

Program EVALPLOT
(Version 2018-1)

by

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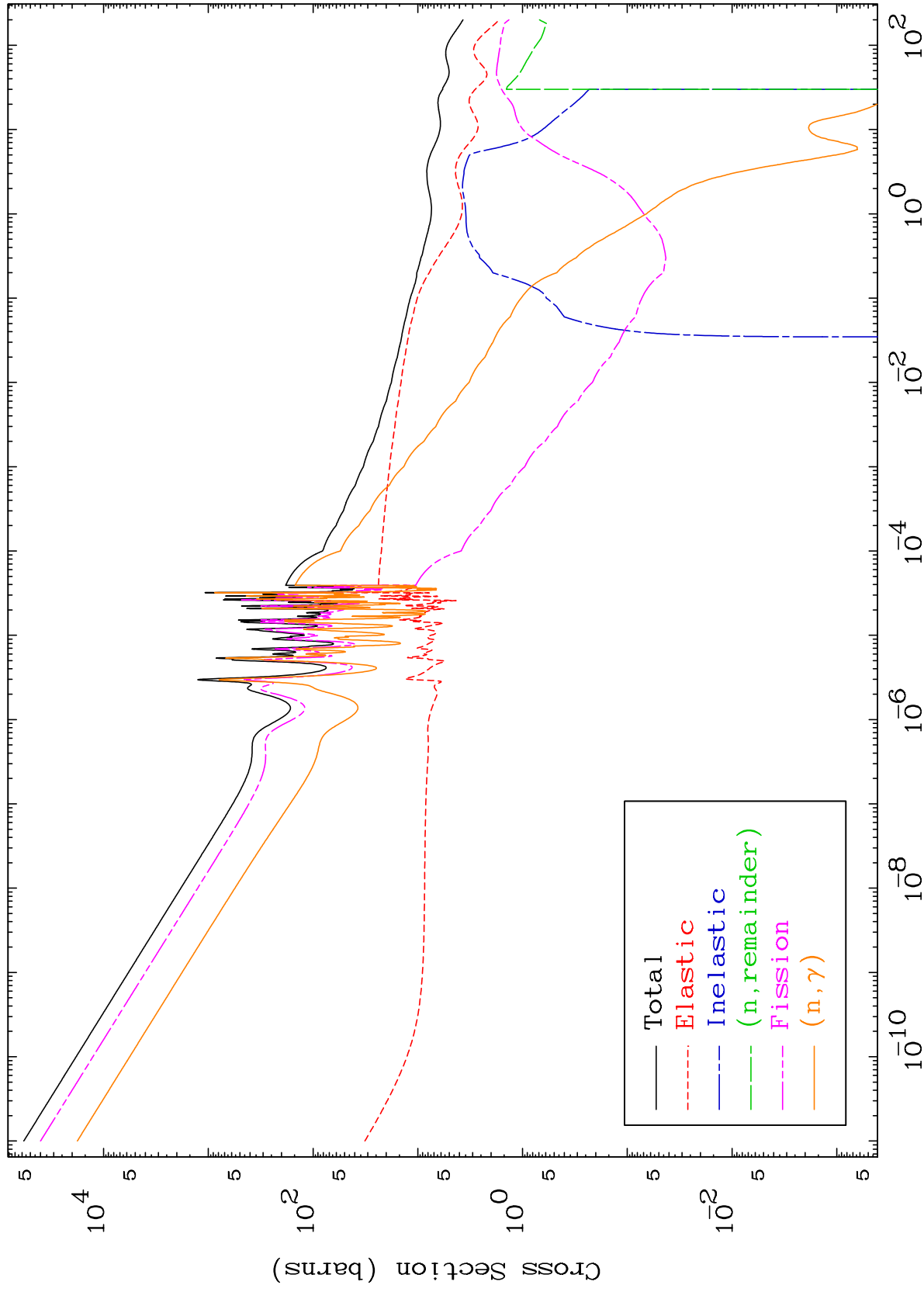
Press Mouse Button to Start

MAT 9755

Major

293 Kelvin Cross Sections

97-Bk-250



1

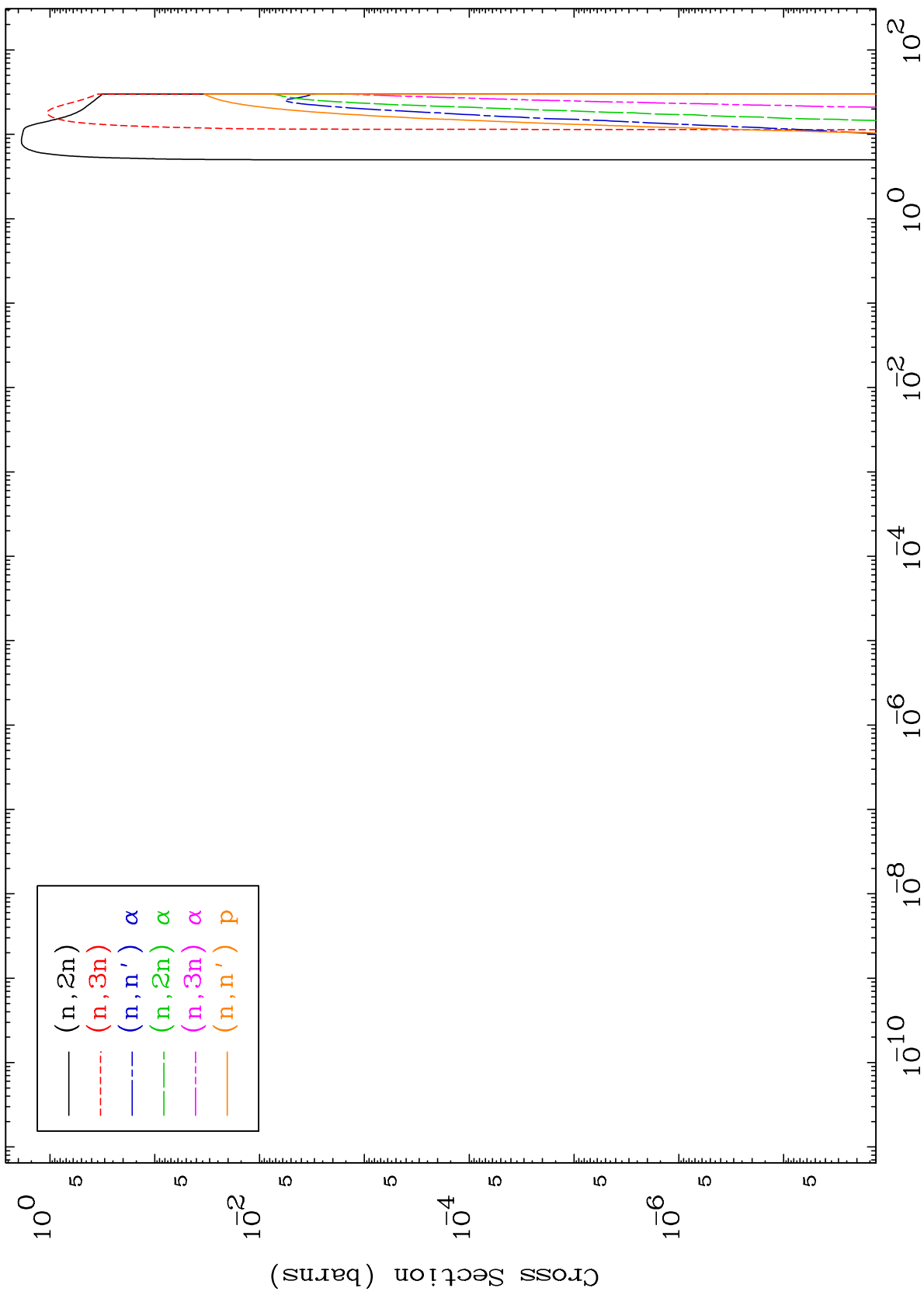
Incident Energy (MeV)

97-Bk-250

MAT 9755

Neutron Production
293 Kelvin Cross Sections

97-Bk-250



2

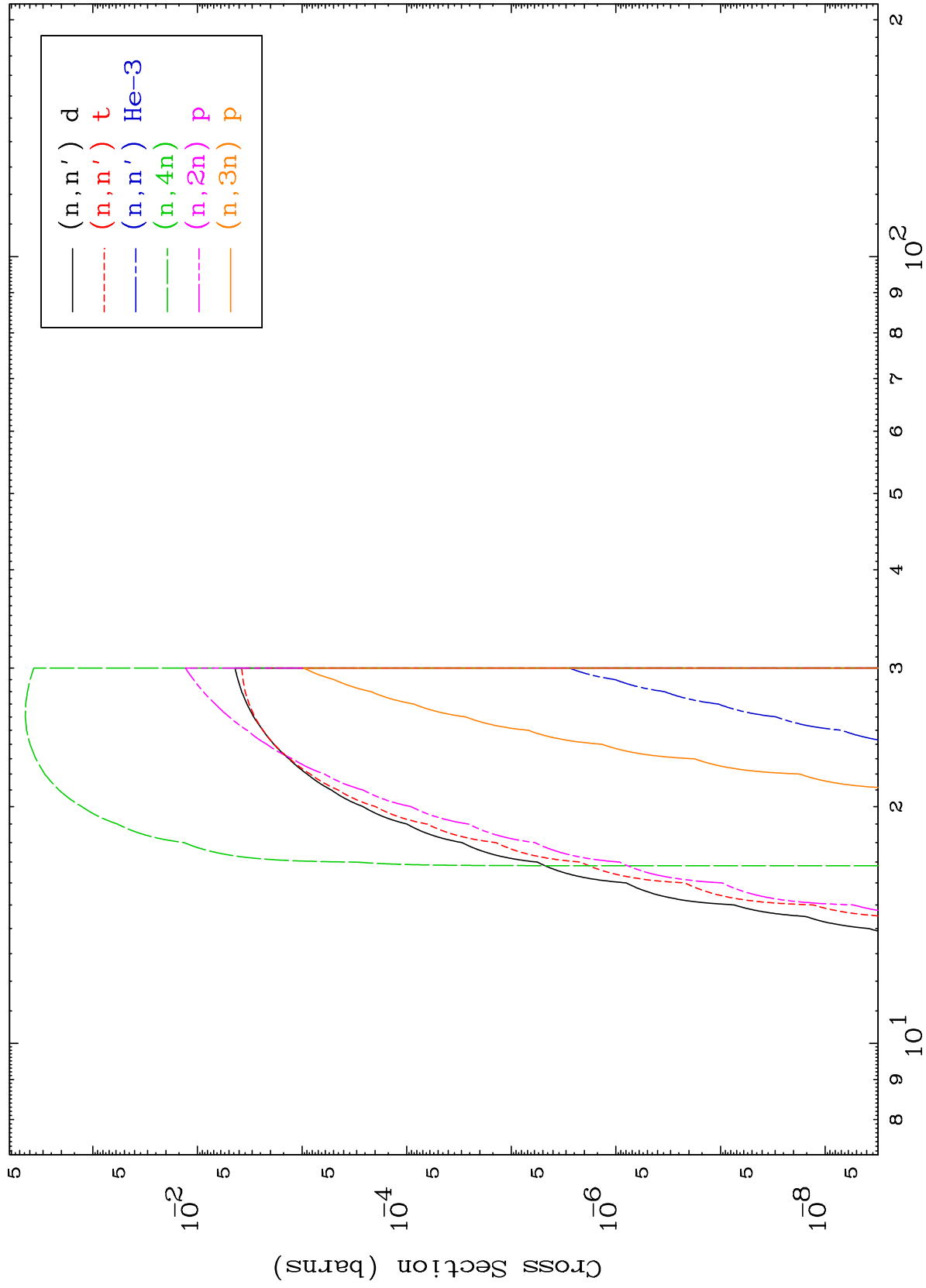
Incident Energy (MeV)

97-Bk-250

MAT 9755

Neutron Production
293 Kelvin Cross Sections

97-Bk-250



3

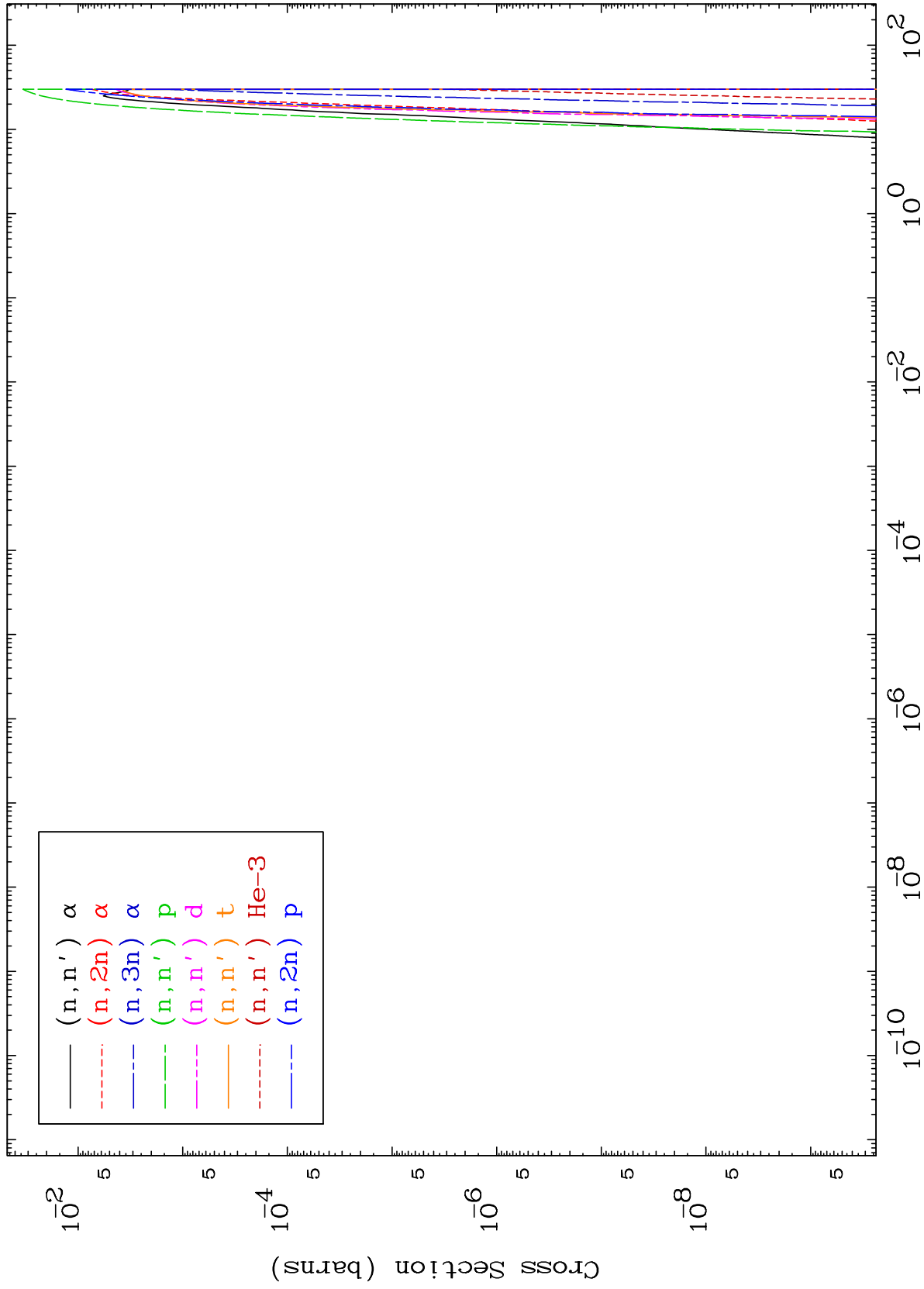
Incident Energy (MeV)

97-Bk-250

MAT 9755

Charged Particle
293 Kelvin Cross Sections

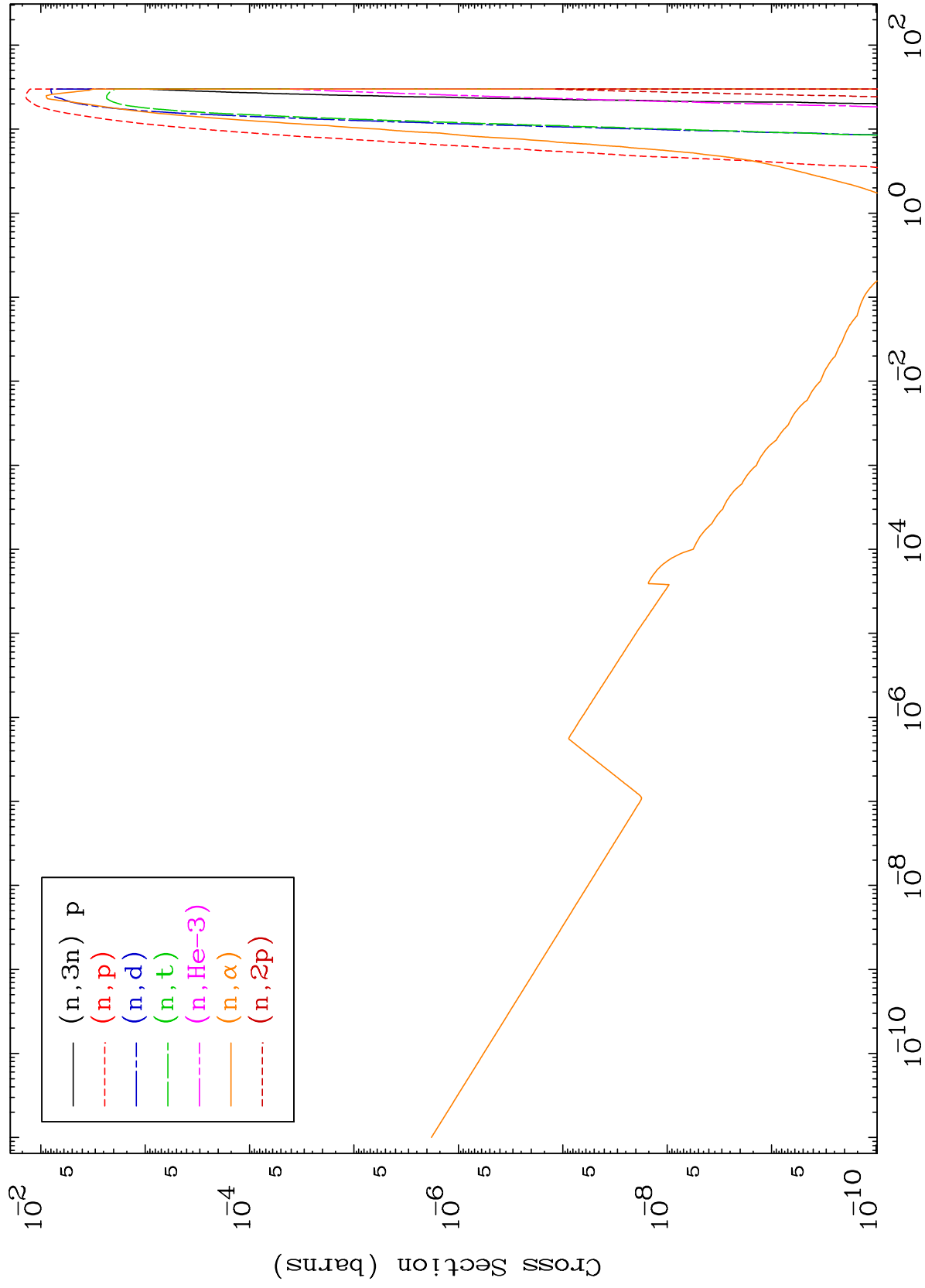
97-Bk-250



MAT 9755

Charged Particle
293 Kelvin Cross Sections

97-Bk-250



5

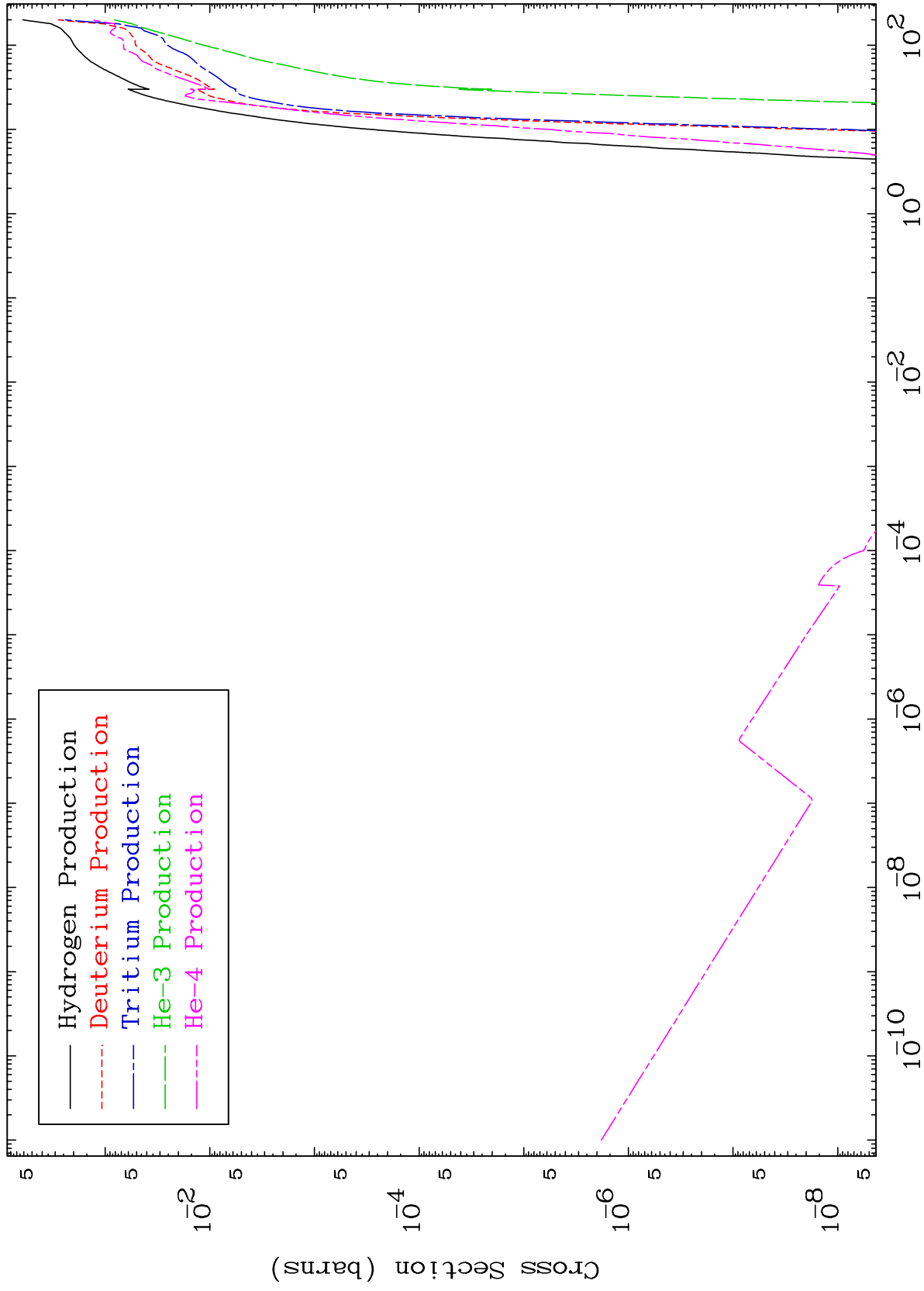
Incident Energy (MeV)

97-Bk-250

MAT 9755

Particle Production
293 Kelvin Cross Sections

97-Bk-250



6

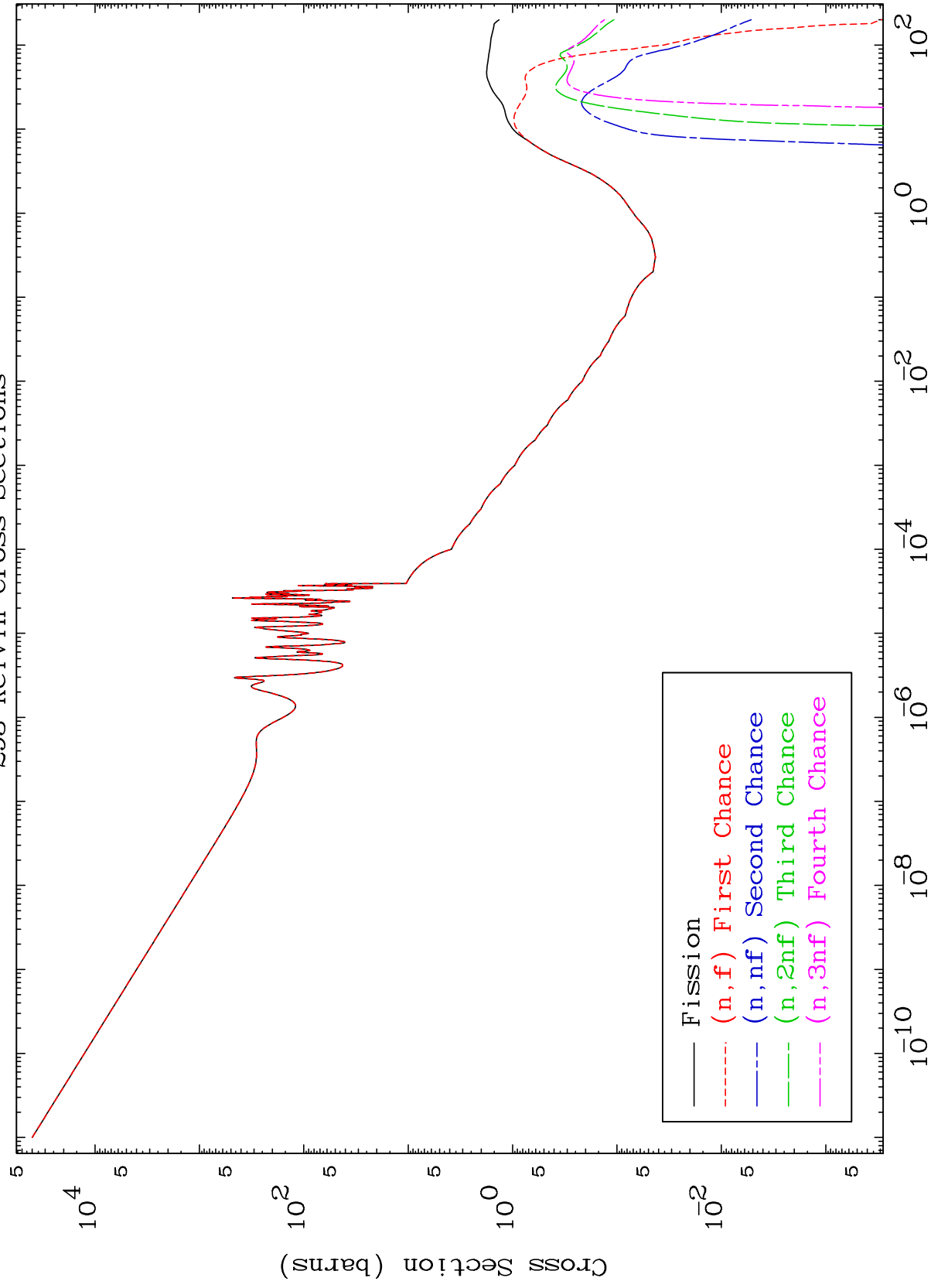
Incident Energy (MeV)

97-Bk-250

MAT 9755

Fission
293 Kelvin Cross Sections

97-Bk-250

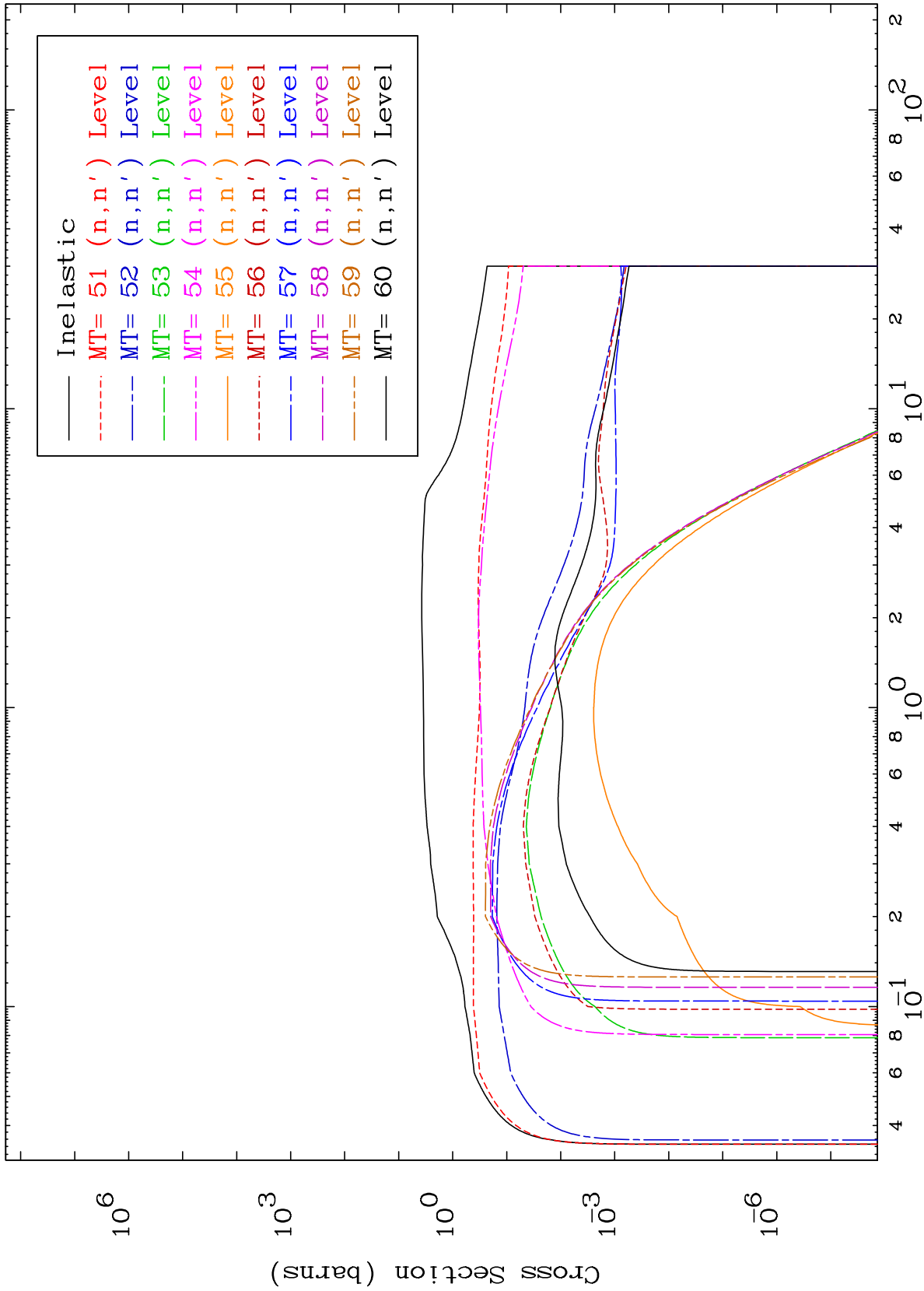


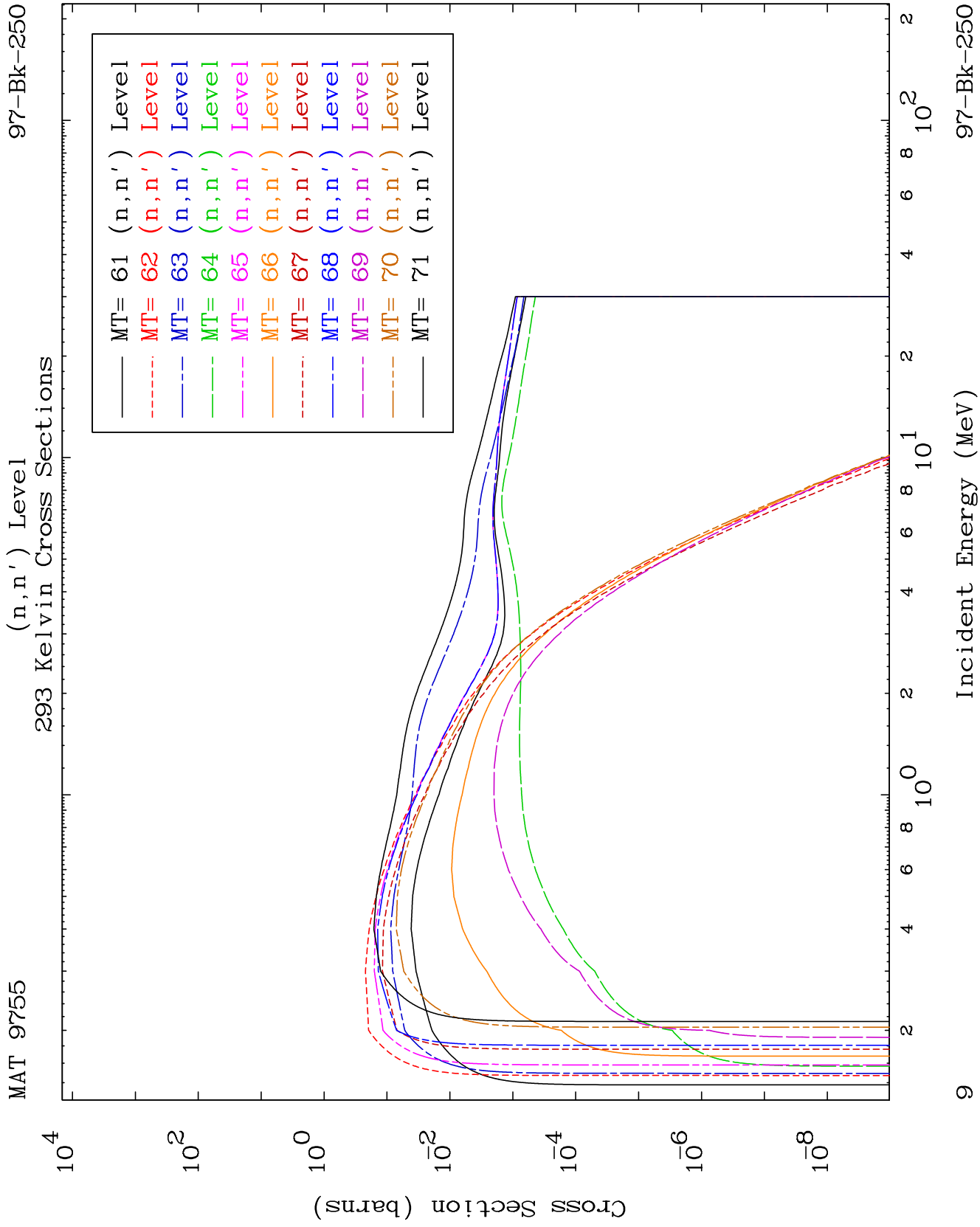
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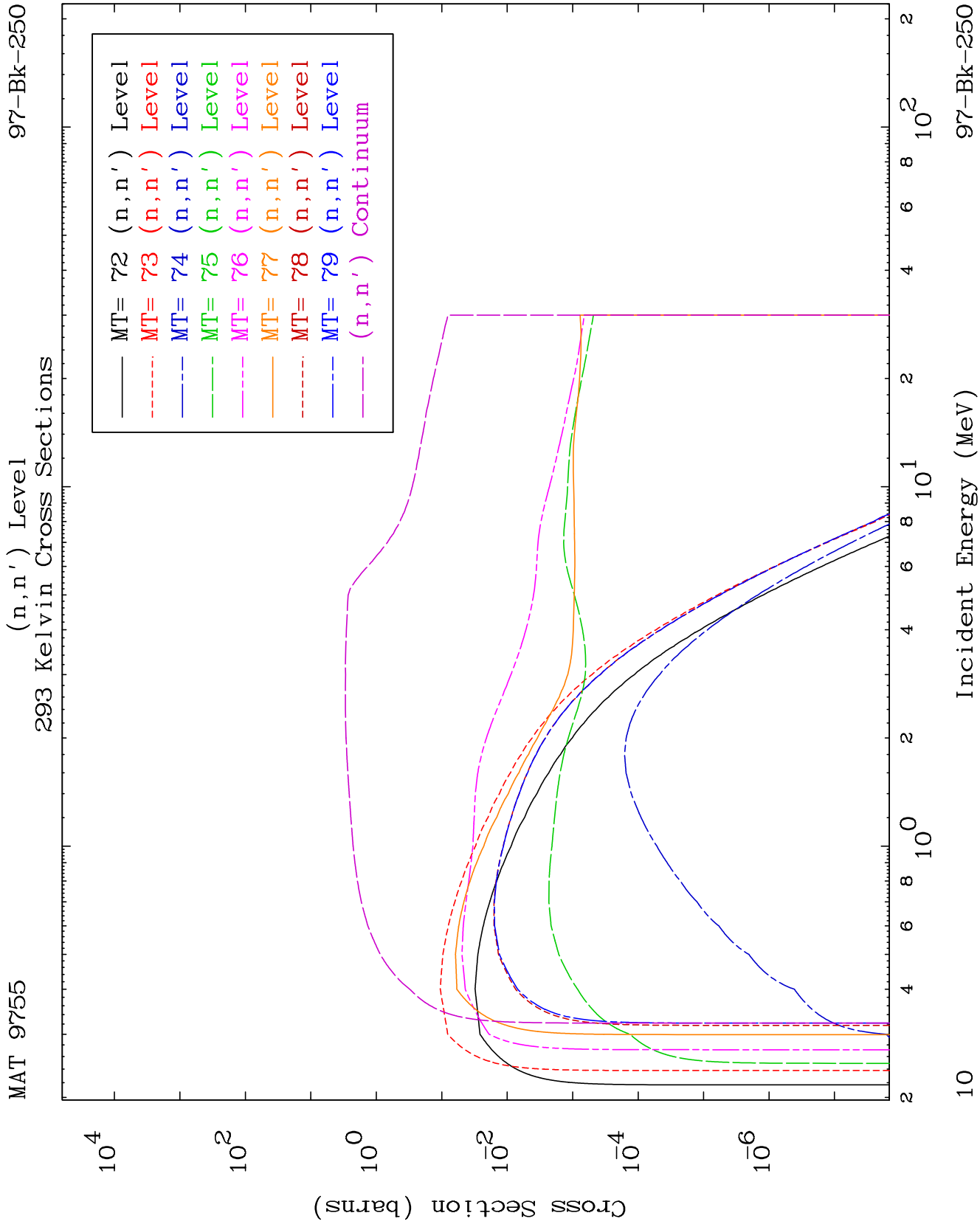
(n,n') Level

97-Bk-250

293 Kelvin Cross Sections



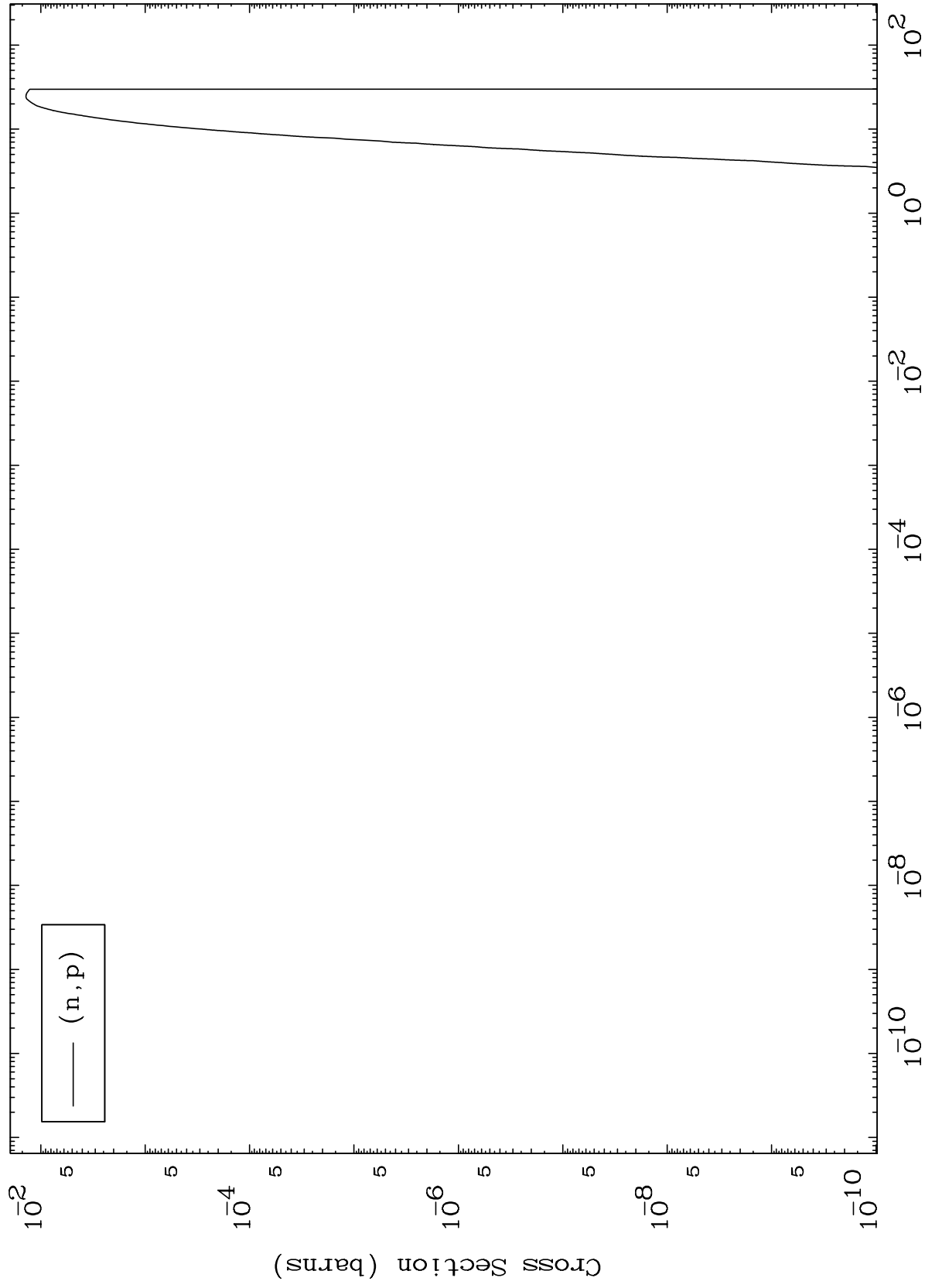




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(n,p) Levels
293 Kelvin Cross Sections

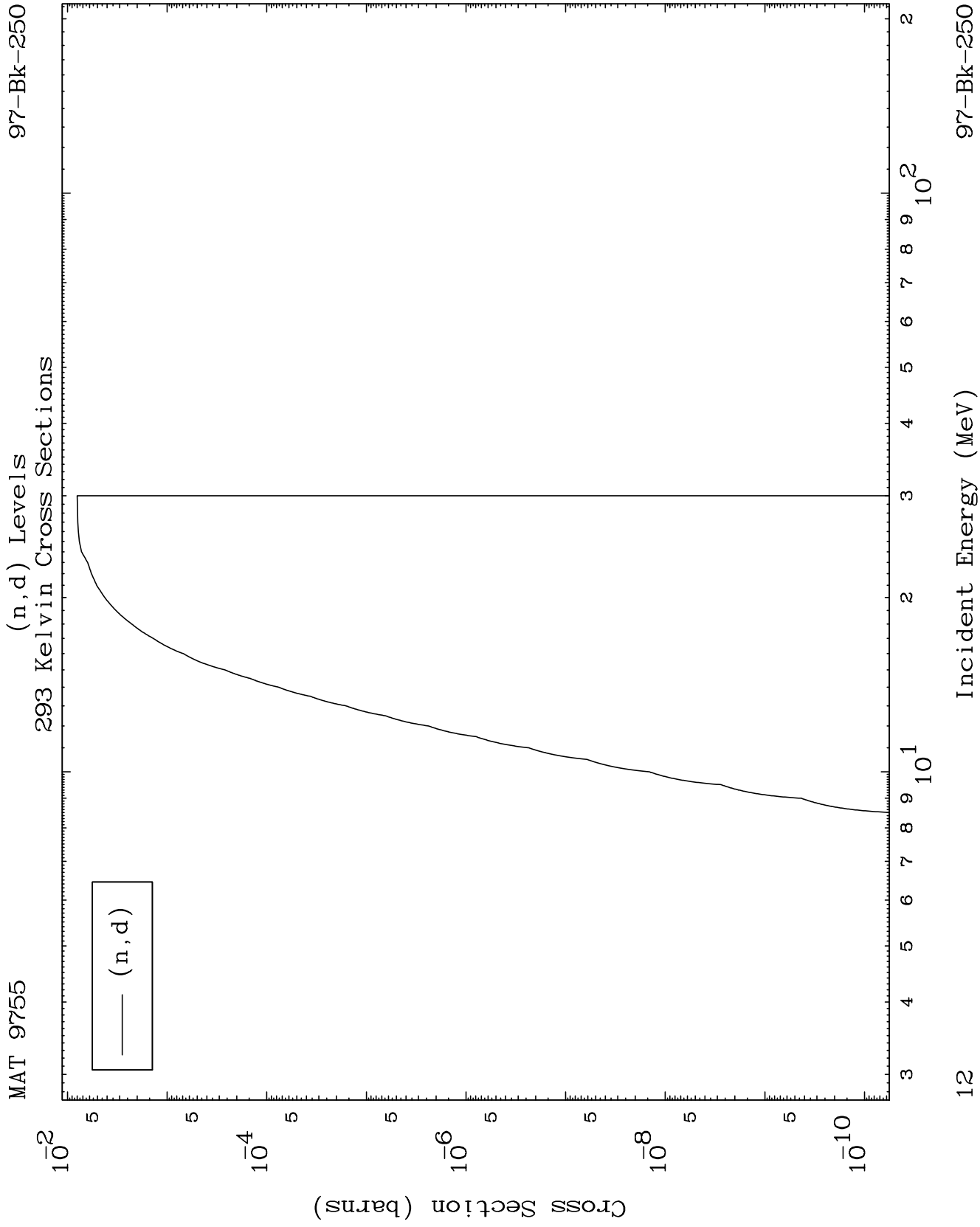
97-Bk-250



11

Incident Energy (MeV)

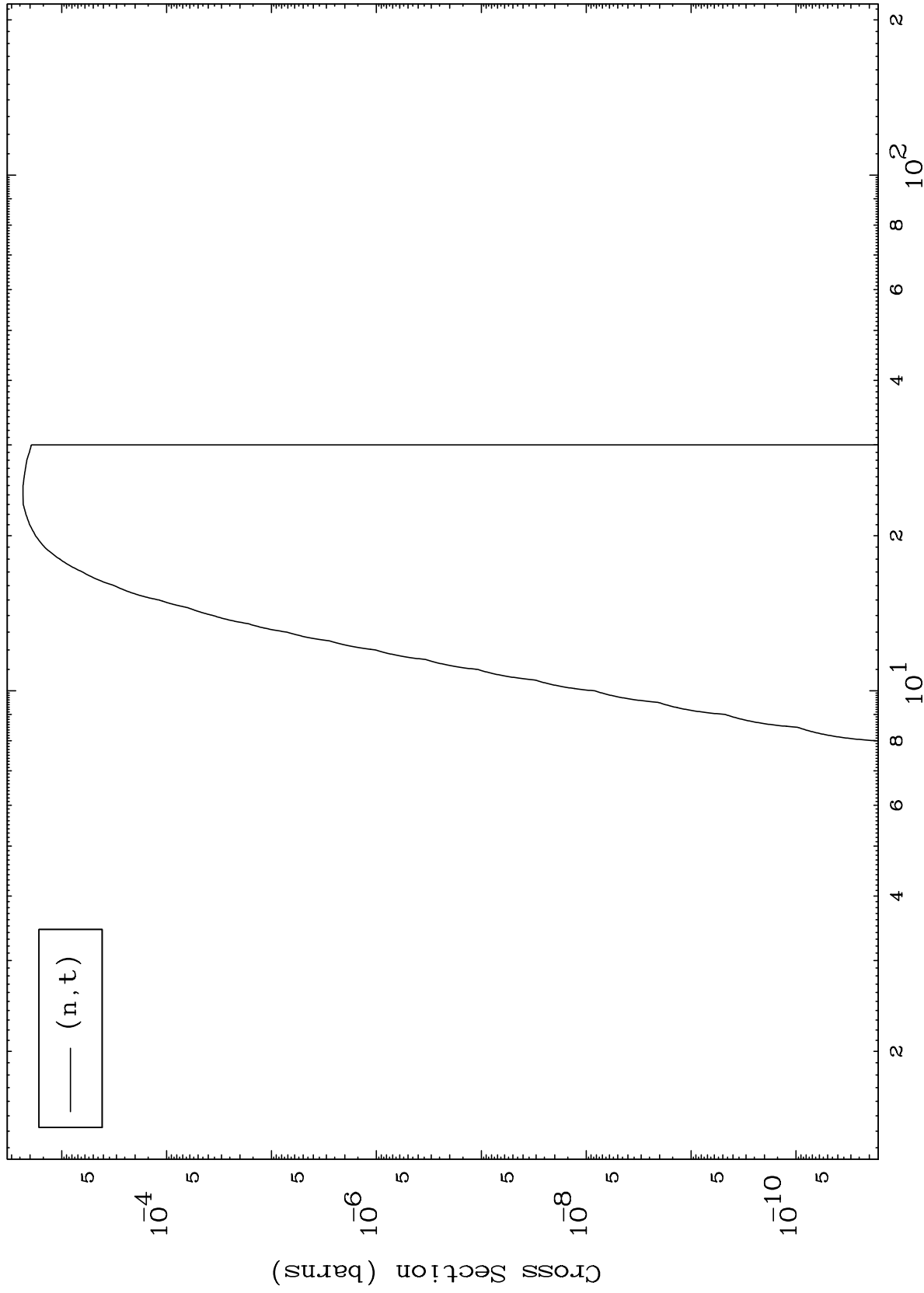
97-Bk-250



MAT 9755

97-Bk-250

(n,t) Levels
293 Kelvin Cross Sections



13

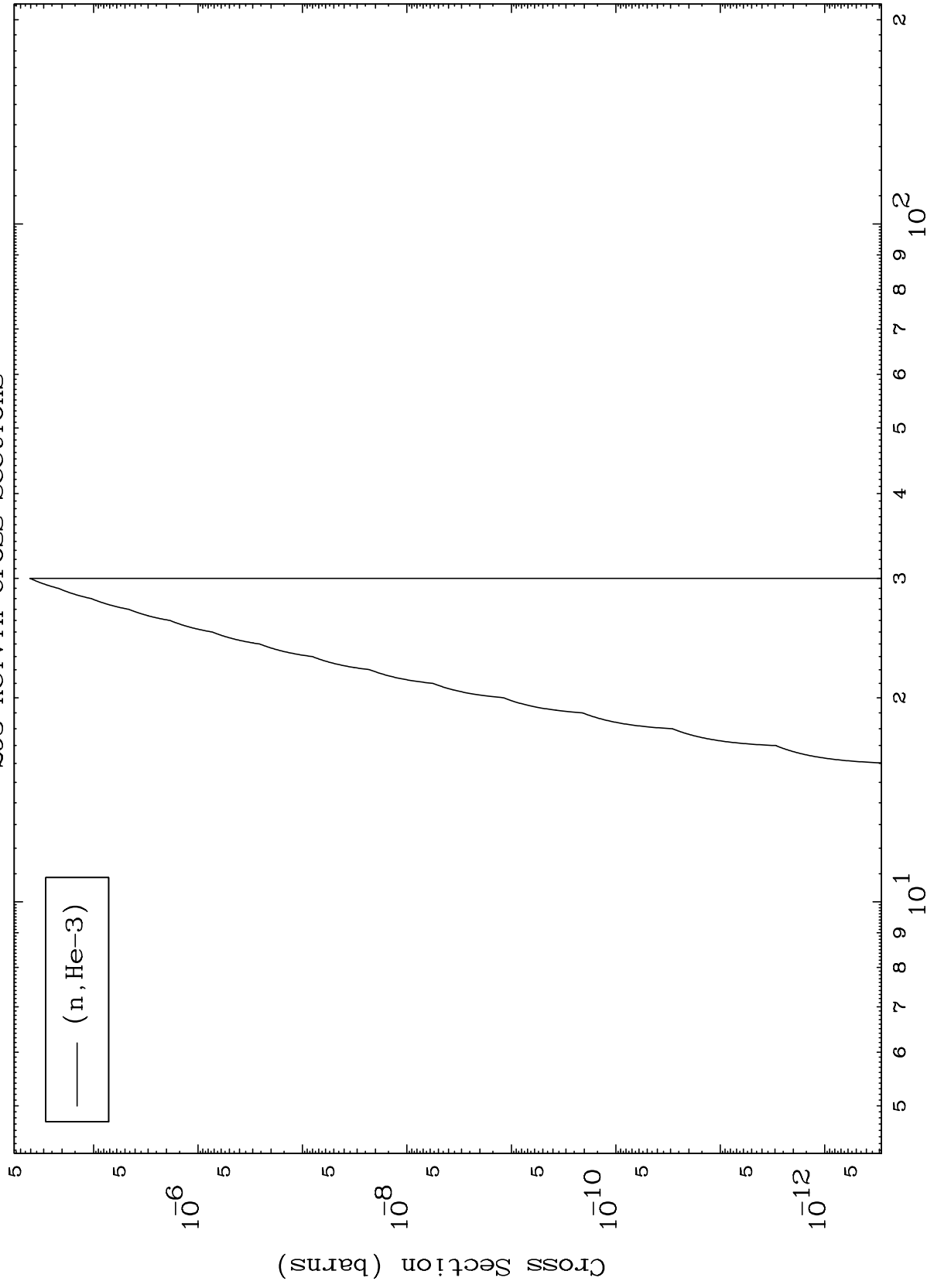
Incident Energy (MeV)

97-Bk-250

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(n,He3) Levels
293 Kelvin Cross Sections

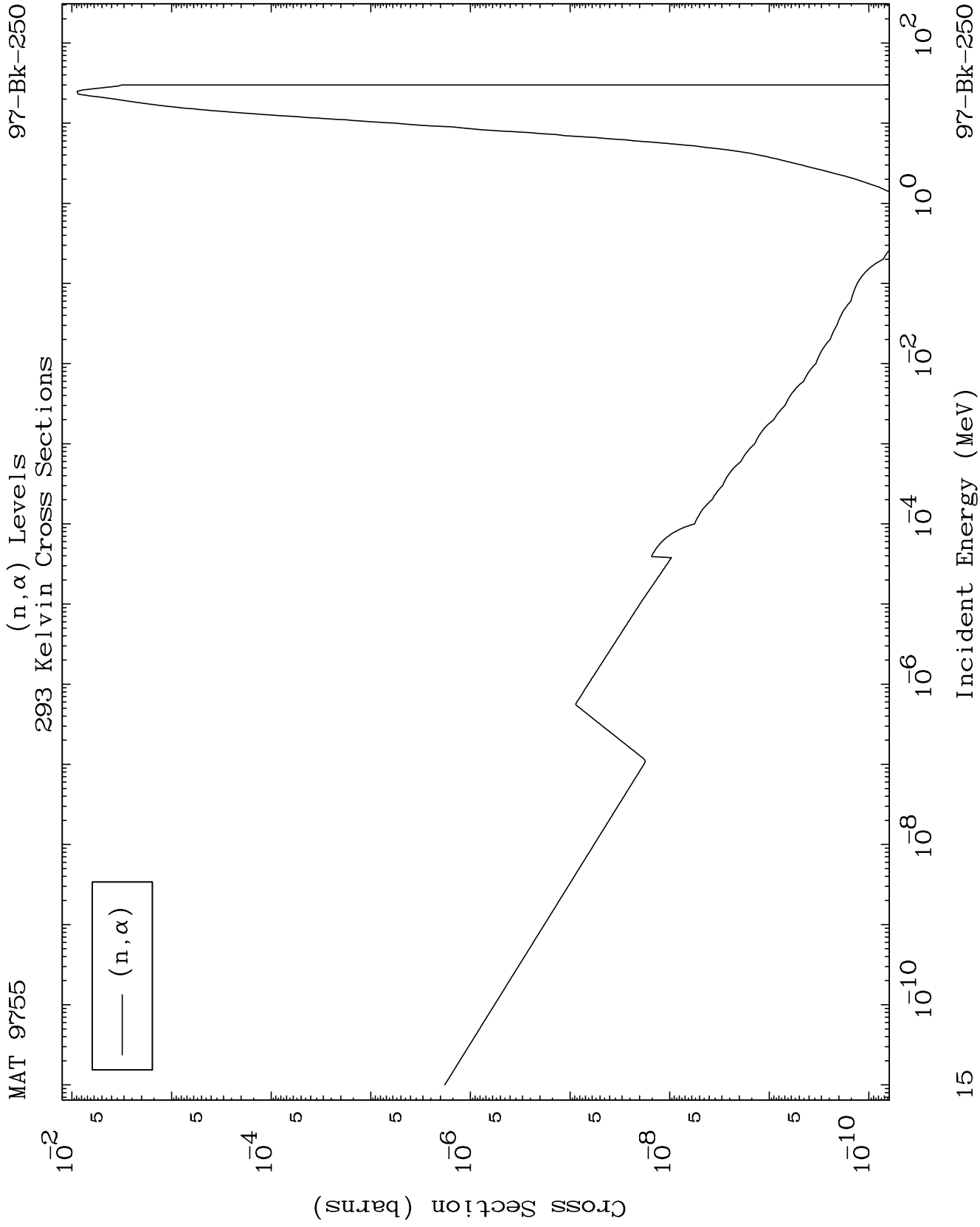
97-Bk-250



14

Incident Energy (MeV)

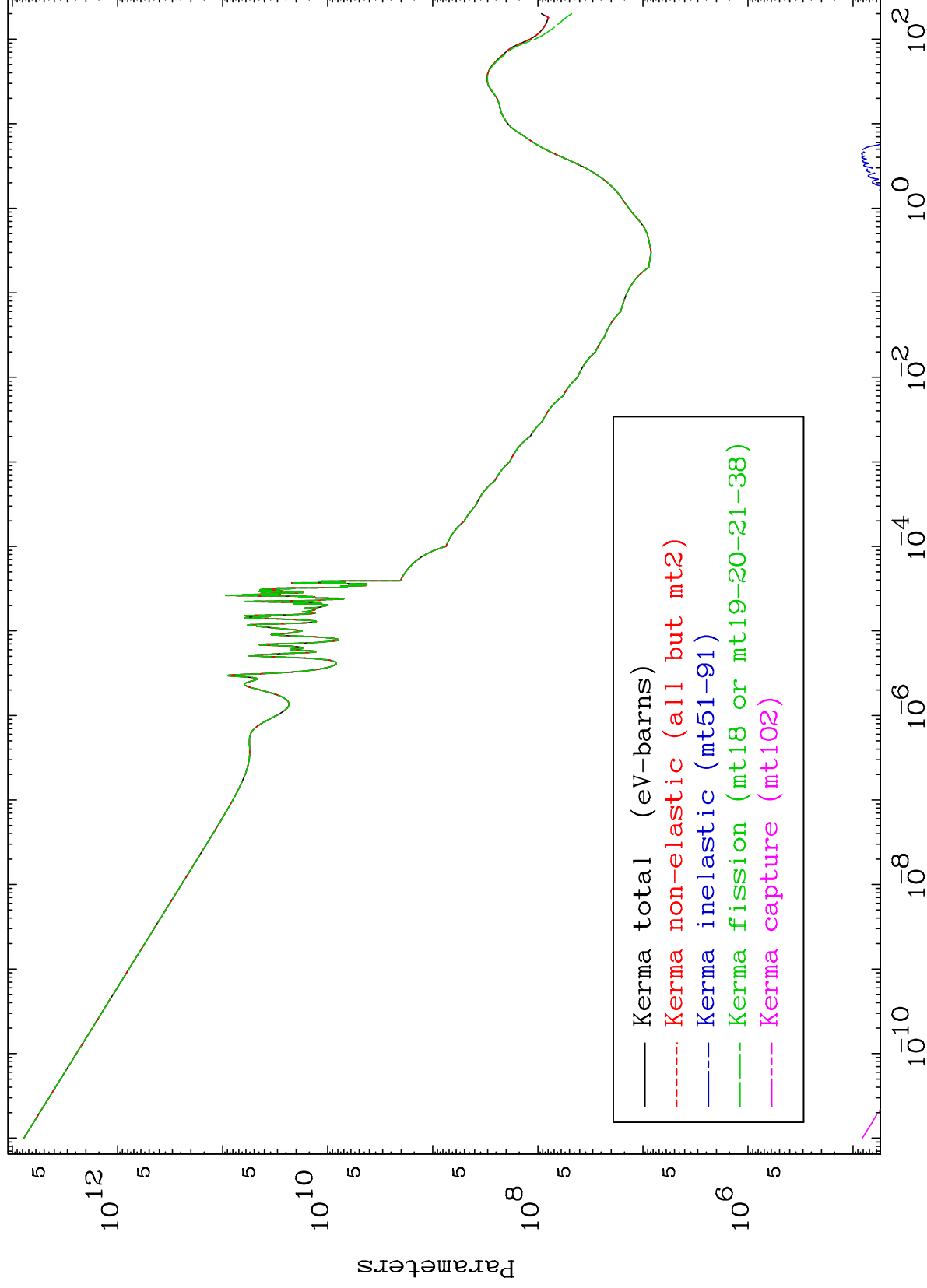
97-Bk-250



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Energy Release
Parameters

97-Bk-250



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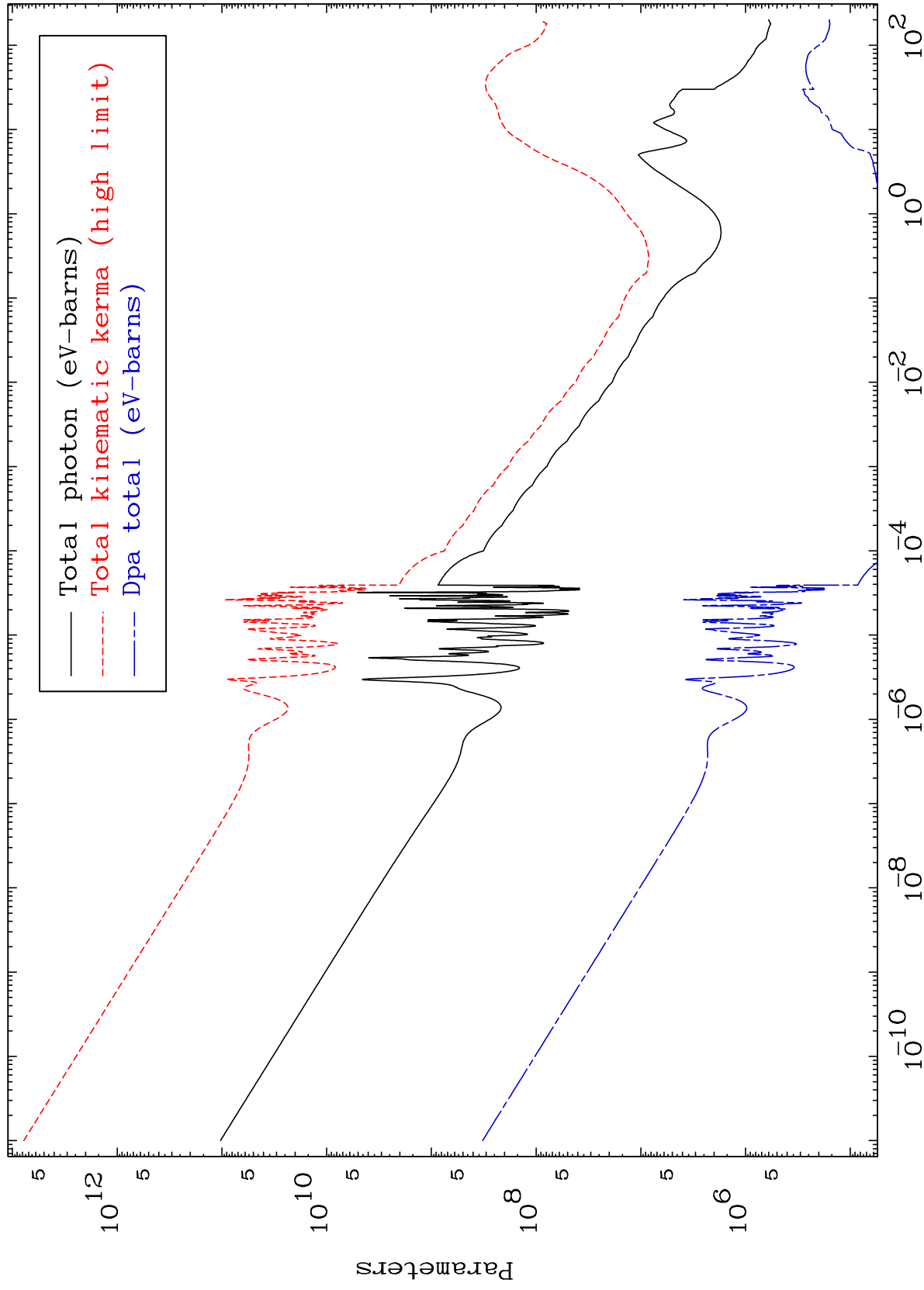
Incident Energy (MeV)

97-Bk-250

MAT 9755

Energy Release
Parameters

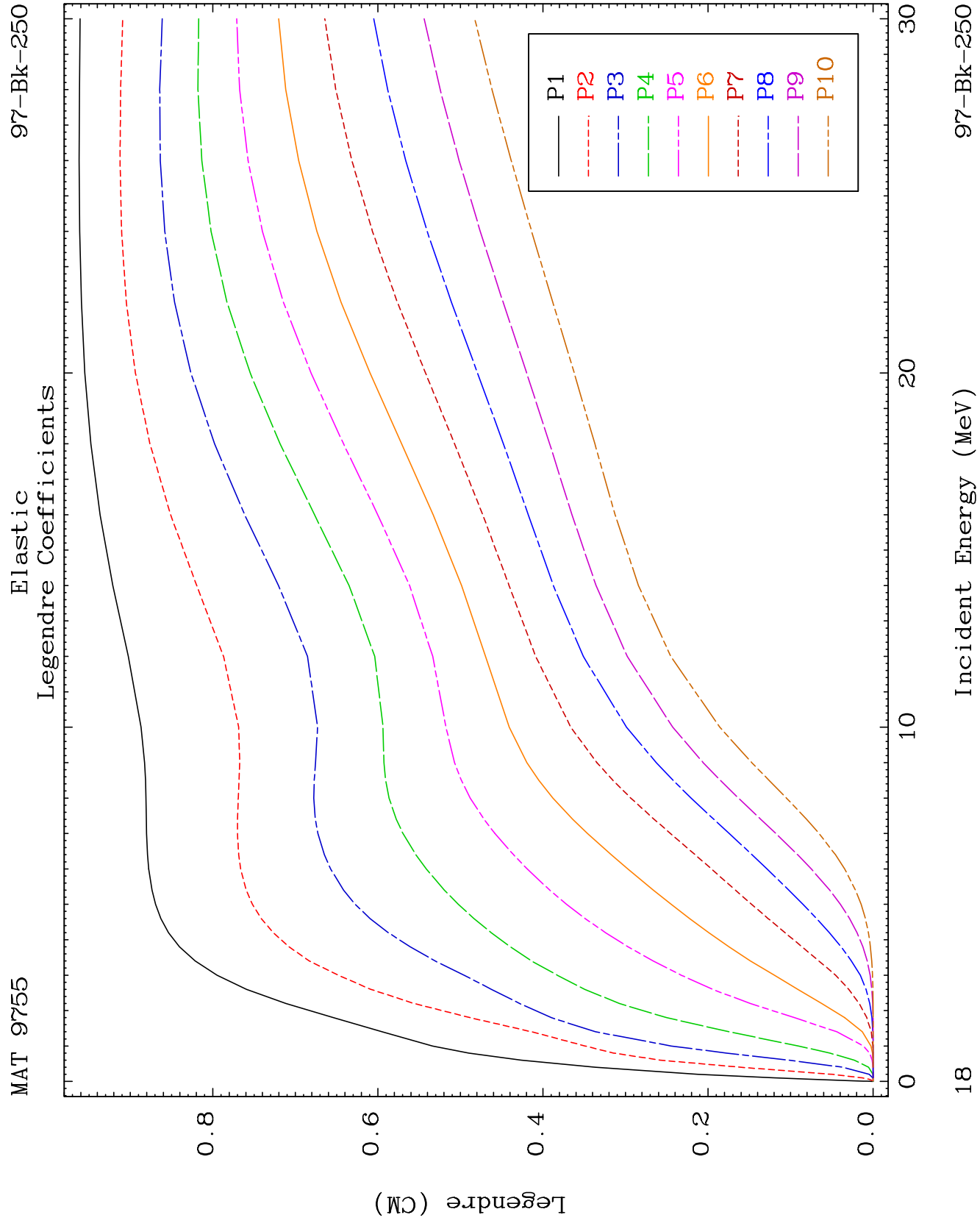
97-Bk-250

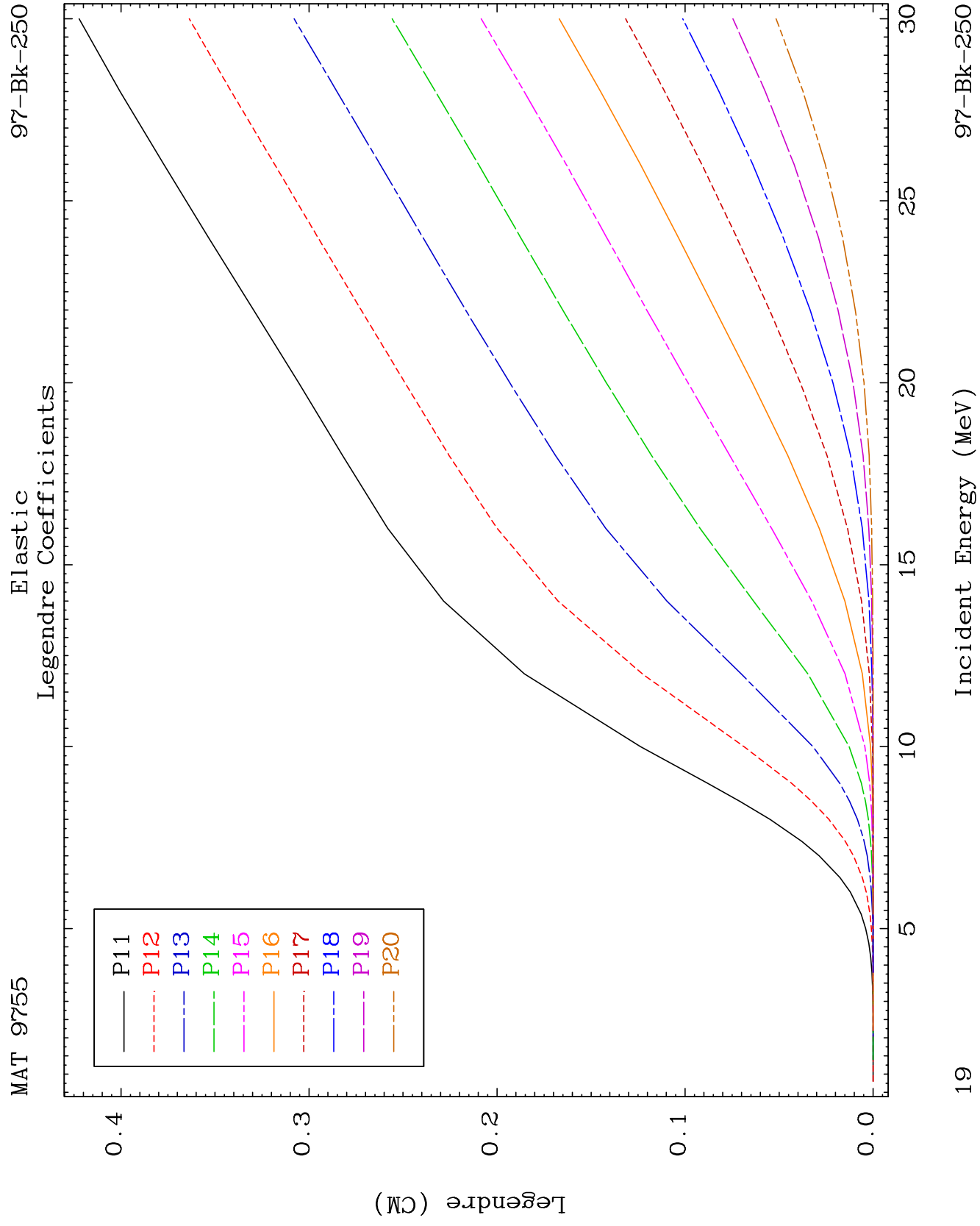


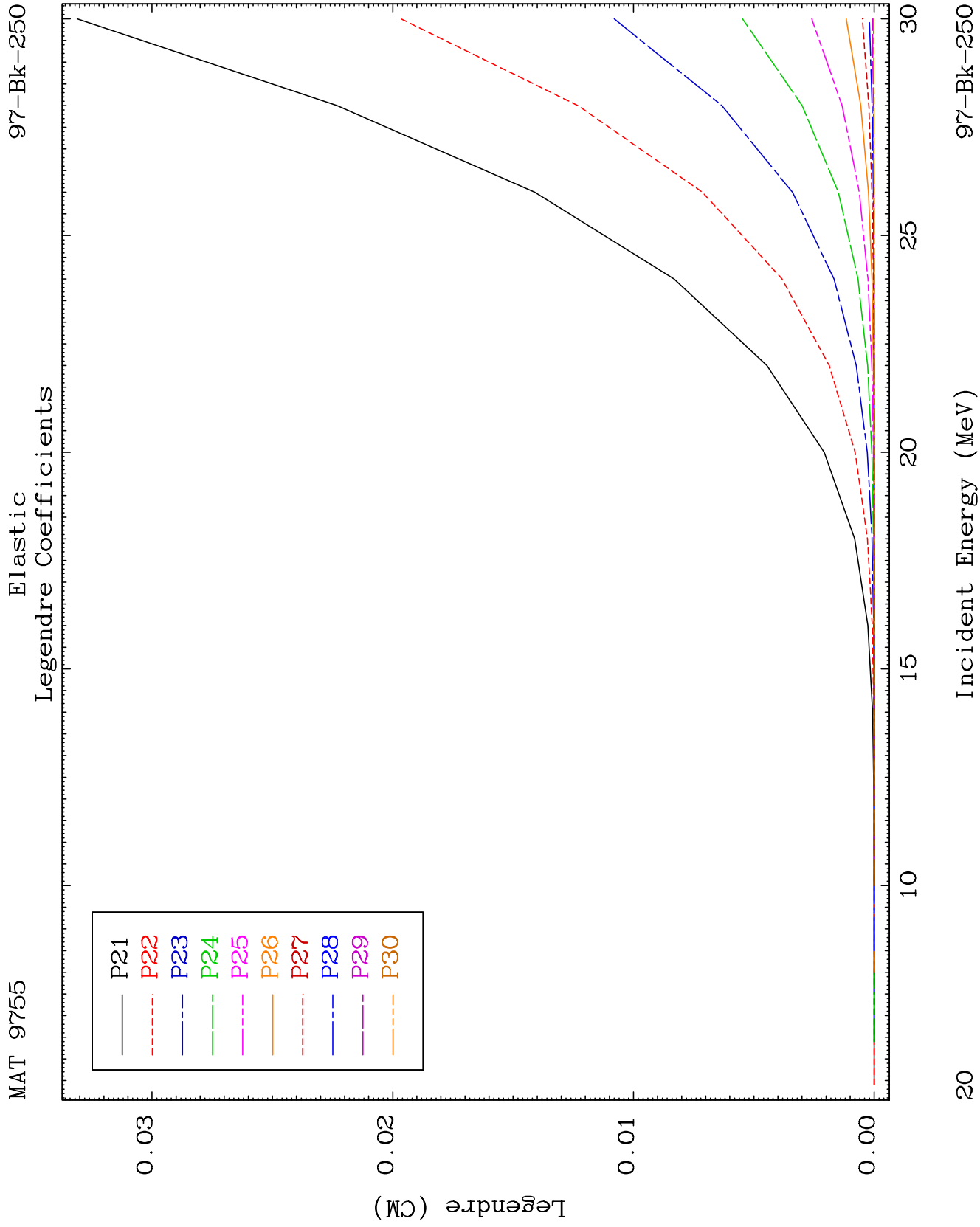
17

Incident Energy (MeV)

97-Bk-250



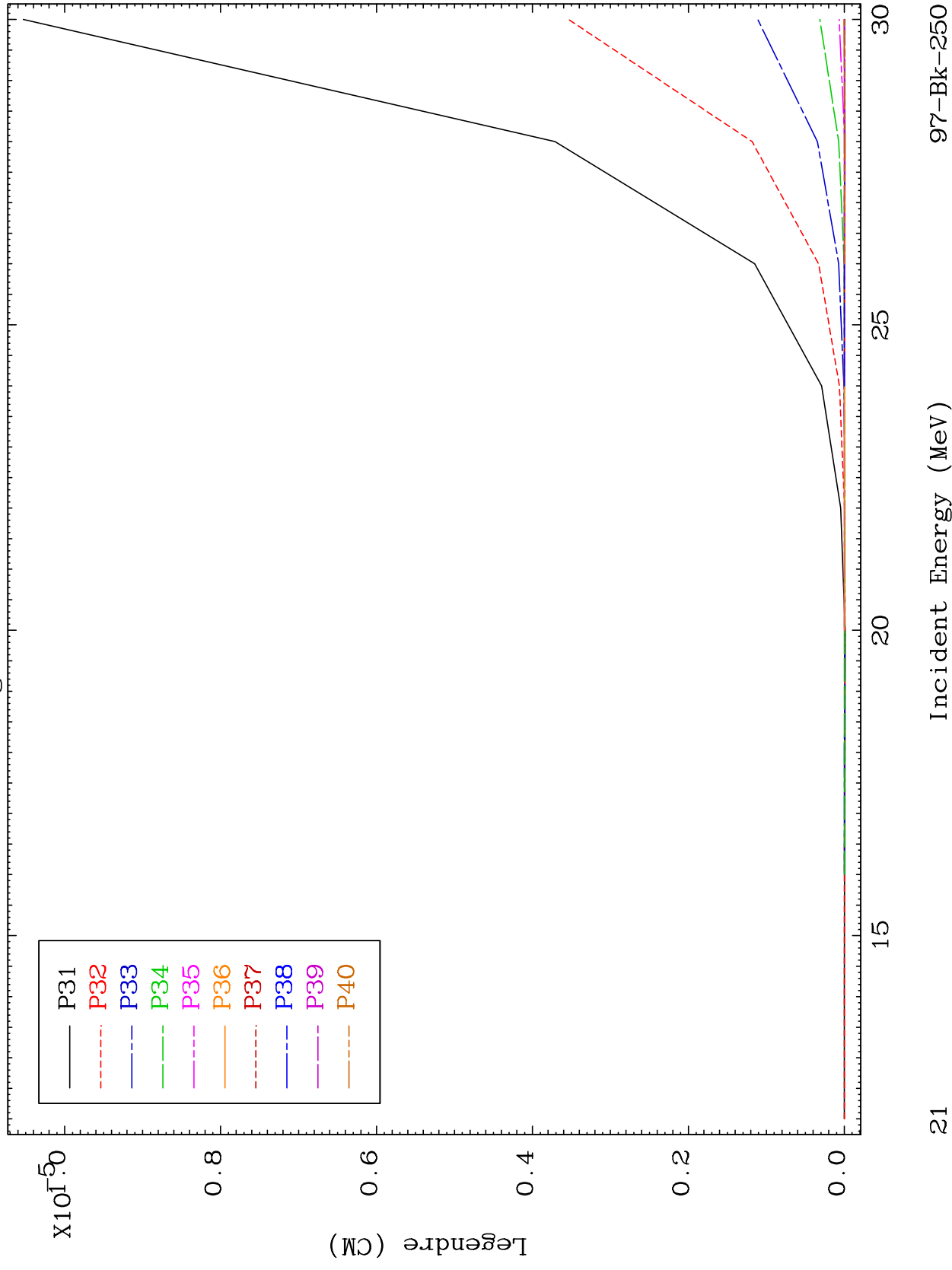




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Elastic Legendre Coefficients

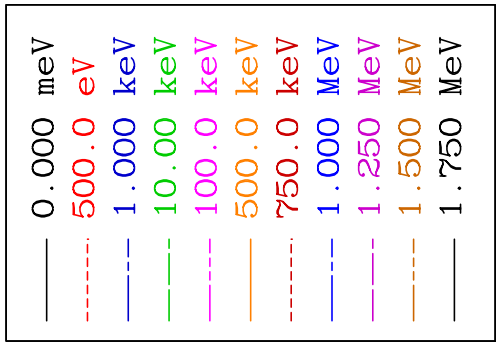
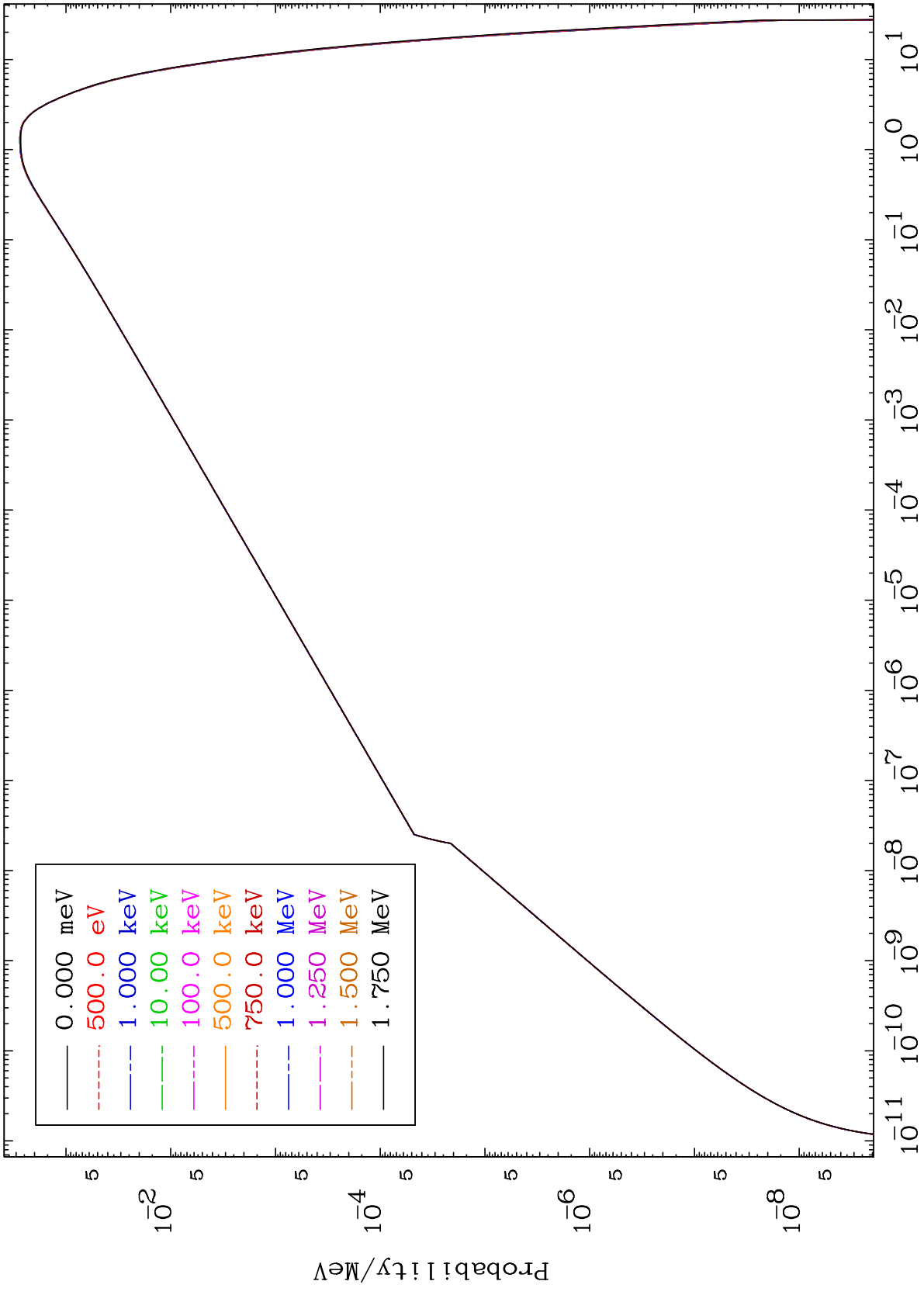
97-Bk-250



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Fission
Energy Distribution

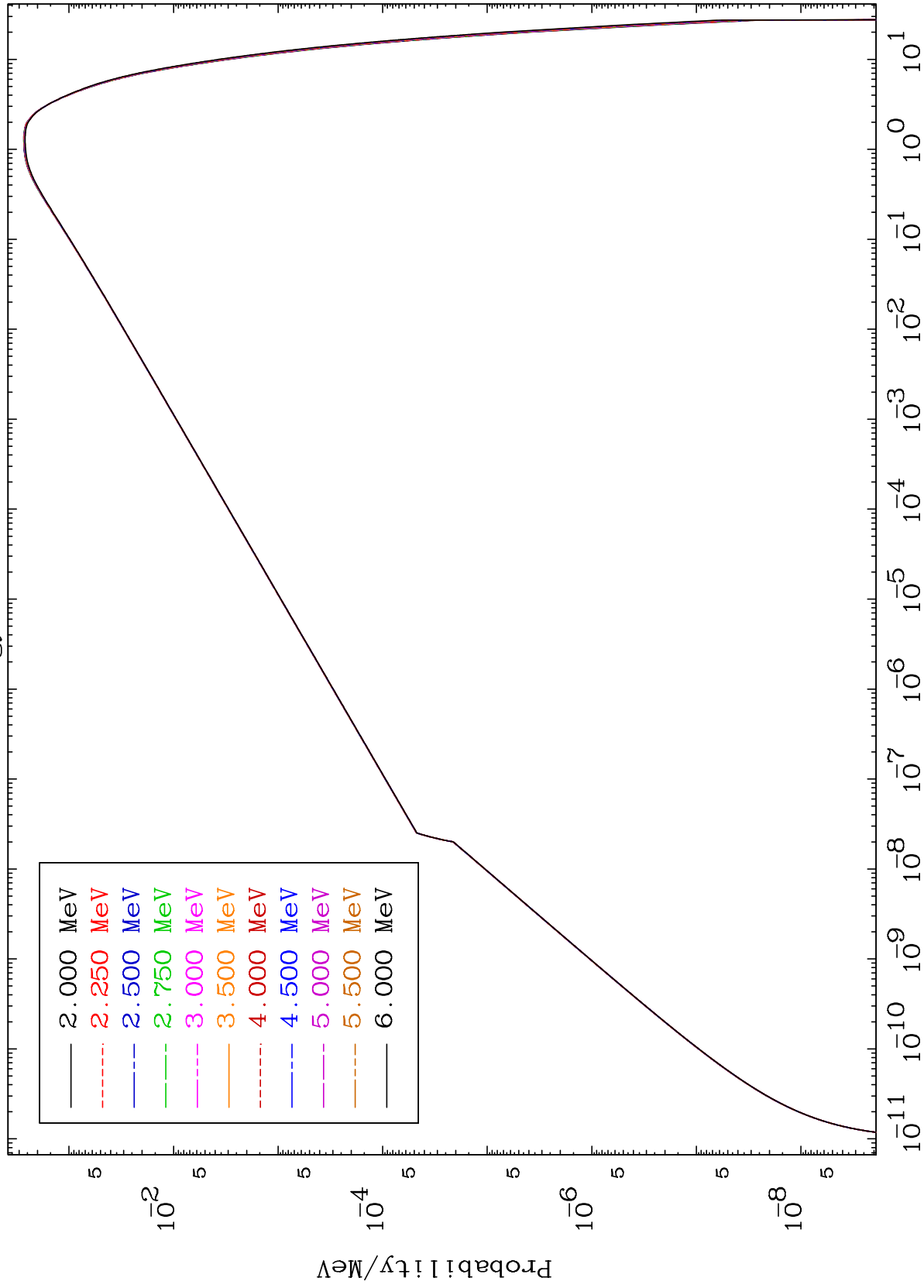
97-Bk-250



MAT 9755

Fission
Energy Distribution

97-Bk-250



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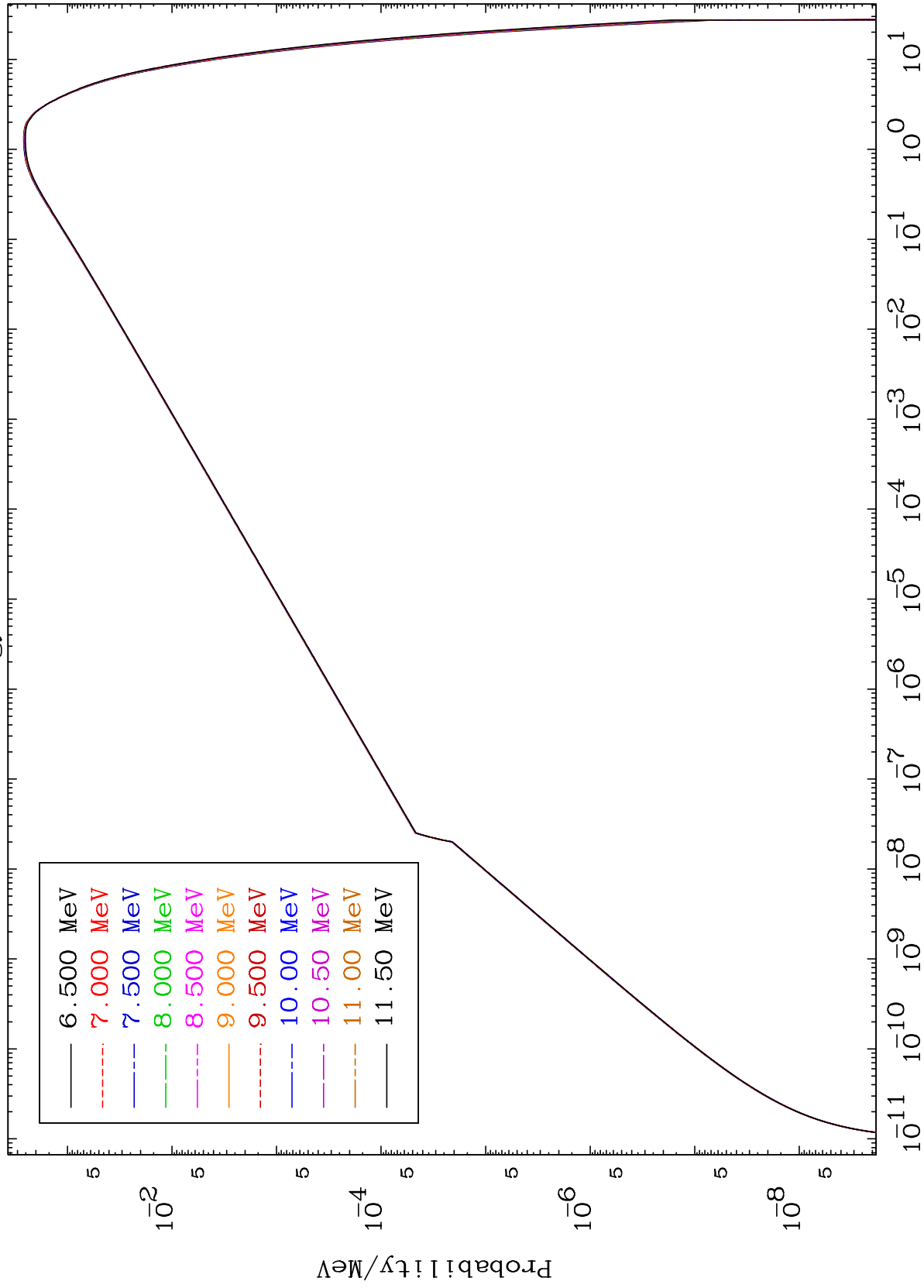
Secondary Energy (MeV)

97-Bk-250

MAT 9755

Fission
Energy Distribution

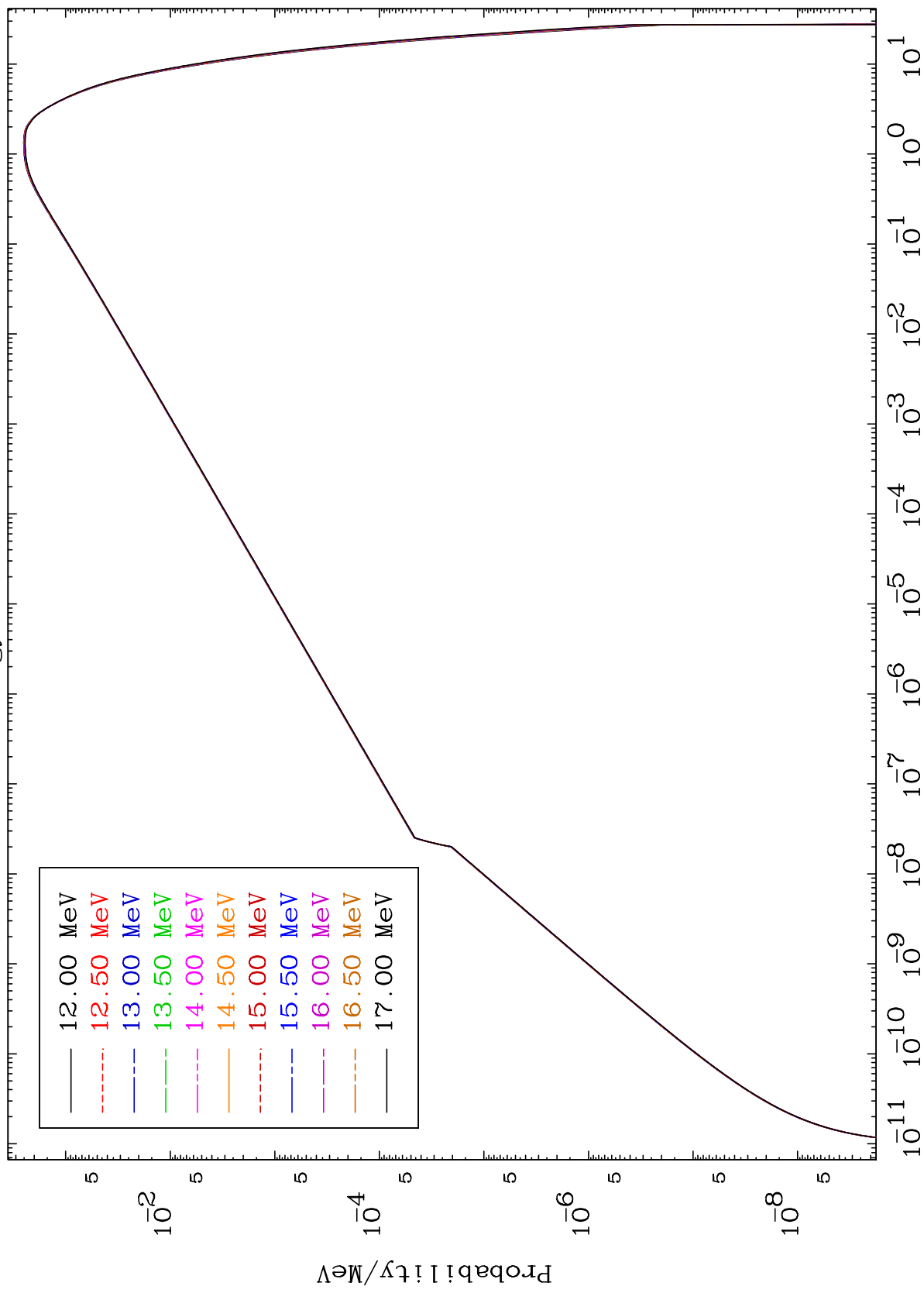
97-Bk-250



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Fission
Energy Distribution

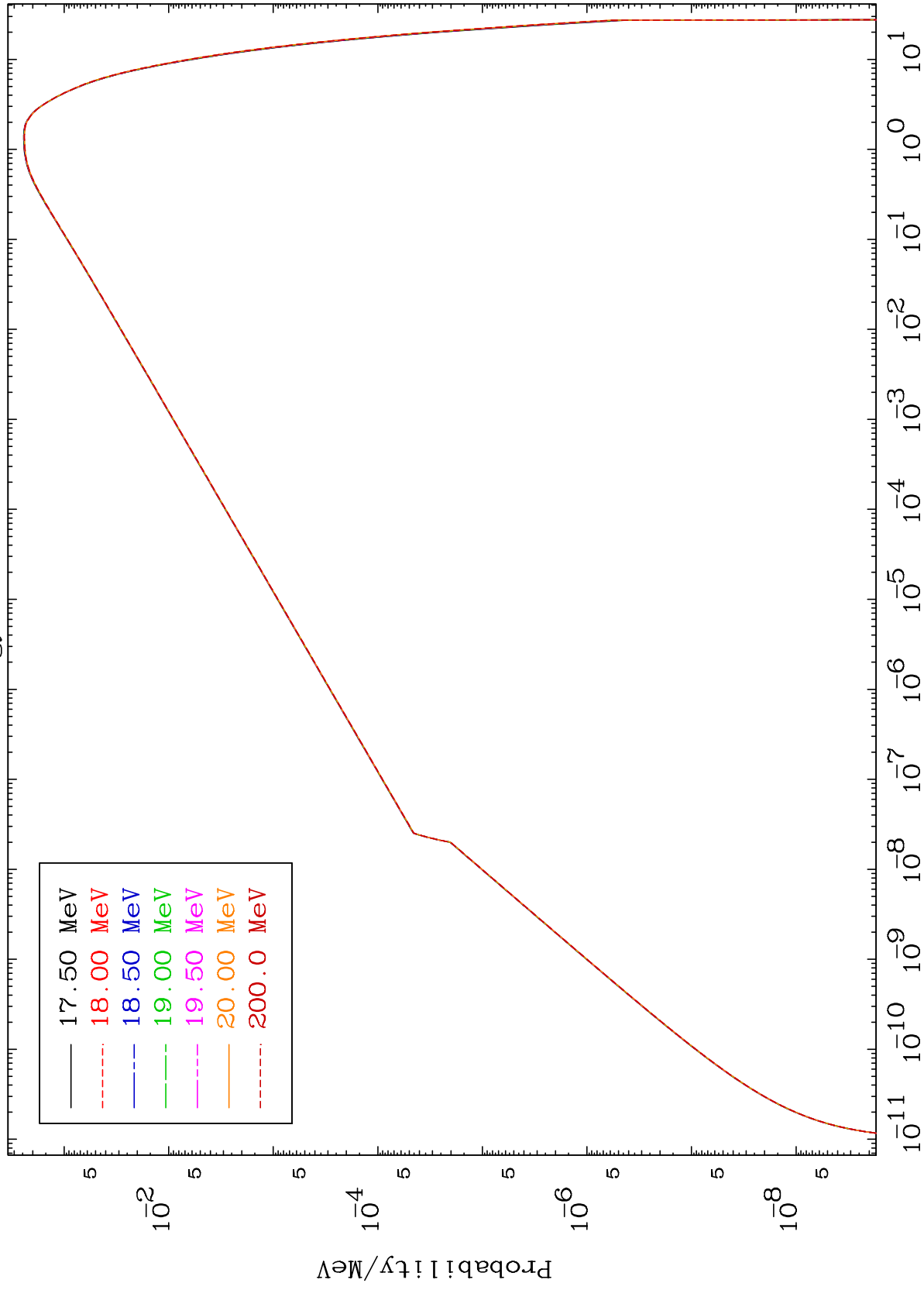
97-Bk-250



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Fission
Energy Distribution

97-Bk-250



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Secondary Energy (MeV)

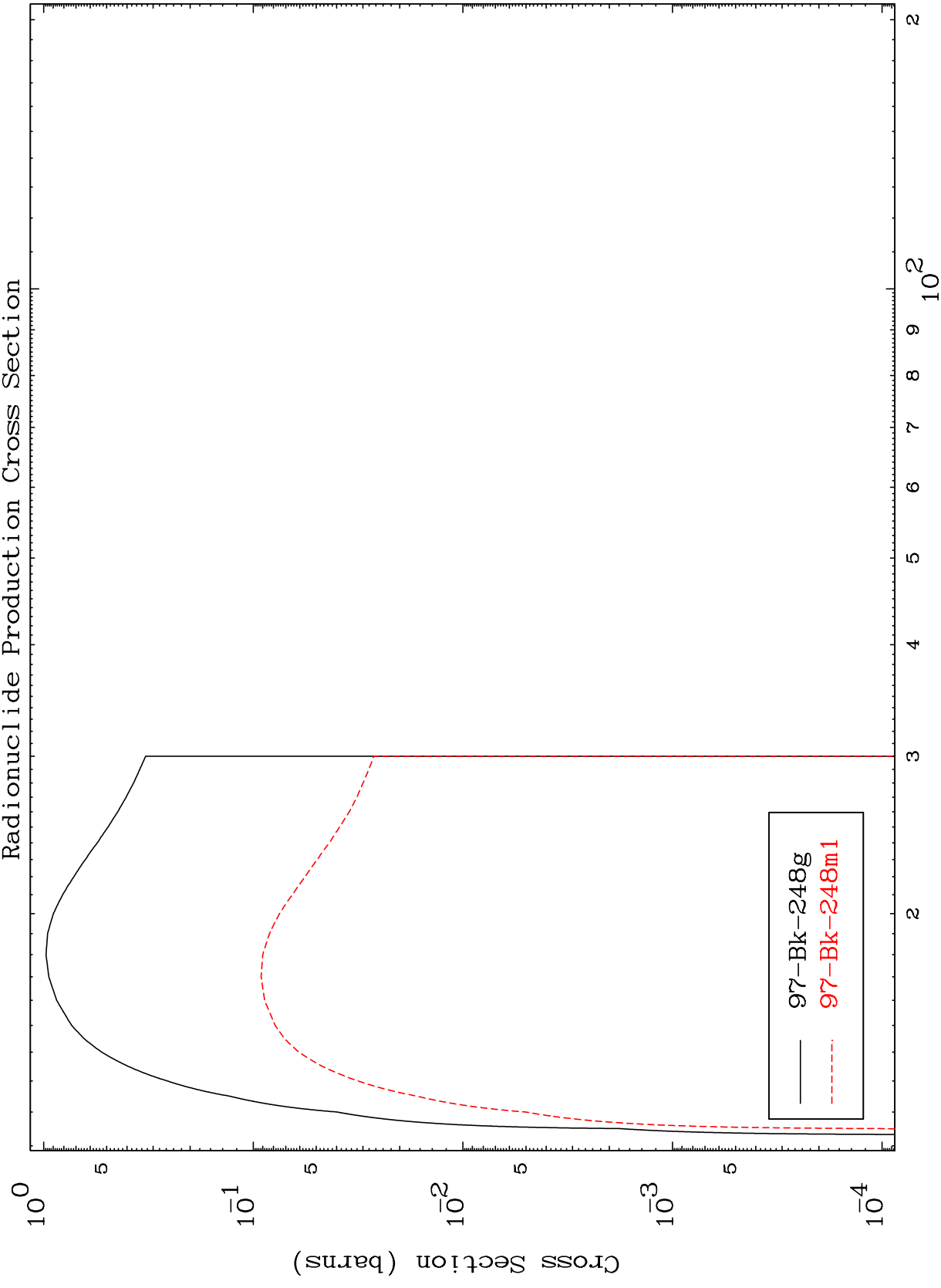
97-Bk-250

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$$(n, 3n)$$

97-Bk-250

Radionuclide Production Cross Section

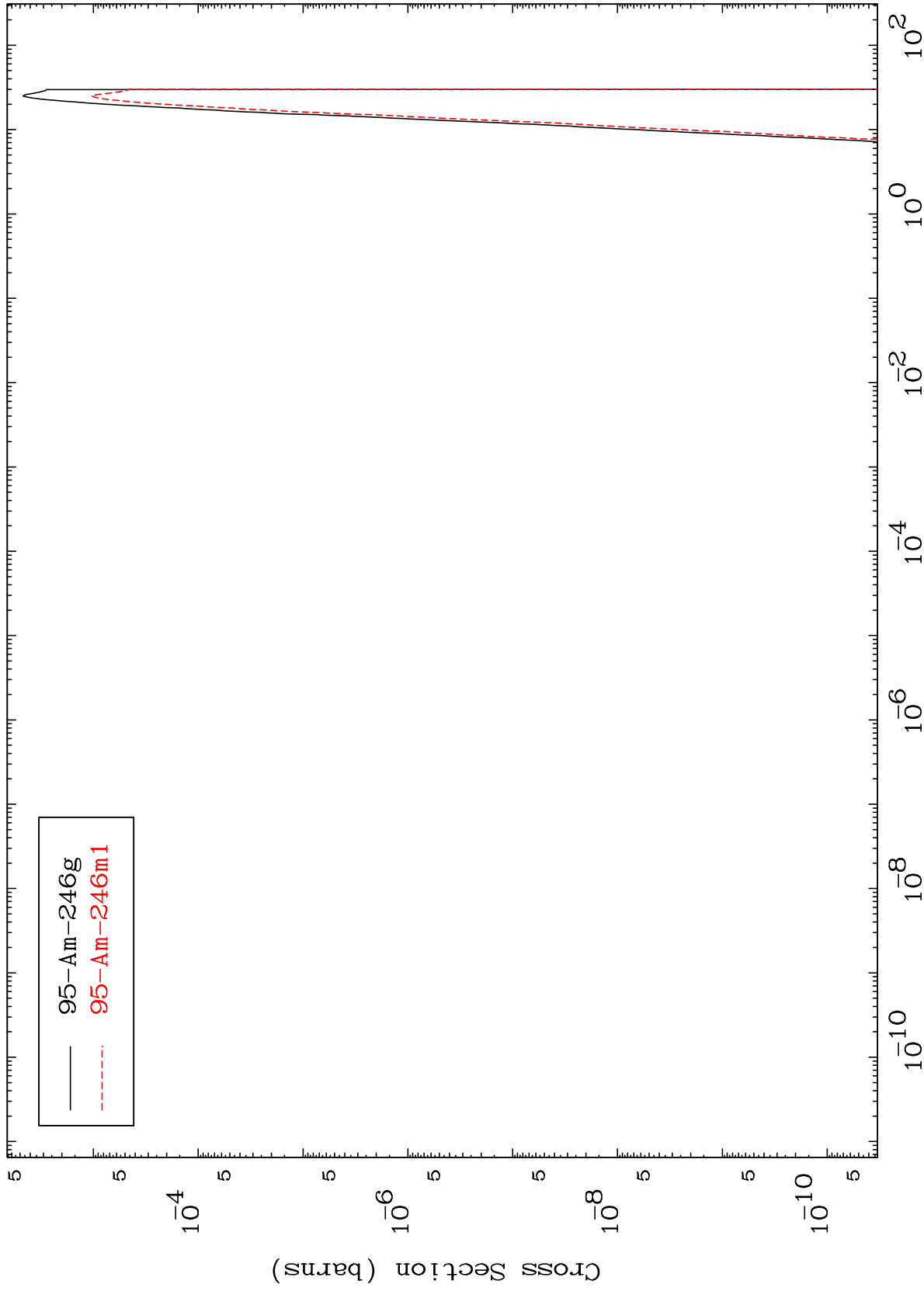


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$(n,n') \alpha$

97-Bk-250

Radionuclide Production Cross Section



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Incident Energy (MeV)

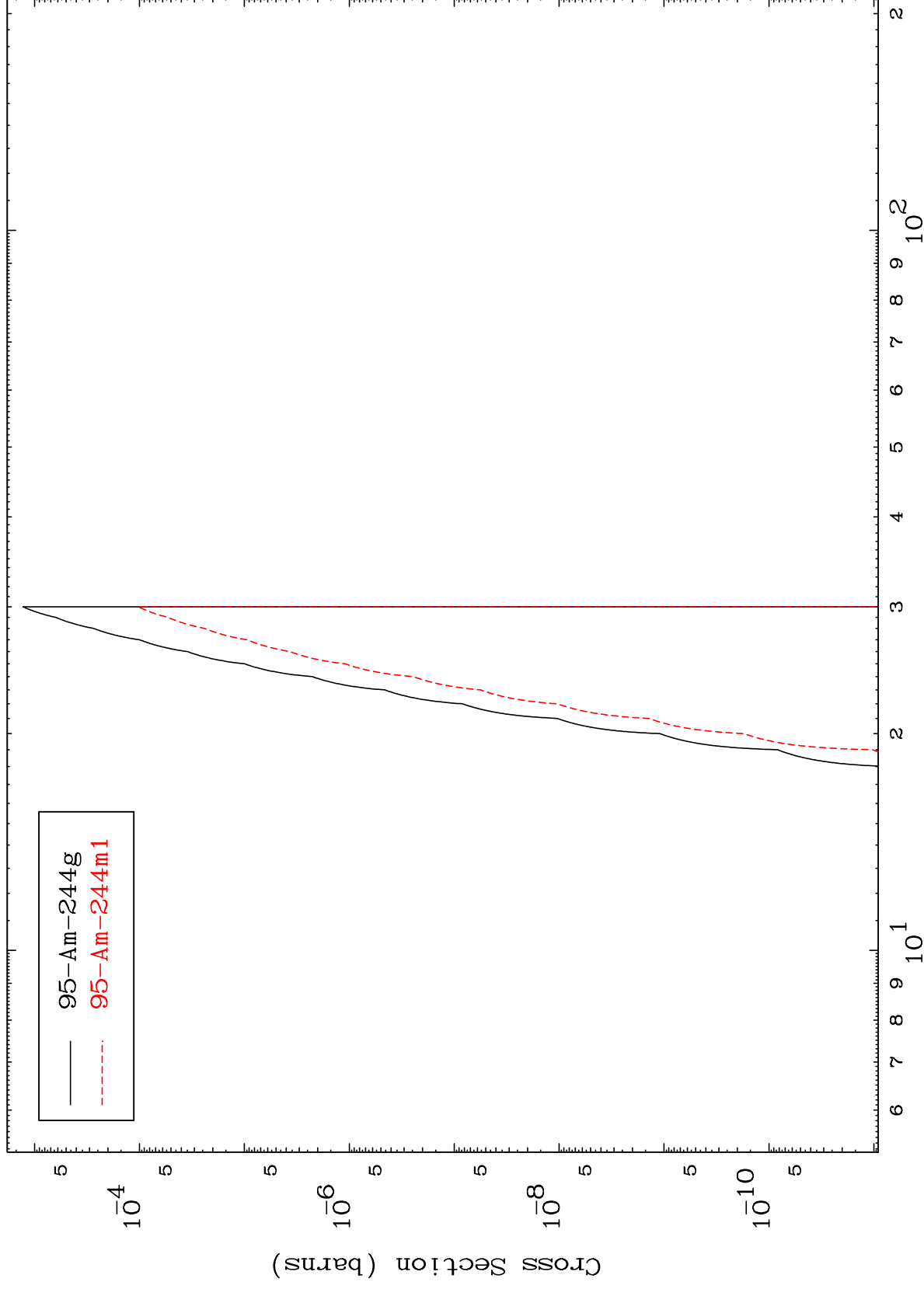
97-Bk-250

MAT 9755

$(n,3n) \alpha$

97-Bk-250

Radionuclide Production Cross Section



29

Incident Energy (MeV)

97-Bk-250