

Program EVALPLOT
(Version 2018-1)

by

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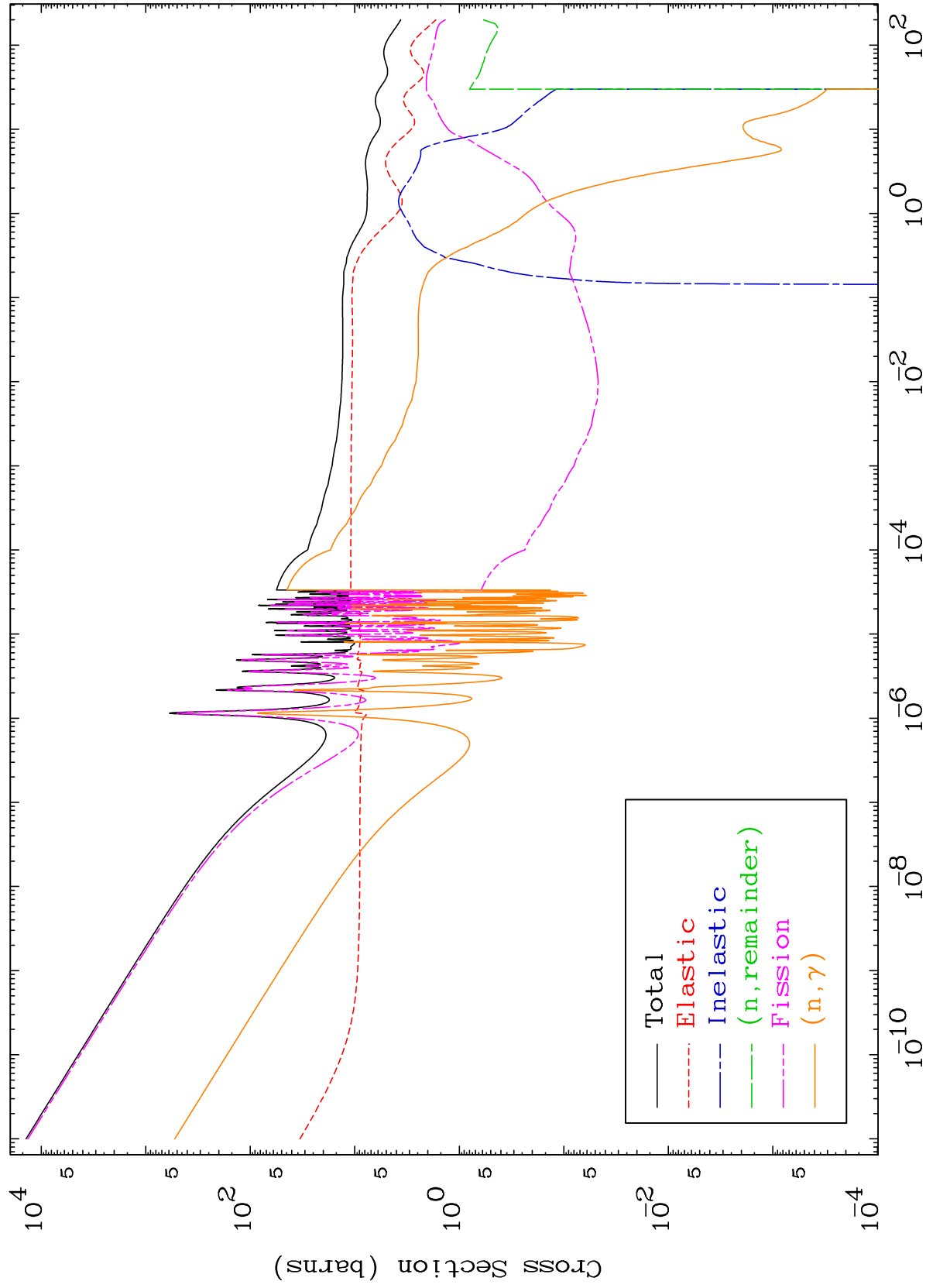
Press Mouse Button to Start

MAT 9749

Major

293 Kelvin Cross Sections

97-Bk-248



1

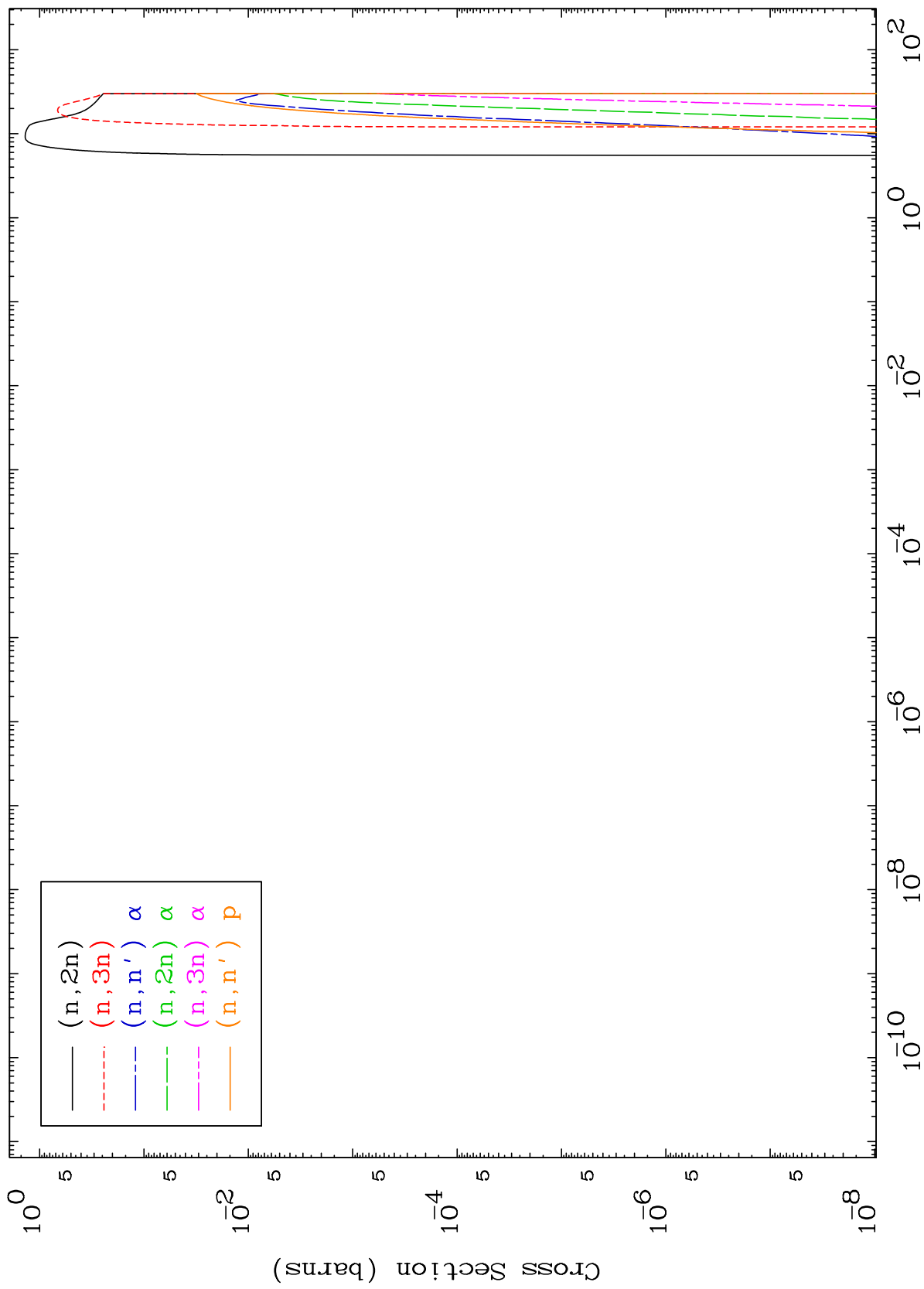
Incident Energy (MeV)

97-Bk-248

MAT 9749

Neutron Production
293 Kelvin Cross Sections

97-Bk-248



2

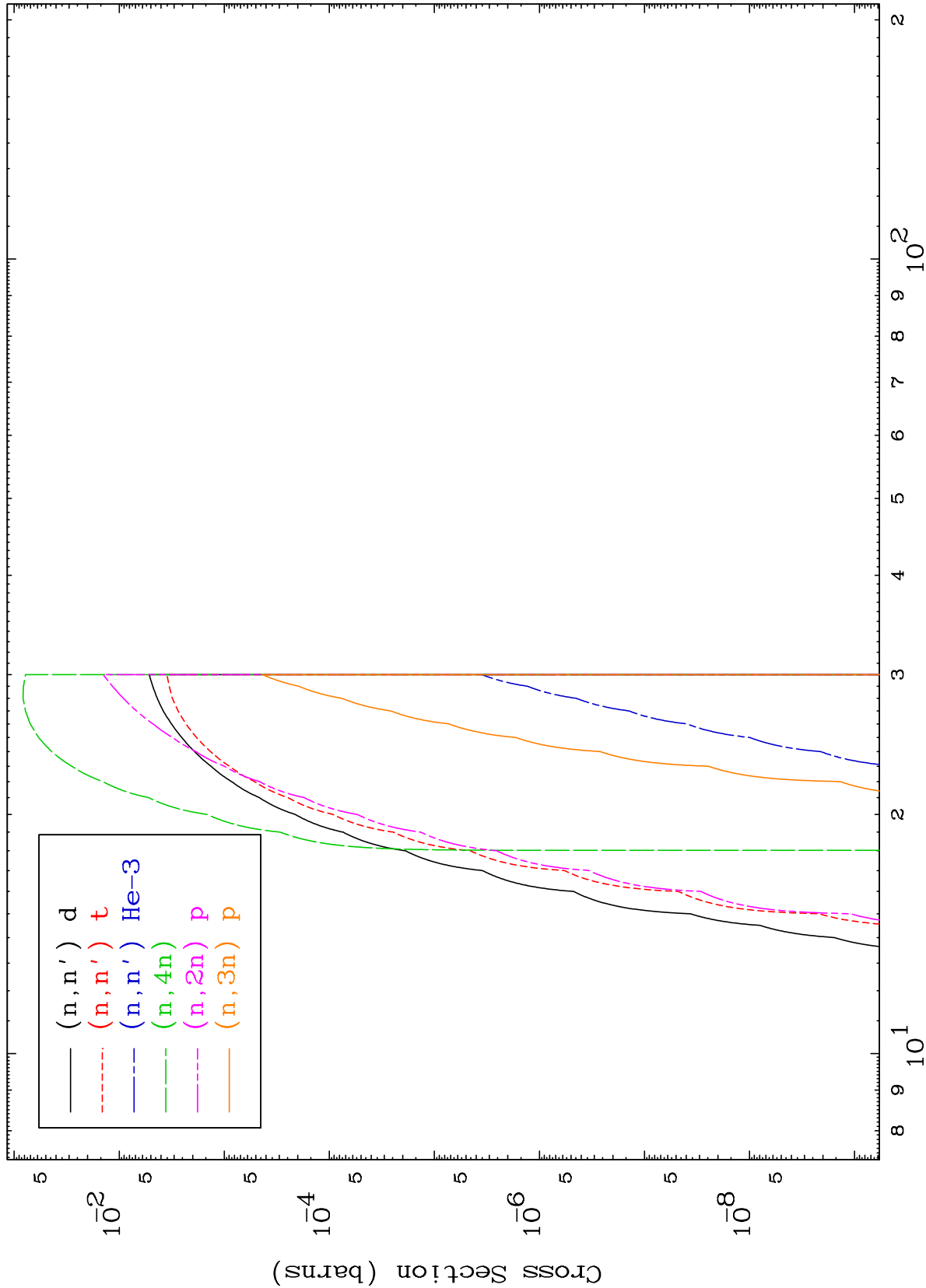
Incident Energy (MeV)

97-Bk-248

MAT 9749

Neutron Production
293 Kelvin Cross Sections

97-Bk-248



3

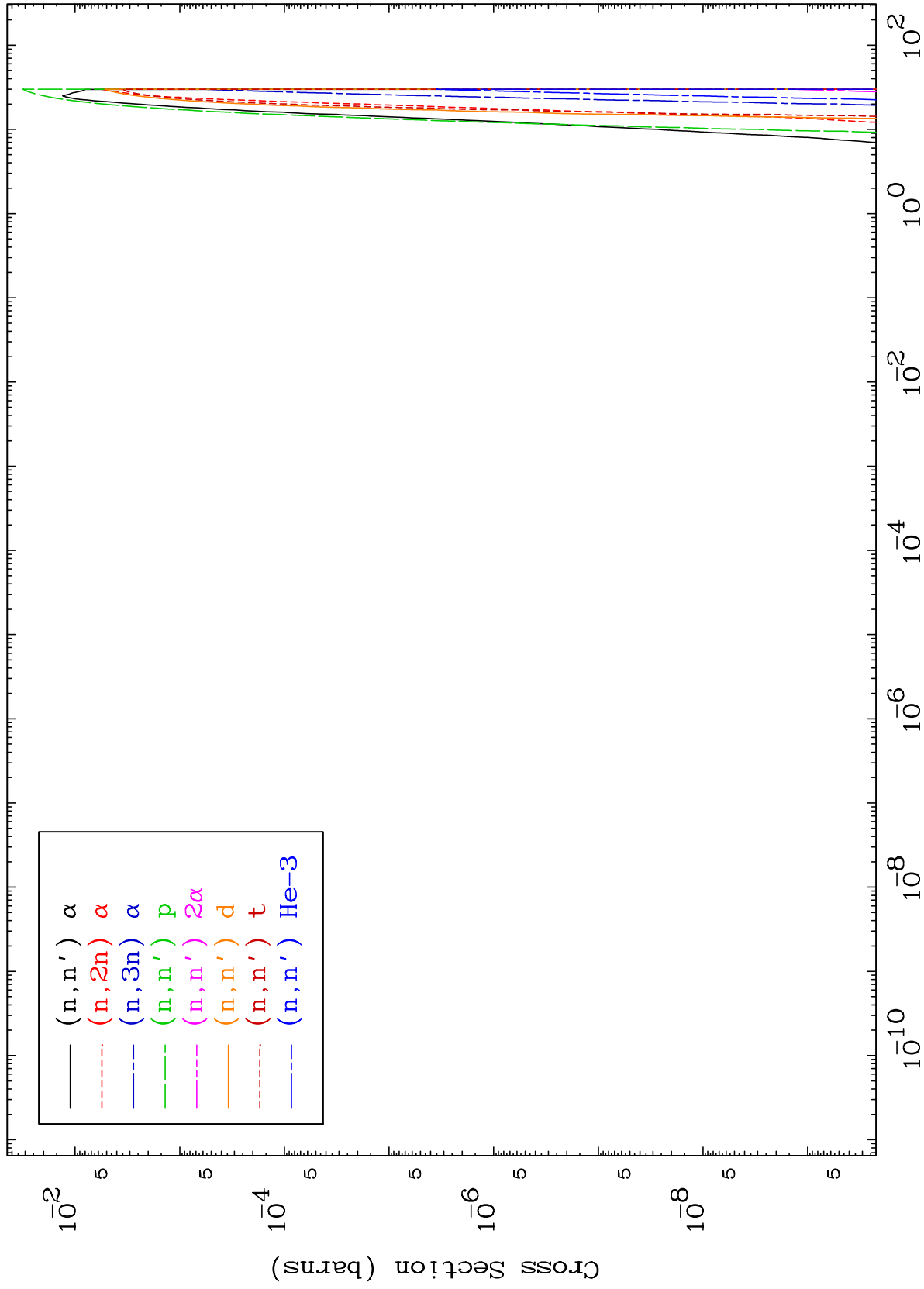
Incident Energy (MeV)

97-Bk-248

MAT 9749

Charged Particle
293 Kelvin Cross Sections

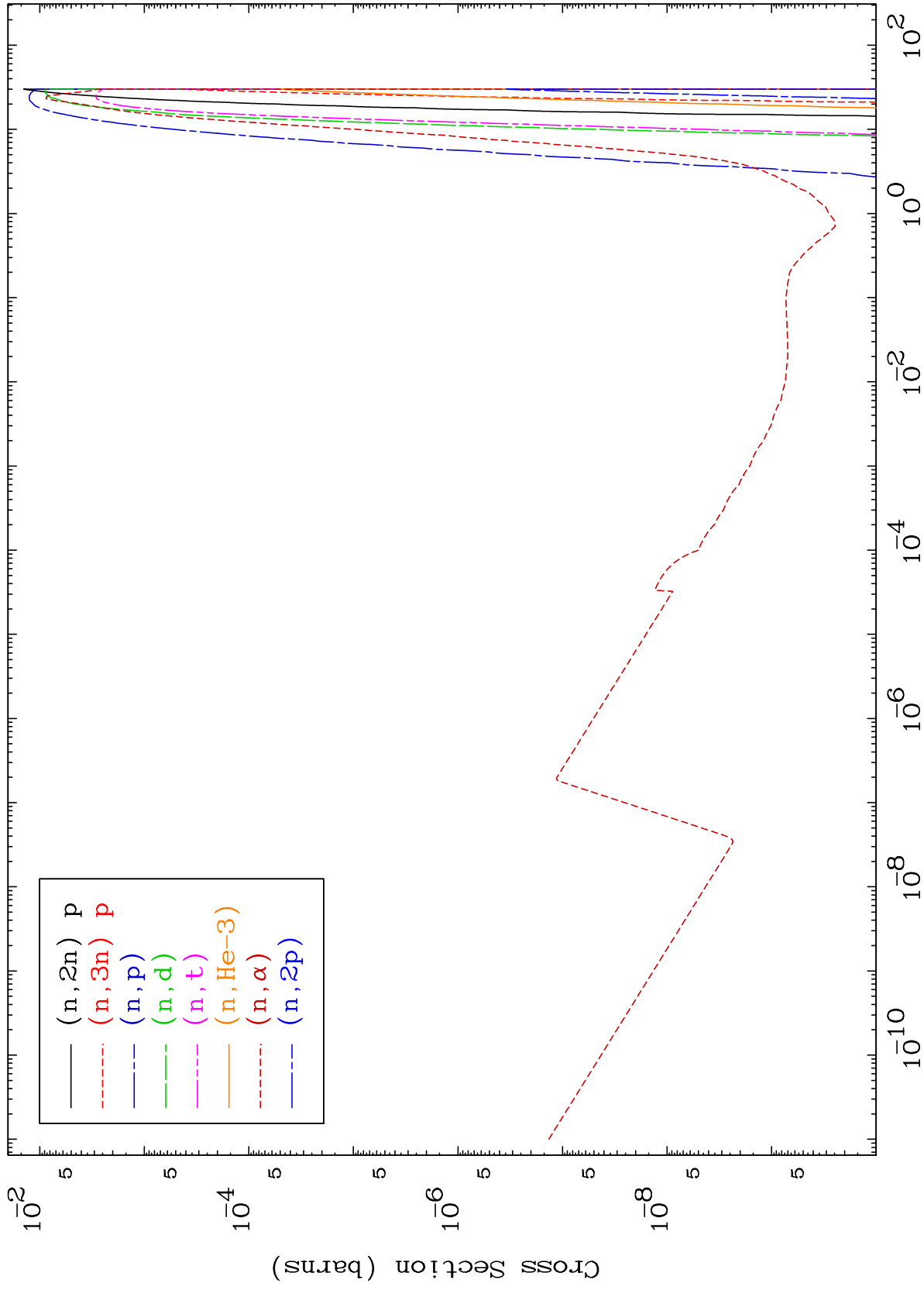
97-Bk-248



MAT 9749

Charged Particle
293 Kelvin Cross Sections

97-Bk-248



5

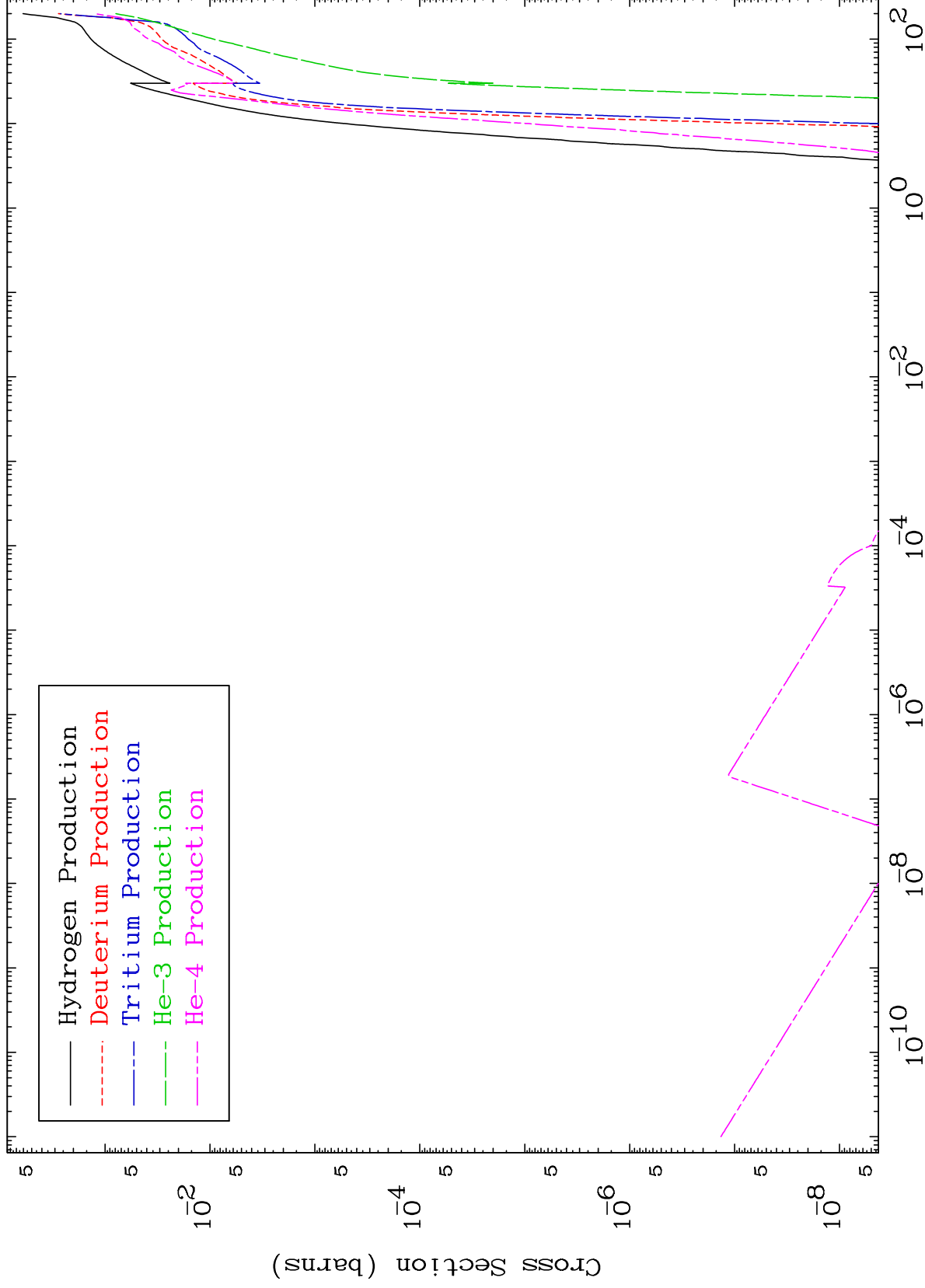
Incident Energy (MeV)

97-Bk-248

MAT 9749

Particle Production
293 Kelvin Cross Sections

97-Bk-248



6

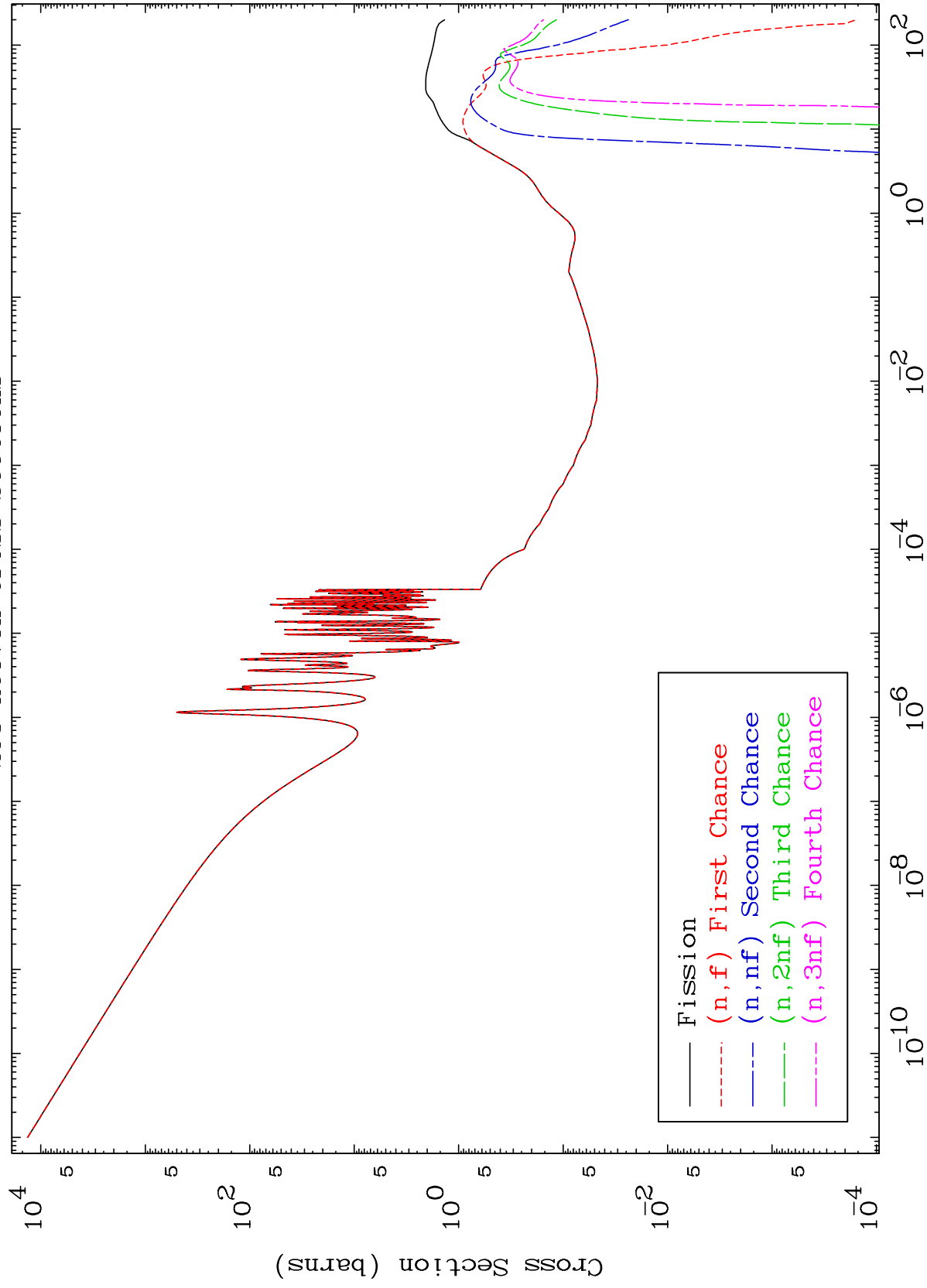
Incident Energy (MeV)

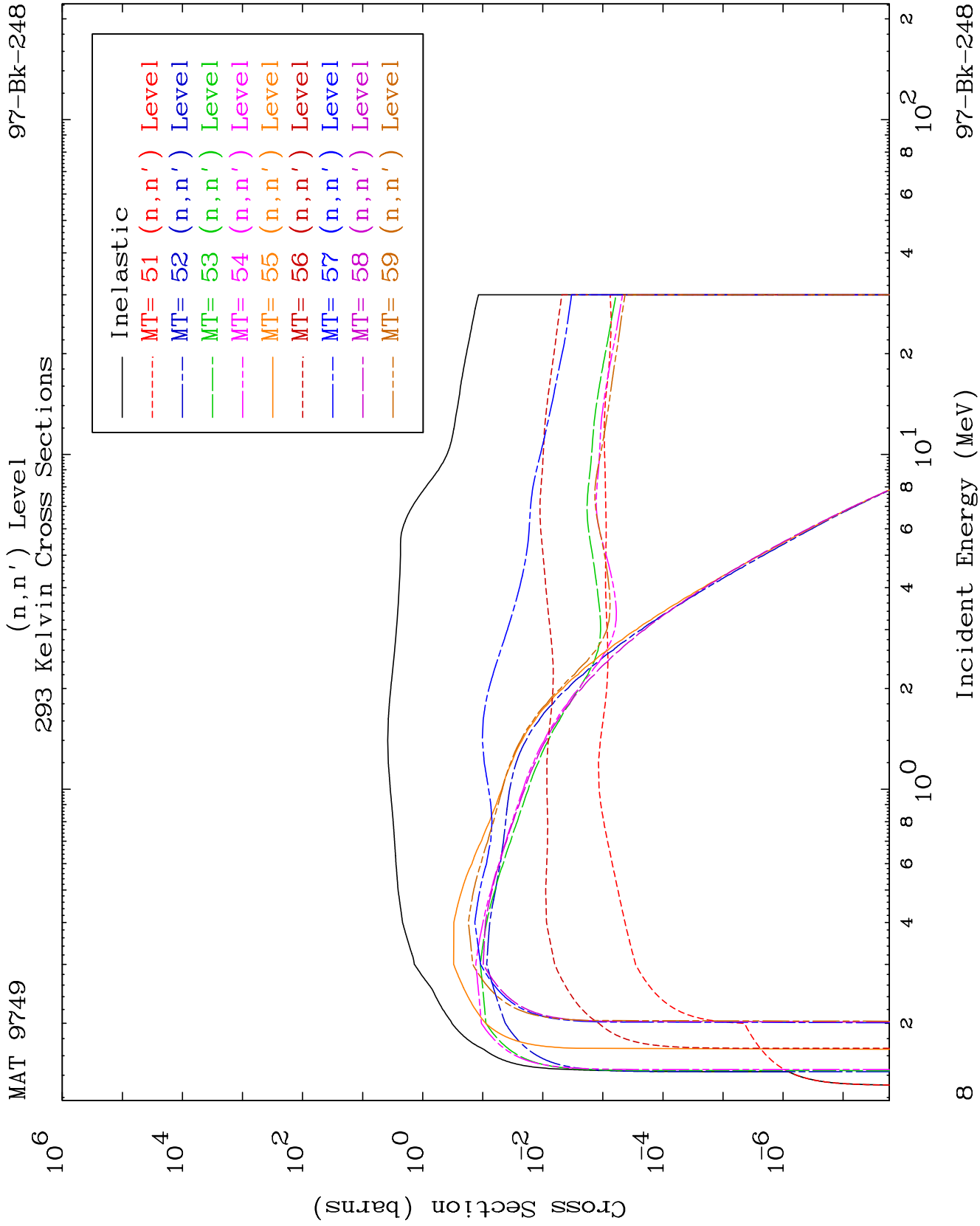
97-Bk-248

MAT 9749

Fission
293 Kelvin Cross Sections

97-Bk-248



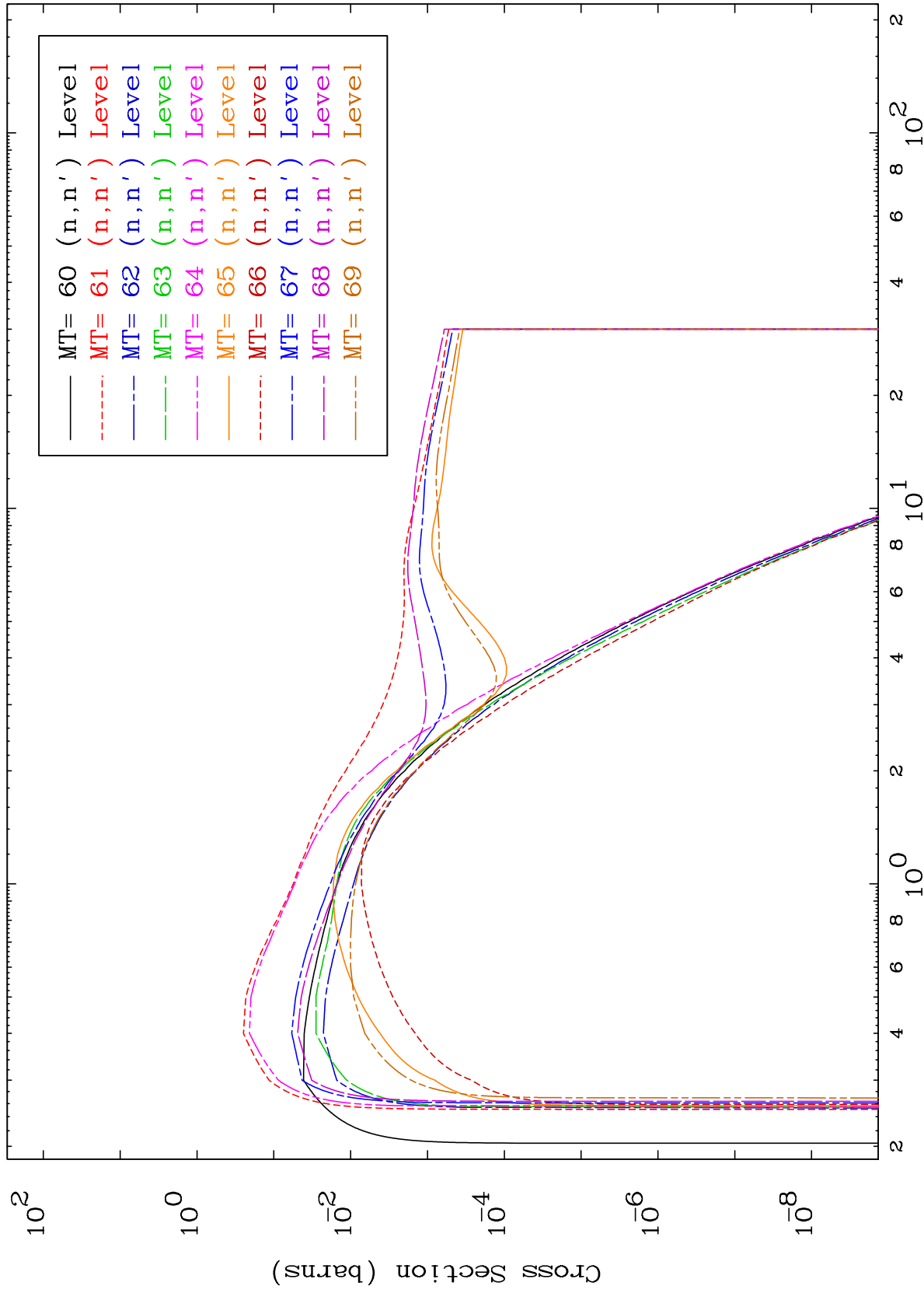


MAT 9749

(n,n') Level

97-Bk-248

293 Kelvin Cross Sections

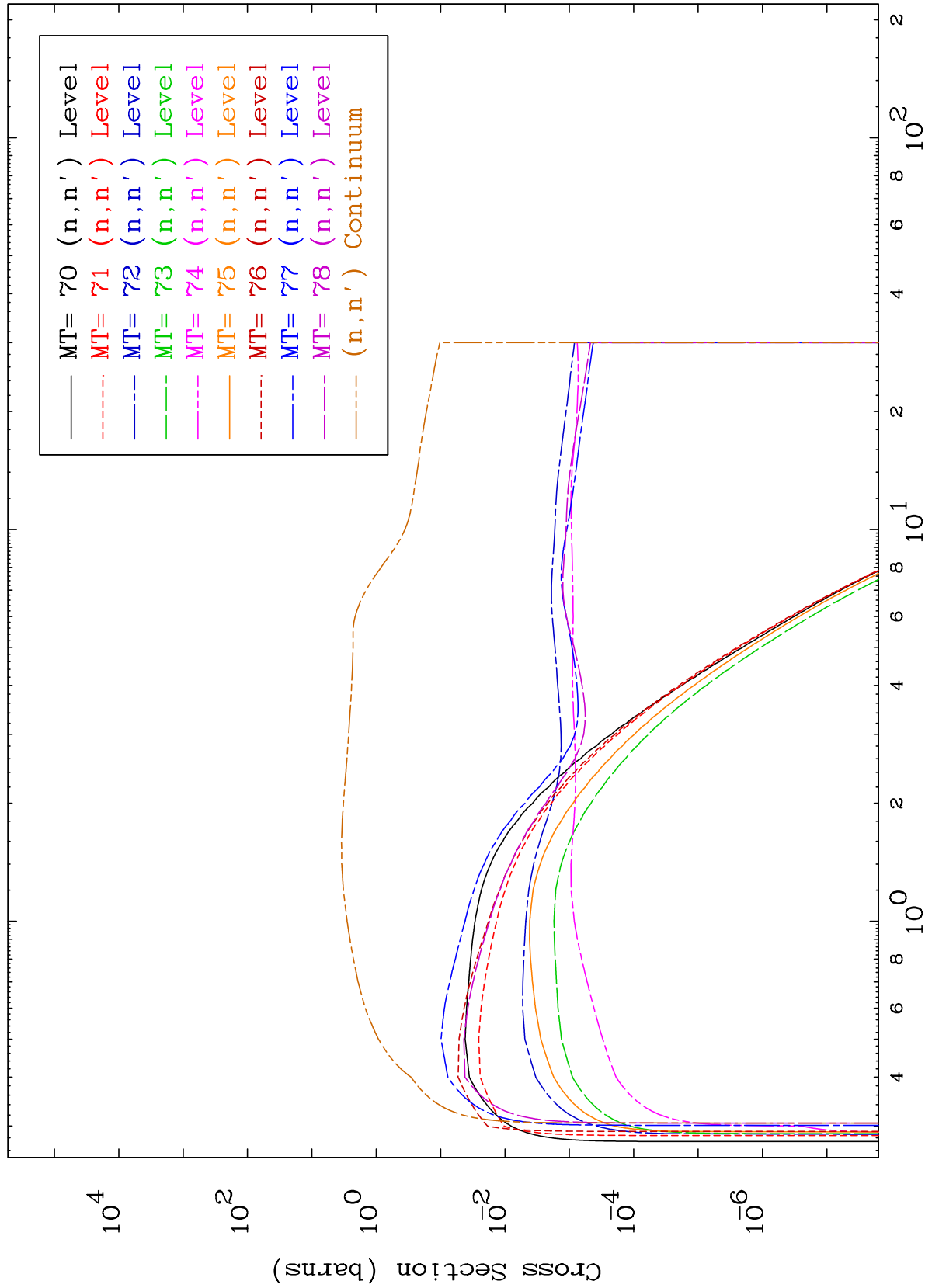


MAT 9749

(n,n') Level

97-Bk-248

293 Kelvin Cross Sections



10

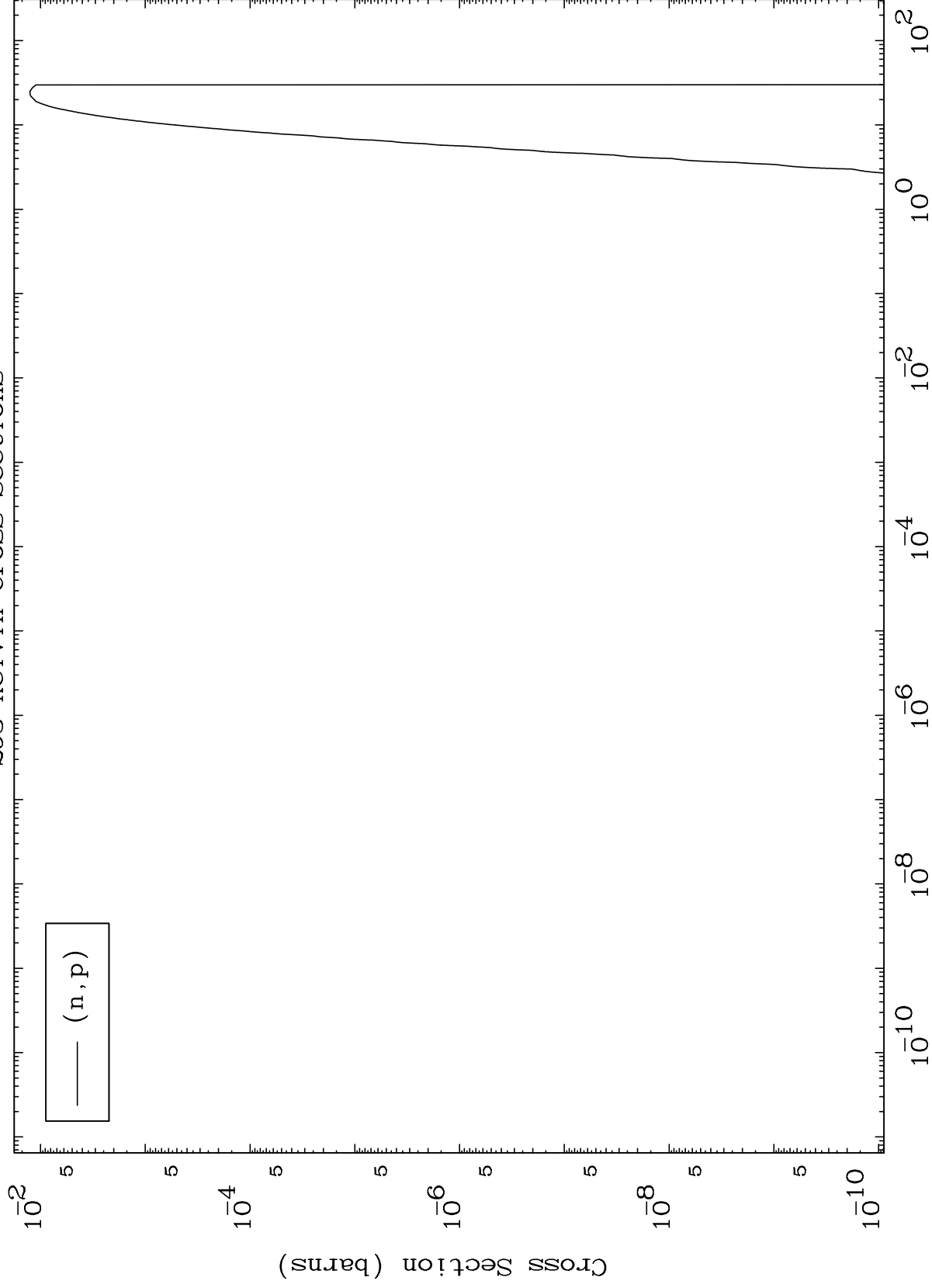
Incident Energy (MeV)

97-Bk-248

MAT 9749

(n,p) Levels
293 Kelvin Cross Sections

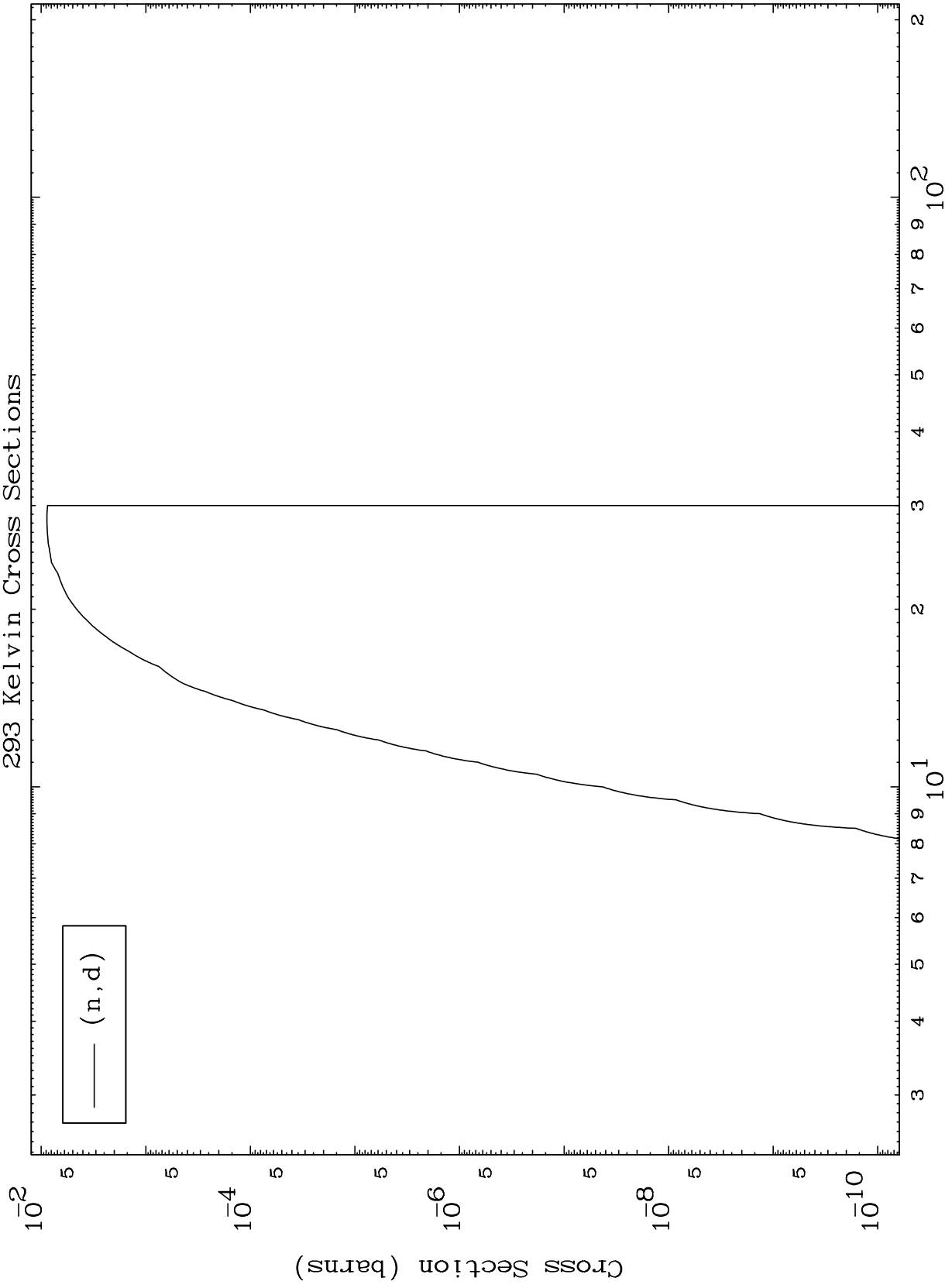
97-Bk-248



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97-Bk-248

(n,d) Levels
293 Kelvin Cross Sections



12

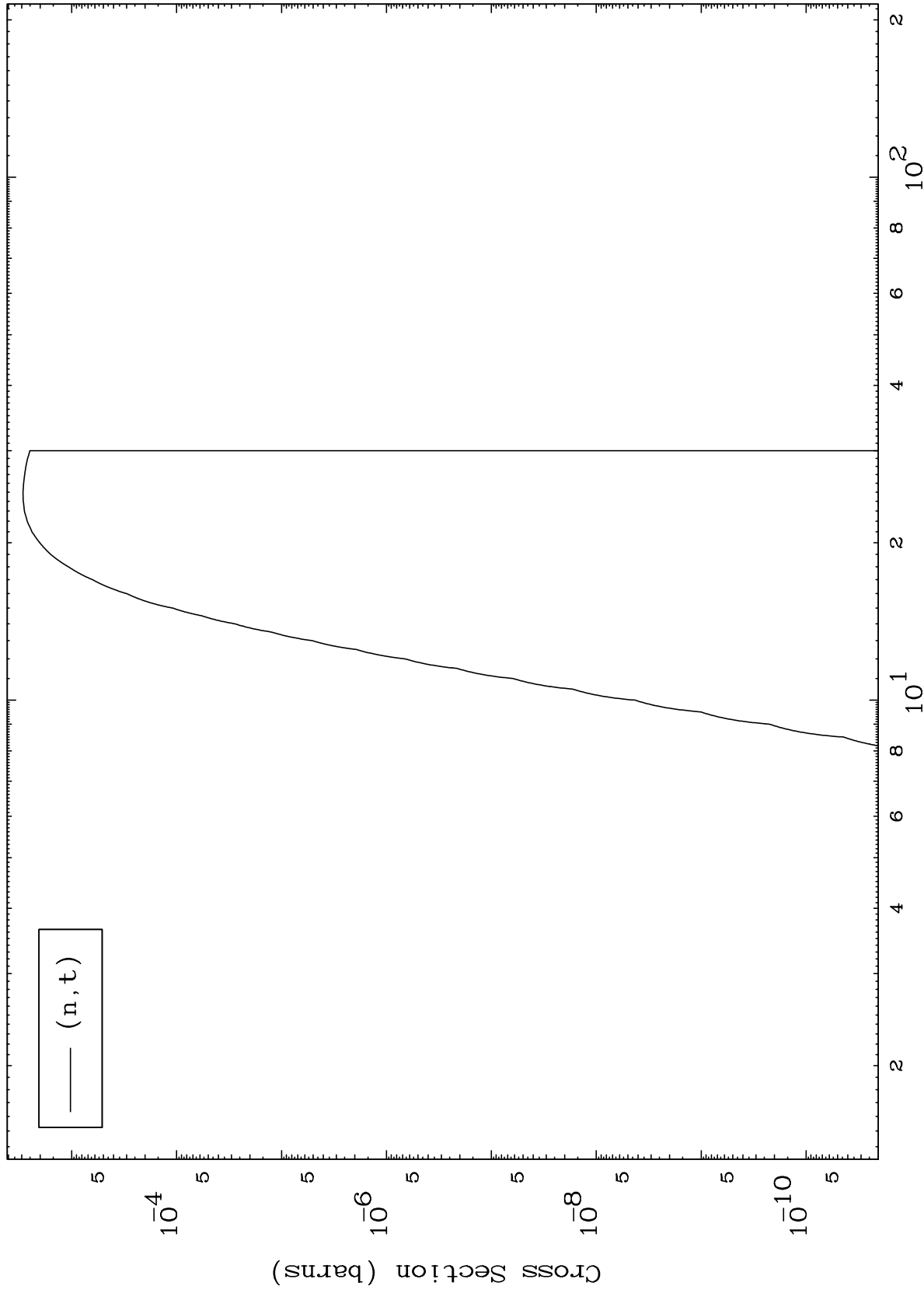
Incident Energy (MeV)

97-Bk-248

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97-Bk-248

(n,t) Levels
293 Kelvin Cross Sections



13

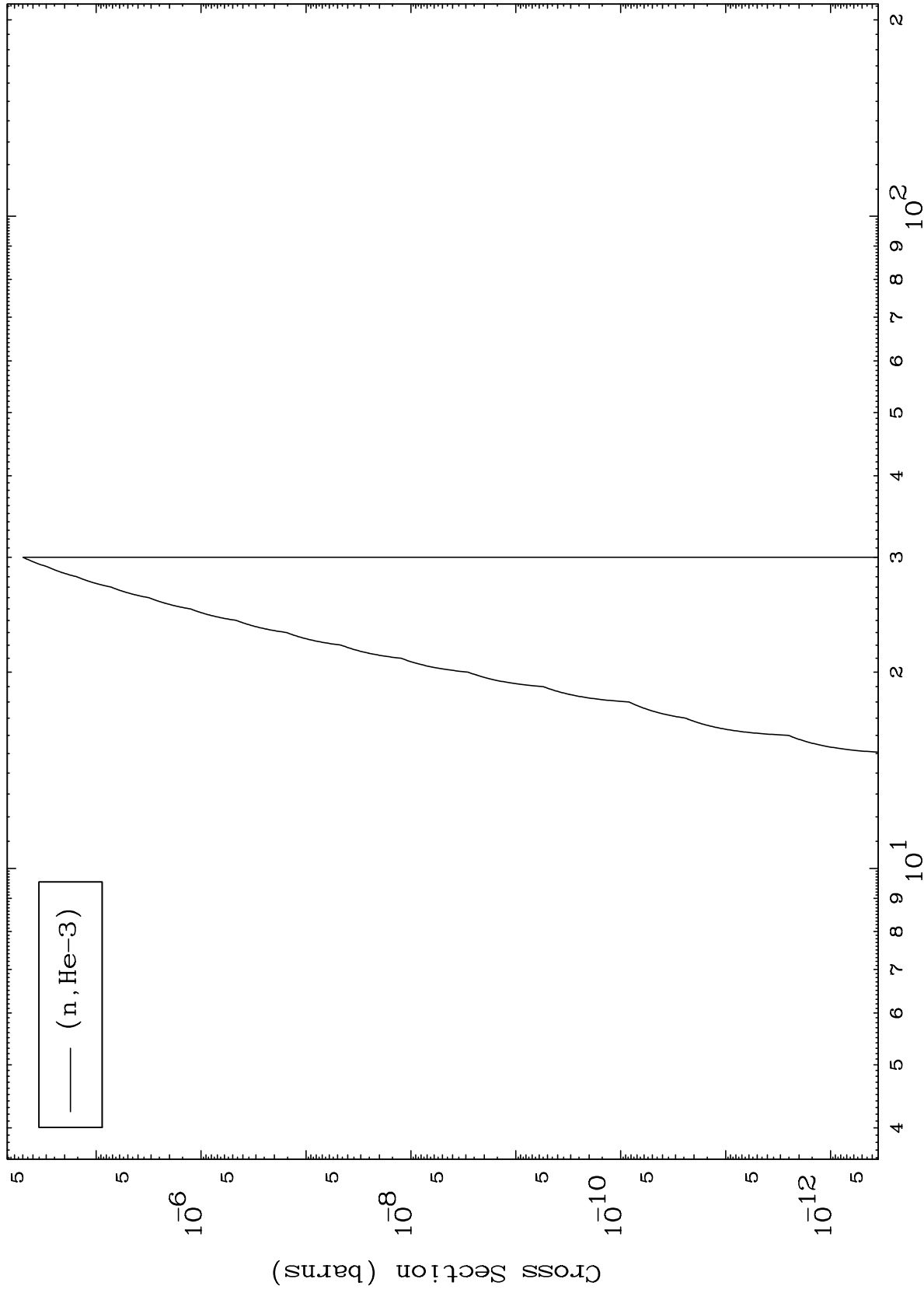
Incident Energy (MeV)

97-Bk-248

MAT 9749

97-Bk-248

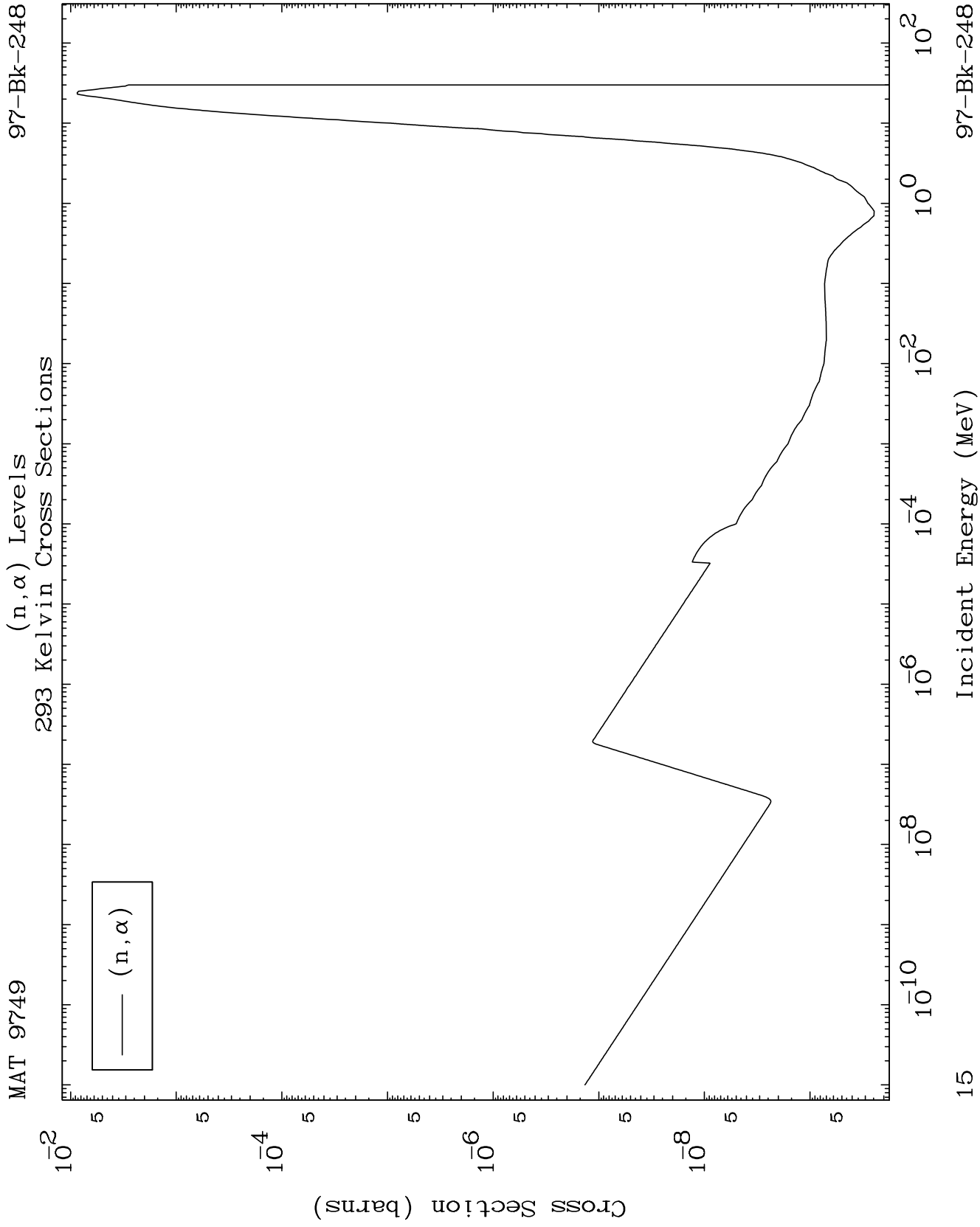
(n,He3) Levels
293 Kelvin Cross Sections

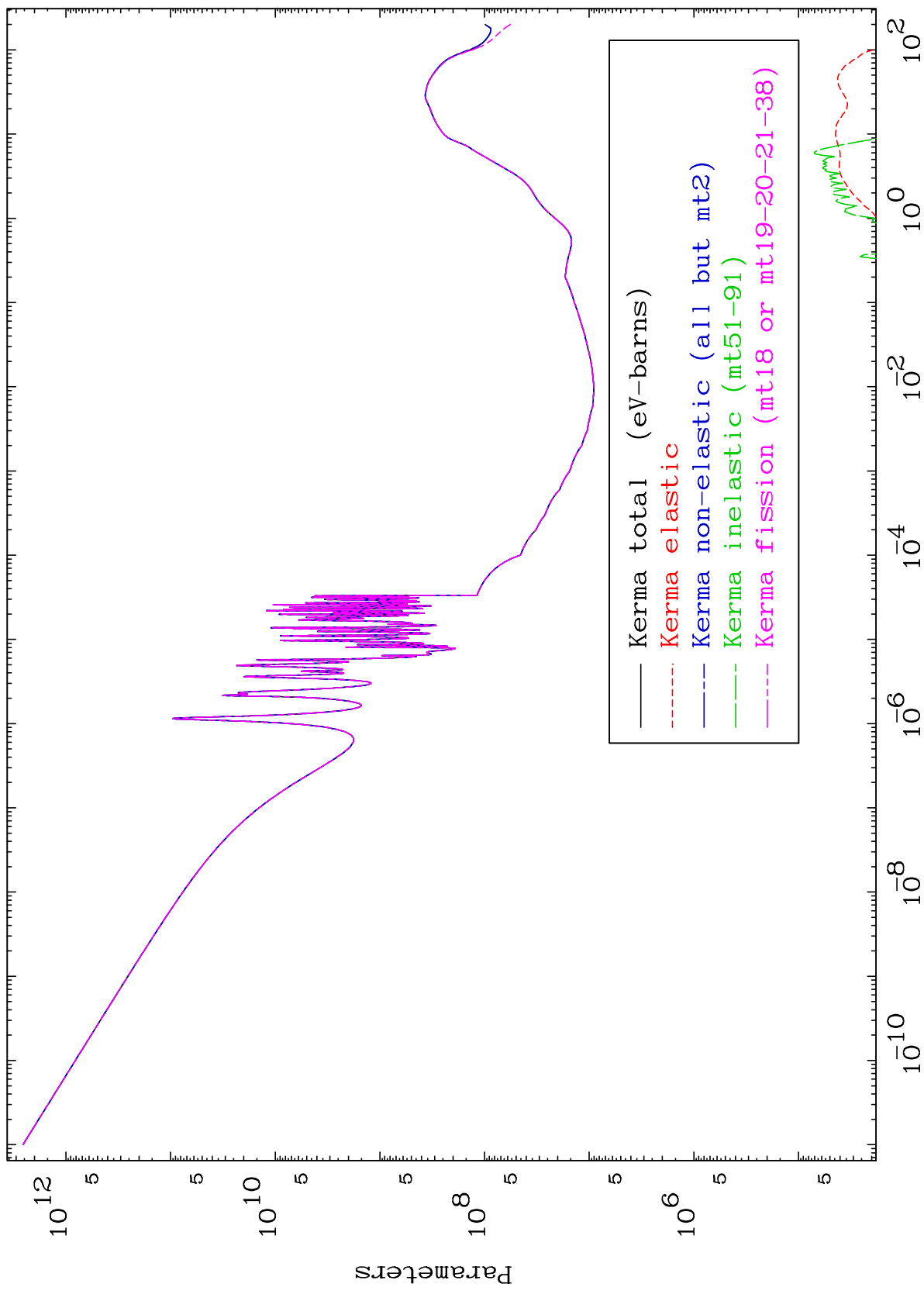


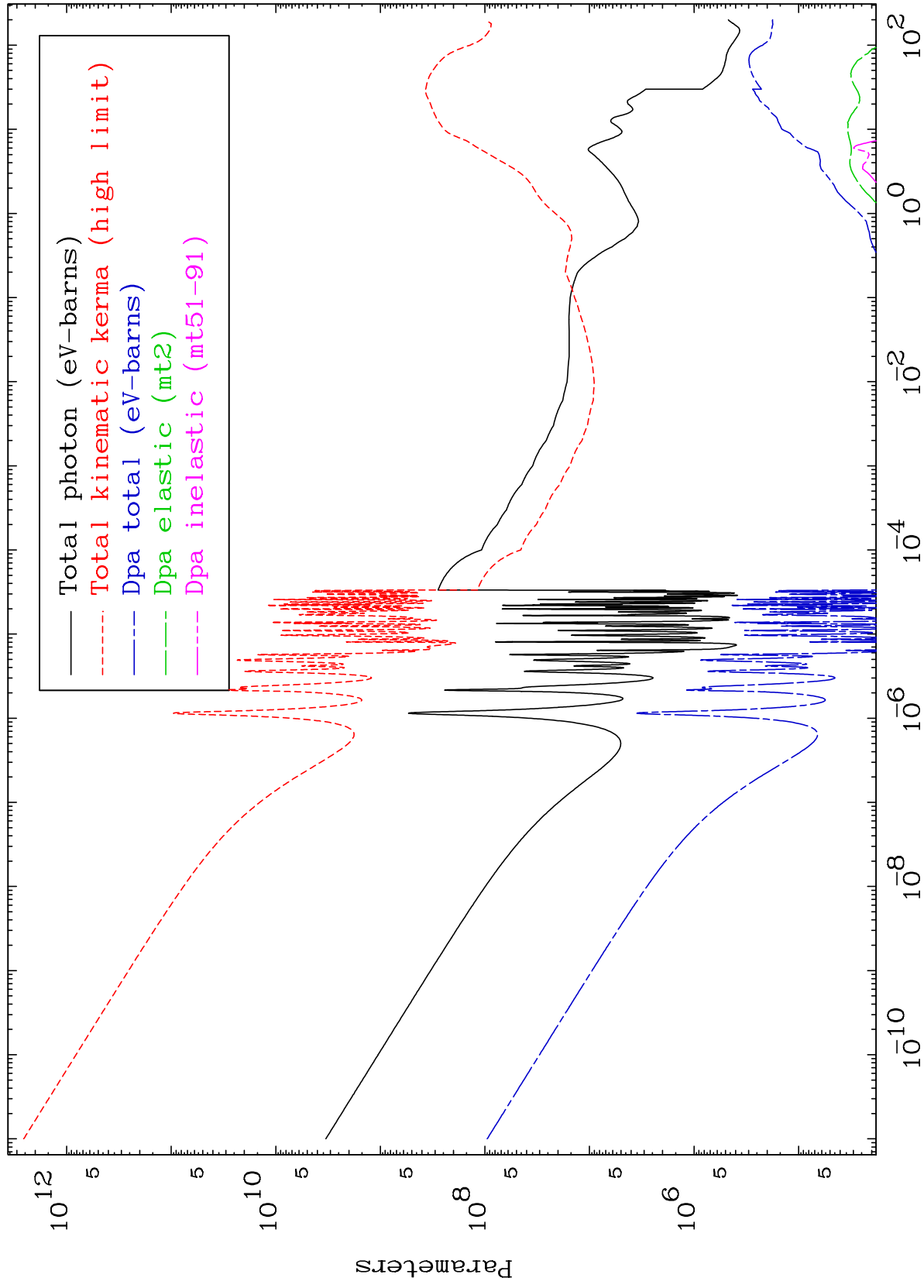
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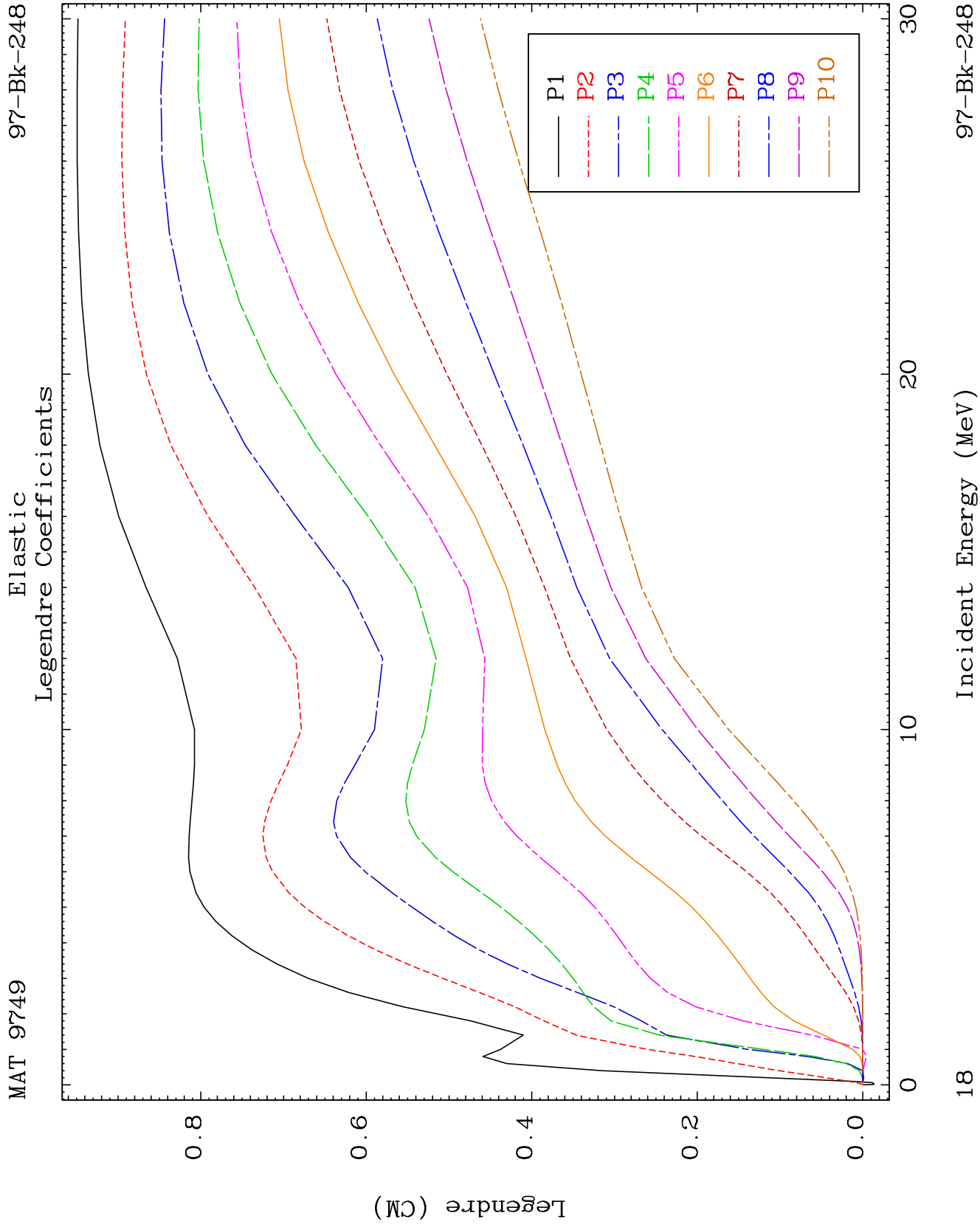
Incident Energy (MeV)

97-Bk-248





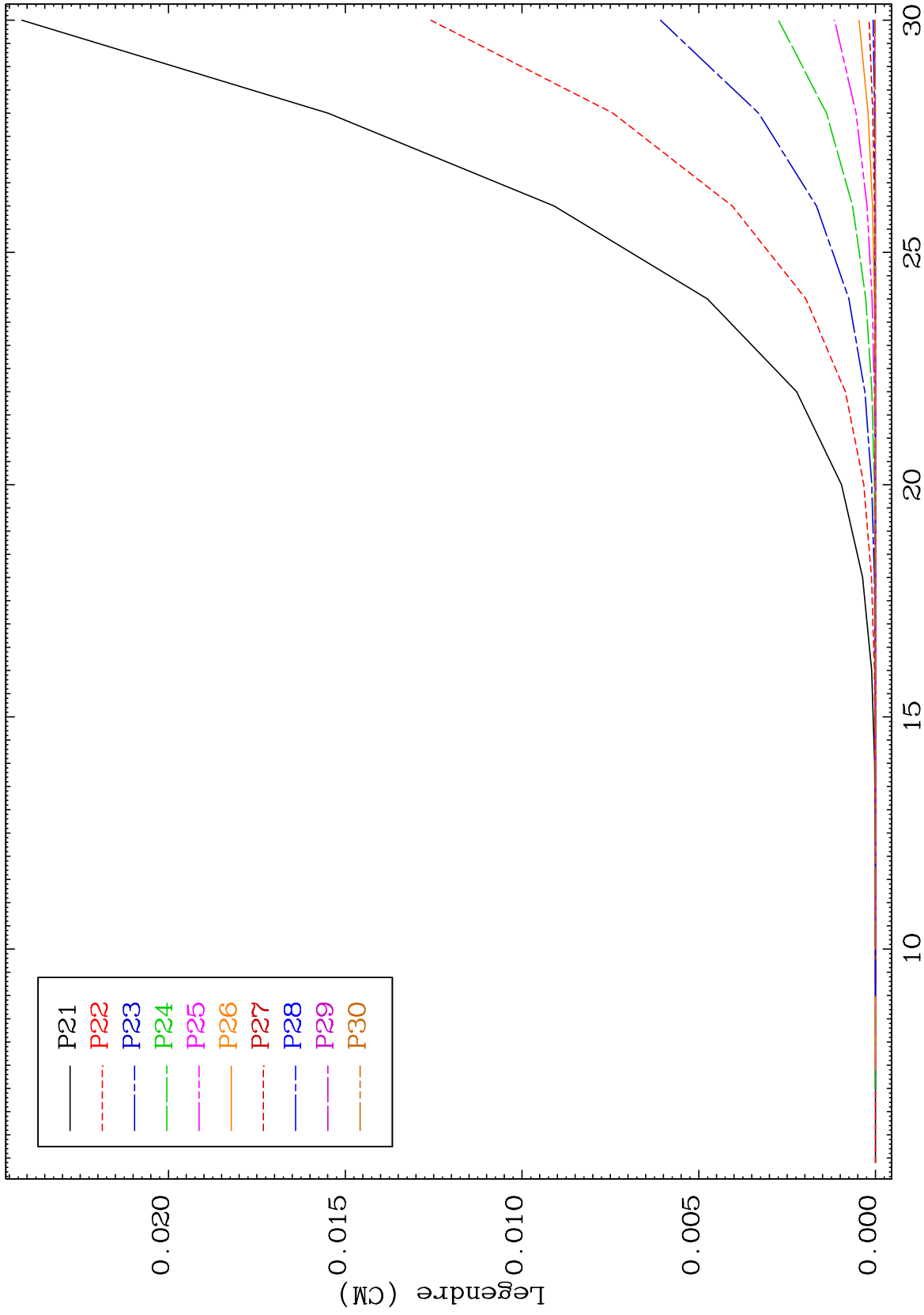




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Elastic
Legendre Coefficients

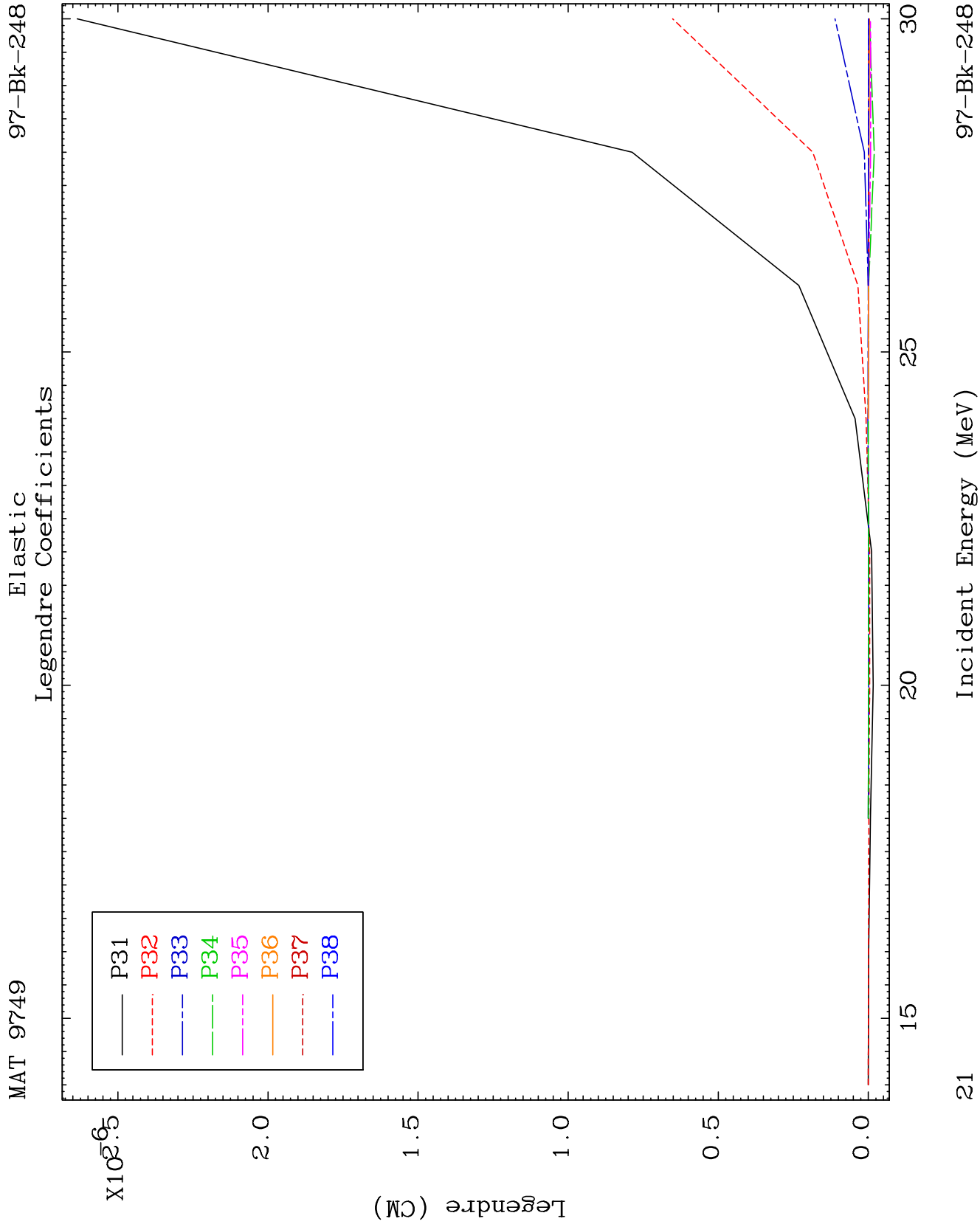
97-Bk-248



20

Incident Energy (MeV)

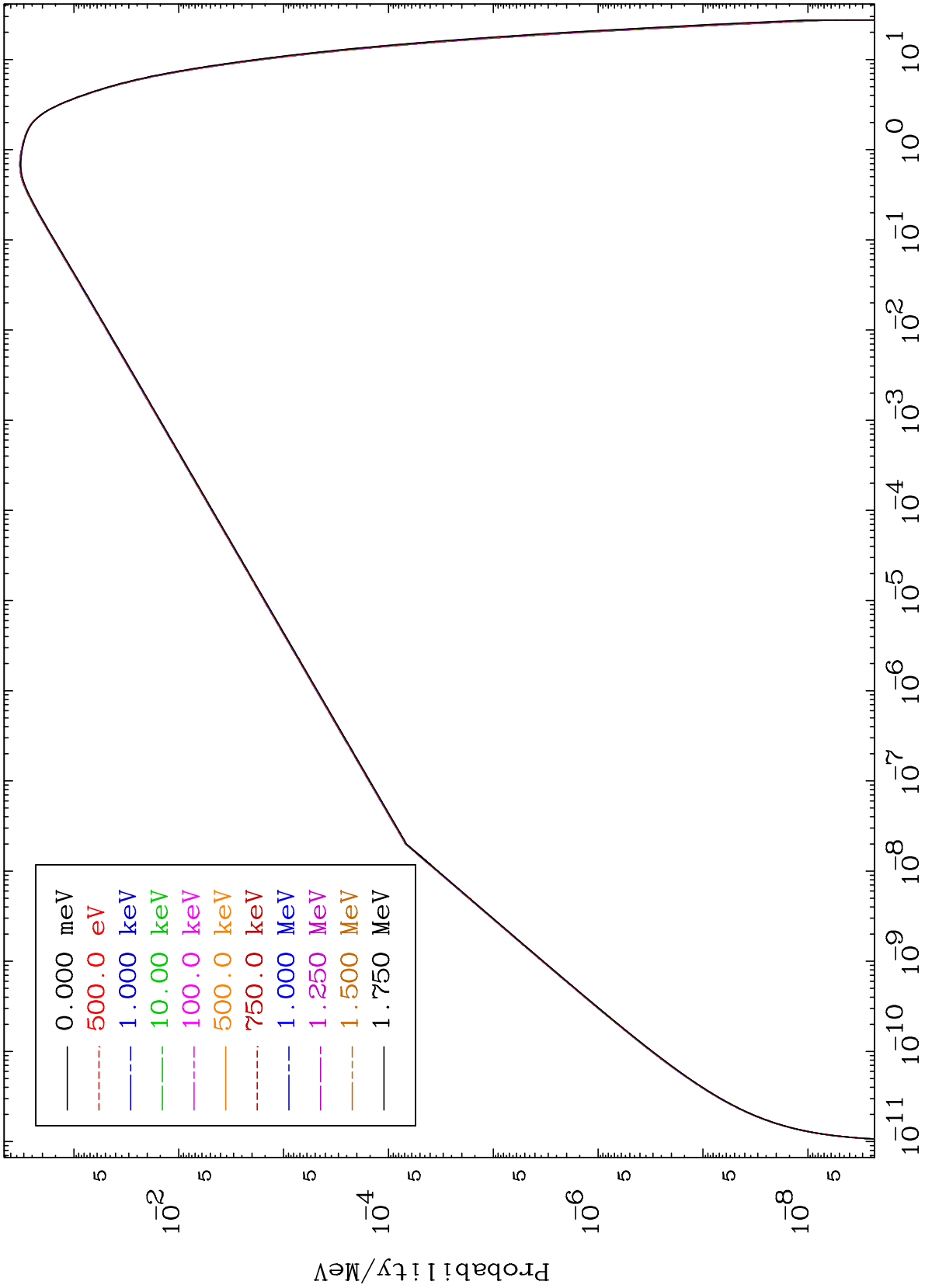
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MAT 9749

Fission
Energy Distribution

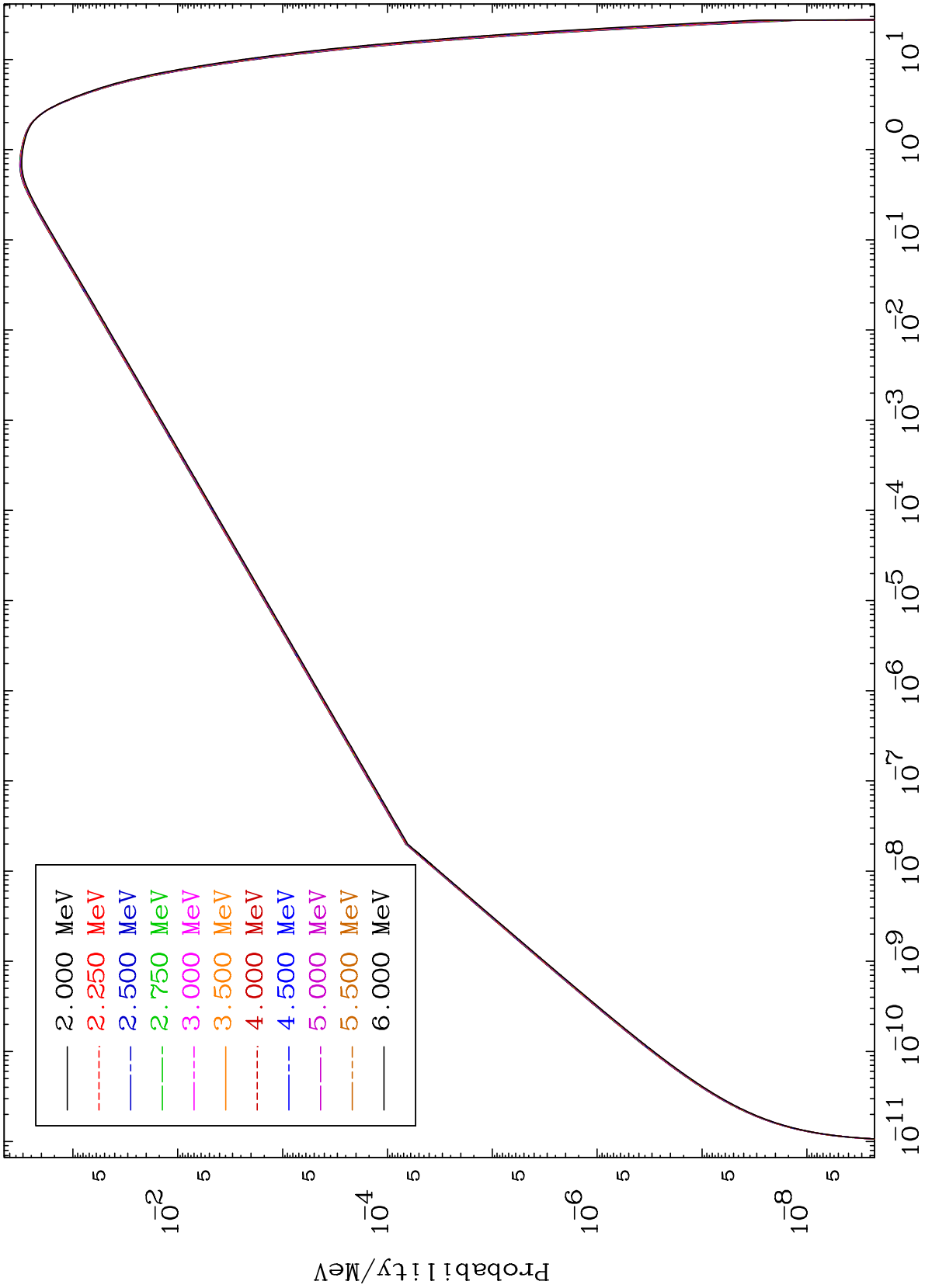
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MAT 9749

Fission
Energy Distribution

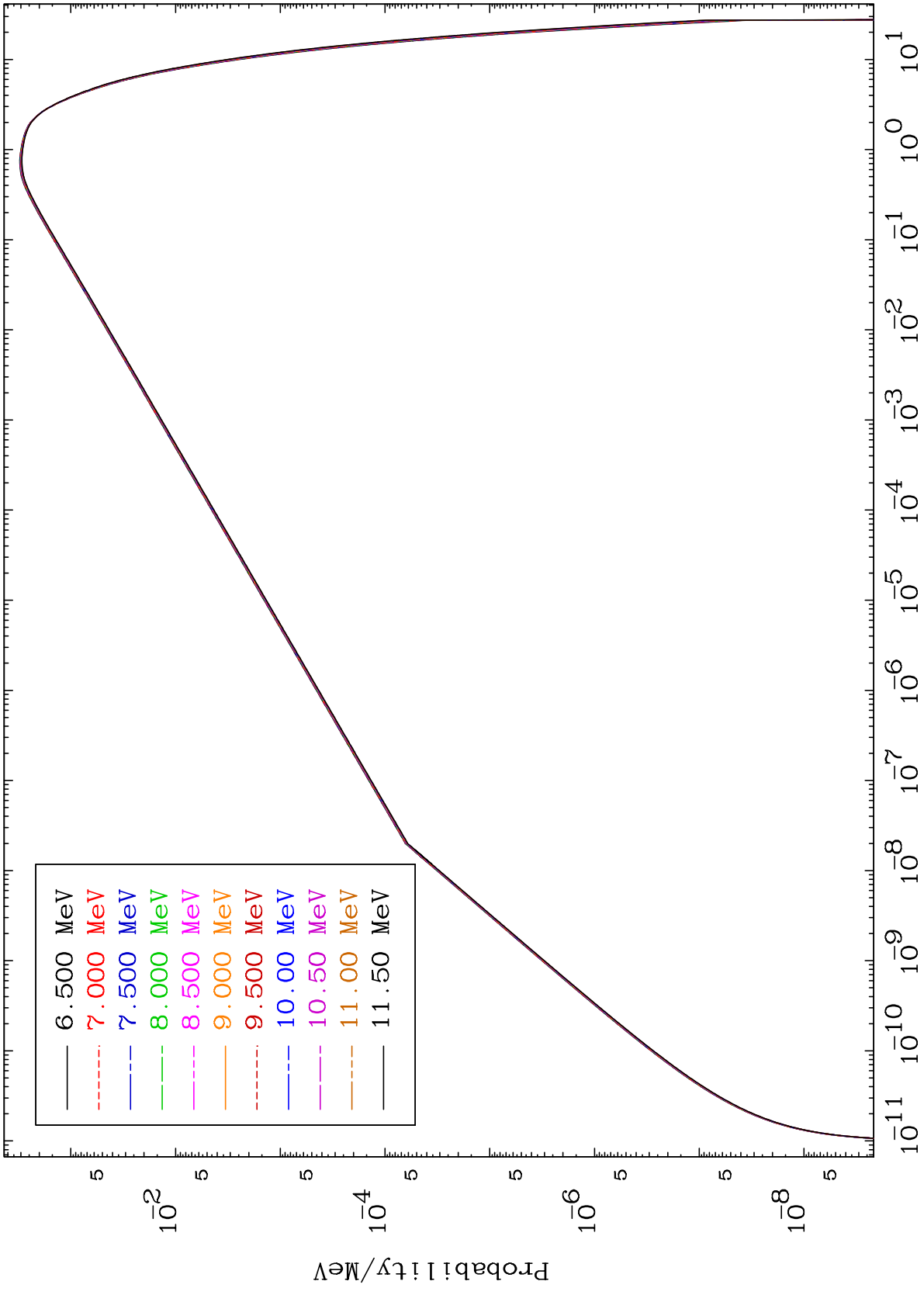
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MAT 9749

Fission
Energy Distribution

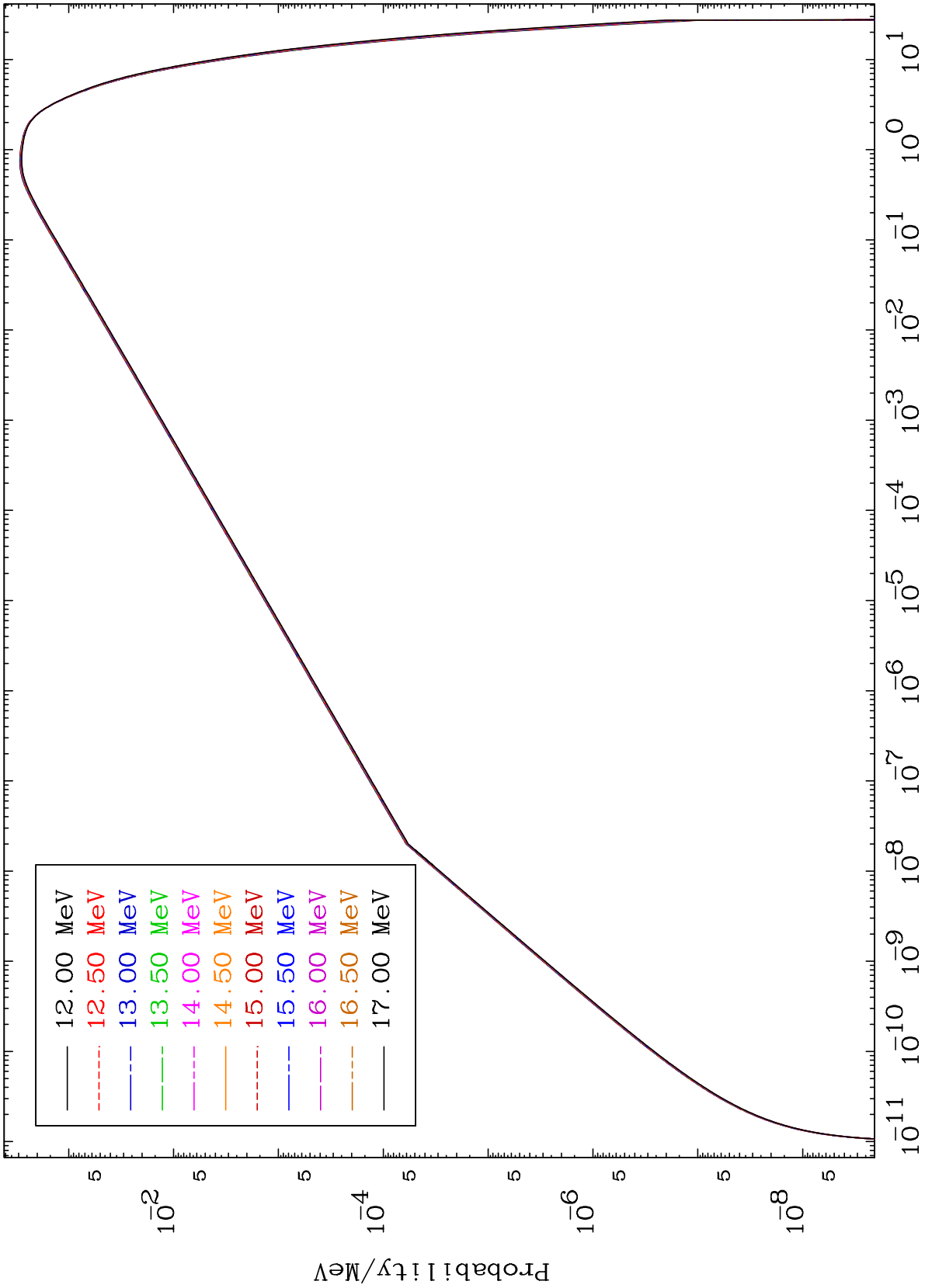
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MAT 9749

Fission
Energy Distribution

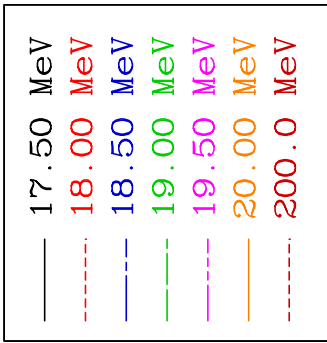
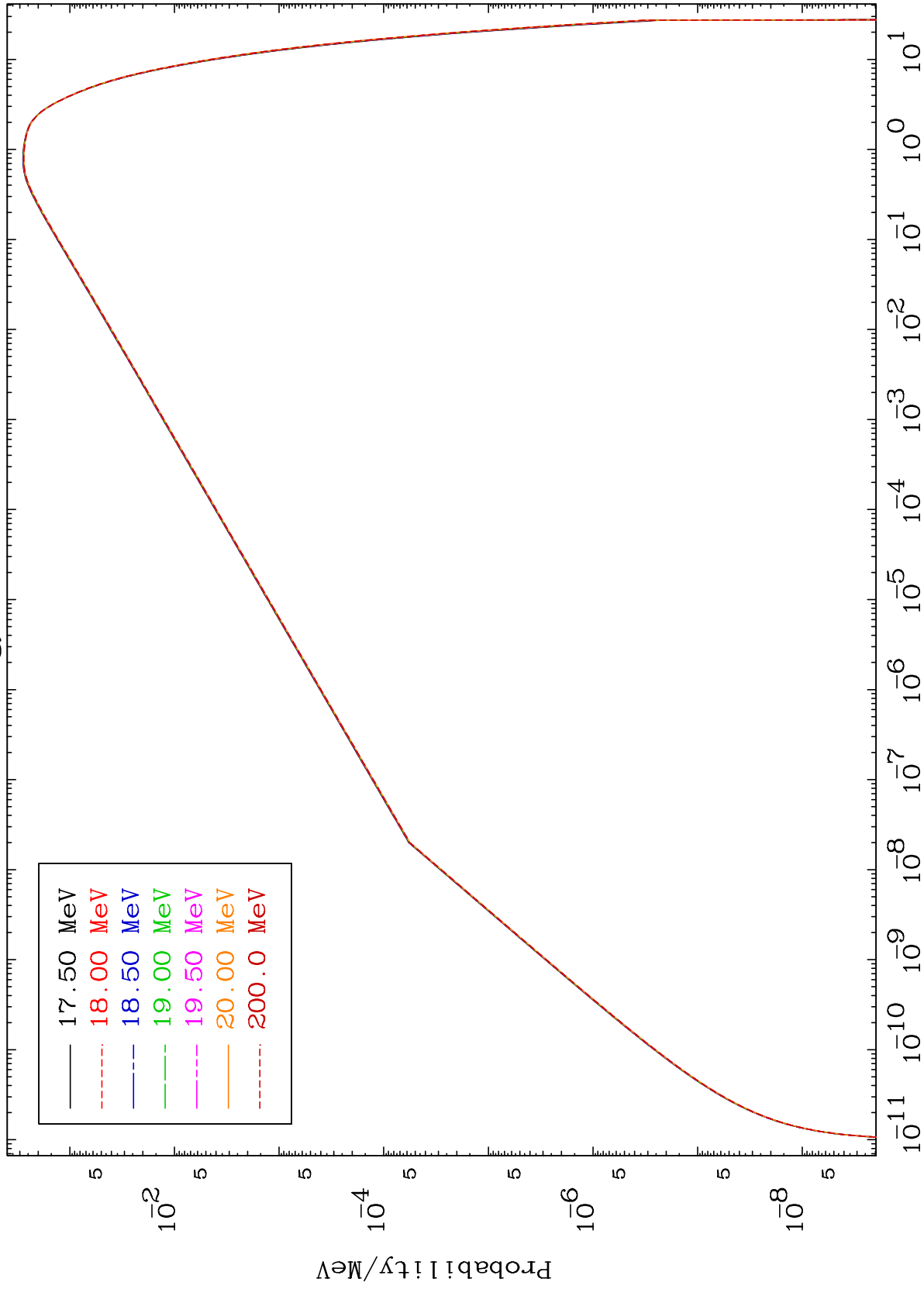
97-Bk-248



MAT 9749

Fission
Energy Distribution

97-Bk-248



26

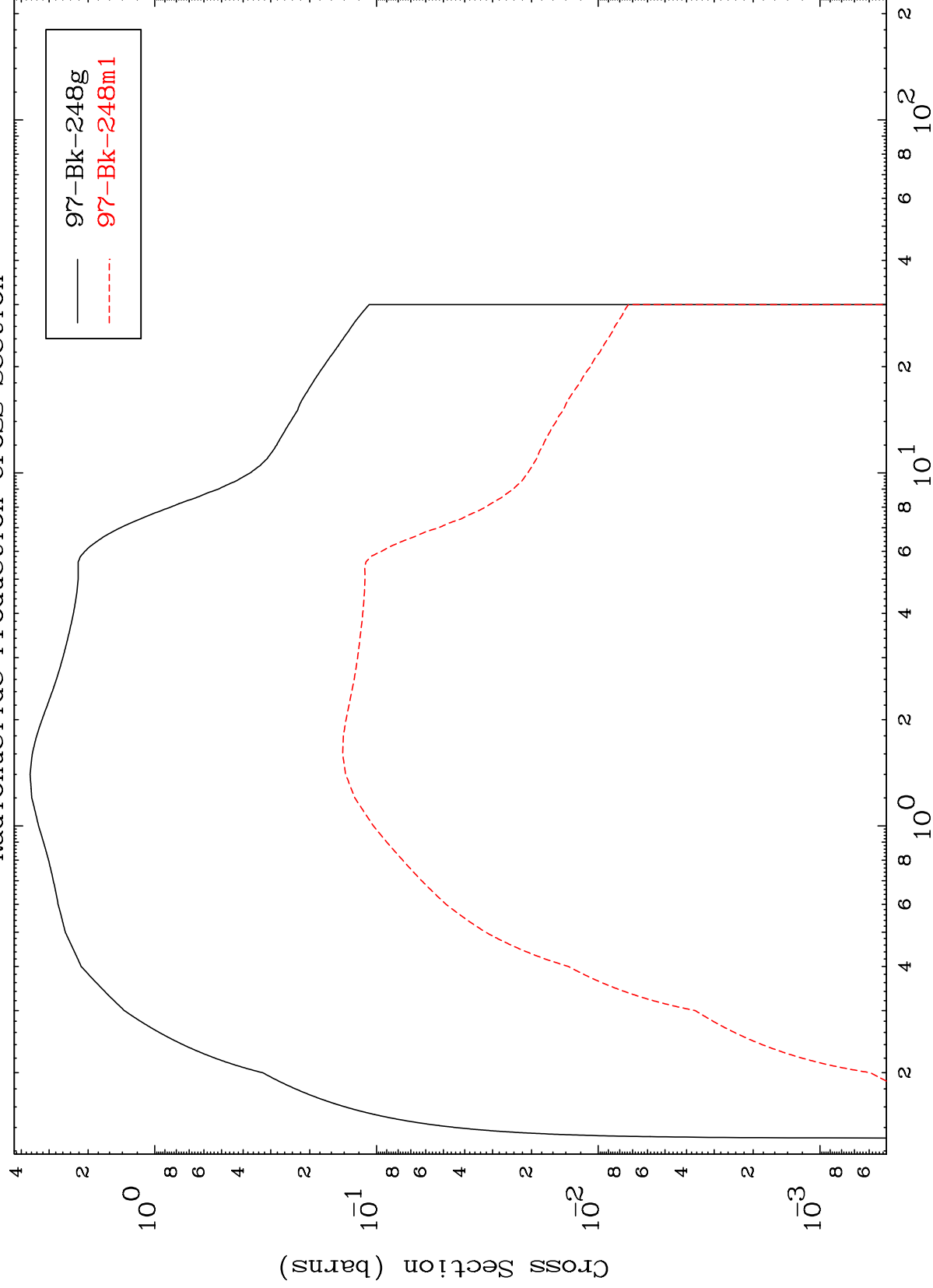
Secondary Energy (MeV)

97-Bk-248

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97-Bk-248

Inelastic
Radionuclide Production Cross Section



27

Incident Energy (MeV)

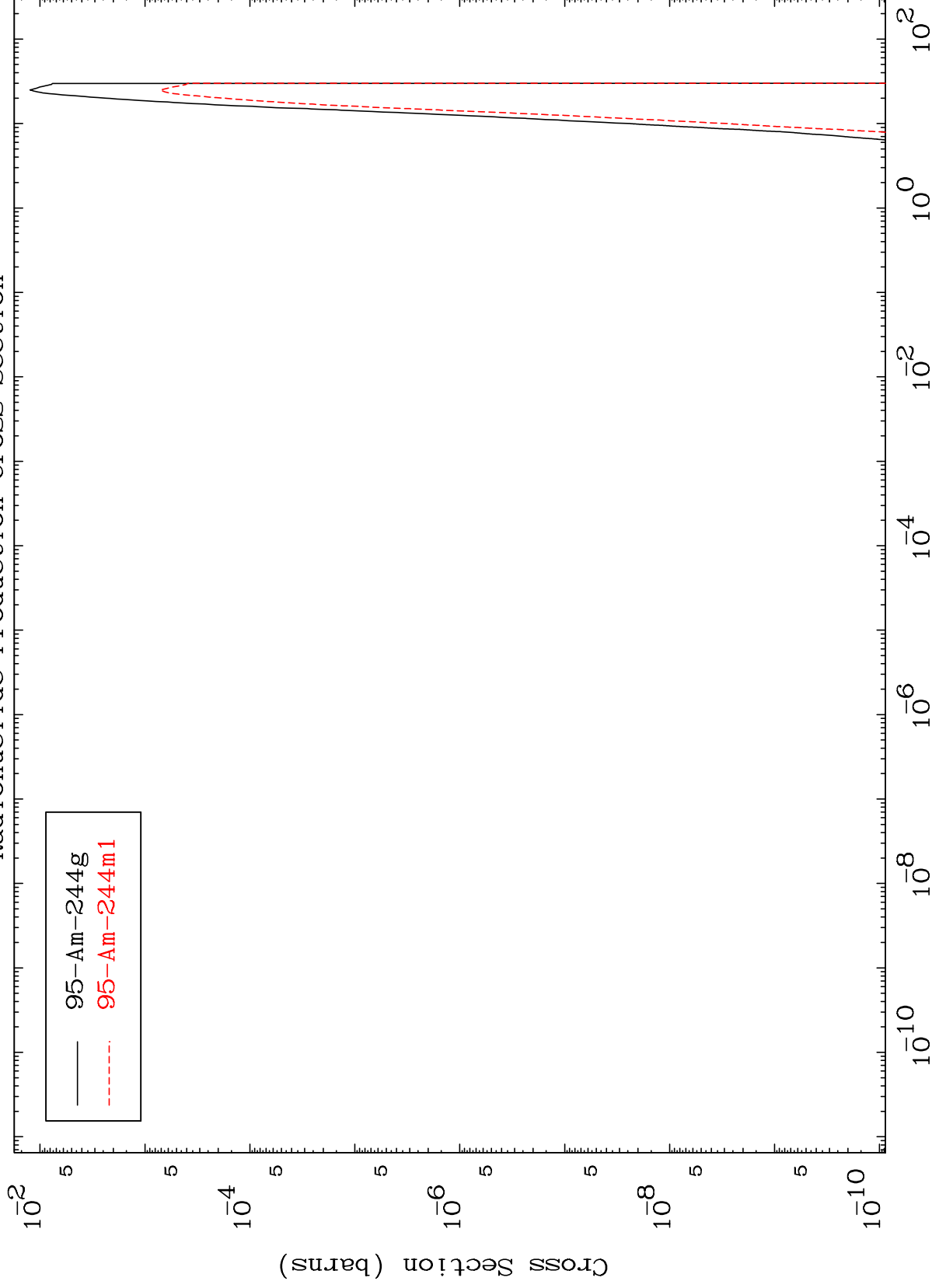
97-Bk-248

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$(n, n') \alpha$

97-Bk-248

Radionuclide Production Cross Section



28

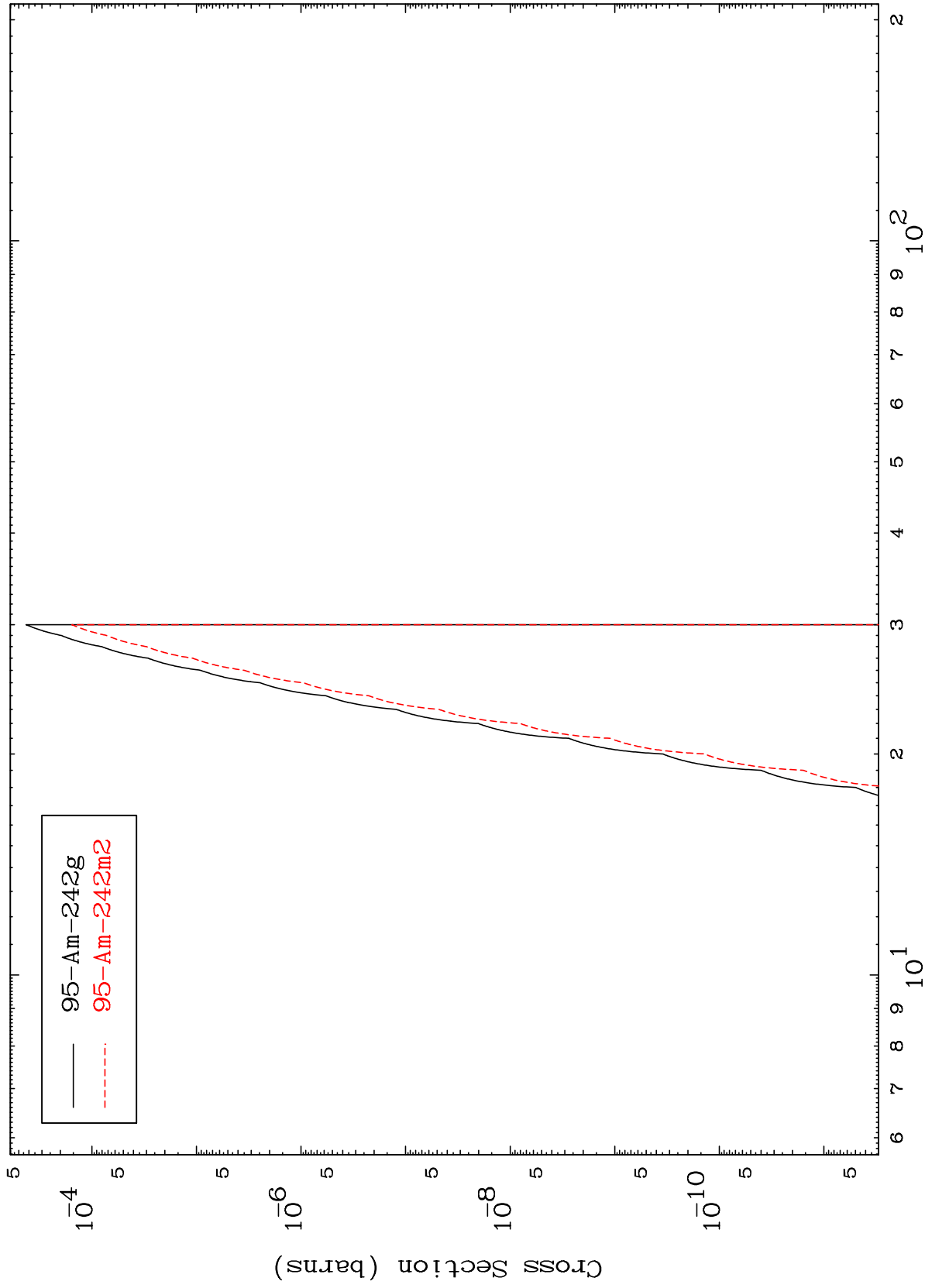
Incident Energy (MeV)

97-Bk-248

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97-Bk-248

(n,3n) α
Radionuclide Production Cross Section



29

Incident Energy (MeV)

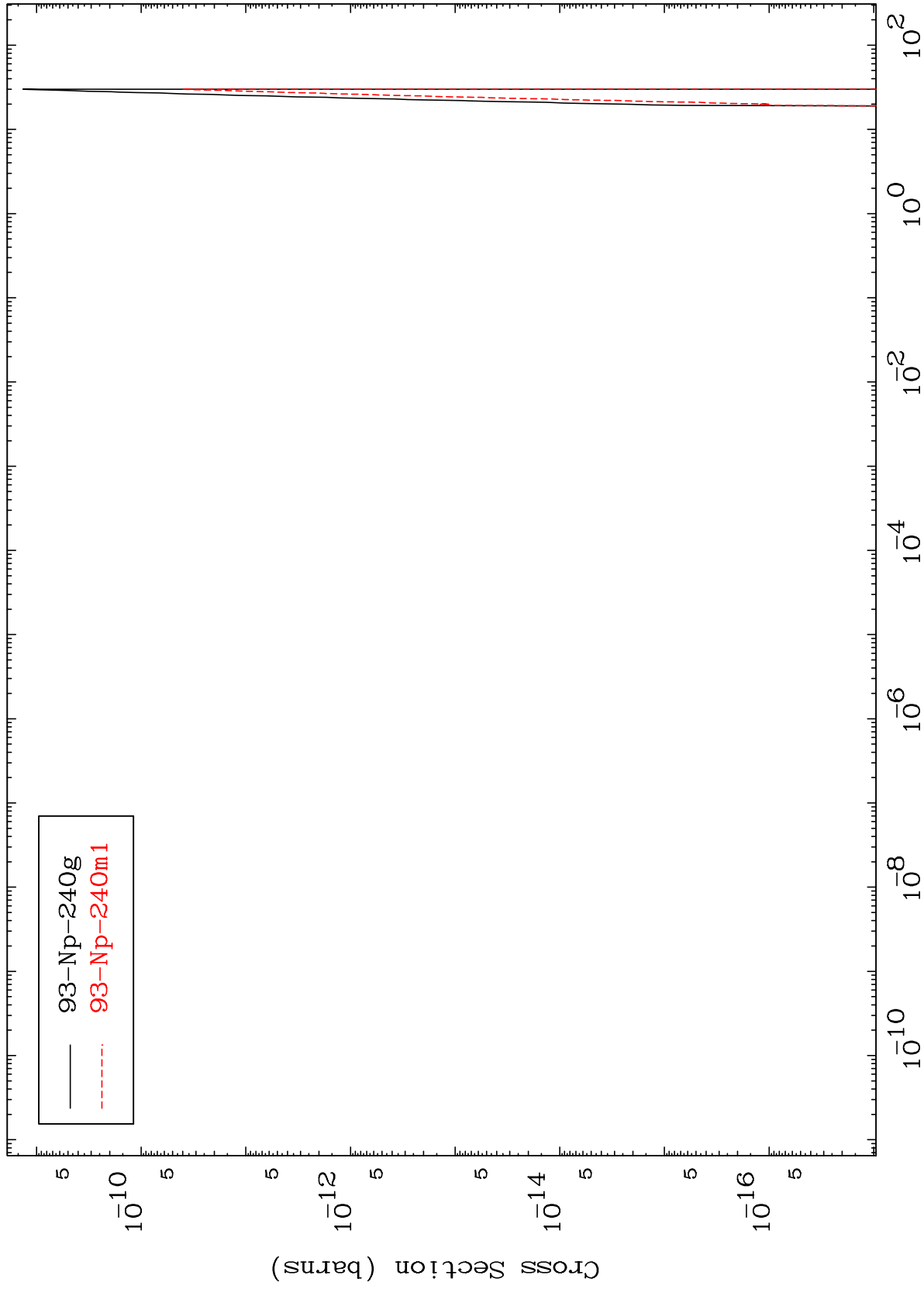
97-Bk-248

MAT 9749

(n,n') 2 α

97-Bk-248

Radionuclide Production Cross Section



30

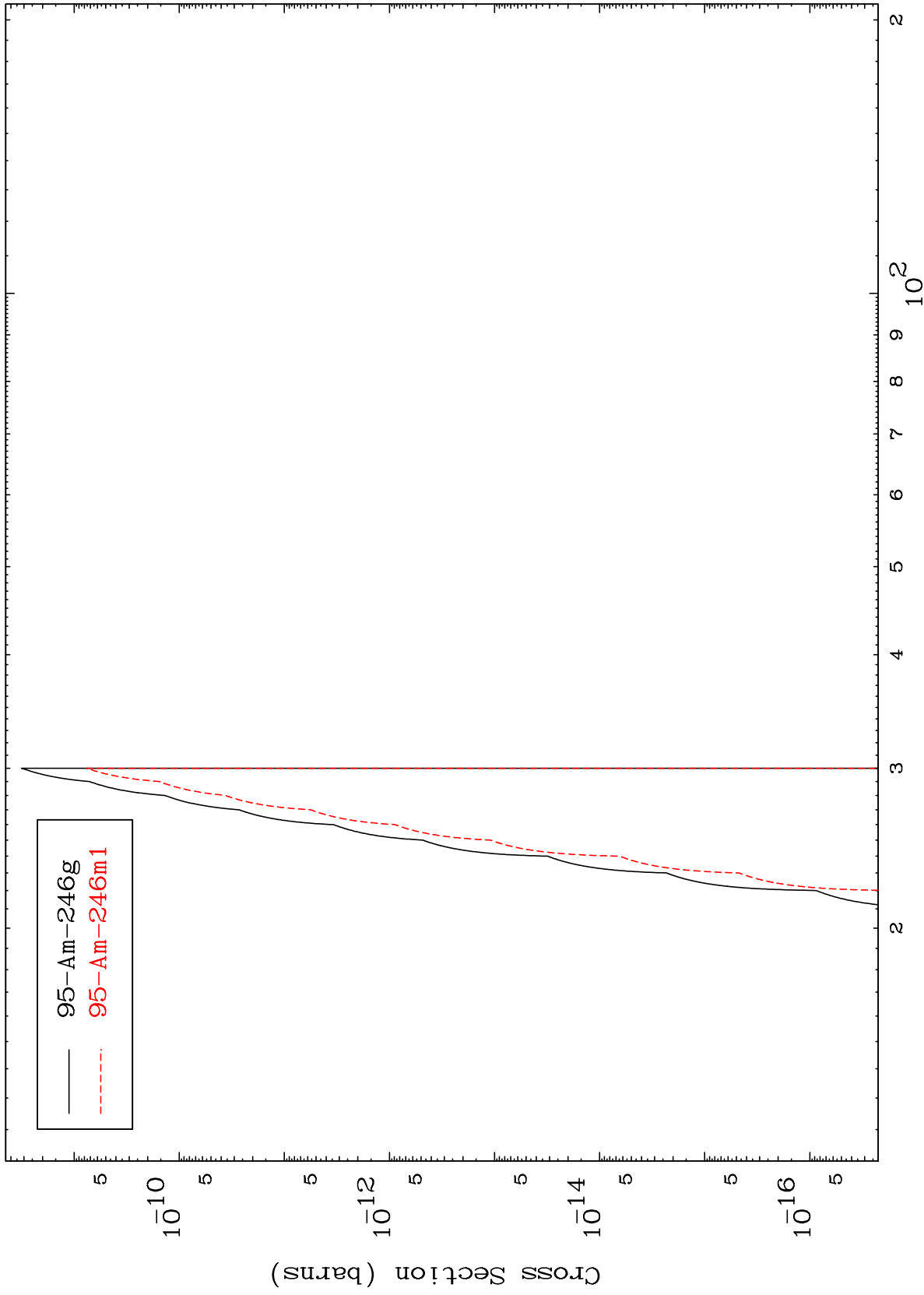
Incident Energy (MeV)

97-Bk-248

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97-Bk-248

(n,2n) p
Radionuclide Production Cross Section



31

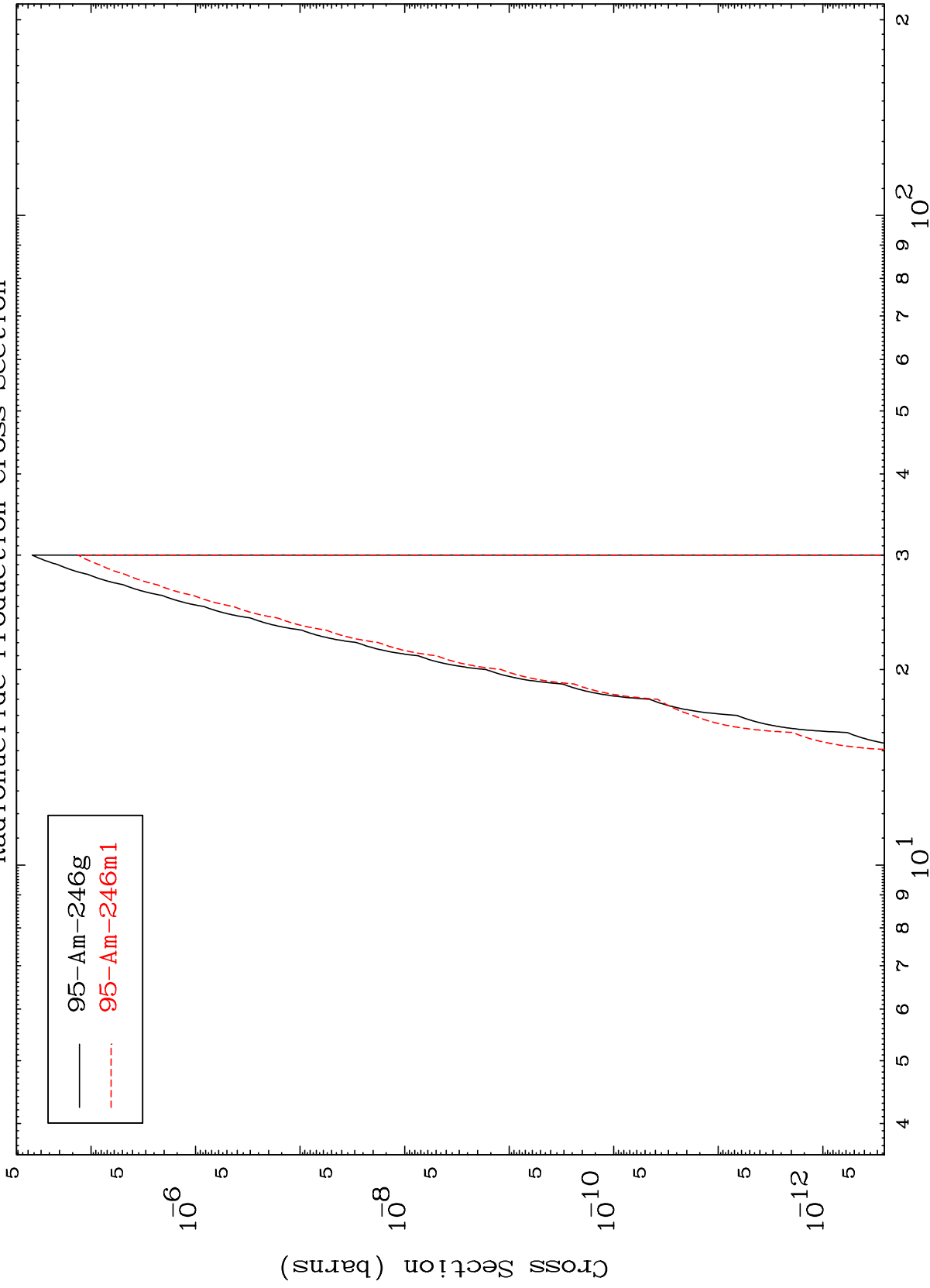
97-Bk-248

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(n,He-3)

97-Bk-248

Radionuclide Production Cross Section



32

Incident Energy (MeV)

97-Bk-248