

Program EVALPLOT
(Version 2018-1)

by

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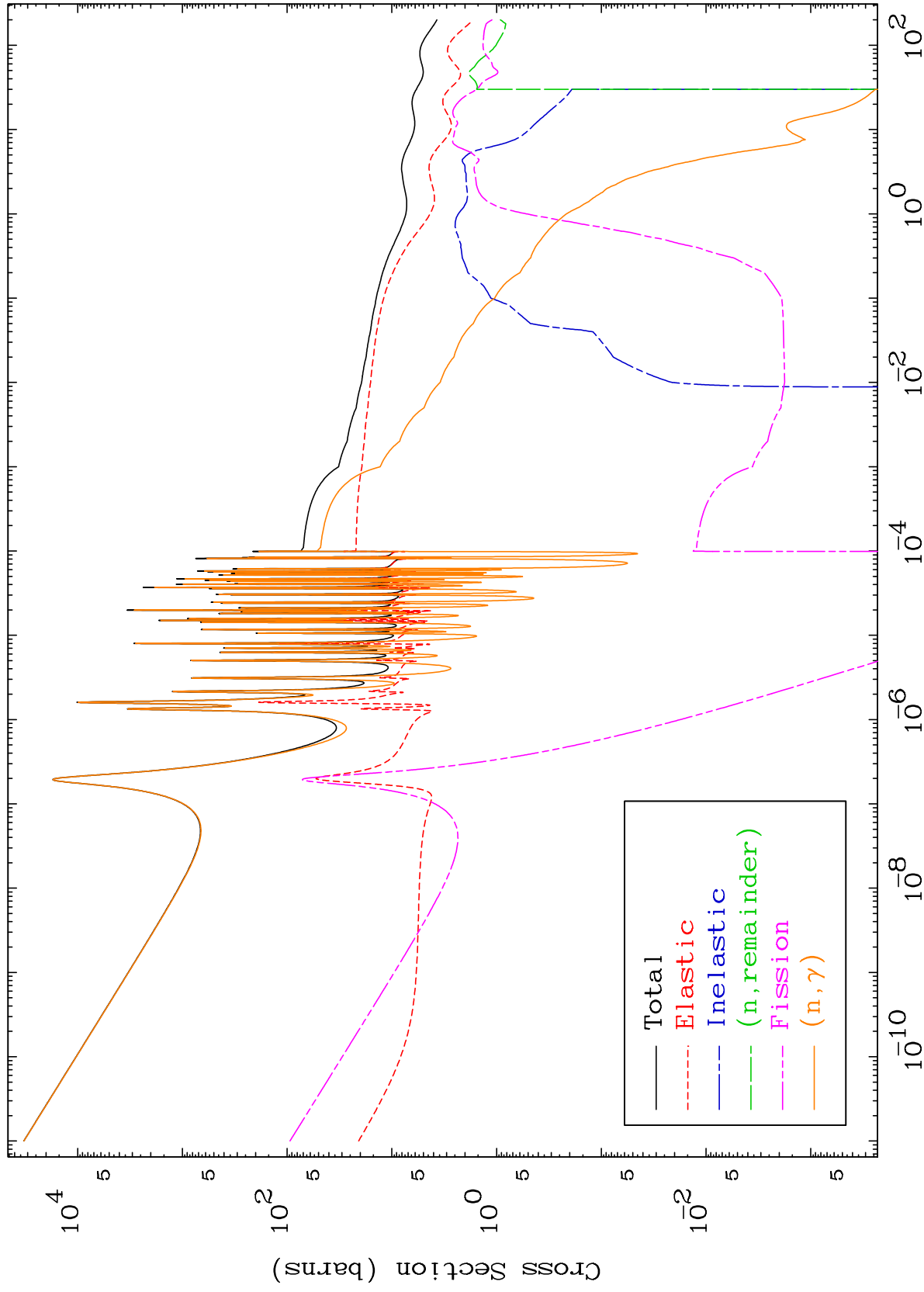
Press Mouse Button to Start

MAT 9752

Major

293 Kelvin Cross Sections

97-Bk-249



1

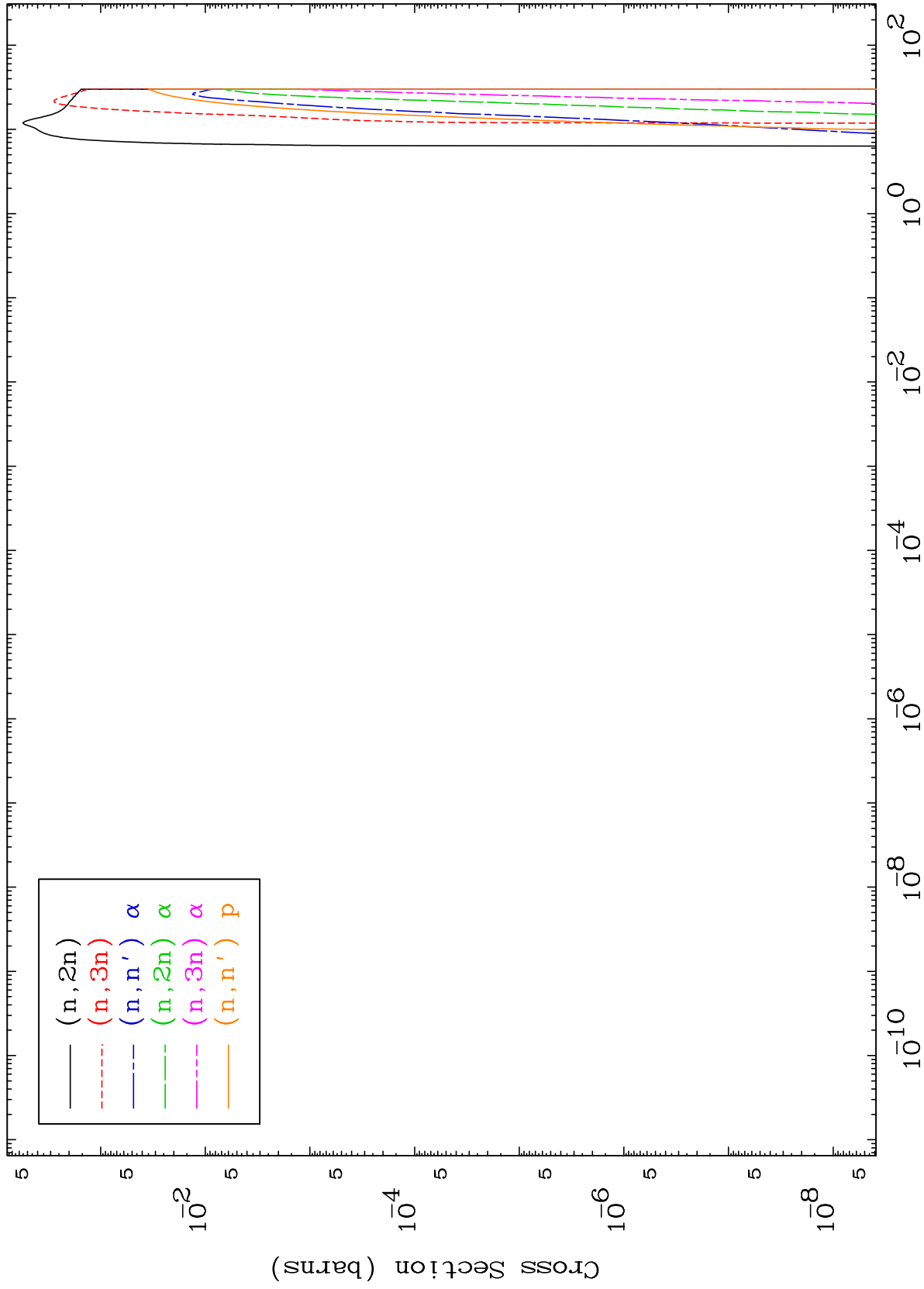
Incident Energy (MeV)

97-Bk-249

MAT 9752

Neutron Production
293 Kelvin Cross Sections

97-Bk-249



2

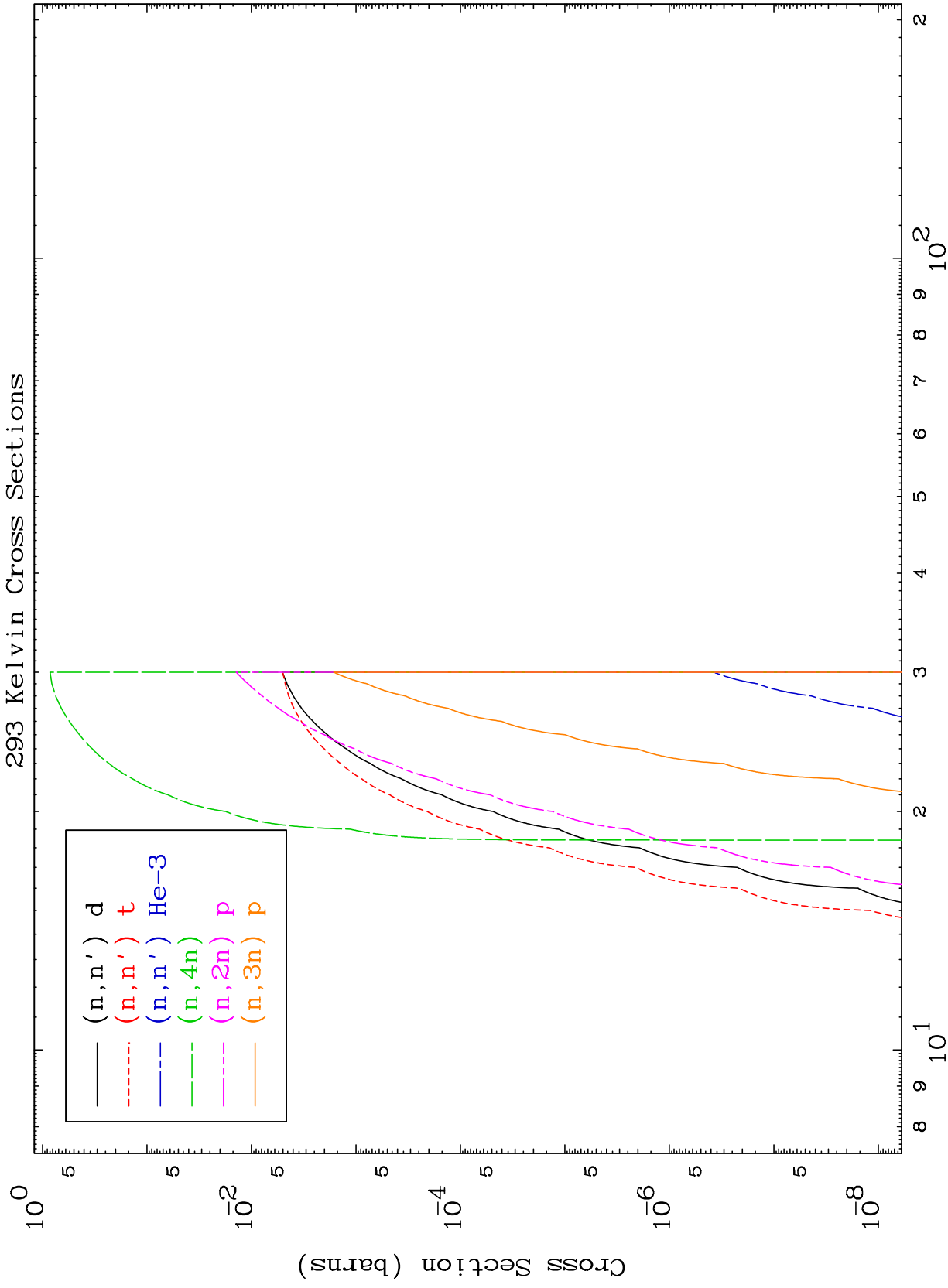
Incident Energy (MeV)

97-Bk-249

MAT 9752

Neutron Production
293 Kelvin Cross Sections

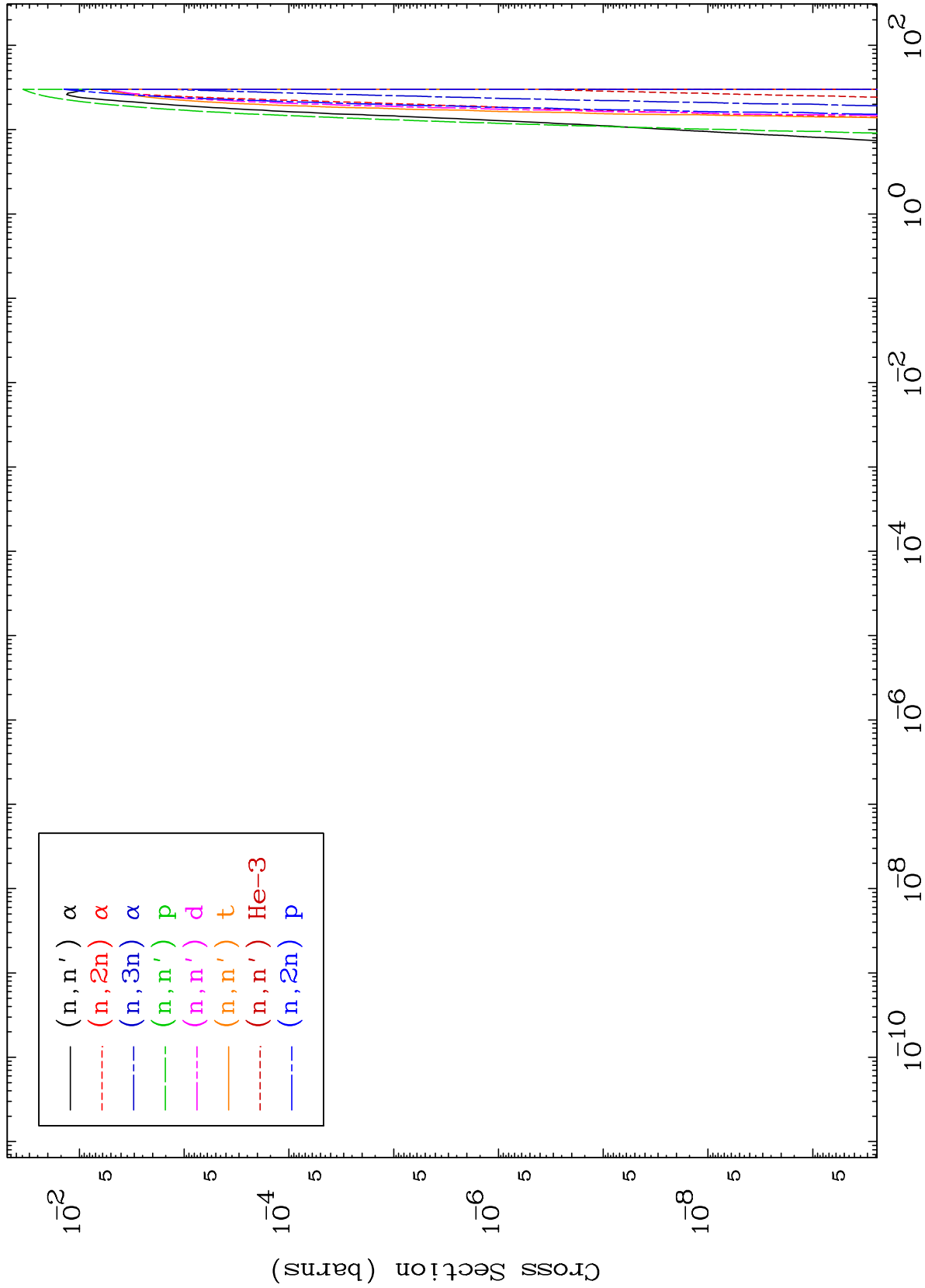
97-Bk-249



3

Incident Energy (MeV)

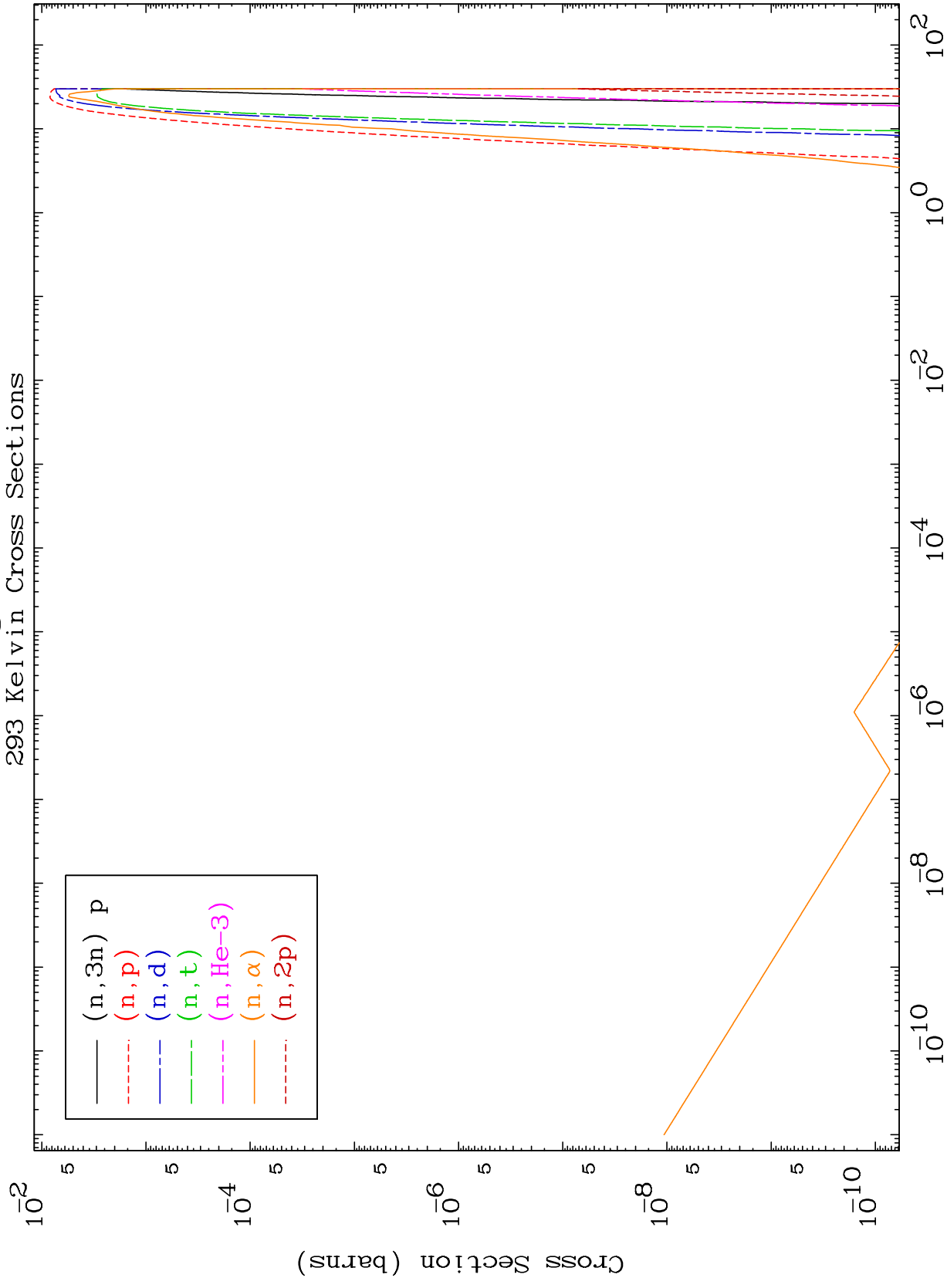
97-Bk-249



MAT 9752

Charged Particle 293 Kelvin Cross Sections

97-Bk-249



5

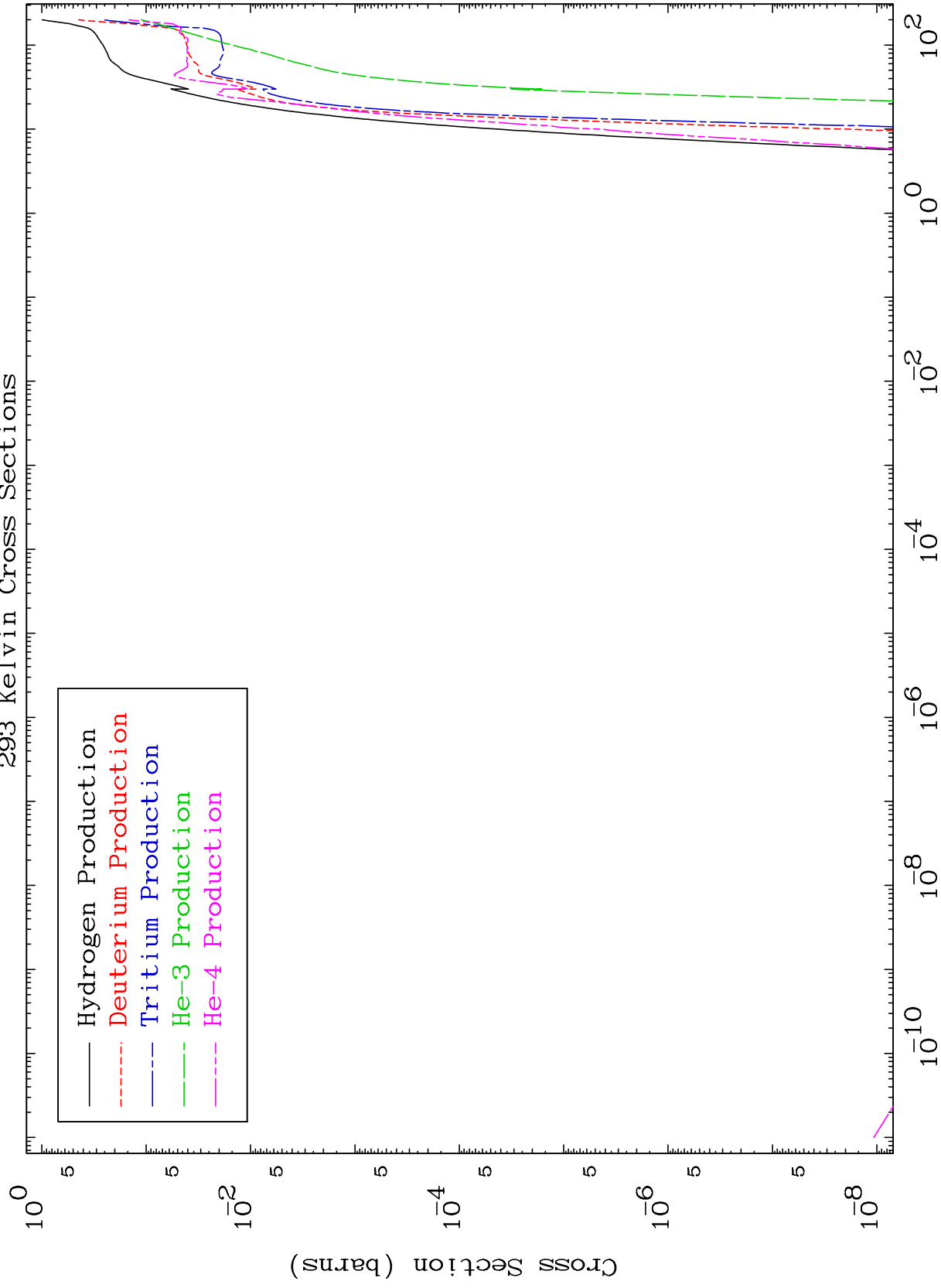
Incident Energy (MeV)

97-Bk-249

MAT 9752

Particle Production
293 Kelvin Cross Sections

97-Bk-249



6

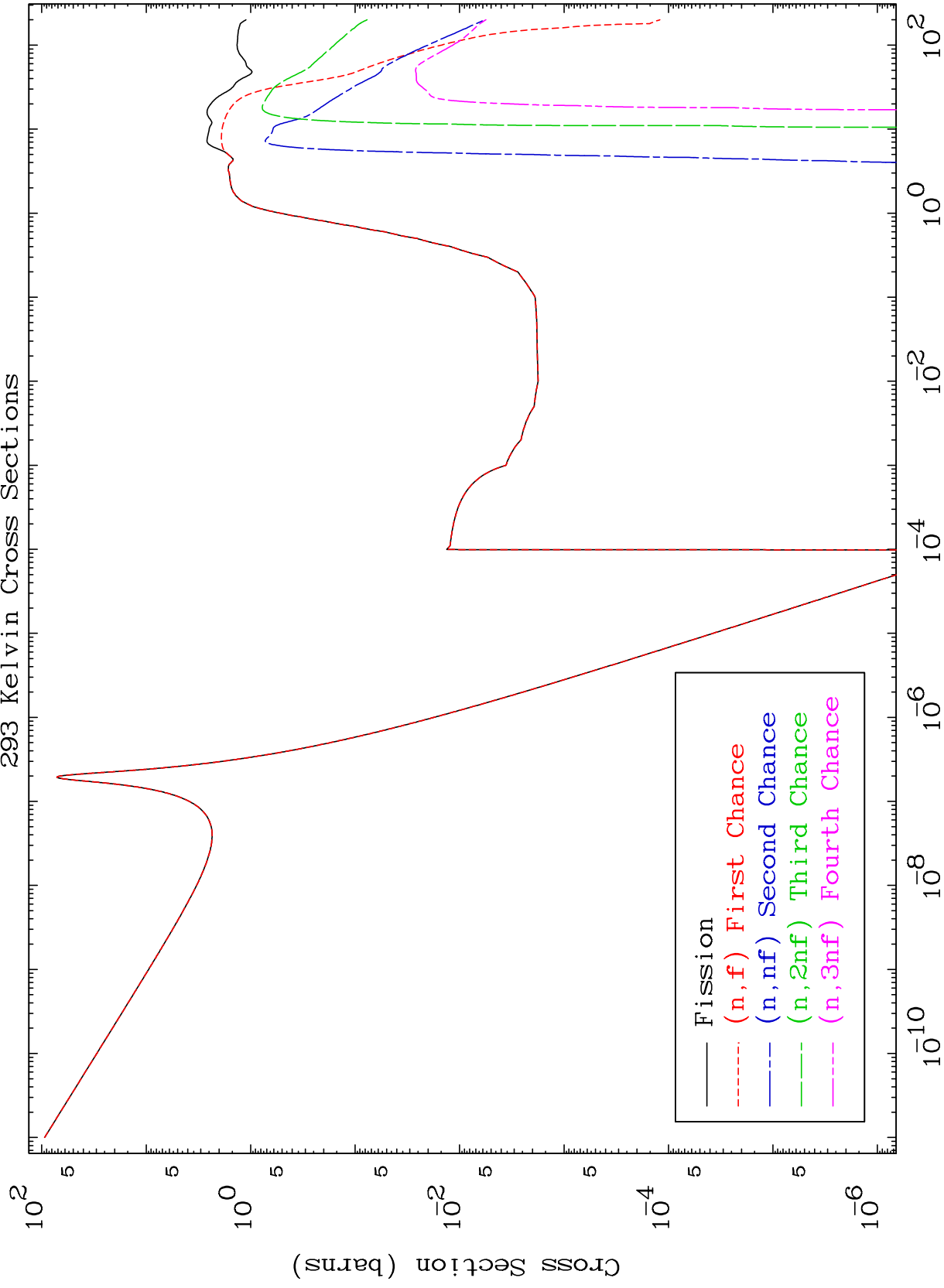
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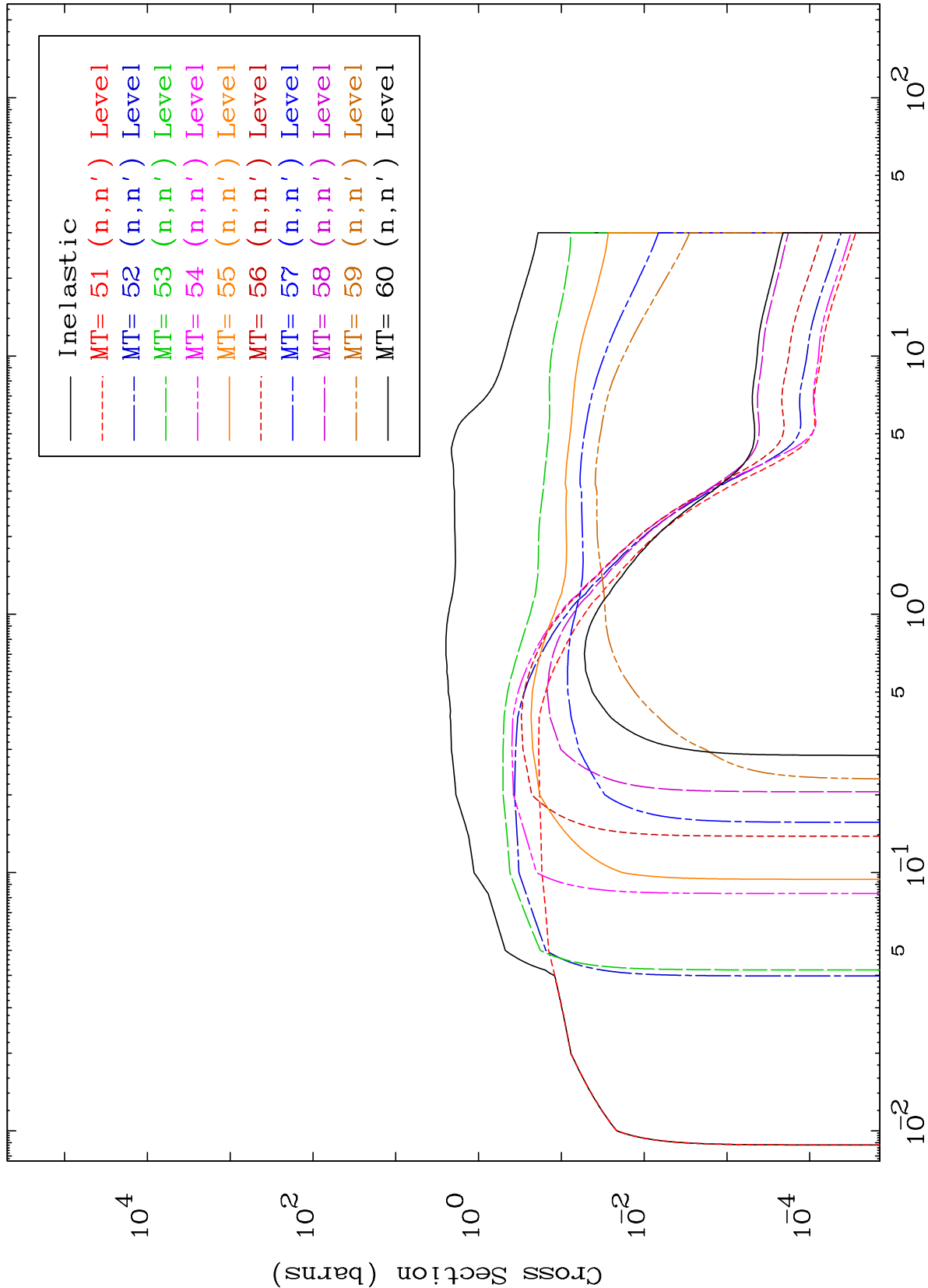
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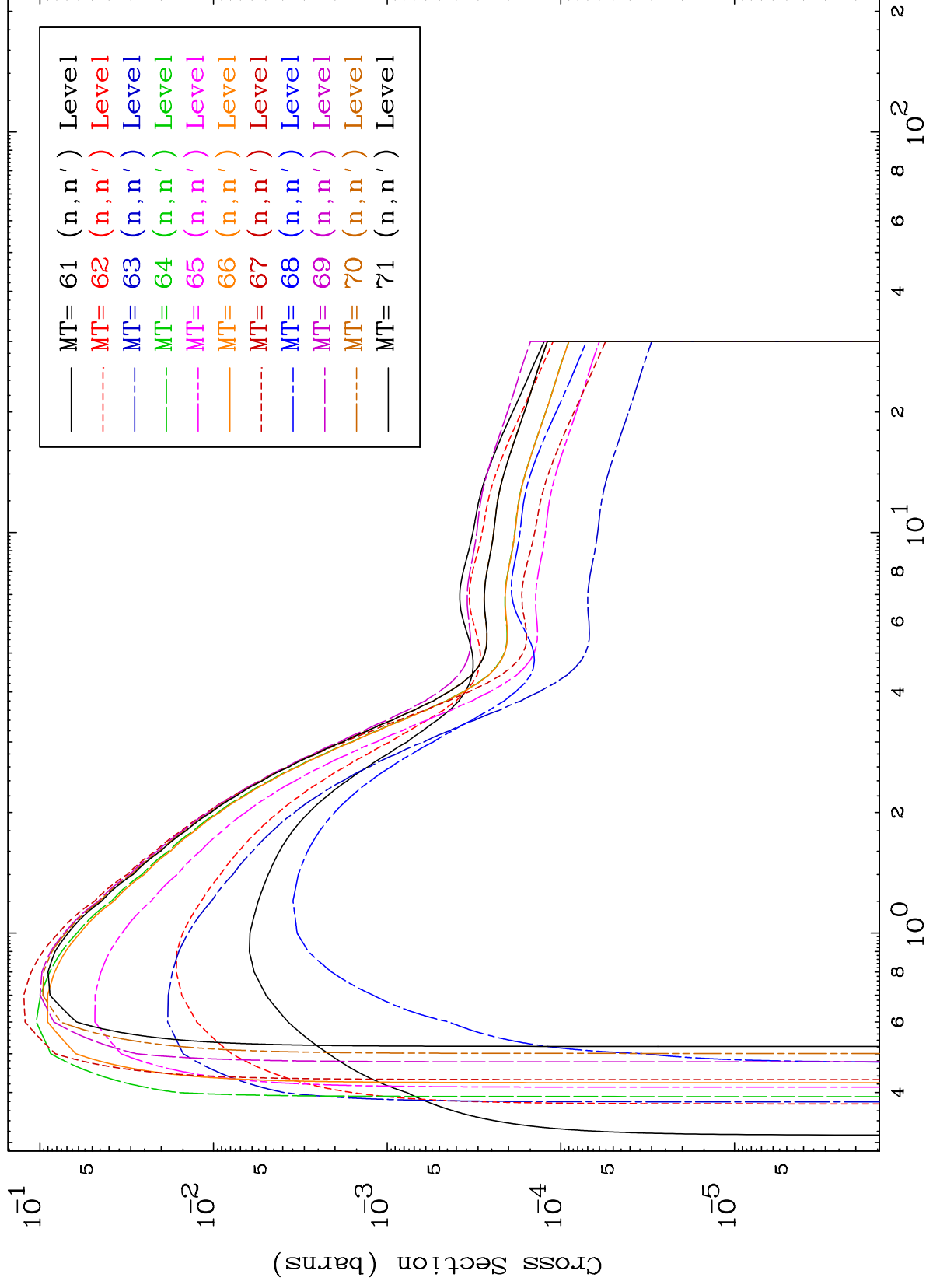
MAT 9752

293 Kelvin Cross Sections

97-Bk-249





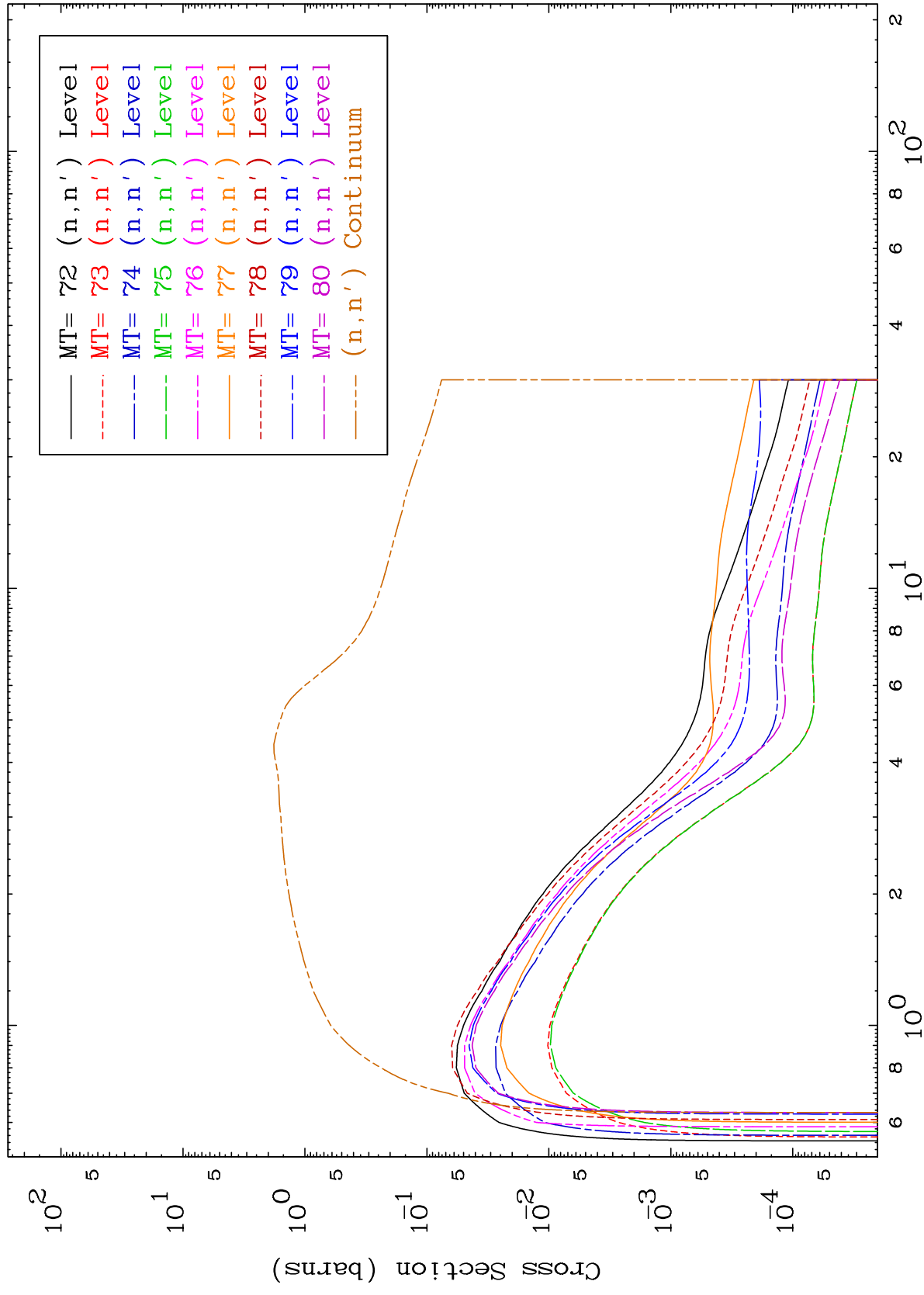


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(n,n') Level

97-Bk-249

293 Kelvin Cross Sections



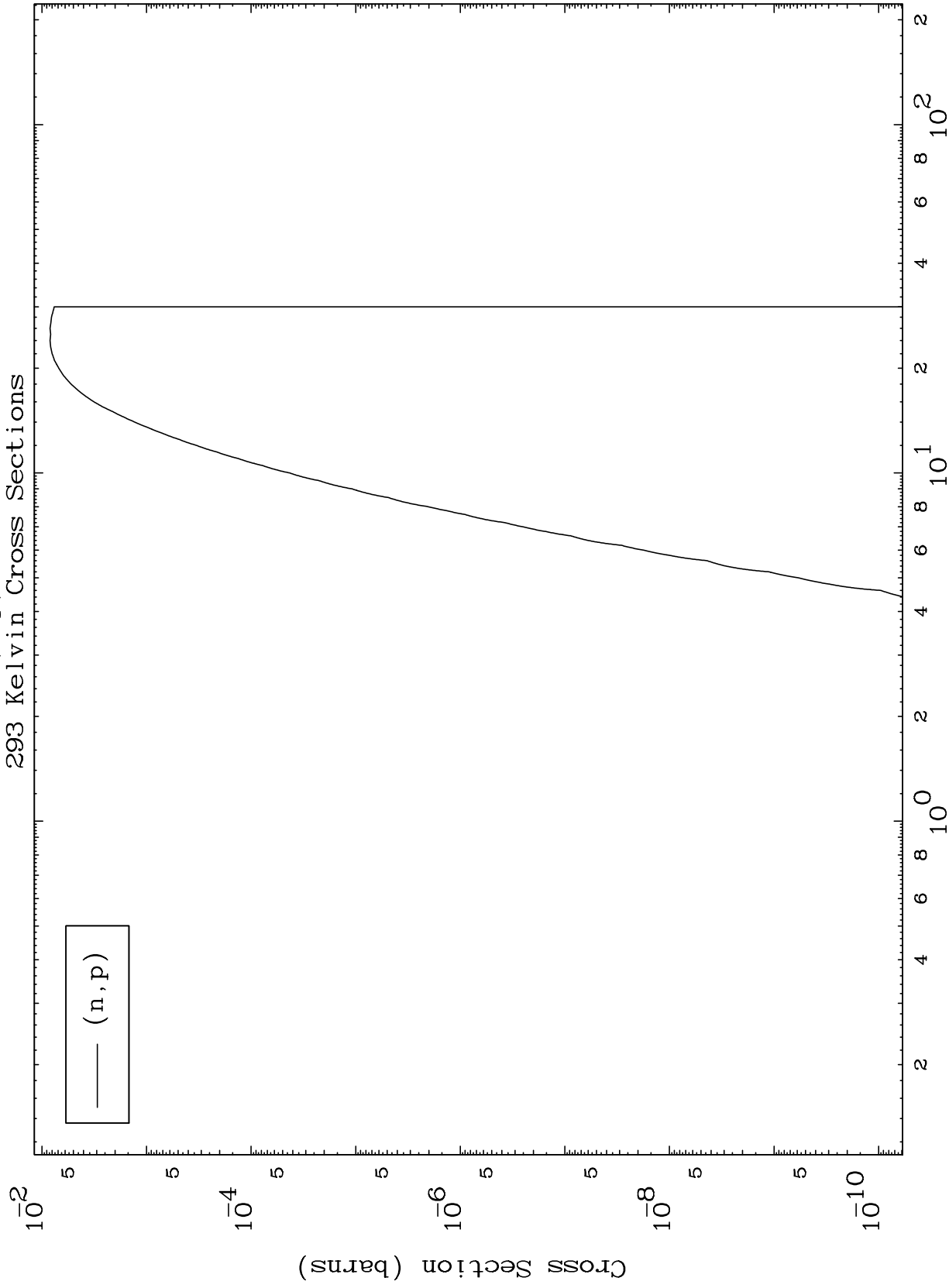
Incident Energy (MeV)

97-Bk-249

MAT 9752

97-Bk-249

(n,p) Levels
293 Kelvin Cross Sections

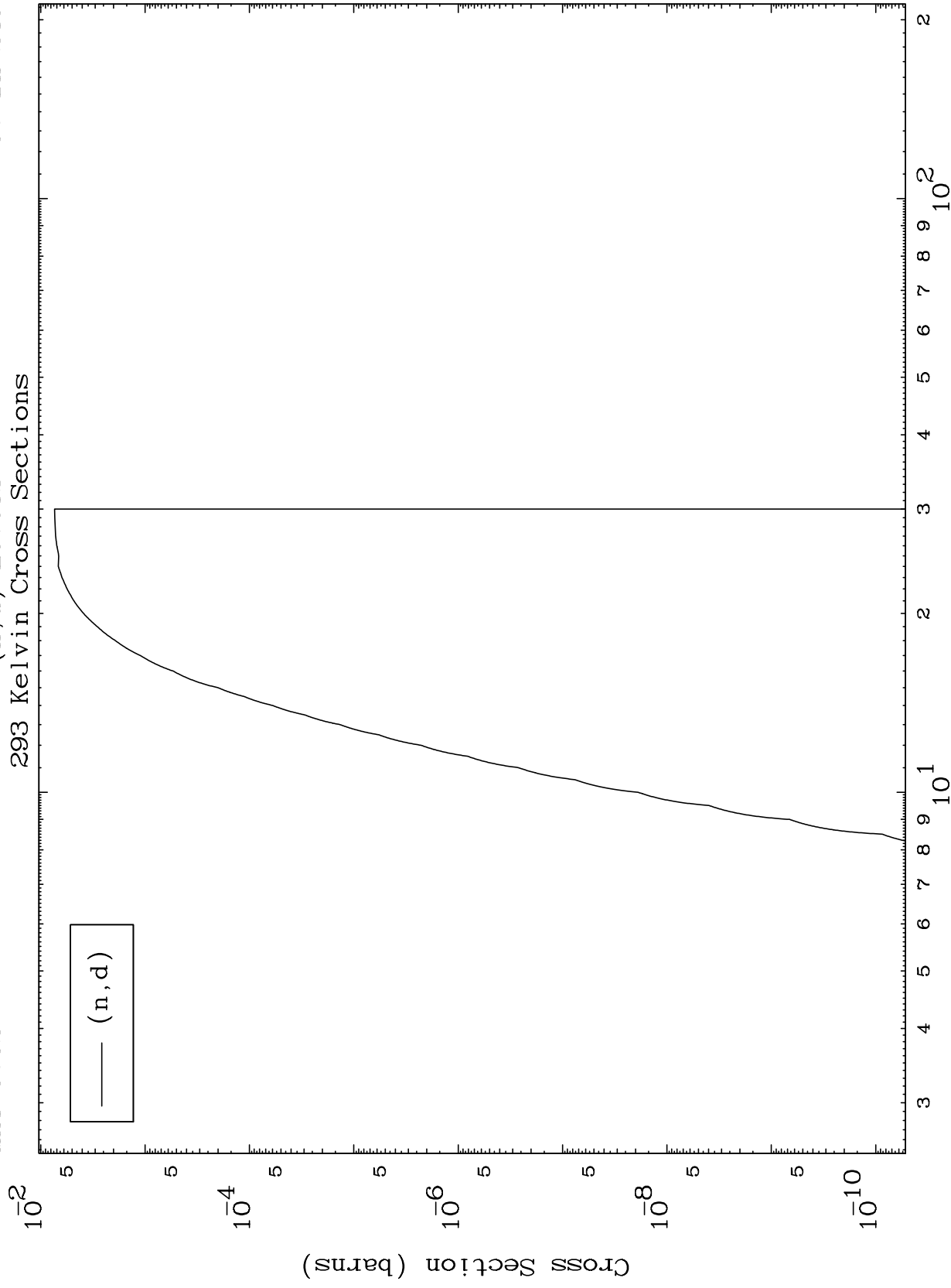


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(n,d) Levels

97-Bk-249

293 Kelvin Cross Sections



12

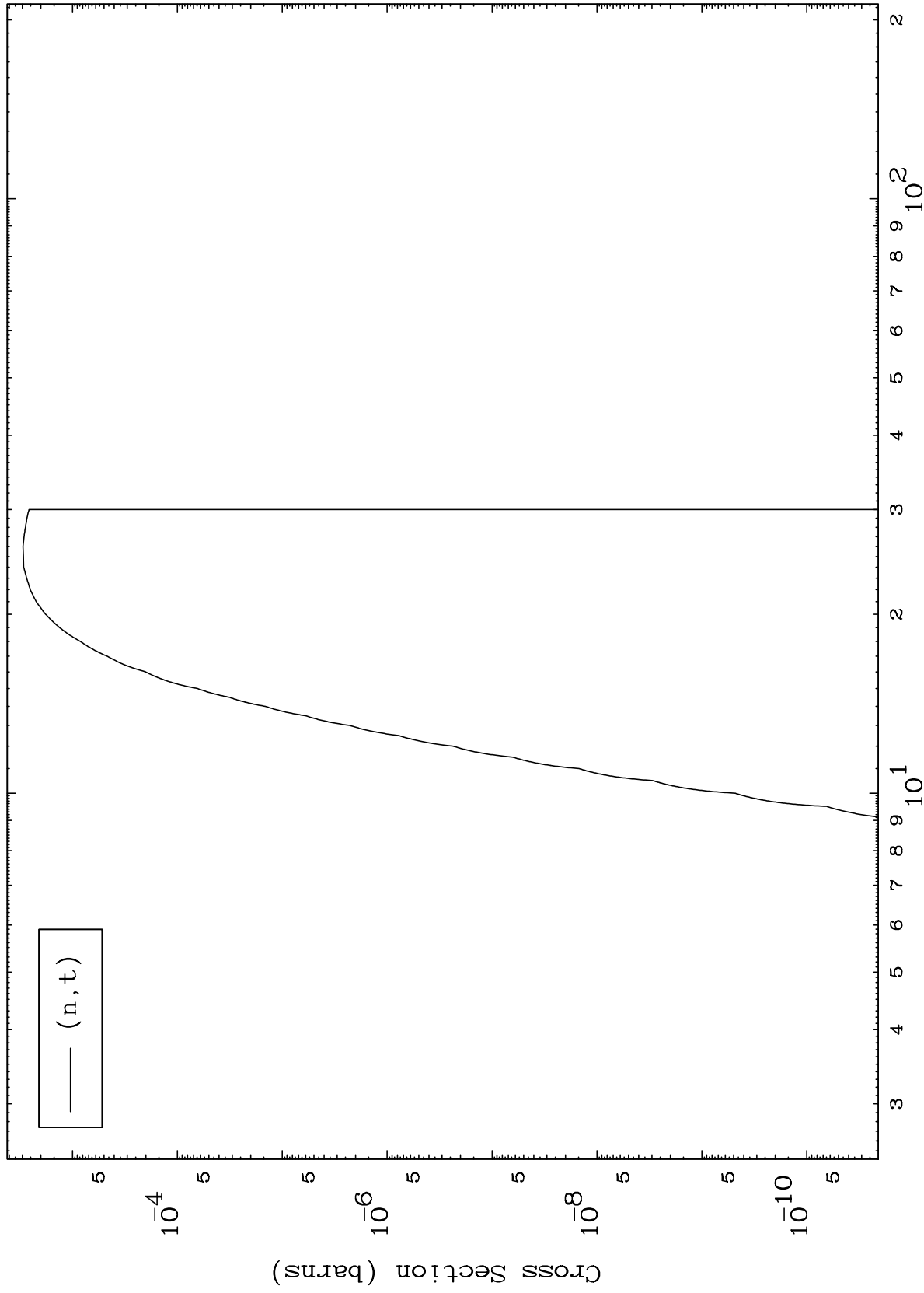
Incident Energy (MeV)

97-Bk-249

MAT 9752

(n,t) Levels
293 Kelvin Cross Sections

97-Bk-249



13

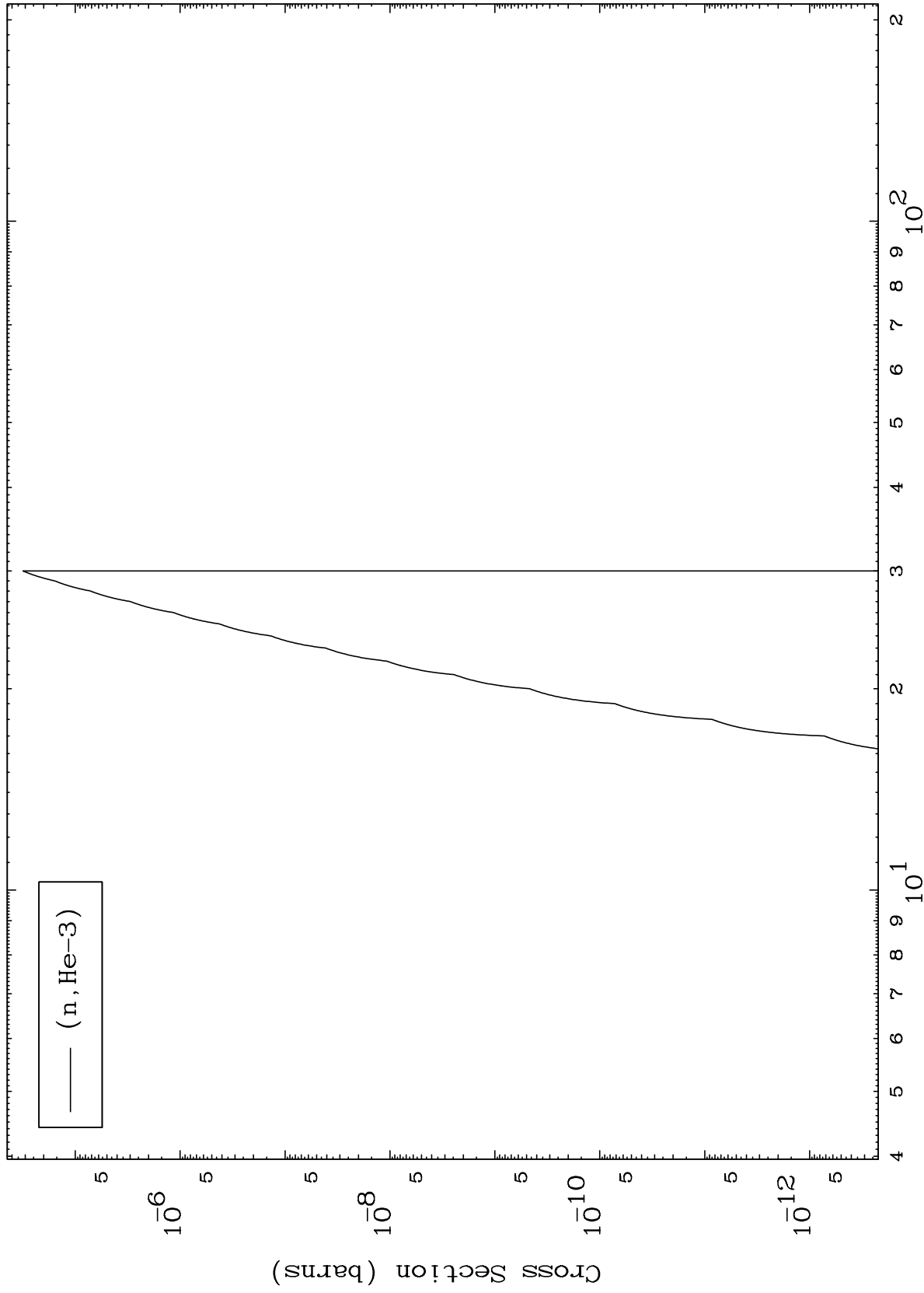
Incident Energy (MeV)

97-Bk-249

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97-Bk-249

(n,He3) Levels
293 Kelvin Cross Sections



14

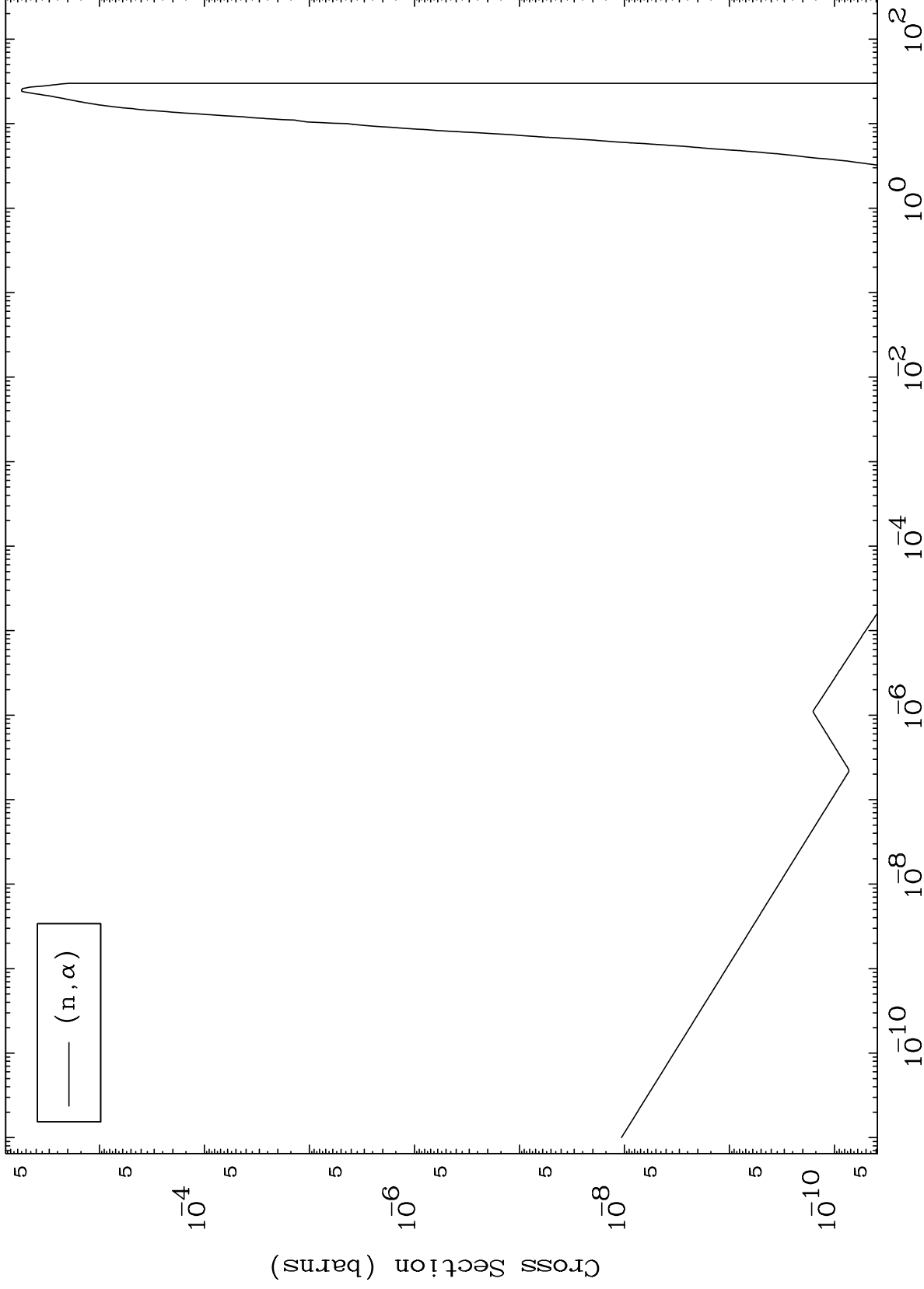
Incident Energy (MeV)

97-Bk-249

MAT 9752

(n, α) Levels
293 Kelvin Cross Sections

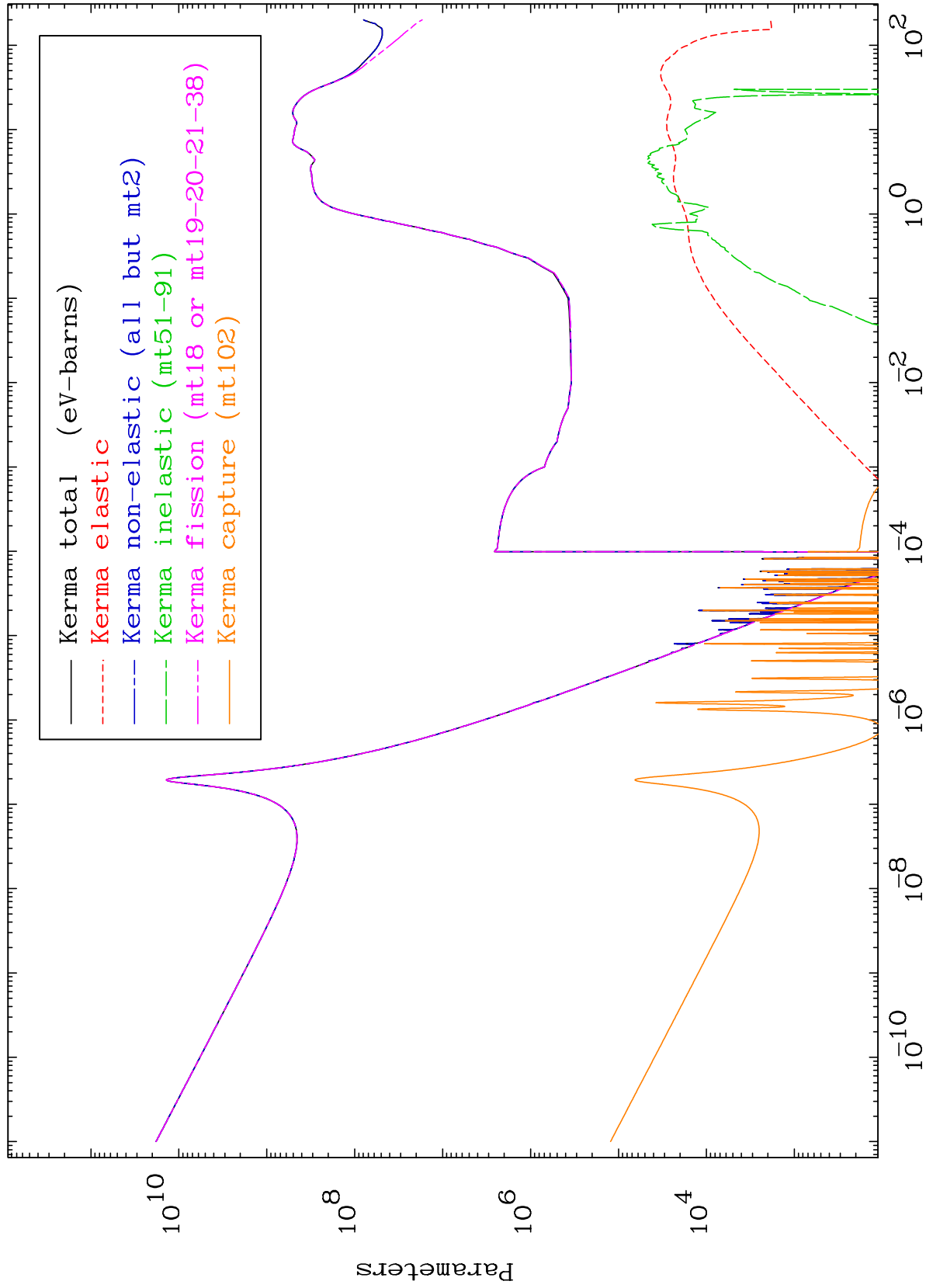
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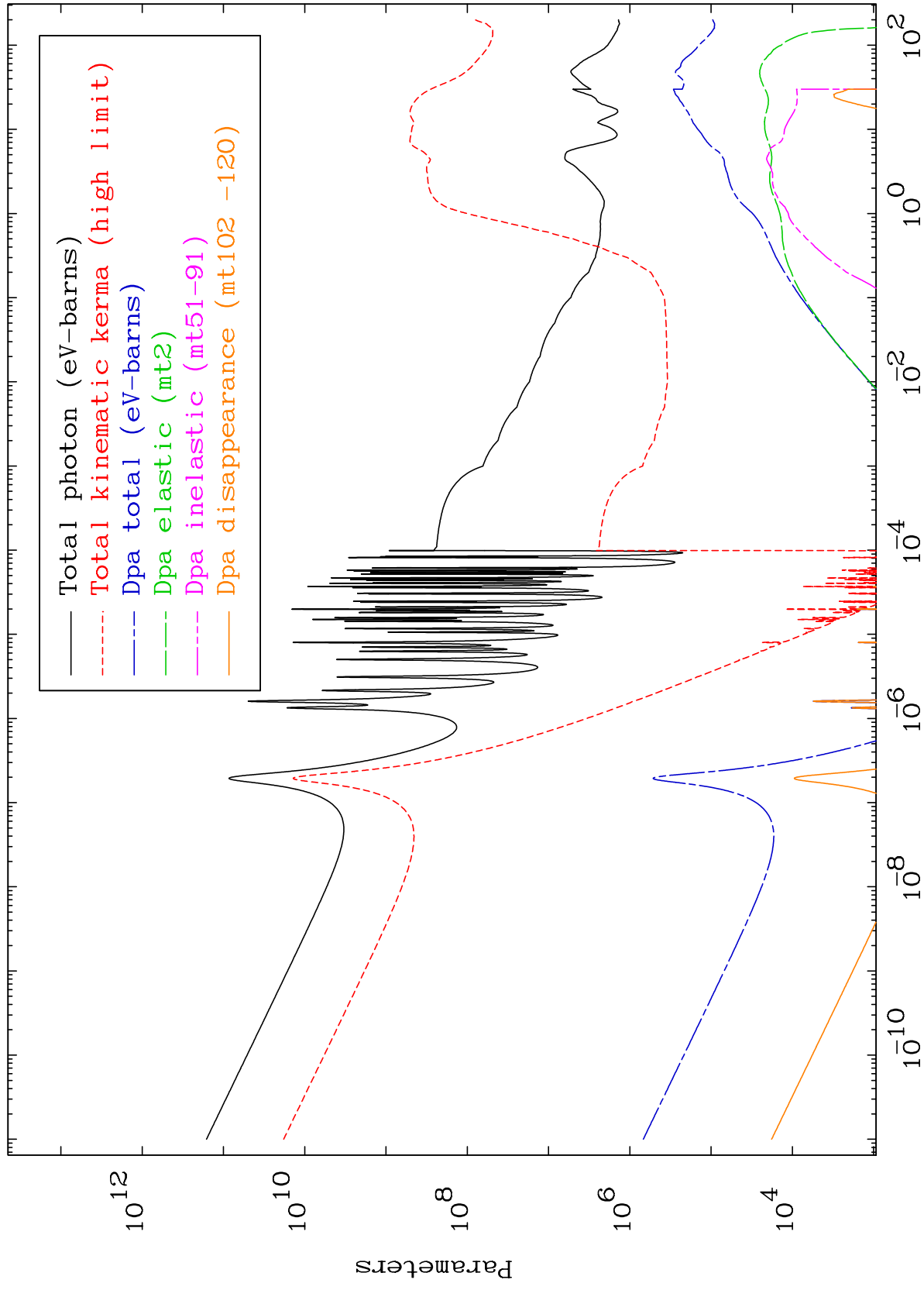


15

Incident Energy (MeV)

97-Bk-249

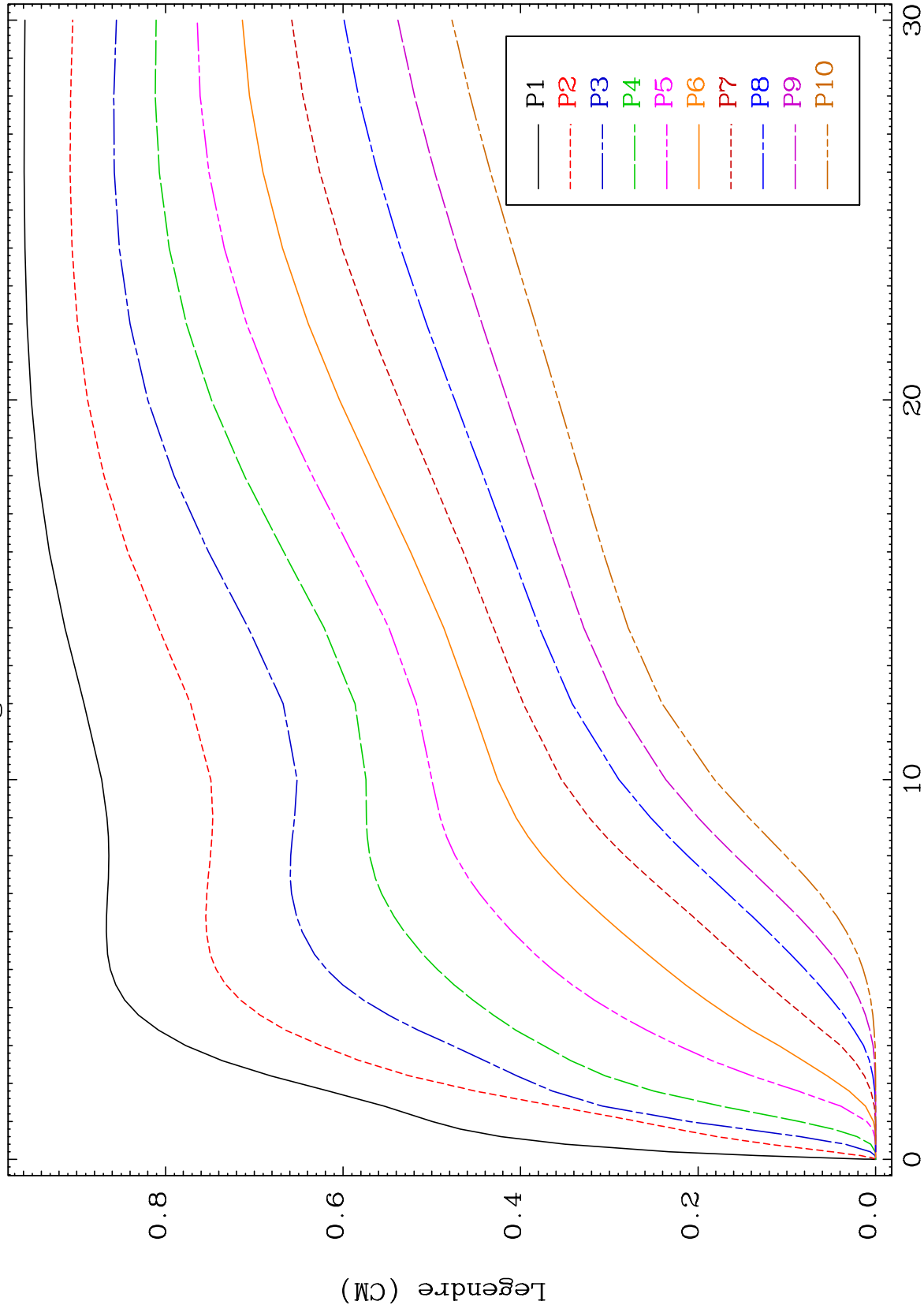




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97-Bk-249

Elastic Legendre Coefficients



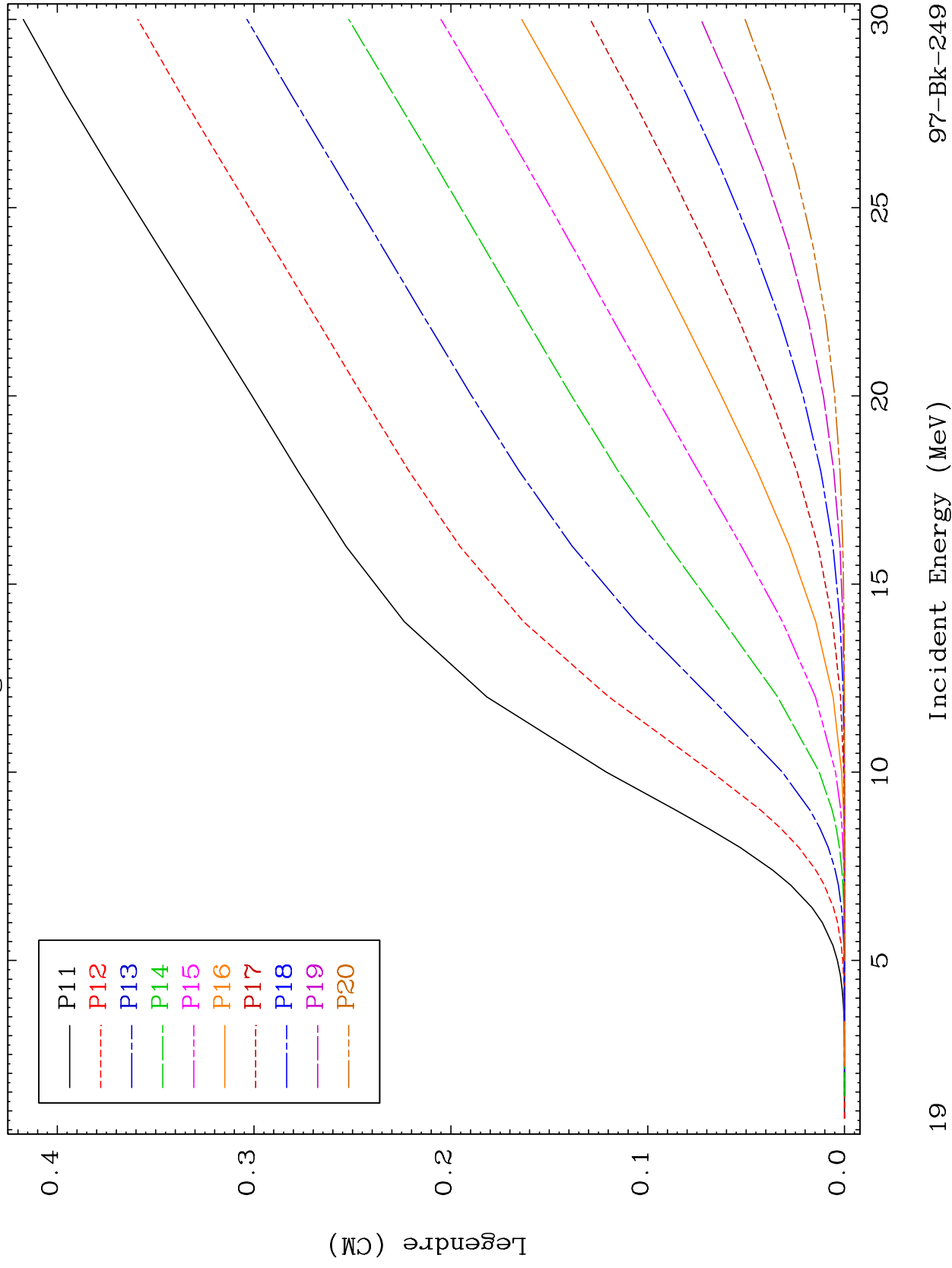
18

Incident Energy (MeV)

97-Bk-249

97-Bk-249

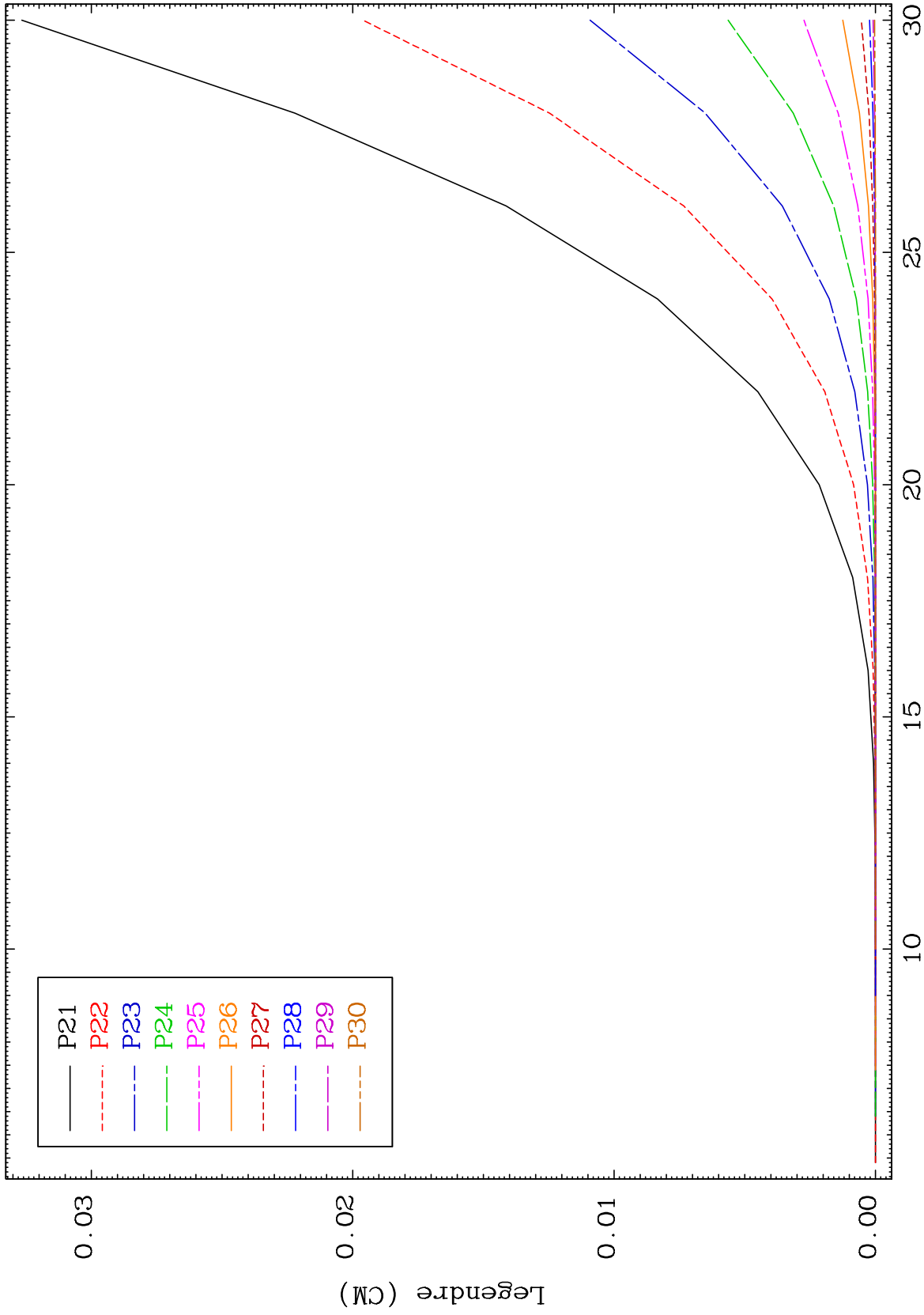
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Elastic
Legendre Coefficients

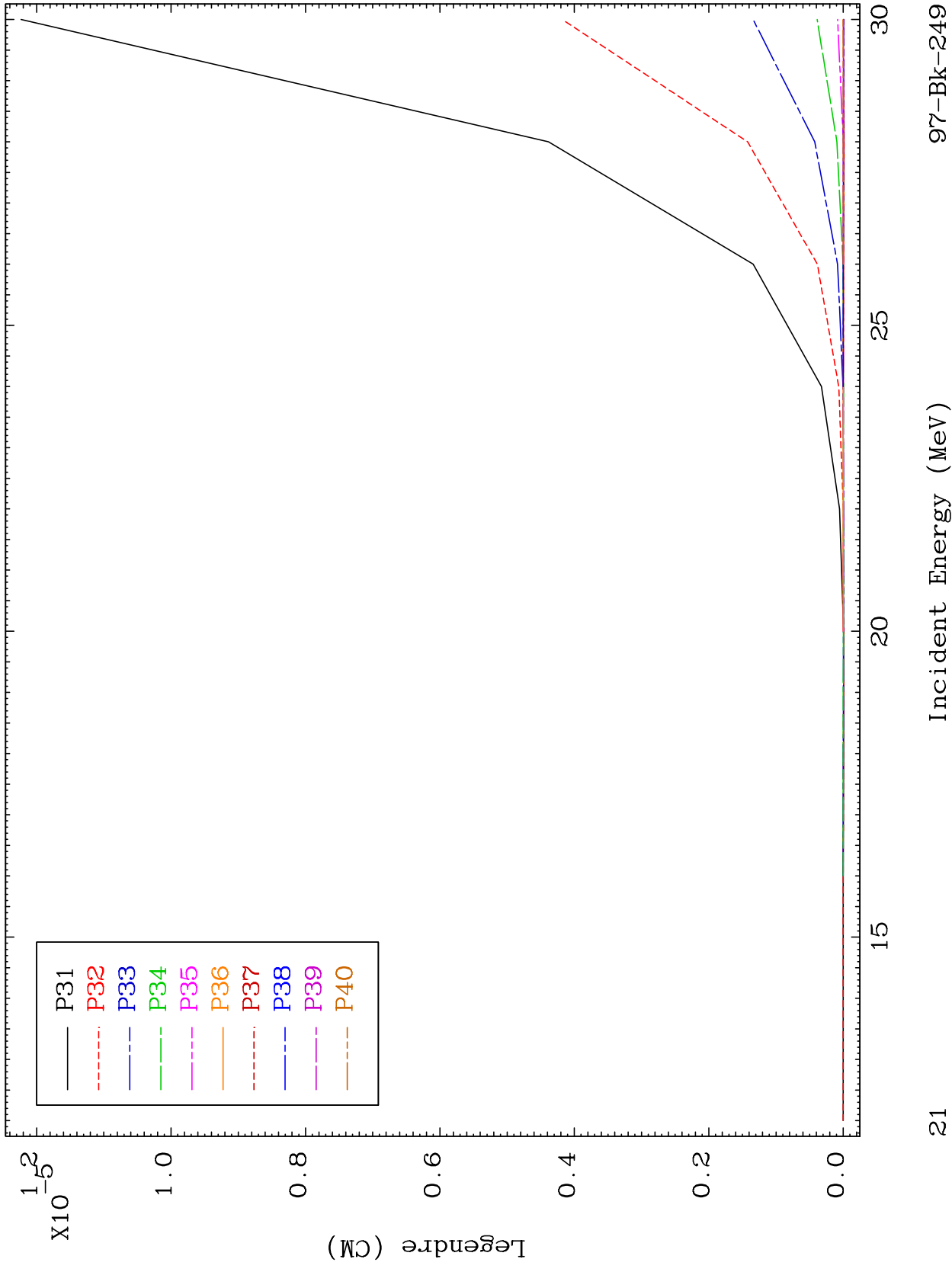
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20

Incident Energy (MeV)

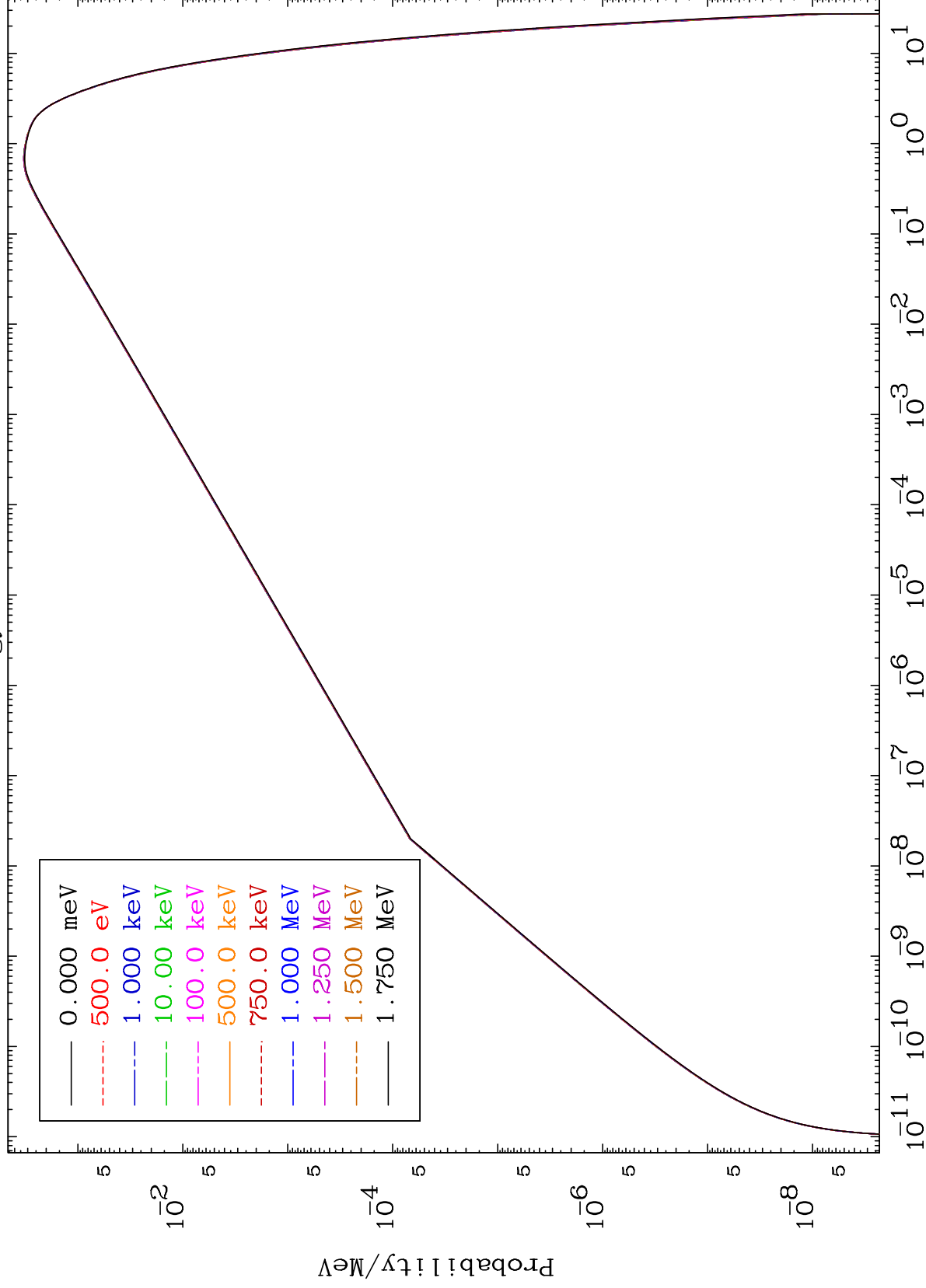
97-Bk-249



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Fission
Energy Distribution

97-Bk-249



22

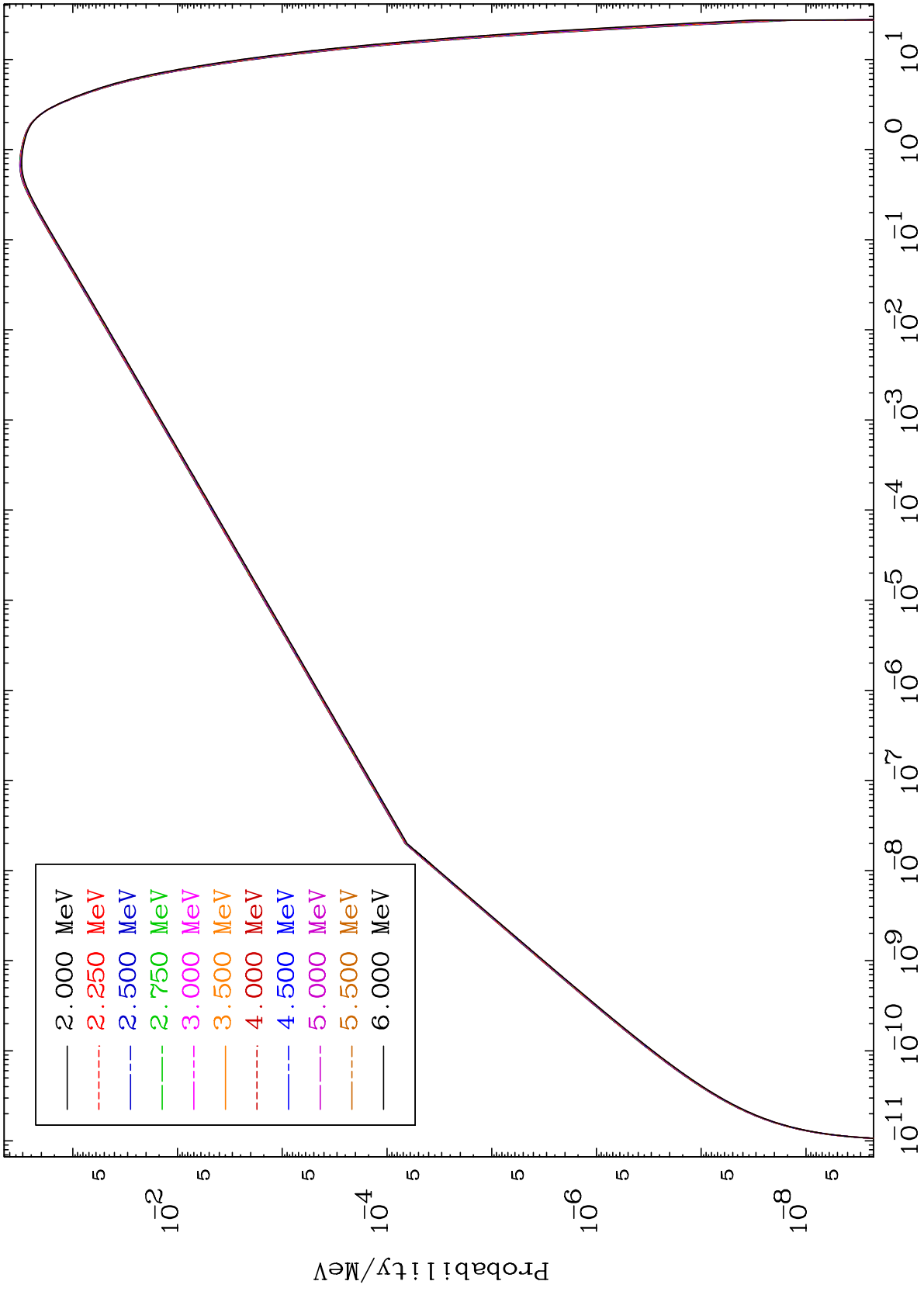
Secondary Energy (MeV)

97-Bk-249

MAT 9752

Fission
Energy Distribution

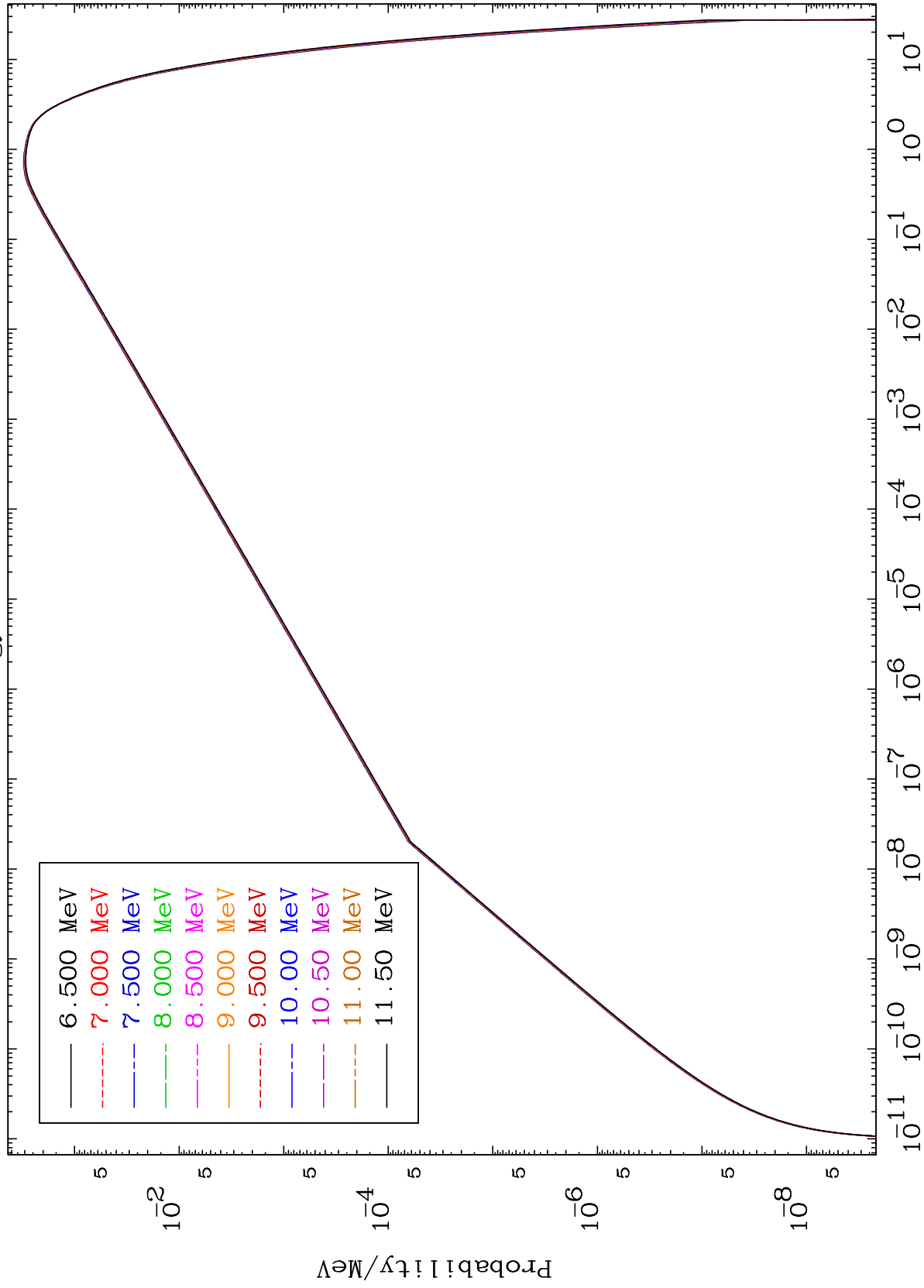
97-Bk-249



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Fission
Energy Distribution

97-Bk-249



24

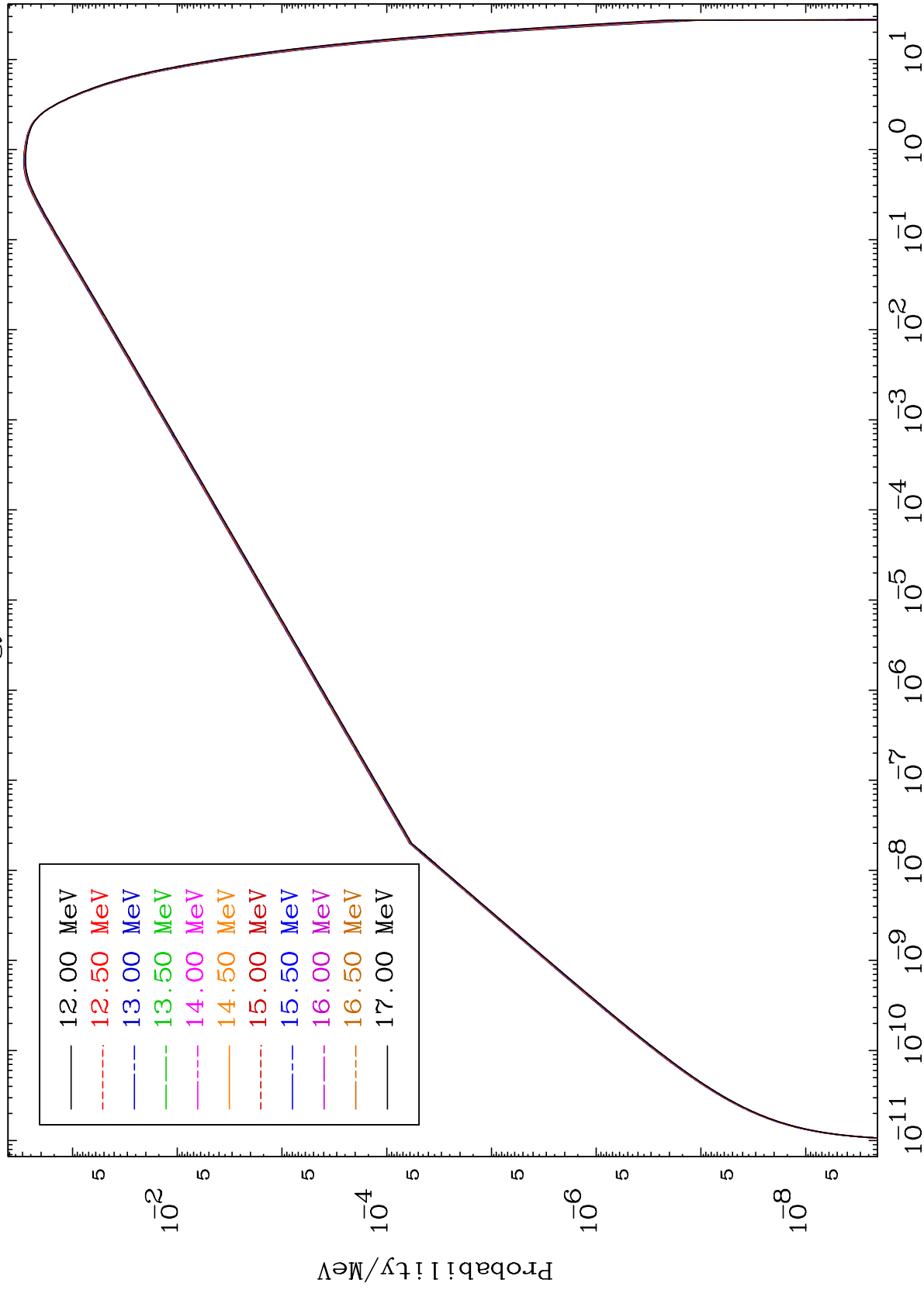
Secondary Energy (MeV)

97-Bk-249

MAT 9752

Fission
Energy Distribution

97-Bk-249



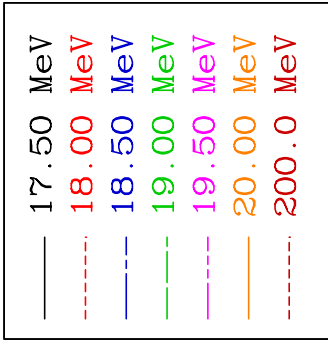
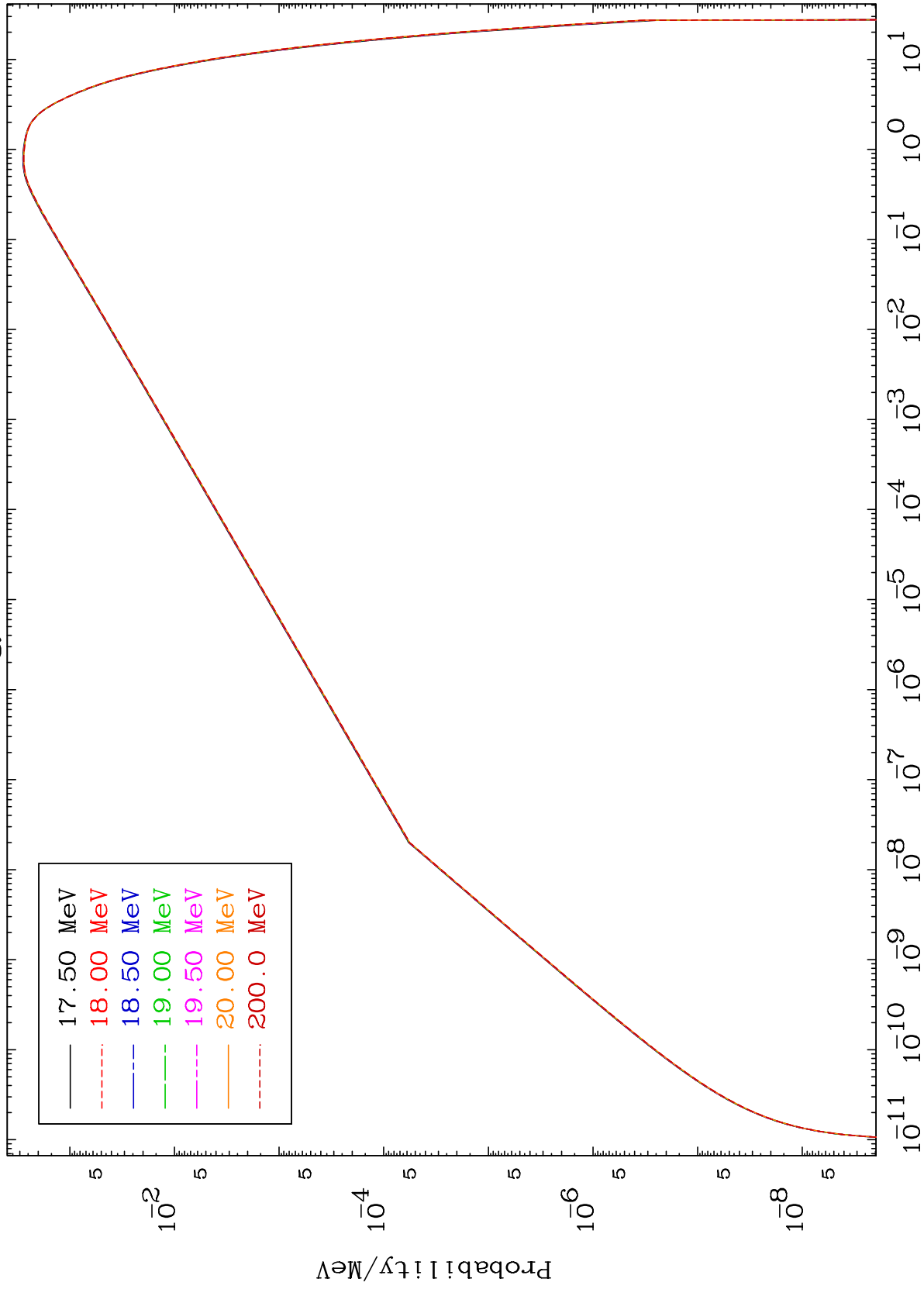
25

97-Bk-249

MAT 9752

Fission
Energy Distribution

97-Bk-249



26

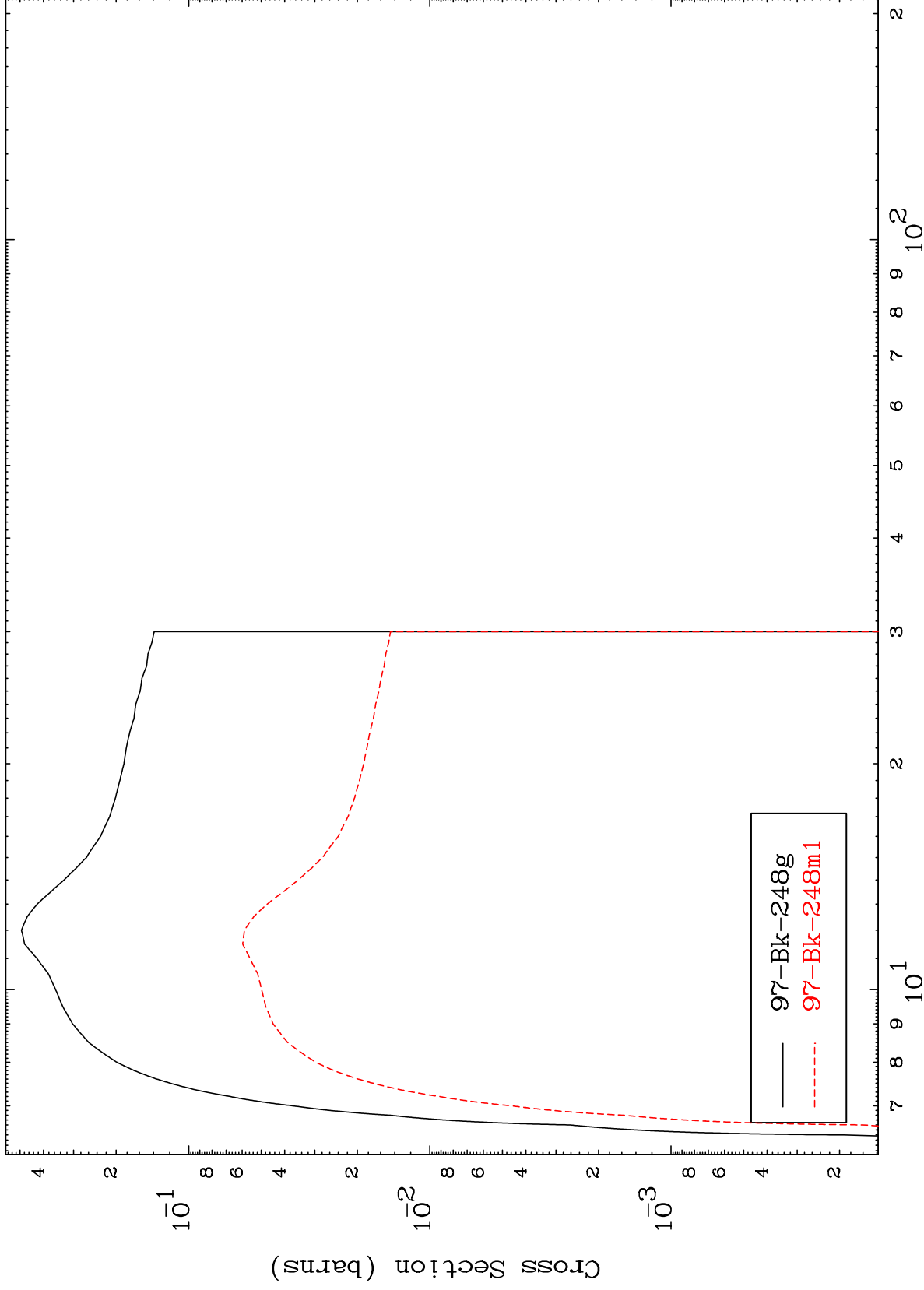
Secondary Energy (MeV)

97-Bk-249

MAT 9752

97-Bk-249

(n,2n)
Radionuclide Production Cross Section



27

Incident Energy (MeV)

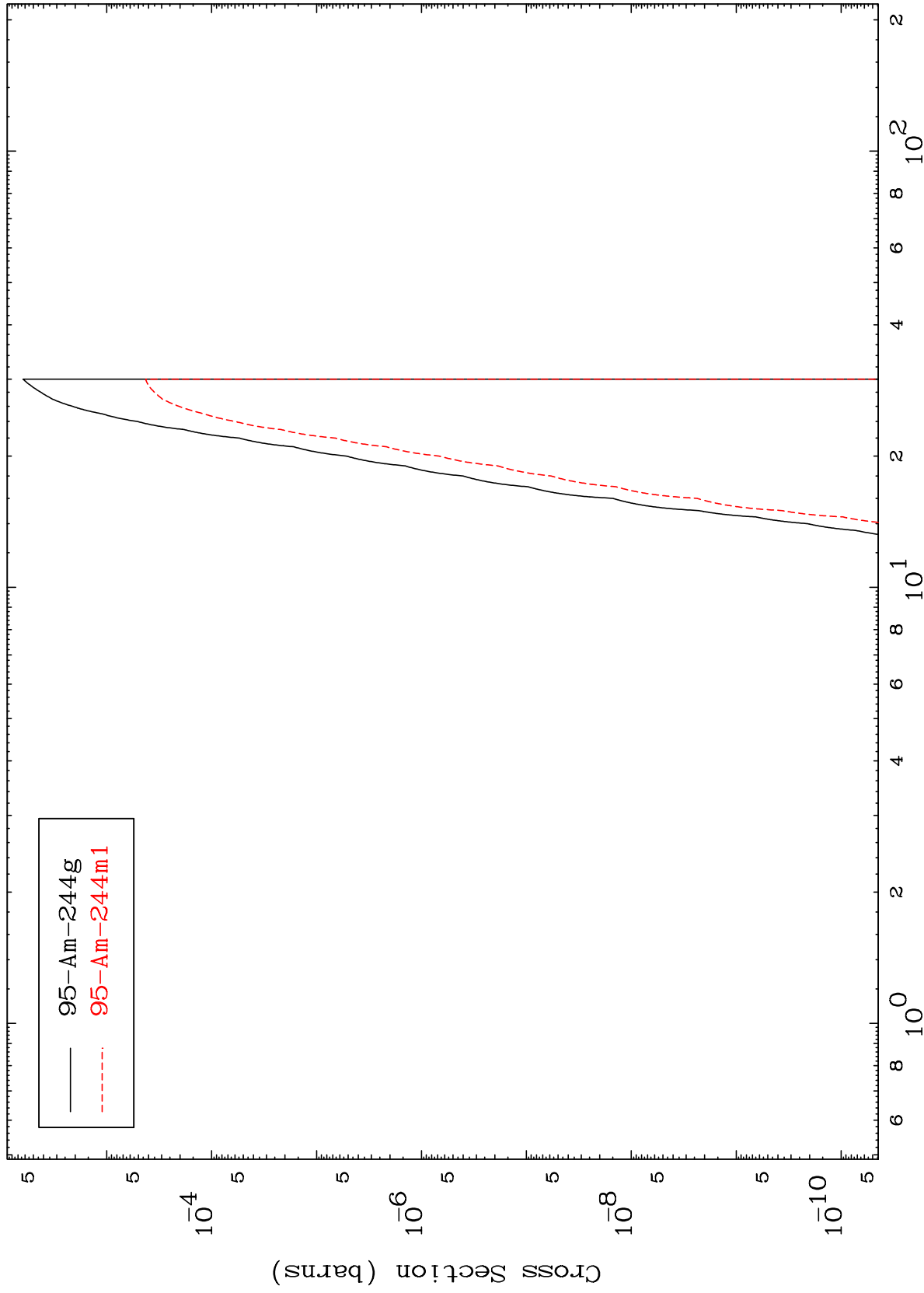
97-Bk-249

MAT 9752

$(n,2n) \alpha$

97-Bk-249

Radionuclide Production Cross Section



28

Incident Energy (MeV)

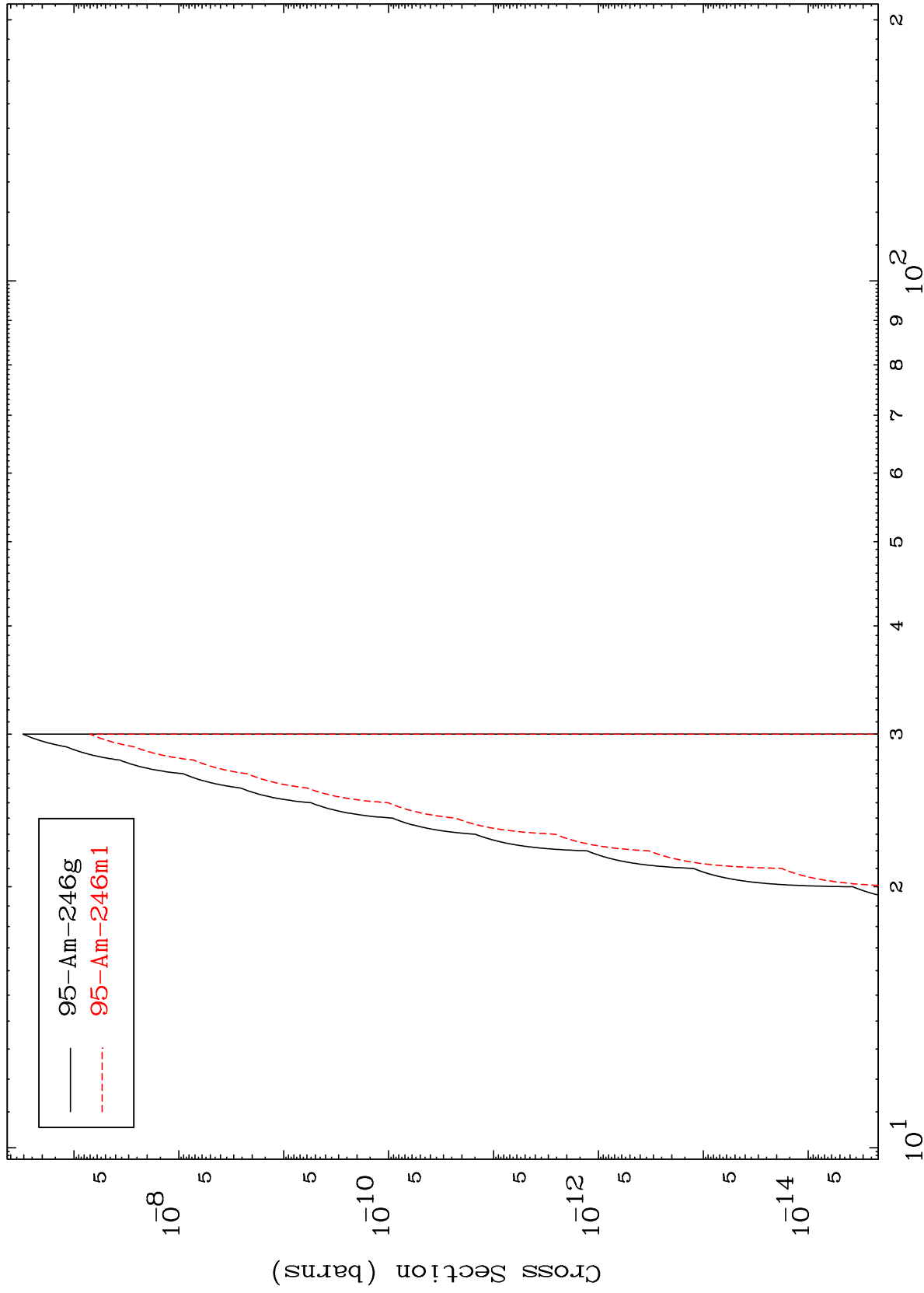
97-Bk-249

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(n,n') He-3

97-Bk-249

Radionuclide Production Cross Section



29

Incident Energy (MeV)

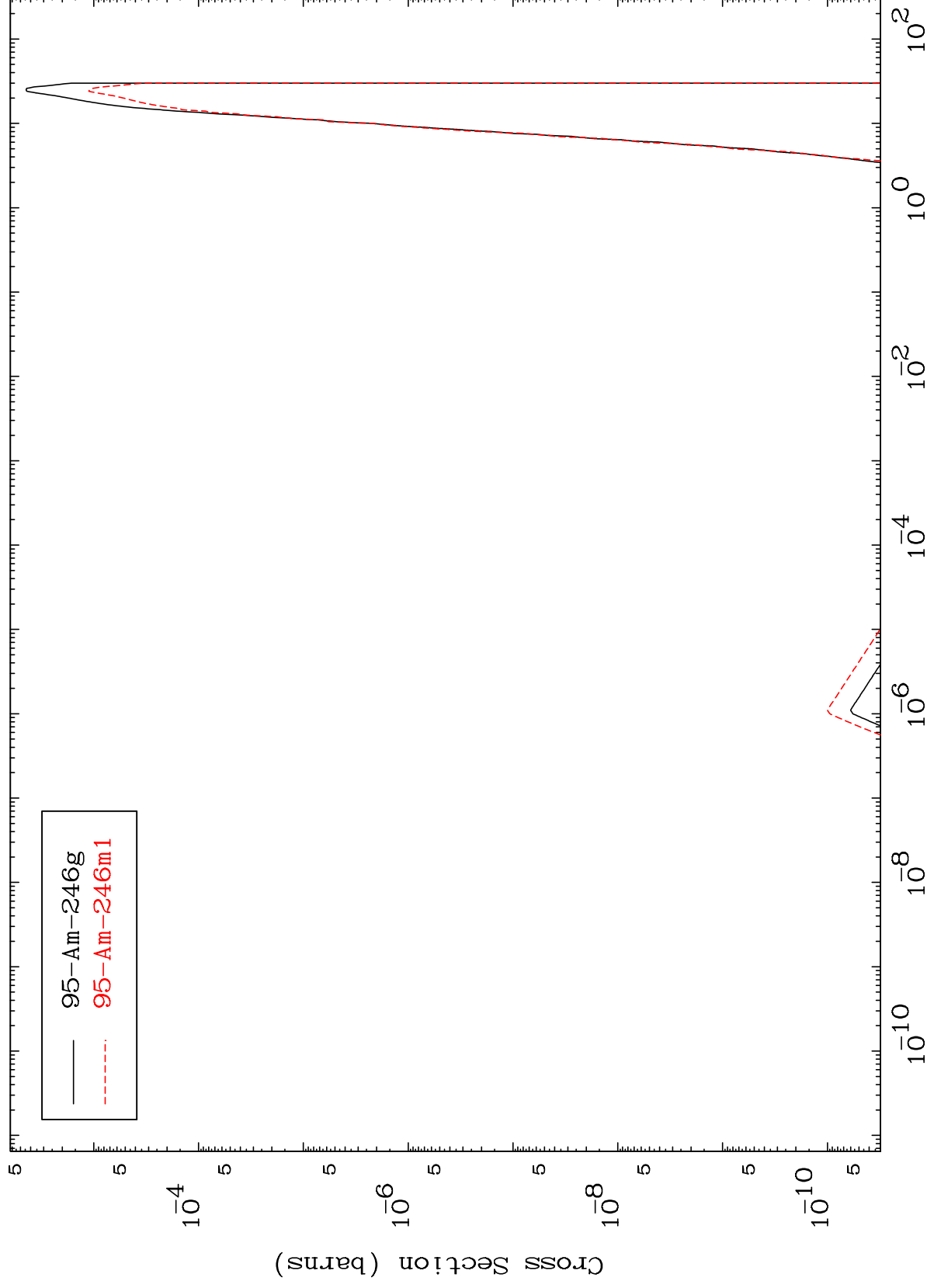
97-Bk-249

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(n, α)

97-Bk-249

Radionuclide Production Cross Section



30

Incident Energy (MeV)

97-Bk-249