

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

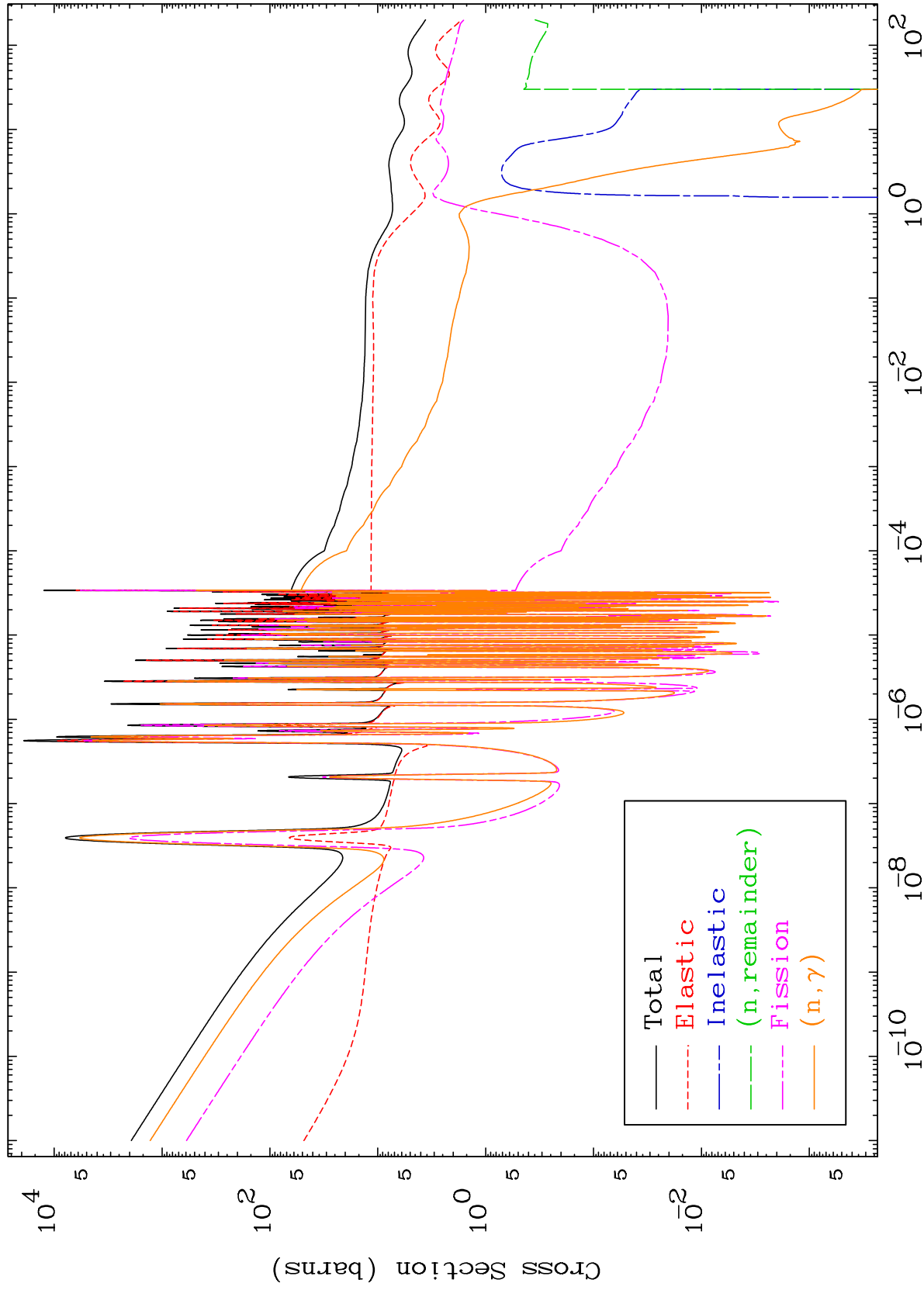
Press Mouse Button to Start

MAT 9740

Major

293 Kelvin Cross Sections

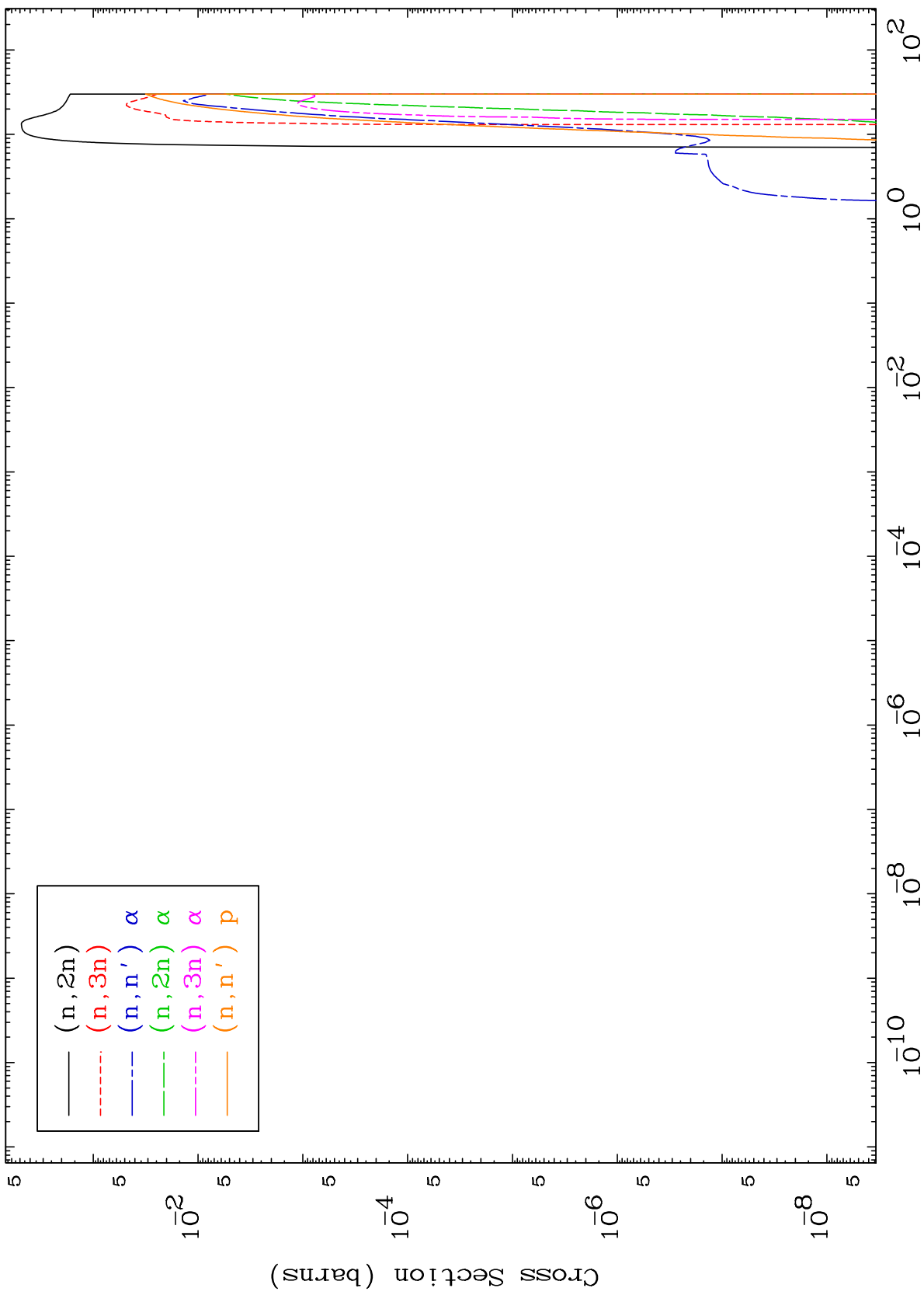
97-Bk-245



MAT 9740

Neutron Production
293 Kelvin Cross Sections

97-Bk-245



2

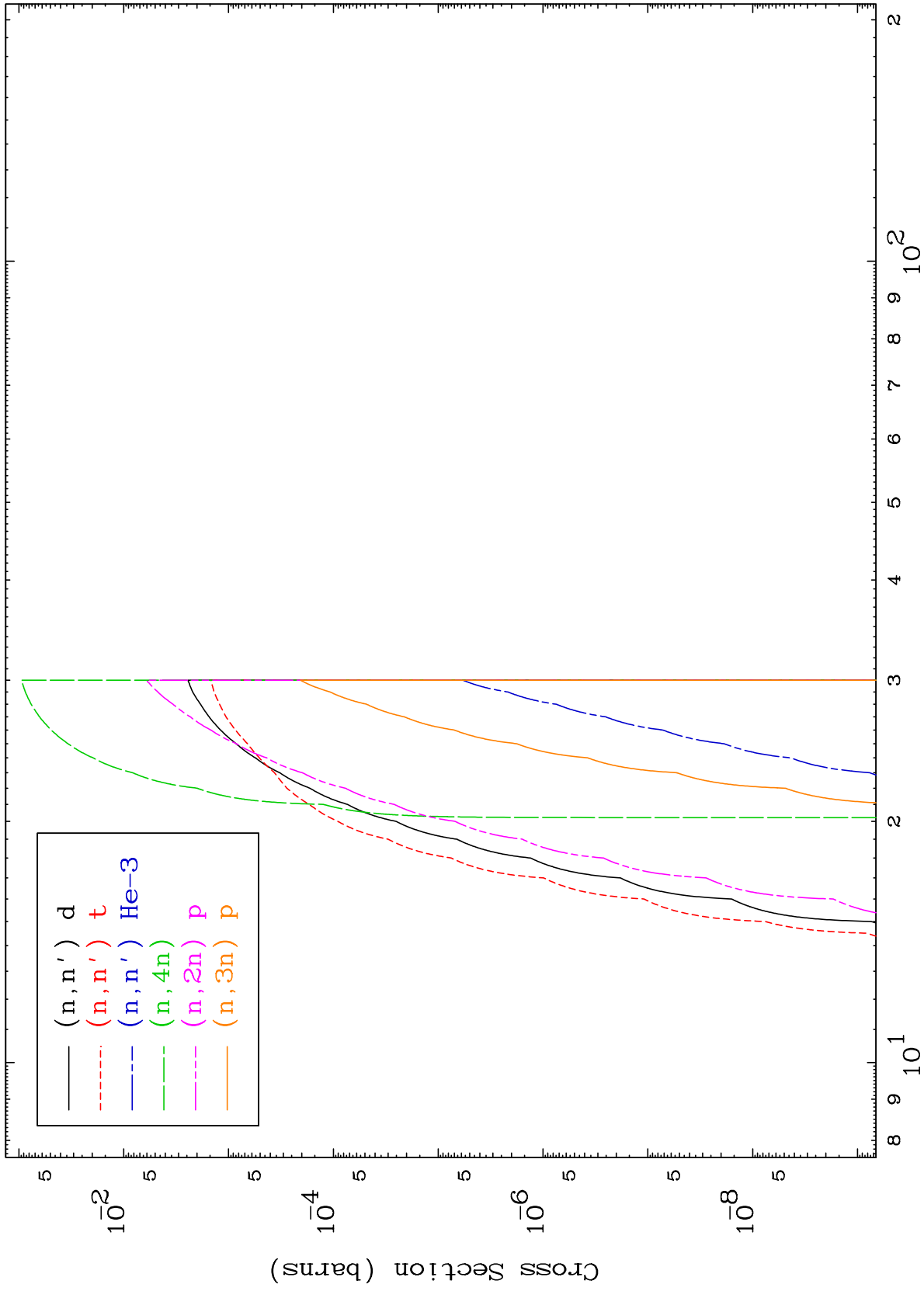
Incident Energy (MeV)

97-Bk-245

MAT 9740

Neutron Production
293 Kelvin Cross Sections

97-Bk-245



3

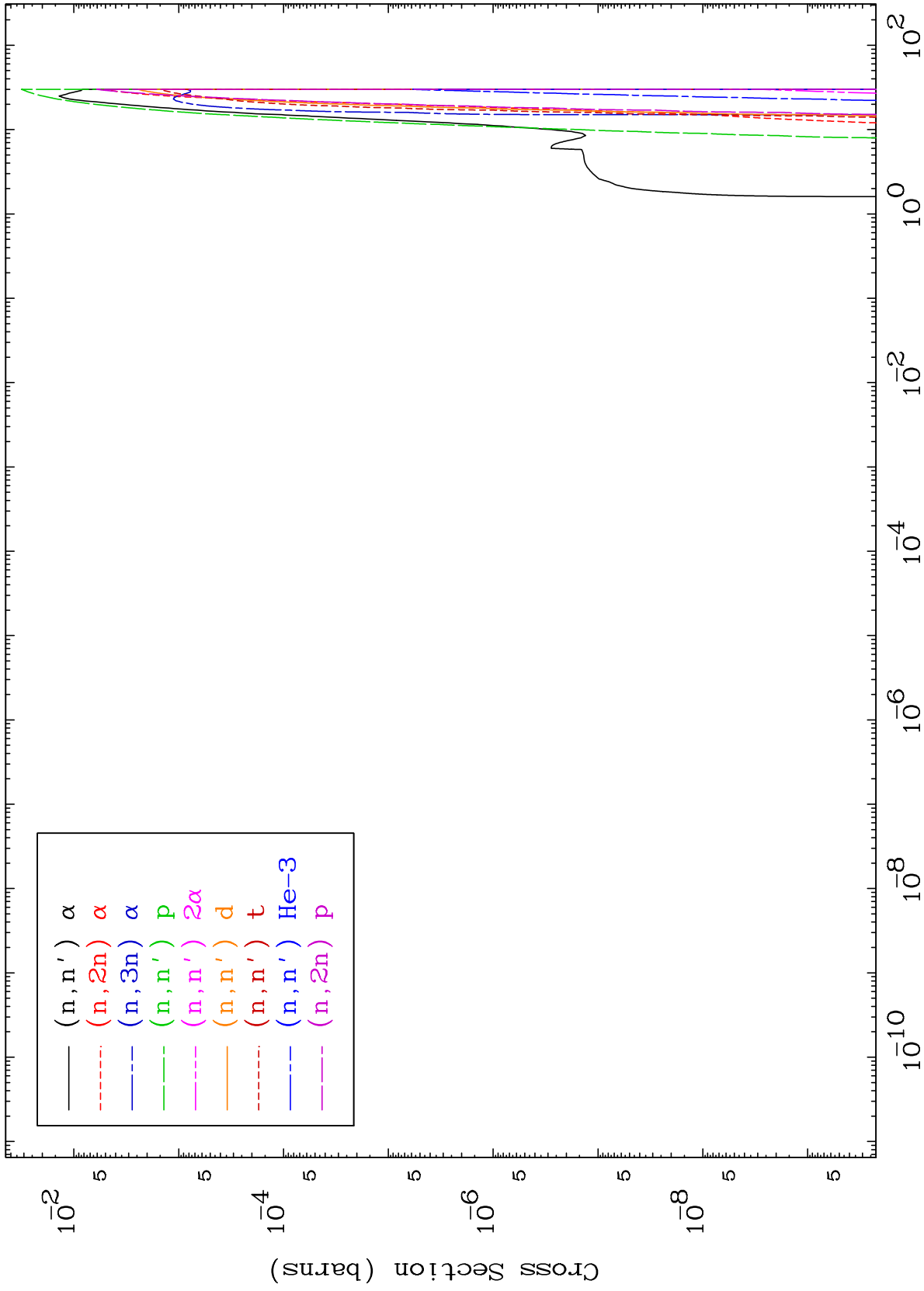
Incident Energy (MeV)

97-Bk-245

MAT 9740

Charged Particle
293 Kelvin Cross Sections

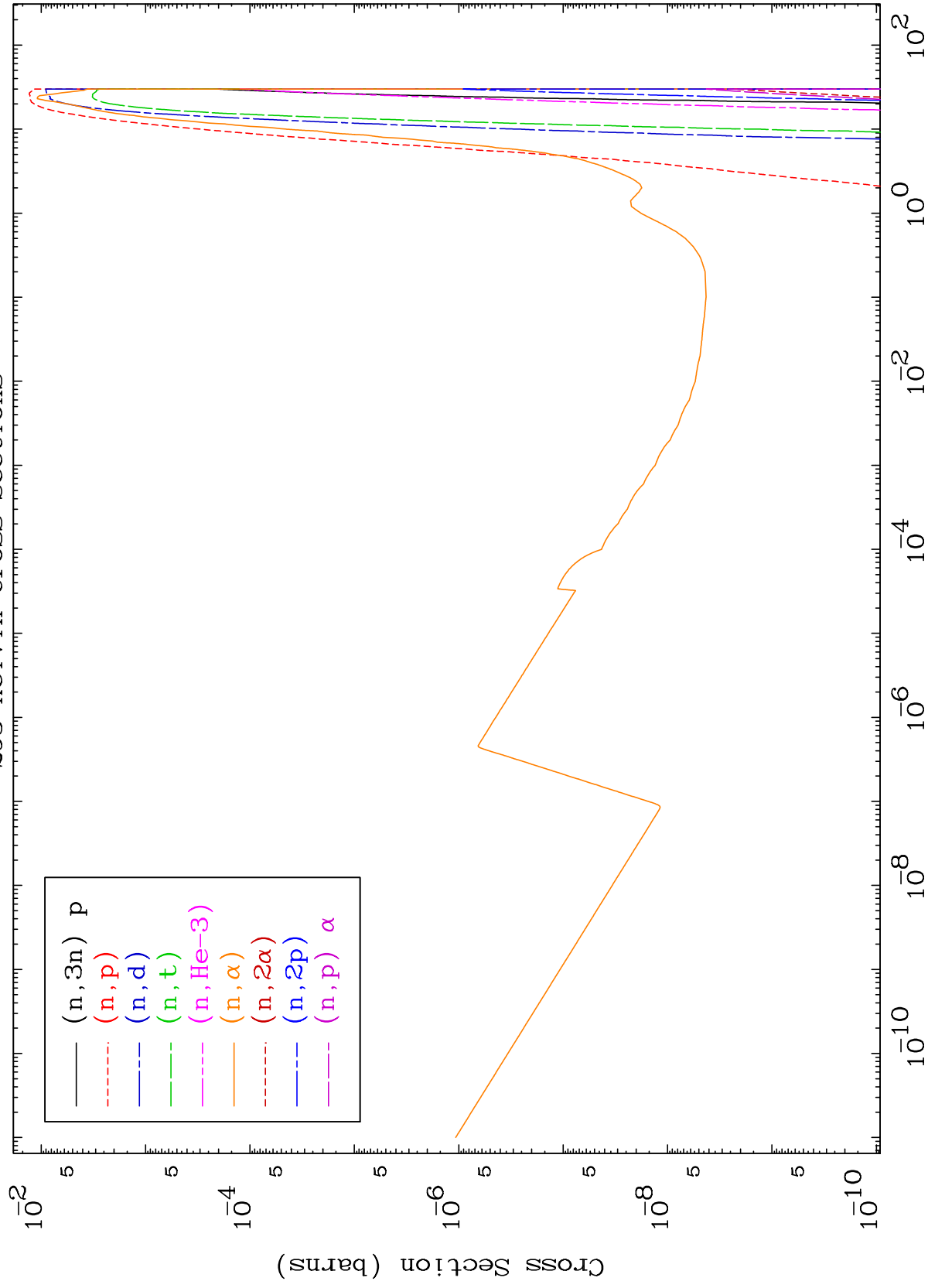
97-Bk-245



MAT 9740

Charged Particle
293 Kelvin Cross Sections

97-Bk-245

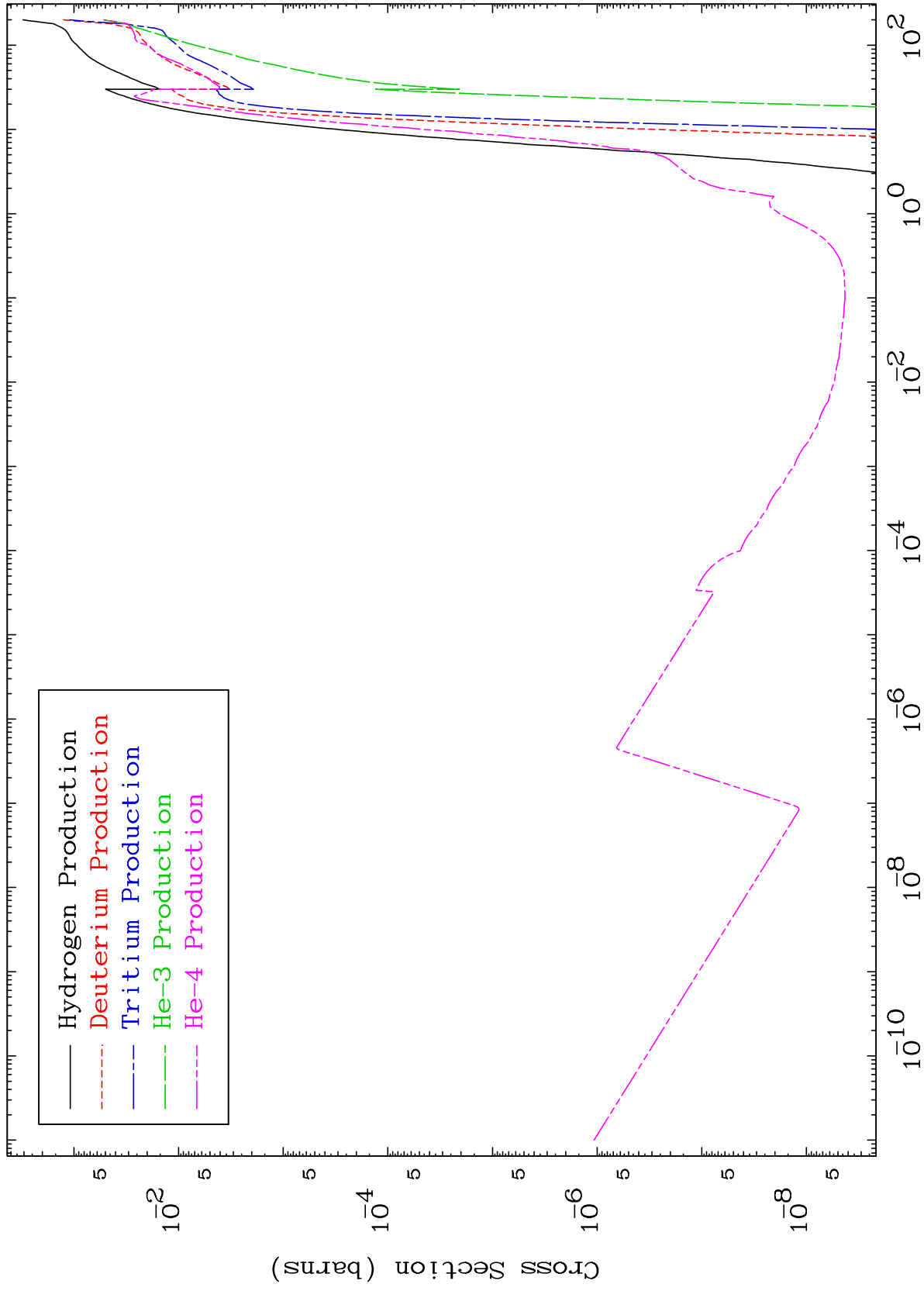


5

MAT 9740

Particle Production
293 Kelvin Cross Sections

97-Bk-245



6

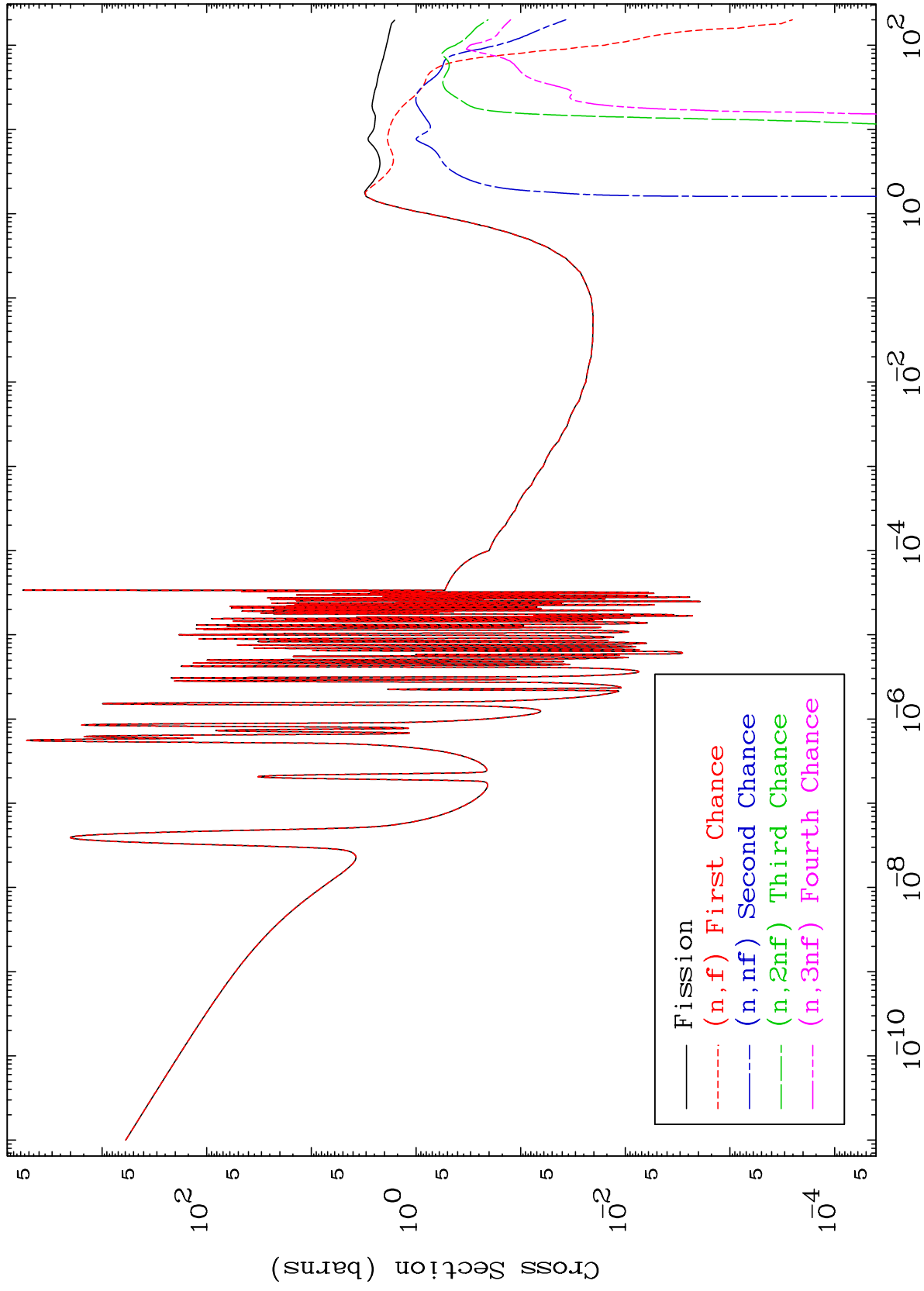
Incident Energy (MeV)

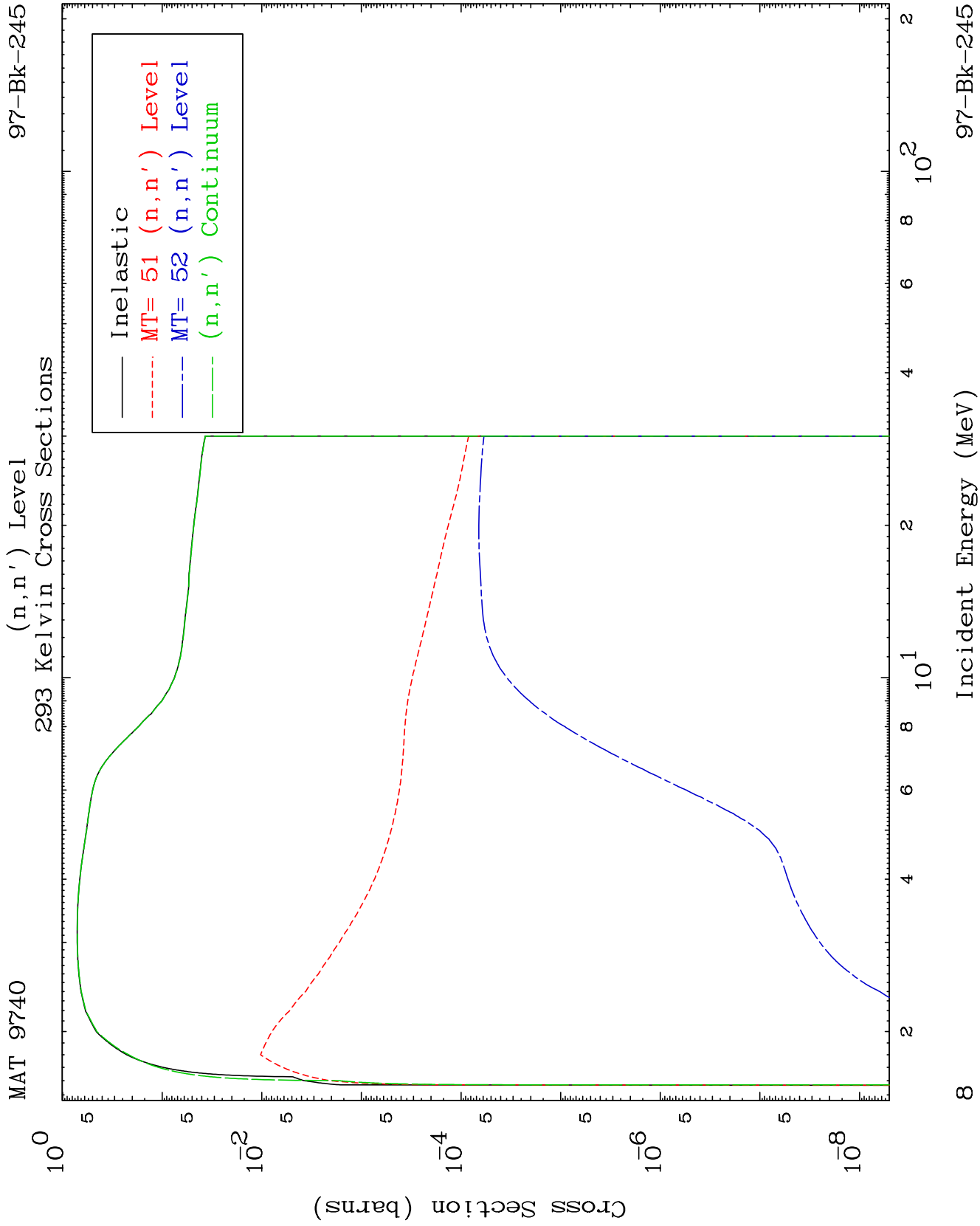
97-Bk-245

MAT 9740

Fission
293 Kelvin Cross Sections

97-Bk-245

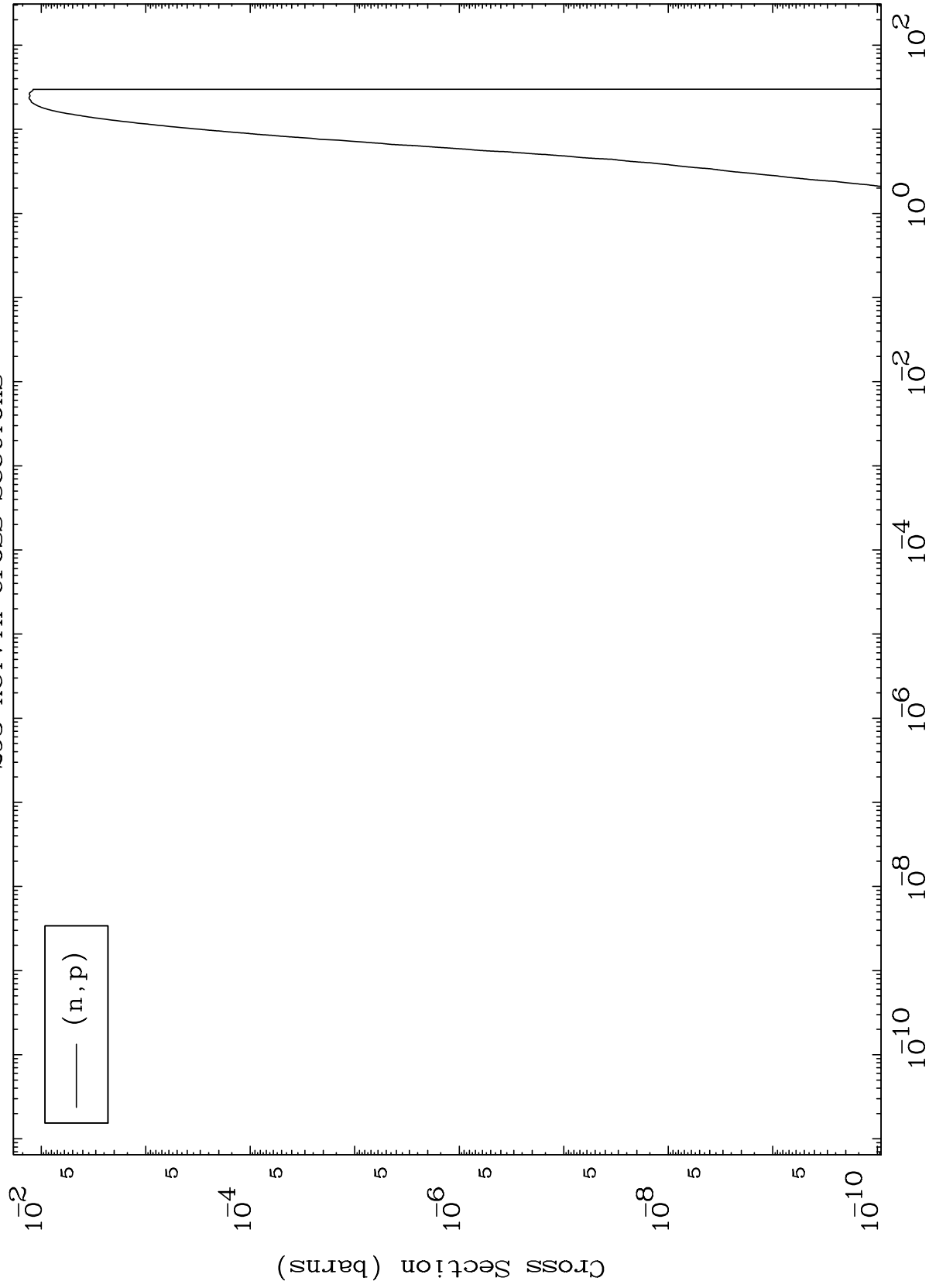


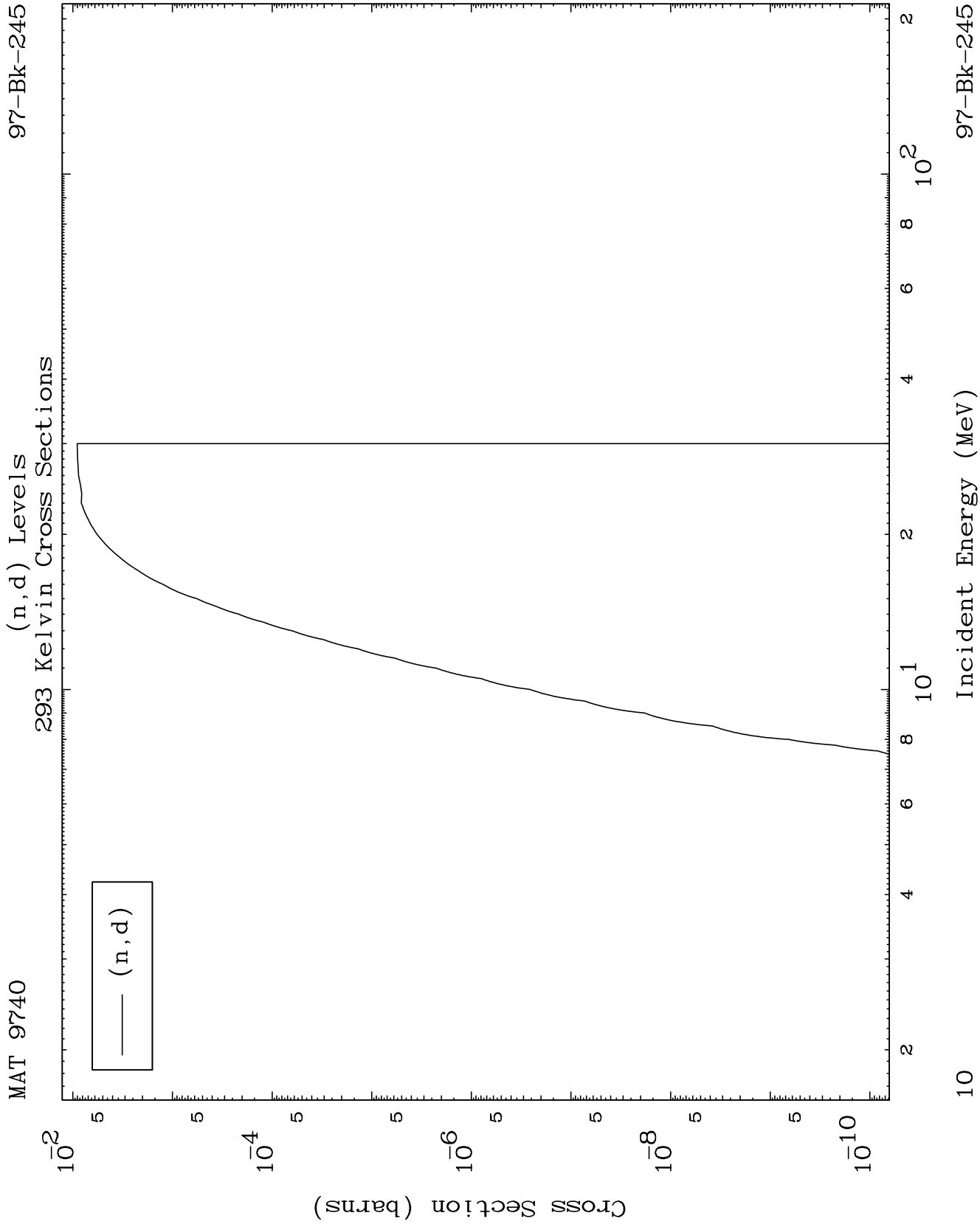


MAT 9740

(n,p) Levels
293 Kelvin Cross Sections

97-Bk-245

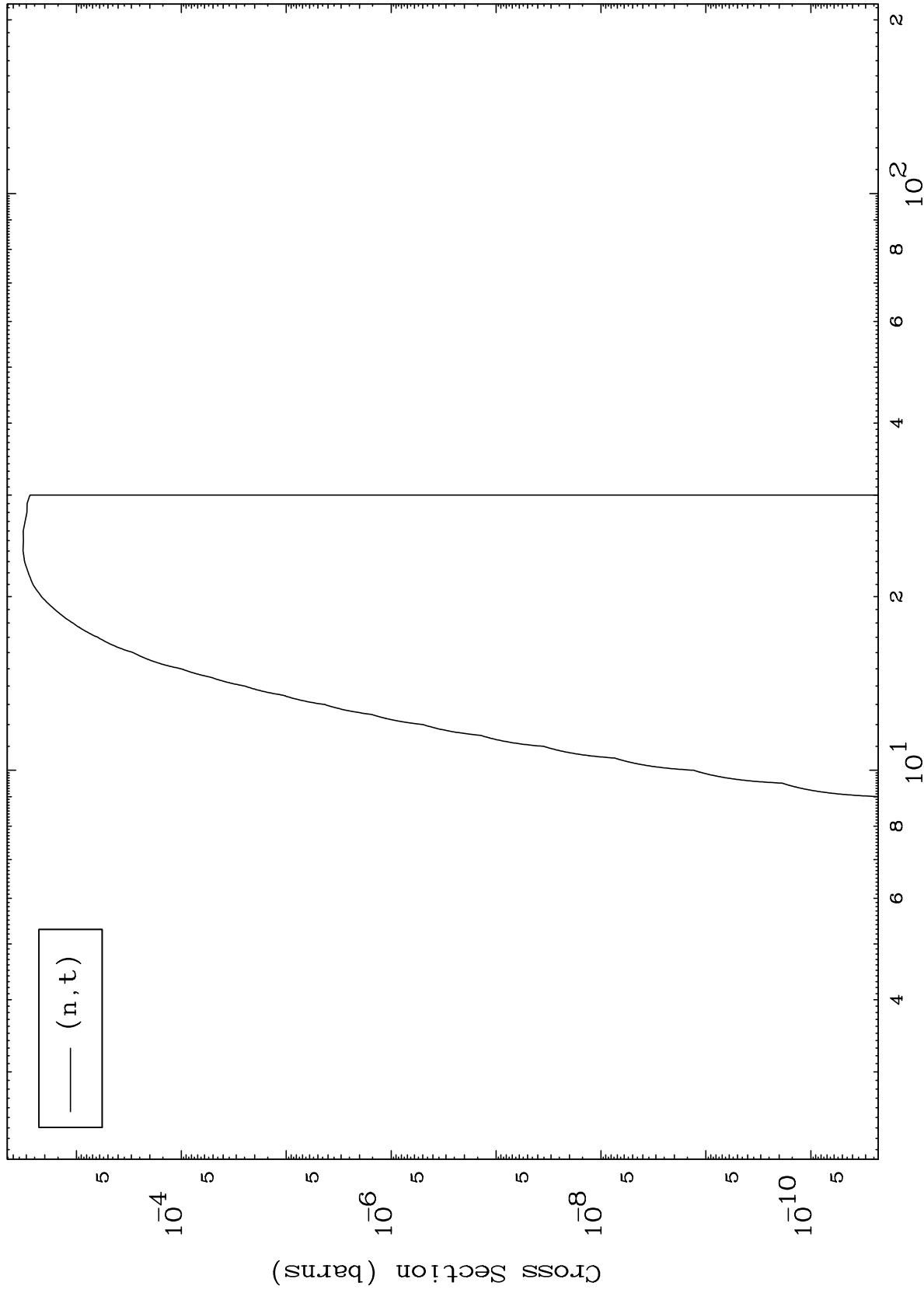




MAT 9740

97-Bk-245

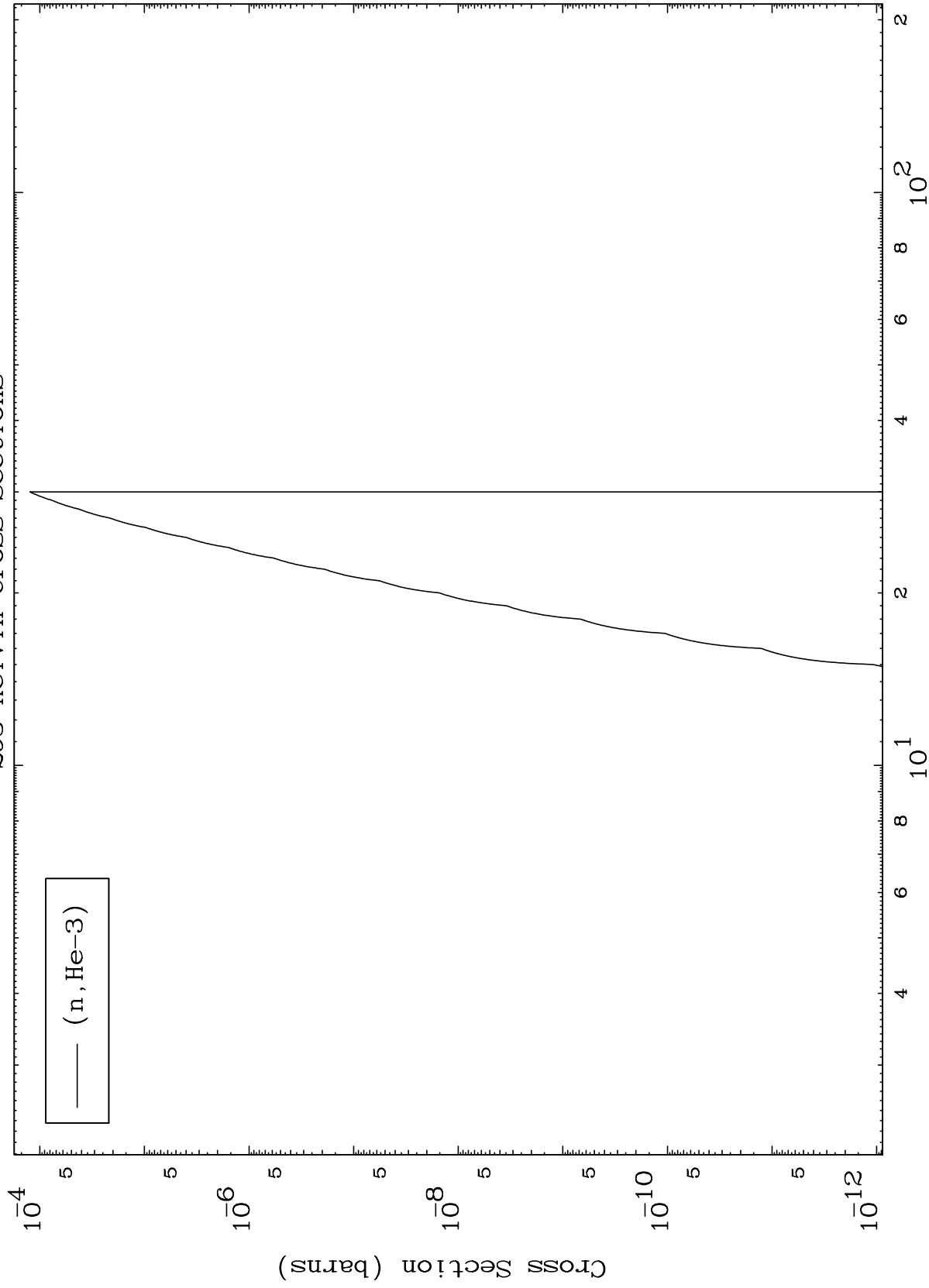
(n,t) Levels
293 Kelvin Cross Sections



MAT 9740

(n,He3) Levels
293 Kelvin Cross Sections

97-Bk-245



12

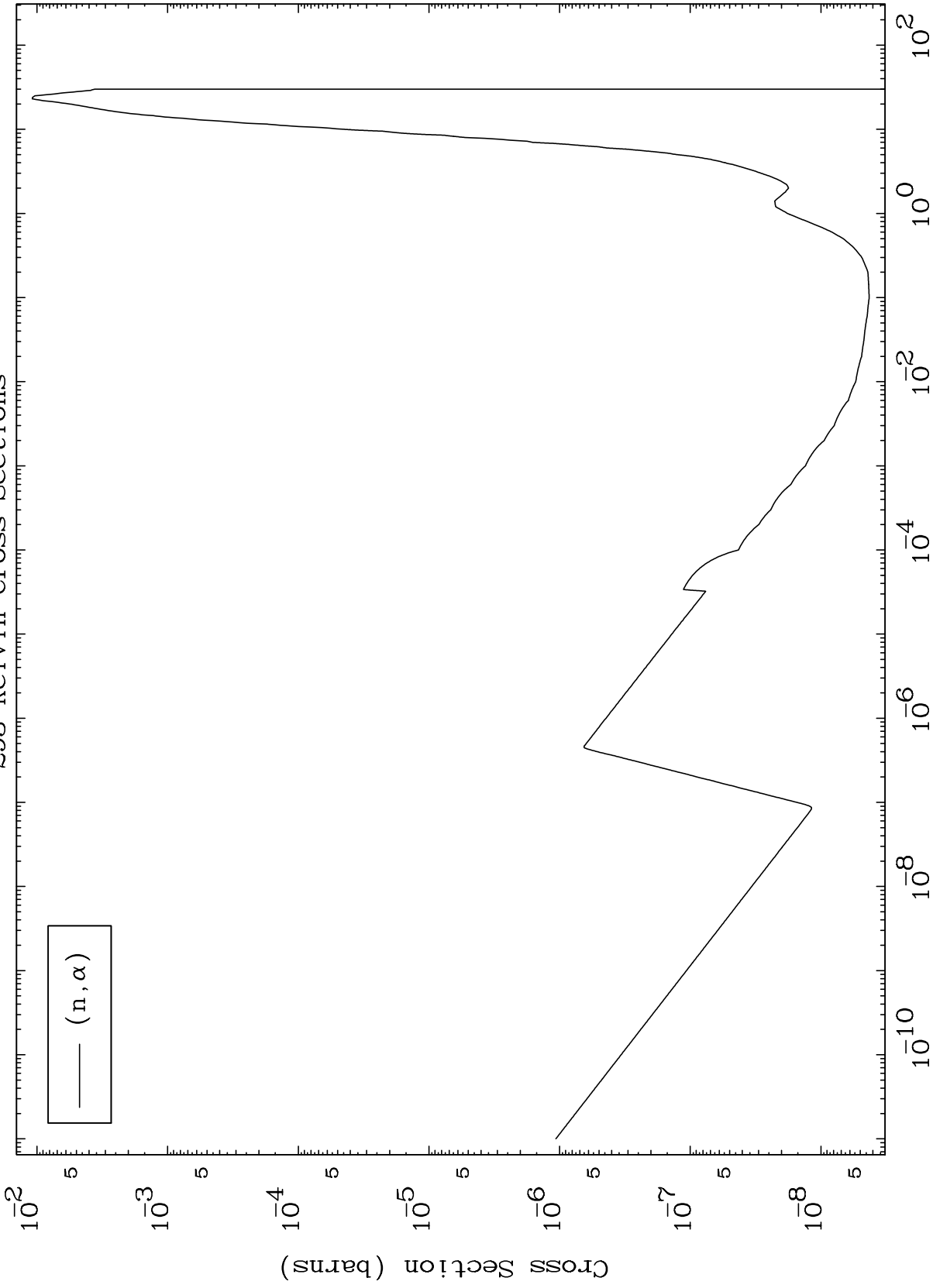
Incident Energy (MeV)

97-Bk-245

MAT 9740

293 Kelvin Cross Sections

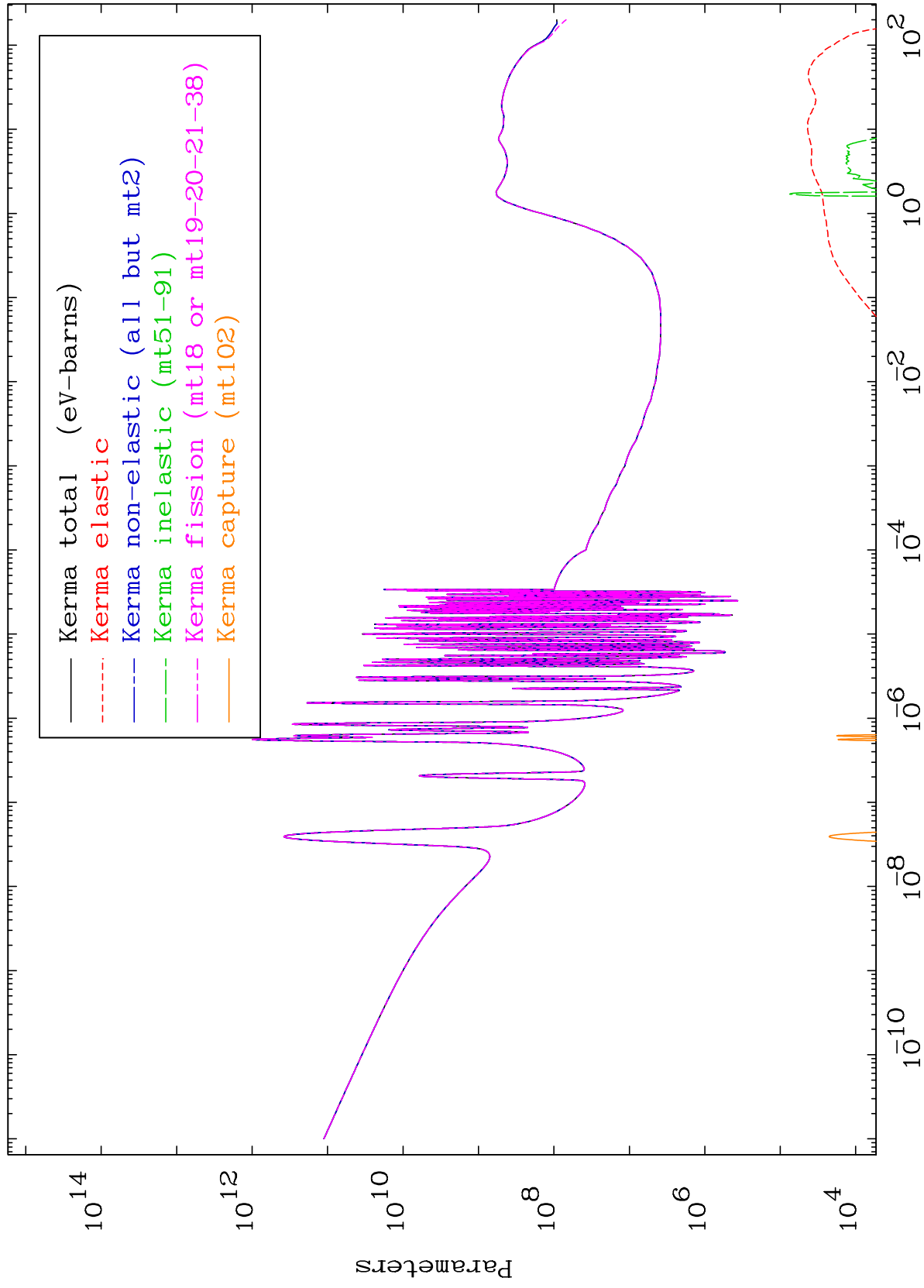
97-Bk-245



13

Incident Energy (MeV)

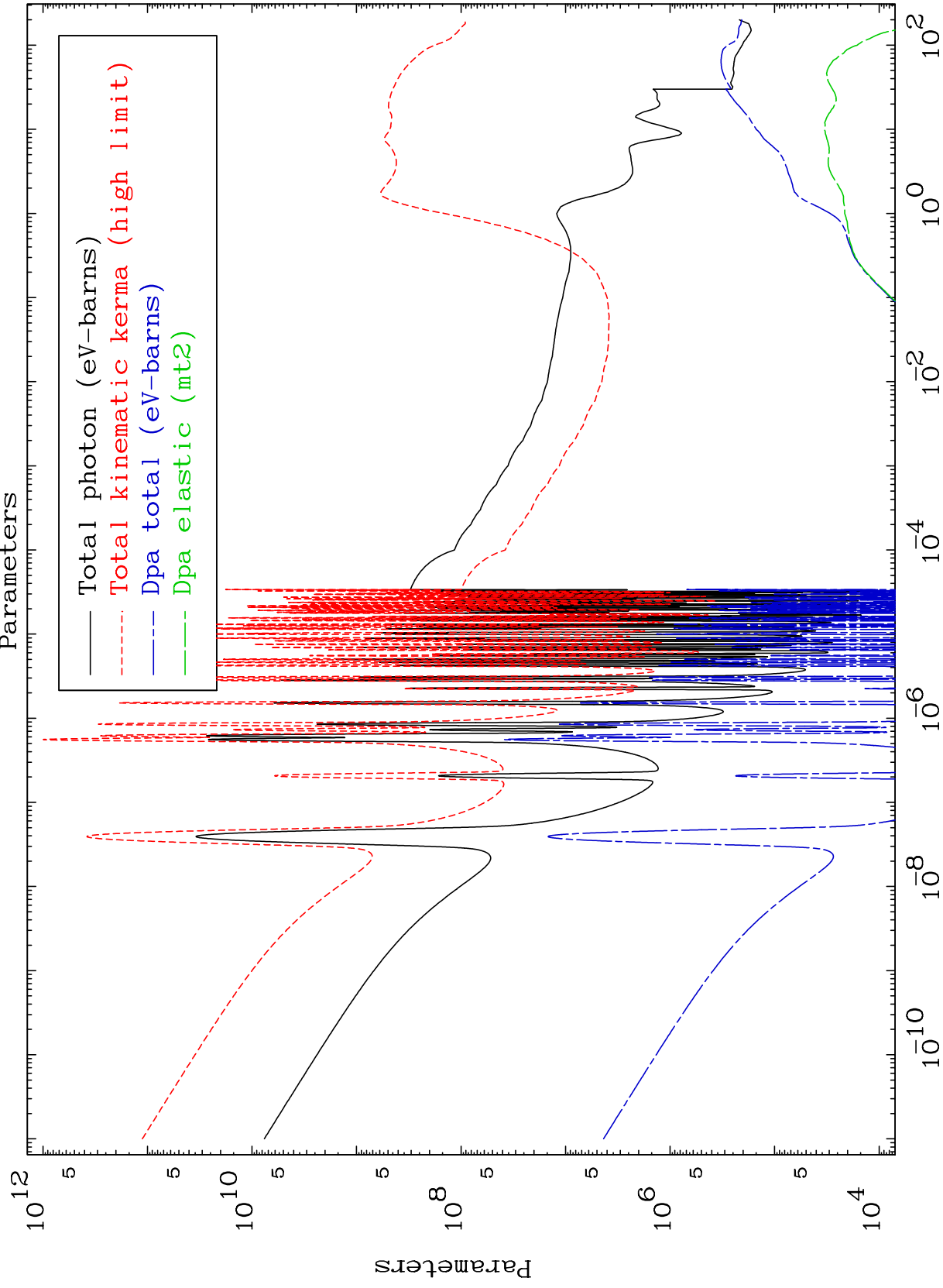
97-Bk-245



MAT 9740

Energy Release
Parameters

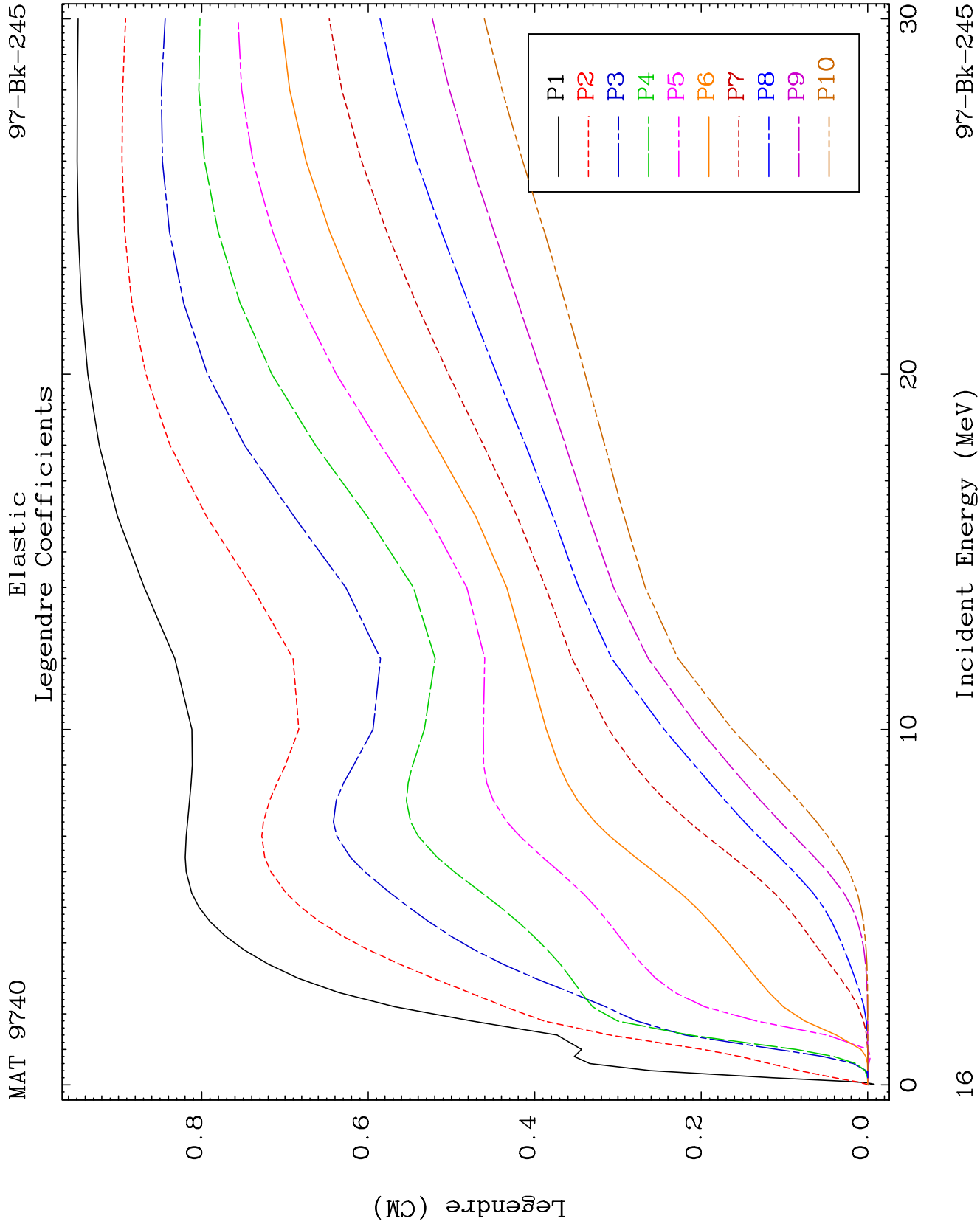
97-Bk-245

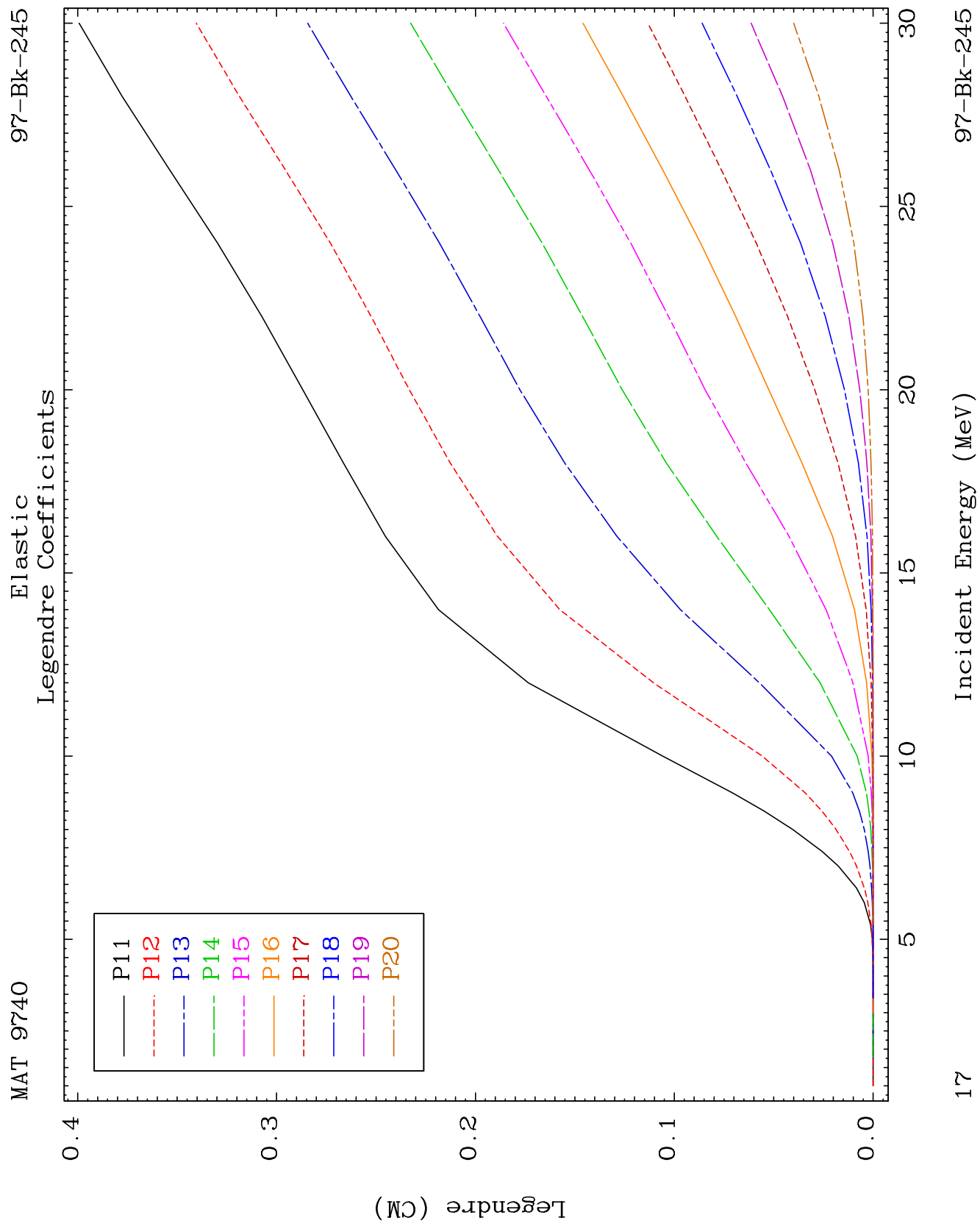


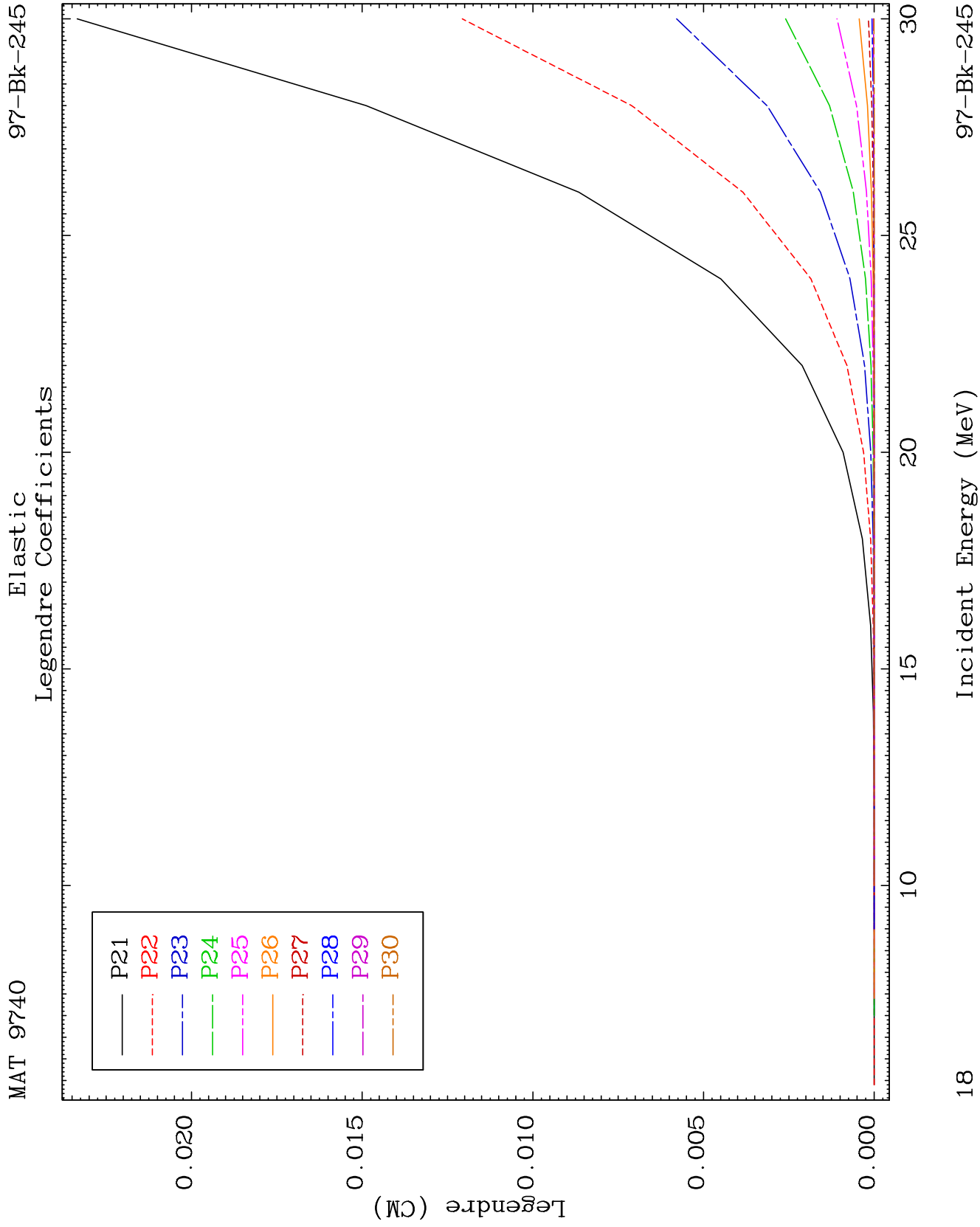
15

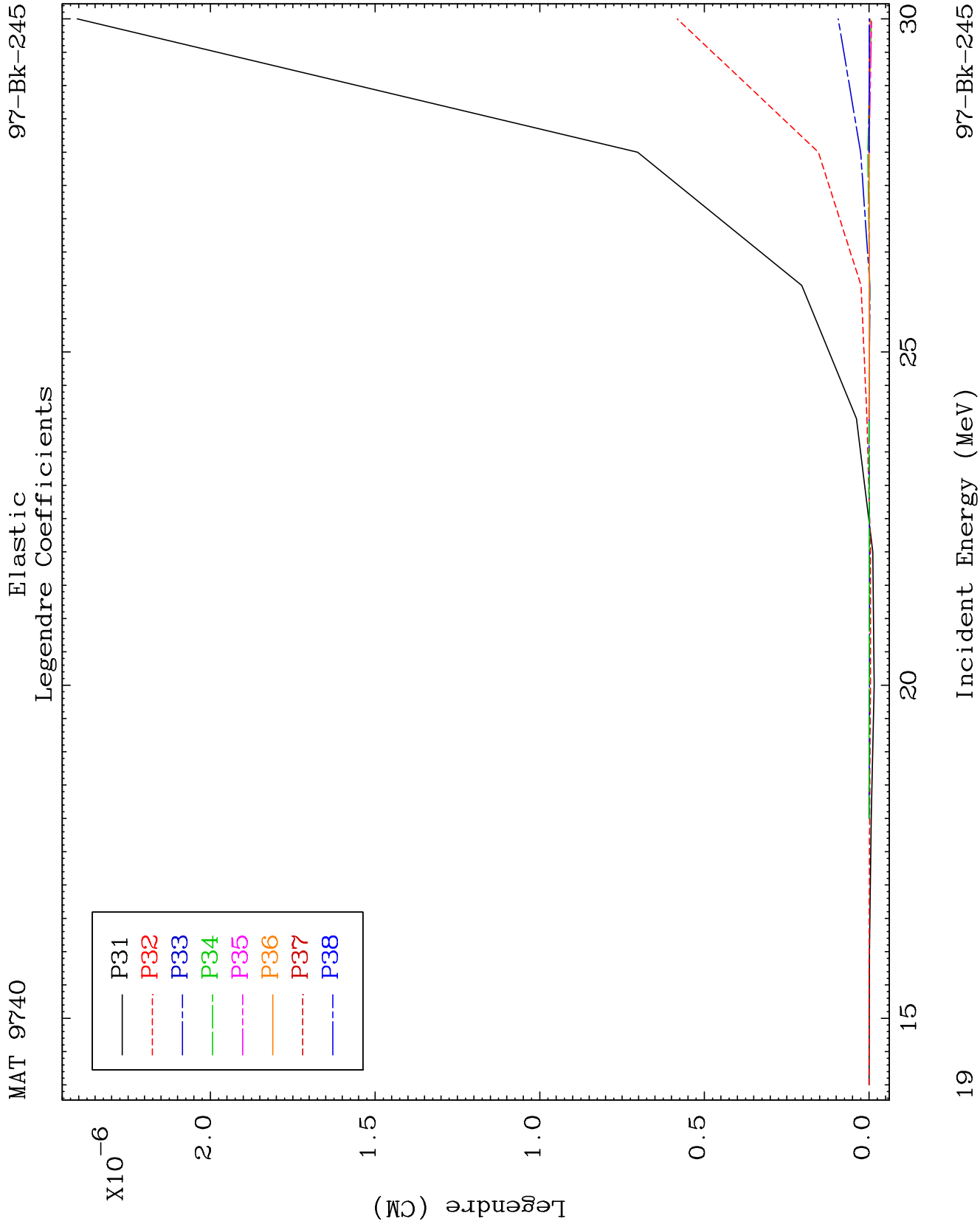
Incident Energy (MeV)

97-Bk-245





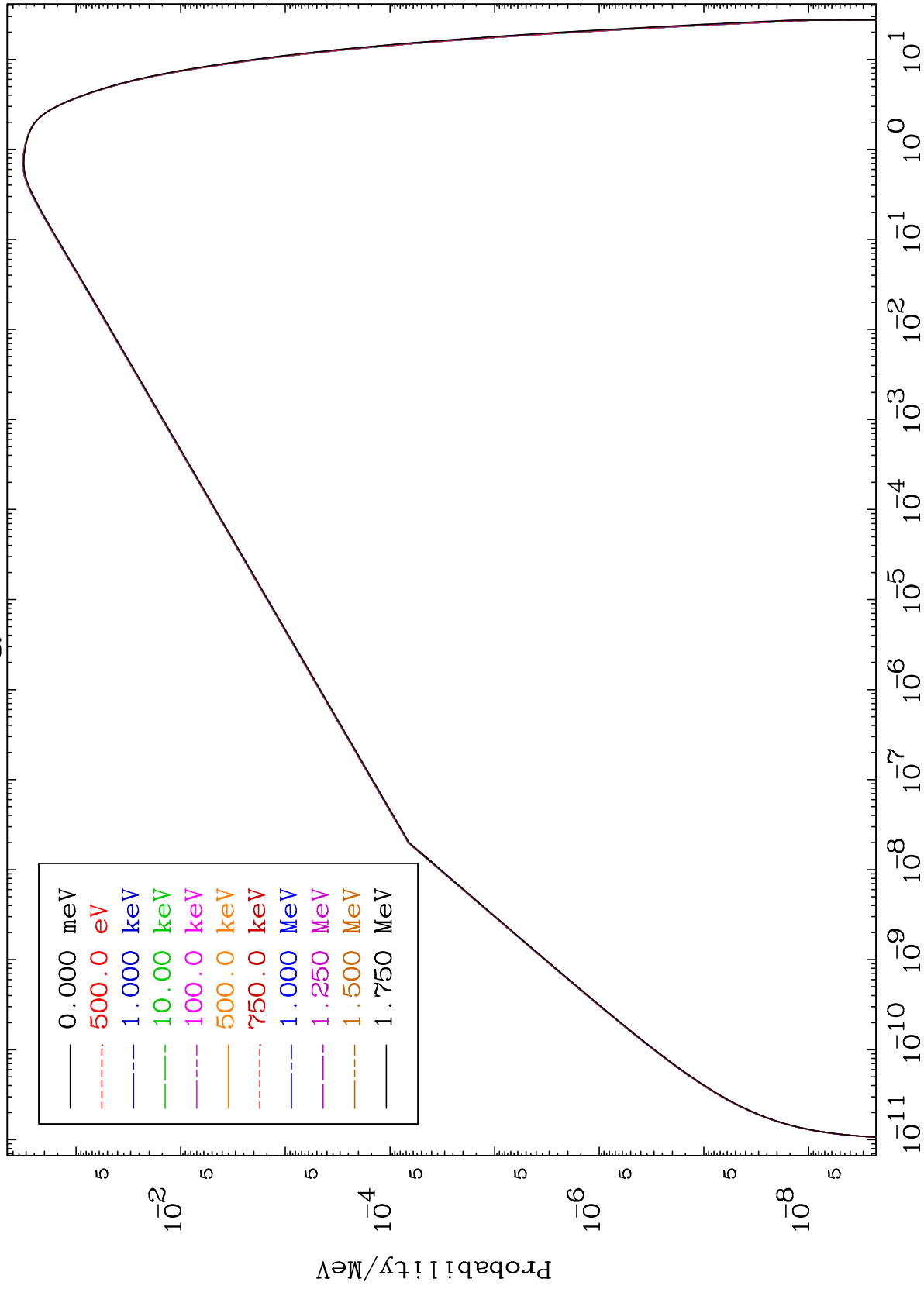




MAT 9740

Fission
Energy Distribution

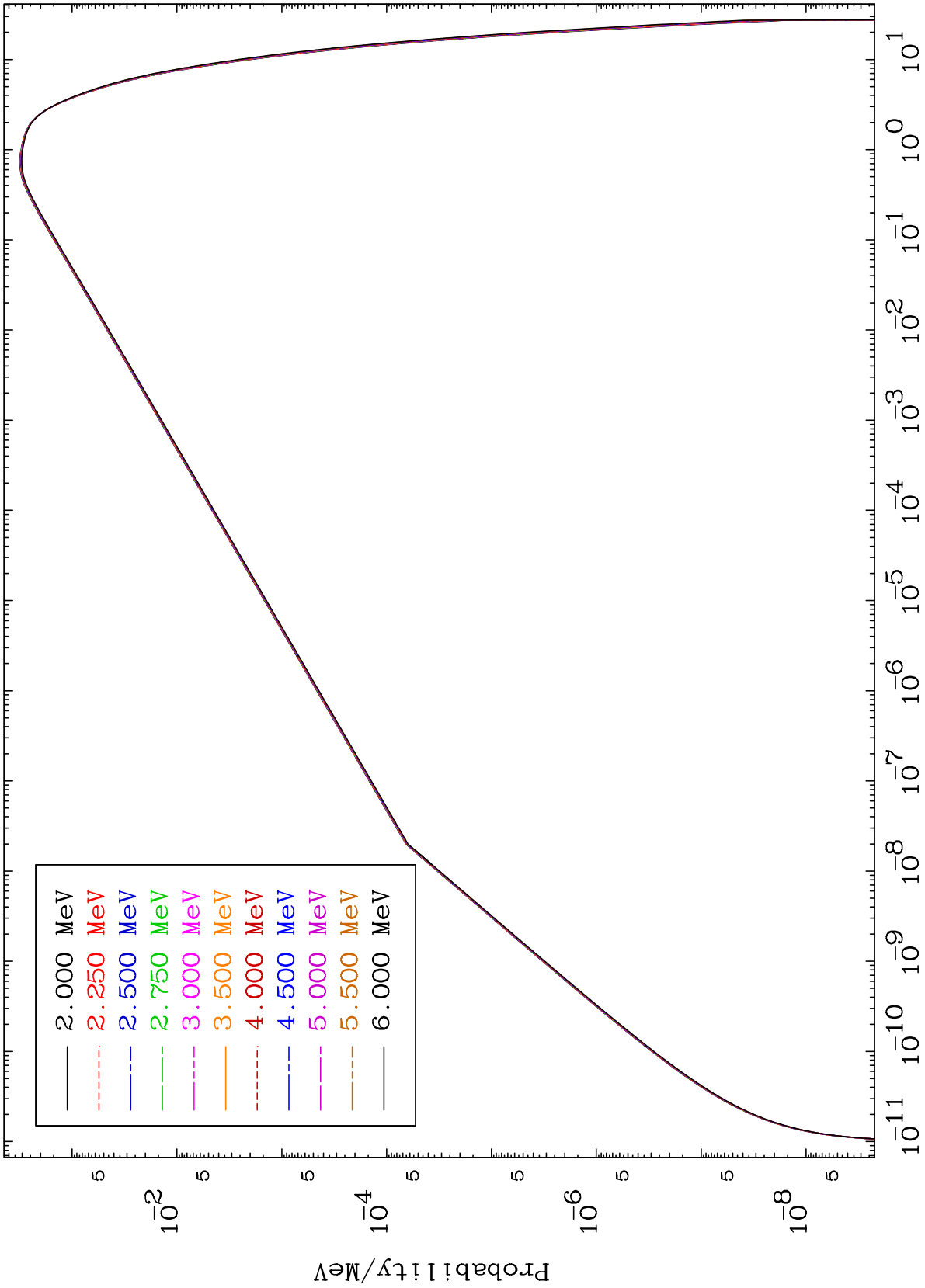
97-Bk-245



MAT 9740

Fission
Energy Distribution

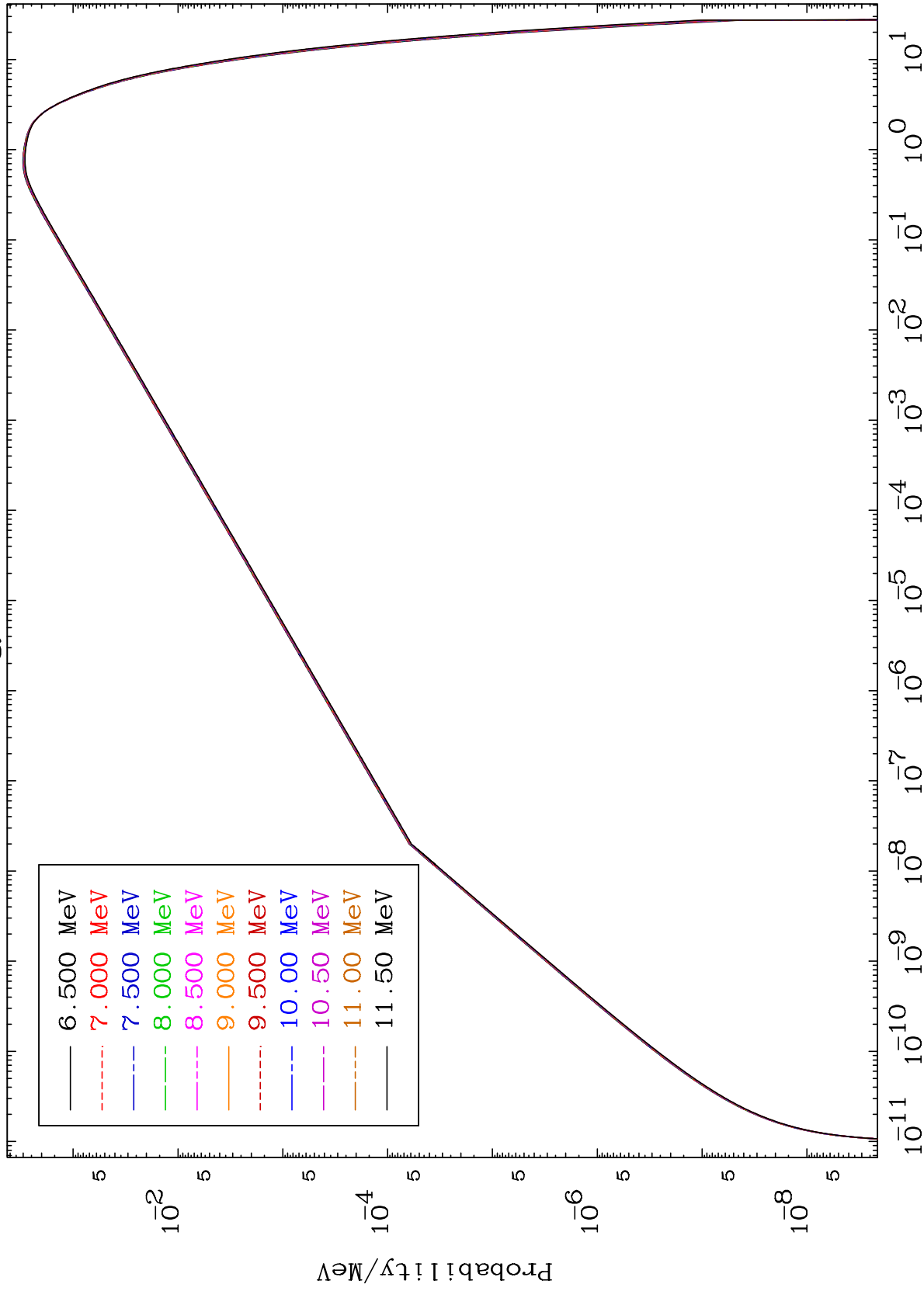
97-Bk-245



MAT 9740

Fission
Energy Distribution

97-Bk-245



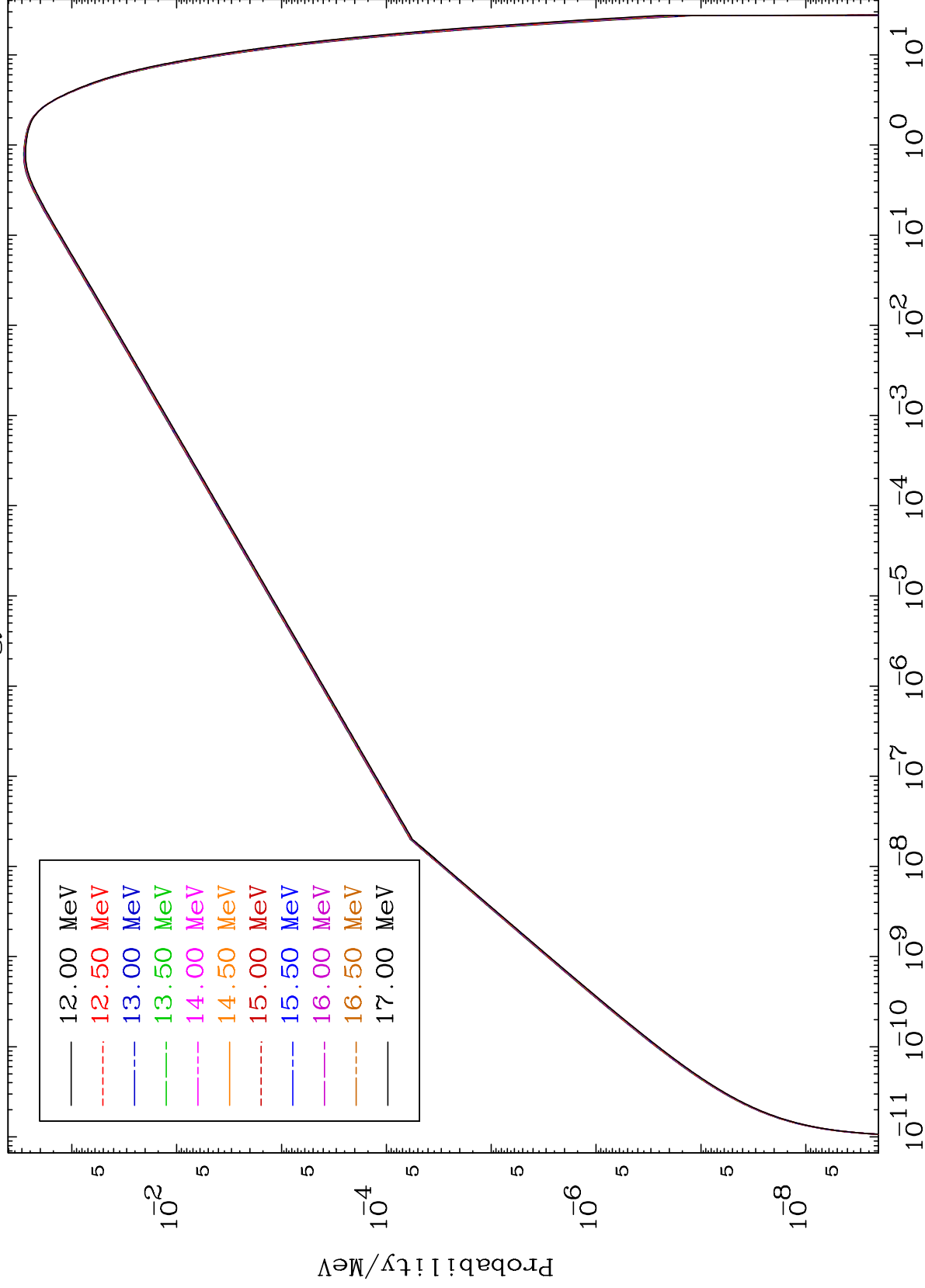
22

97-Bk-245

MAT 9740

Fission
Energy Distribution

97-Bk-245



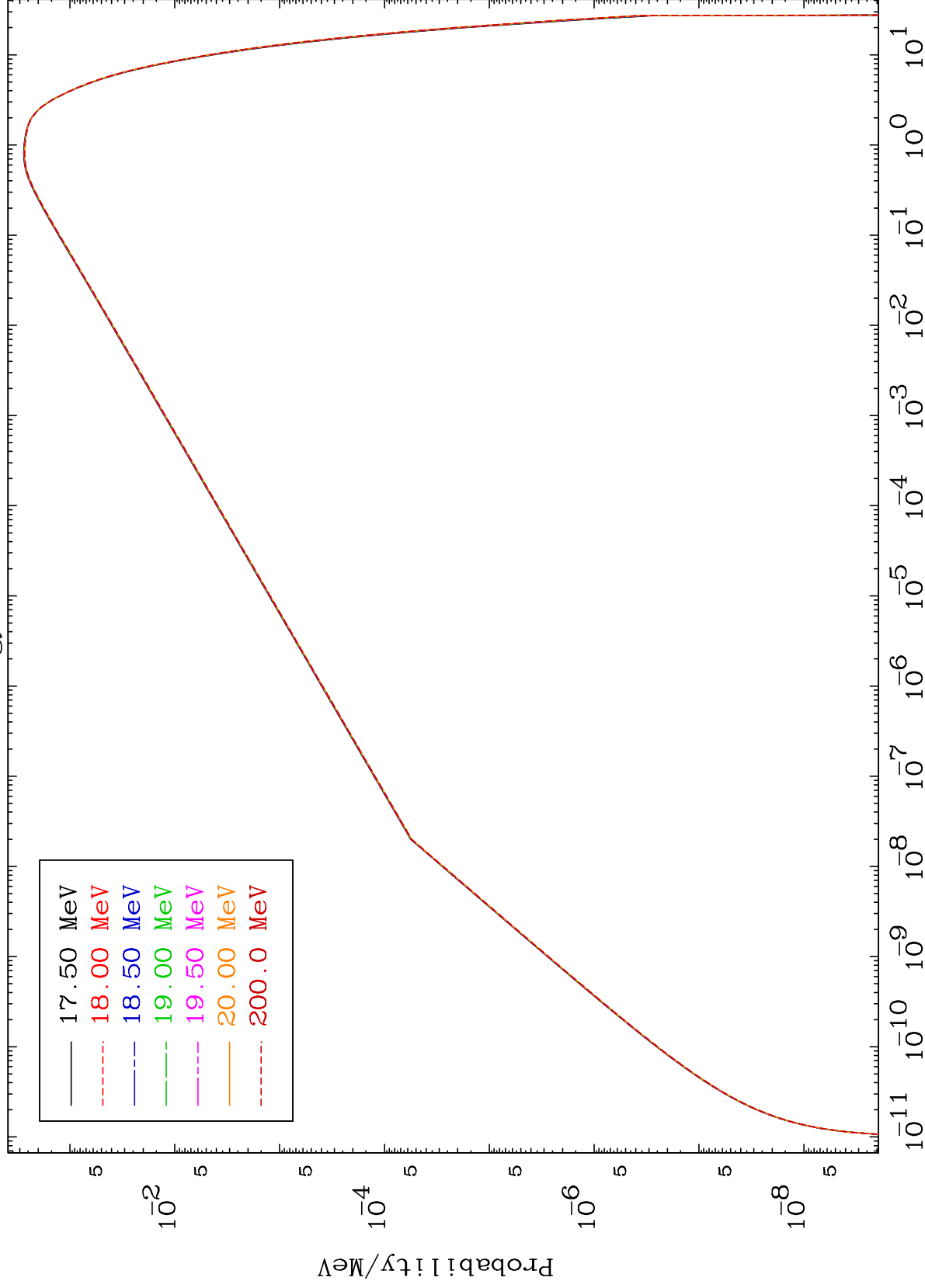
23

97-Bk-245

MAT 9740

Fission
Energy Distribution

97-Bk-245



24

Secondary Energy (MeV)

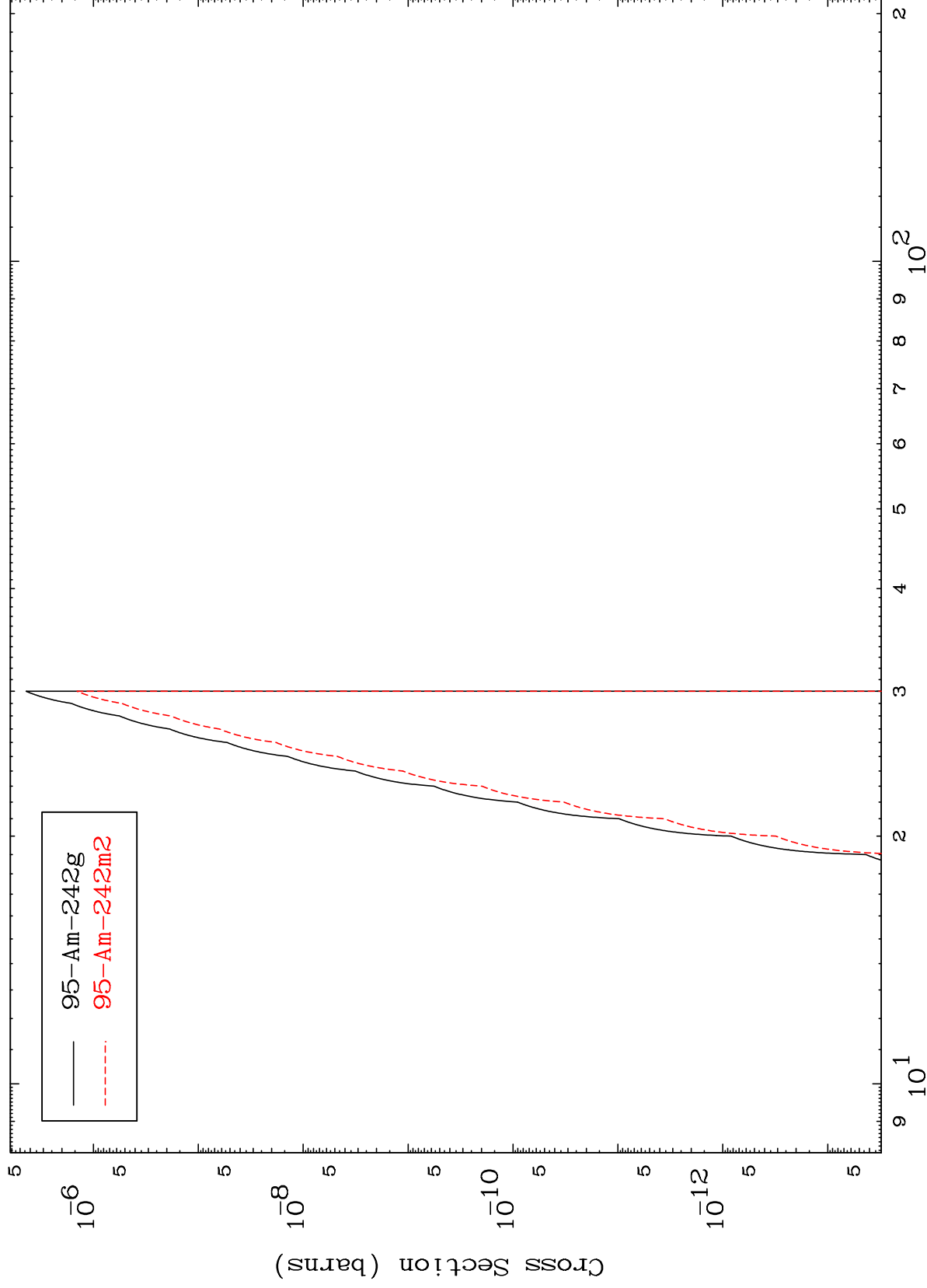
97-Bk-245

MAT 9740

(n,n') He-3

97-Bk-245

Radionuclide Production Cross Section



25

Incident Energy (MeV)

97-Bk-245

