

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550

U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

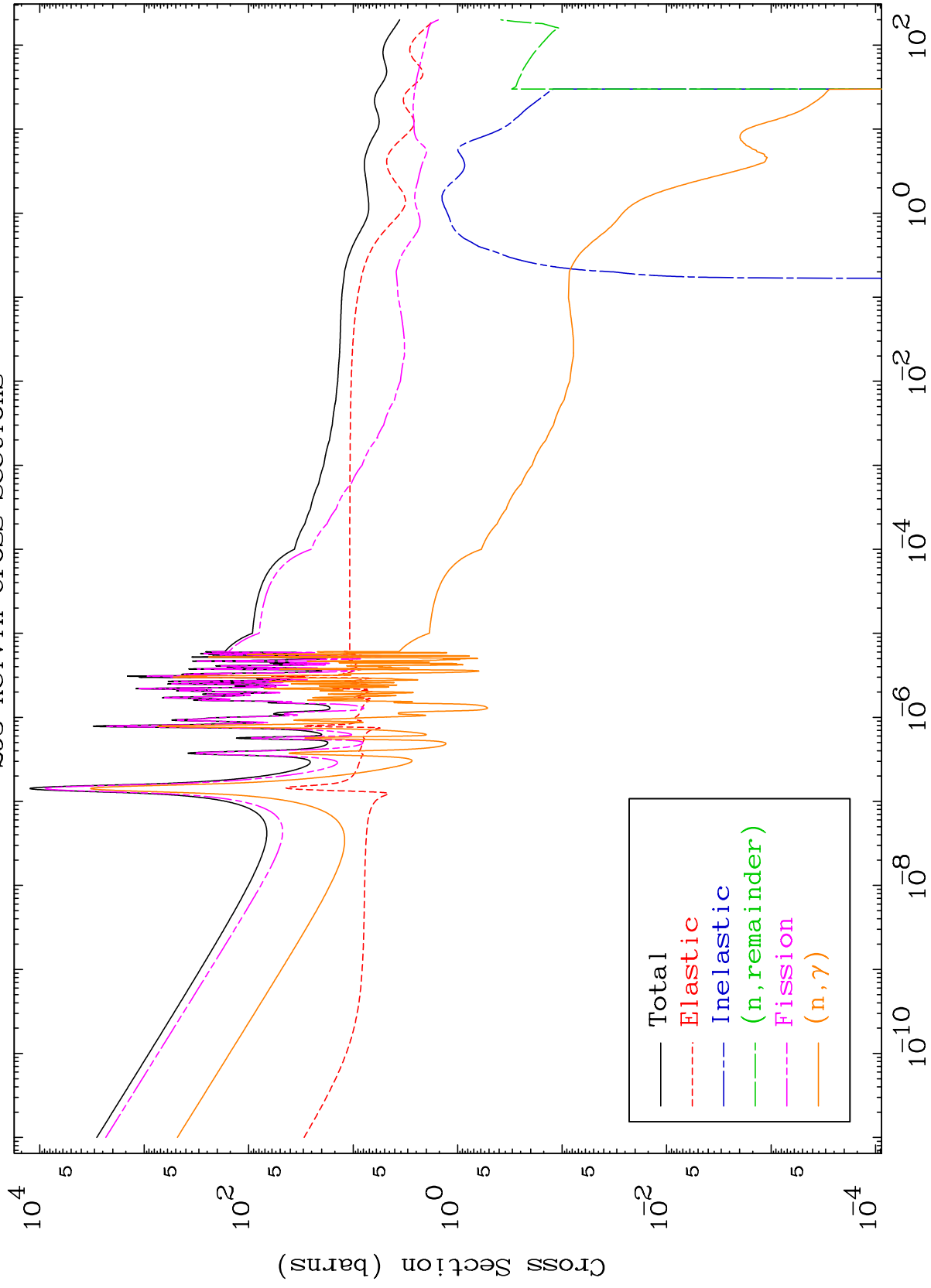
Press Mouse Button to Start

MAT 9828

Major

293 Kelvin Cross Sections

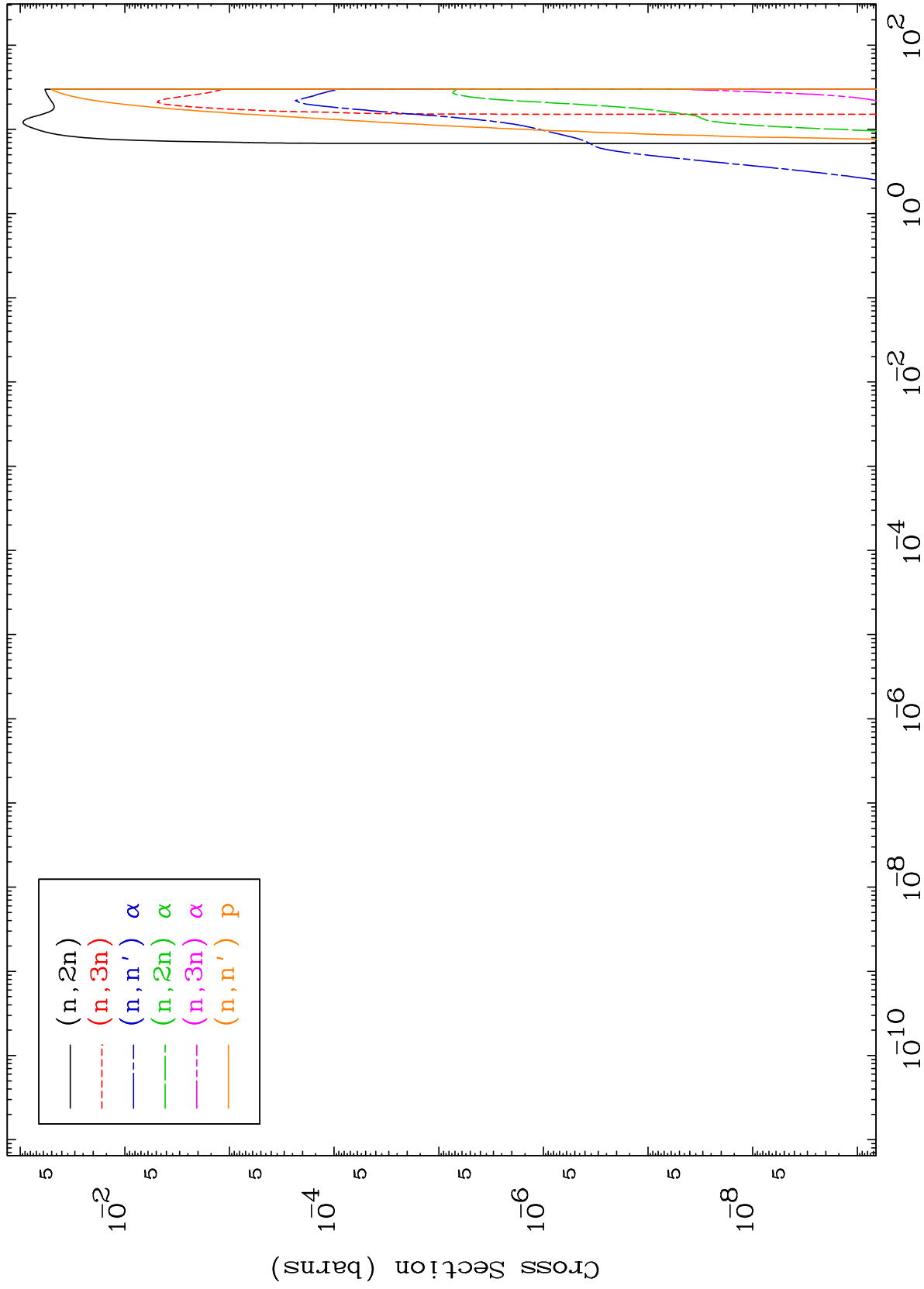
98-Cf-241



MAT 9828

Neutron Production  
293 Kelvin Cross Sections

98-Cf-241



2

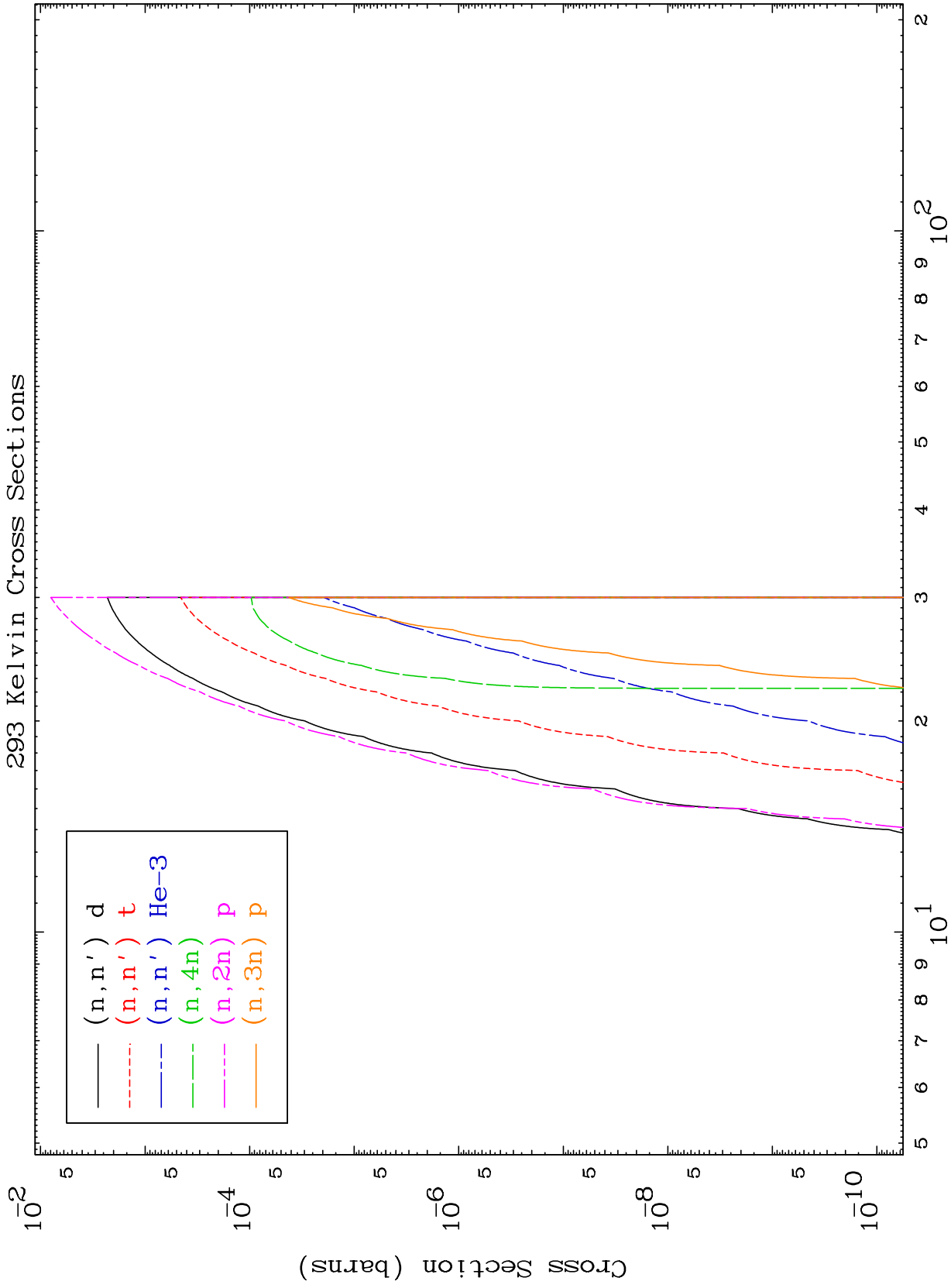
Incident Energy (MeV)

98-Cf-241

MAT 9828

Neutron Production  
293 Kelvin Cross Sections

98-Cf-241



3

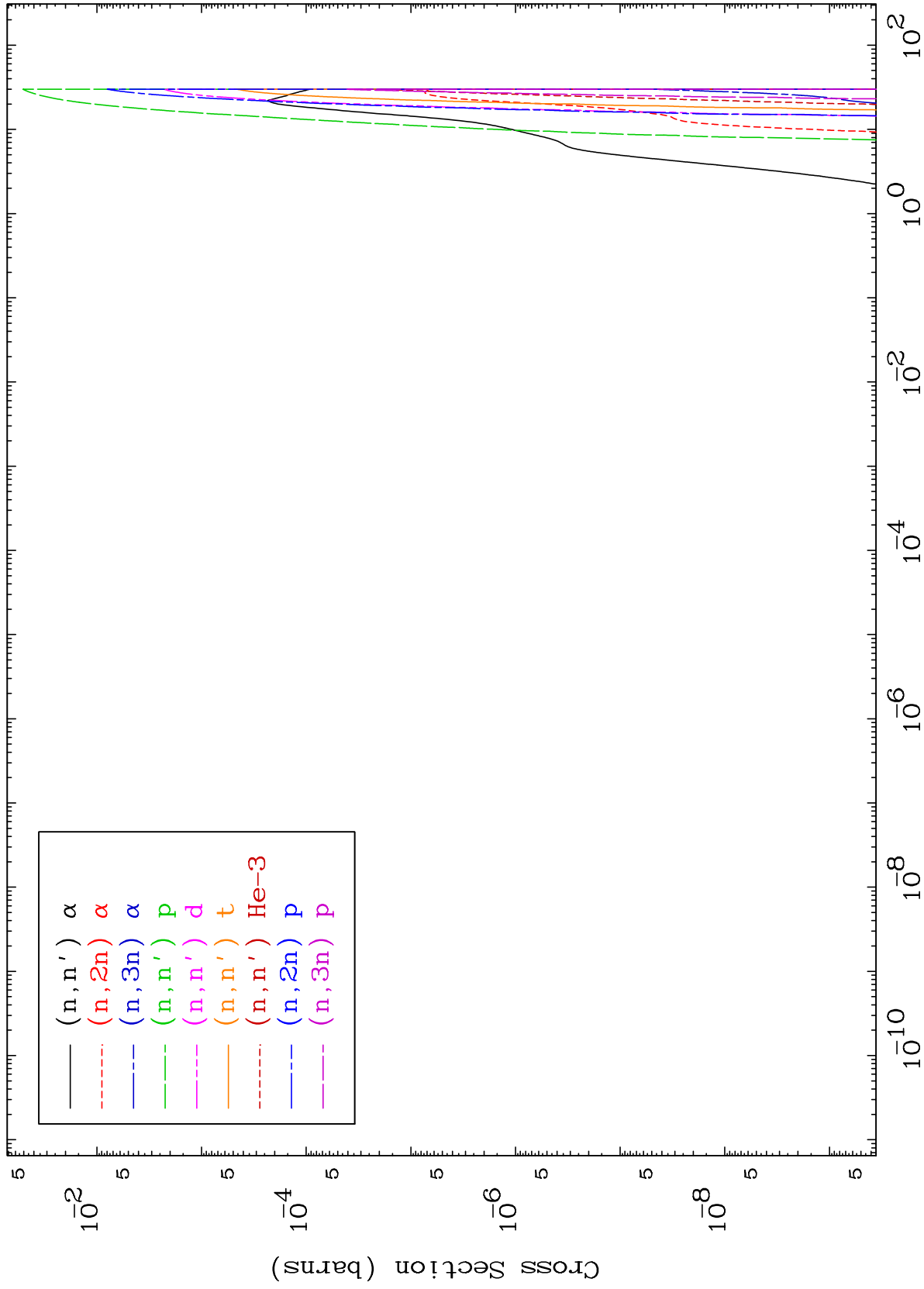
Incident Energy (MeV)

98-Cf-241

MAT 9828

293 Kelvin Cross Sections

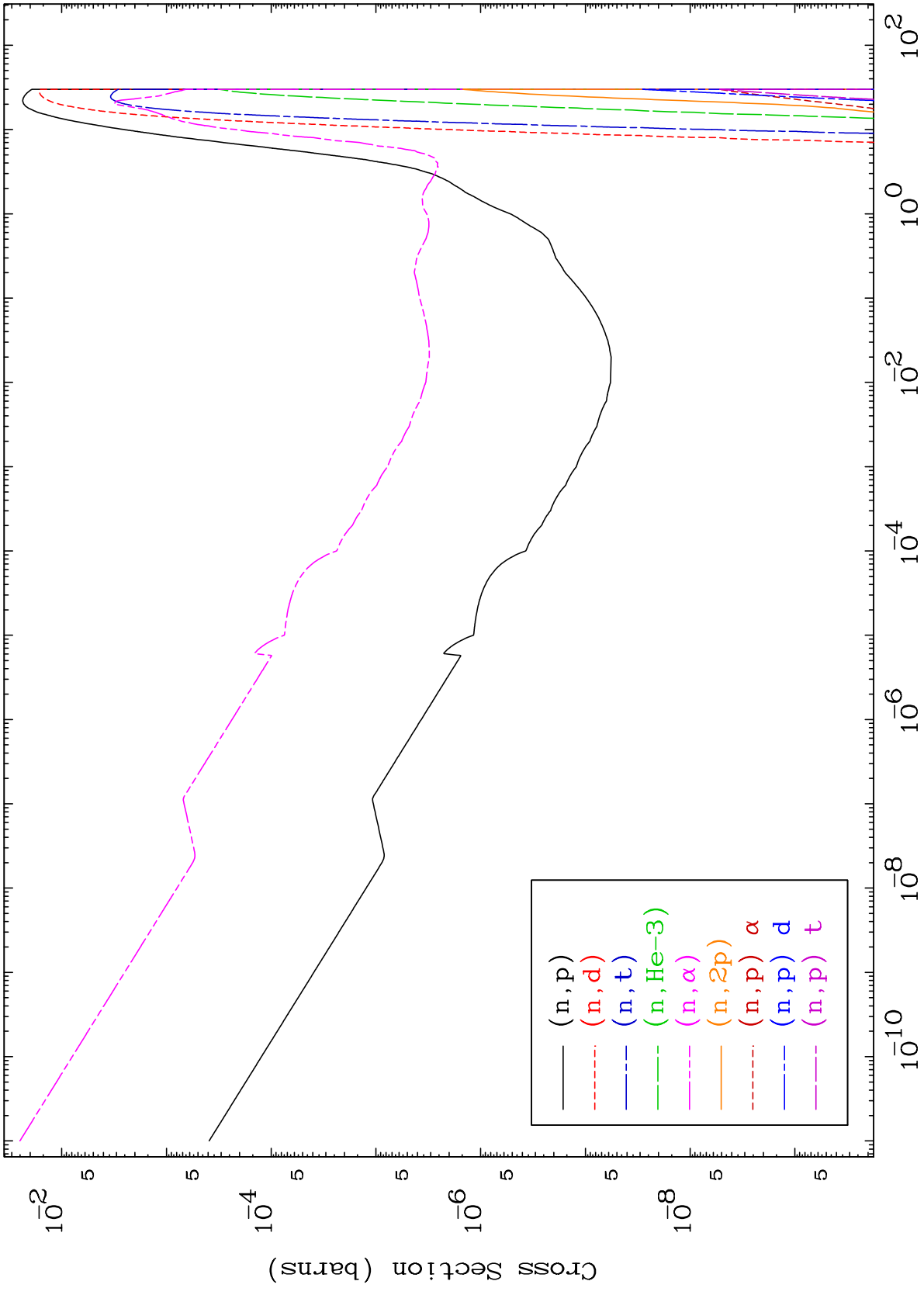
98-Cf-241



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Charged Particle  
293 Kelvin Cross Sections

98-Cf-241



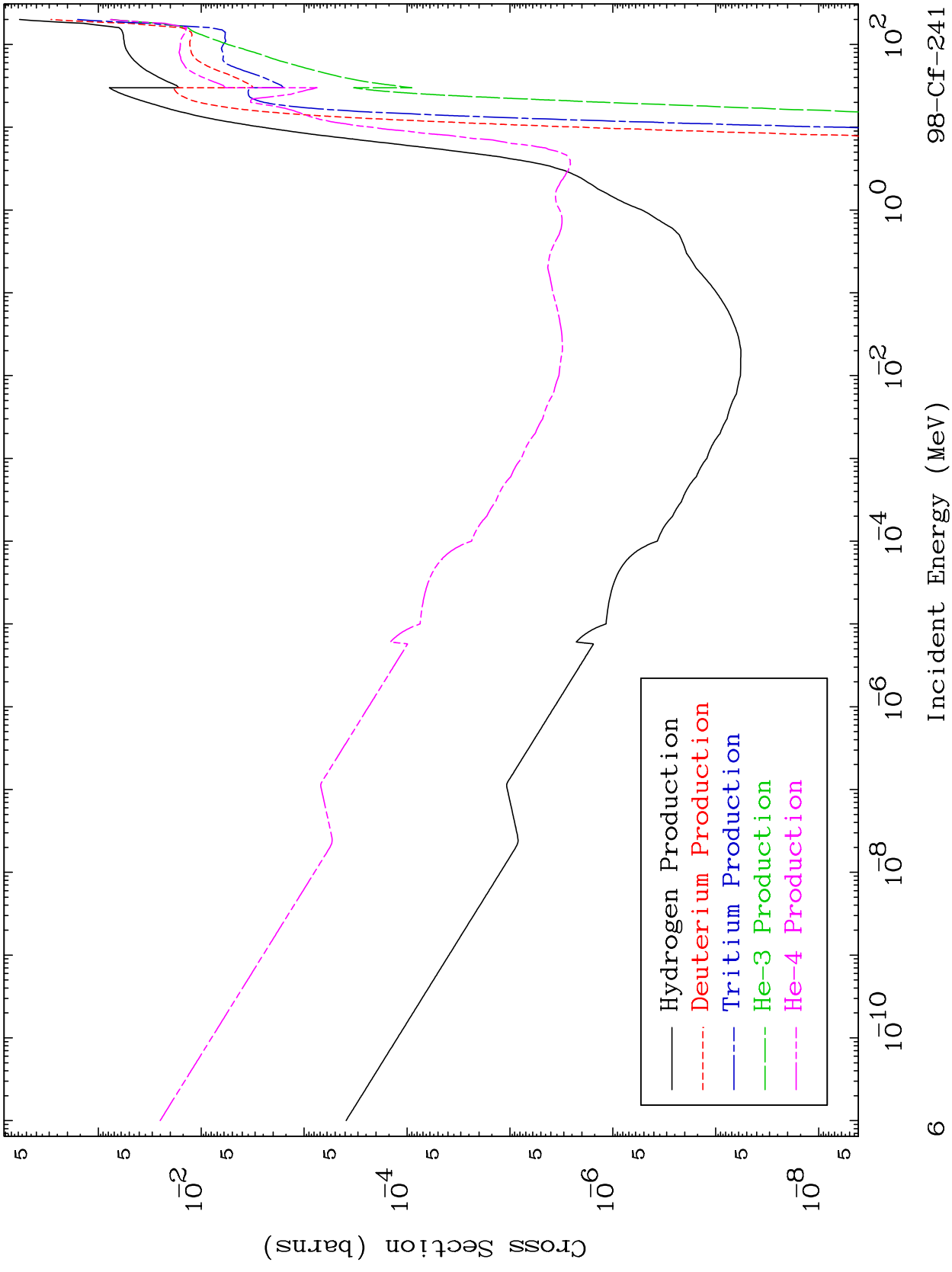
5

98-Cf-241

MAT 9828

Particle Production  
293 Kelvin Cross Sections

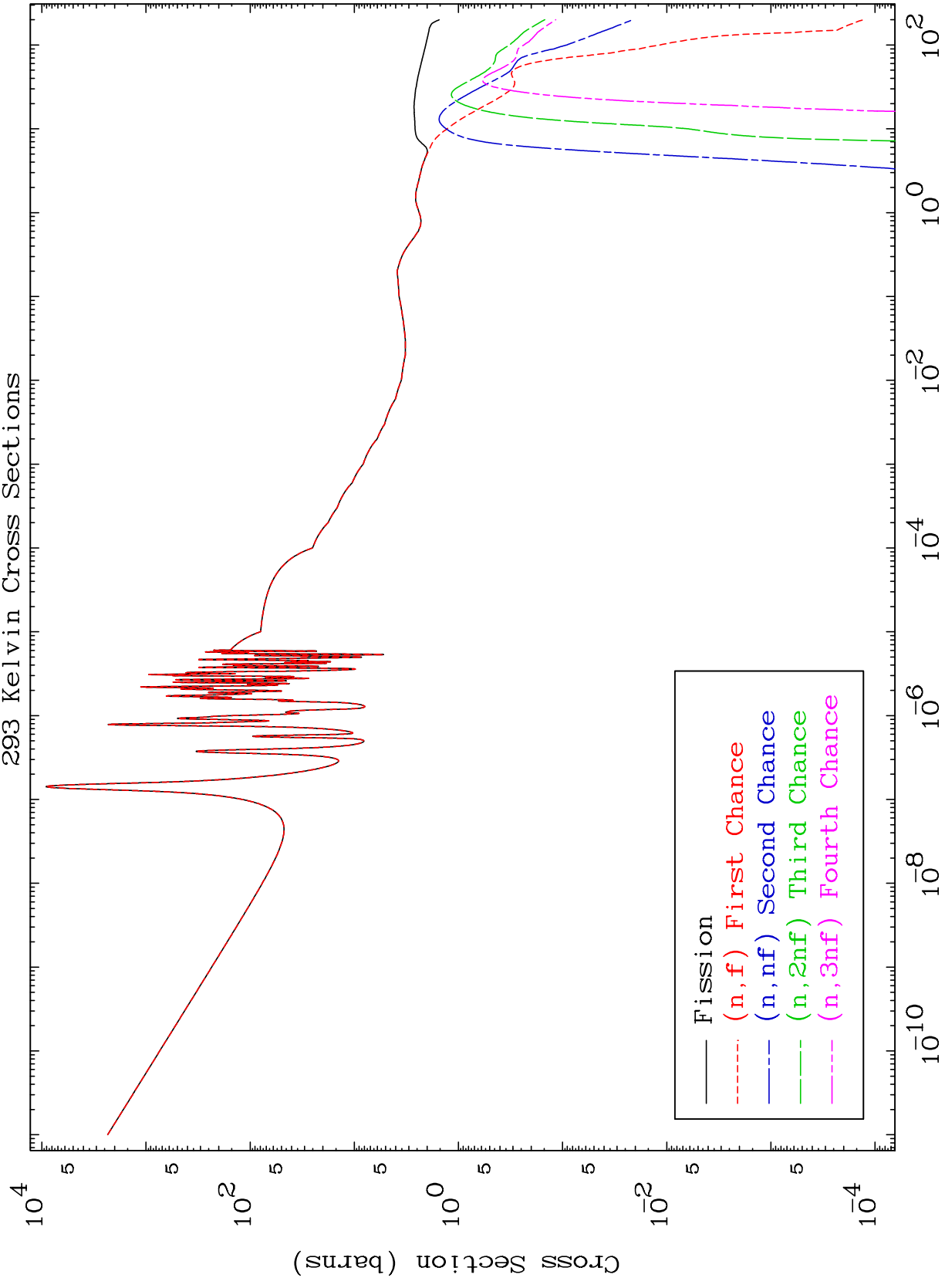
98-Cf-241



MAT 9828

Fission  
293 Kelvin Cross Sections

98-Cf-241



98-Cf-241

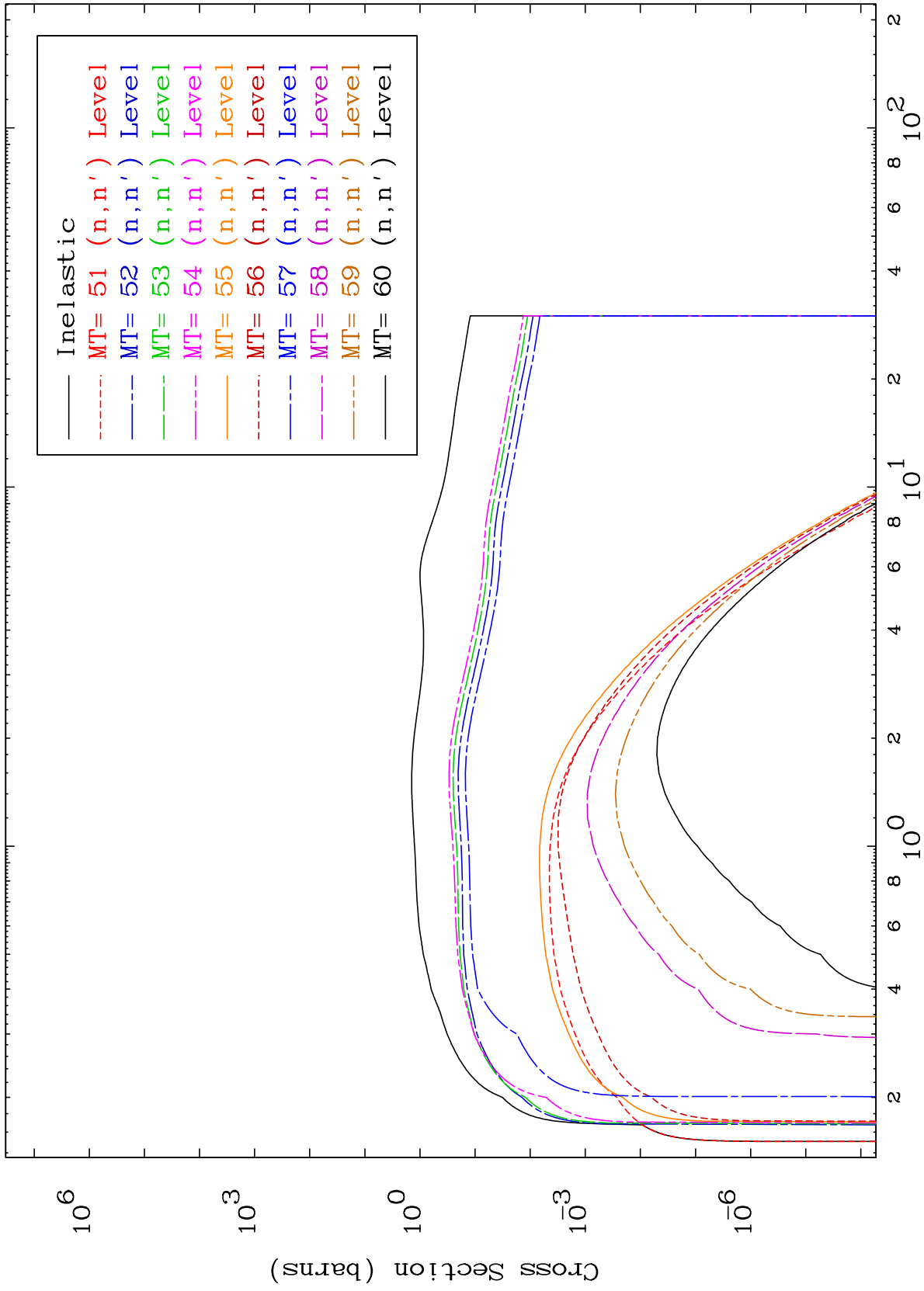


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(n,n') Level

98-Cf-241

293 Kelvin Cross Sections



8

Incident Energy (MeV)

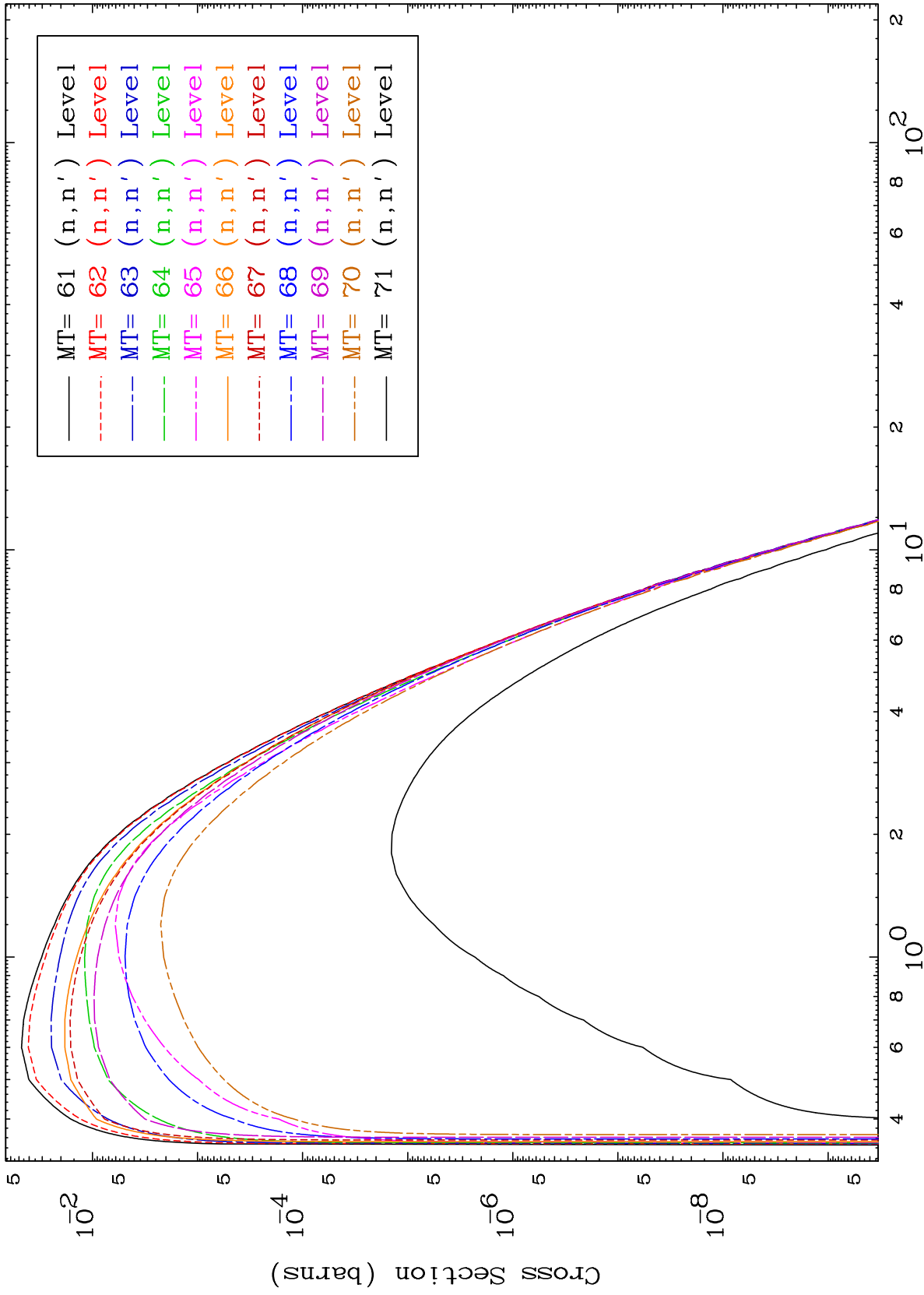
98-Cf-241

MAT 9828

(n,n') Level

98-Cf-241

293 Kelvin Cross Sections



9

Incident Energy (MeV)

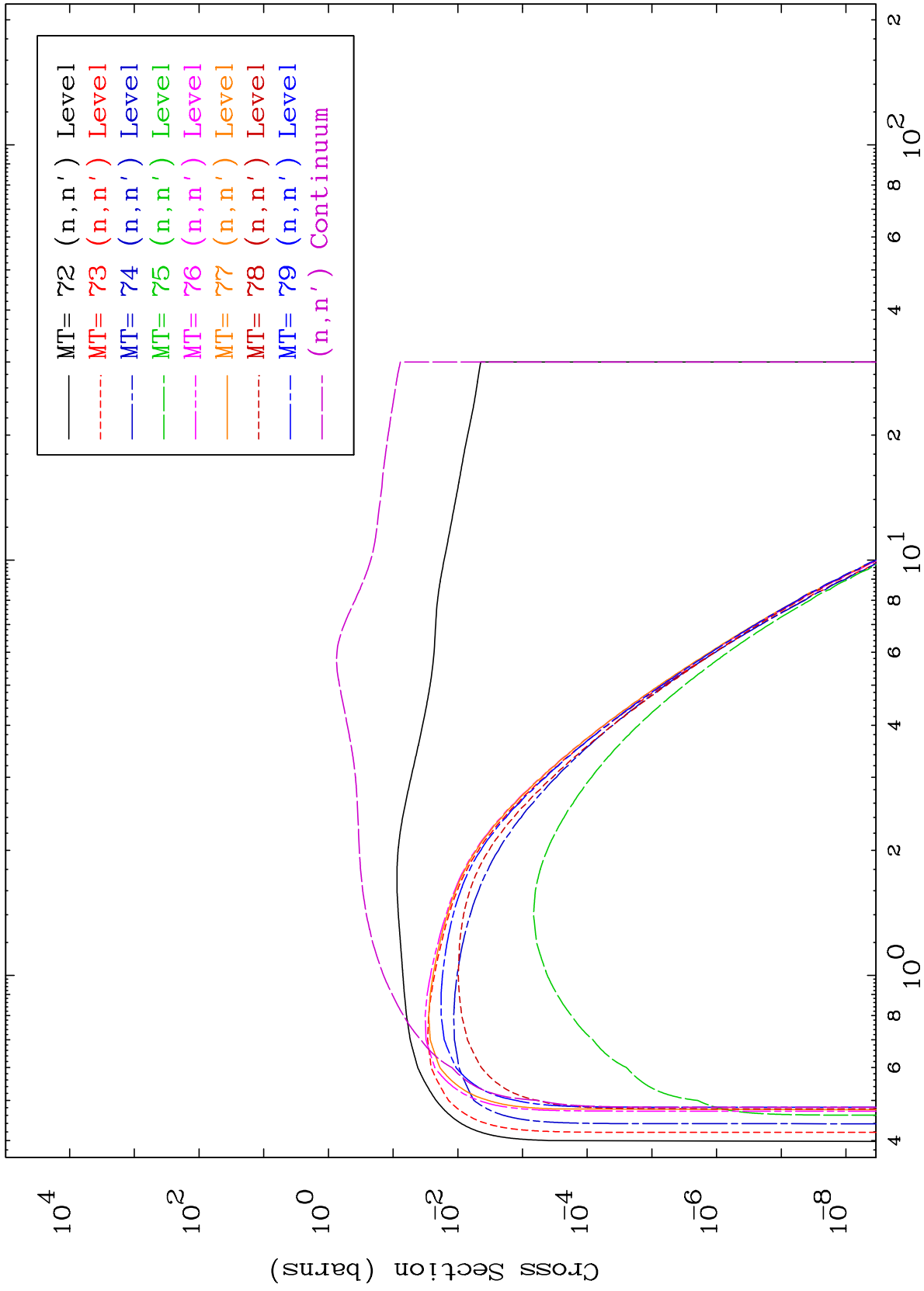
98-Cf-241

MAT 9828

(n,n') Level

98-Cf-241

293 Kelvin Cross Sections



10

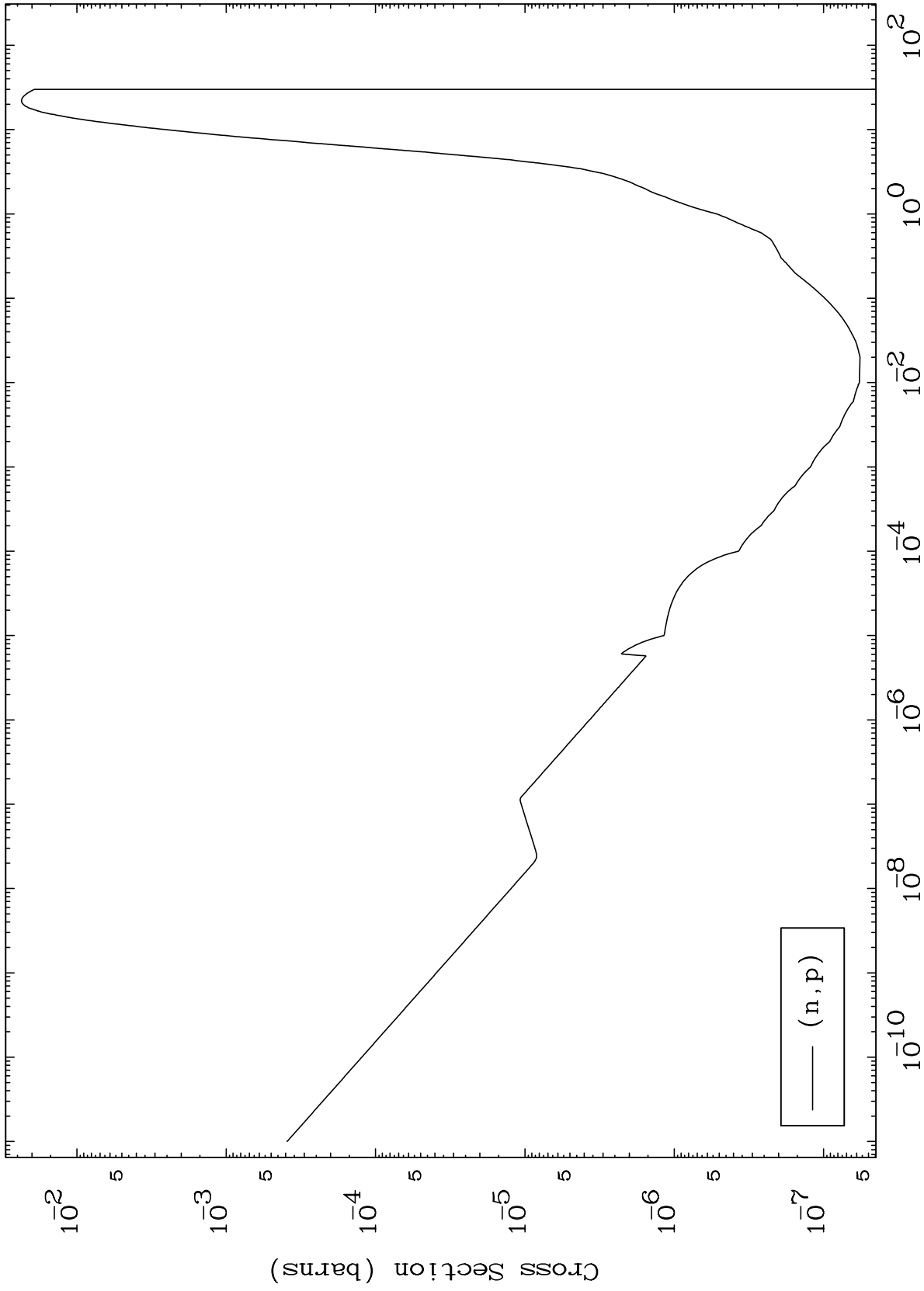
Incident Energy (MeV)

98-Cf-241

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(n,p) Levels  
293 Kelvin Cross Sections

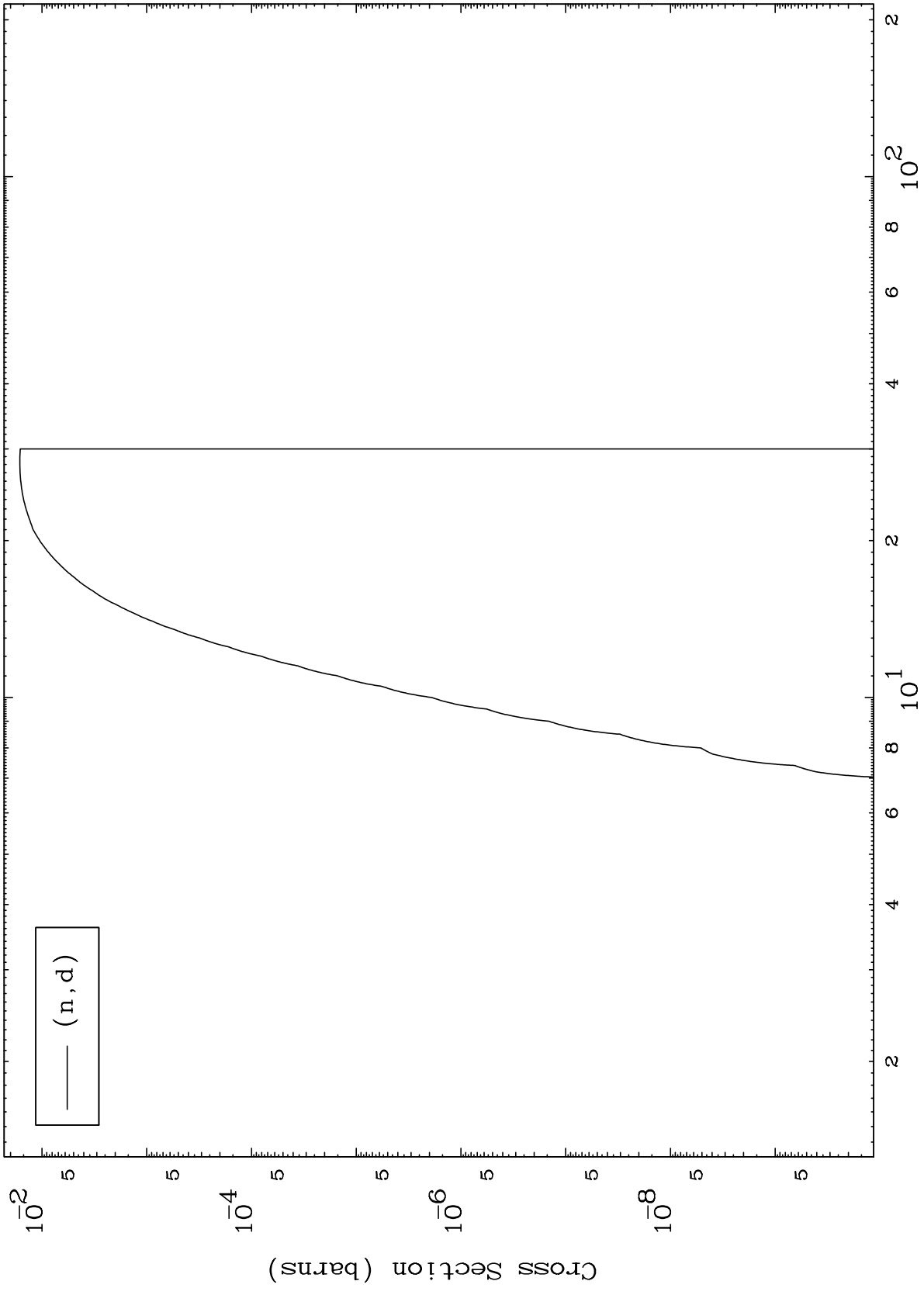
98-Cf-241



MAT 9828

(n,d) Levels  
293 Kelvin Cross Sections

98-Cf-241



12

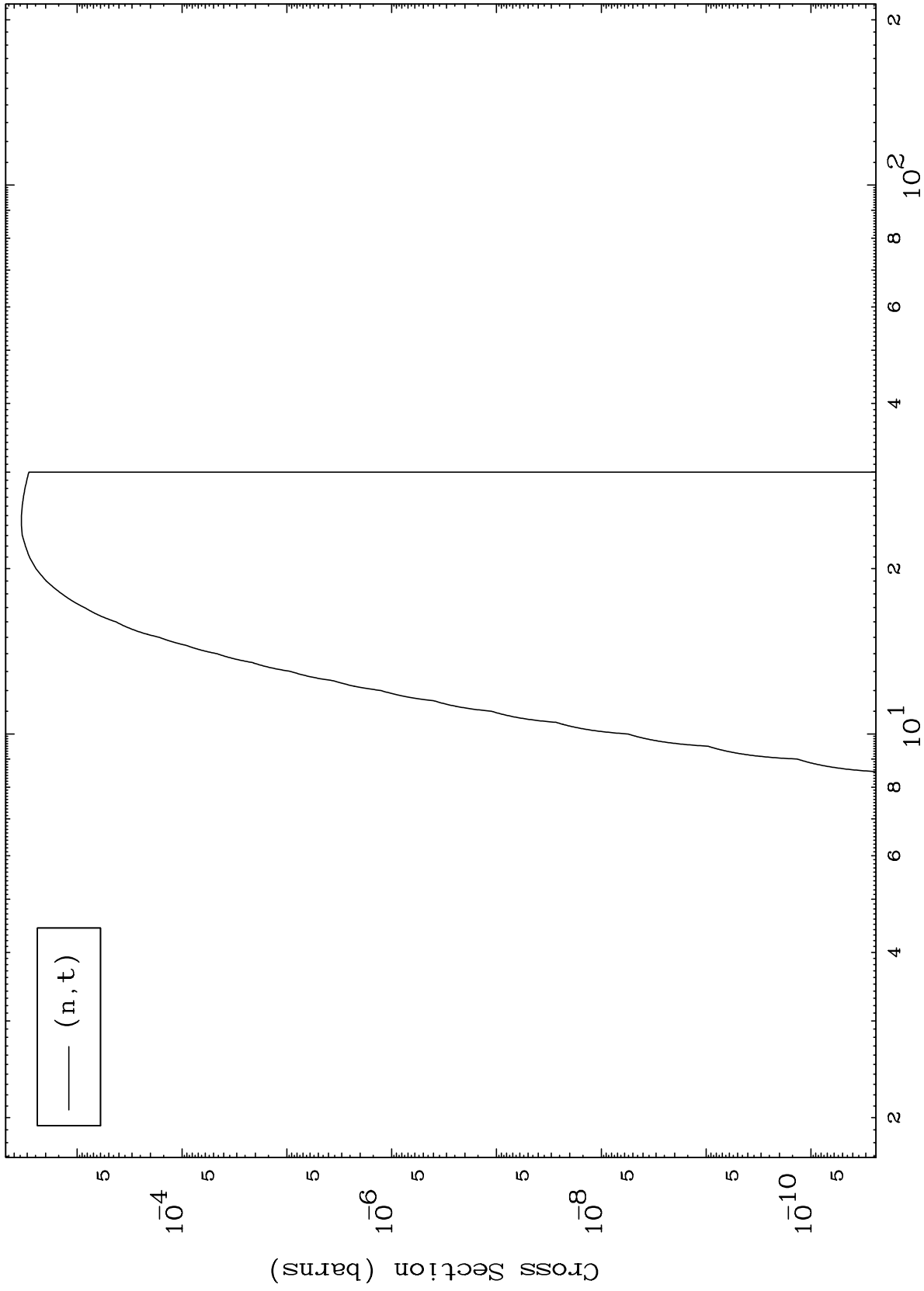
Incident Energy (MeV)

98-Cf-241

MAT 9828

(n,t) Levels  
293 Kelvin Cross Sections

98-Cf-241



13

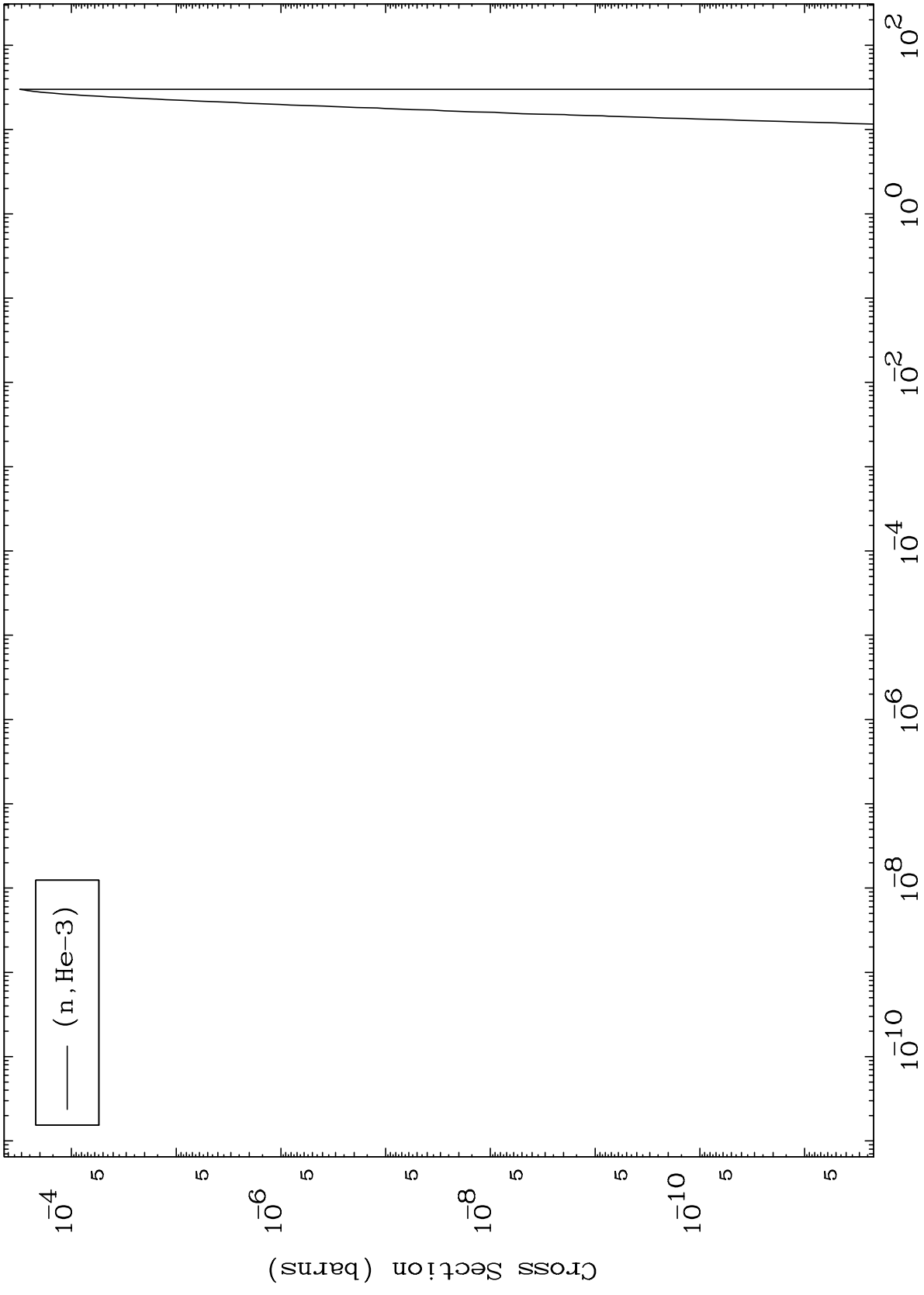
Incident Energy (MeV)

98-Cf-241

MAT 9828

(n,He3) Levels  
293 Kelvin Cross Sections

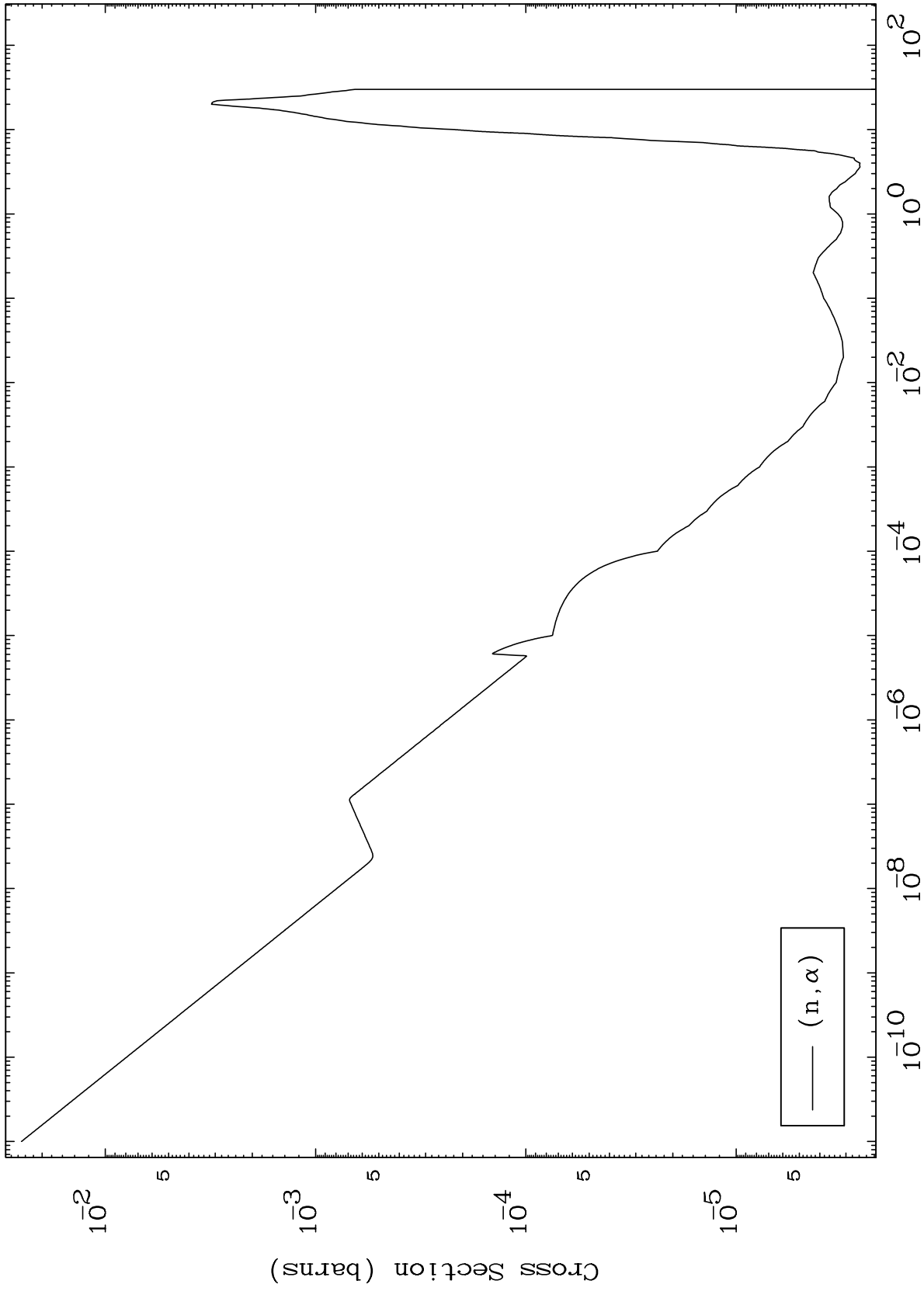
98-Cf-241



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(n, $\alpha$ ) Levels  
293 Kelvin Cross Sections

98-Cf-241



15

Incident Energy (MeV)

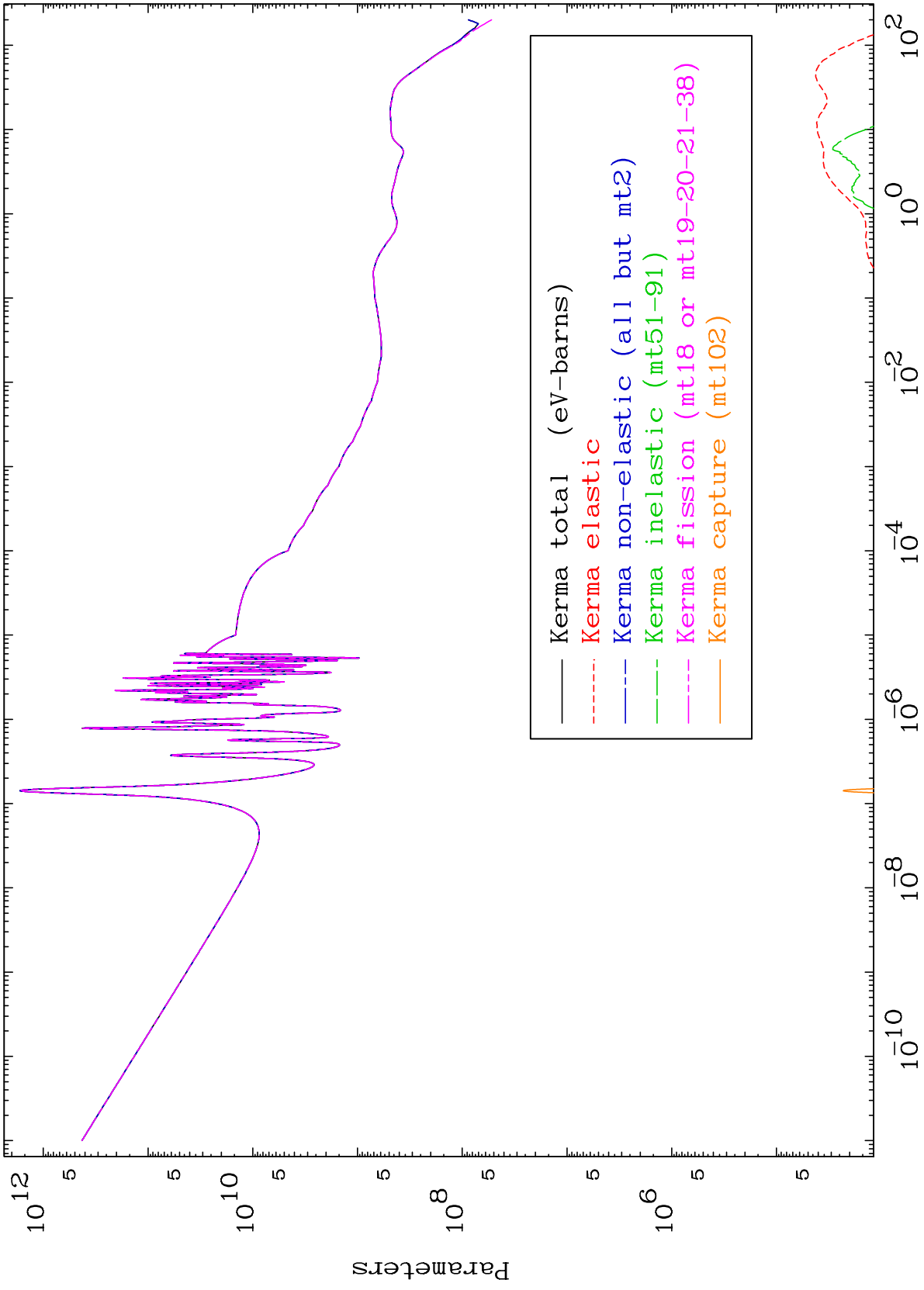
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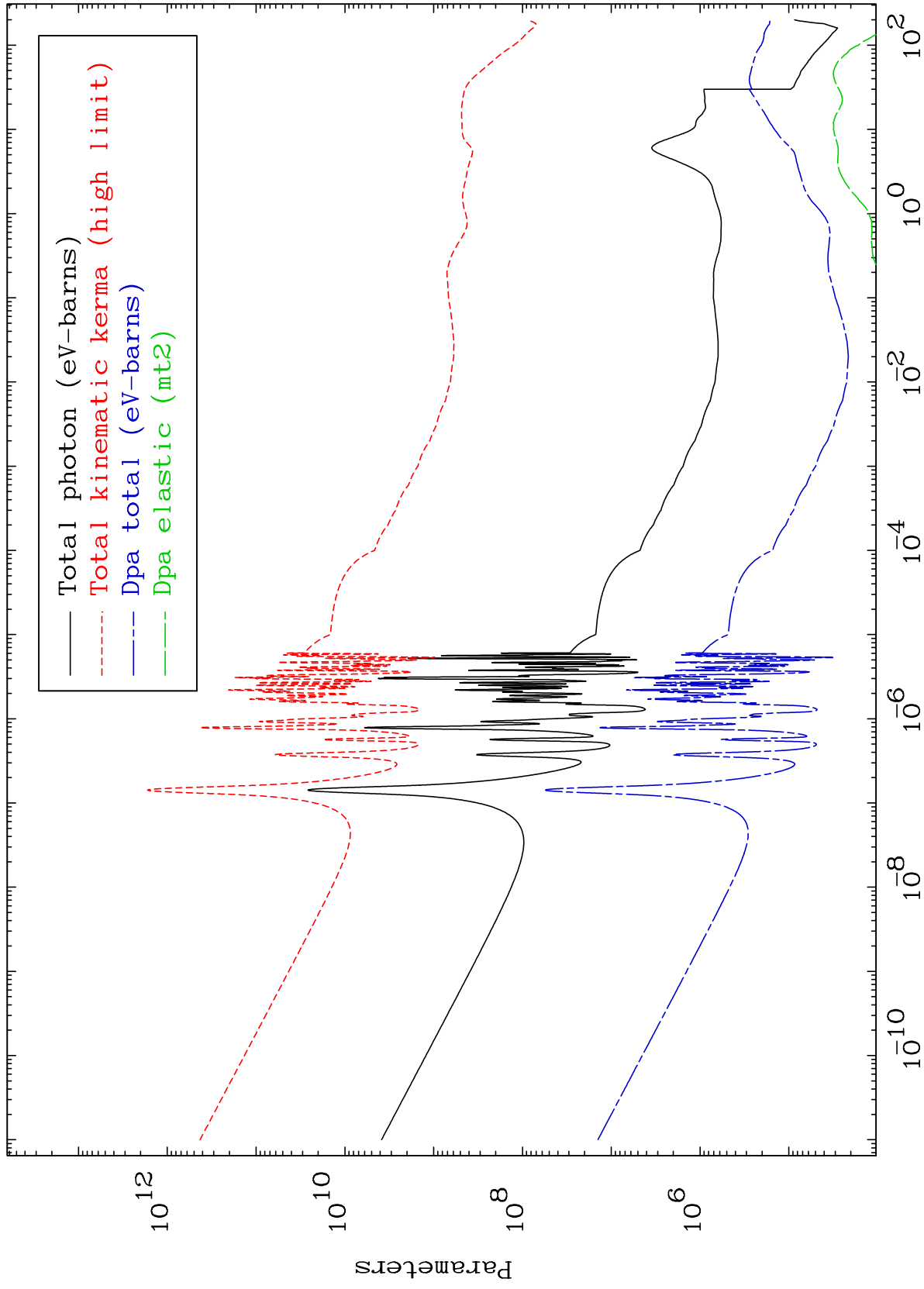


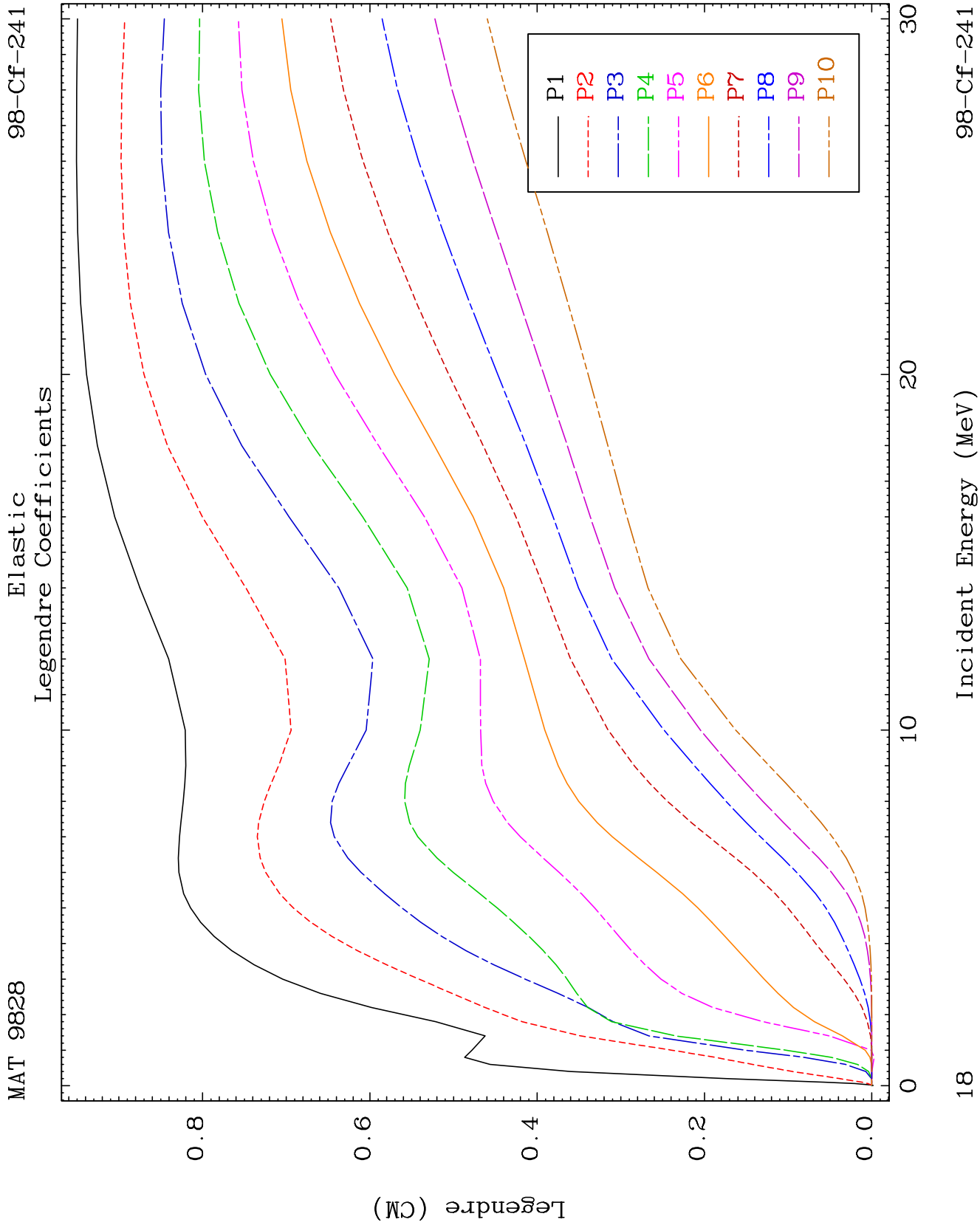
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Energy Release  
Parameters

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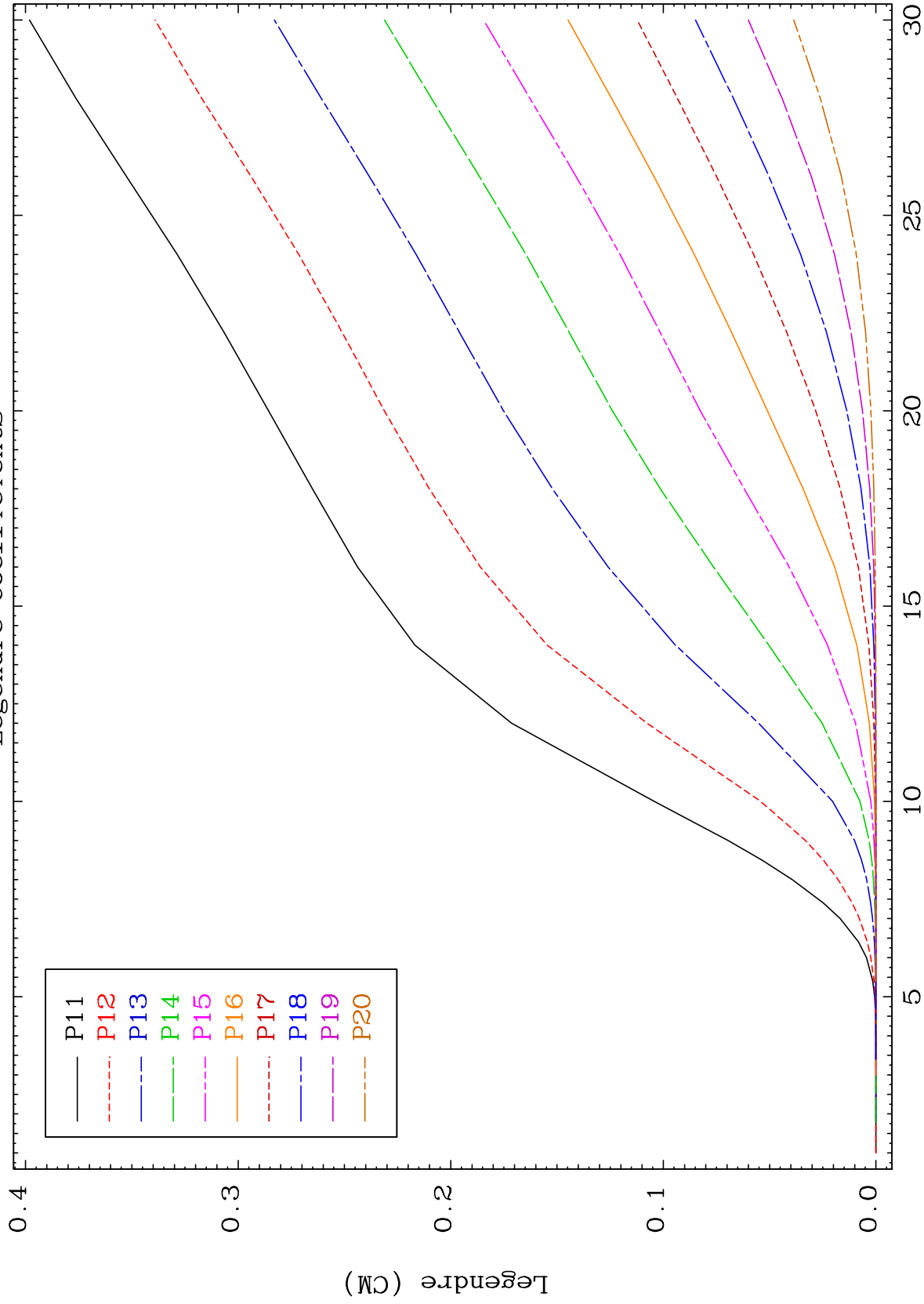




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Elastic Legendre Coefficients

98-Cf-241



19

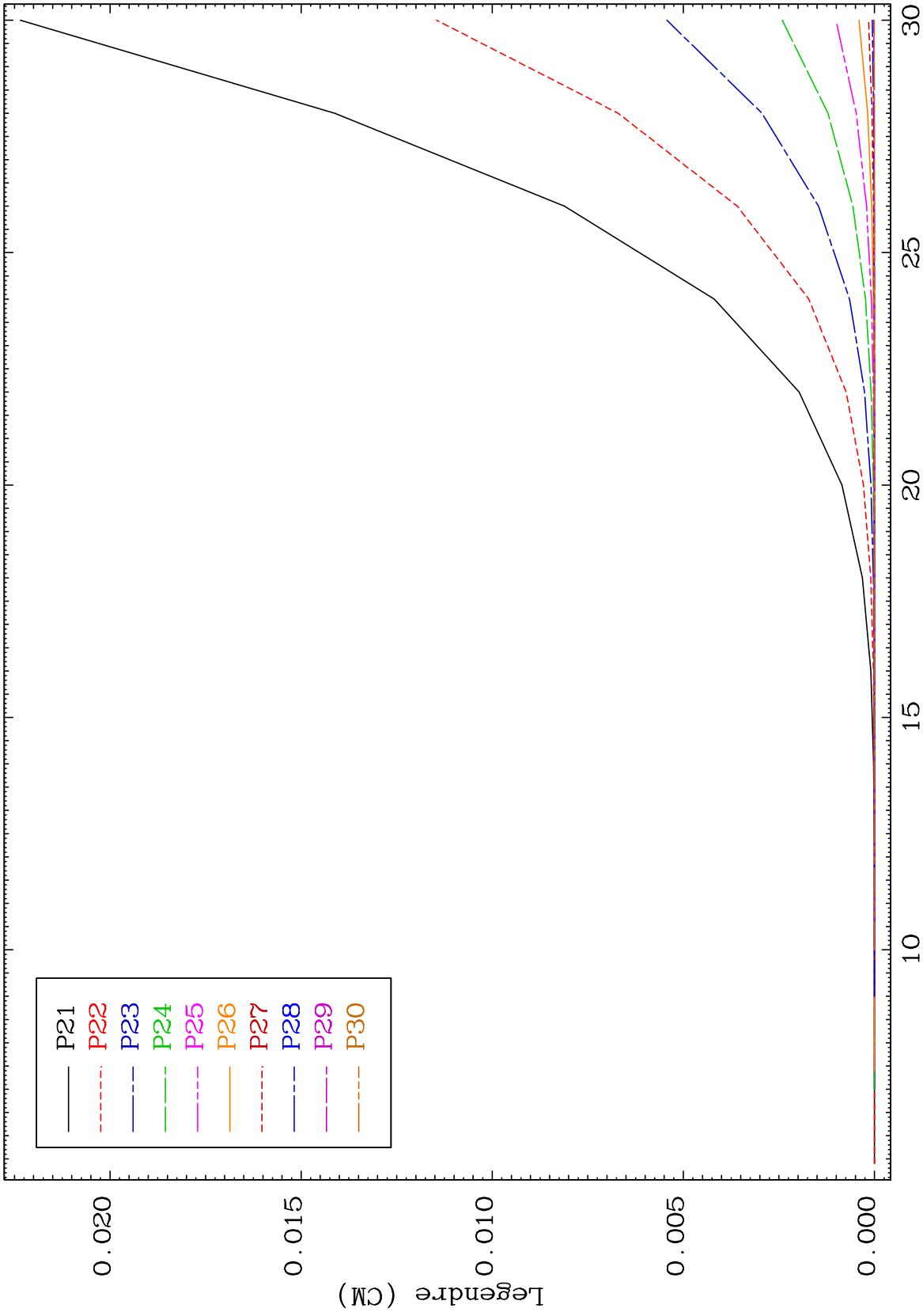
Incident Energy (MeV)

98-Cf-241

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### Elastic Legendre Coefficients

98-Cf-241



98-Cf-241

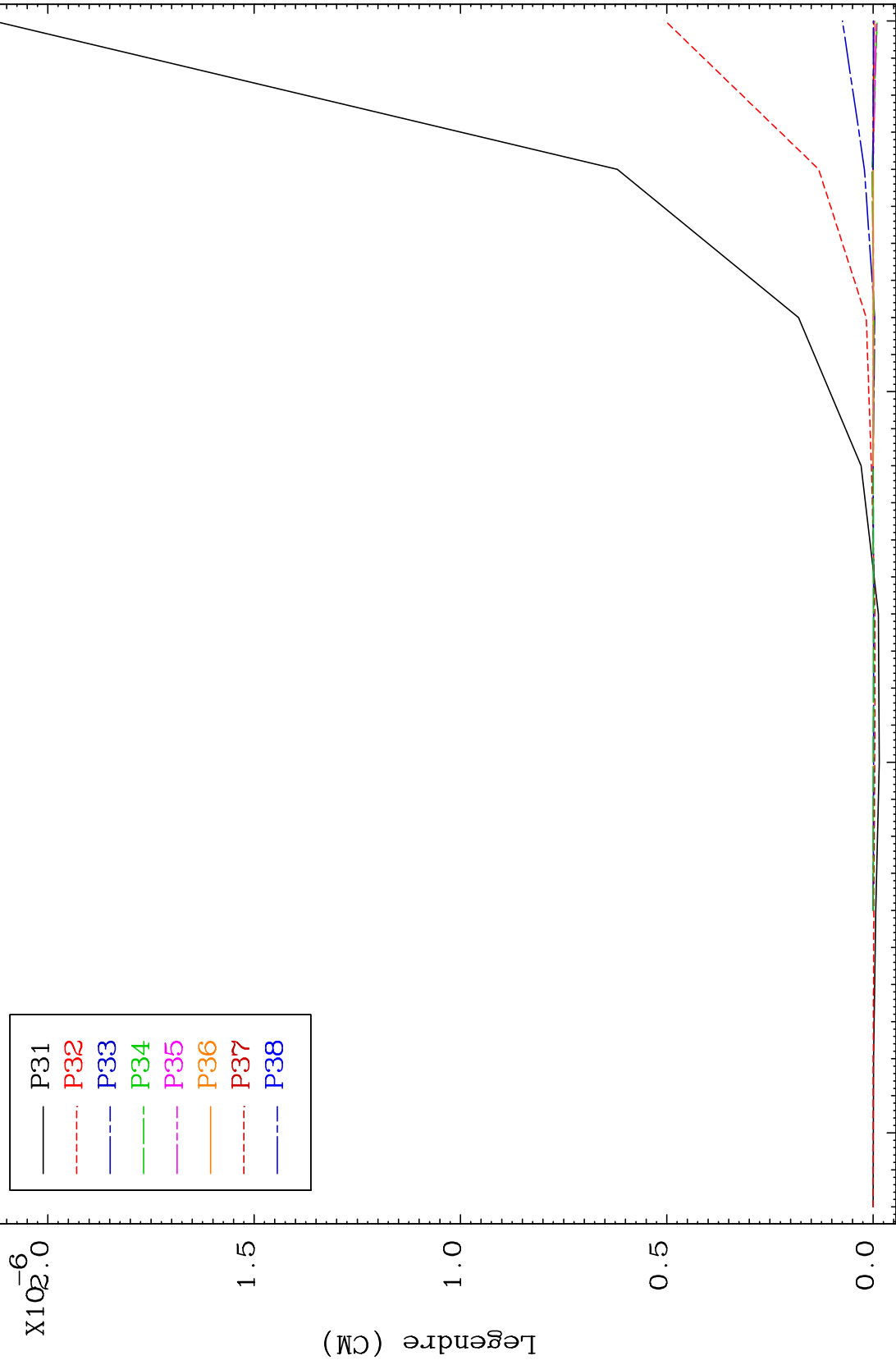
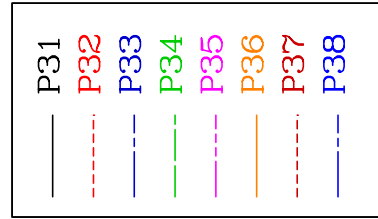
Incident Energy (MeV)

20

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Elastic Legendre Coefficients

98-Cf-241



21

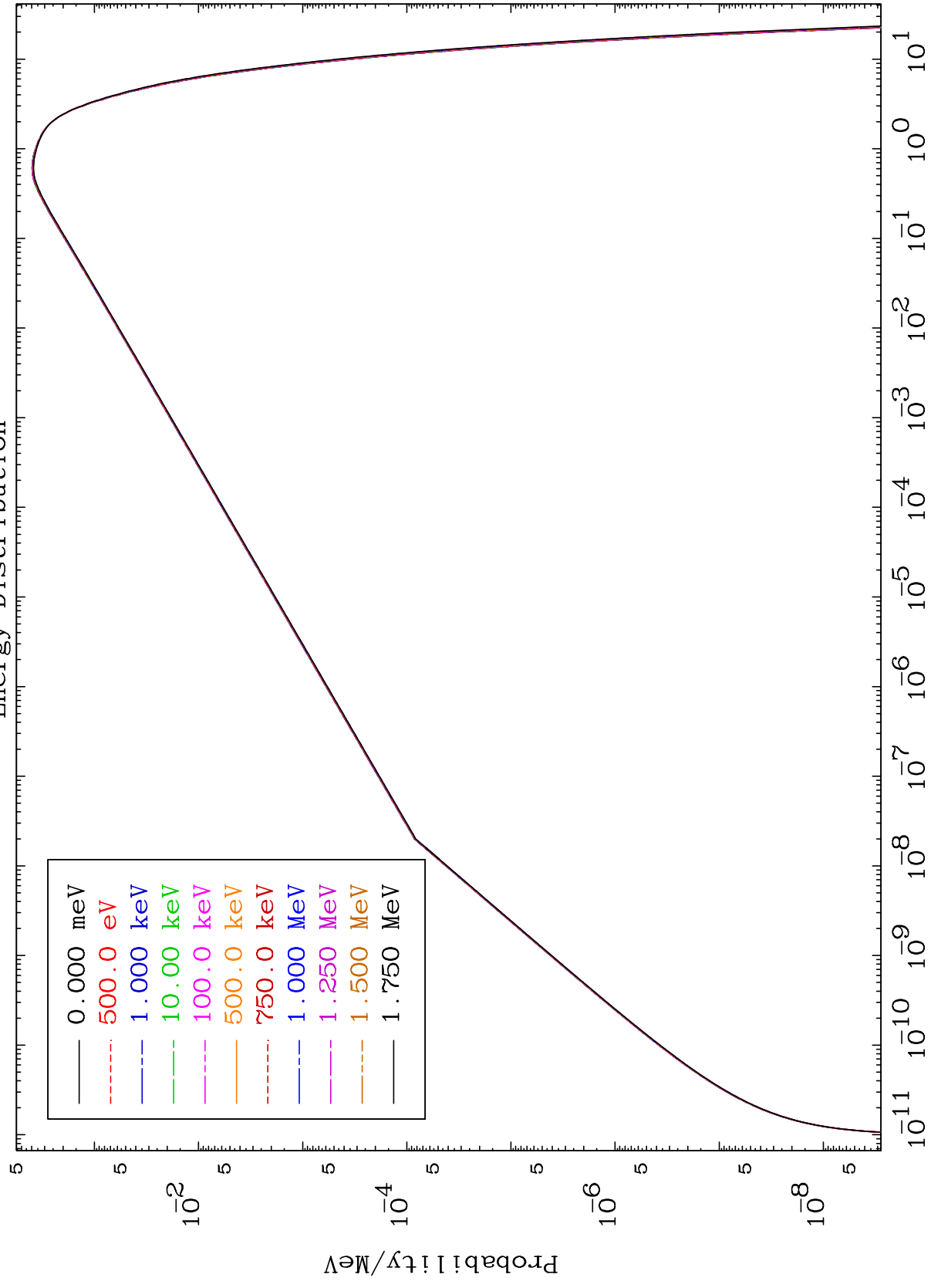
Incident Energy (MeV)

98-Cf-241

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### Fission Energy Distribution

98-Cf-241



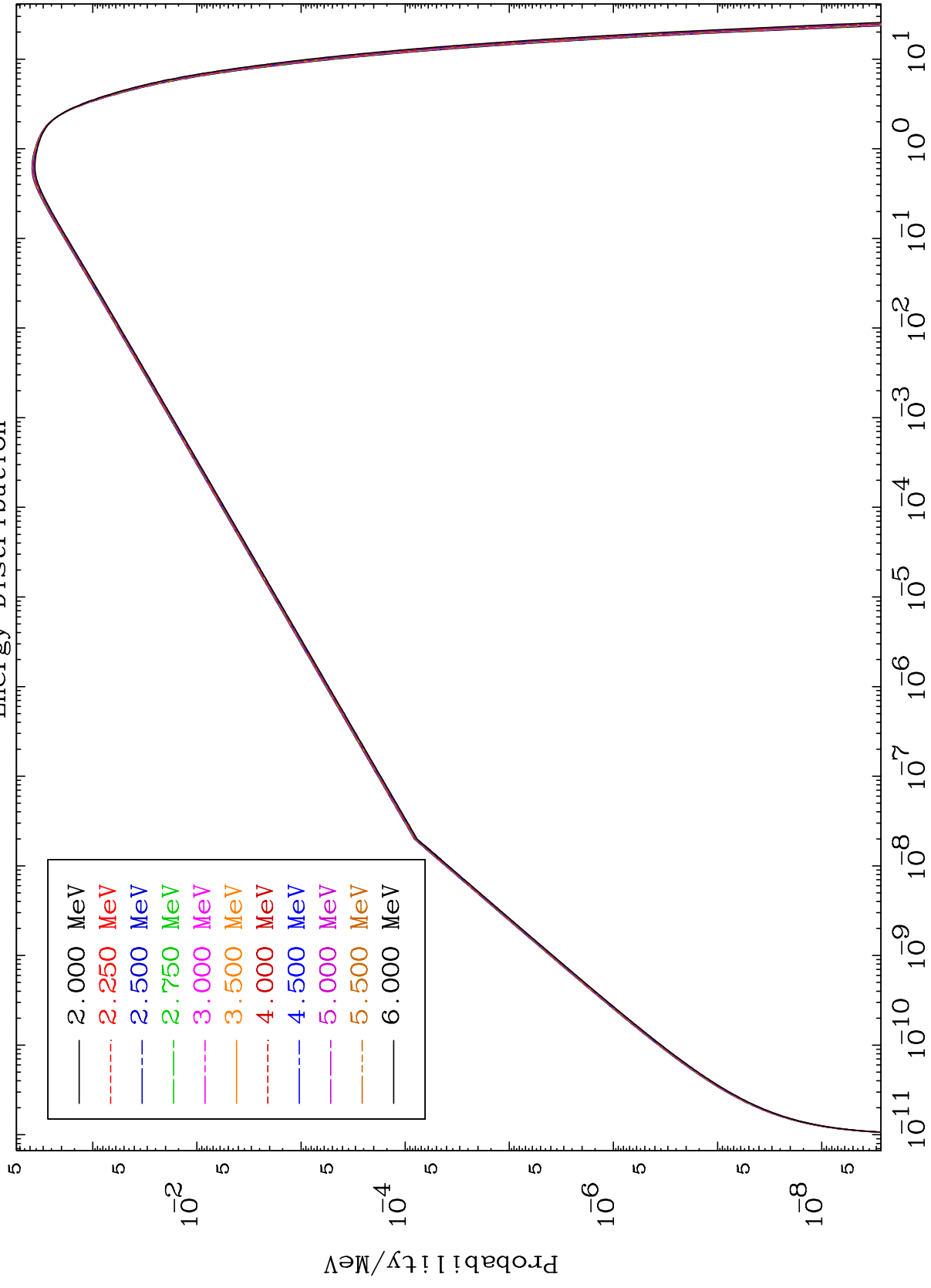
98-Cf-241

Secondary Energy (MeV)

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### Fission Energy Distribution

98-Cf-241



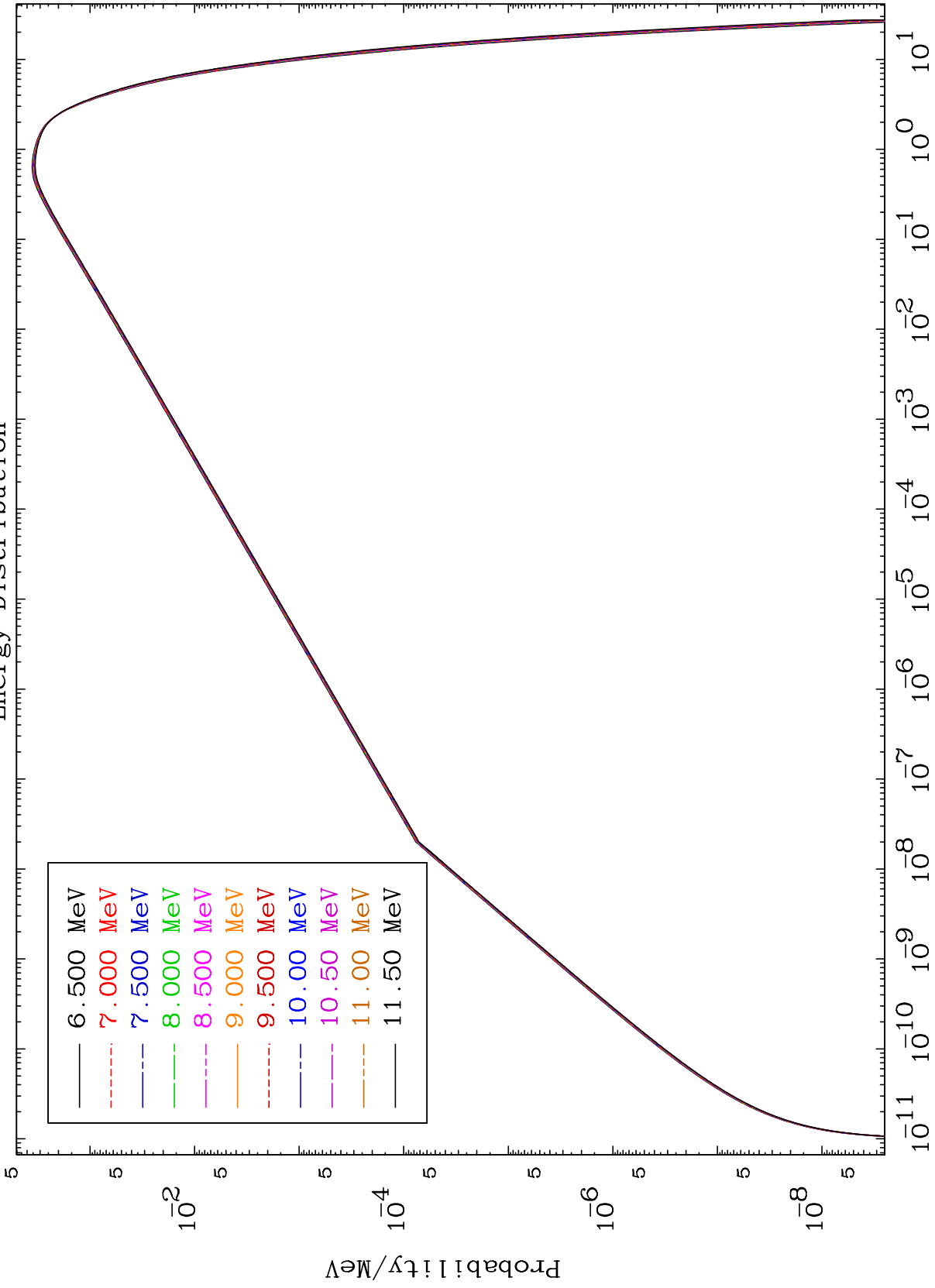
98-Cf-241



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### Fission Energy Distribution

98-Cf-241

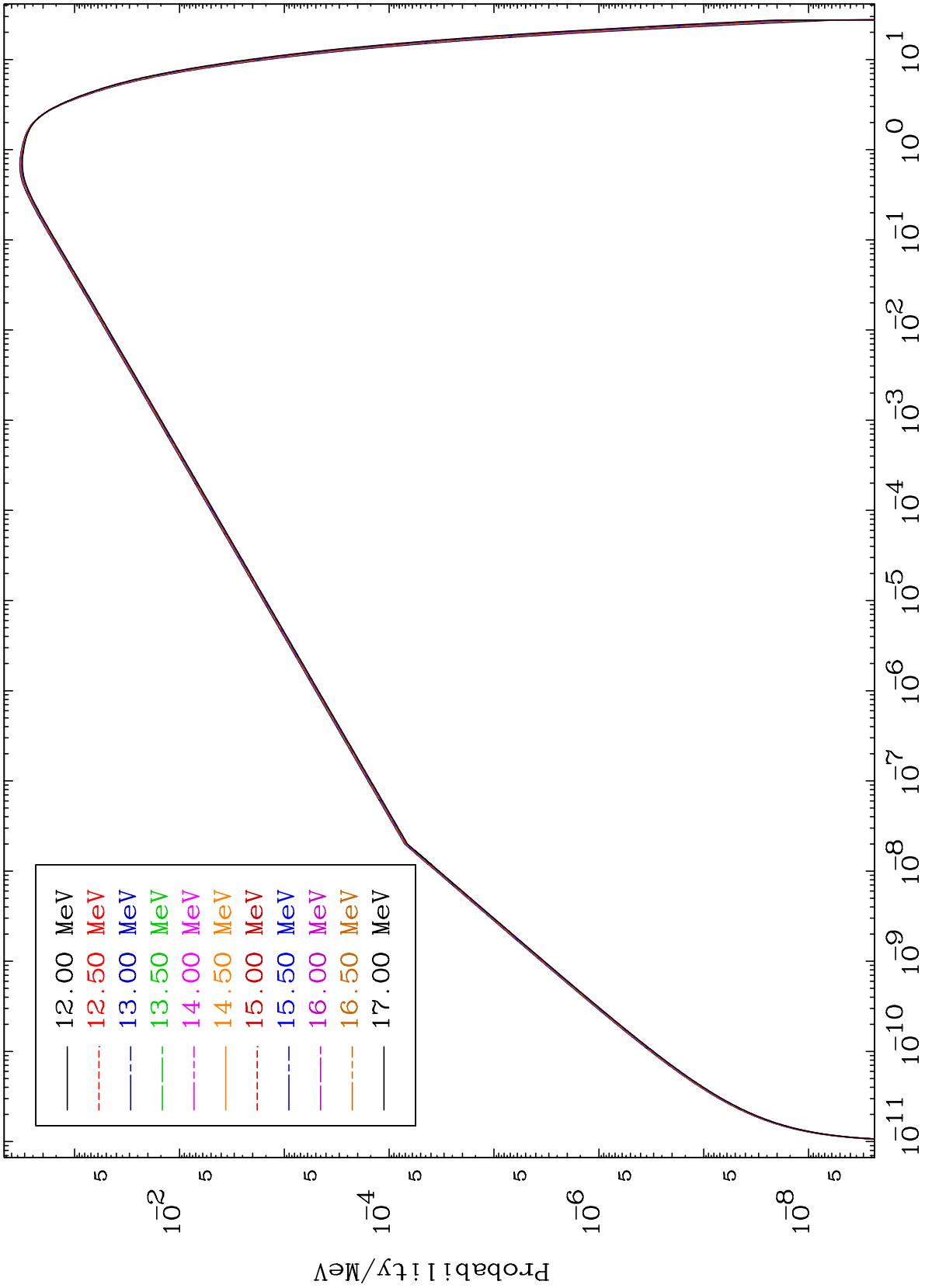


98-Cf-241

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### Fission Energy Distribution

98-Cf-241



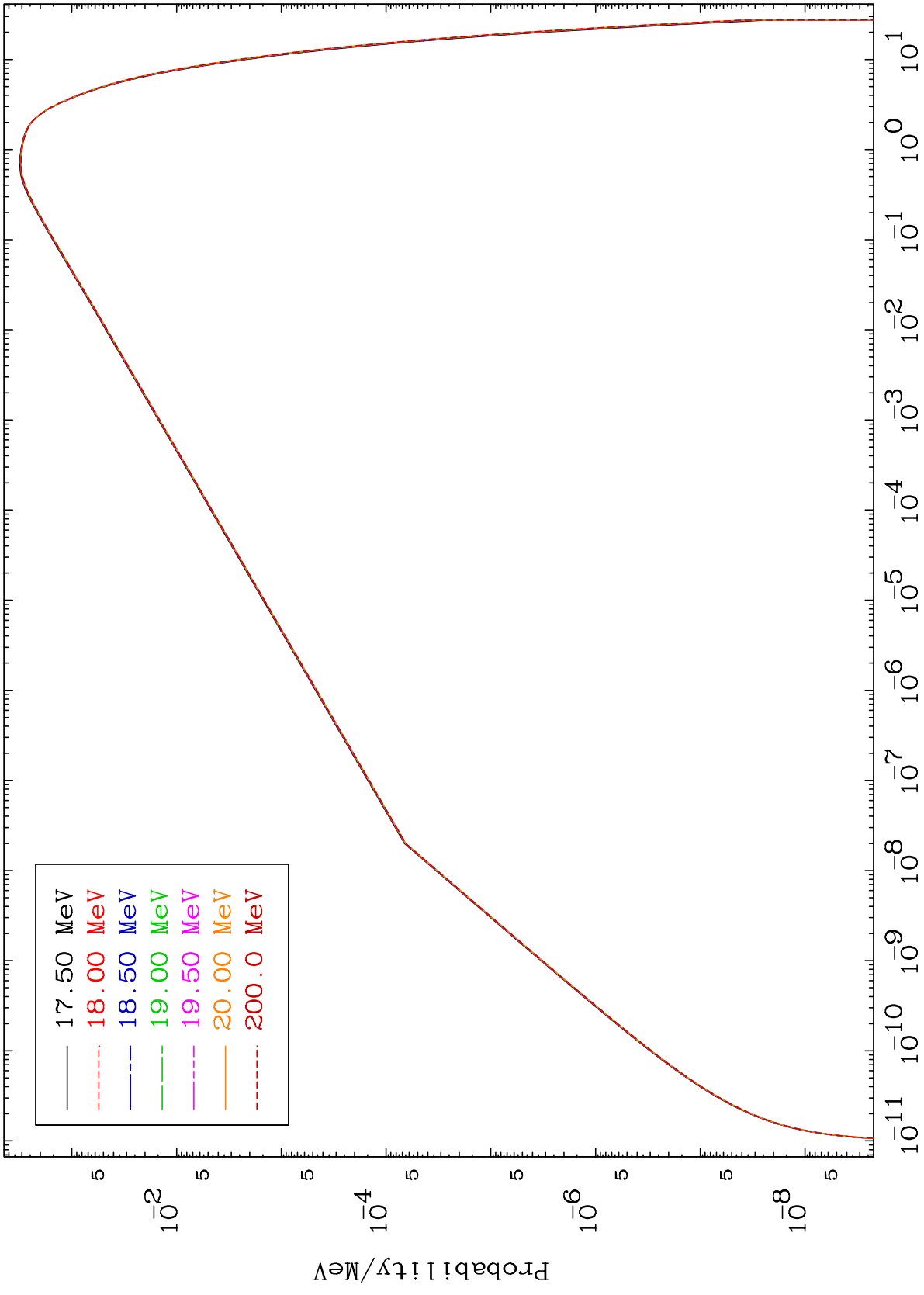
98-Cf-241

Secondary Energy (MeV)

MAT 9828

### Fission Energy Distribution

98-Cf-241



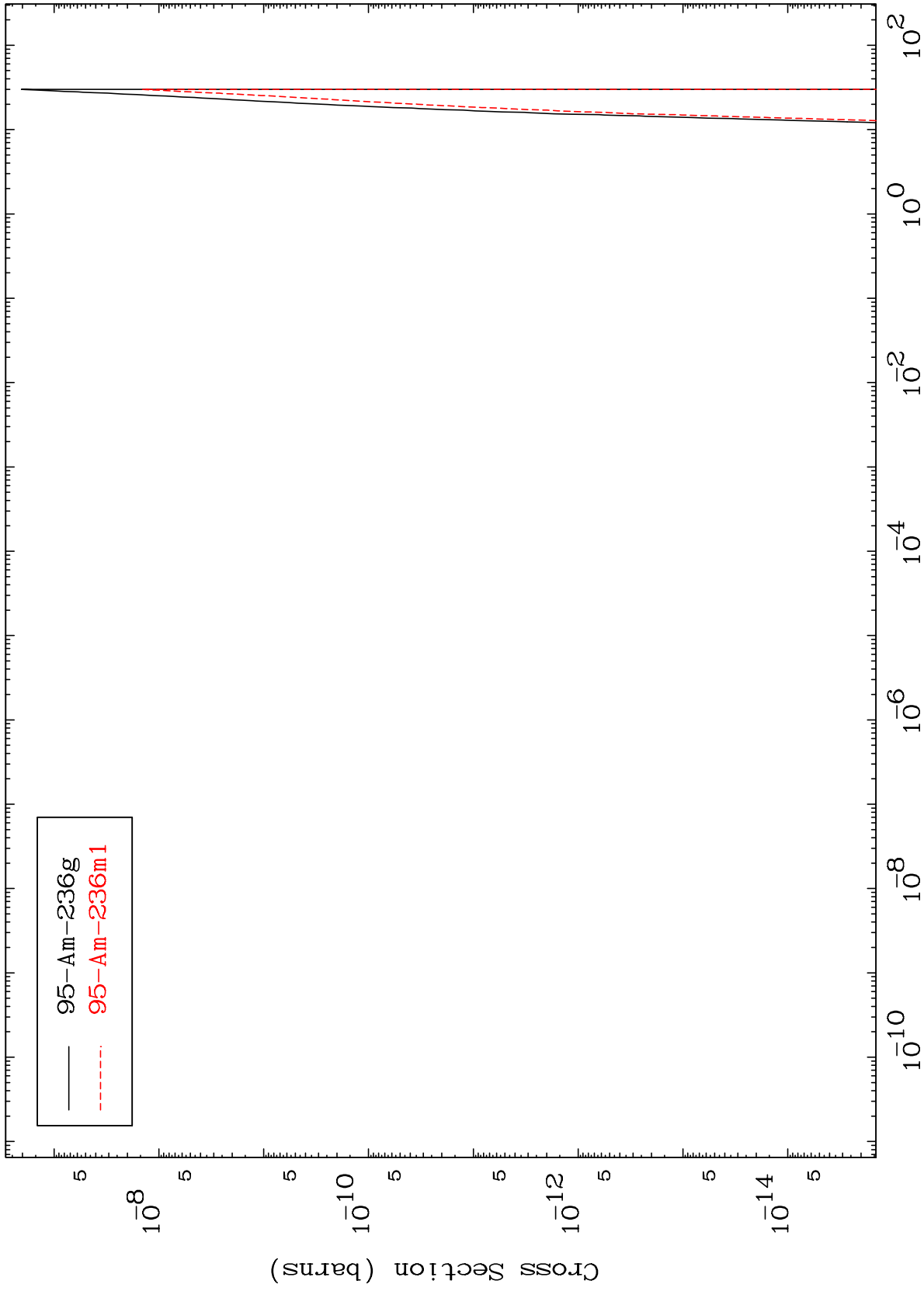
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(n,n') p  $\alpha$

98-Cf-241

Radionuclide Production Cross Section



27

Incident Energy (MeV)

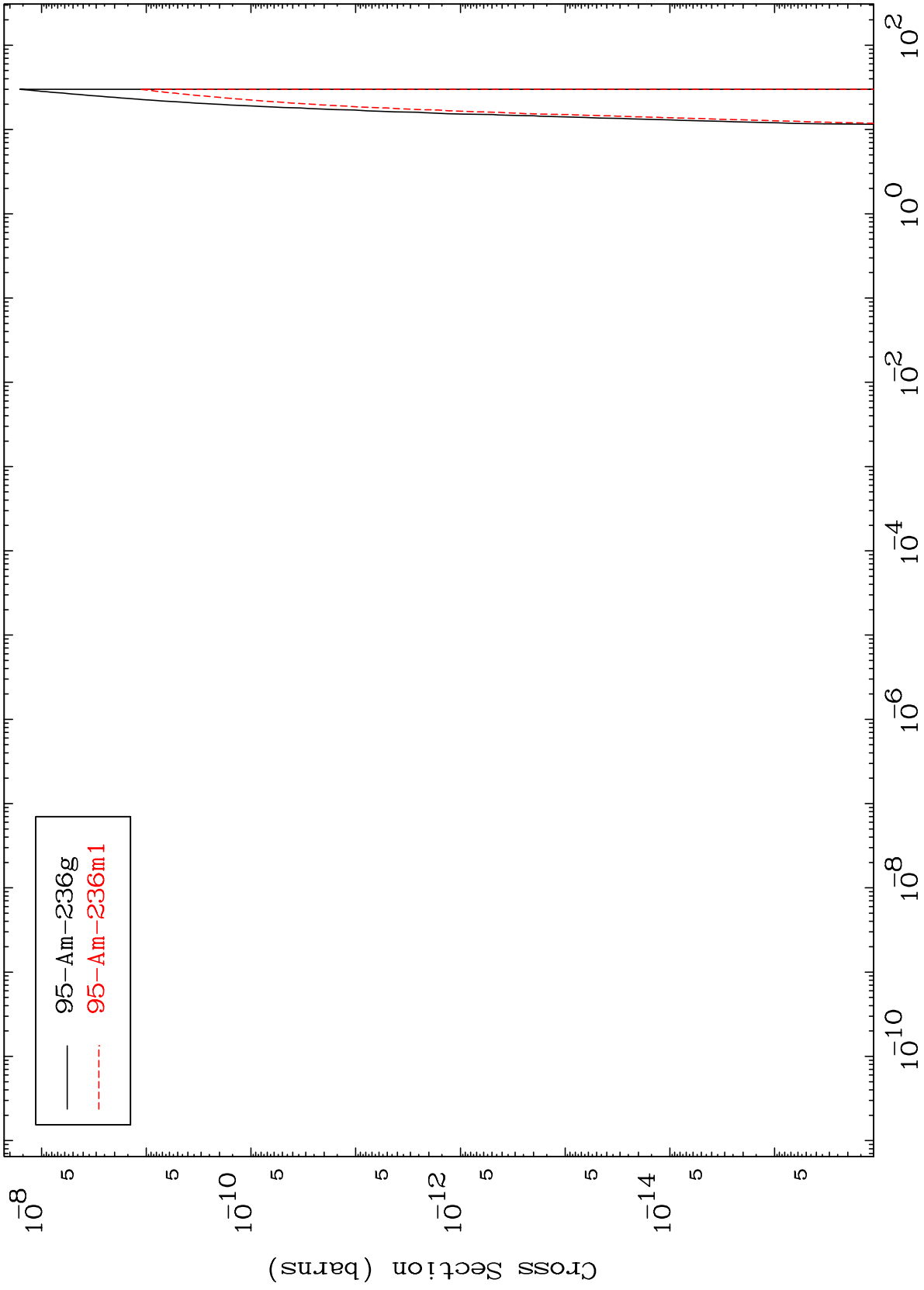
98-Cf-241

MAT 9828

(n,d)  $\alpha$

98-Cf-241

Radionuclide Production Cross Section



28

Incident Energy (MeV)

98-Cf-241