

Program EVALPLOT
(Version 2018-1)

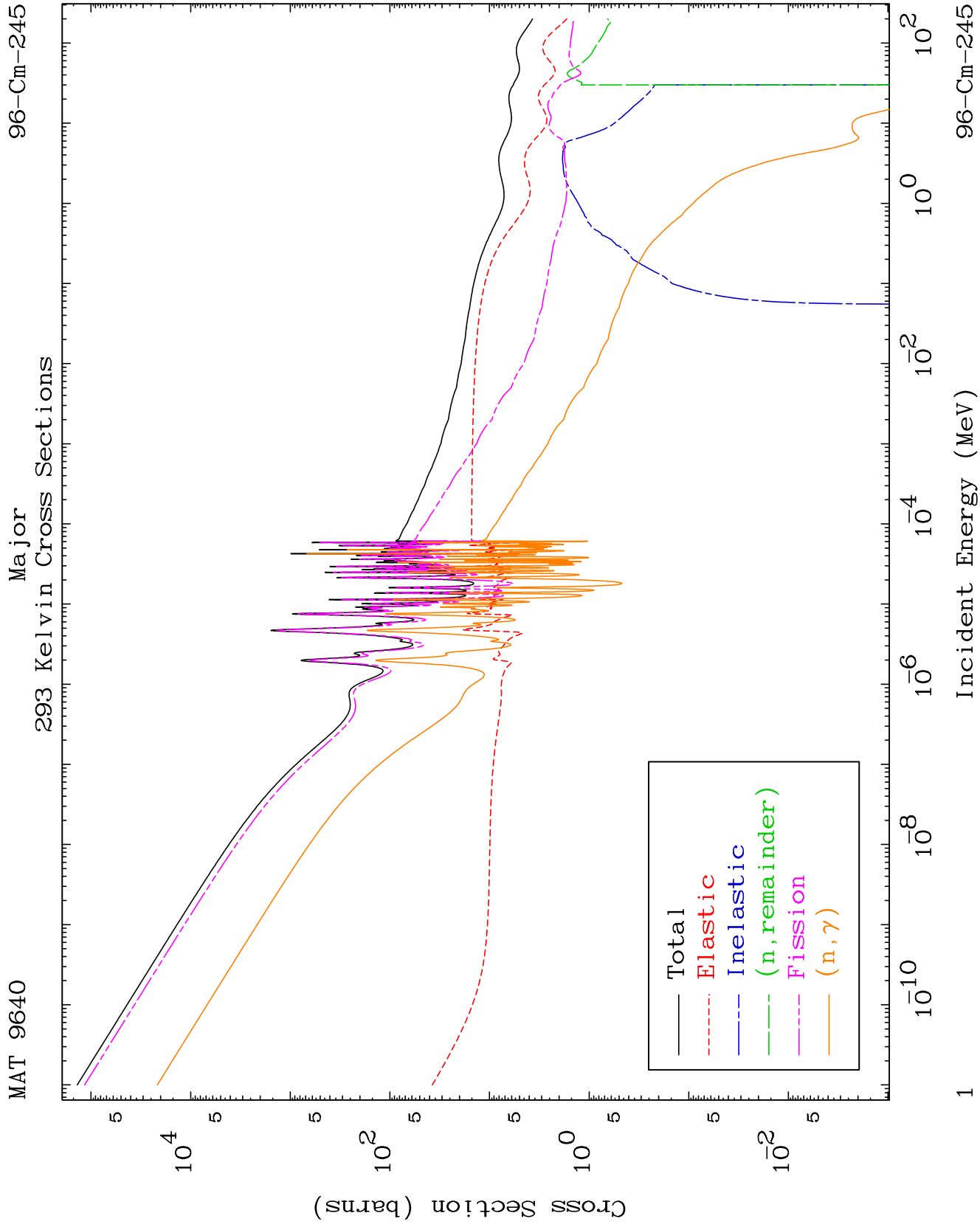
by

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Web: redcullen1.net/HOMEPAGE.NEW

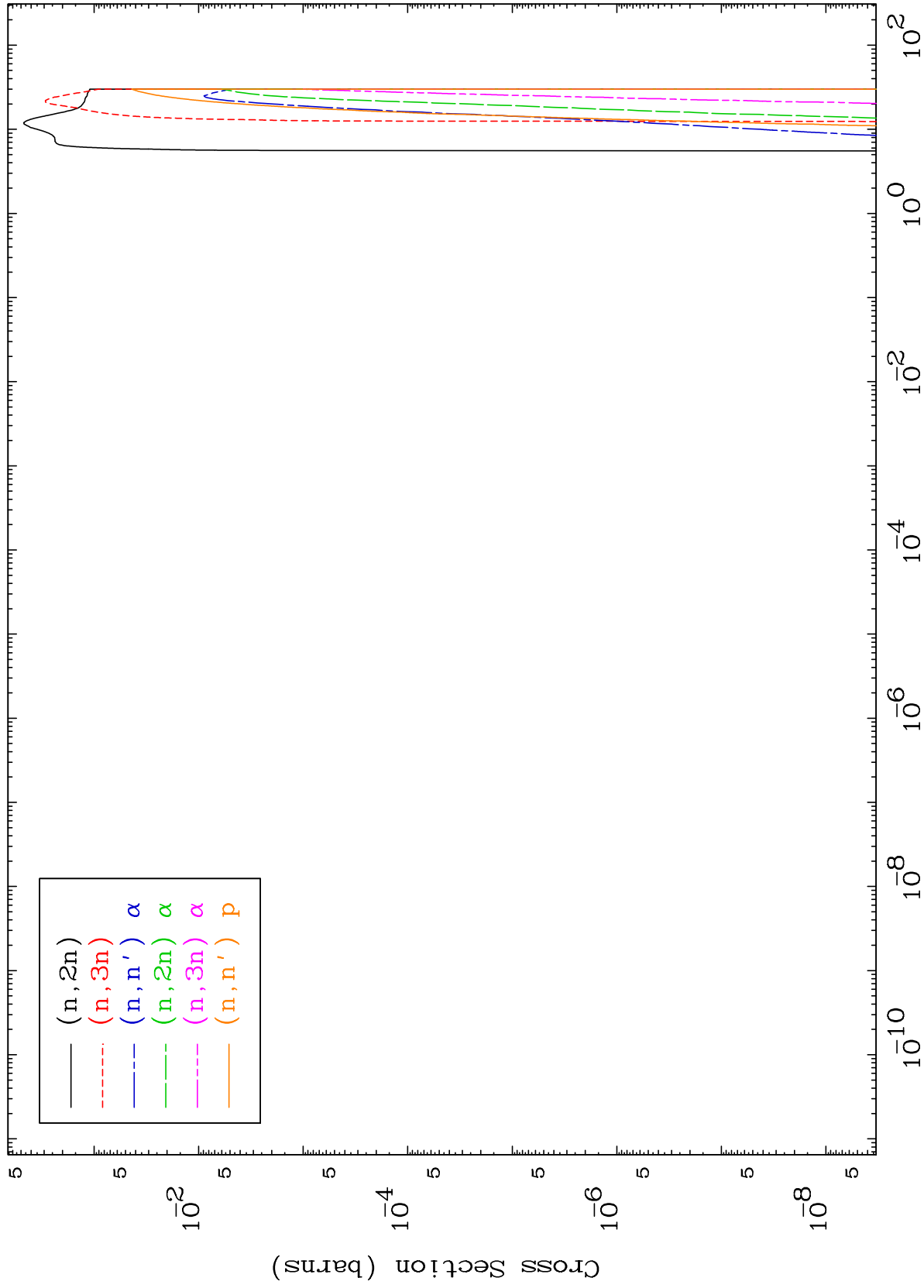
Press Mouse Button to Start



MAT 9640

Neutron Production
293 Kelvin Cross Sections

96-Cm-245



2

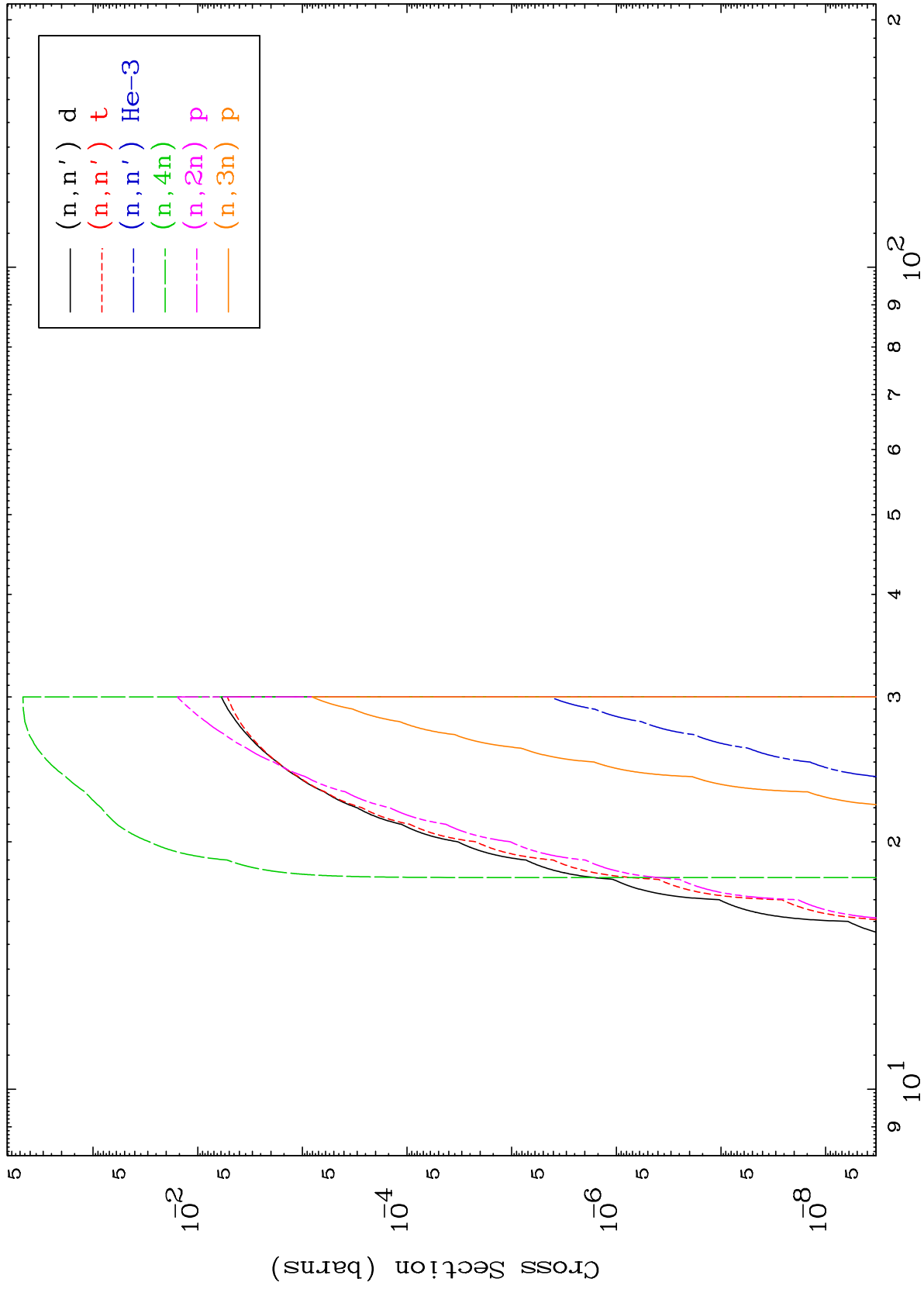
Incident Energy (MeV)

96-Cm-245

MAT 9640

Neutron Production
293 Kelvin Cross Sections

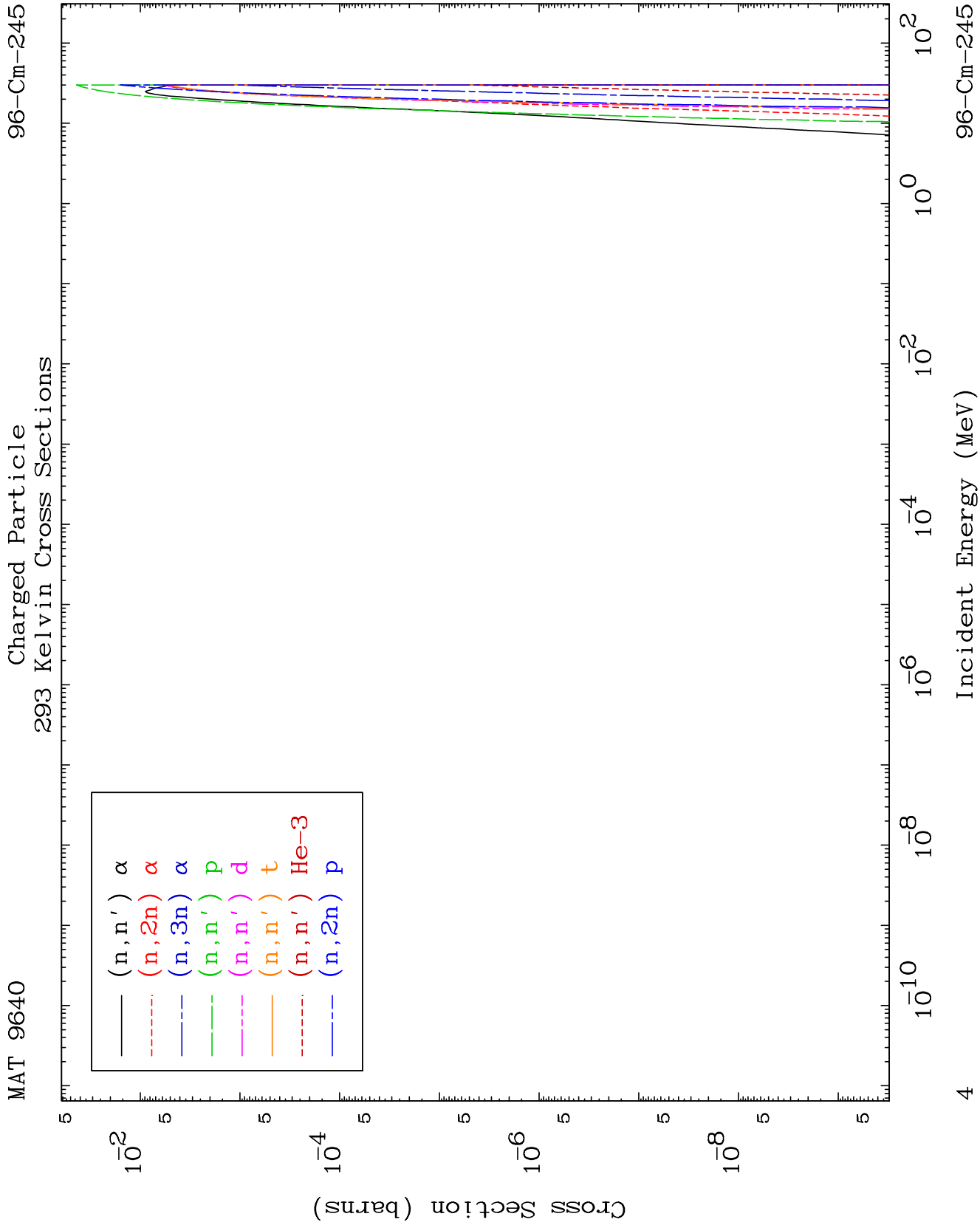
96-Cm-245



3

Incident Energy (MeV)

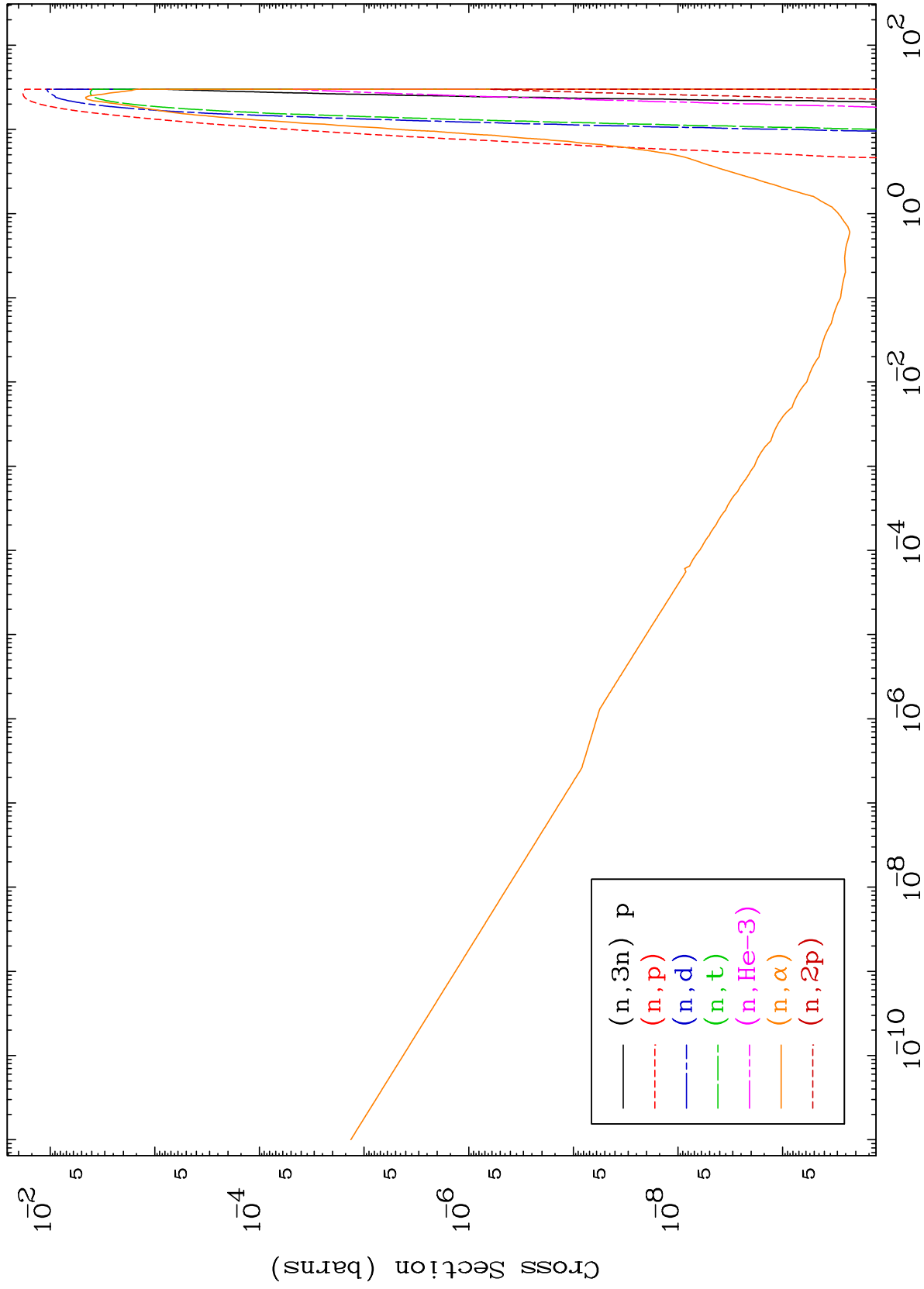
96-Cm-245



MAT 9640

Charged Particle
293 Kelvin Cross Sections

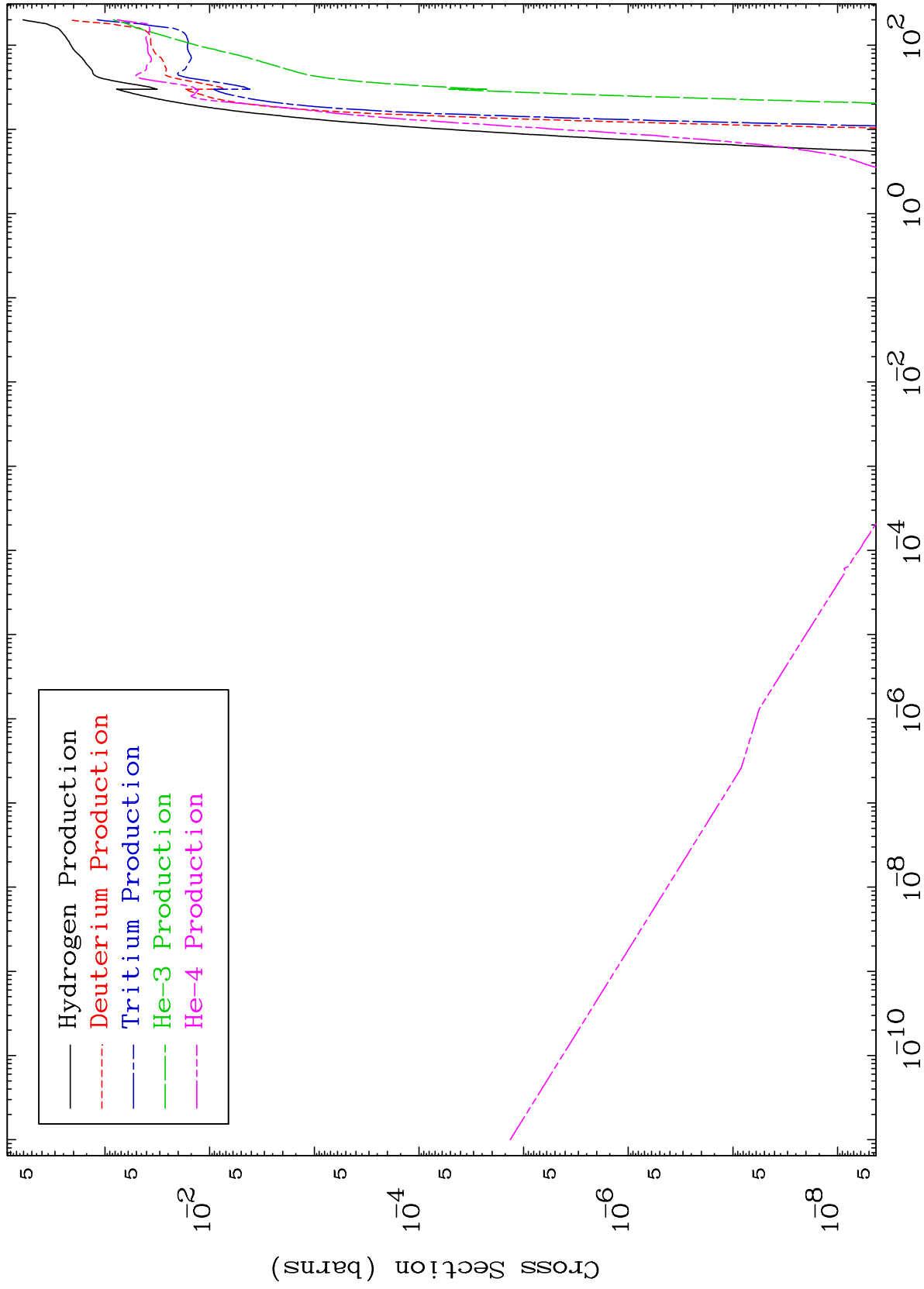
96-Cm-245



MAT 9640

Particle Production
293 Kelvin Cross Sections

96-Cm-245



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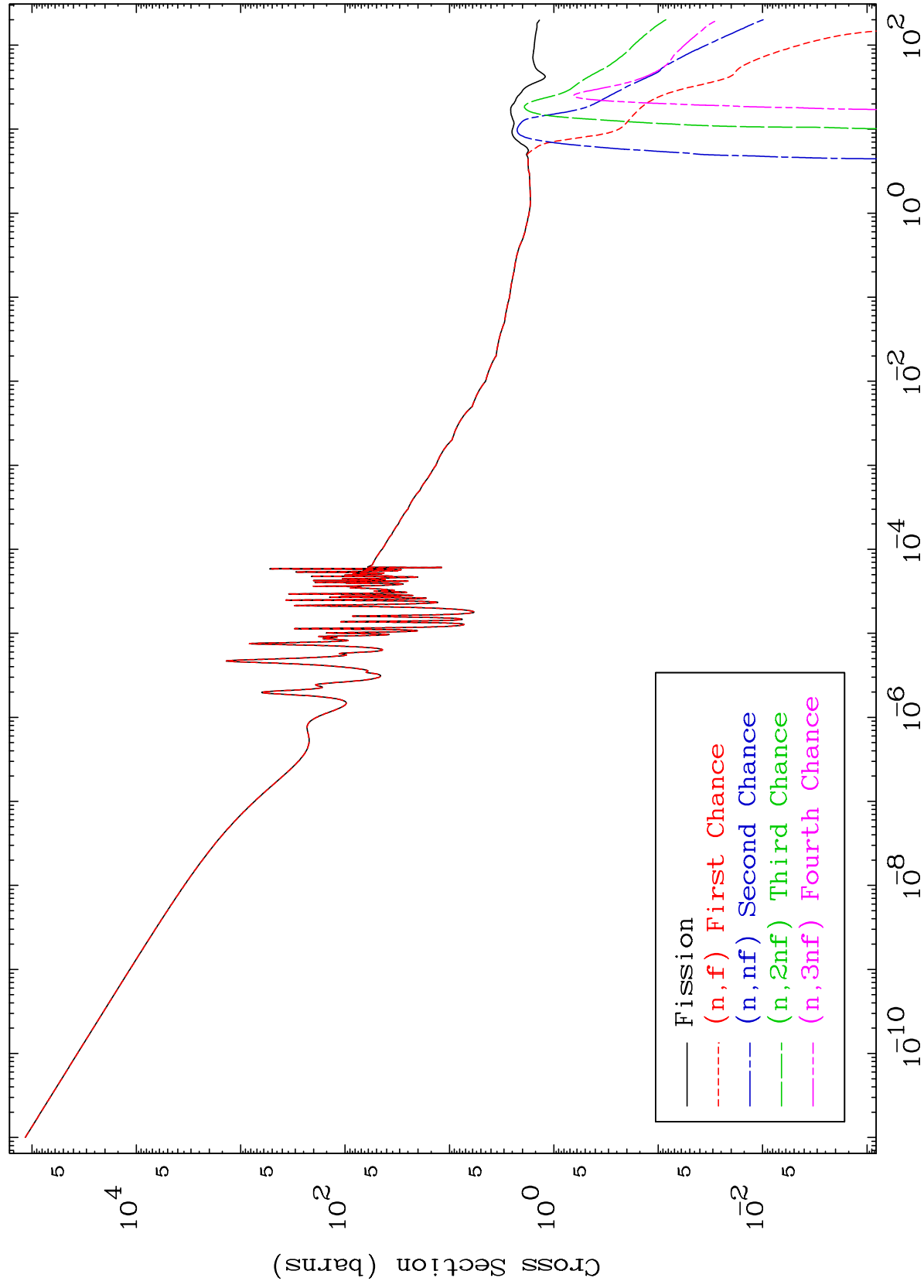
Incident Energy (MeV)

96-Cm-245

MAT 9640

Fission
293 Kelvin Cross Sections

96-Cm-245

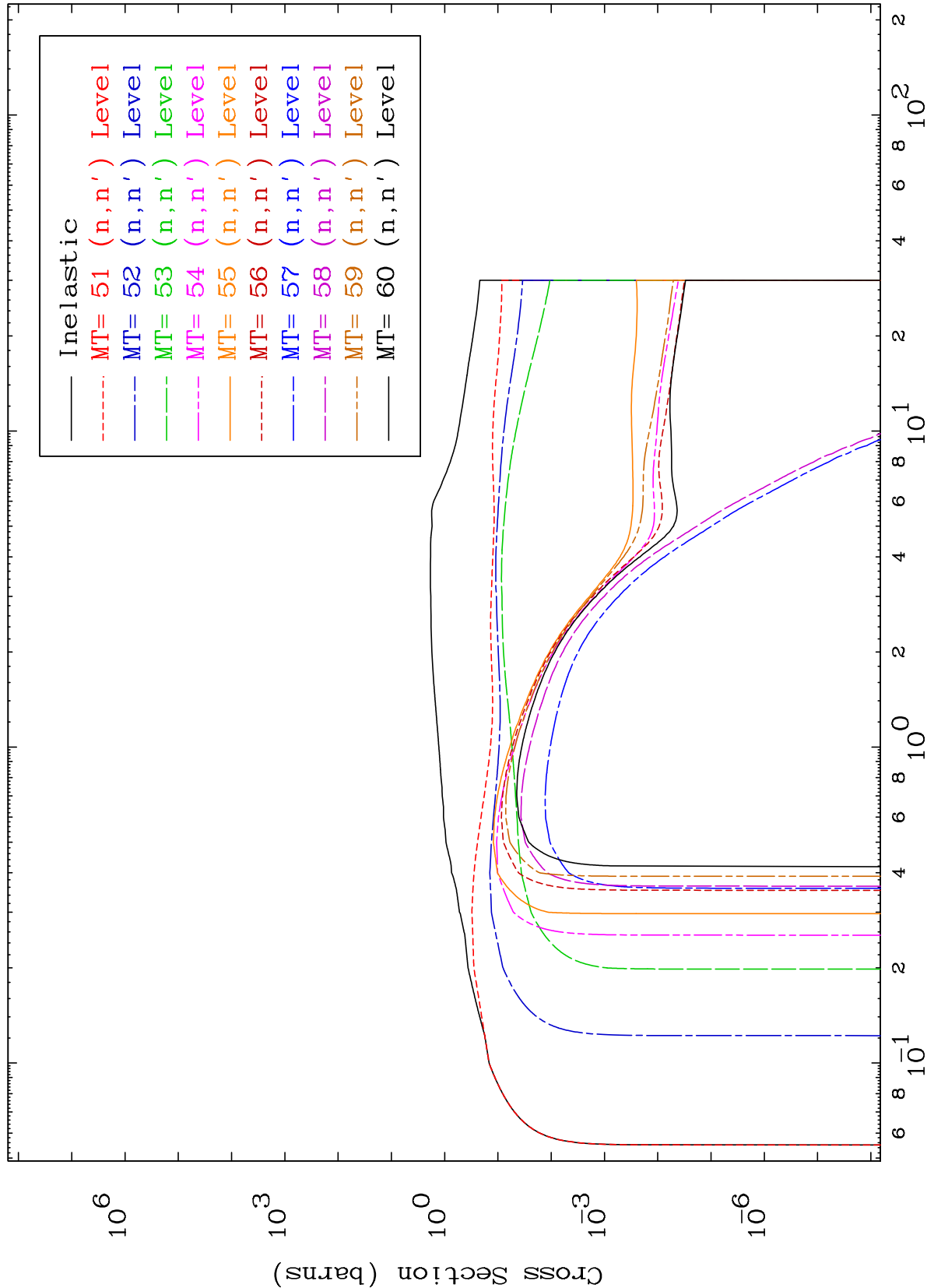


MAT 9640

(n,n') Level

96-Cm-245

293 Kelvin Cross Sections

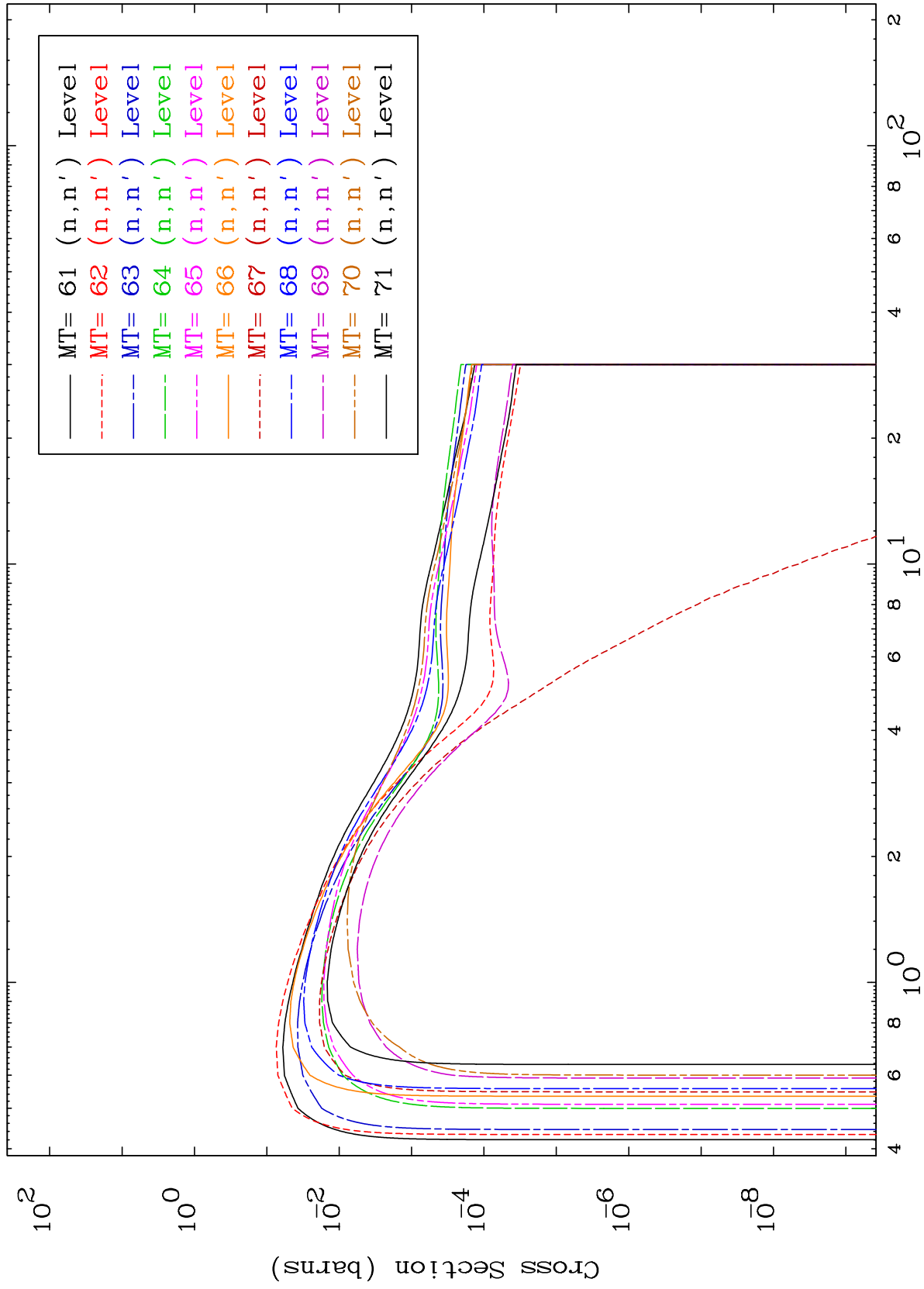


MAT 9640

(n,n') Level

96-Cm-245

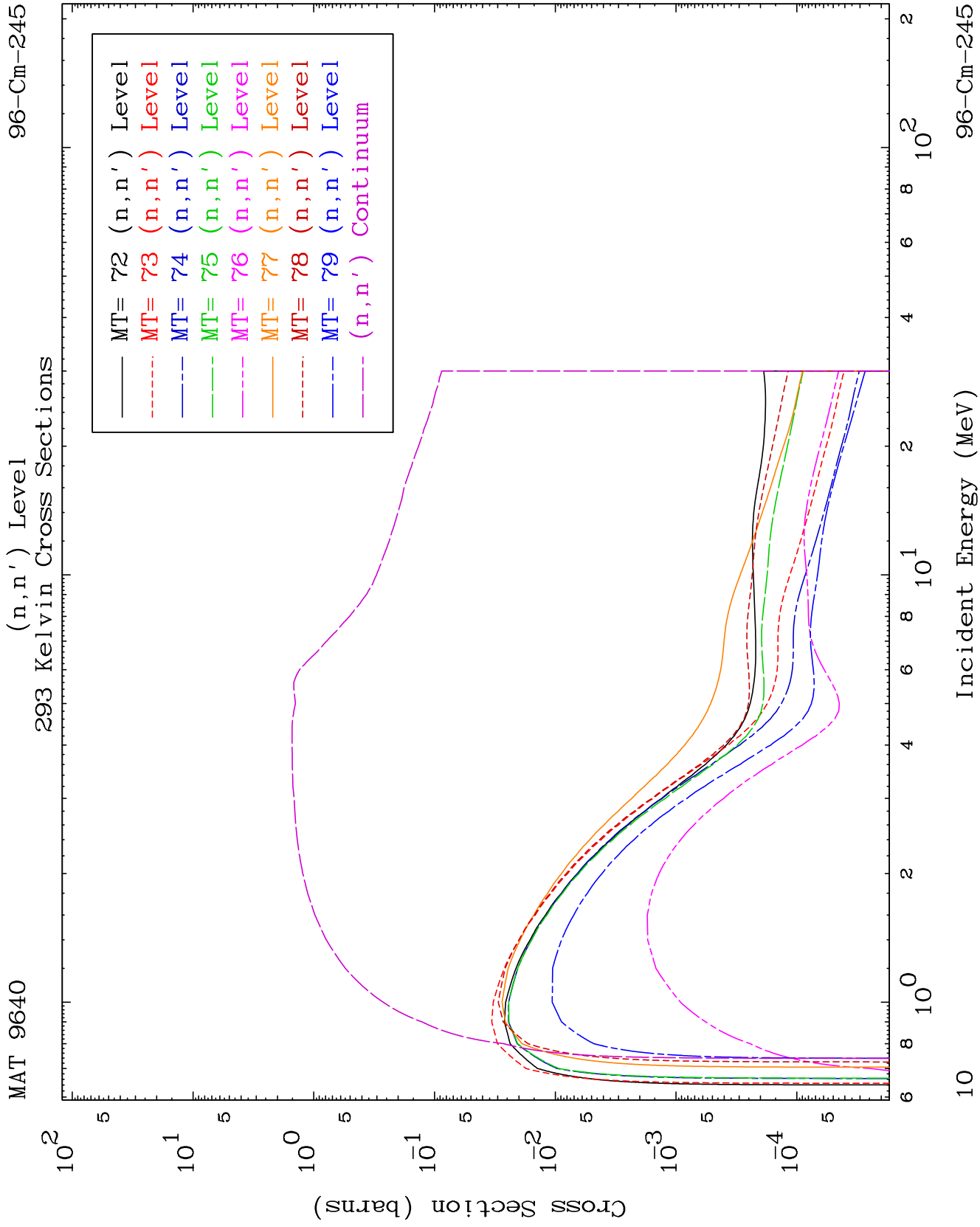
293 Kelvin Cross Sections



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Incident Energy (MeV)

96-Cm-245

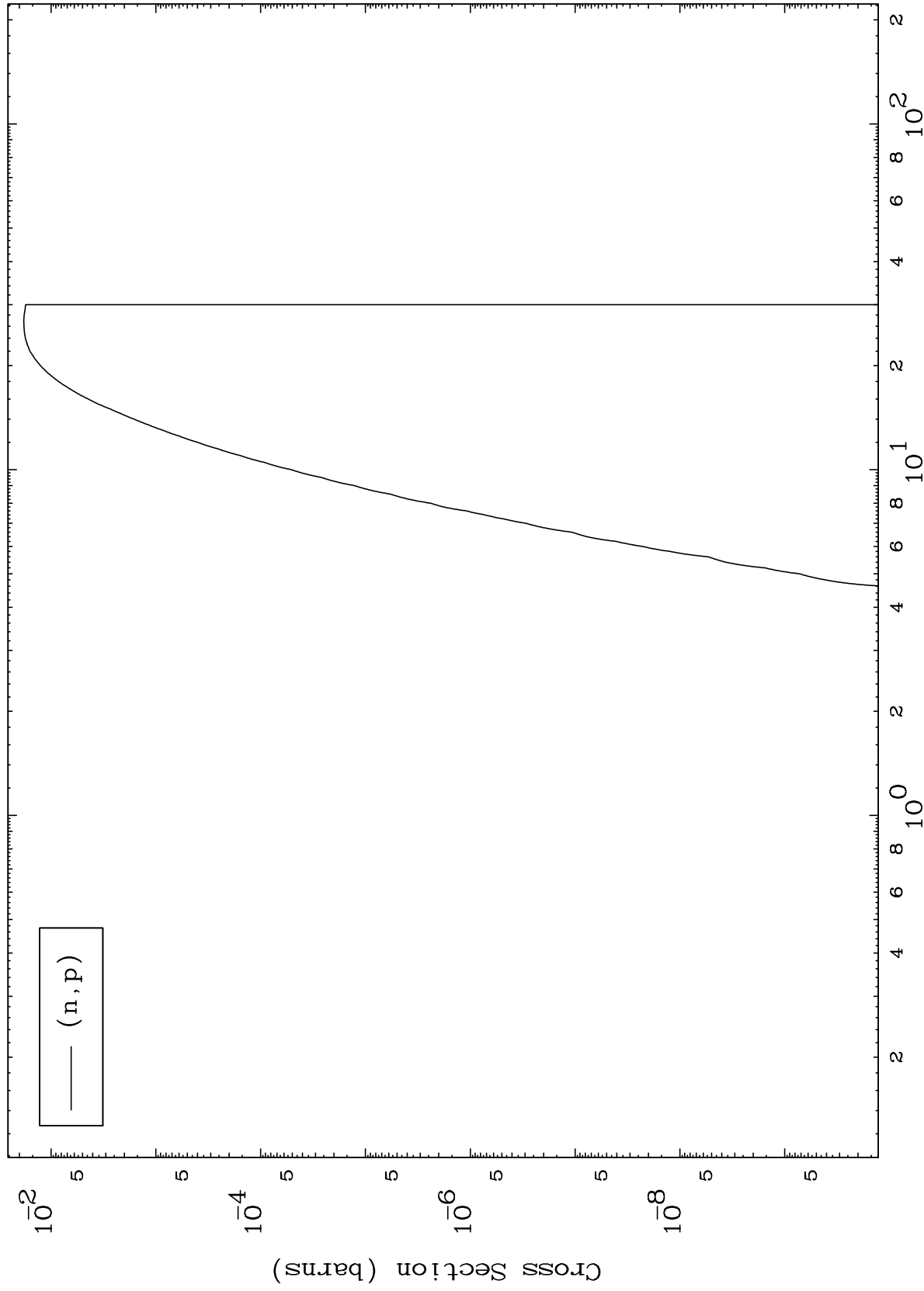


MAT 9640

(n,p) Levels

96-Cm-245

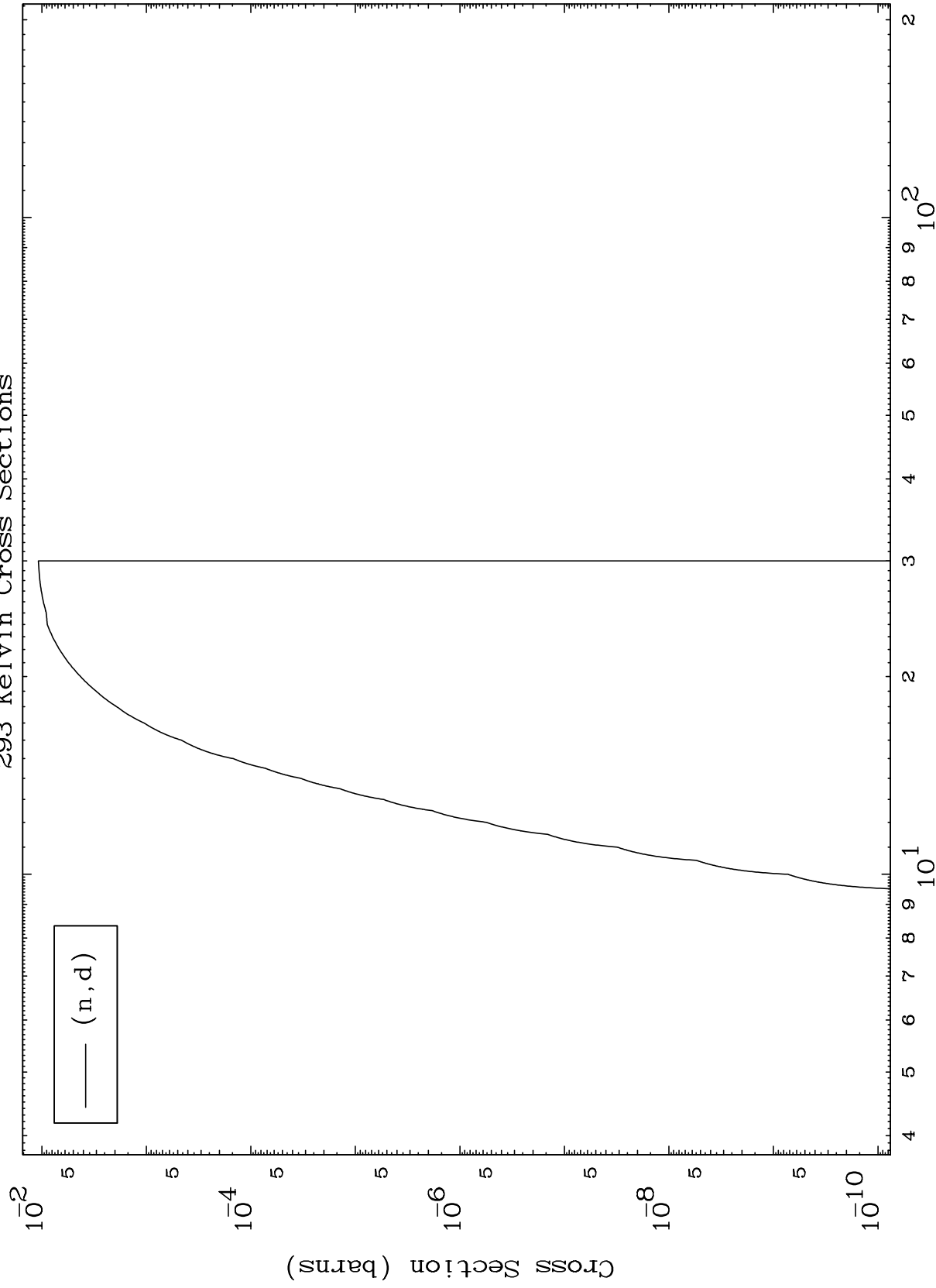
293 Kelvin Cross Sections



MAT 9640

96-Cm-245

(n,d) Levels
293 Kelvin Cross Sections



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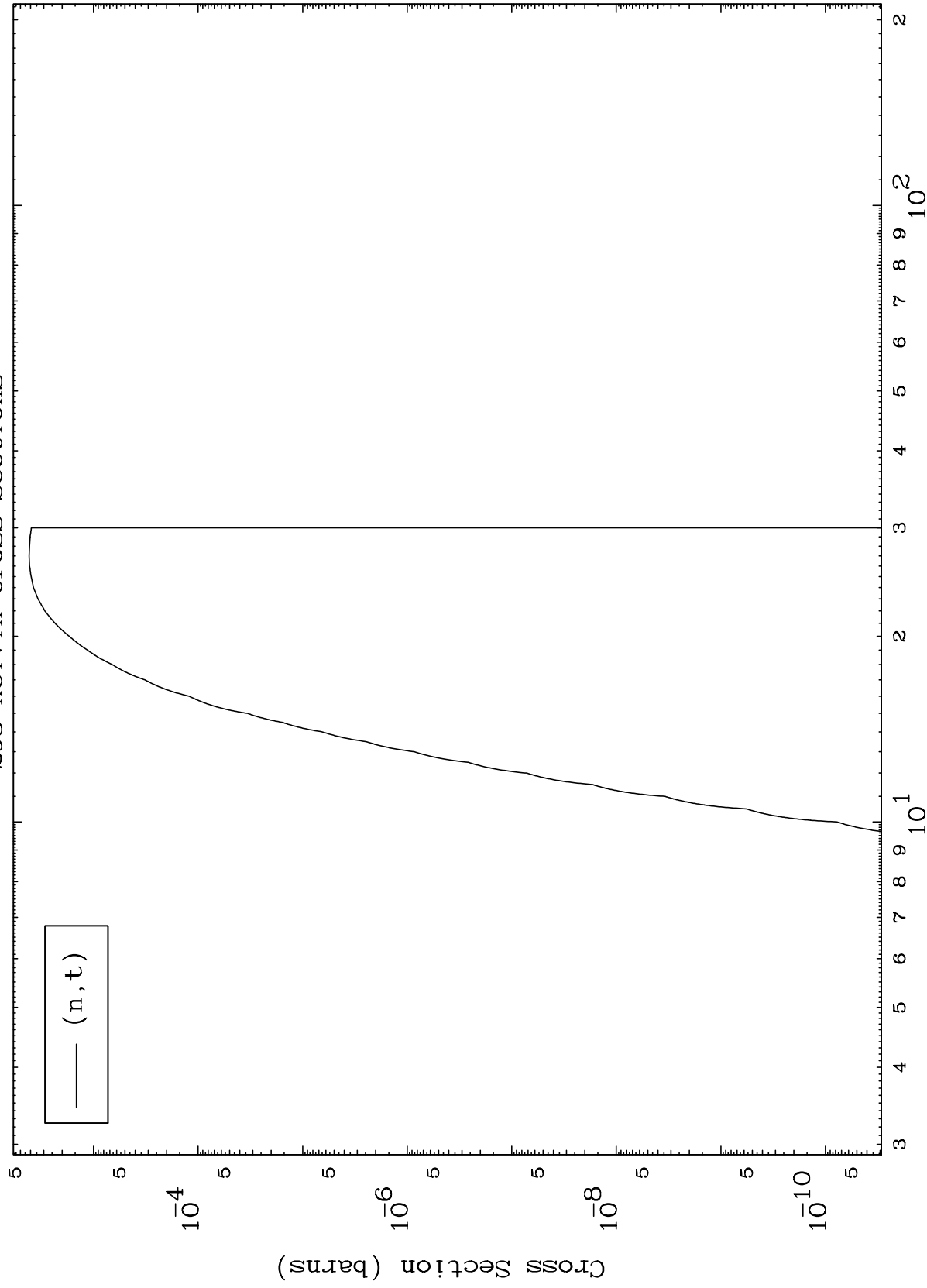
Incident Energy (MeV)

96-Cm-245

MAT 9640

96-Cm-245

(n,t) Levels
293 Kelvin Cross Sections



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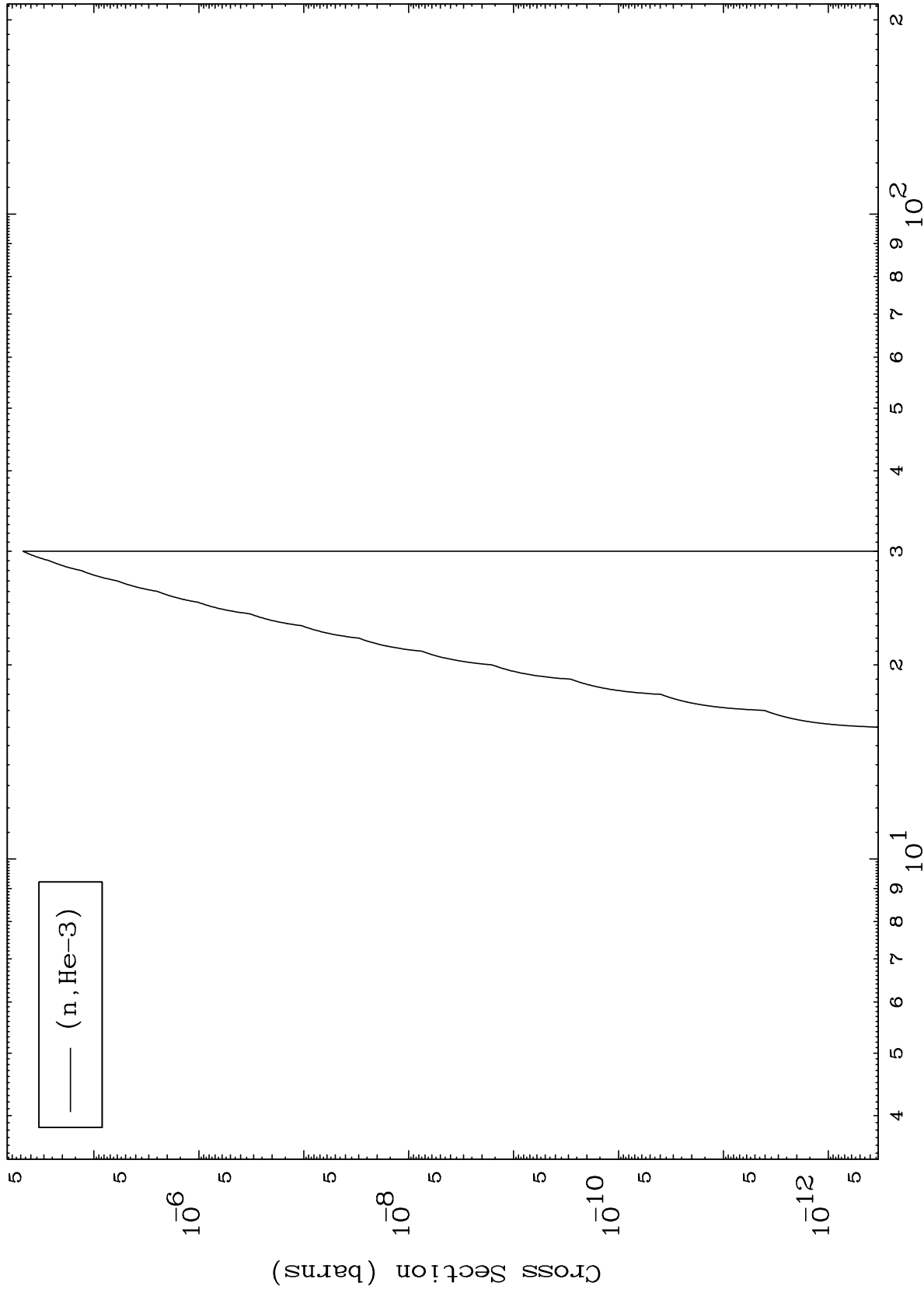
Incident Energy (MeV)

96-Cm-245

MAT 9640

96-Cm-245

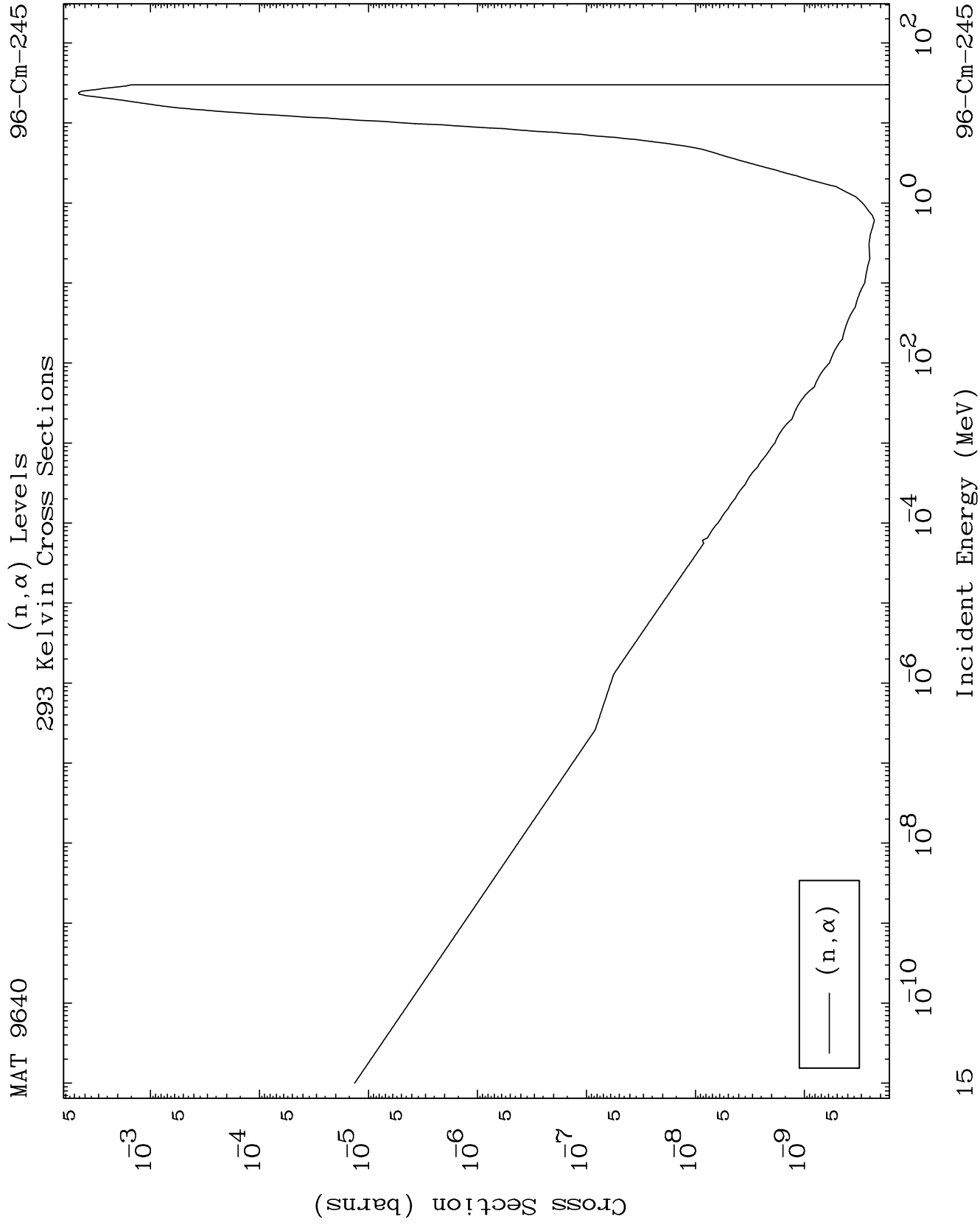
(n,He3) Levels
293 Kelvin Cross Sections



14

Incident Energy (MeV)

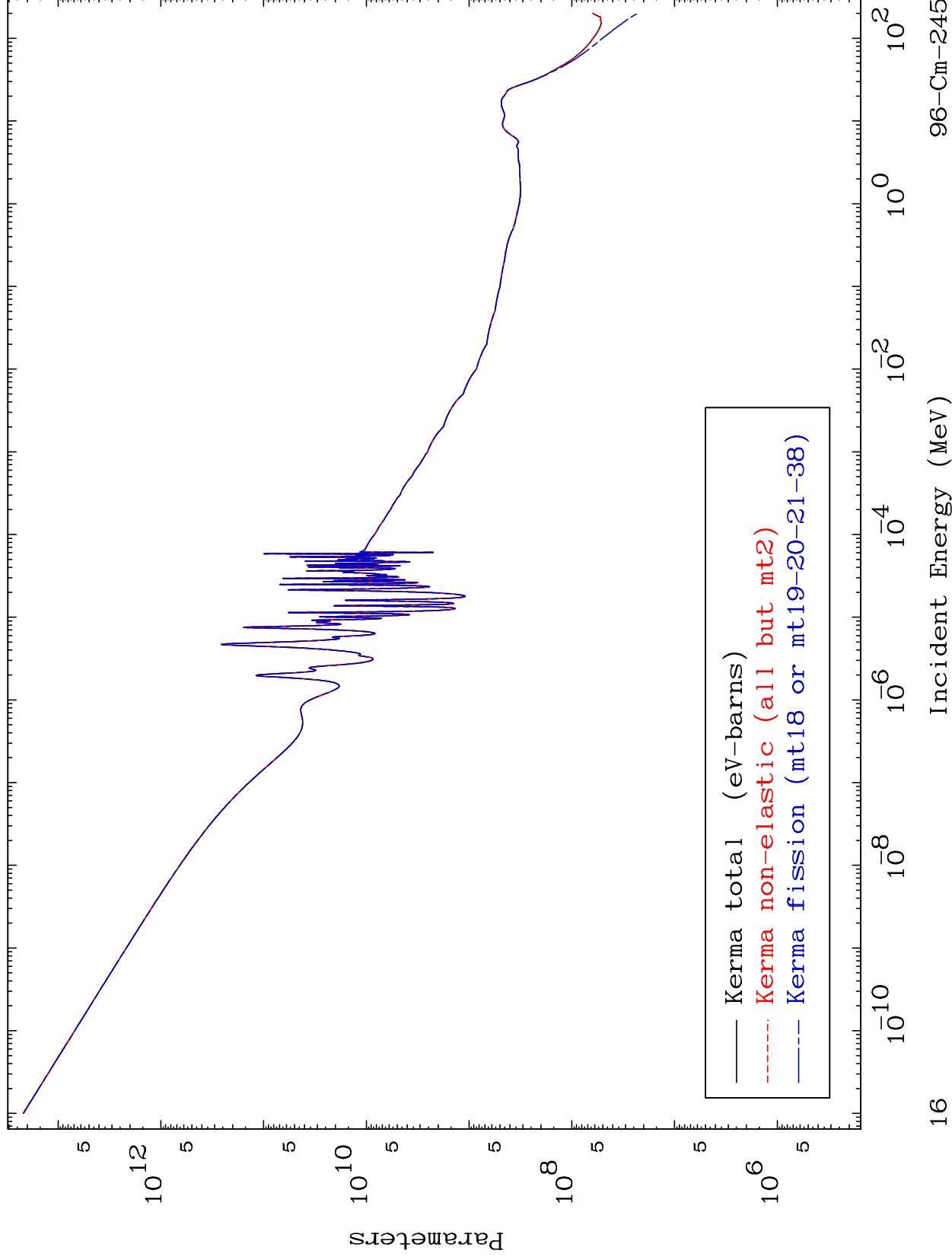
96-Cm-245



MAT 9640

Energy Release
Parameters

96-Cm-245

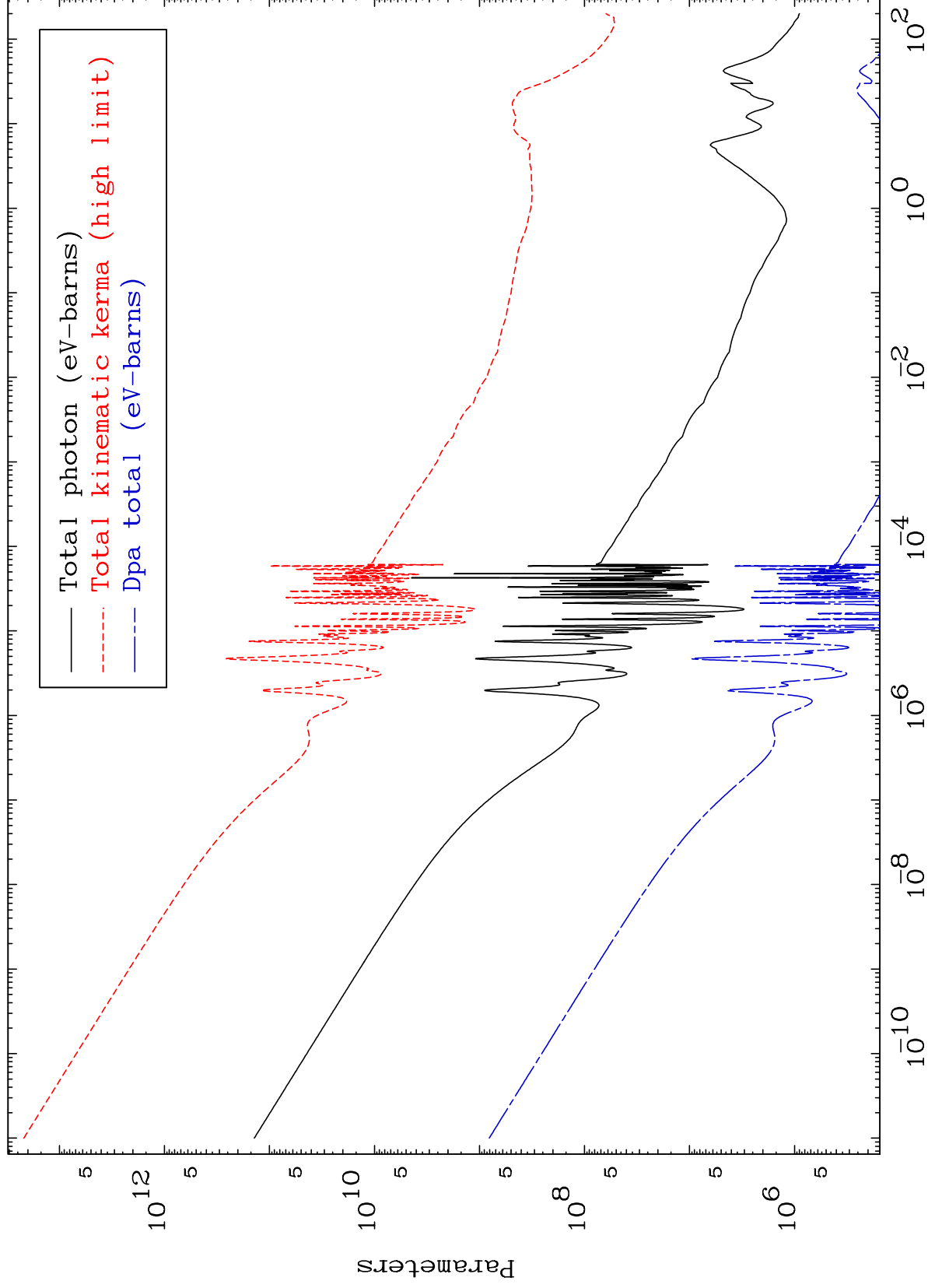


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MAT 9640

Energy Release
Parameters

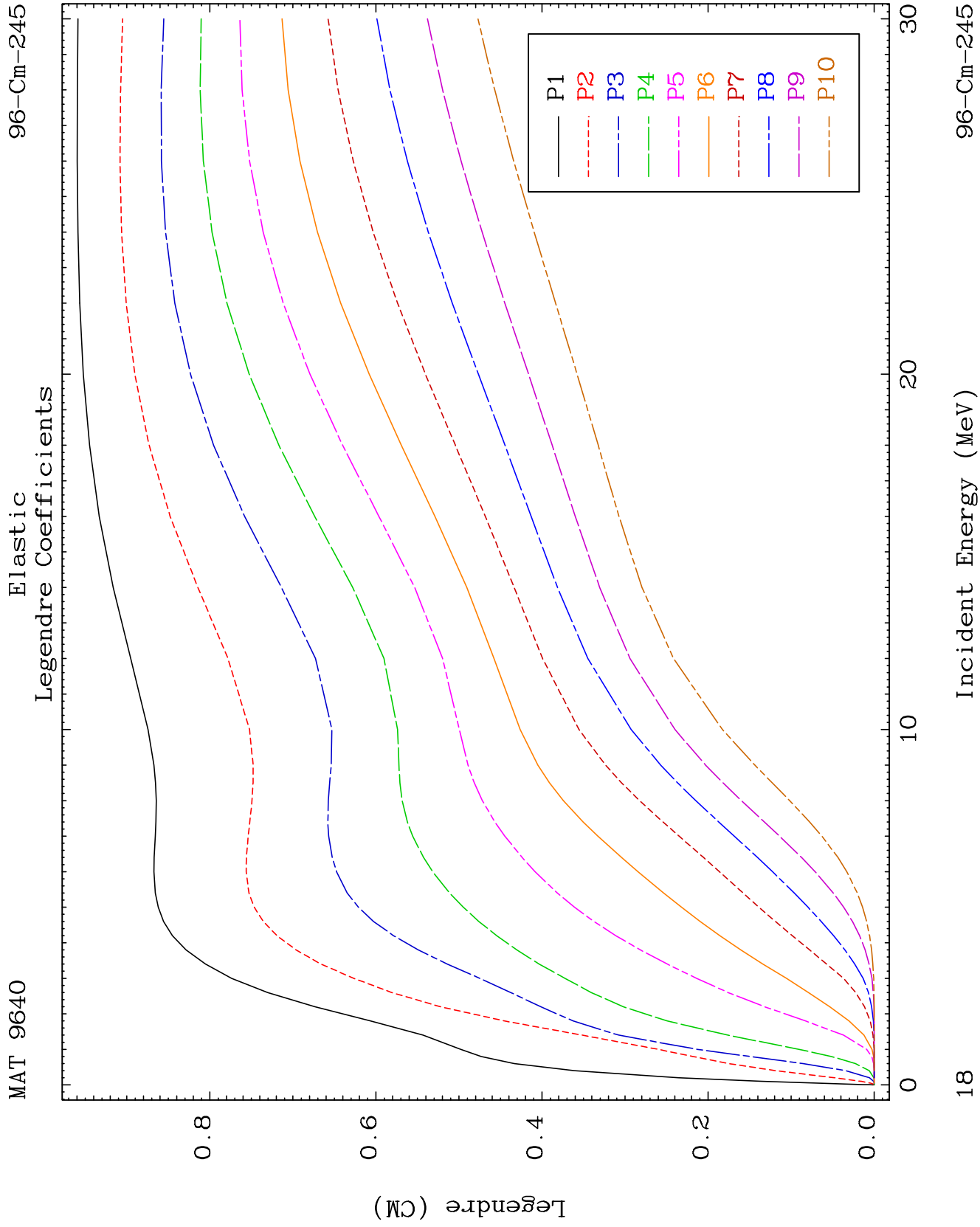
96-Cm-245

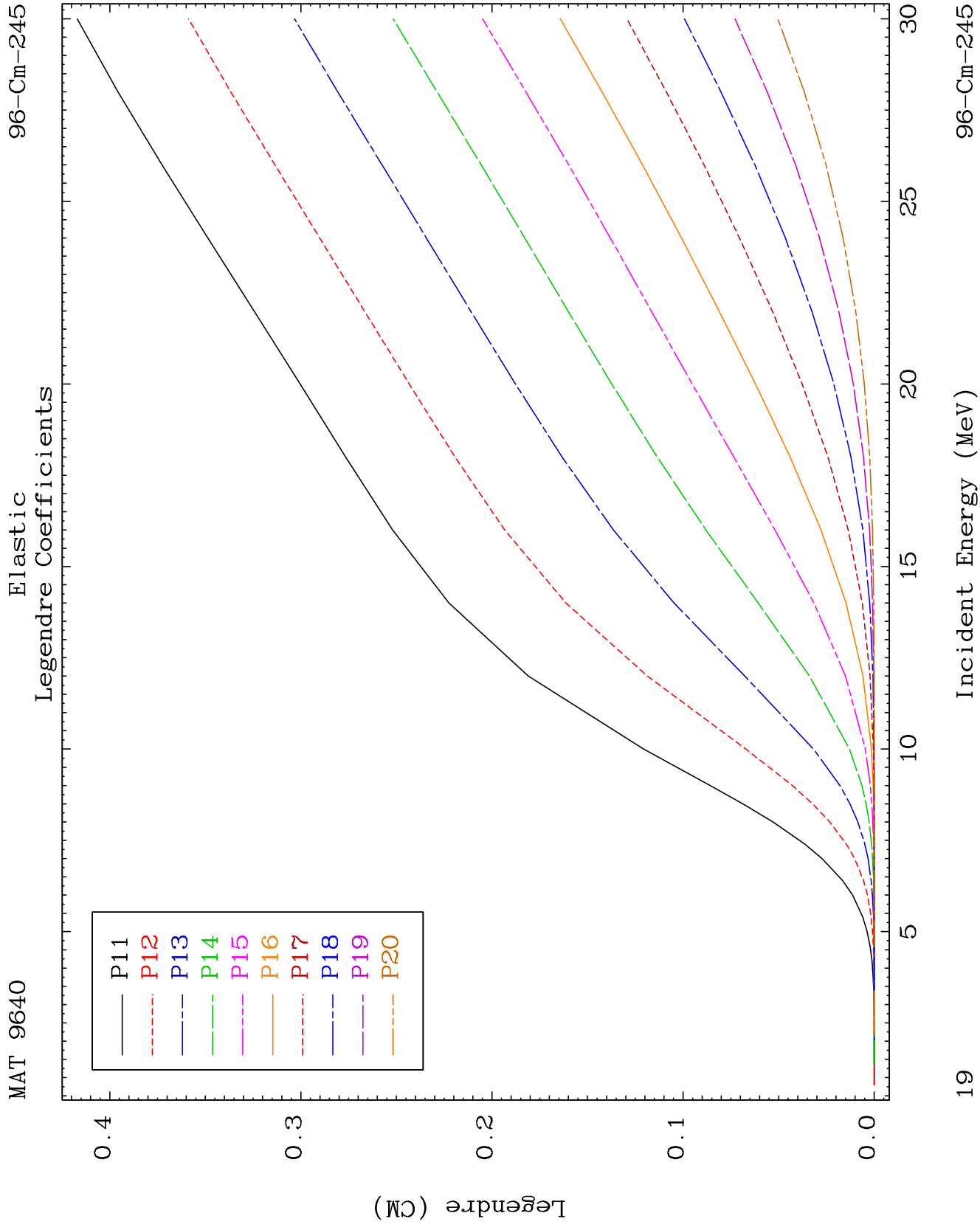


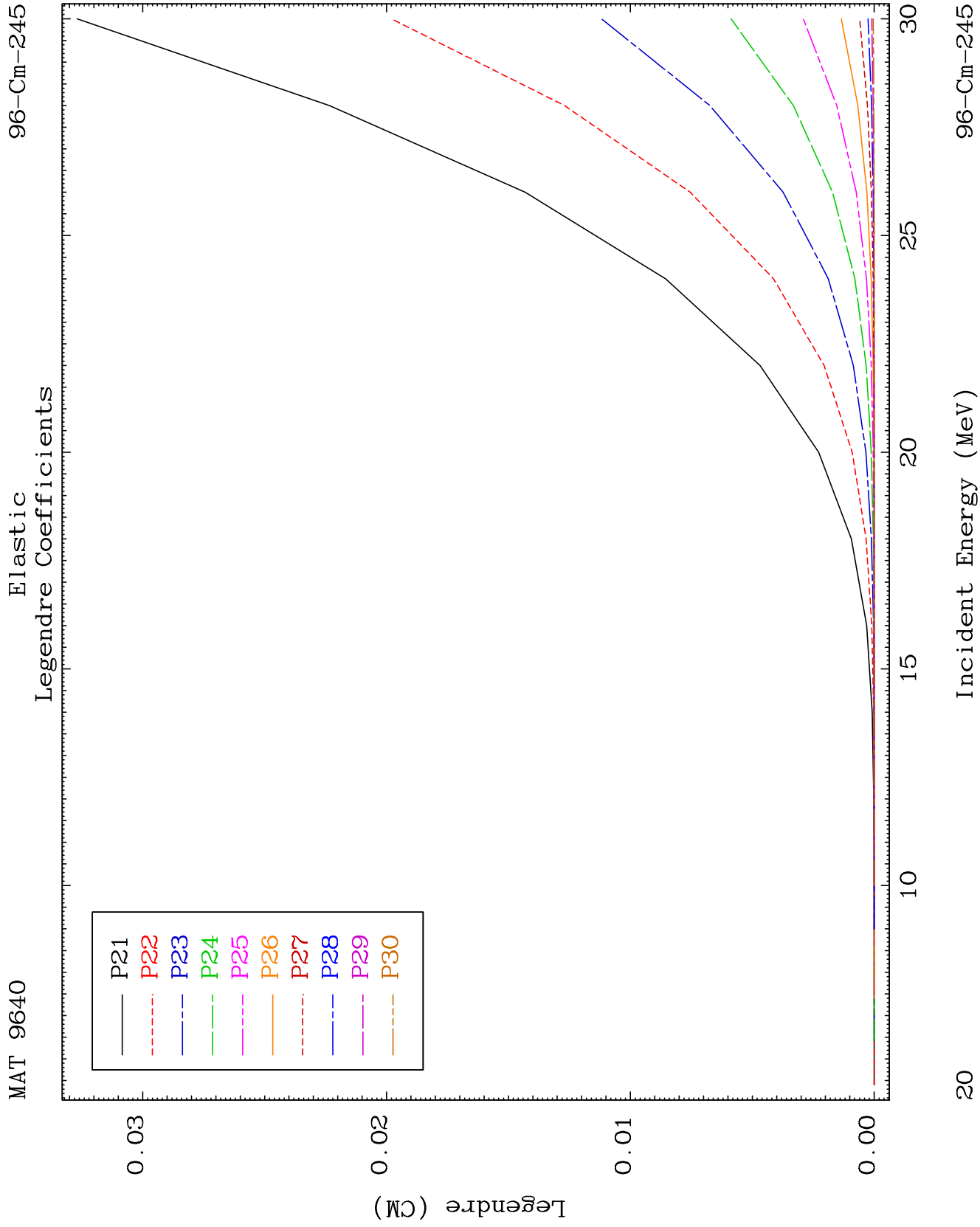
17

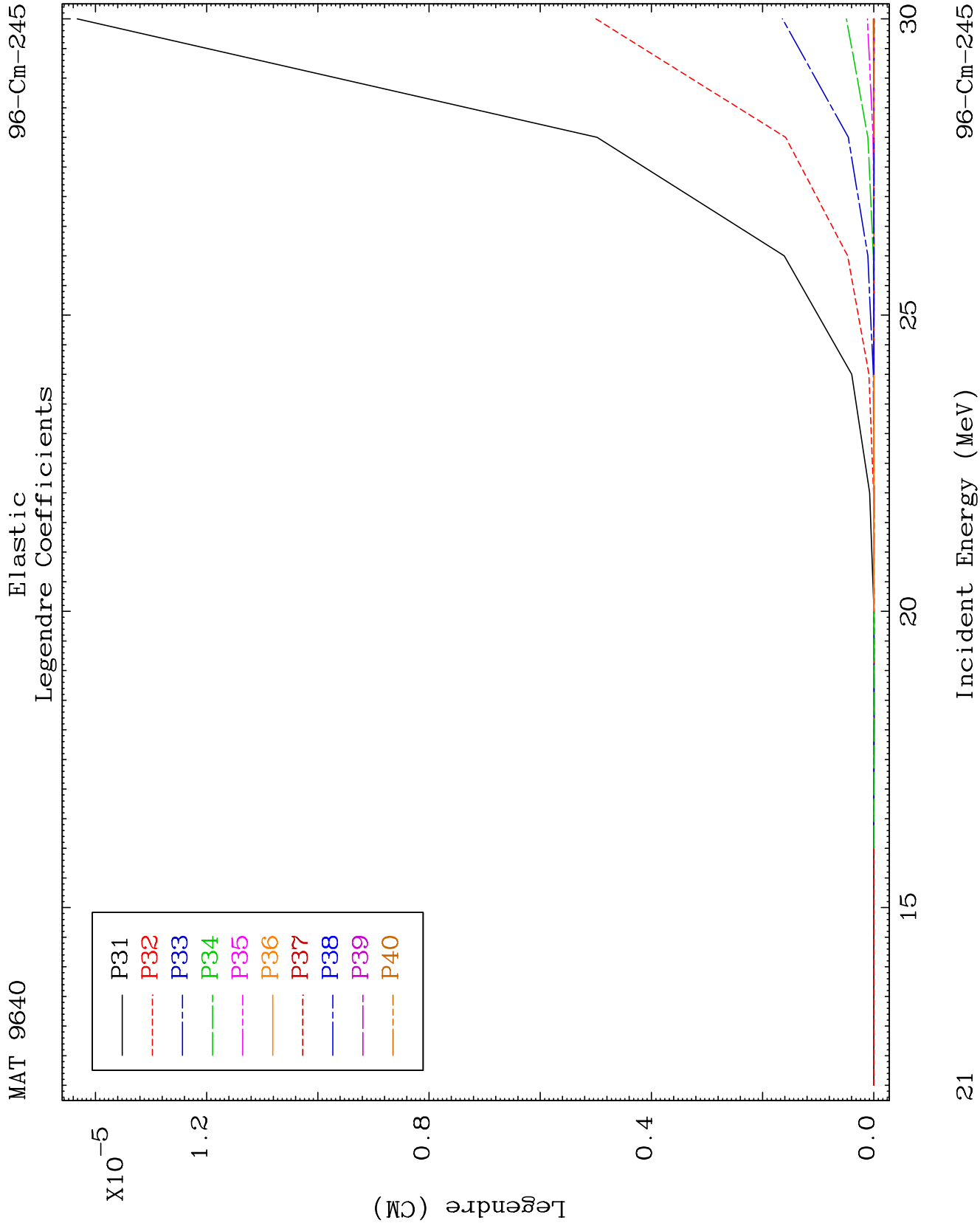
Incident Energy (MeV)

96-Cm-245





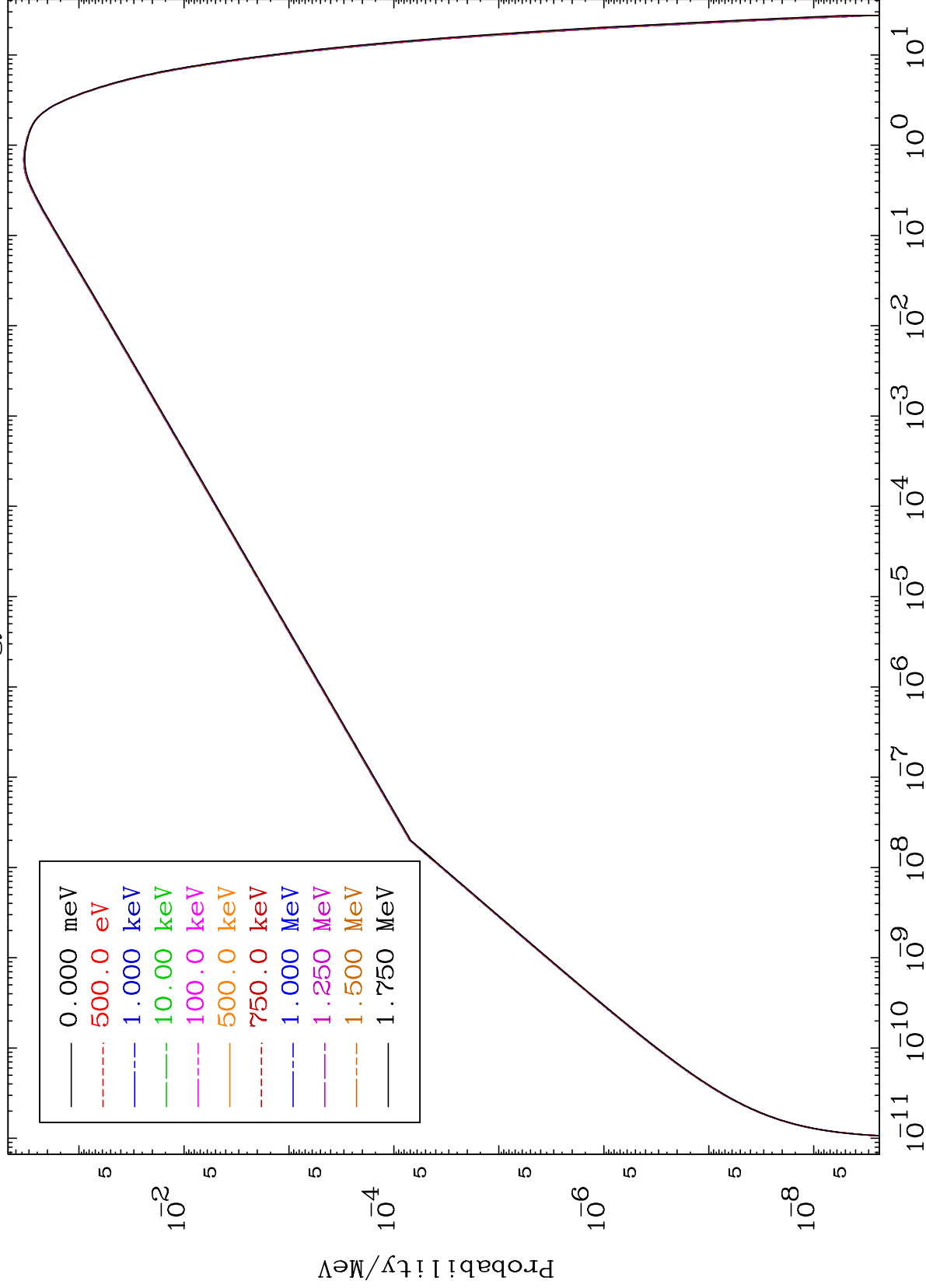




MAT 9640

Fission
Energy Distribution

96-Cm-245



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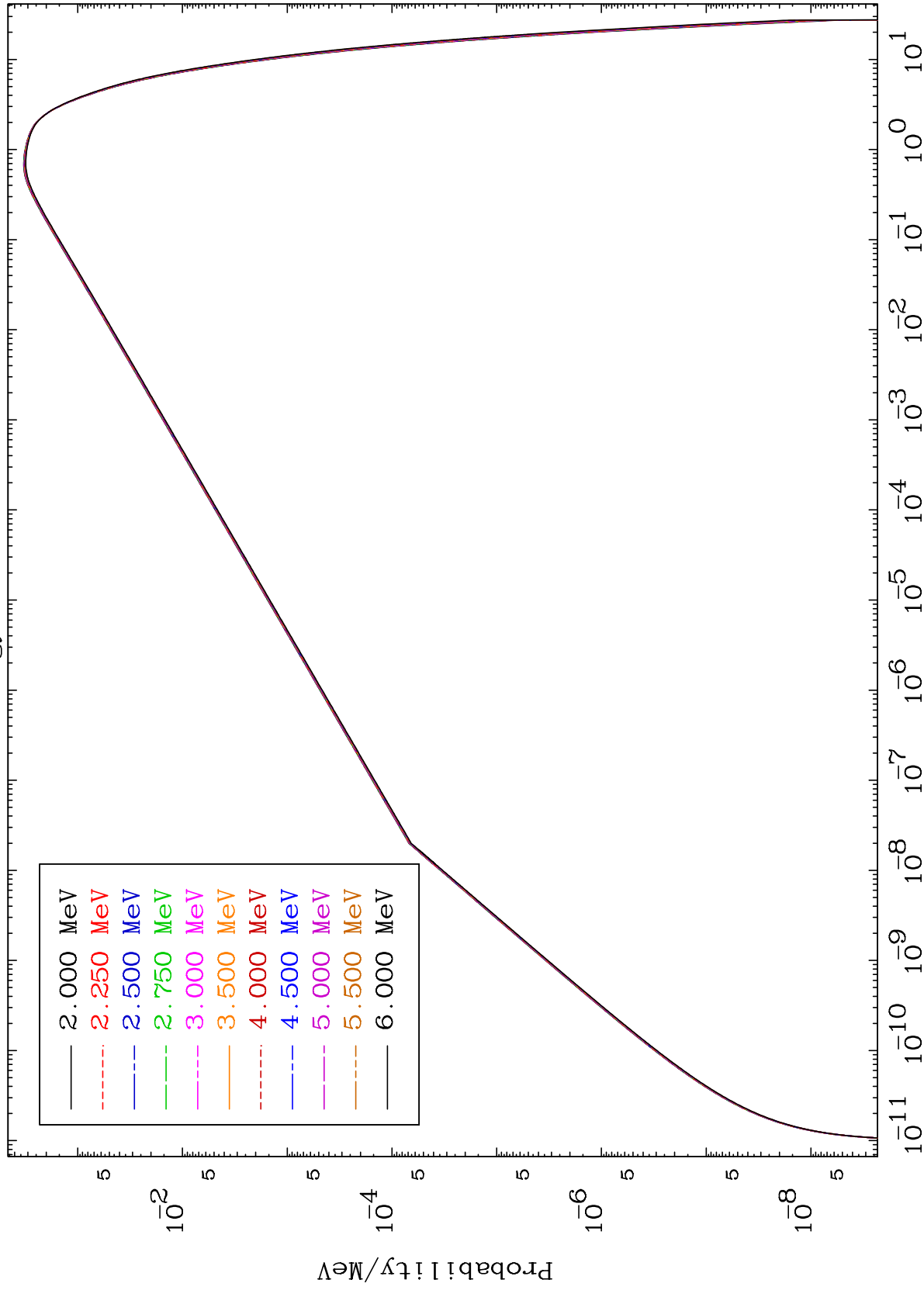
Secondary Energy (MeV)

96-Cm-245

MAT 9640

Fission
Energy Distribution

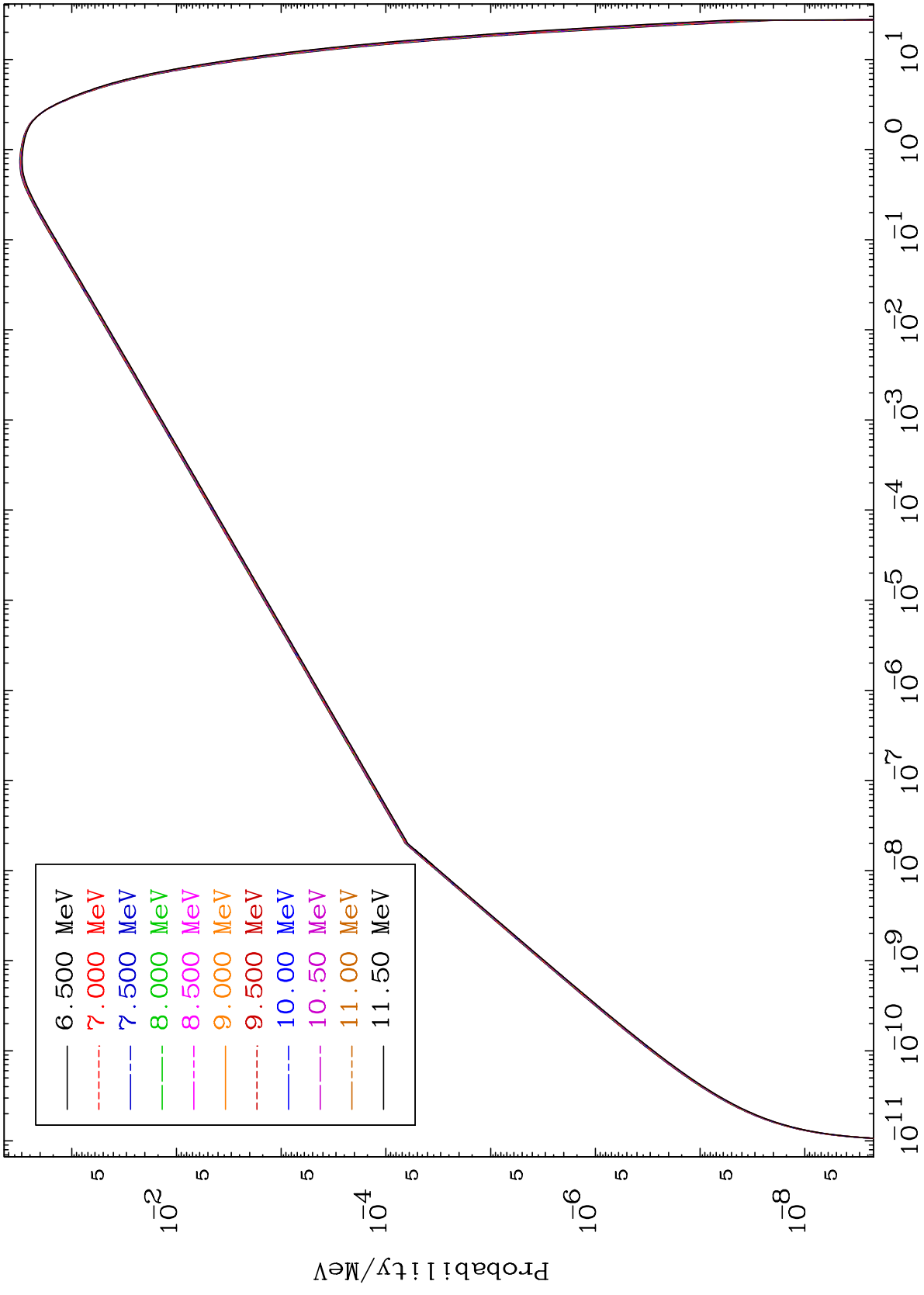
96-Cm-245



MAT 9640

Fission
Energy Distribution

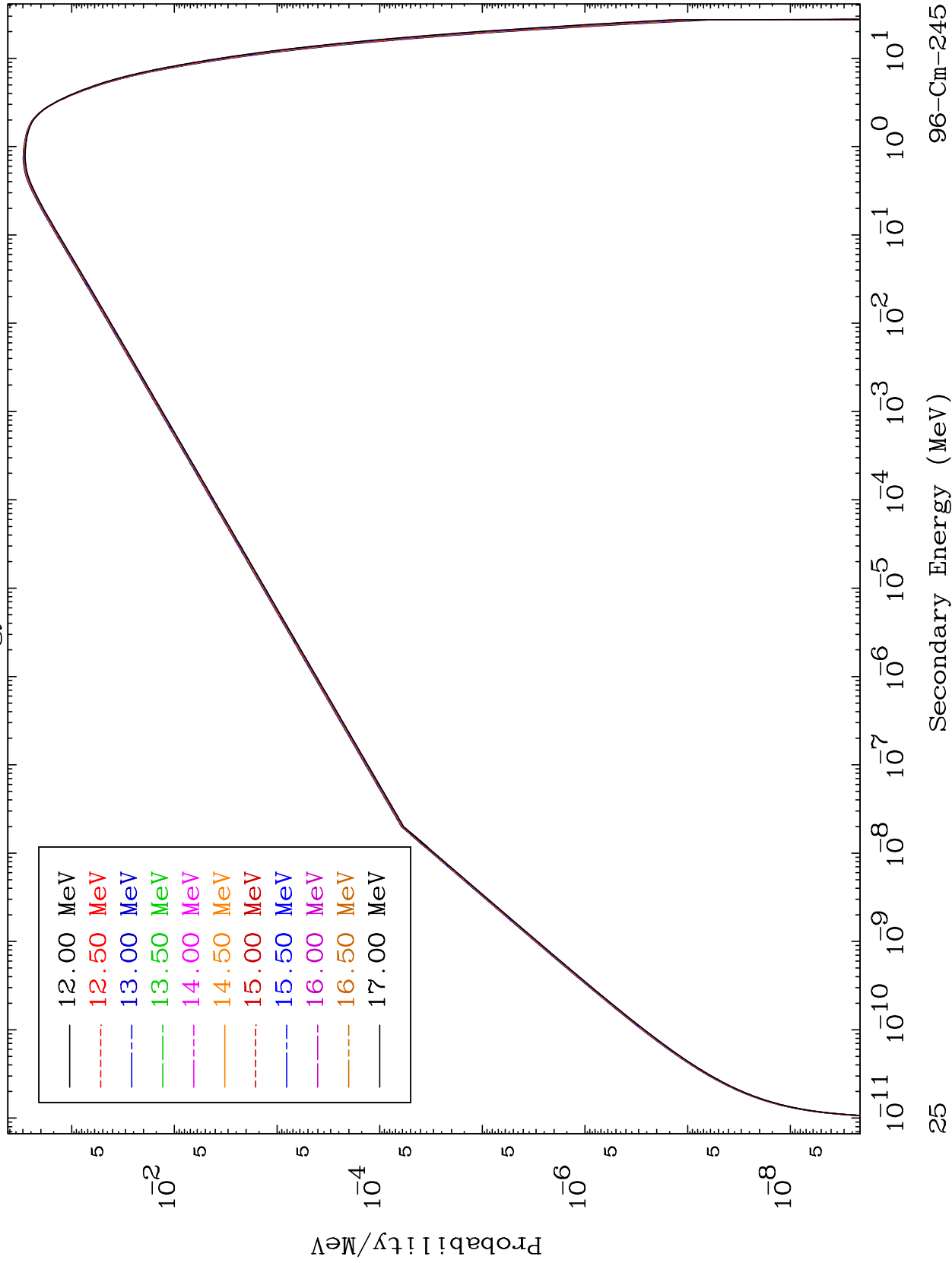
96-Cm-245



MAT 9640

Fission
Energy Distribution

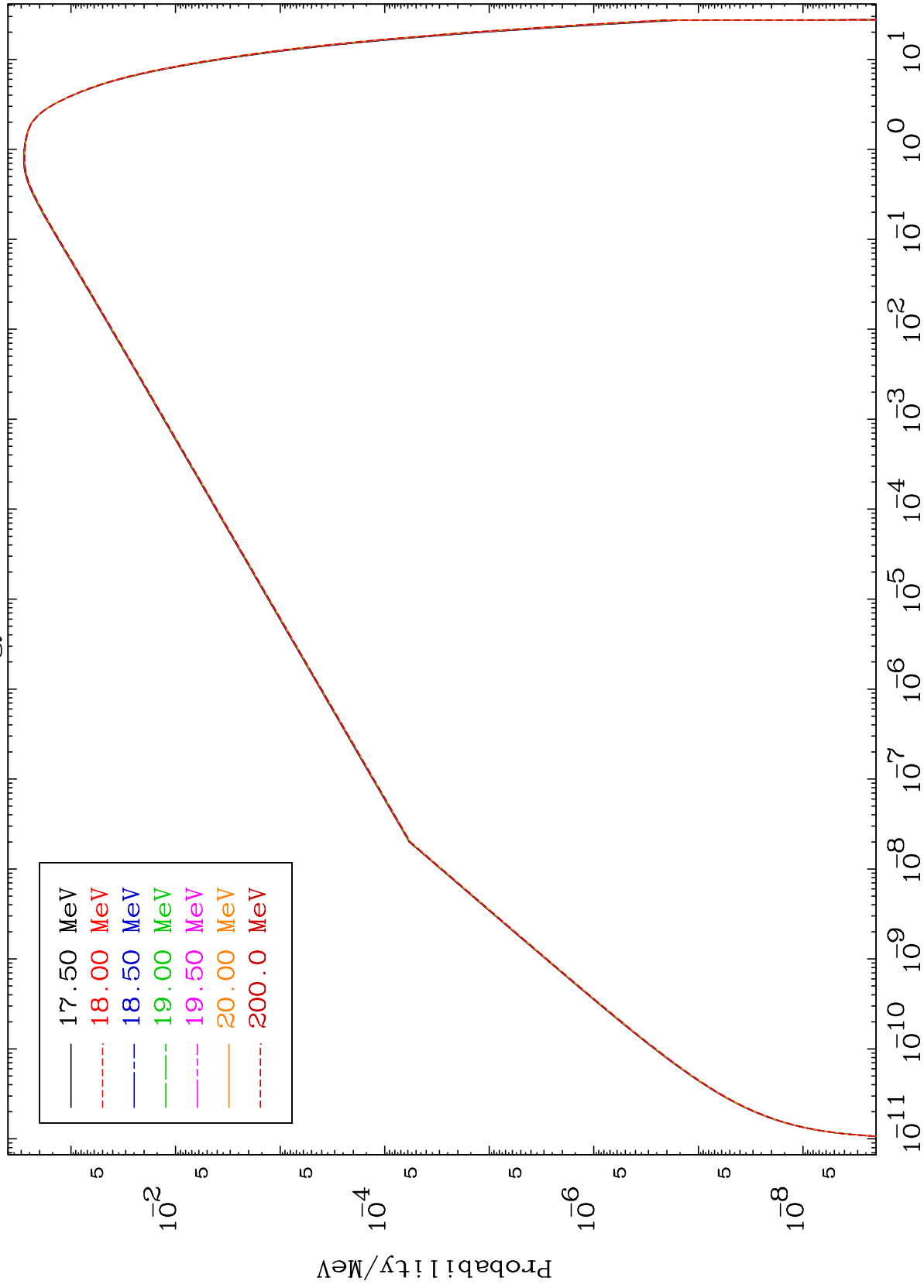
96-Cm-245



MAT 9640

Fission
Energy Distribution

96-Cm-245



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Secondary Energy (MeV)

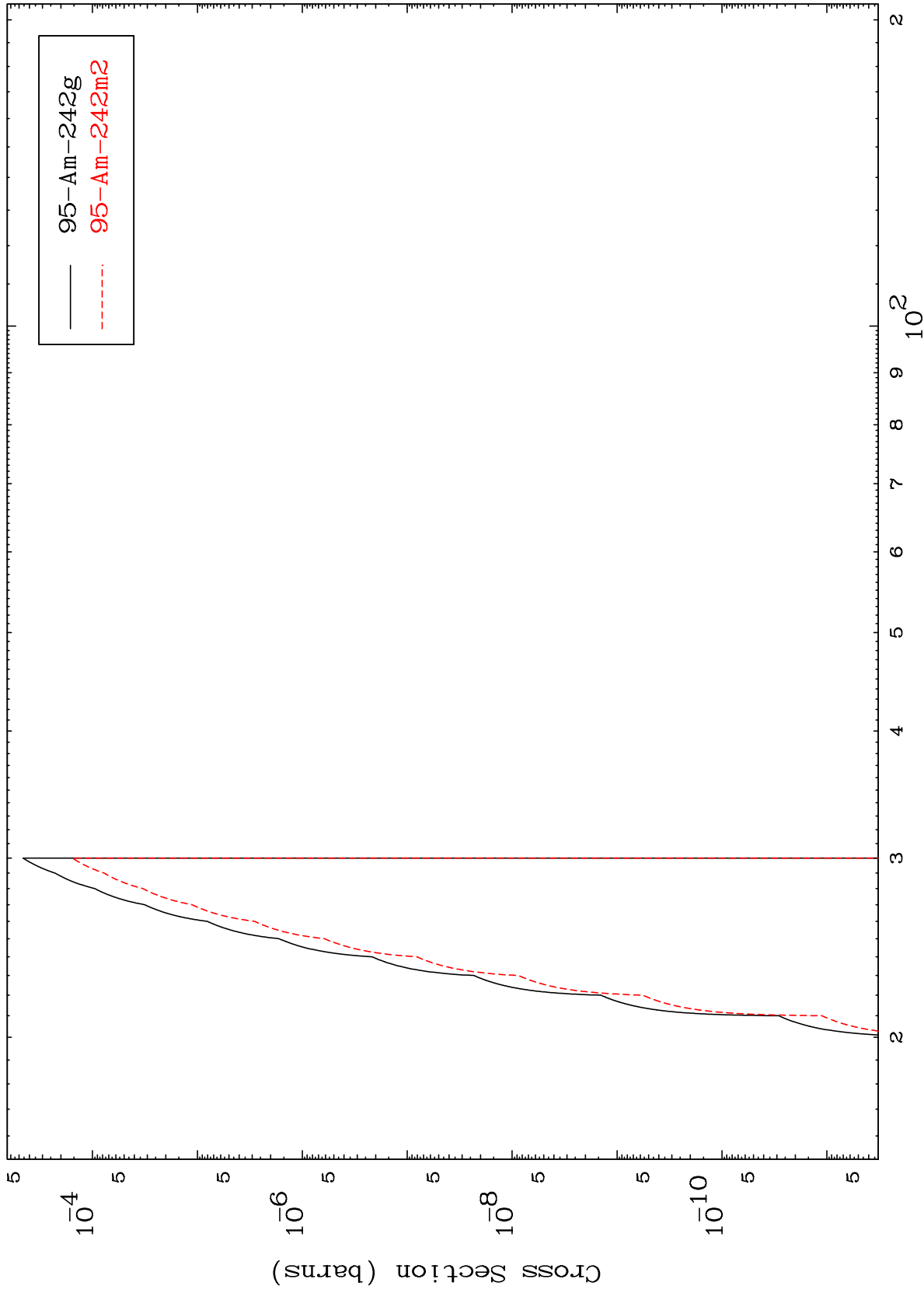
96-Cm-245

MAT 9640

(n,2n) d

96-Cm-245

Radionuclide Production Cross Section



27

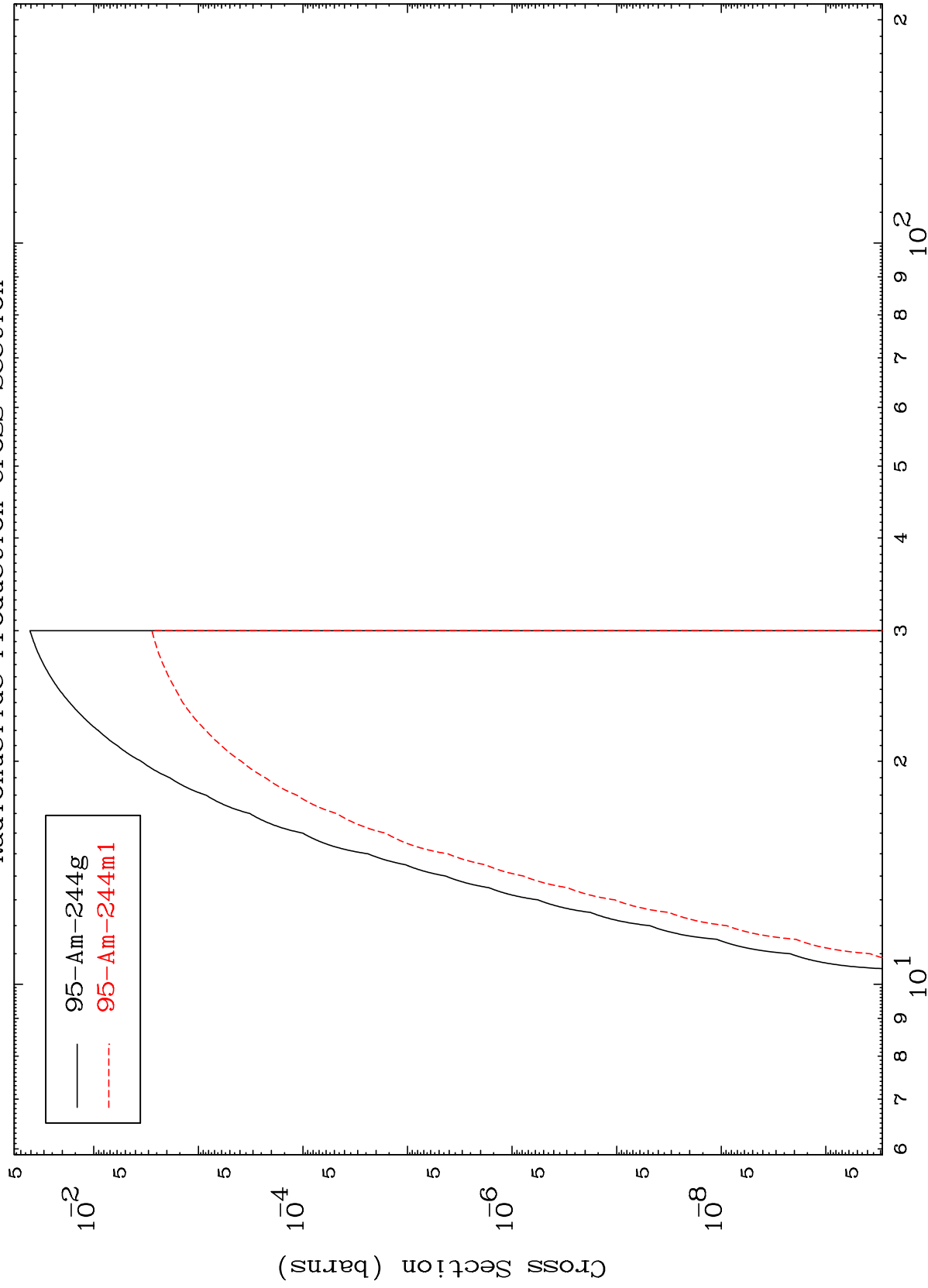
Incident Energy (MeV)

96-Cm-245

MAT 9640

96-Cm-245

(n,n') p
Radionuclide Production Cross Section



28

Incident Energy (MeV)

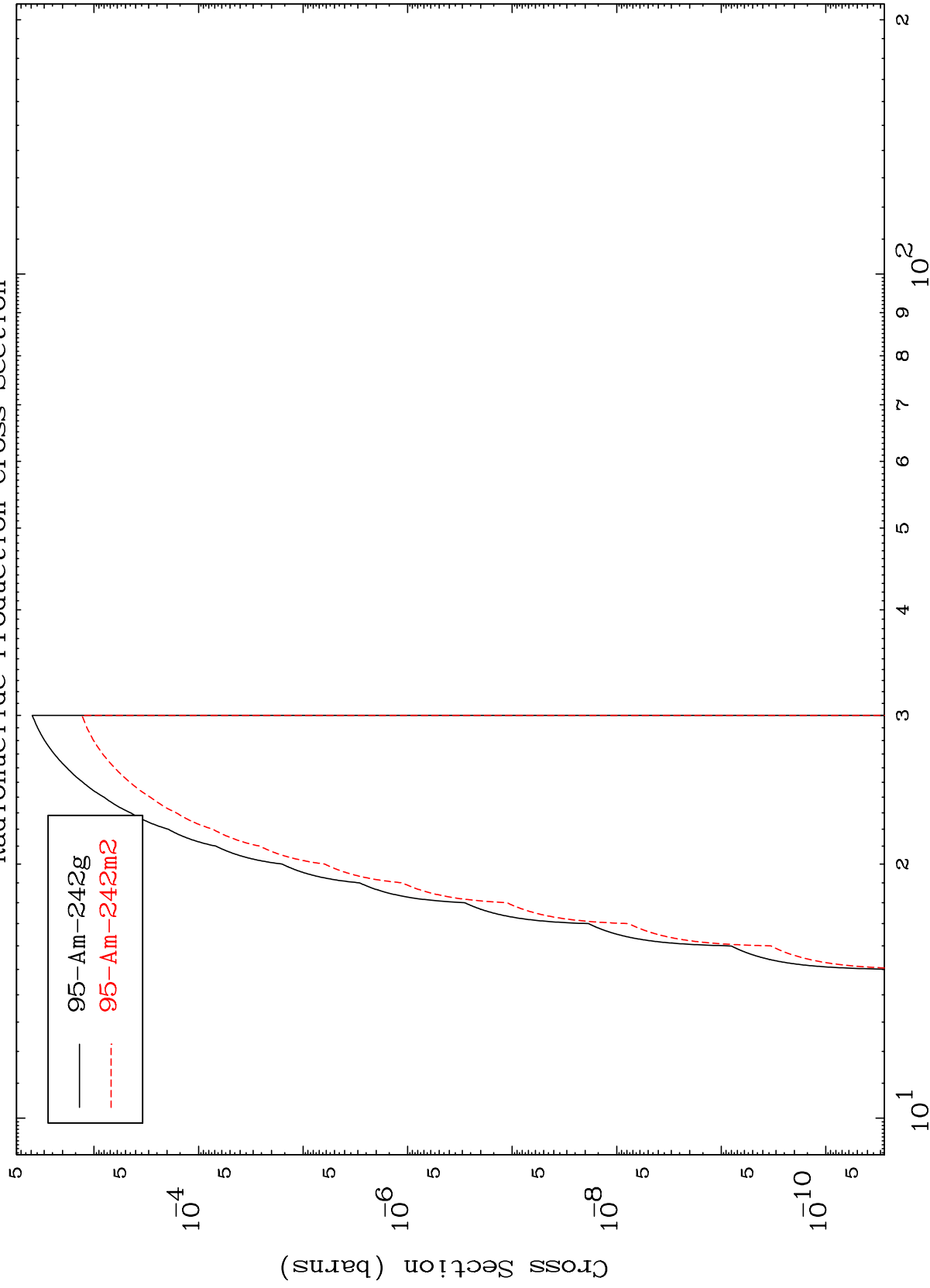
96-Cm-245

MAT 9640

(n,n') t

96-Cm-245

Radionuclide Production Cross Section



29

Incident Energy (MeV)

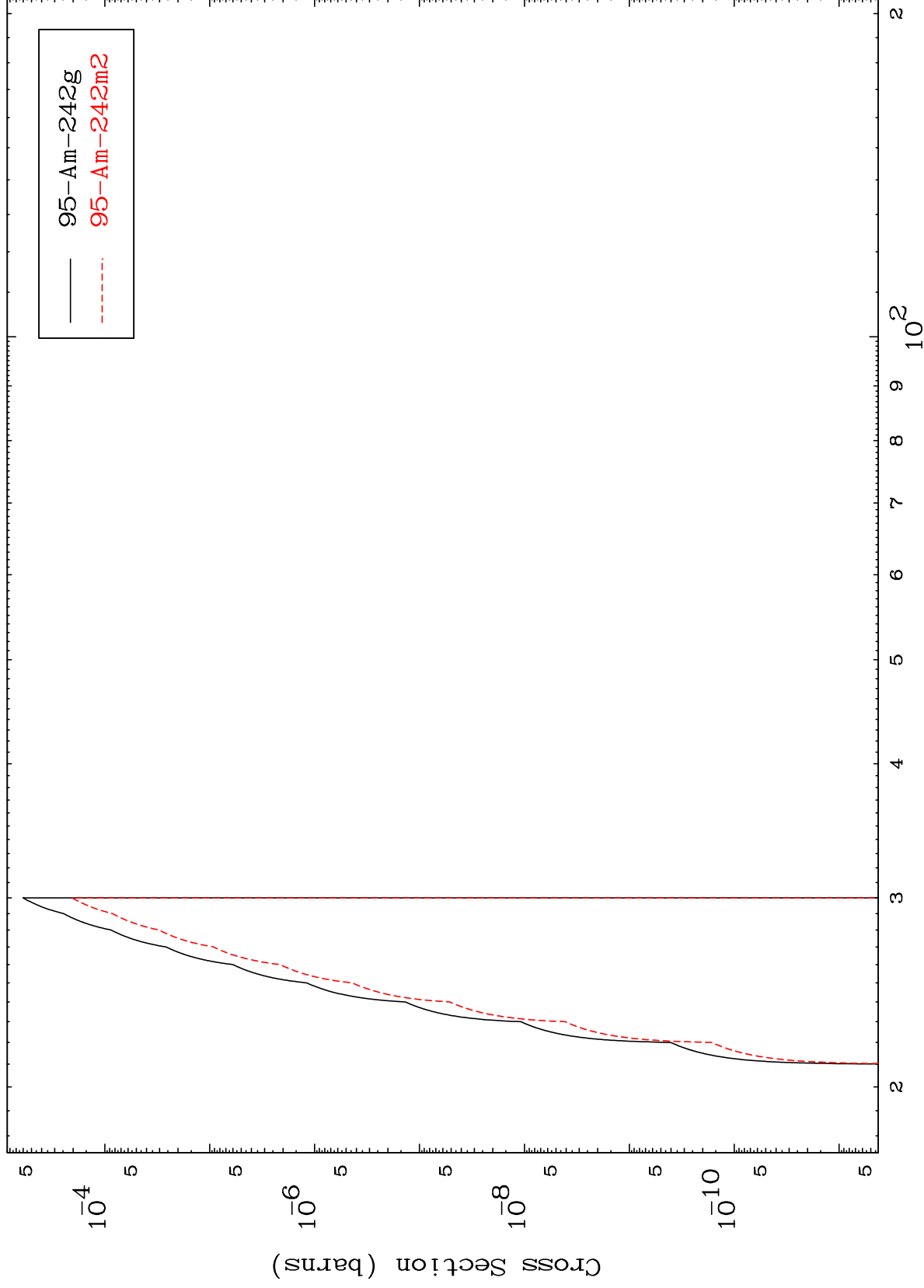
96-Cm-245

MAT 9640

(n,3n) p

96-Cm-245

Radionuclide Production Cross Section



30

Incident Energy (MeV)

96-Cm-245

