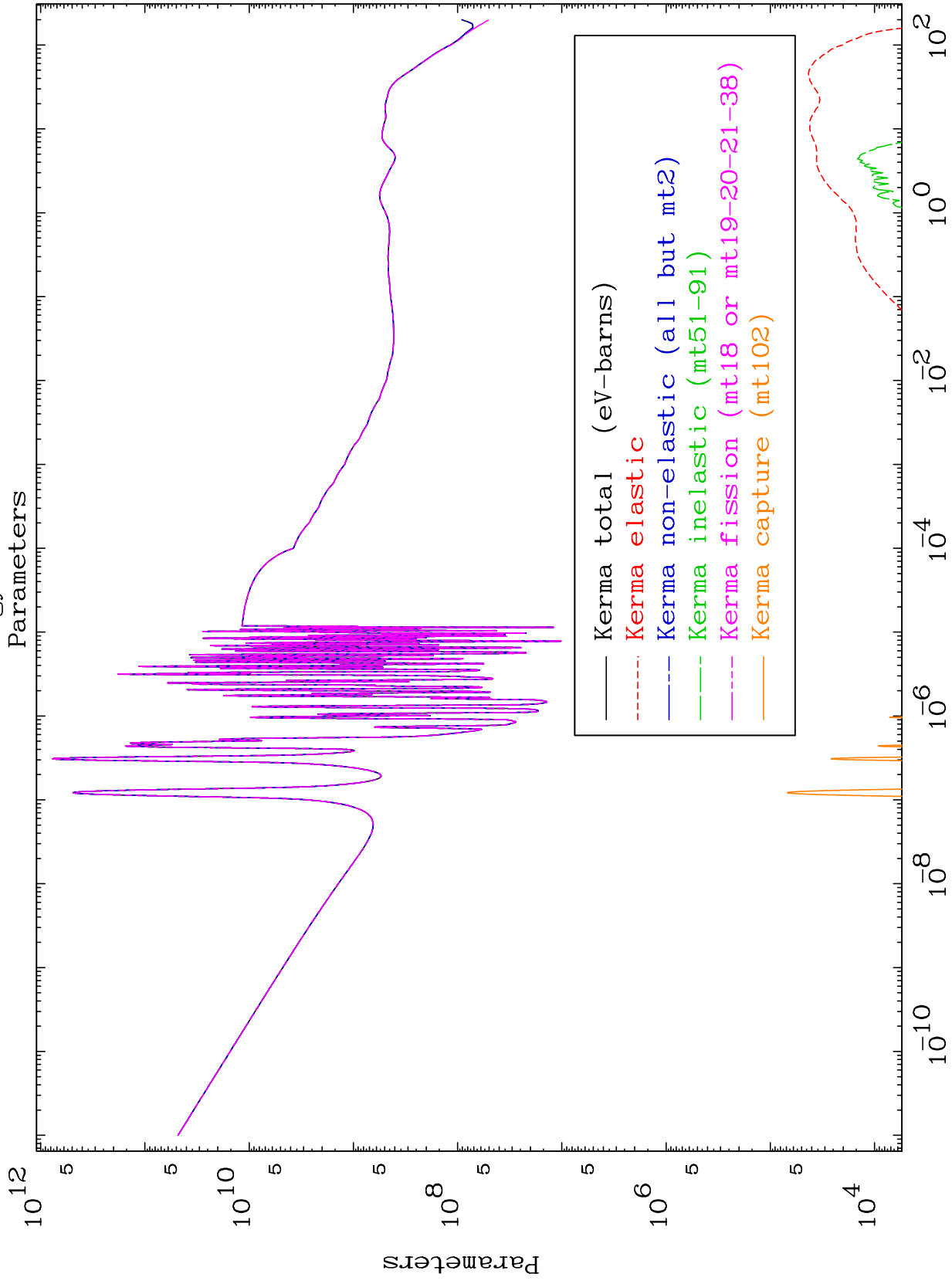
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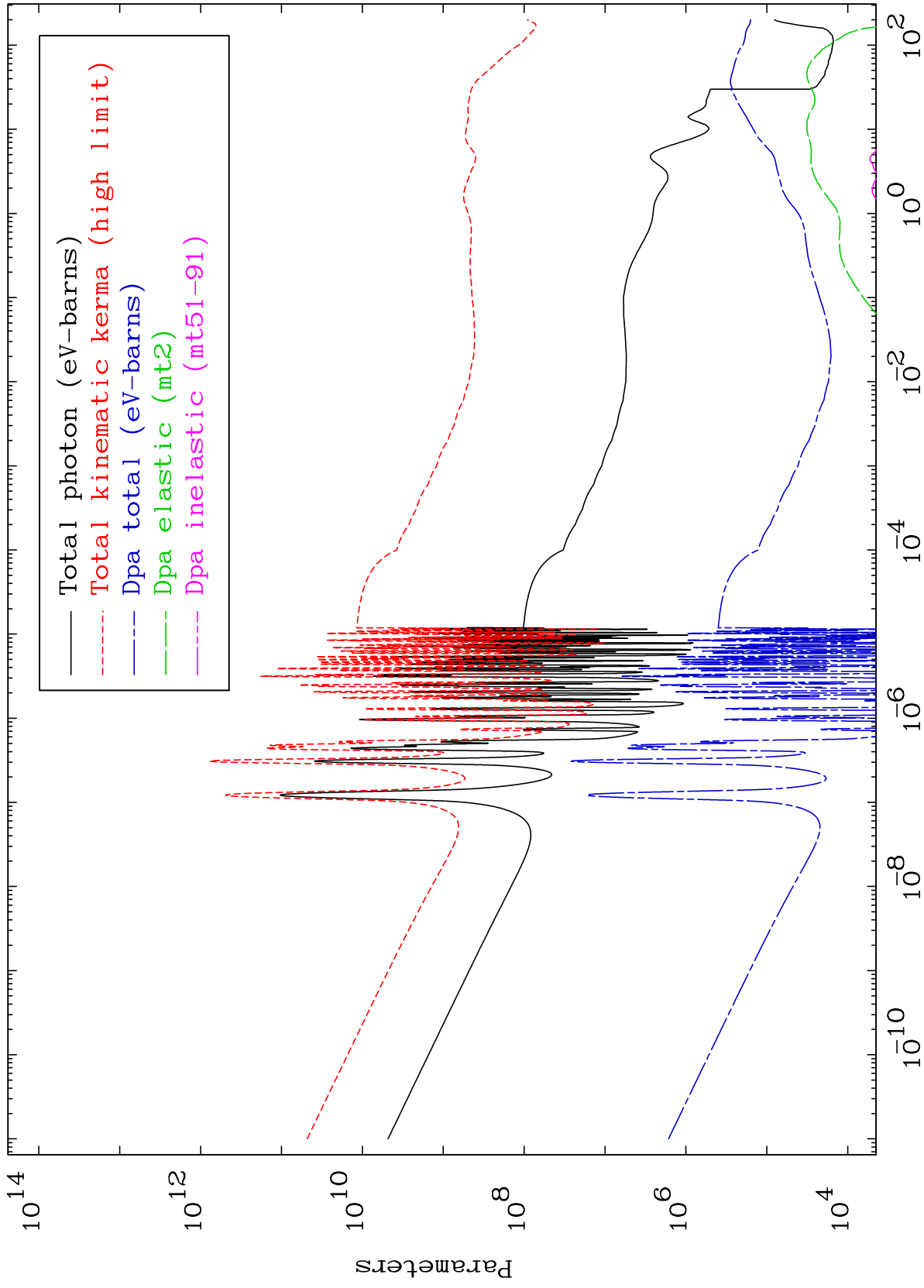


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Incident Energy (MeV)

97-Bk-239



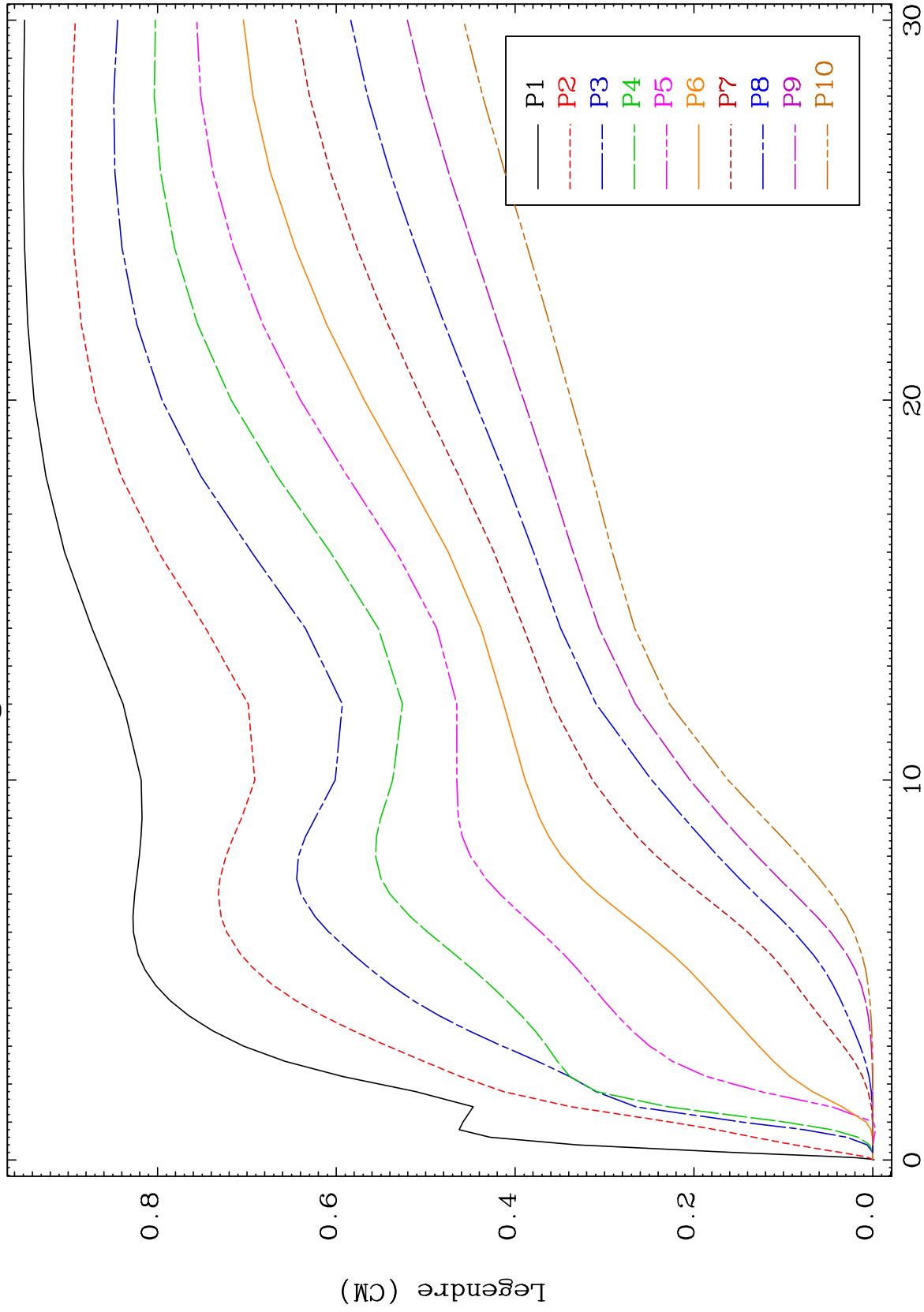




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Elastic Legendre Coefficients

97-Bk-239



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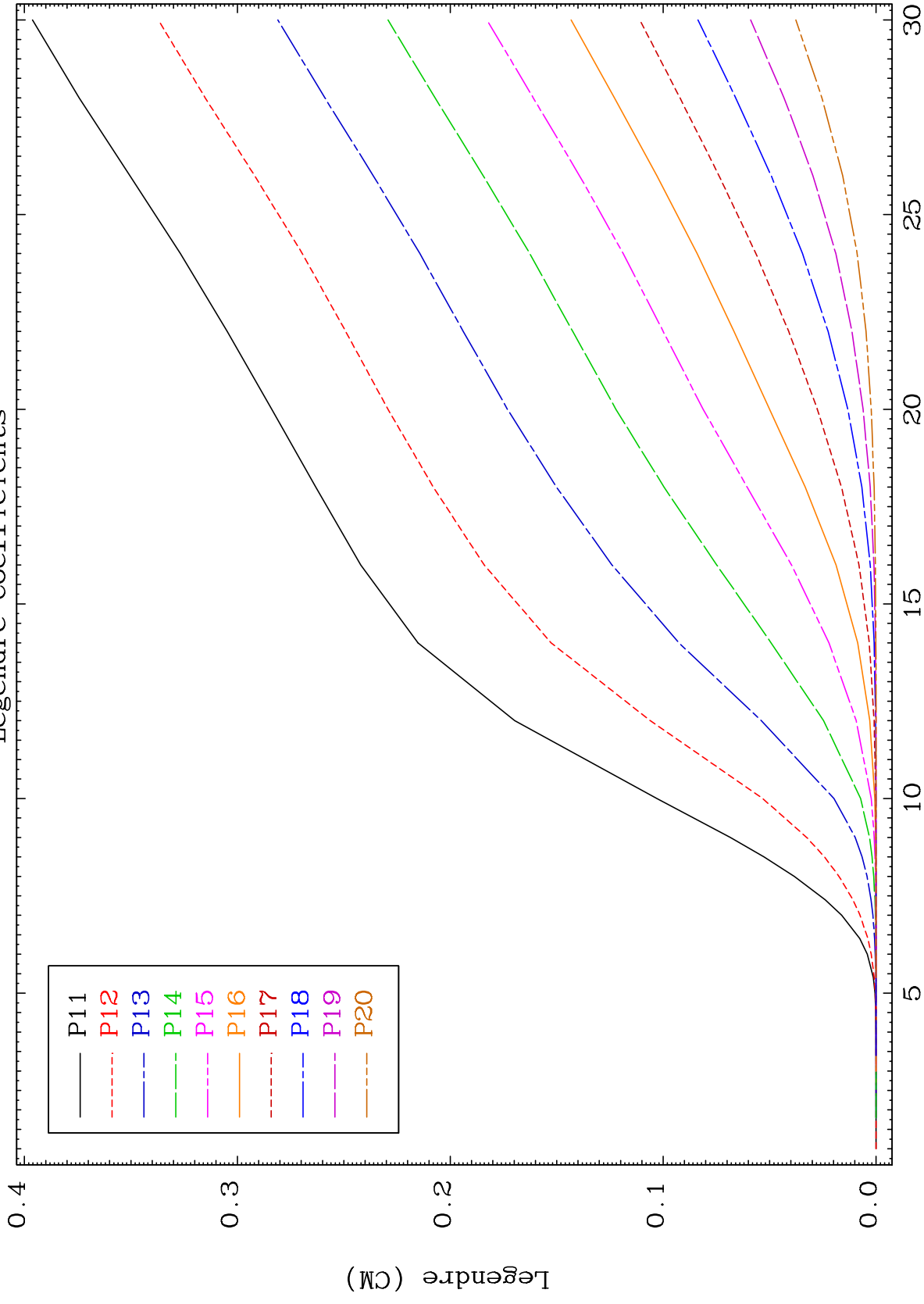
Incident Energy (MeV)

97-Bk-239

MAT 9722

Elastic Legendre Coefficients

97-Bk-239



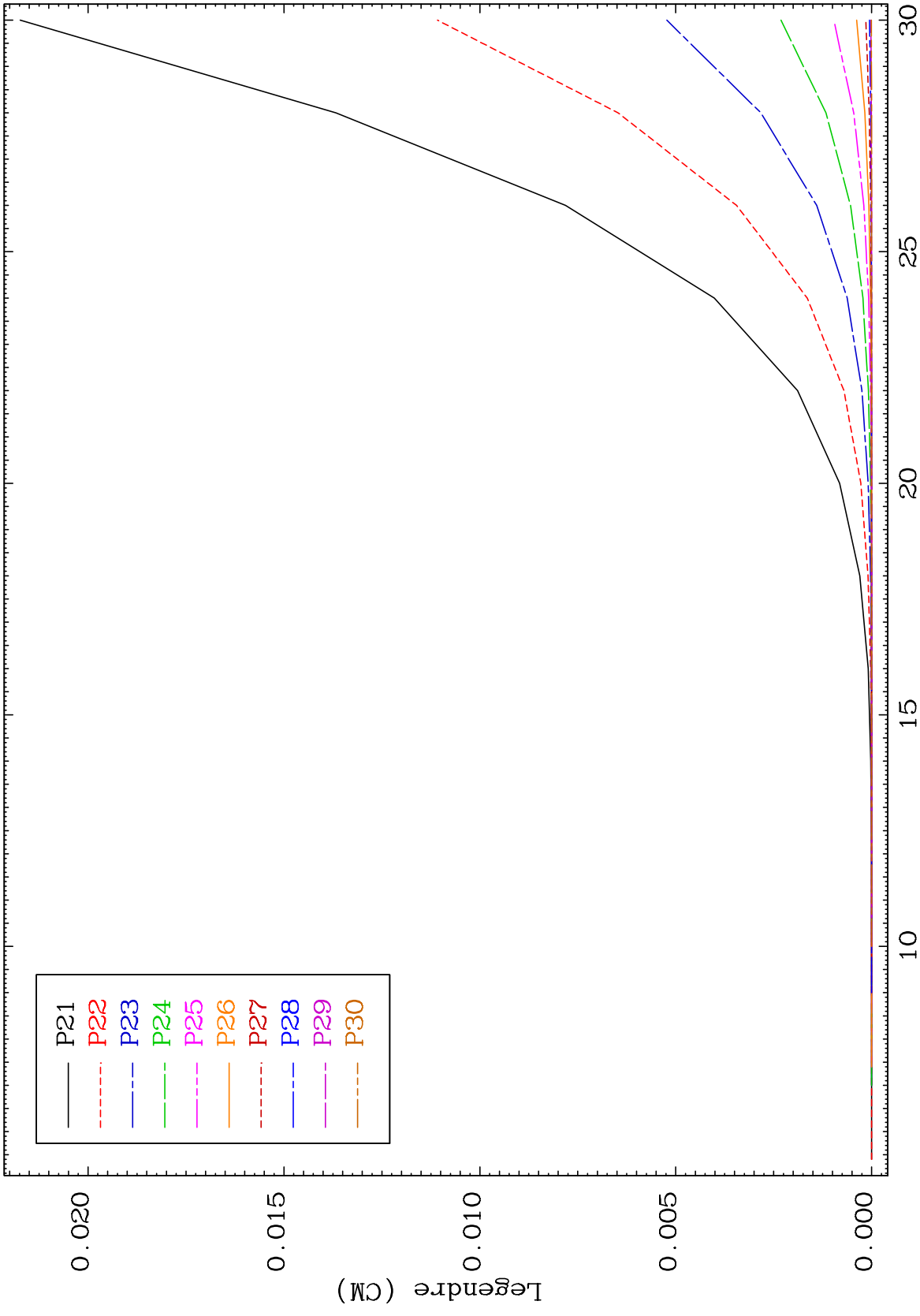
17

97-Bk-239

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### Elastic Legendre Coefficients

97-Bk-239



18

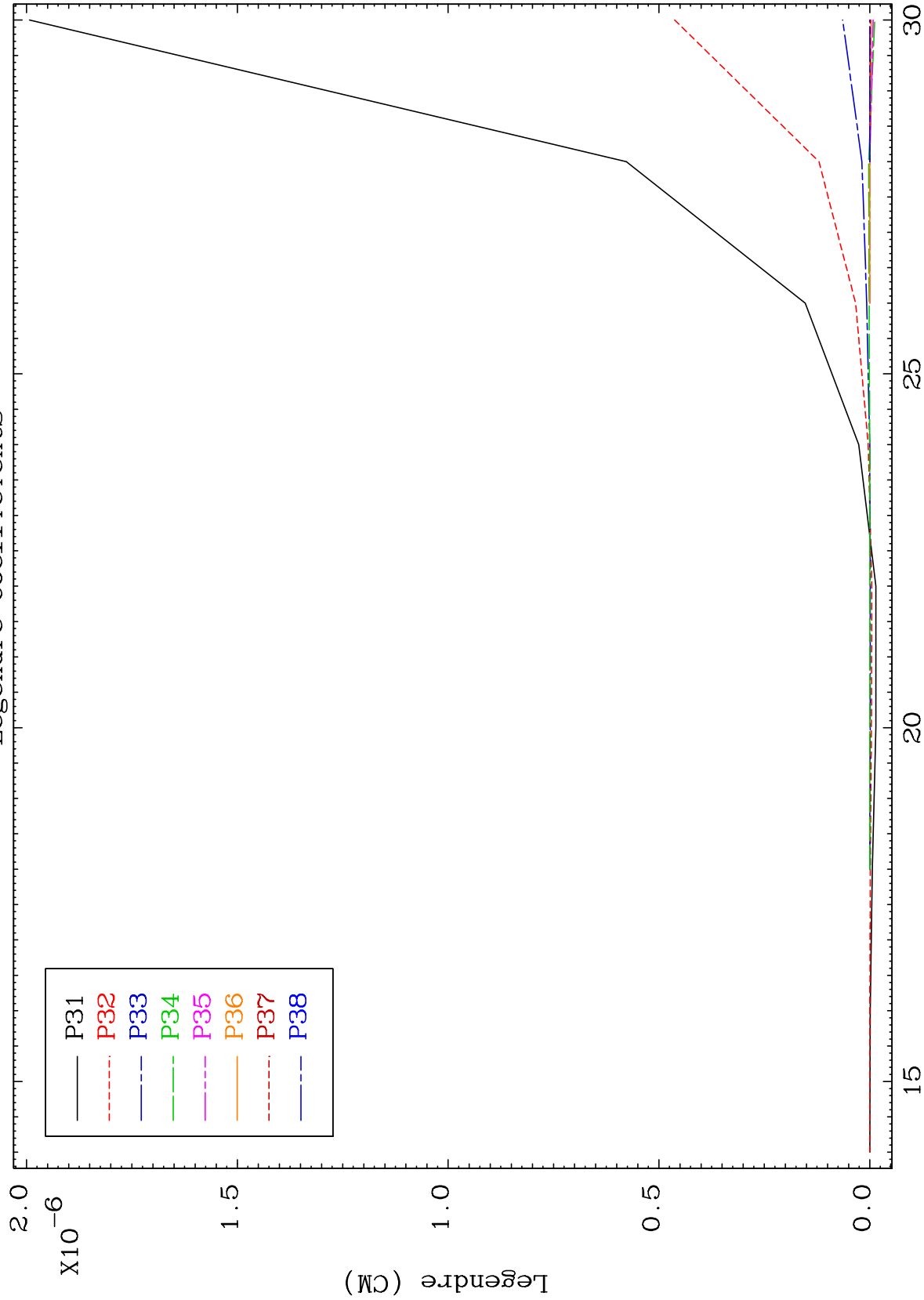
Incident Energy (MeV)

97-Bk-239

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Elastic Legendre Coefficients

97-Bk-239



19

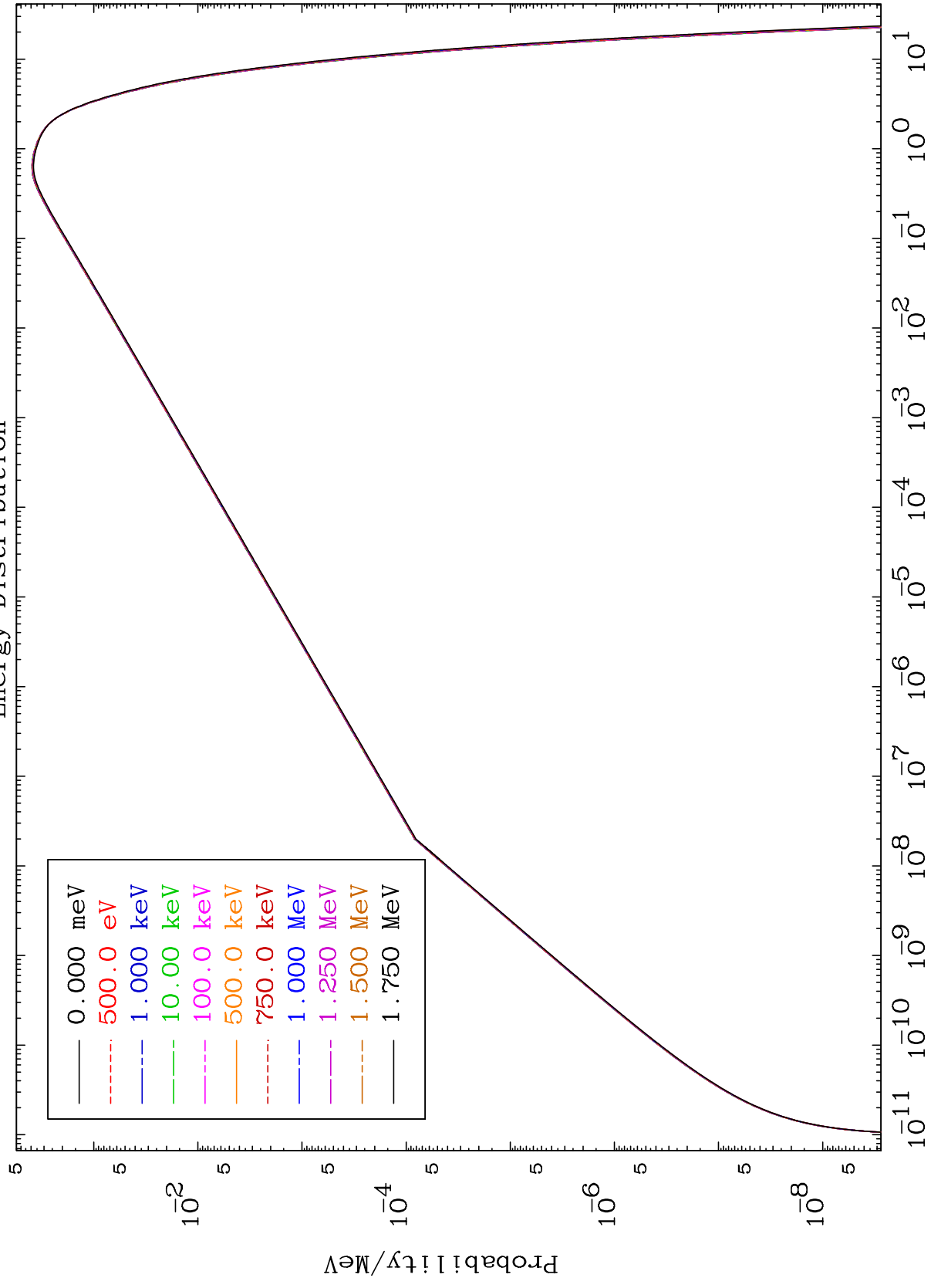
Incident Energy (MeV)

97-Bk-239

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### Fission Energy Distribution

97-Bk-239



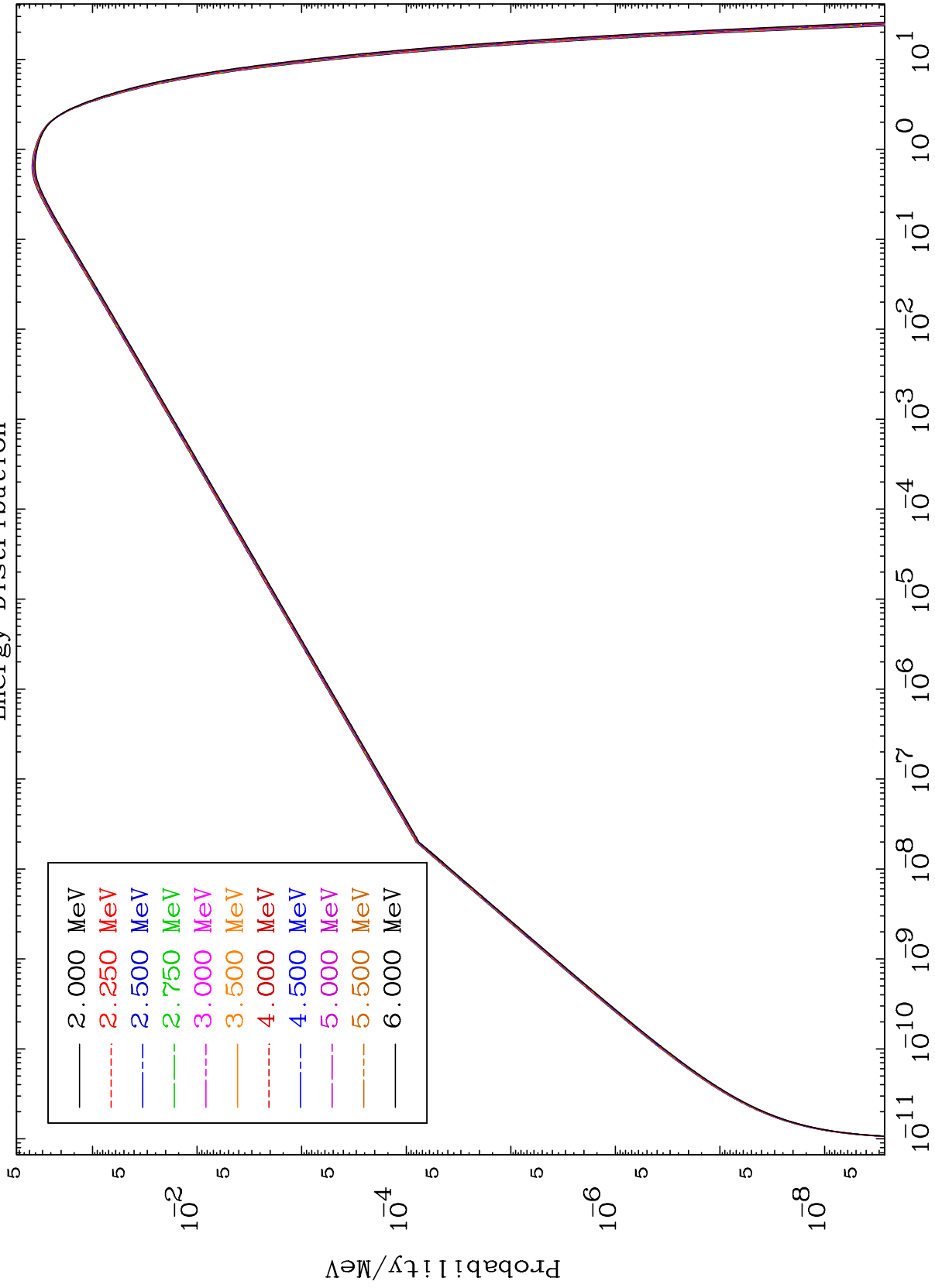
97-Bk-239

Secondary Energy (MeV)

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### Fission Energy Distribution

97-Bk-239

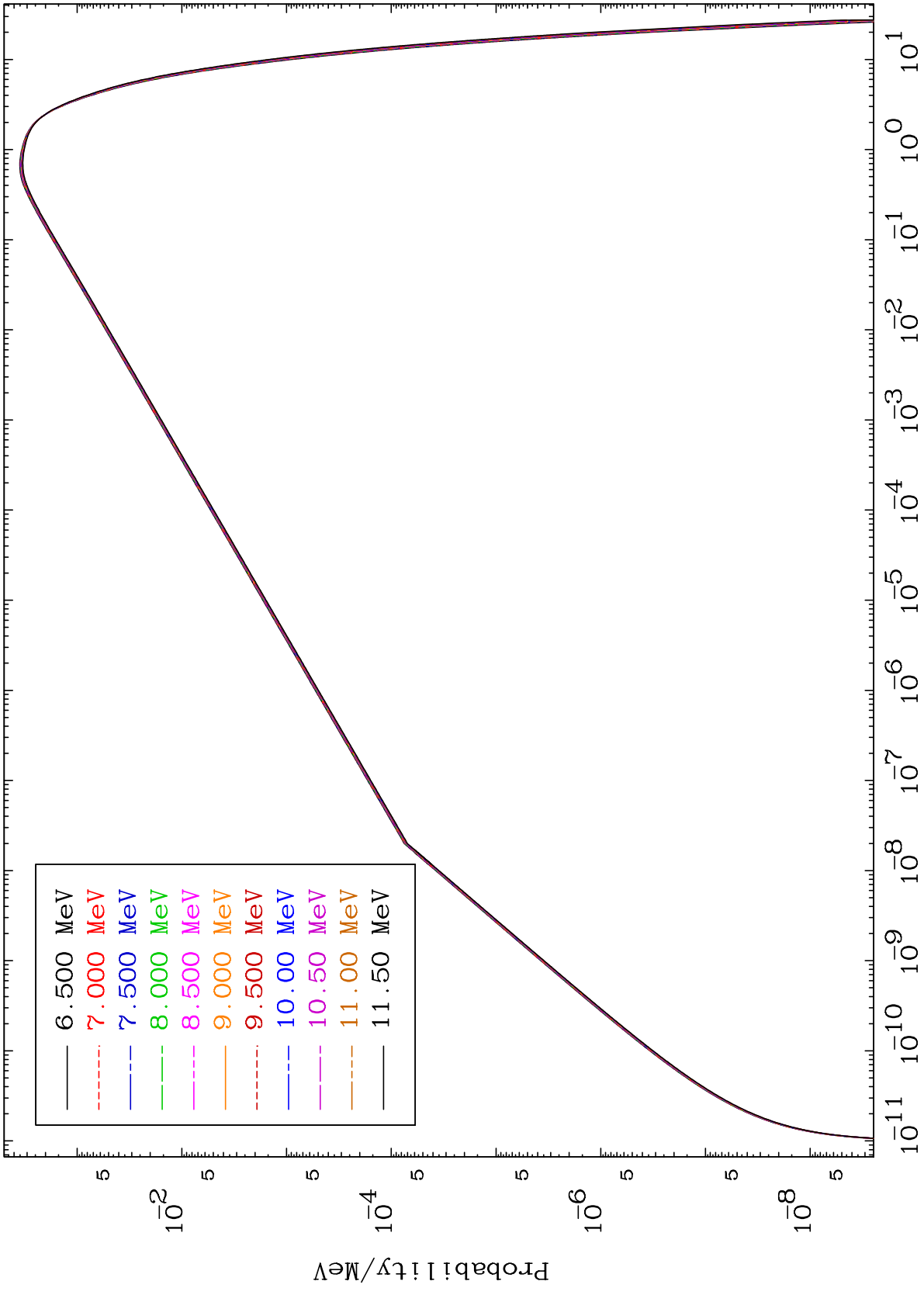


97-Bk-239

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### Fission Energy Distribution

97-Bk-239

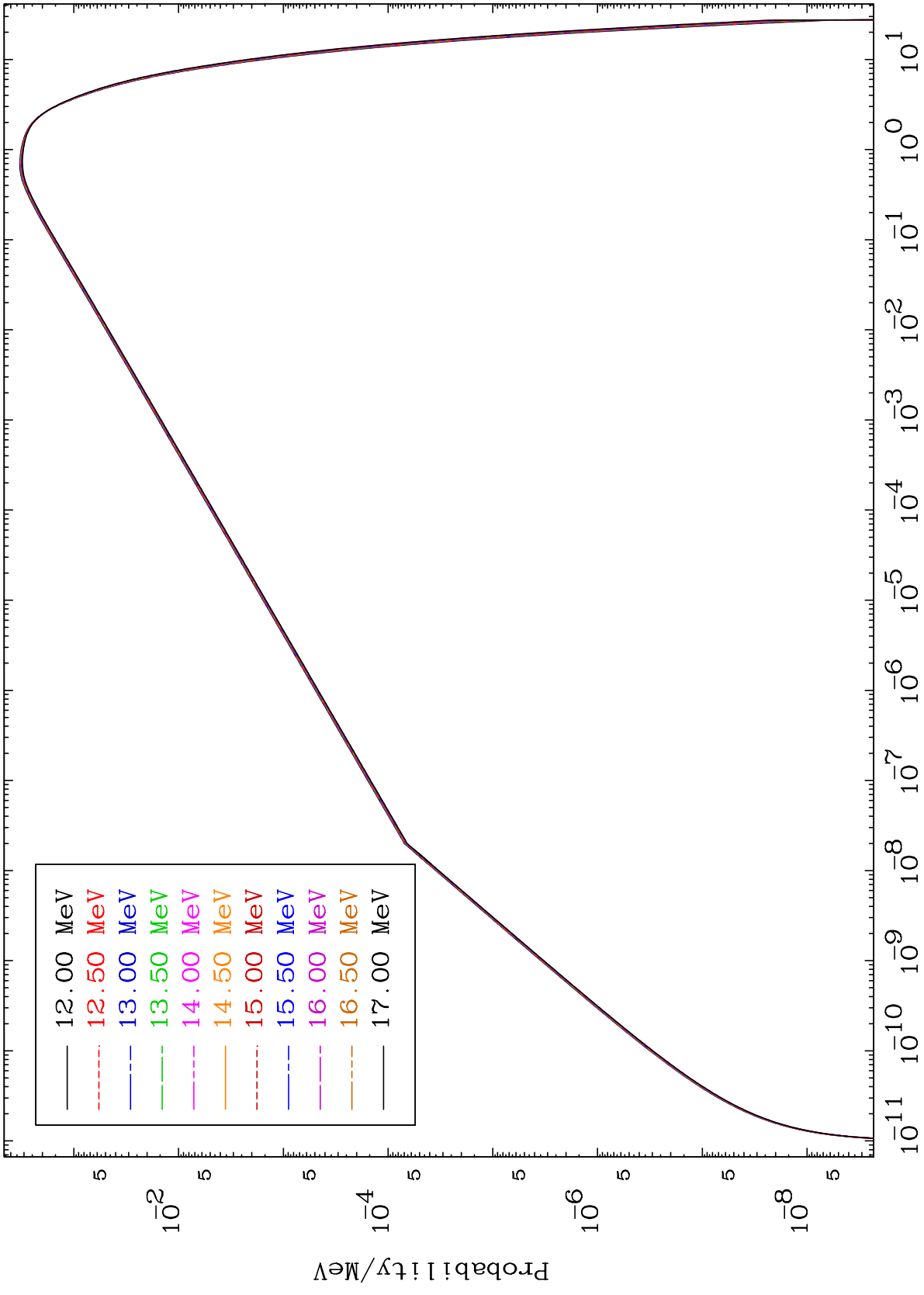


97-Bk-239

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### Fission Energy Distribution

97-Bk-239



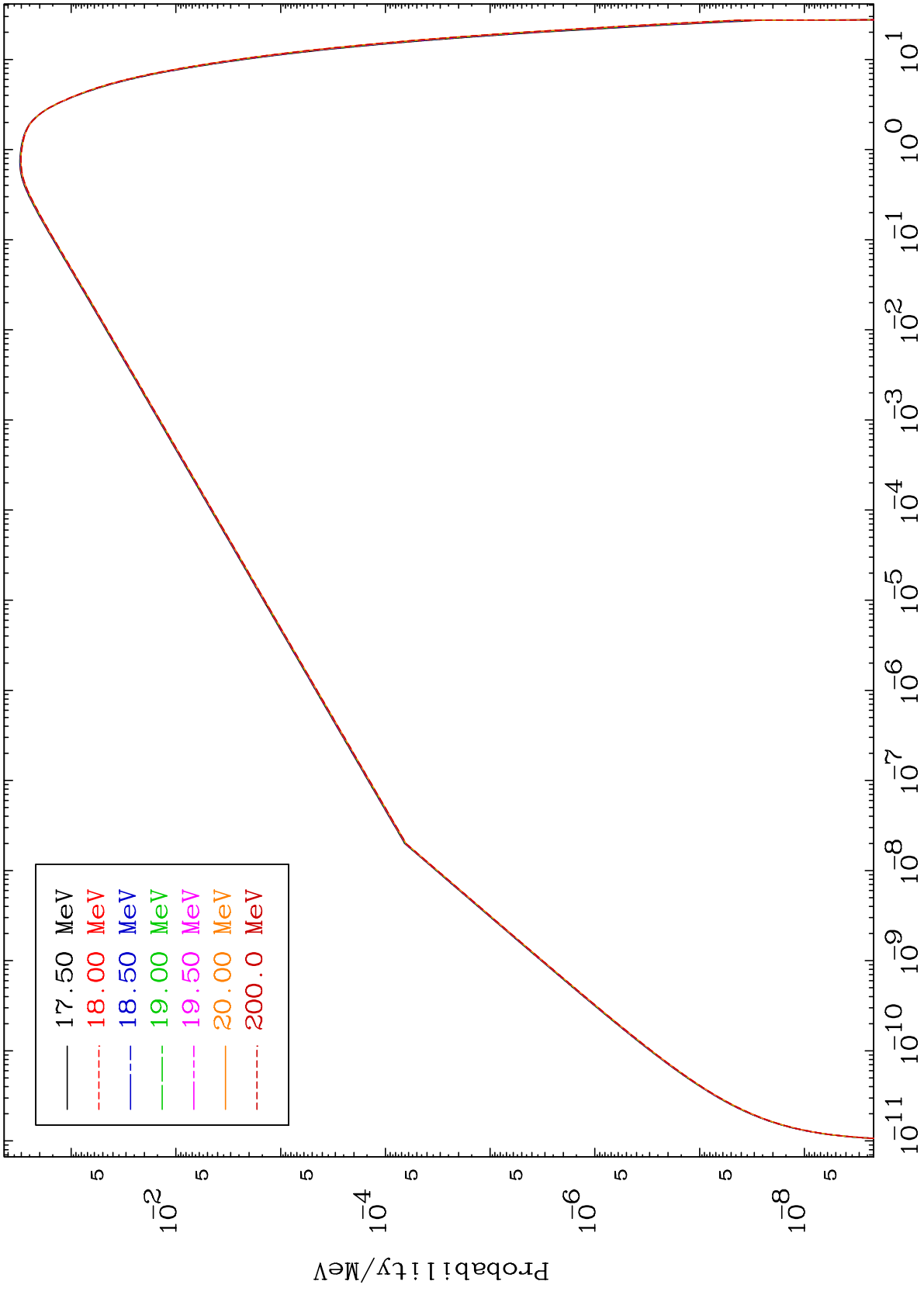
97-Bk-239



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### Fission Energy Distribution

97-Bk-239



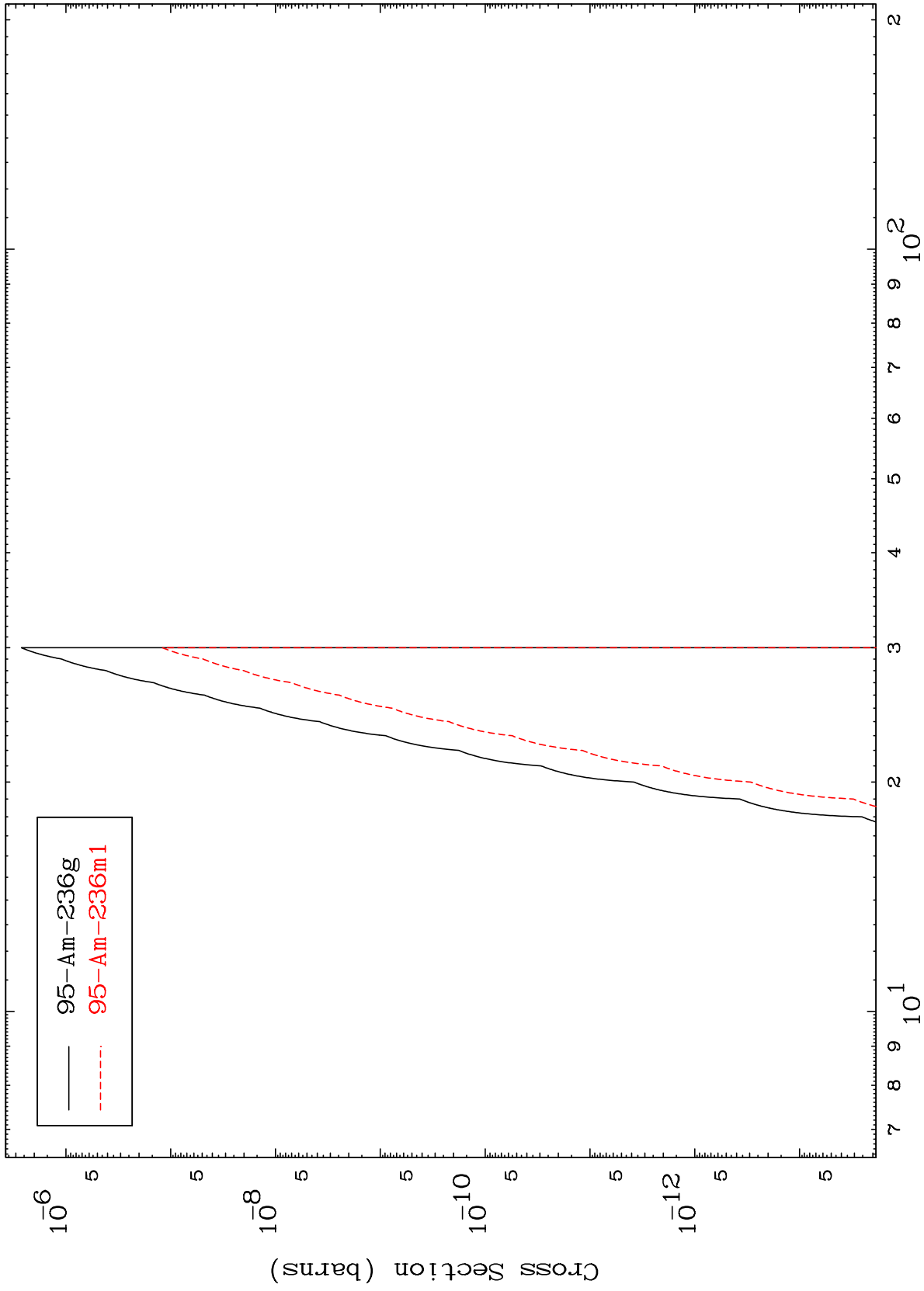
97-Bk-239

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(n, n') He-3

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Radionuclide Production Cross Section



25

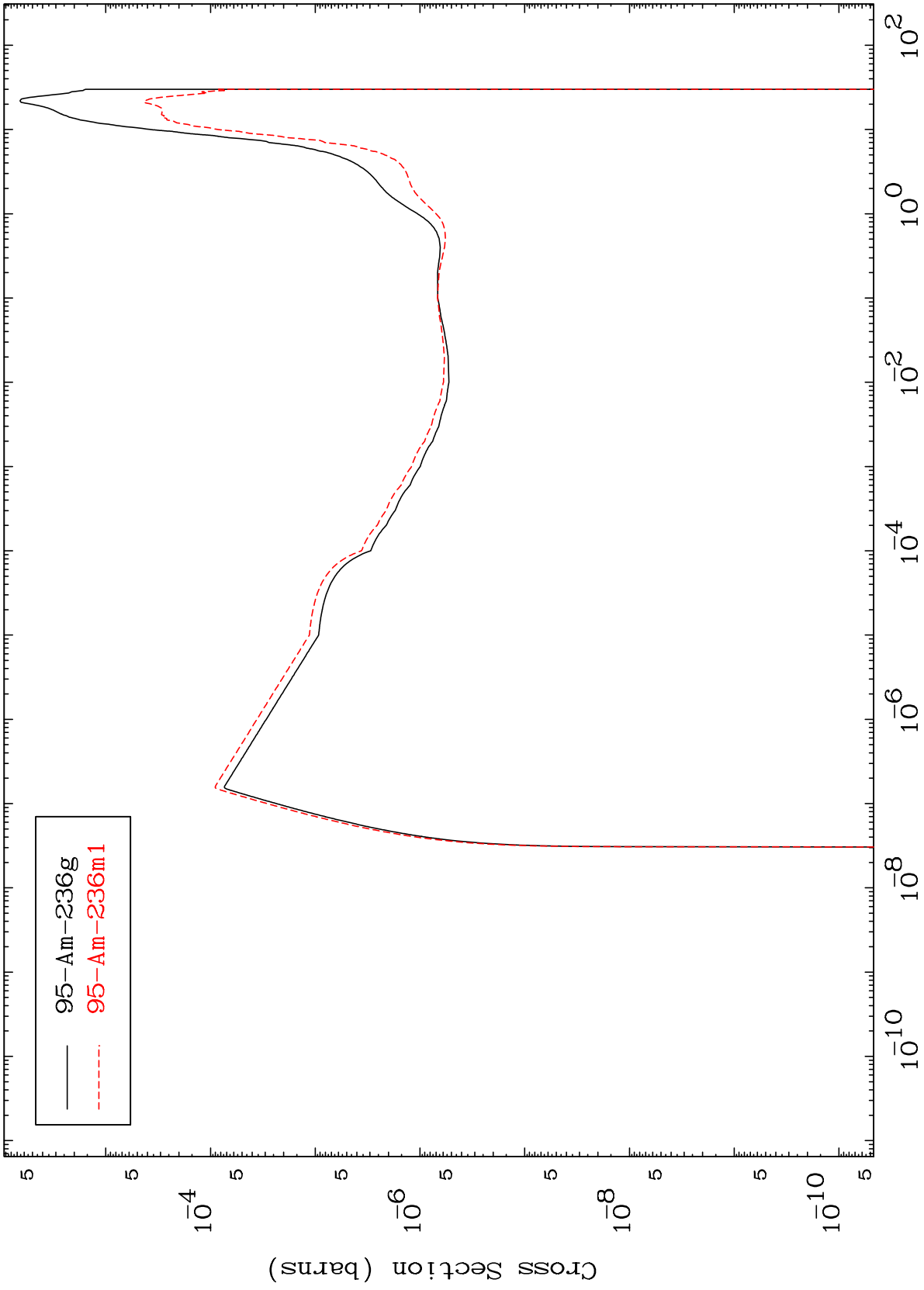
Incident Energy (MeV)

97-Bk-239

MAT 9722

97-Bk-239

(n,  $\alpha$ )  
Radionuclide Production Cross Section

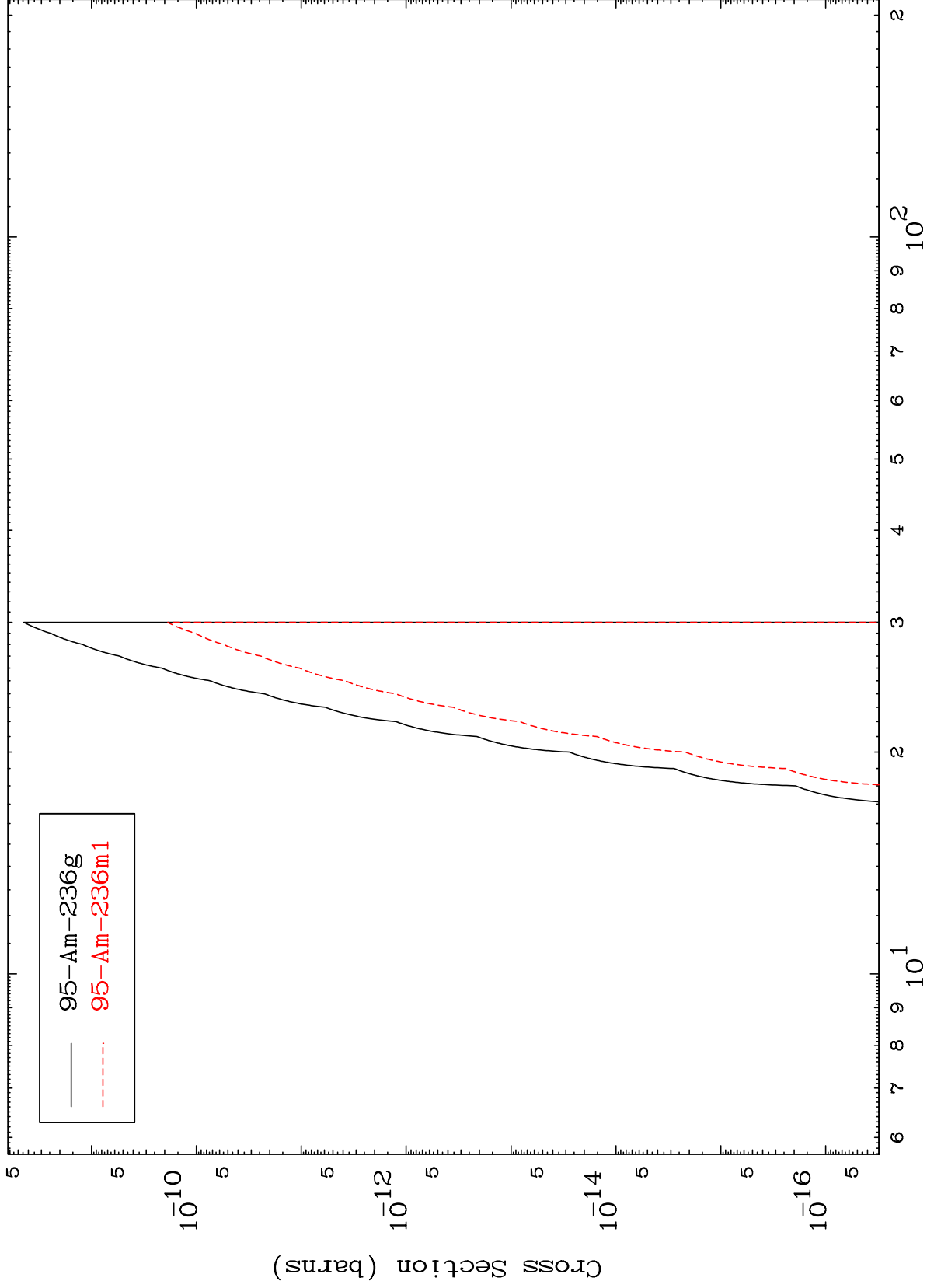


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(n,p) t

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Radionuclide Production Cross Section



27

Incident Energy (MeV)

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