

Program EVALPLOT
(Version 2018-1)

by

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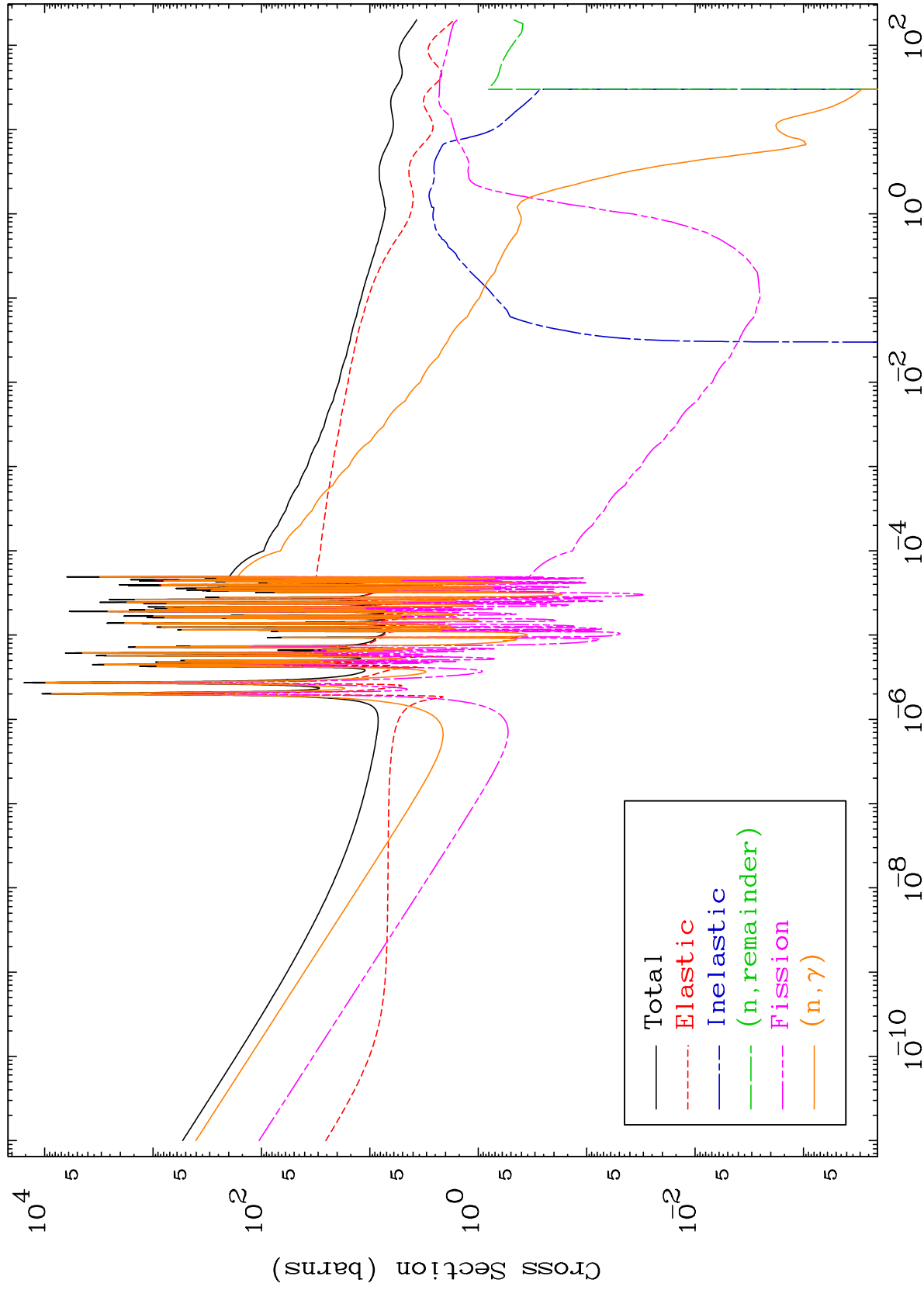
Press Mouse Button to Start

MAT 9746

Major

293 Kelvin Cross Sections

97-Bk-247



1

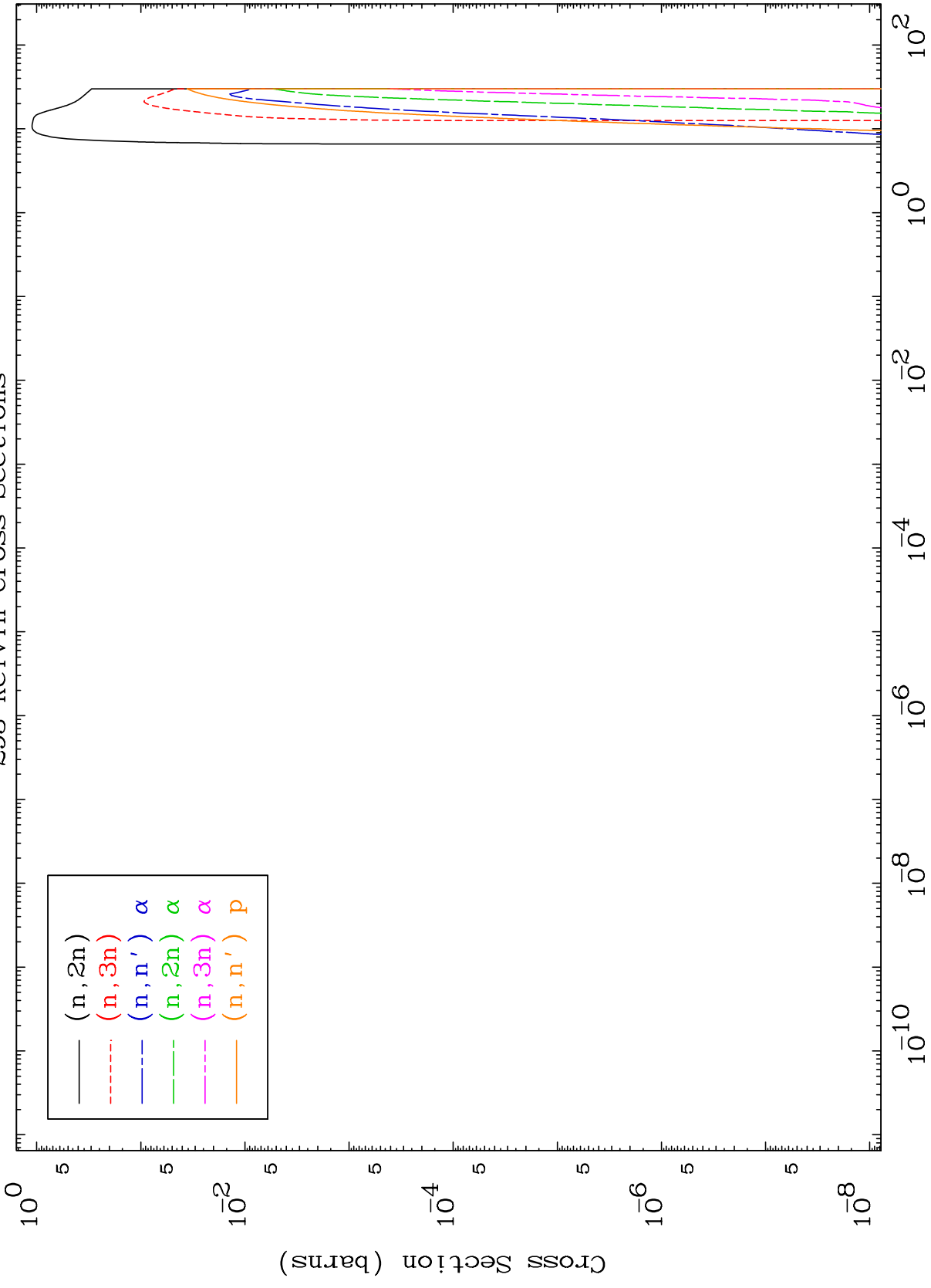
Incident Energy (MeV)

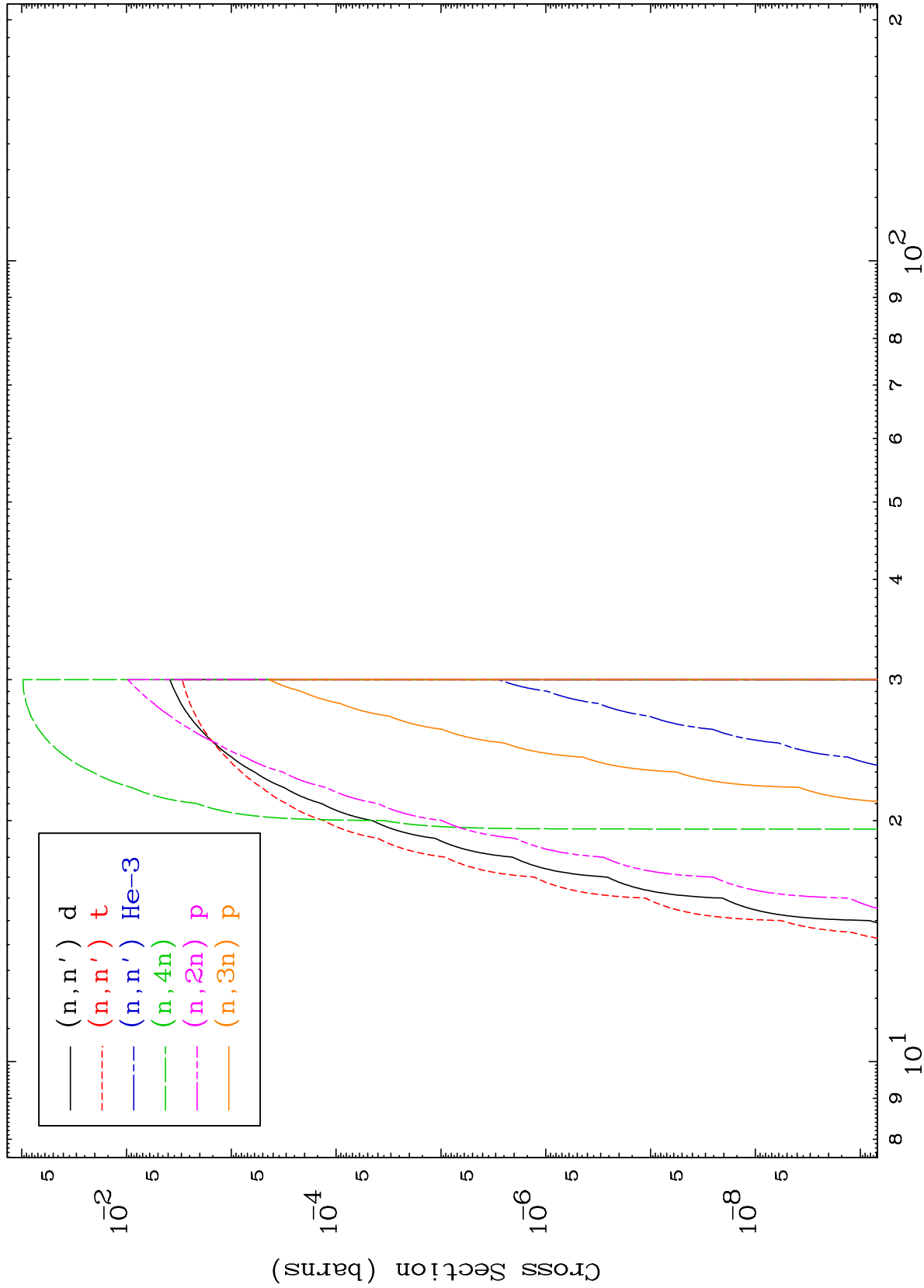
97-Bk-247

MAT 9746

Neutron Production
293 Kelvin Cross Sections

97-Bk-247

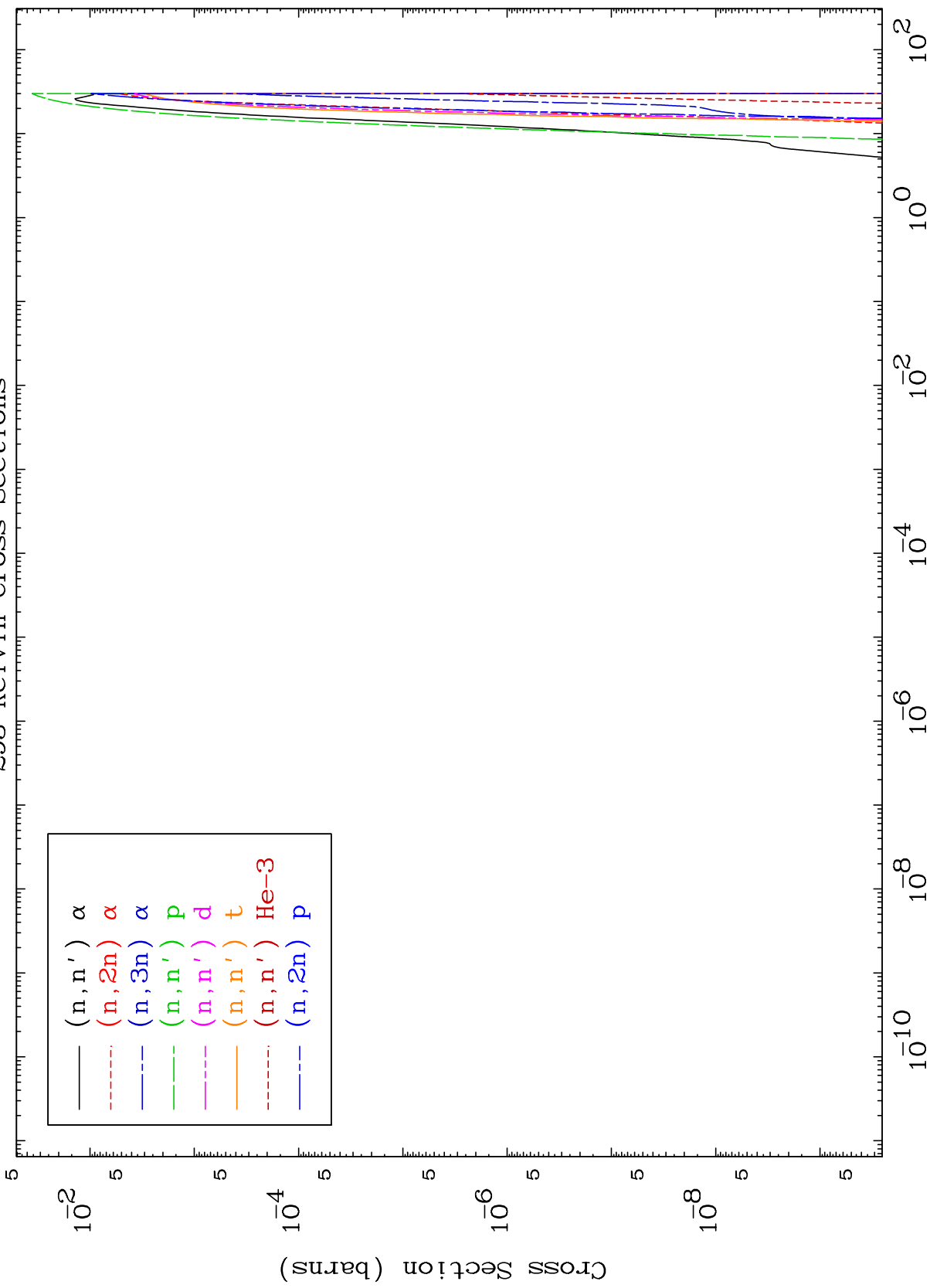




MAT 9746

Charged Particle
293 Kelvin Cross Sections

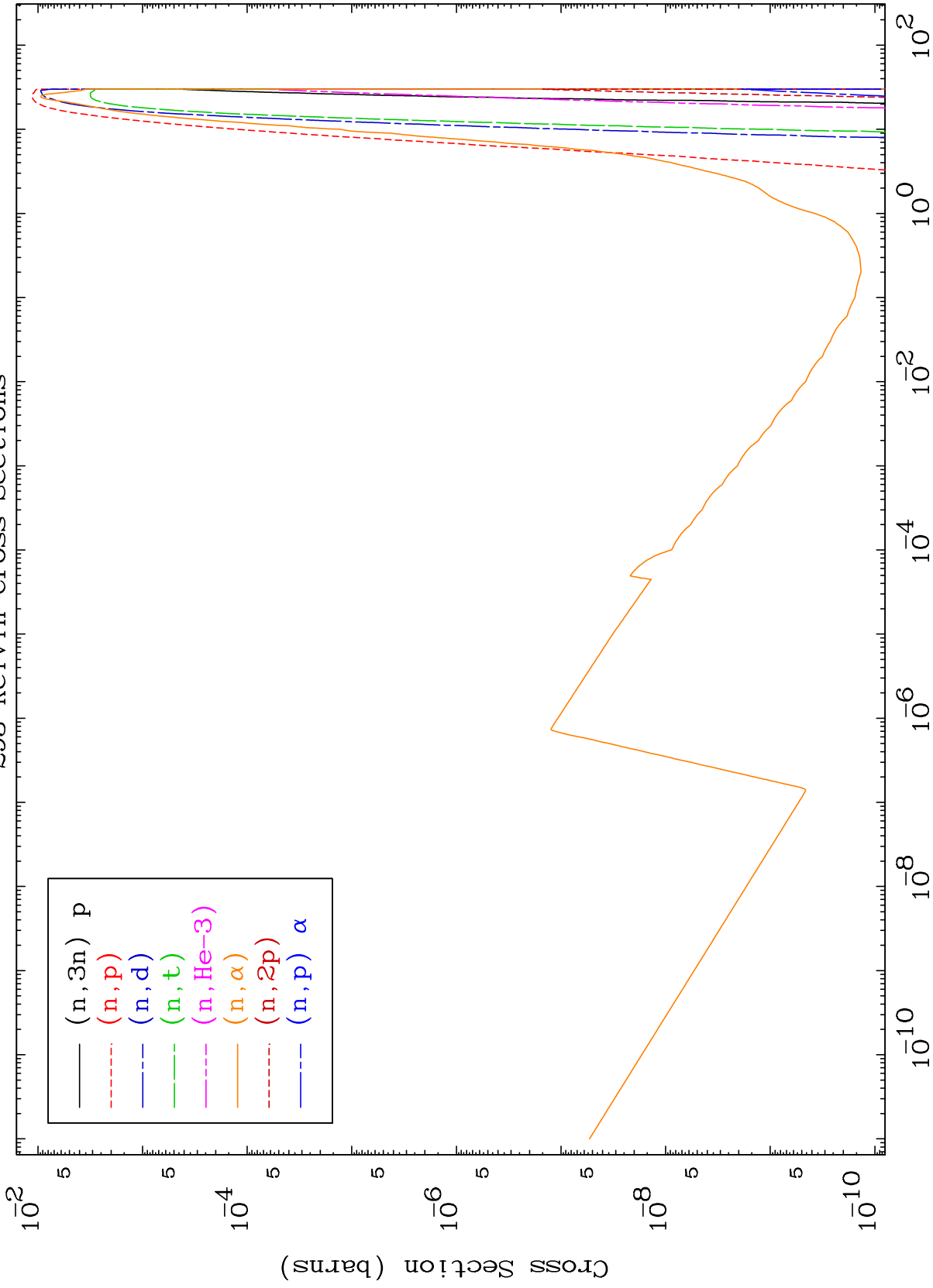
97-Bk-247



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Charged Particle
293 Kelvin Cross Sections

97-Bk-247



5

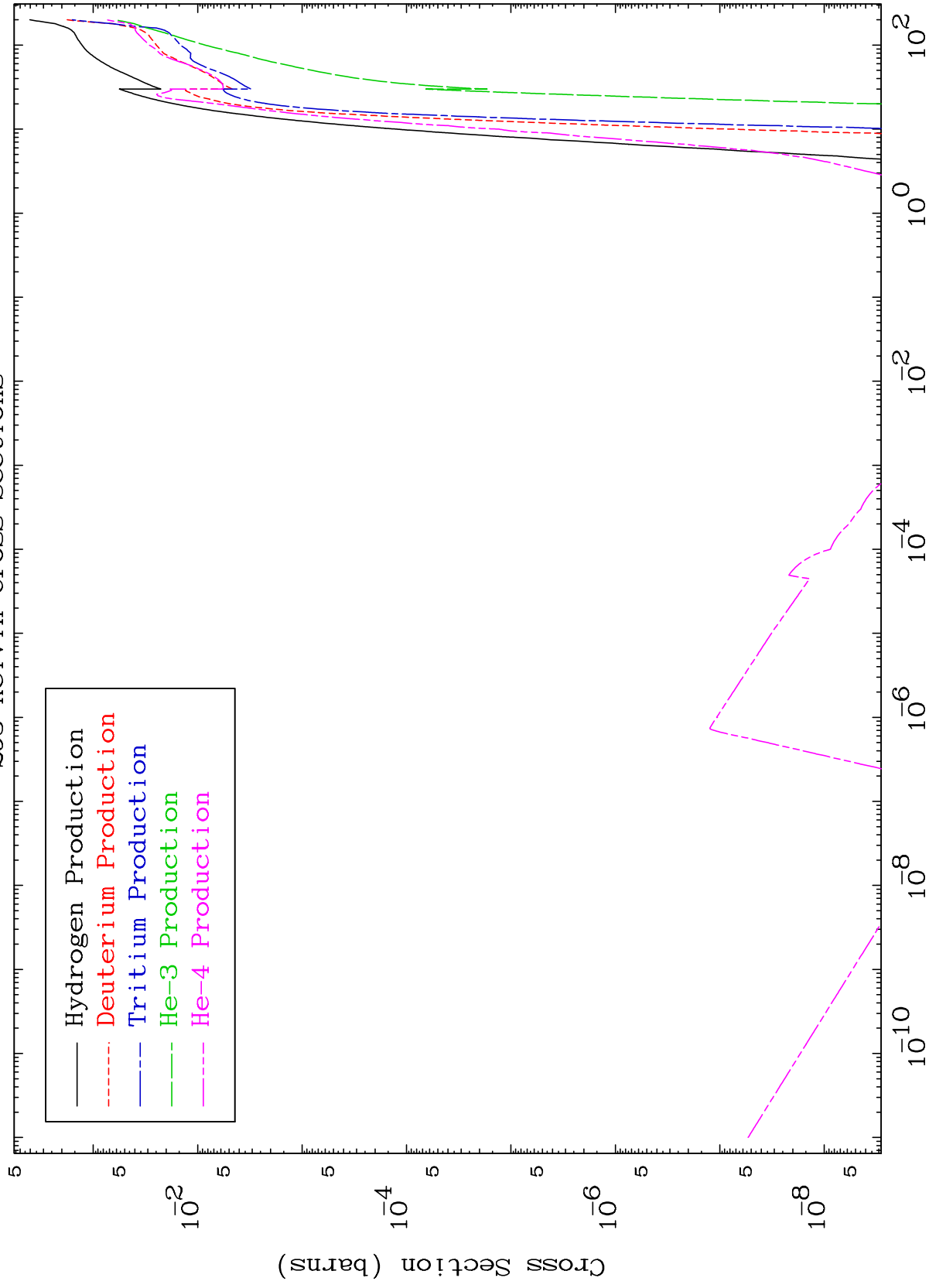
Incident Energy (MeV)

97-Bk-247

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Particle Production
293 Kelvin Cross Sections

97-Bk-247



6

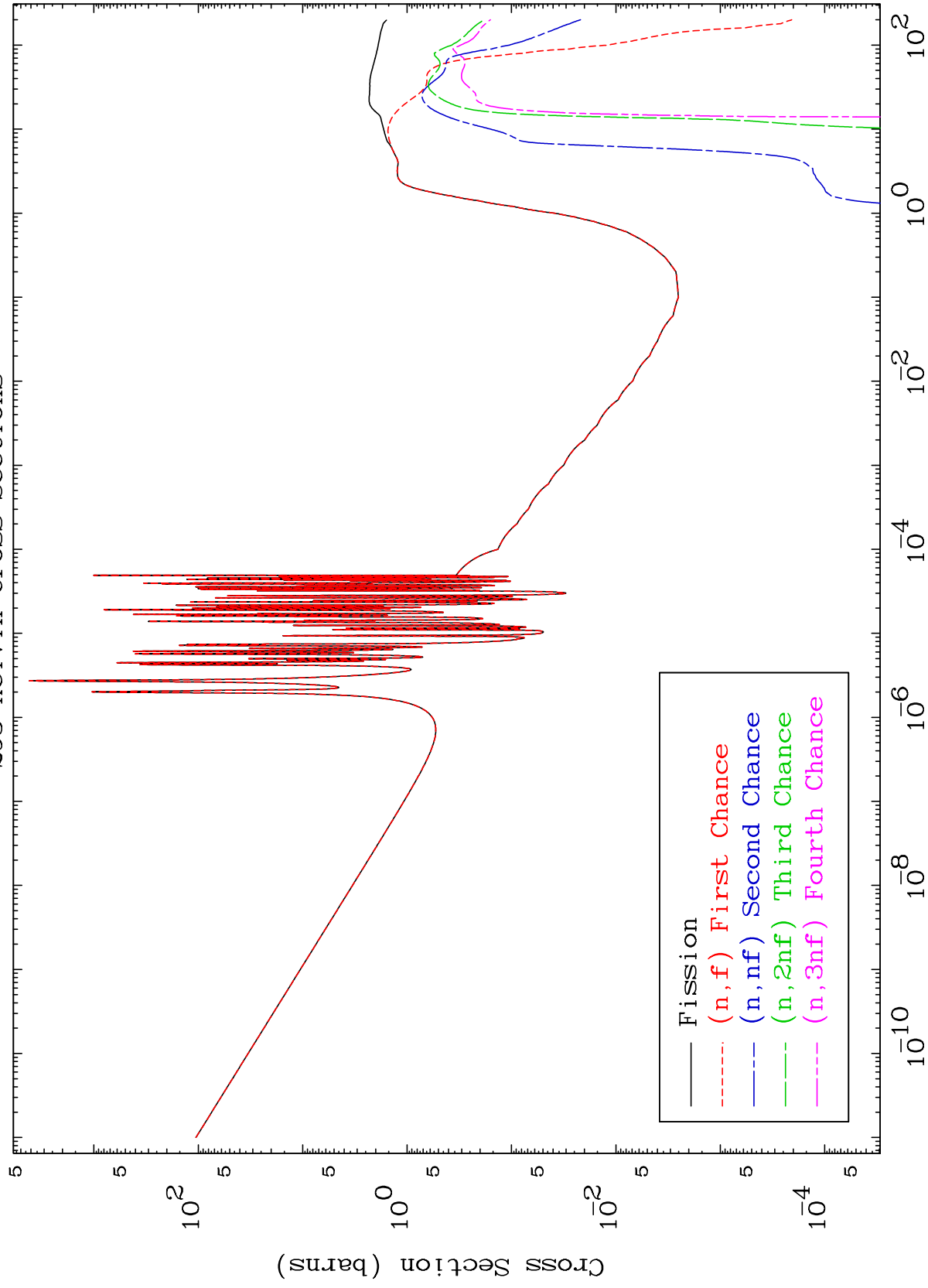
Incident Energy (MeV)

97-Bk-247

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Fission
293 Kelvin Cross Sections

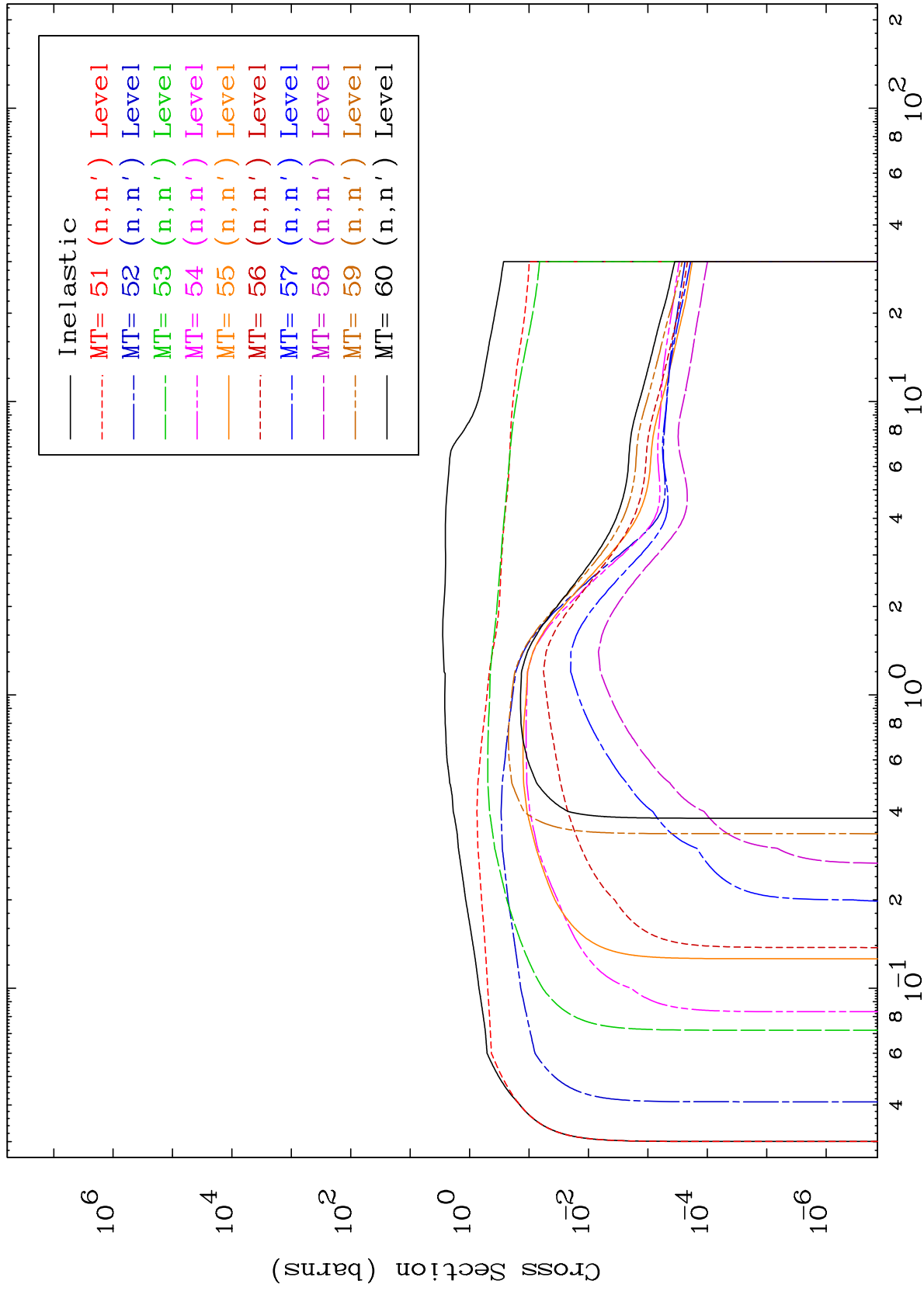
97-Bk-247



7

Incident Energy (MeV)

97-Bk-247

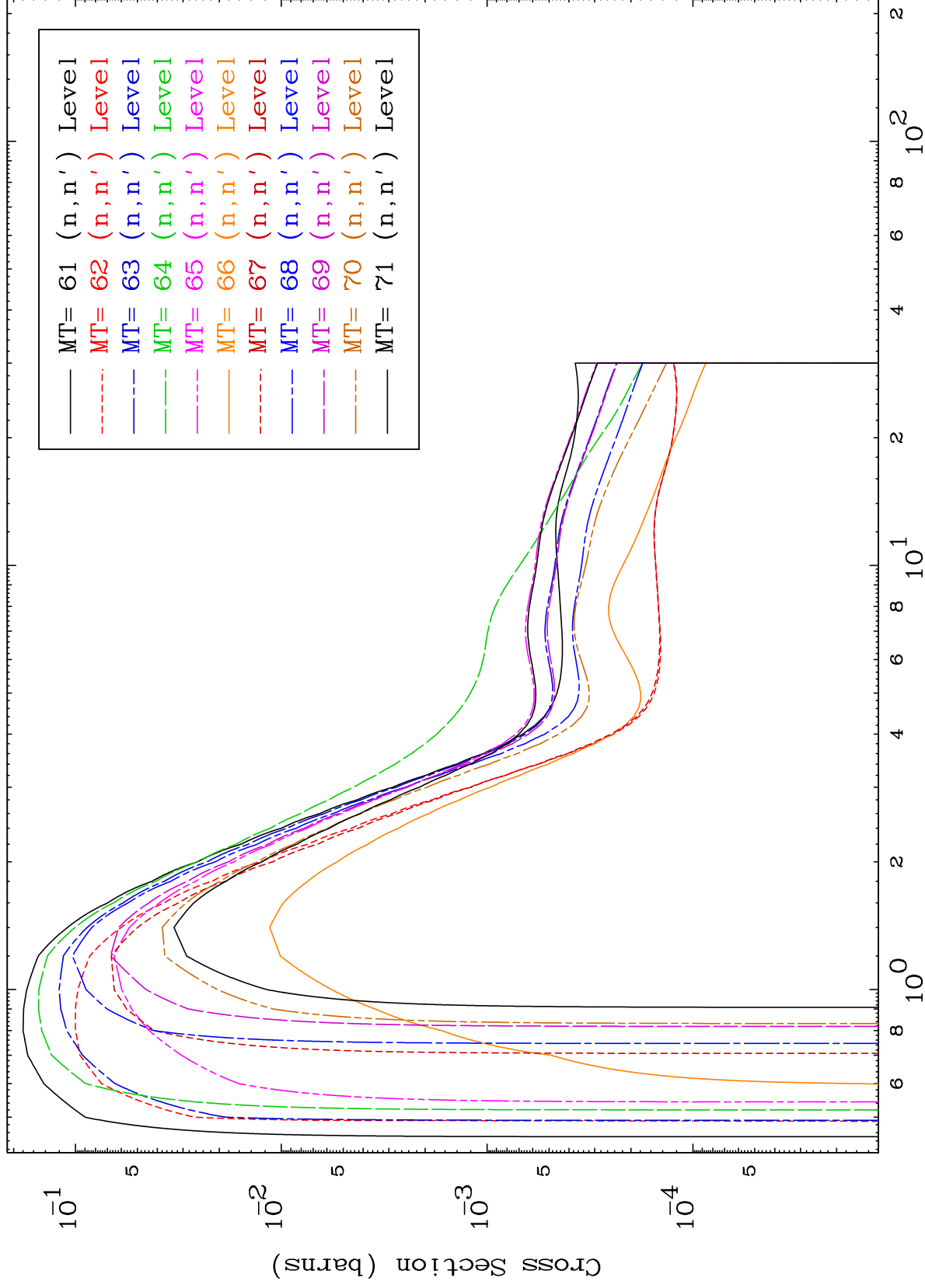


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(n,n') Level

97-Bk-247

293 Kelvin Cross Sections

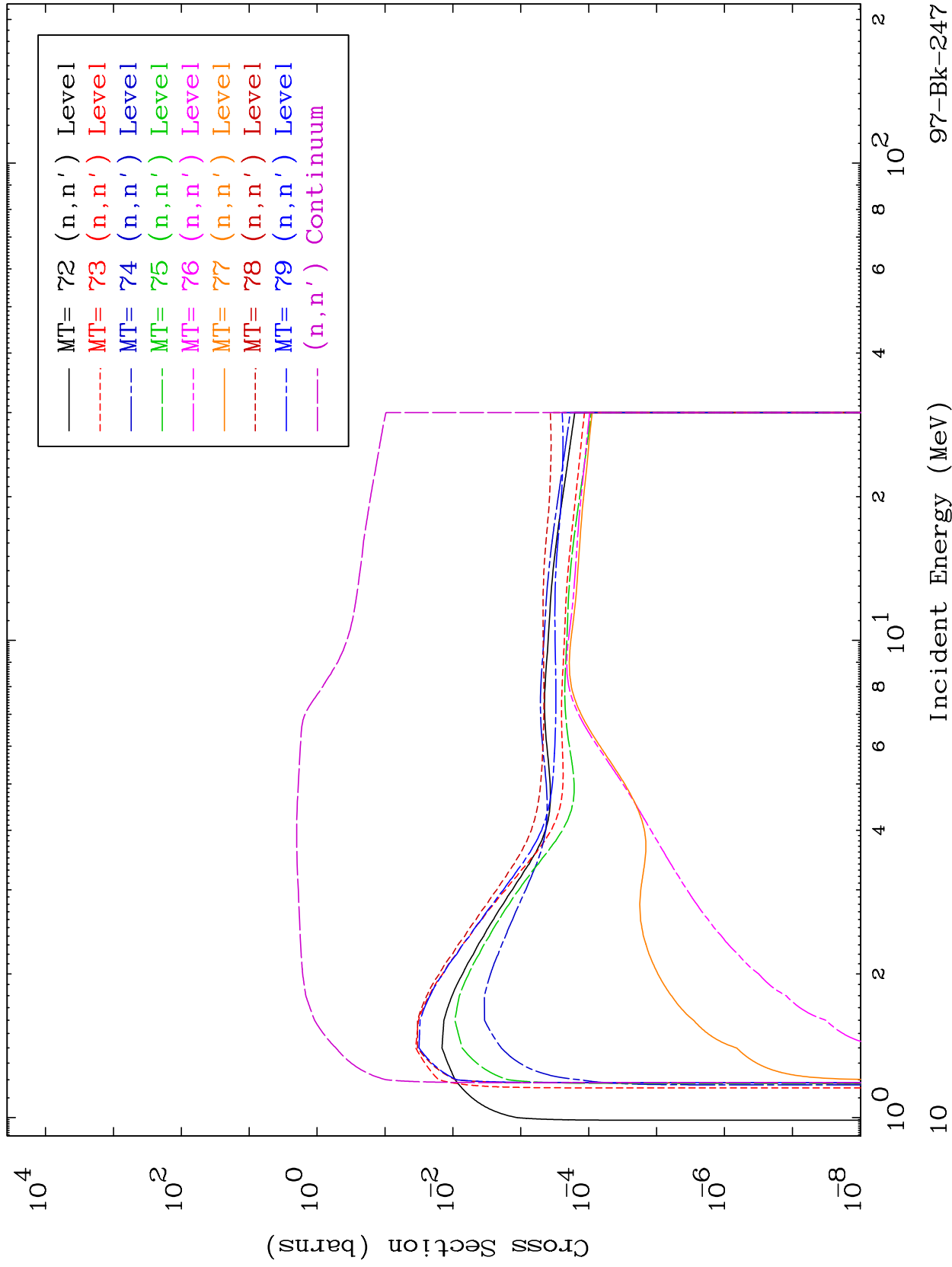


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(n,n') Level

293 Kelvin Cross Sections

97-Bk-247

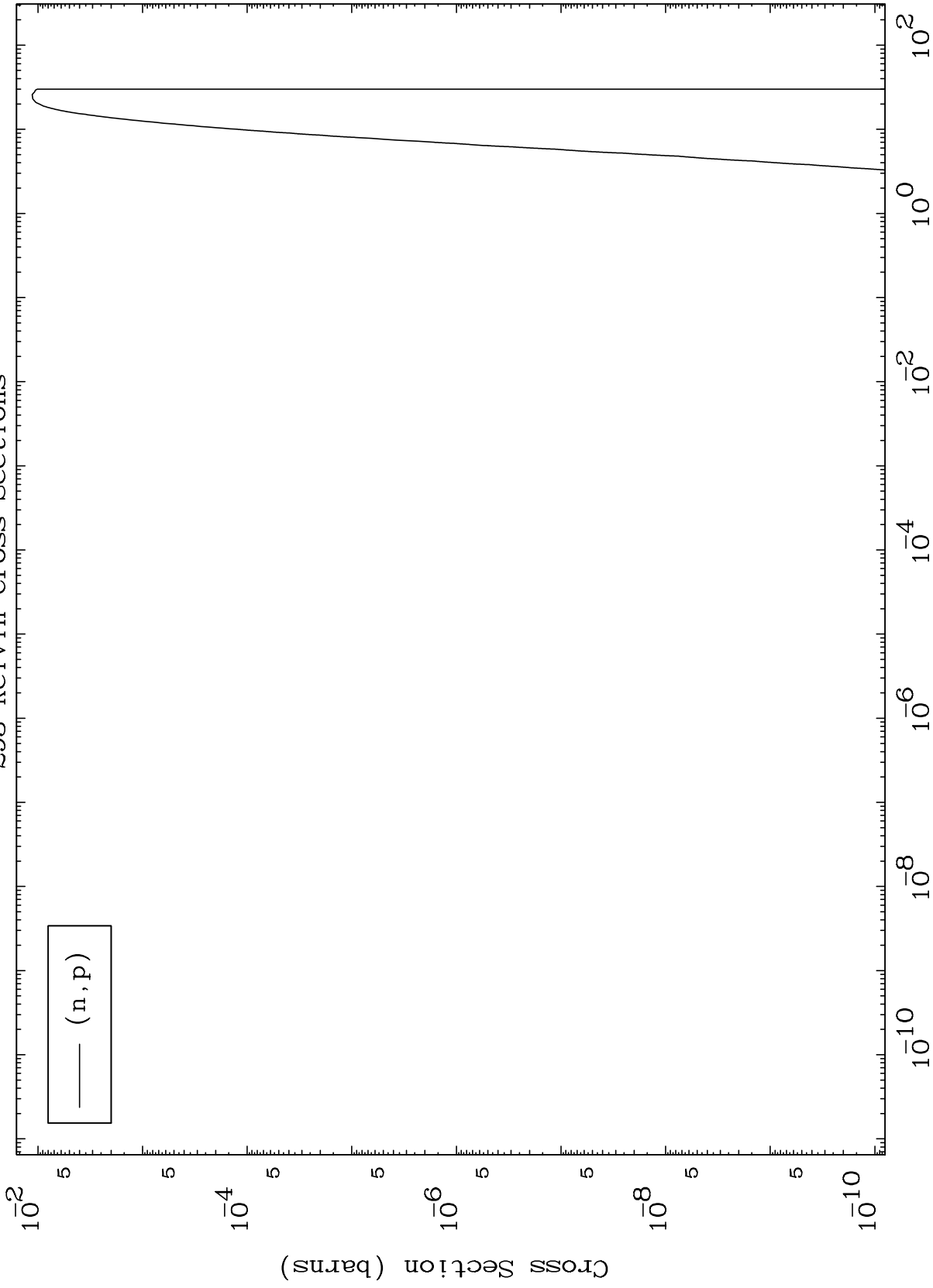


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(n,p) Levels
293 Kelvin Cross Sections

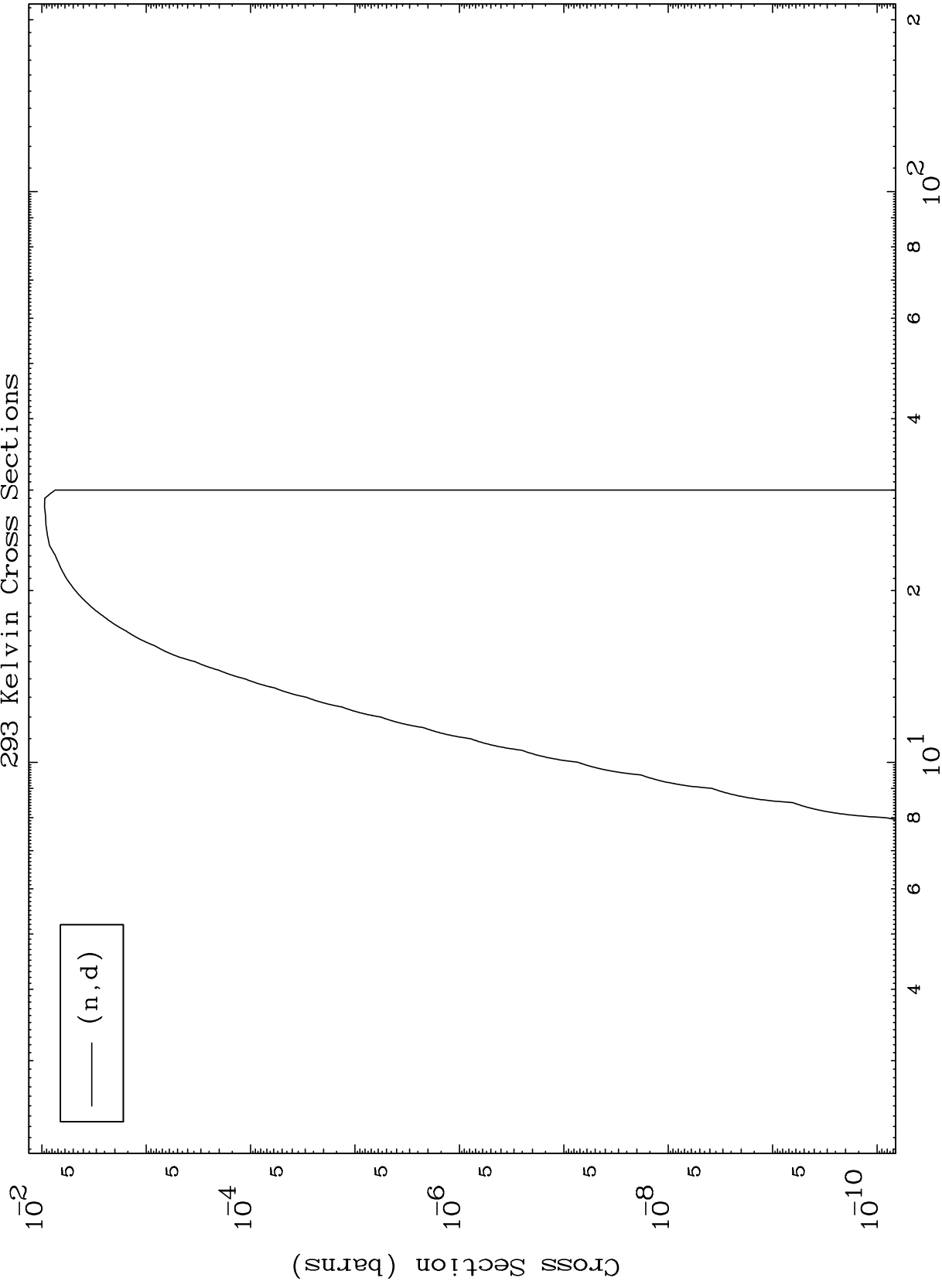
97-Bk-247



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(n,d) Levels
293 Kelvin Cross Sections



12

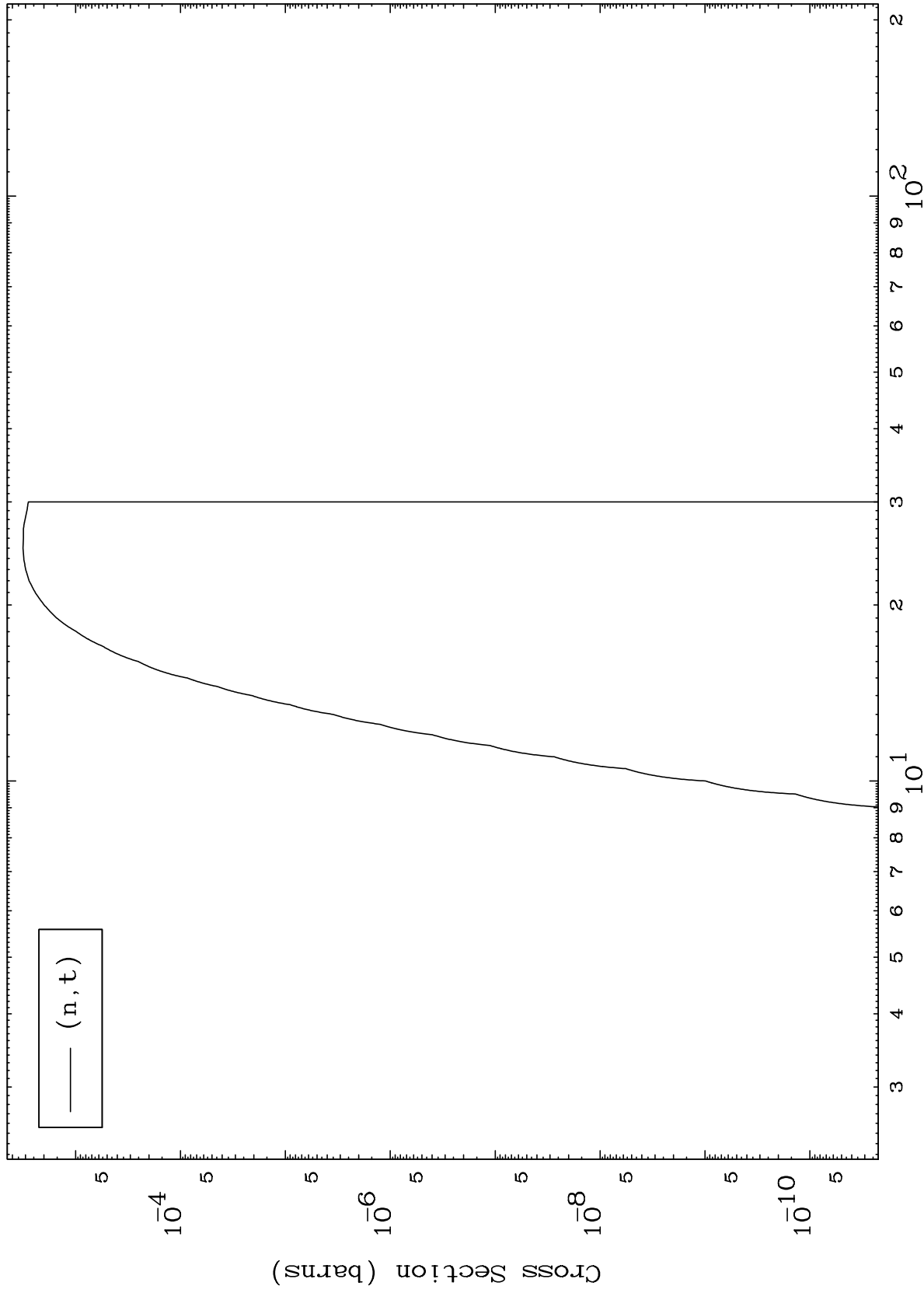
Incident Energy (MeV)

97-Bk-247

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97-Bk-247

(n,t) Levels
293 Kelvin Cross Sections



13

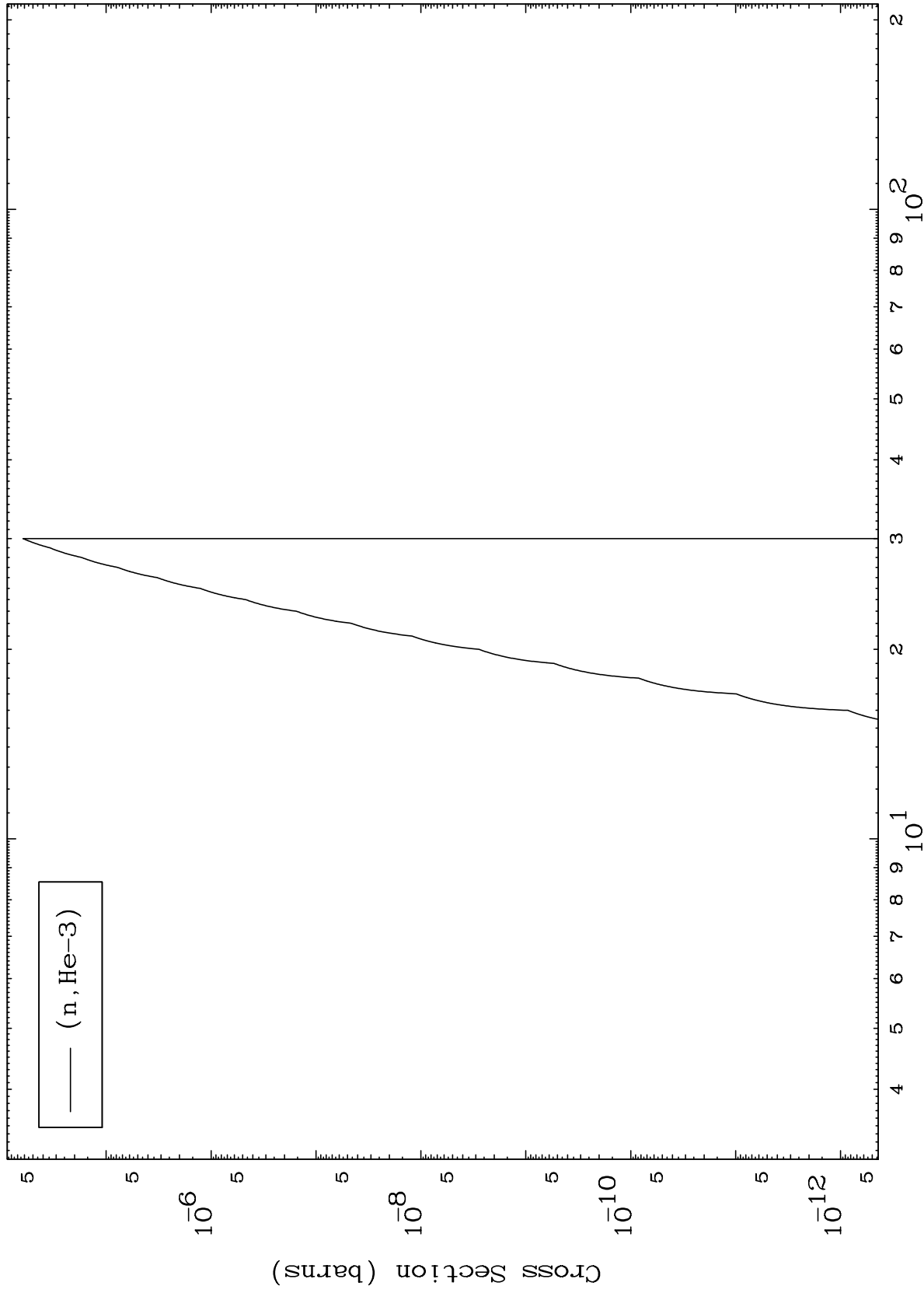
Incident Energy (MeV)

97-Bk-247

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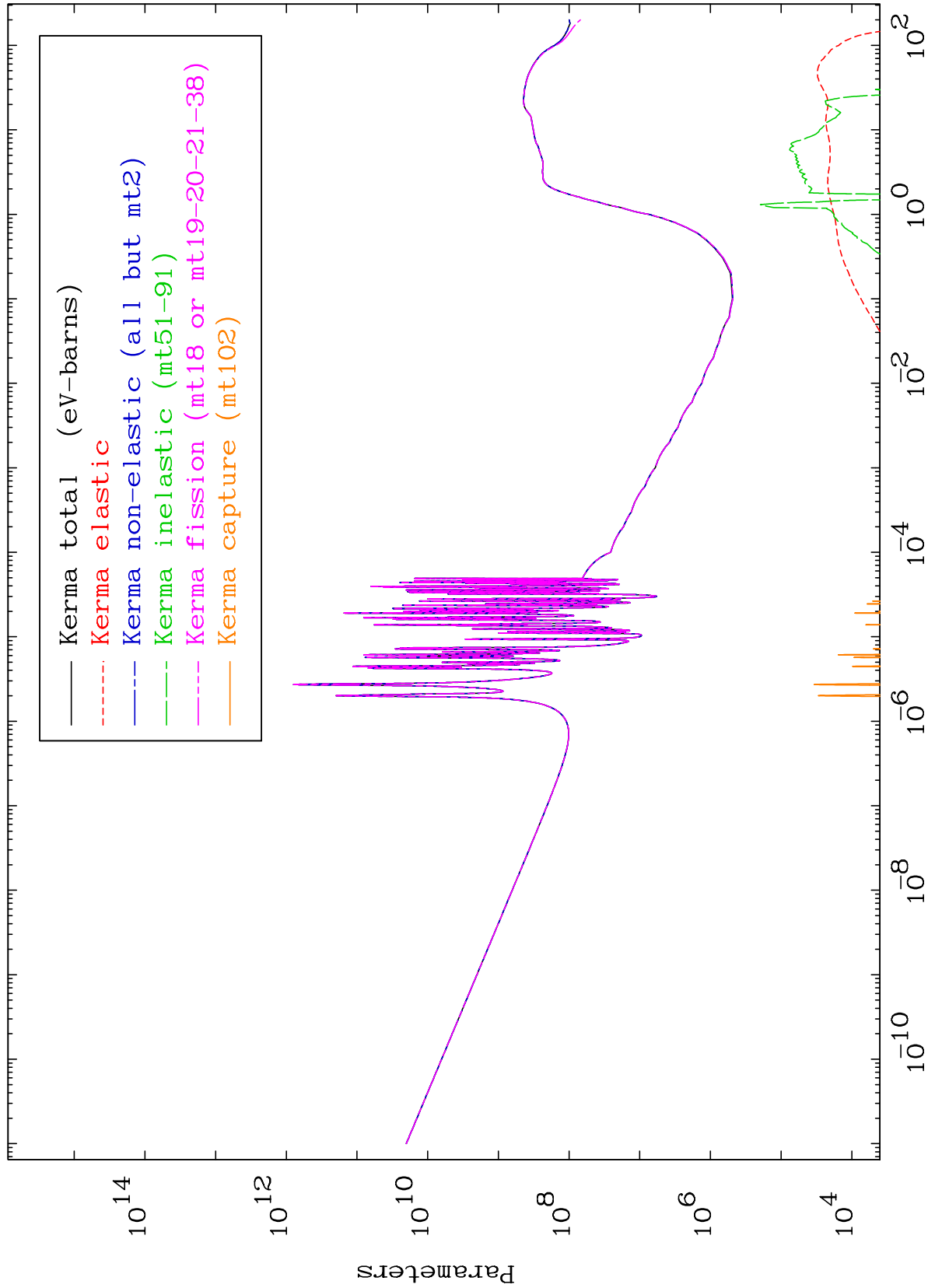
(n,He3) Levels
293 Kelvin Cross Sections

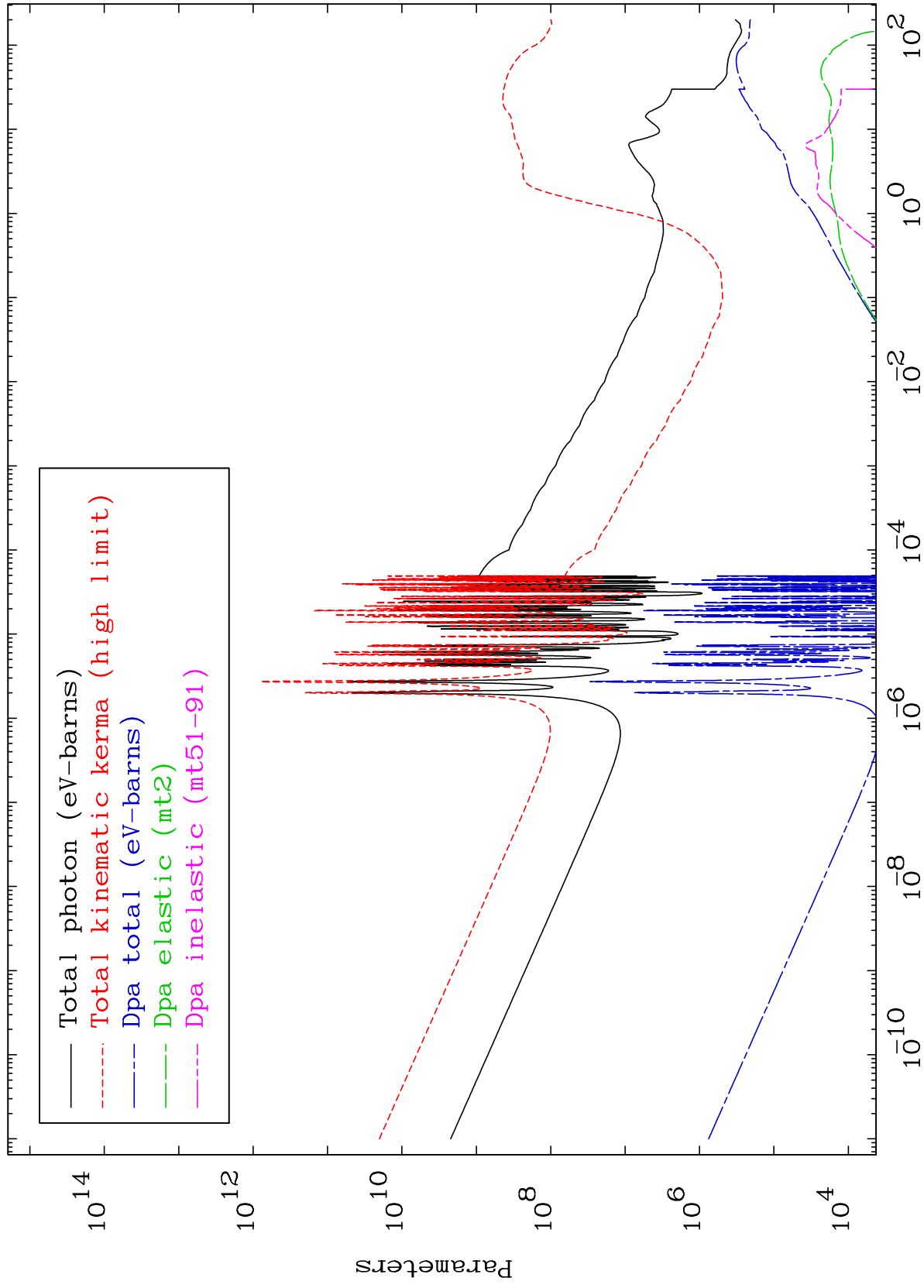


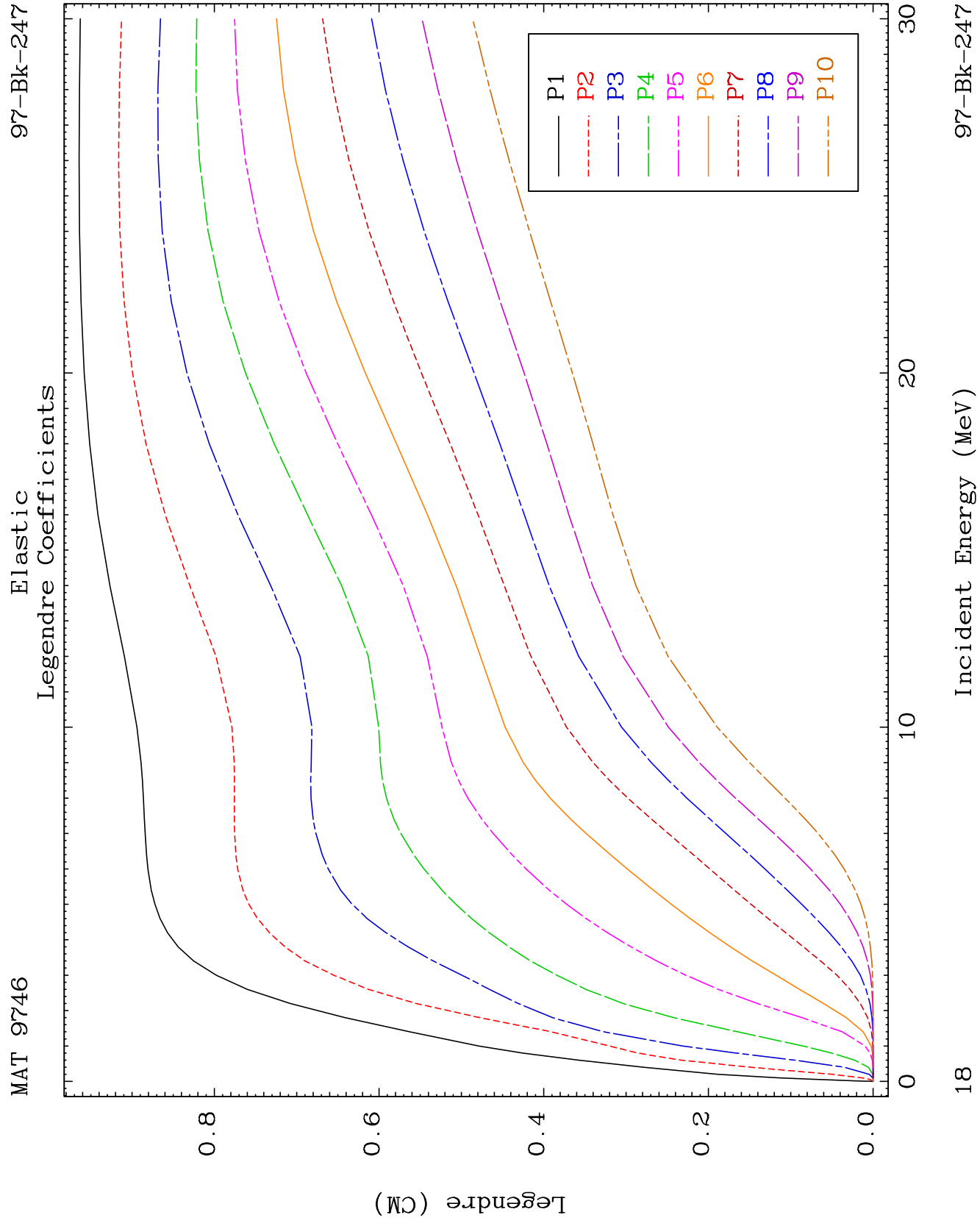
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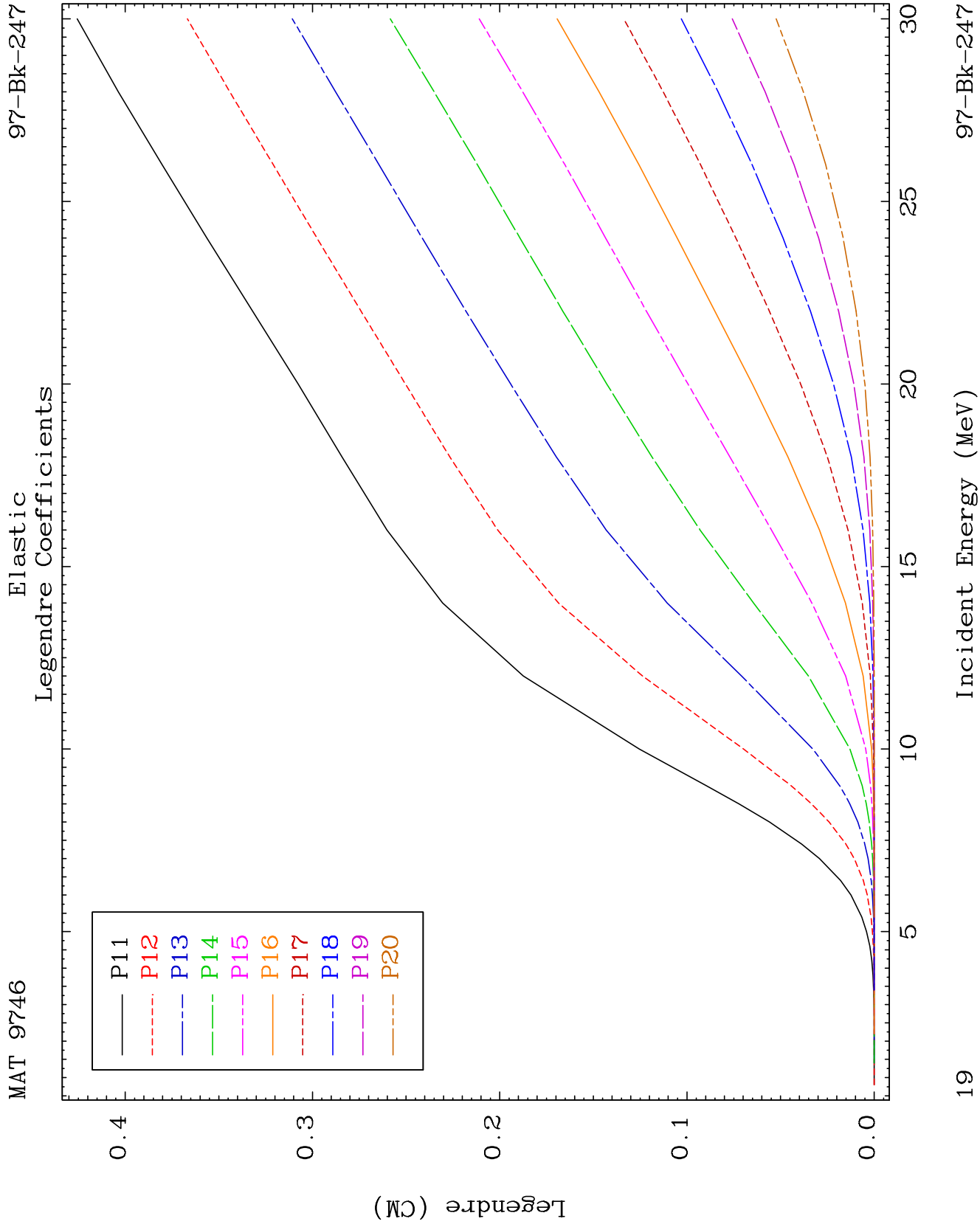
Incident Energy (MeV)

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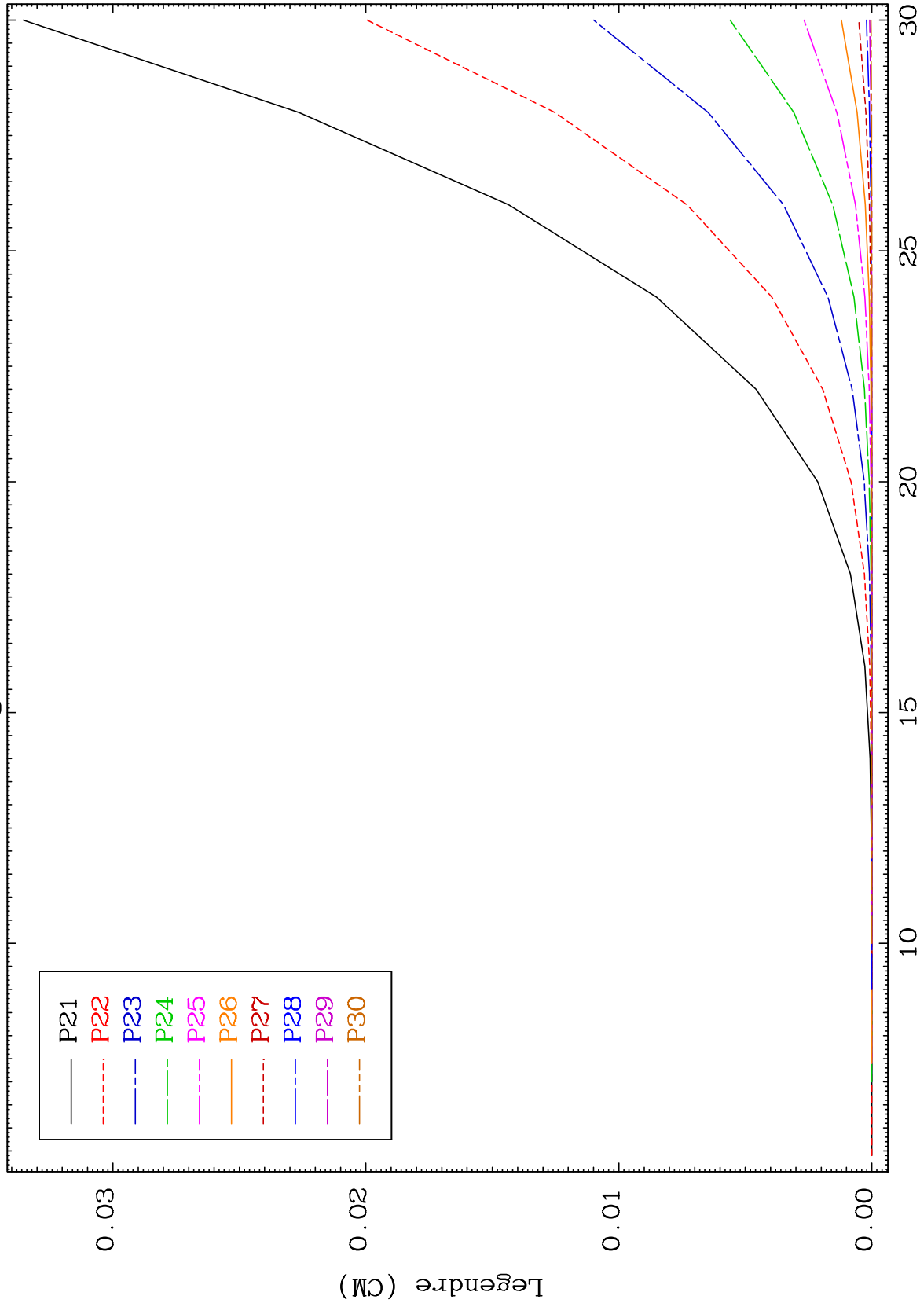




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Elastic
Legendre Coefficients

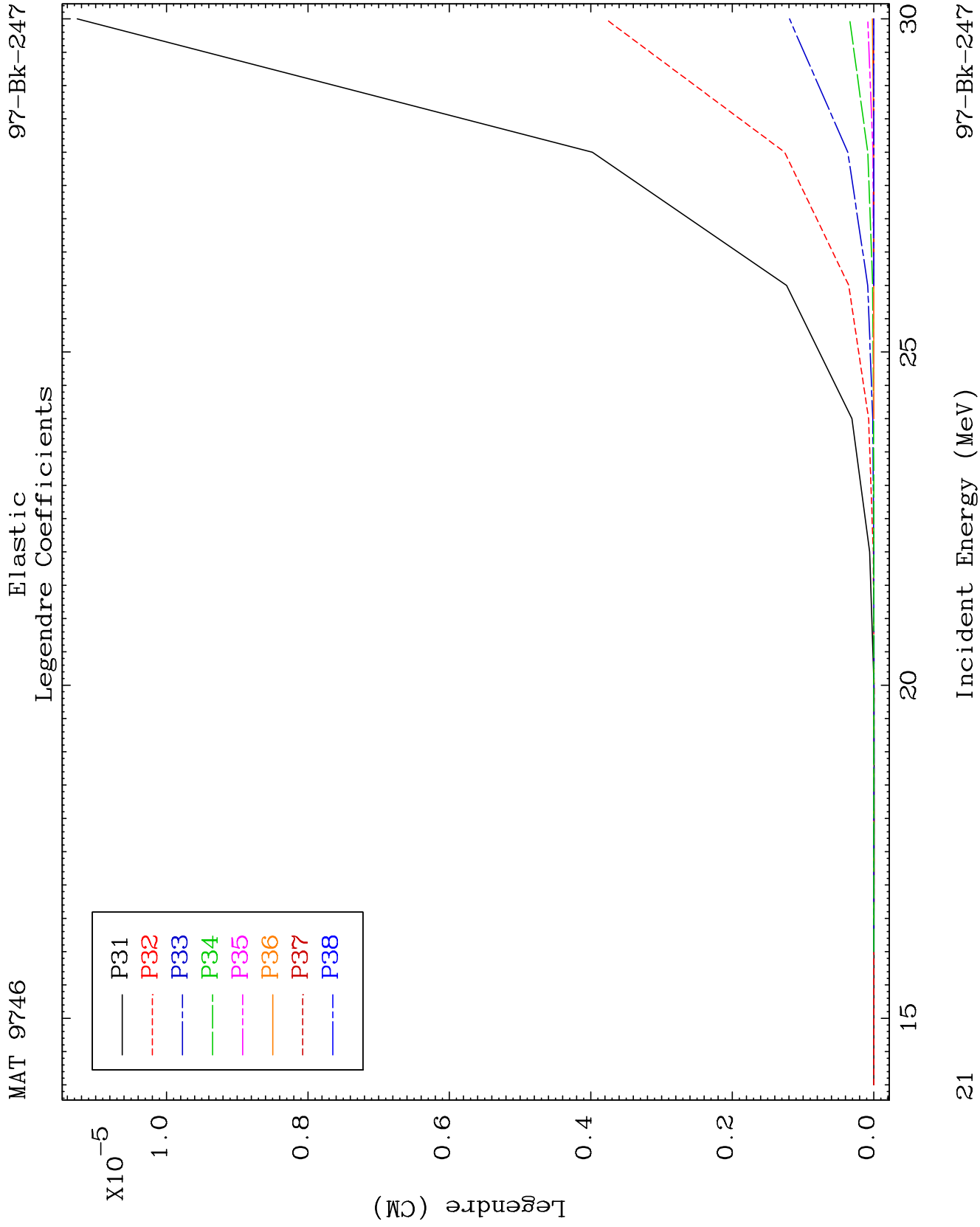
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20

Incident Energy (MeV)

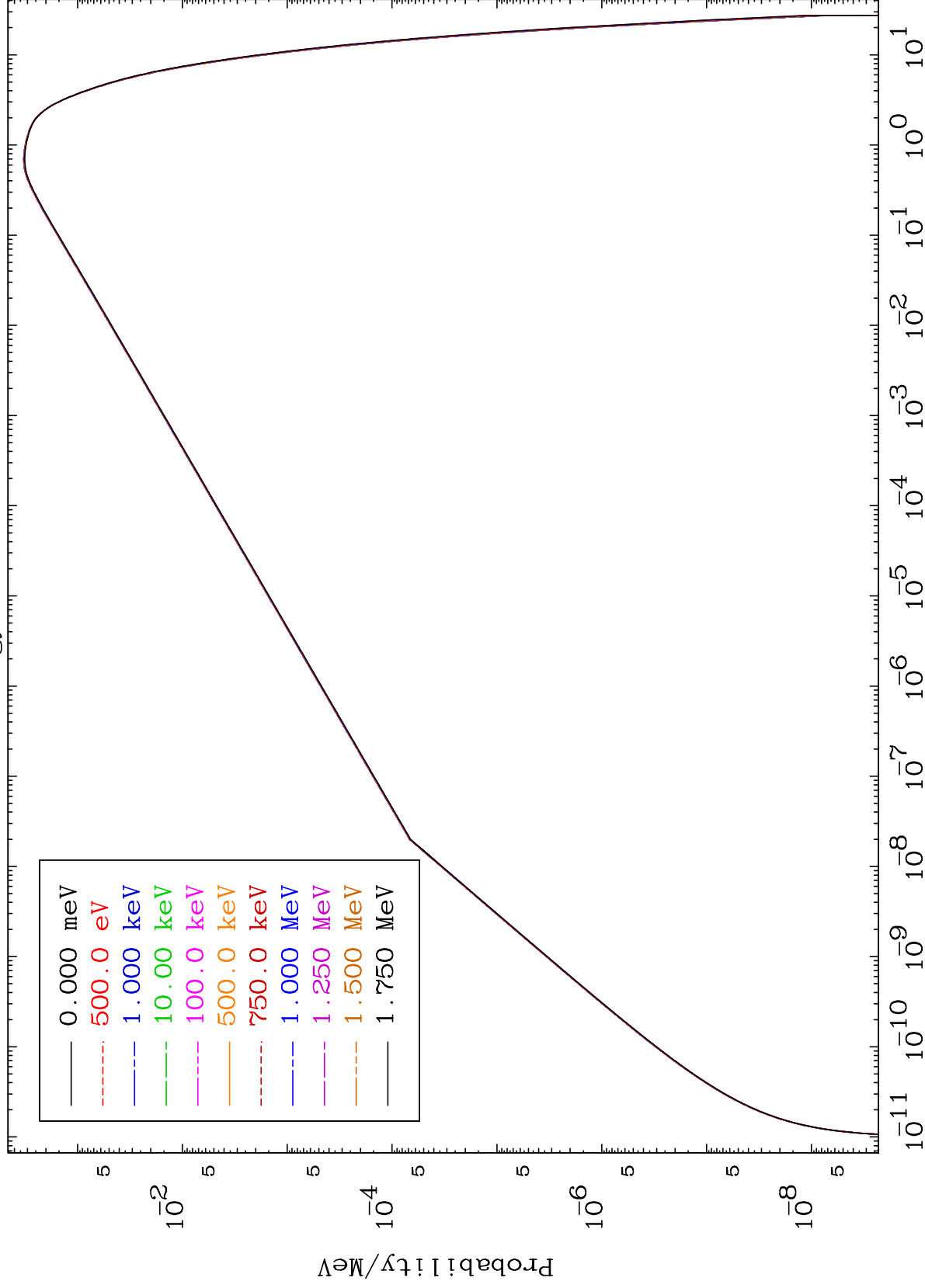
97-Bk-247



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Fission
Energy Distribution

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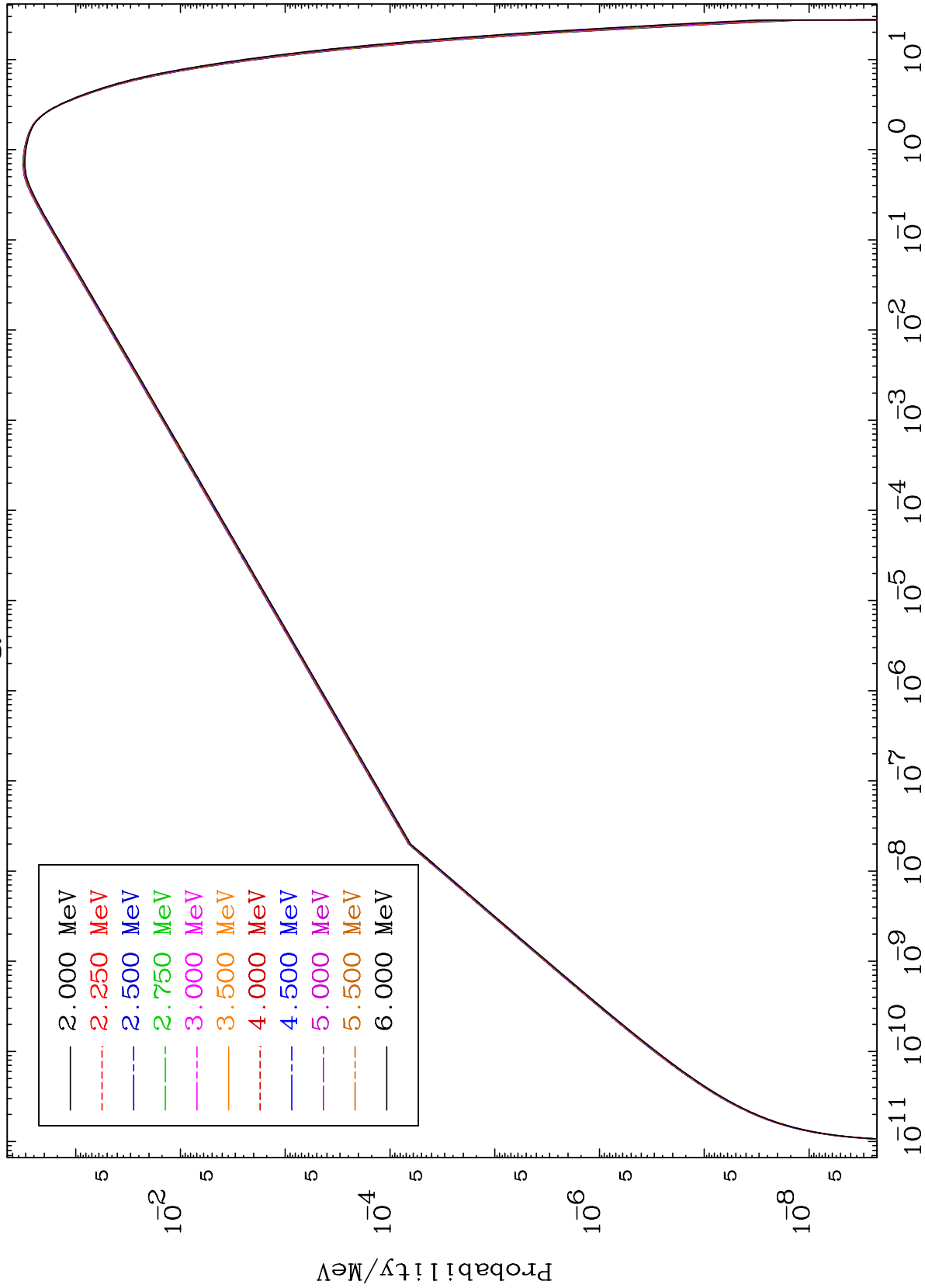
Secondary Energy (MeV)

97-Bk-247

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Fission
Energy Distribution

97-Bk-247



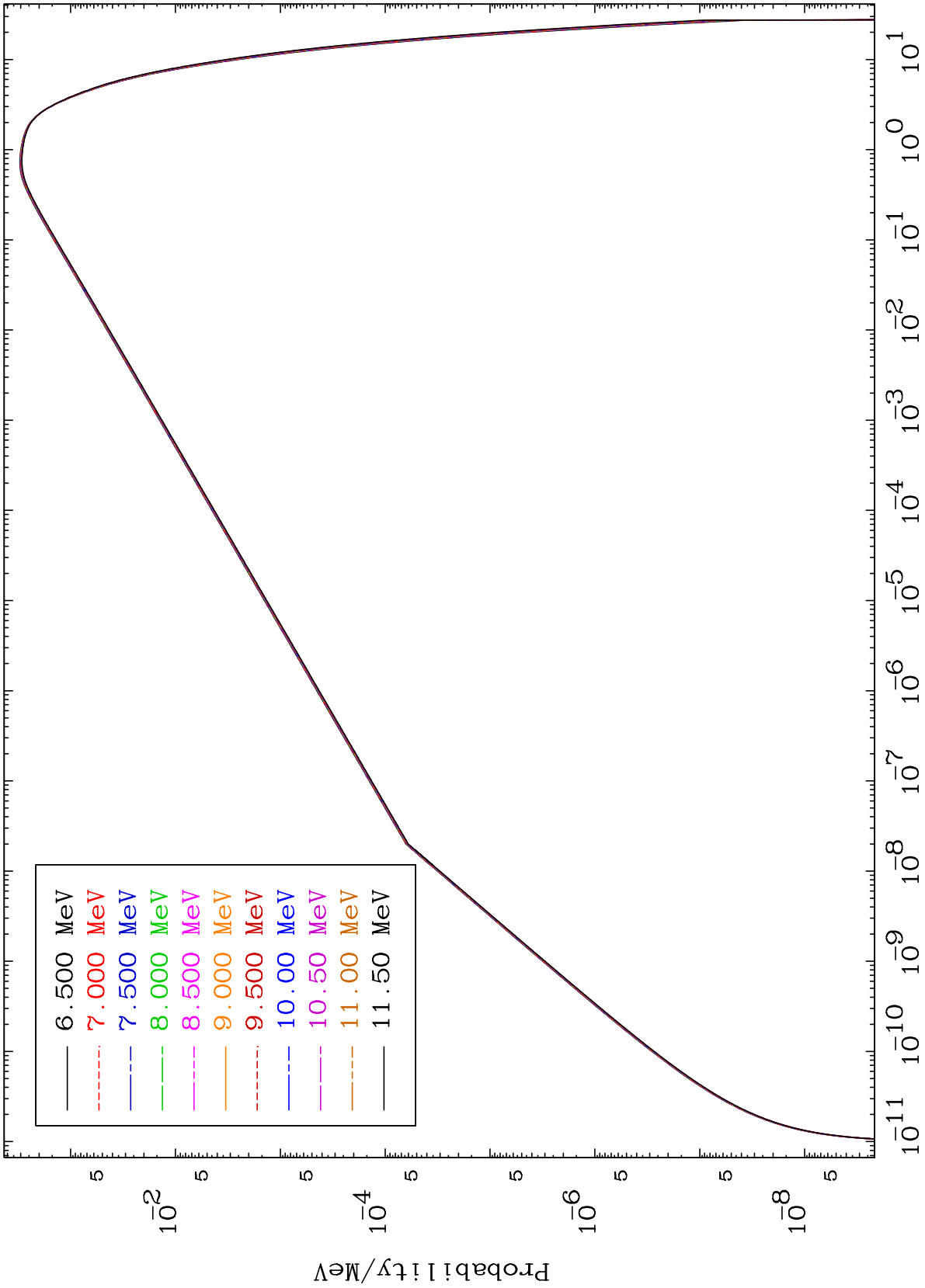
23

97-Bk-247

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Fission
Energy Distribution

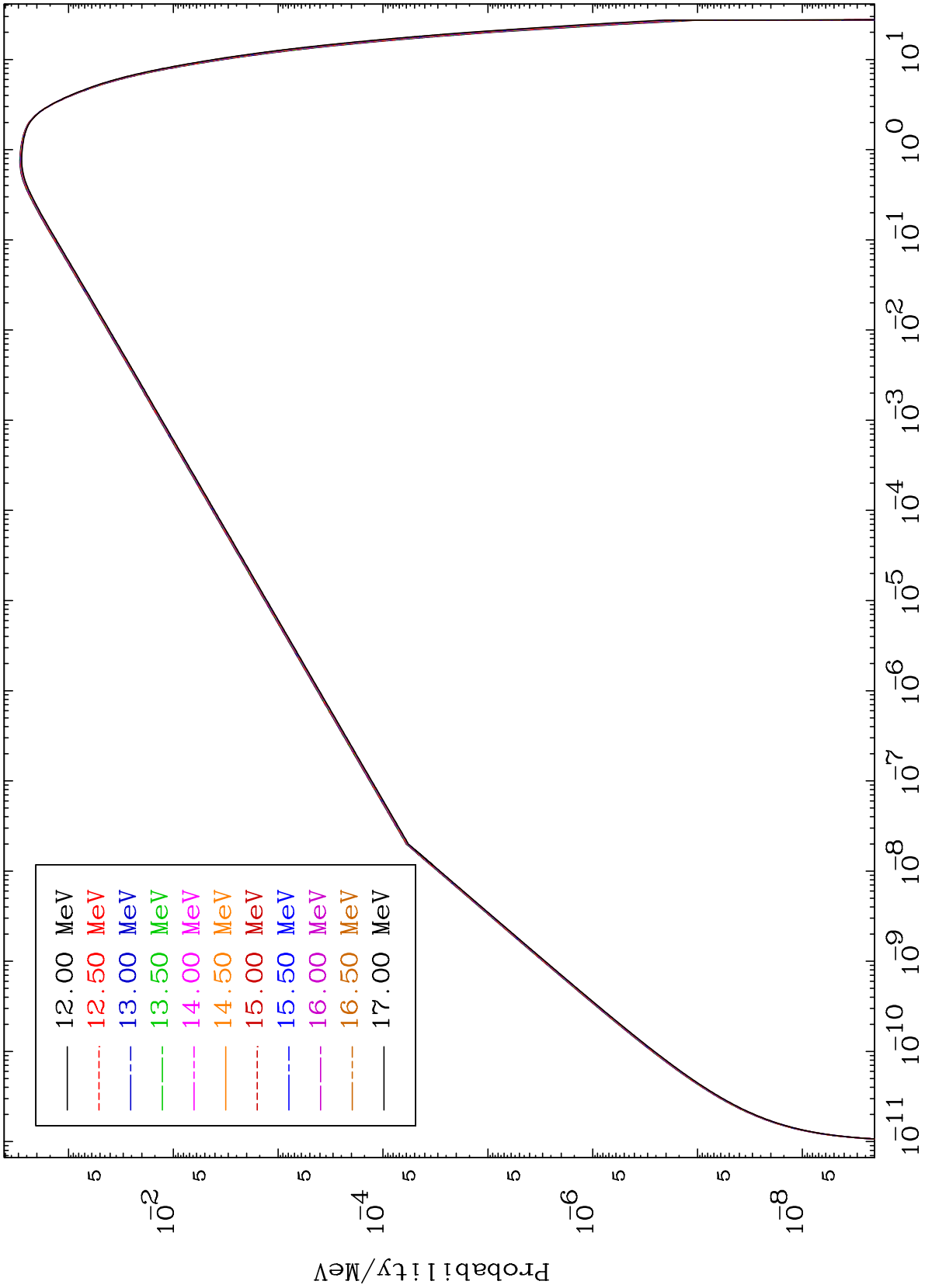
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Fission
Energy Distribution

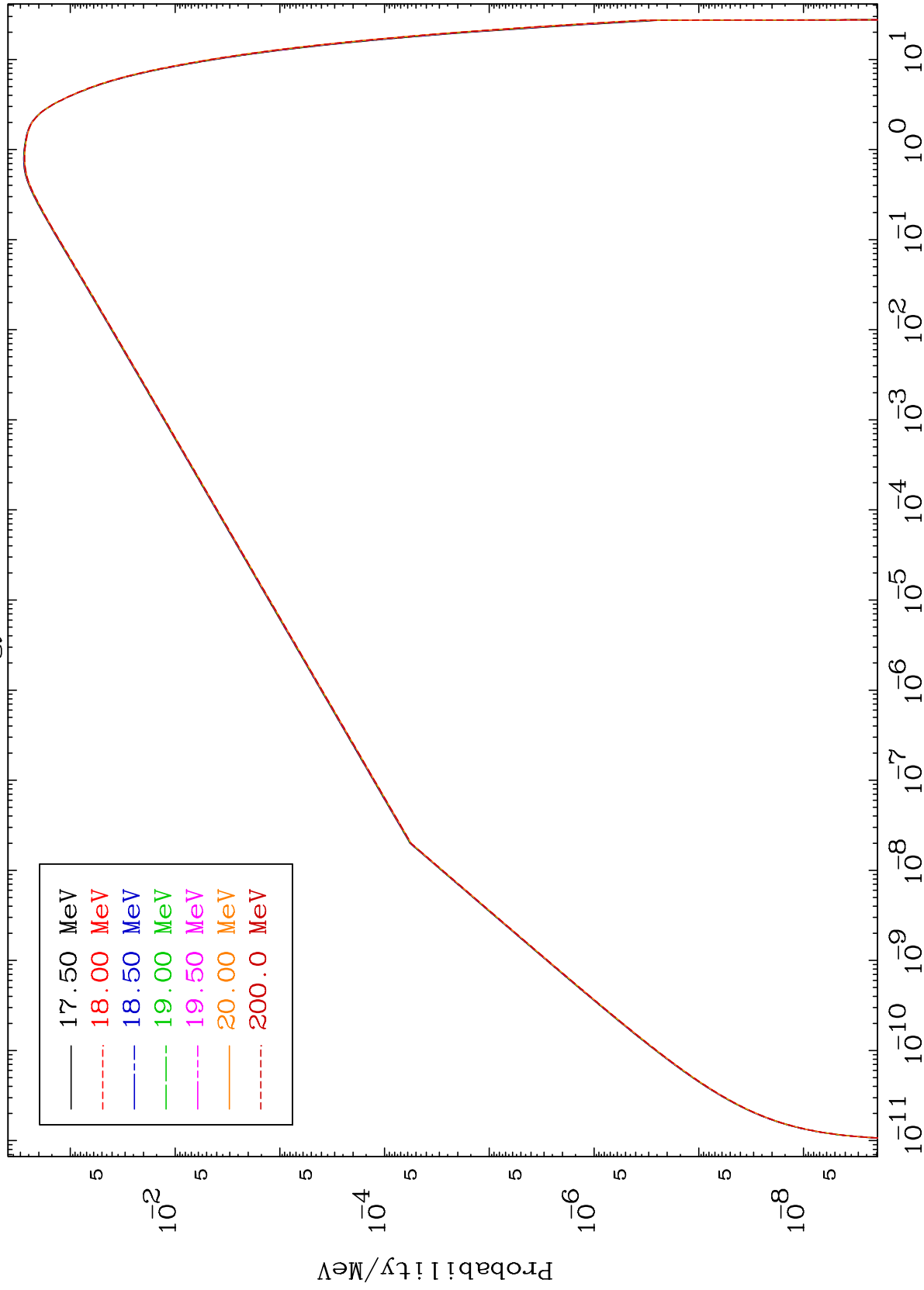
97-Bk-247



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Fission
Energy Distribution

97-Bk-247



26

Secondary Energy (MeV)

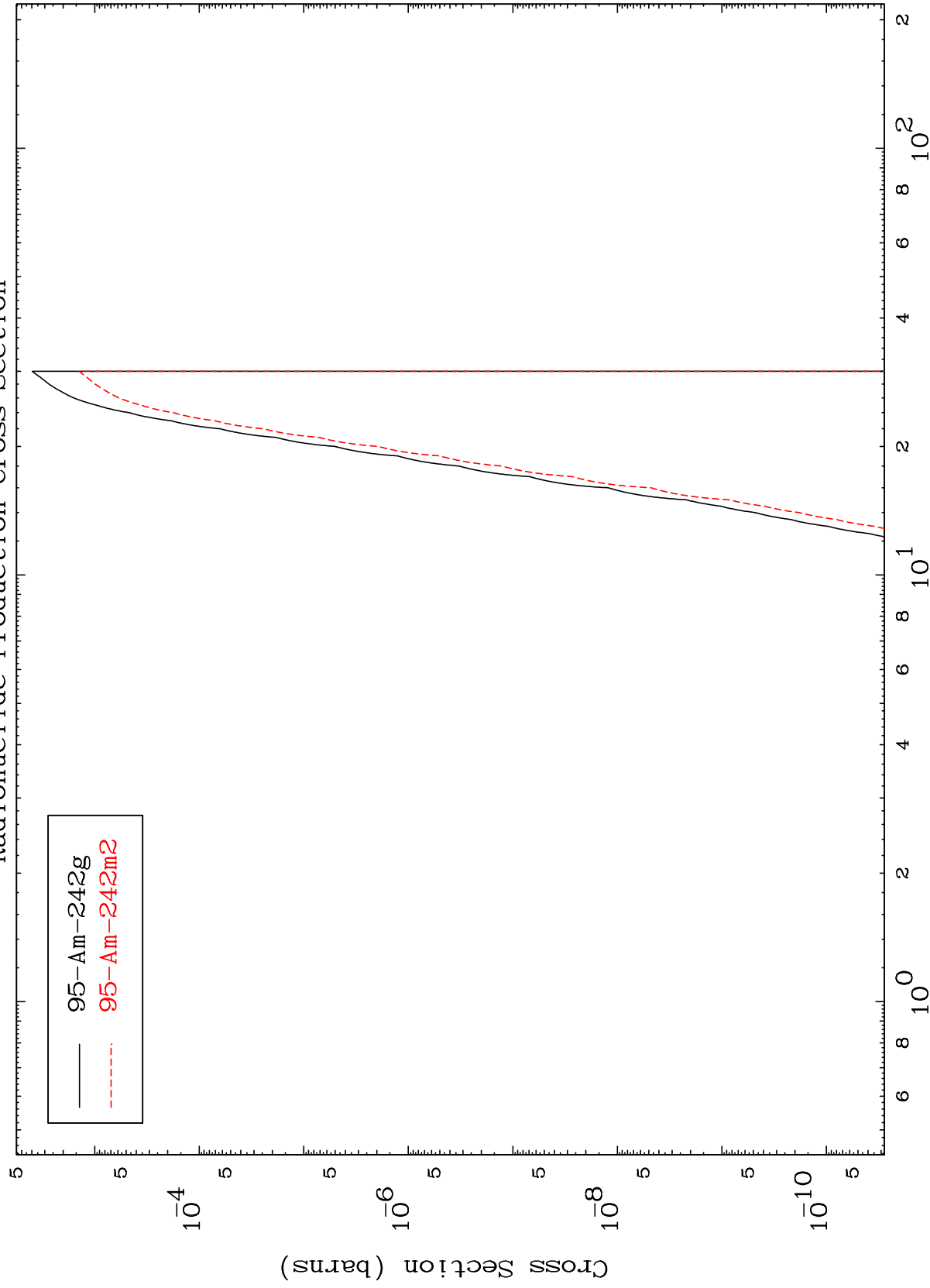
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$(n,2n) \alpha$

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Radionuclide Production Cross Section



27

Incident Energy (MeV)

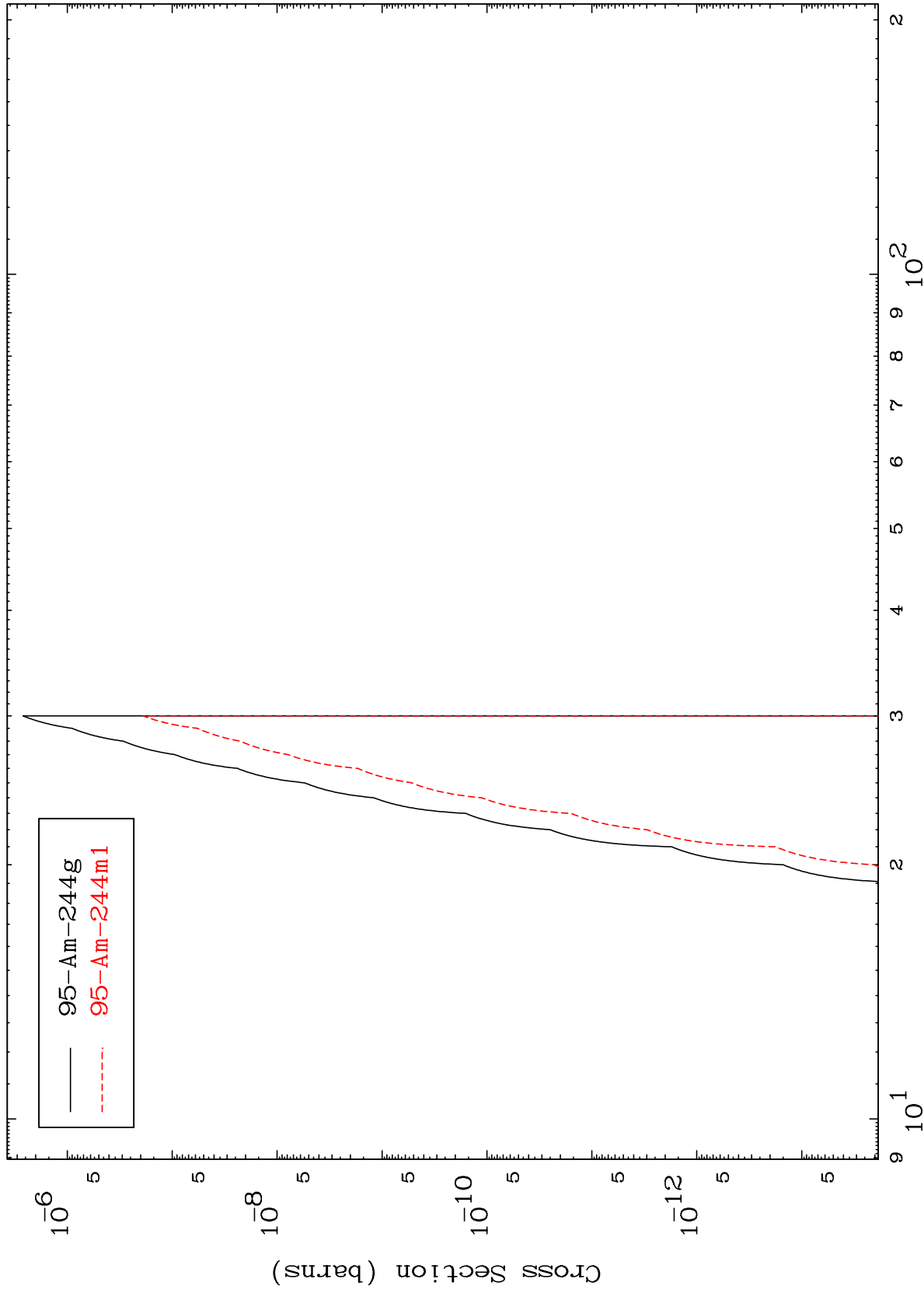
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(n,n') He-3

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Radionuclide Production Cross Section



28

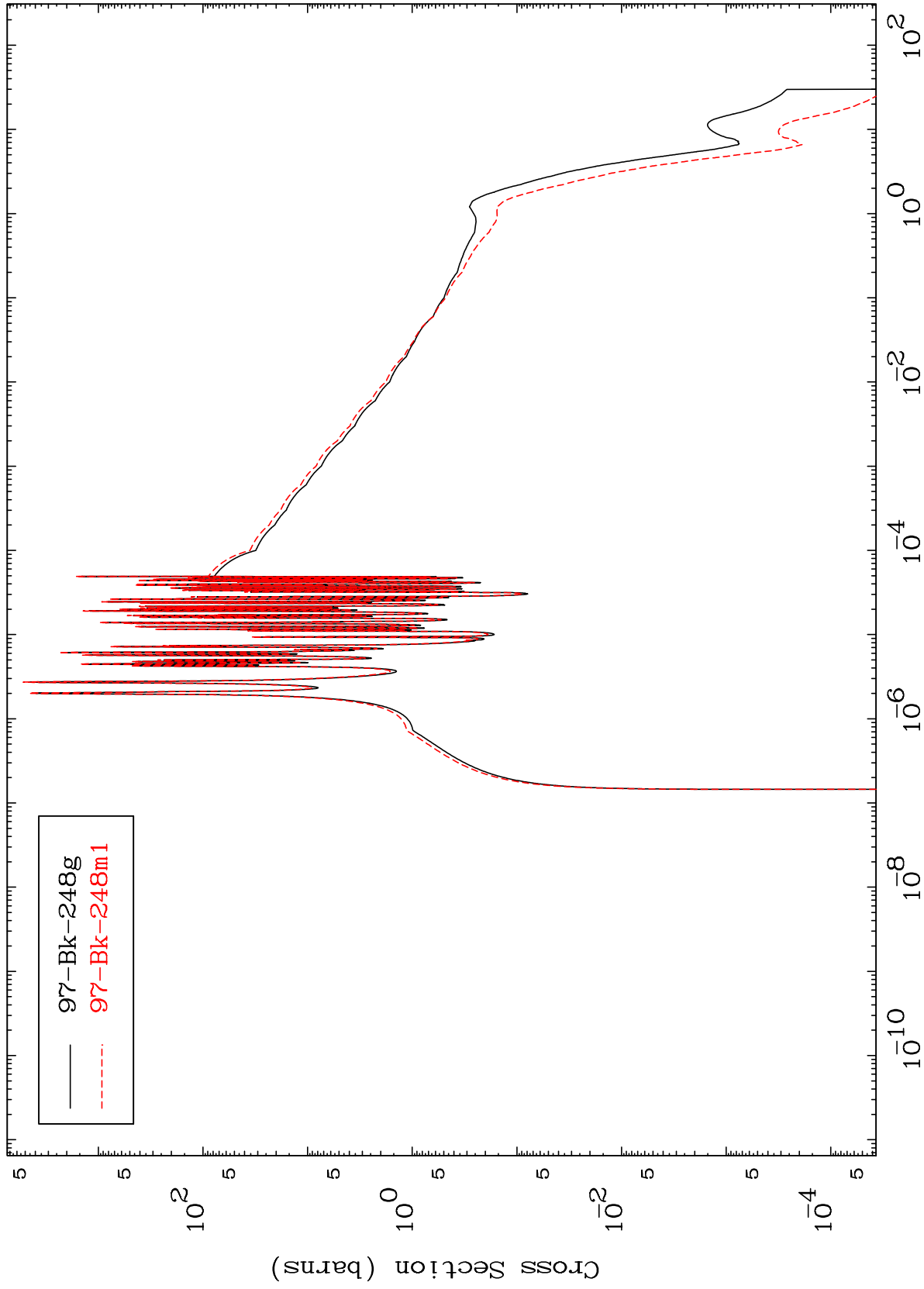
Incident Energy (MeV)

97-Bk-247

MAT 9746

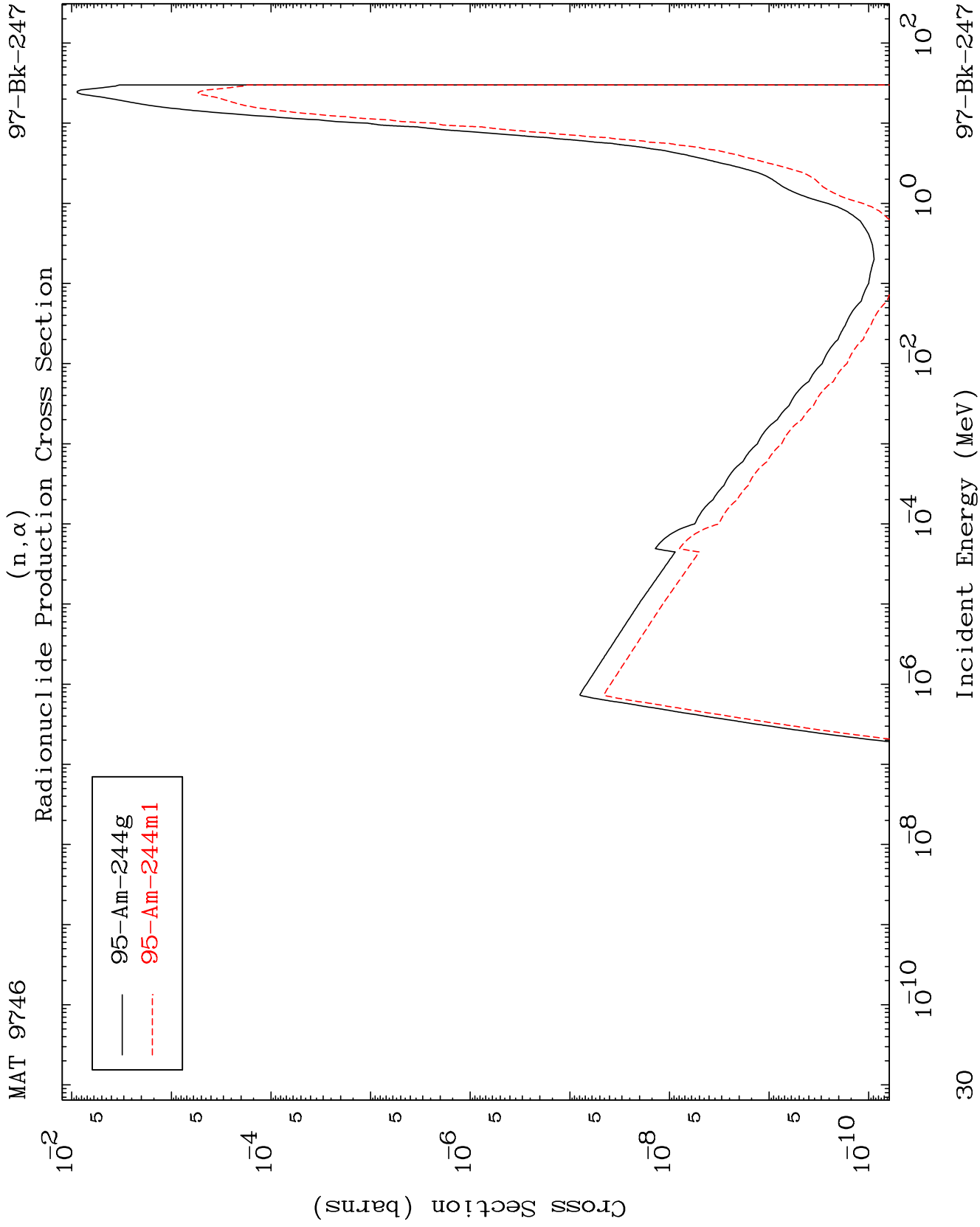
97-Bk-247

Radionuclide Production Cross Section



29

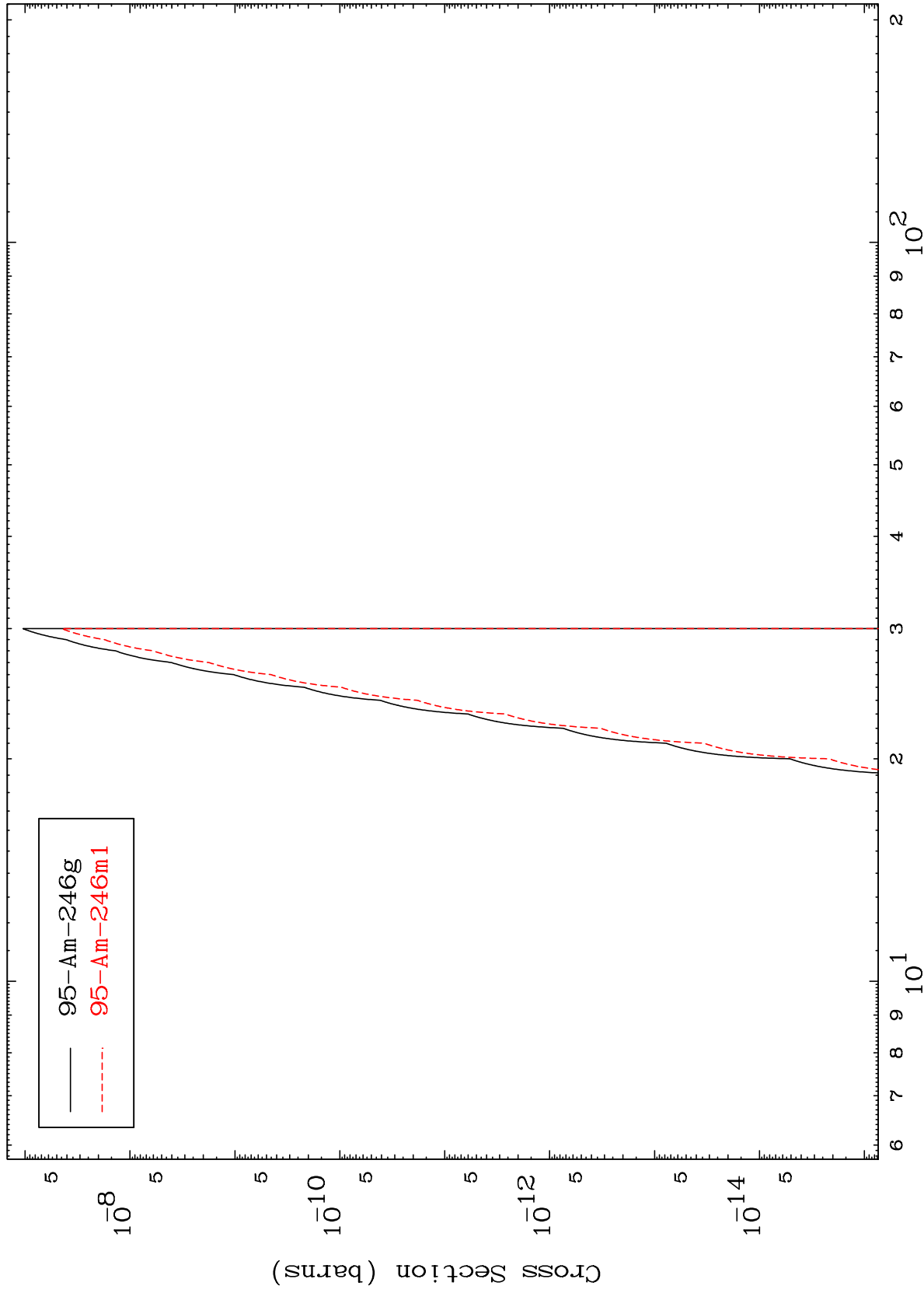
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97-Bk-247

Radionuclide Production Cross Section
(n,2p)



31

Incident Energy (MeV)

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