

Program EVALPLOT  
(Version 2018-1)

by

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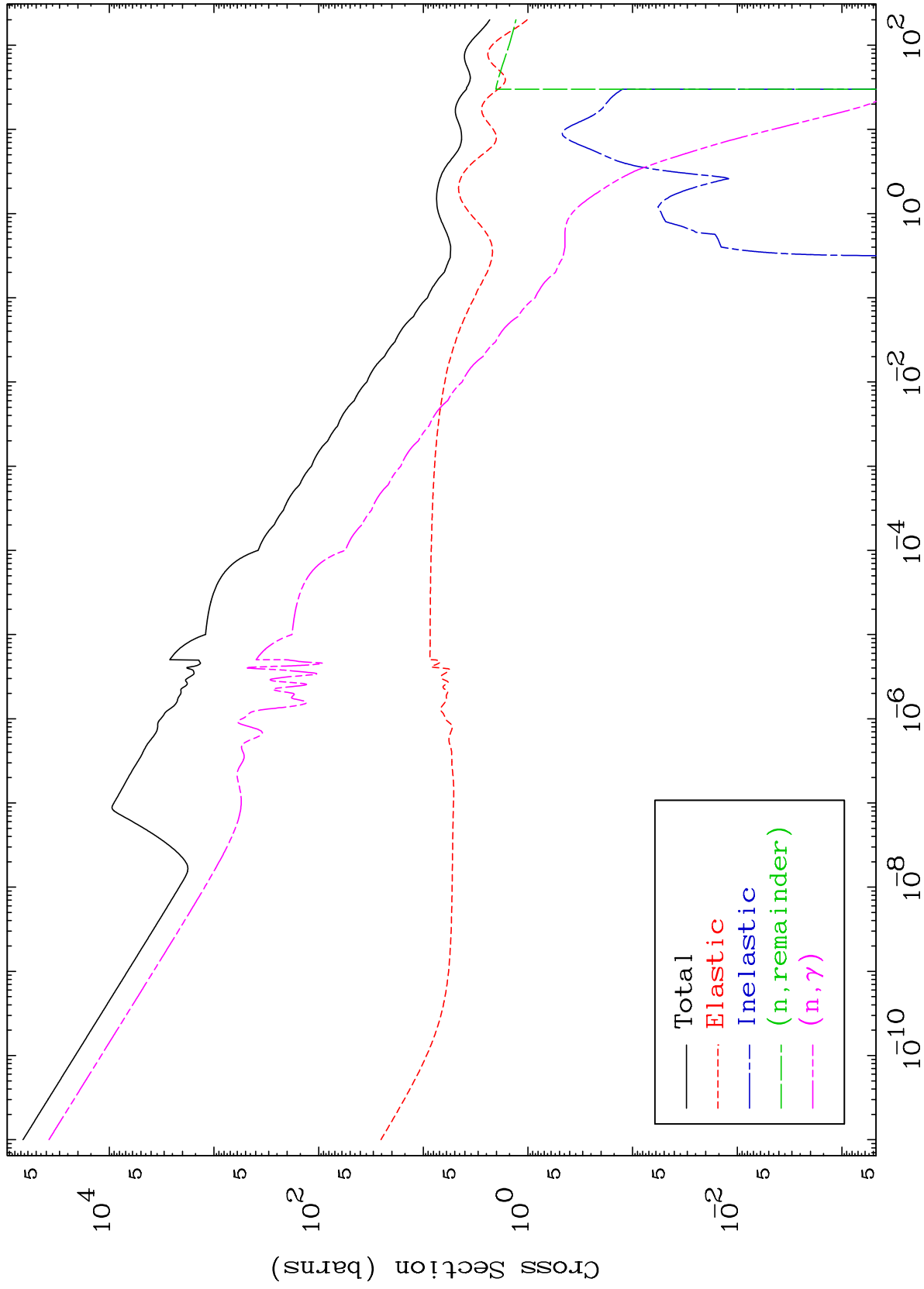
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 6593

Major  
293 Kelvin Cross Sections

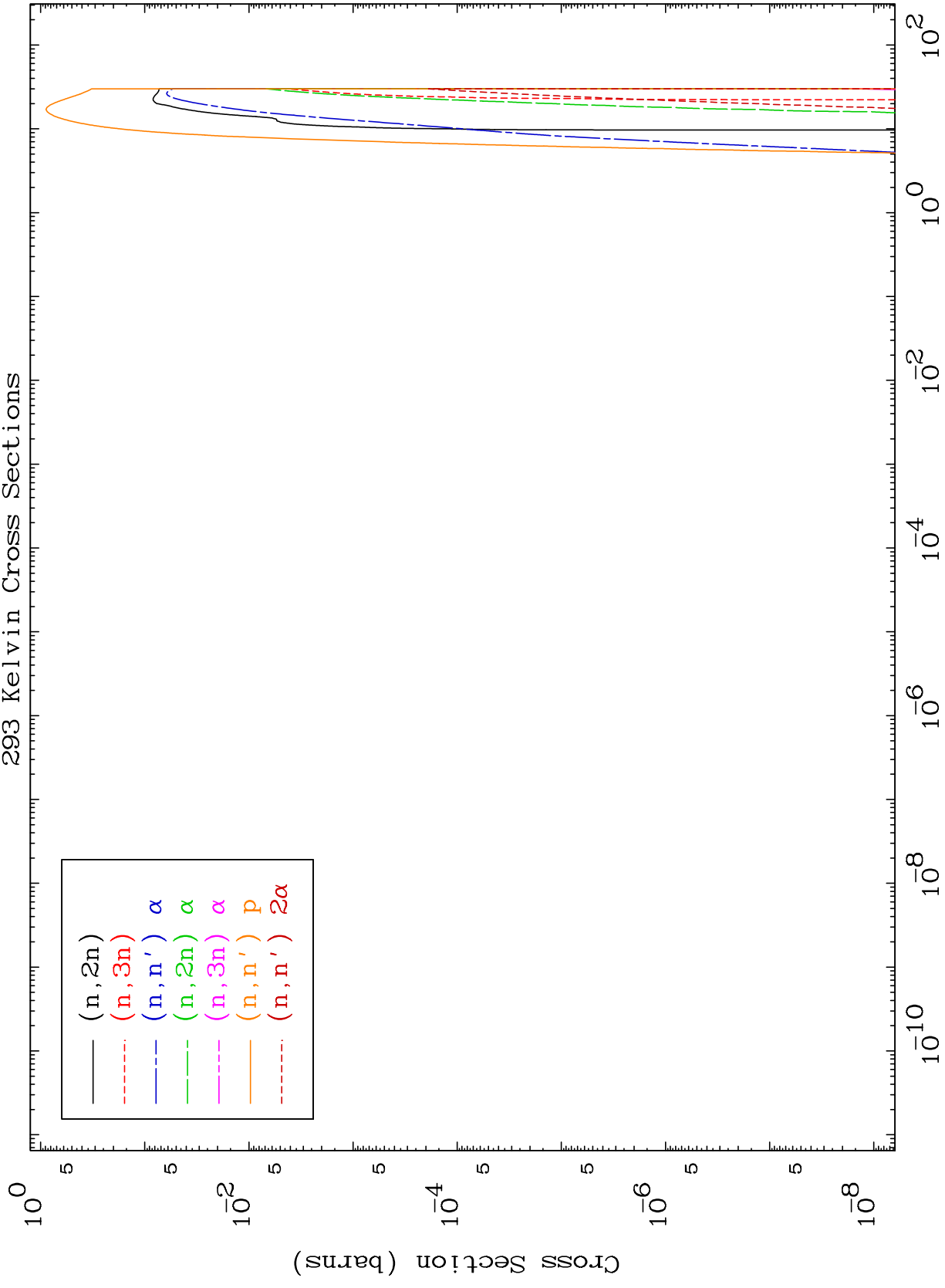
66-Dy-145



MAT 6593

Neutron Production  
293 Kelvin Cross Sections

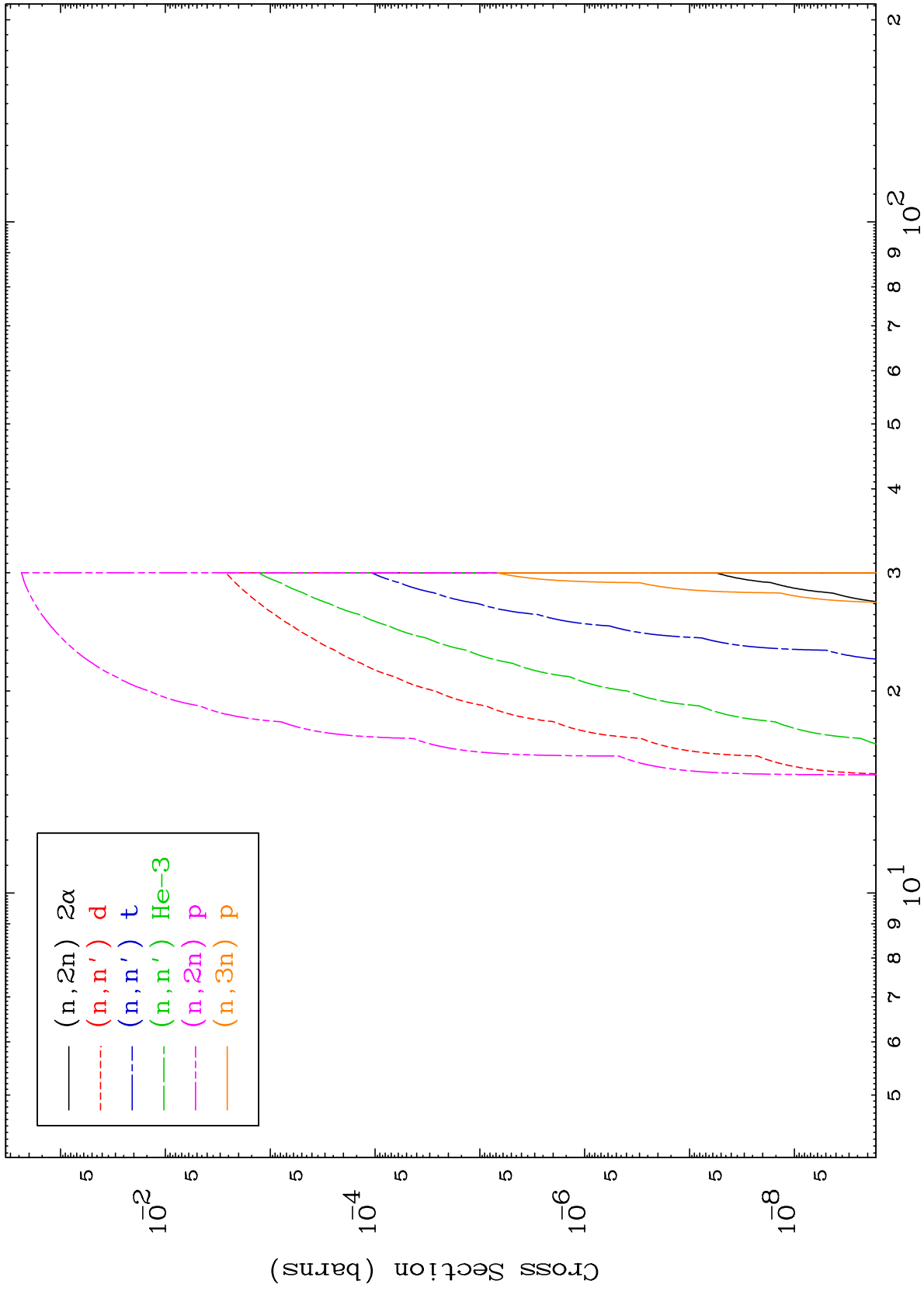
66-Dy-145



MAT 6593

Neutron Production  
293 Kelvin Cross Sections

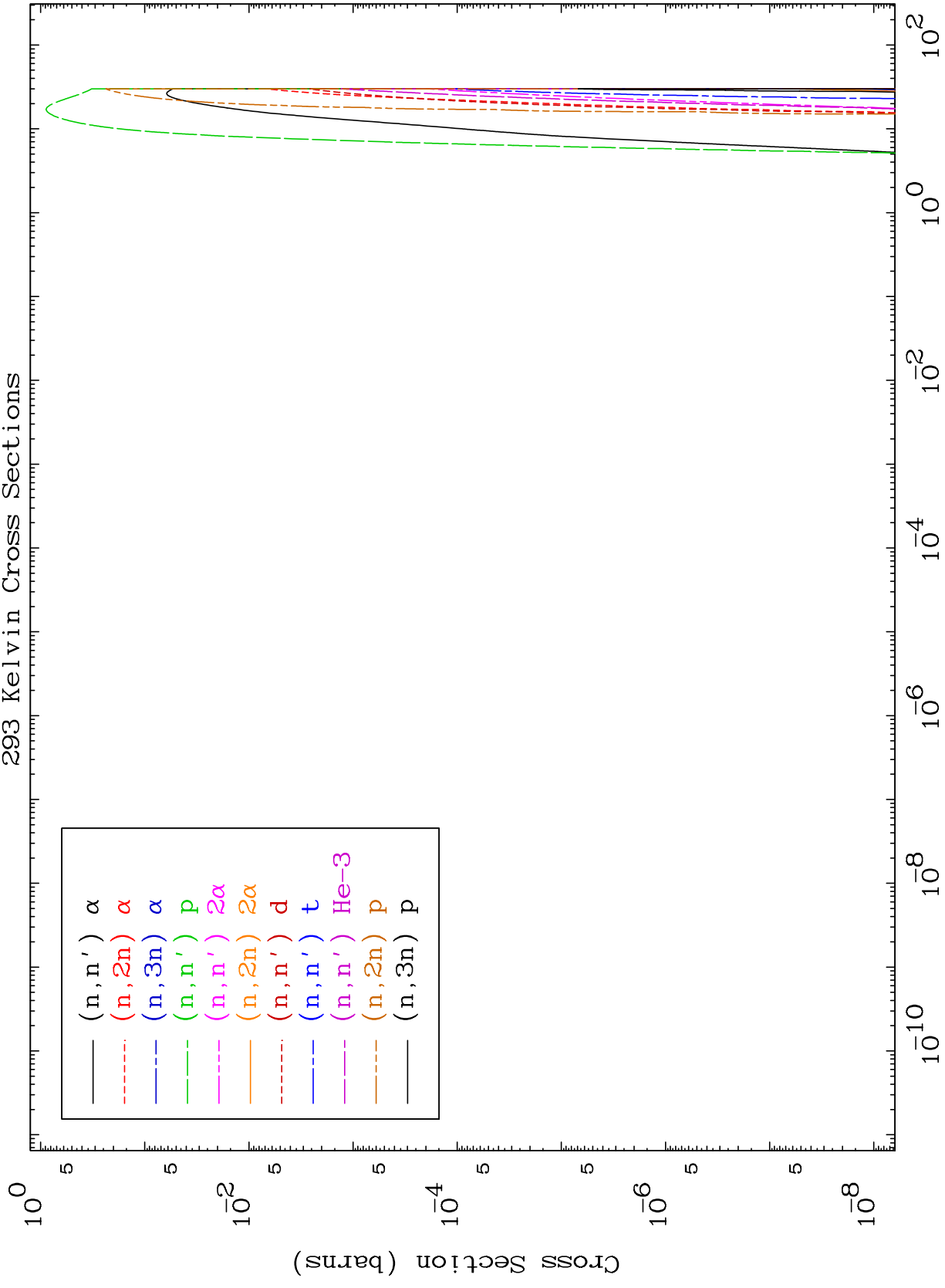
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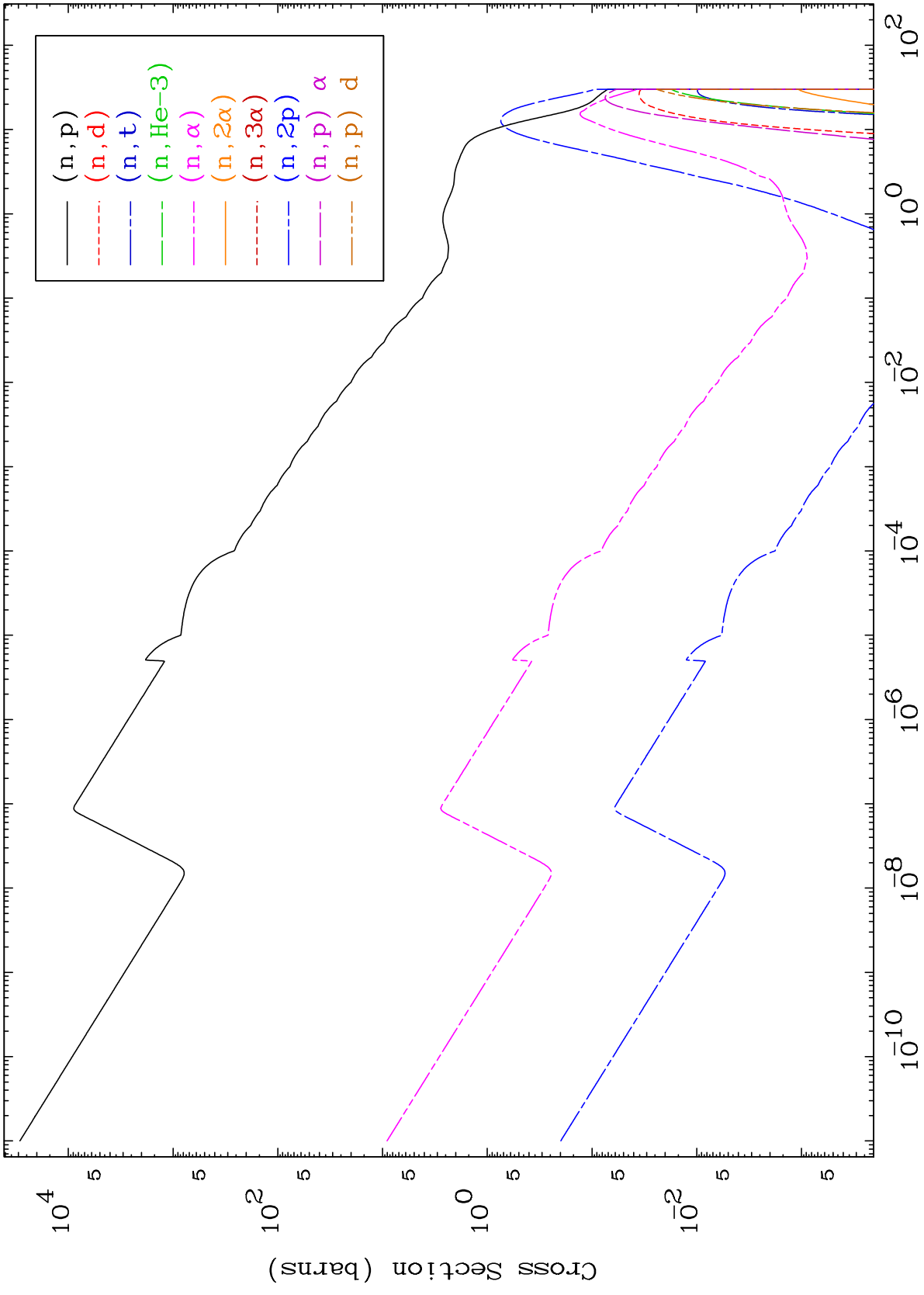
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293 Kelvin Cross Sections

66-Dy-145



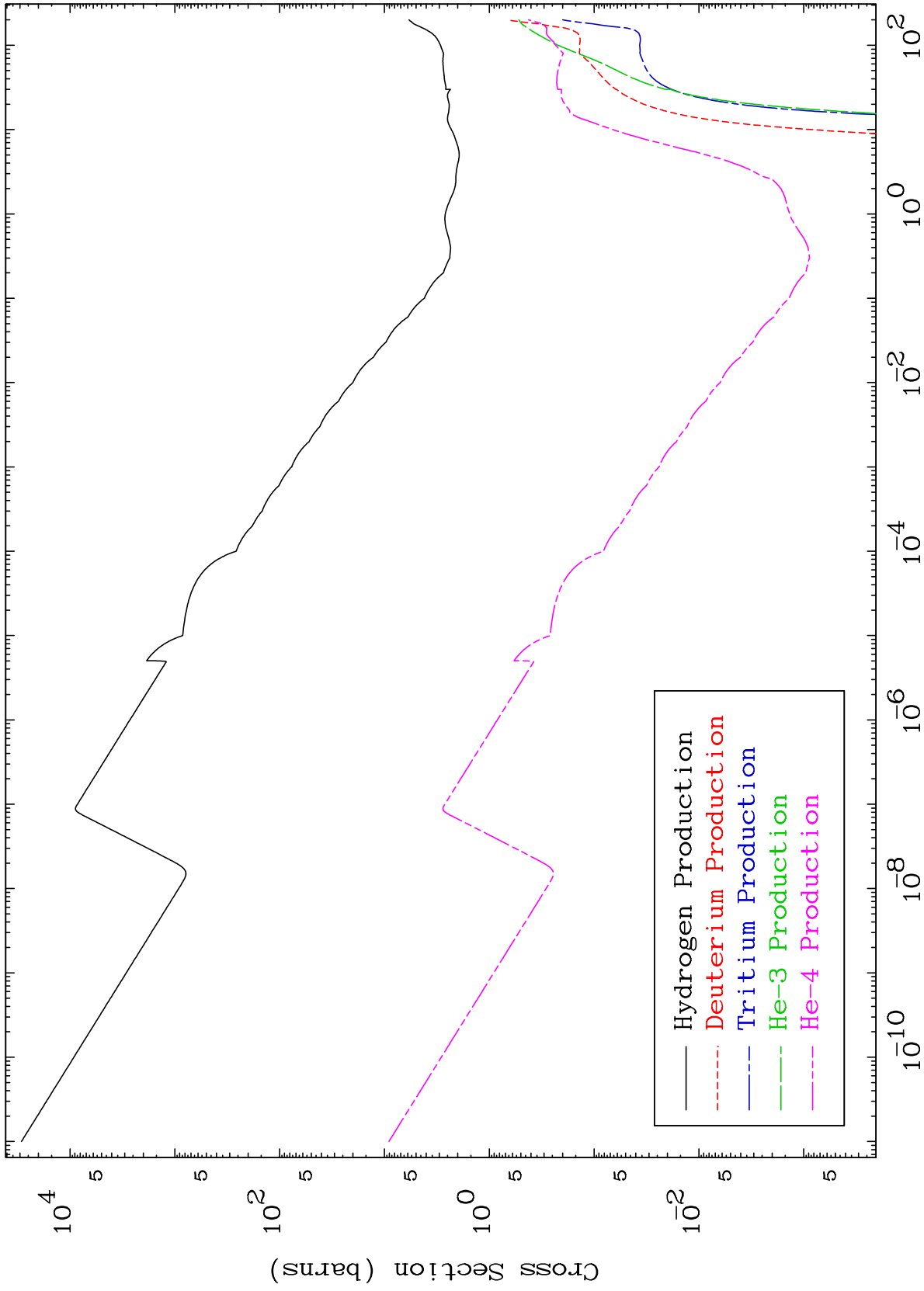
66-Dy-145



MAT 6593

Particle Production  
293 Kelvin Cross Sections

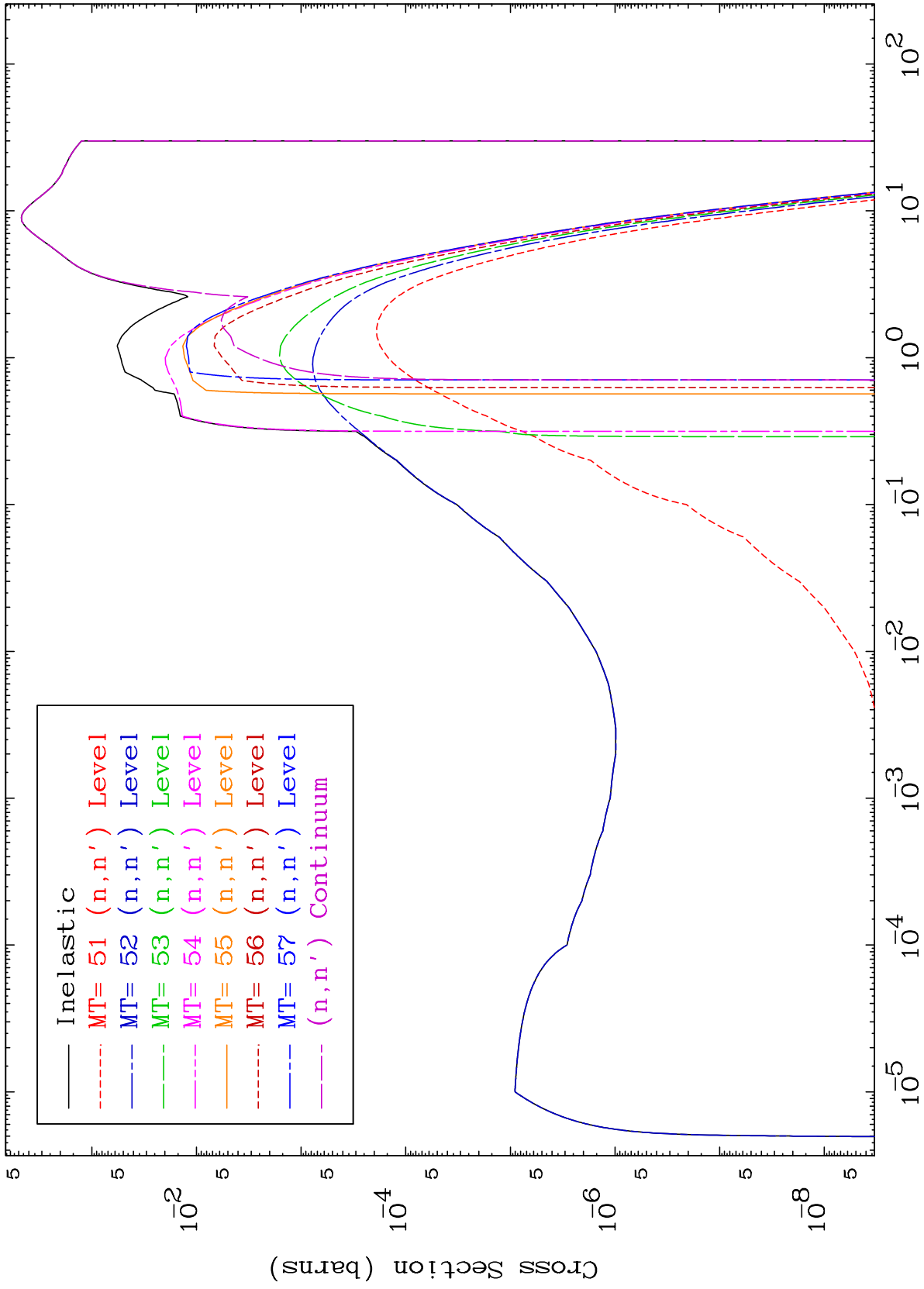
66-Dy-145



MAT 6593

293 Kelvin Cross Sections

66-Dy-145

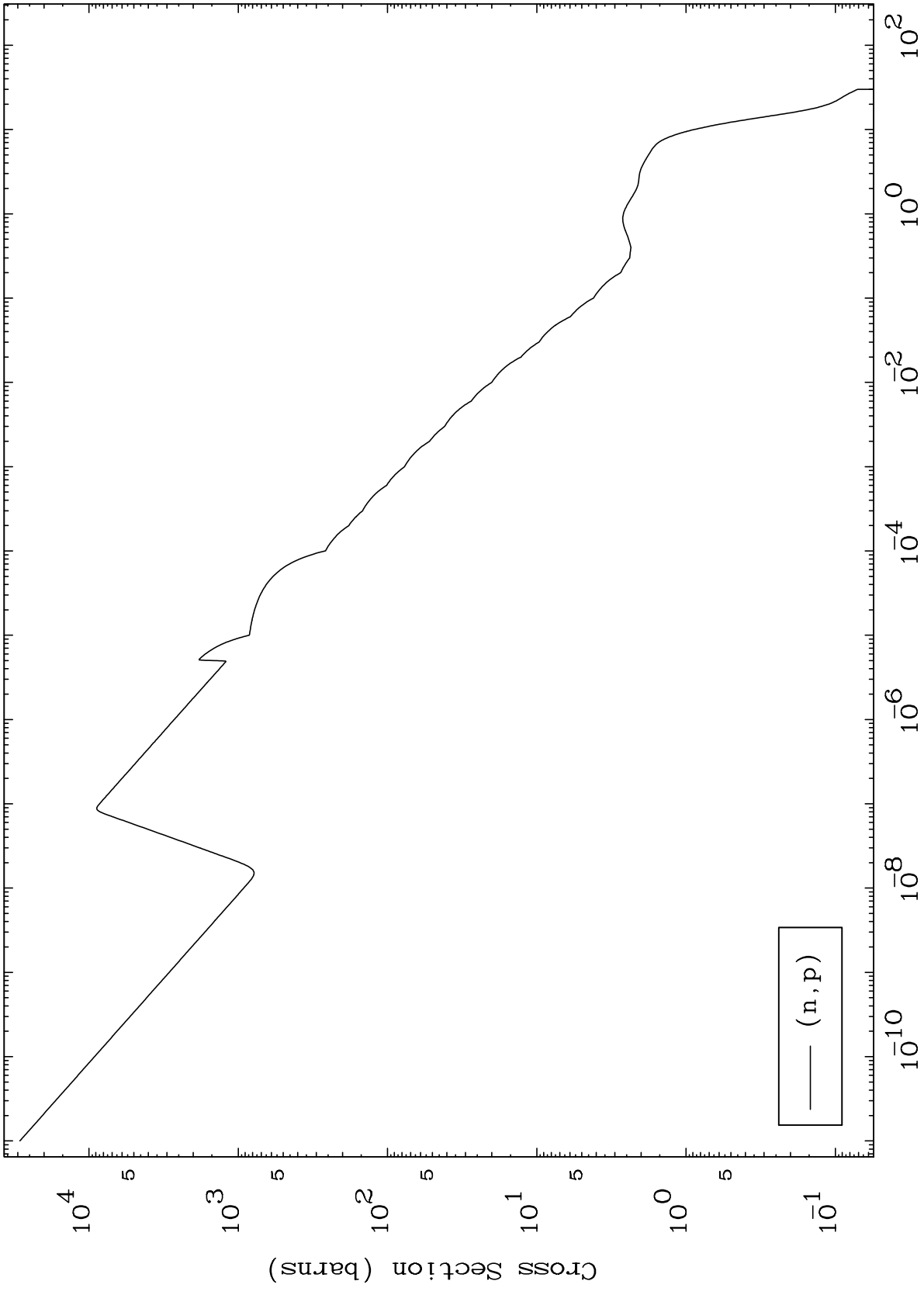




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(n,p) Levels  
293 Kelvin Cross Sections

66-Dy-145

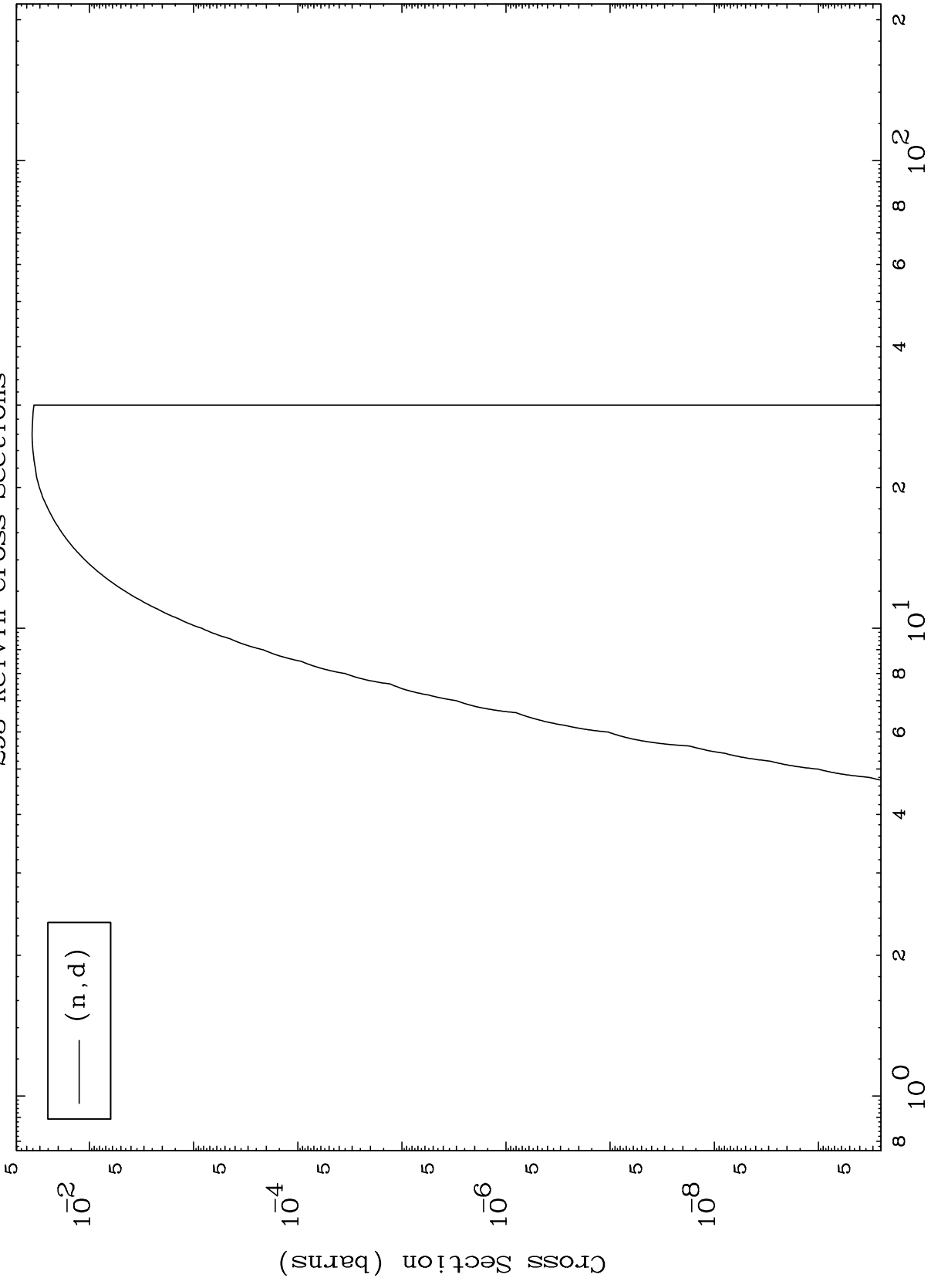


66-Dy-145

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(n,d) Levels  
293 Kelvin Cross Sections

66-Dy-145



9

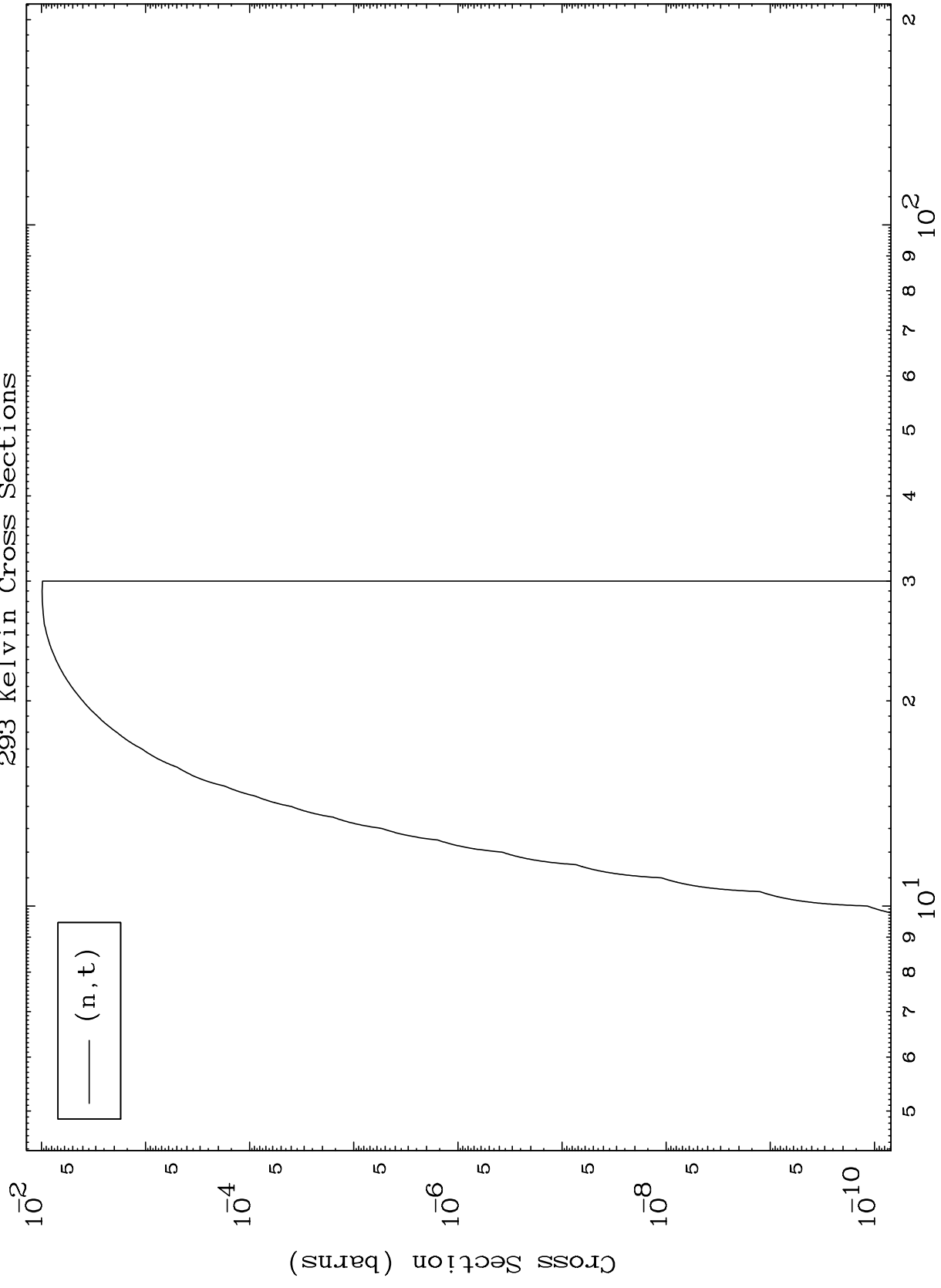
Incident Energy (MeV)

66-Dy-145

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(n,t) Levels  
293 Kelvin Cross Sections

66-Dy-145



10

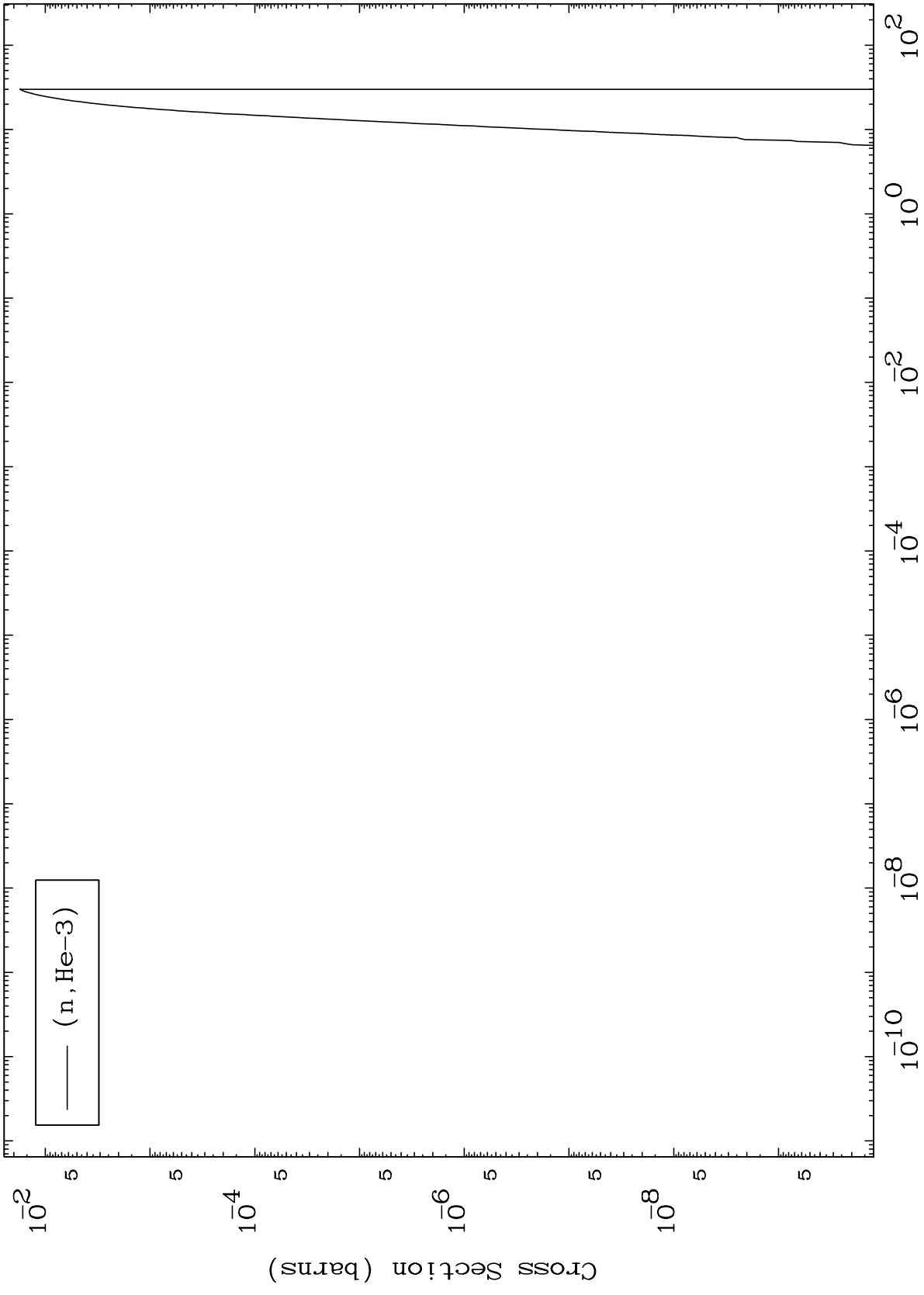
Incident Energy (MeV)

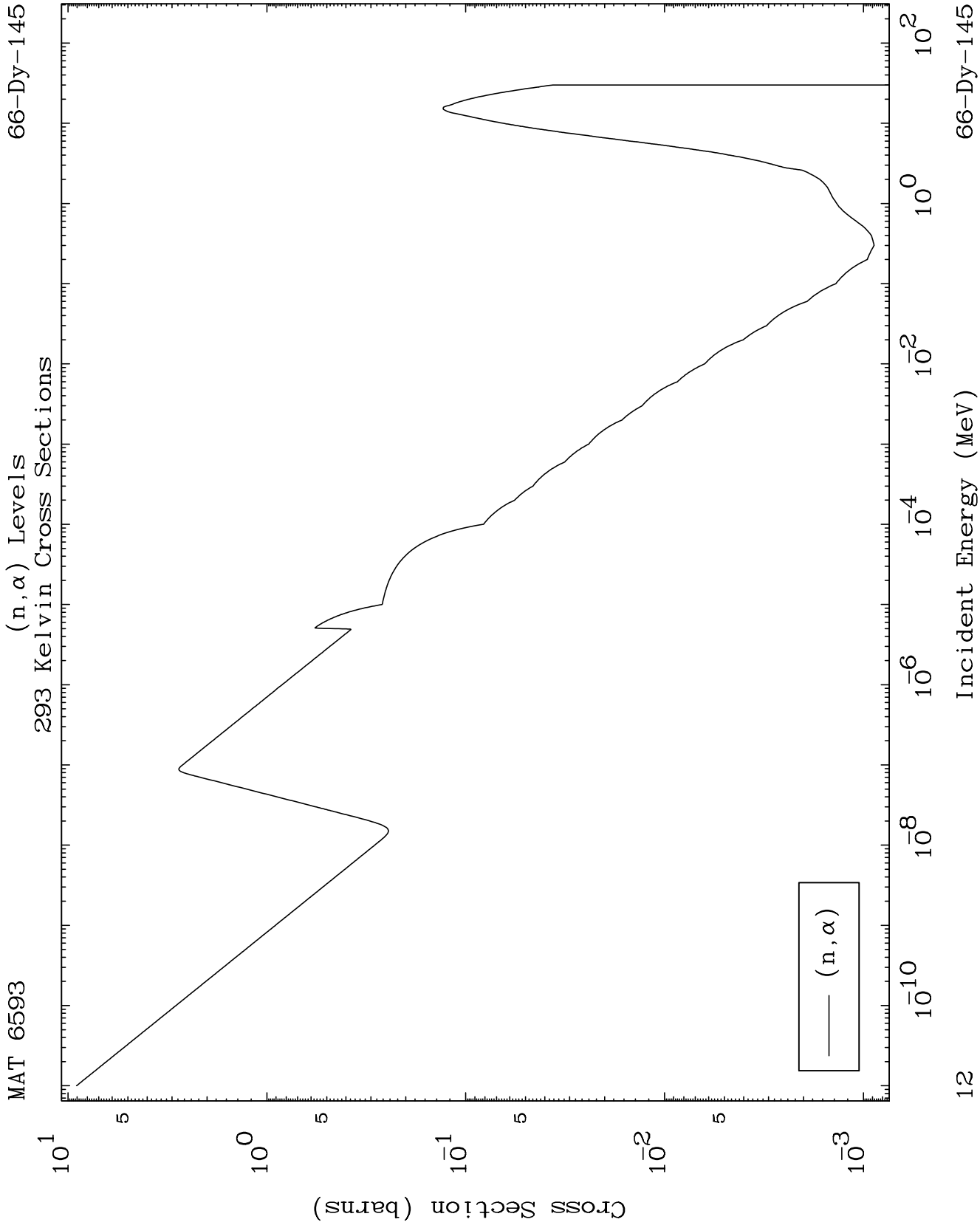
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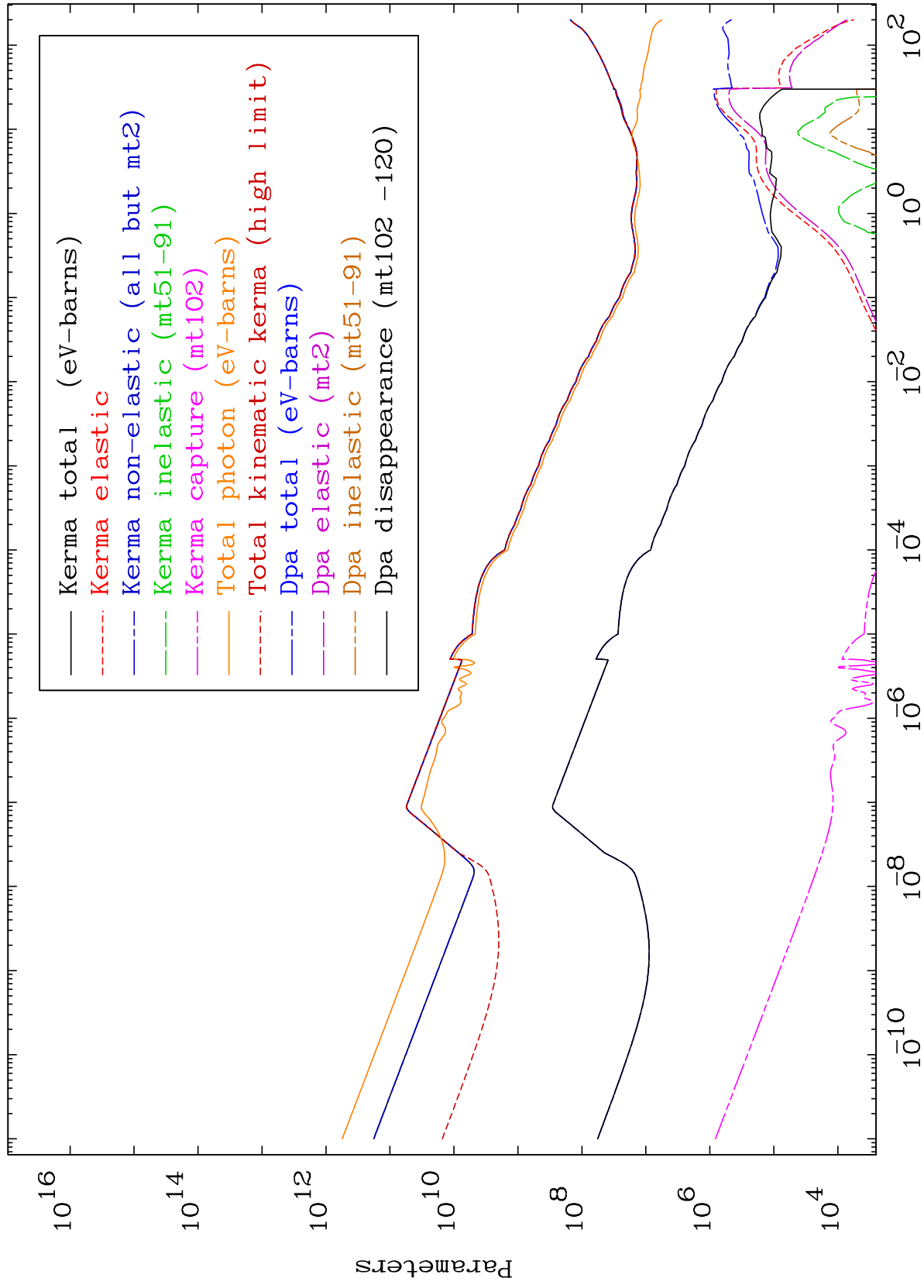
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(n,He3) Levels  
293 Kelvin Cross Sections

66-Dy-145





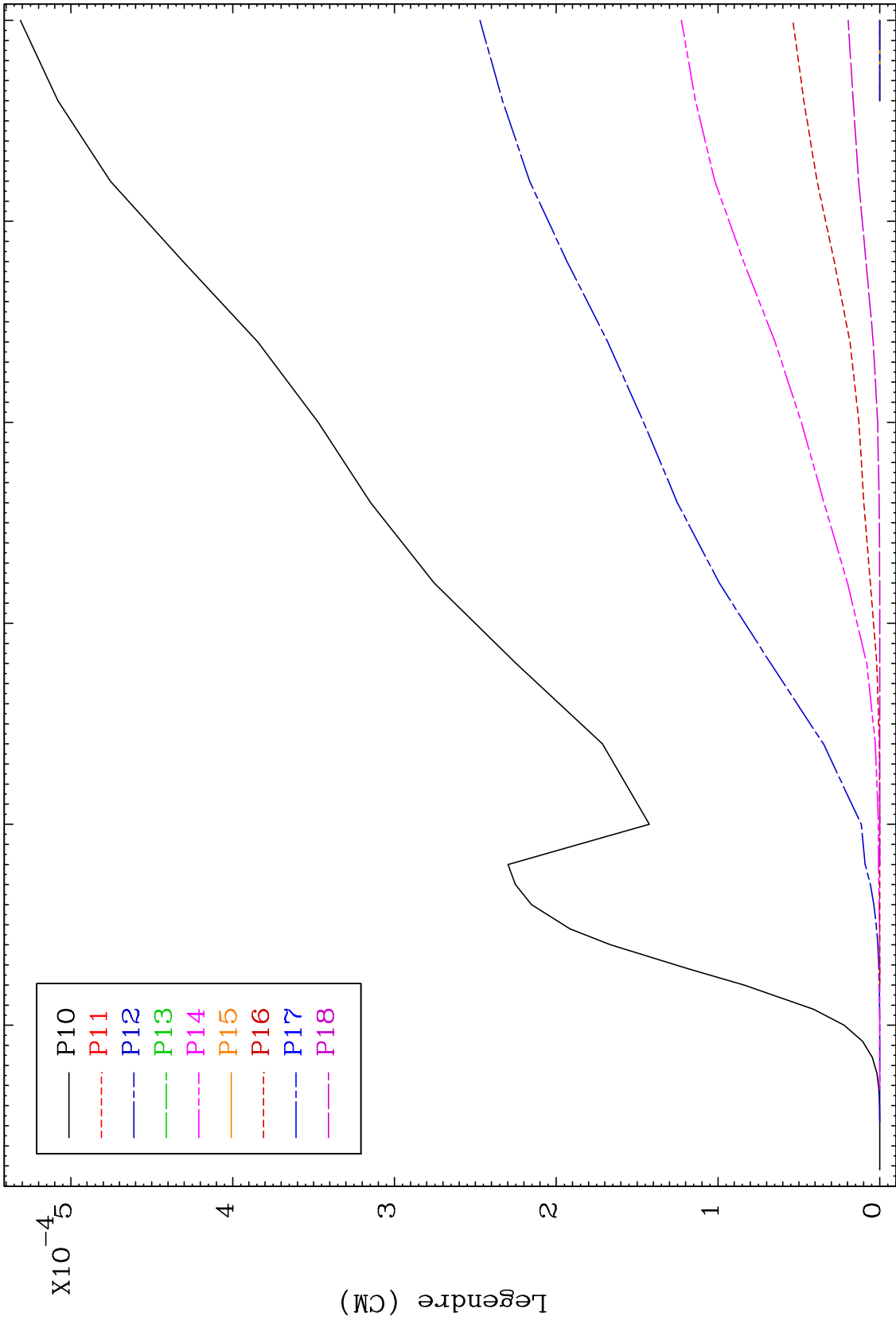




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Elastic Legendre Coefficients

66-Dy-145



15

Incident Energy (MeV)

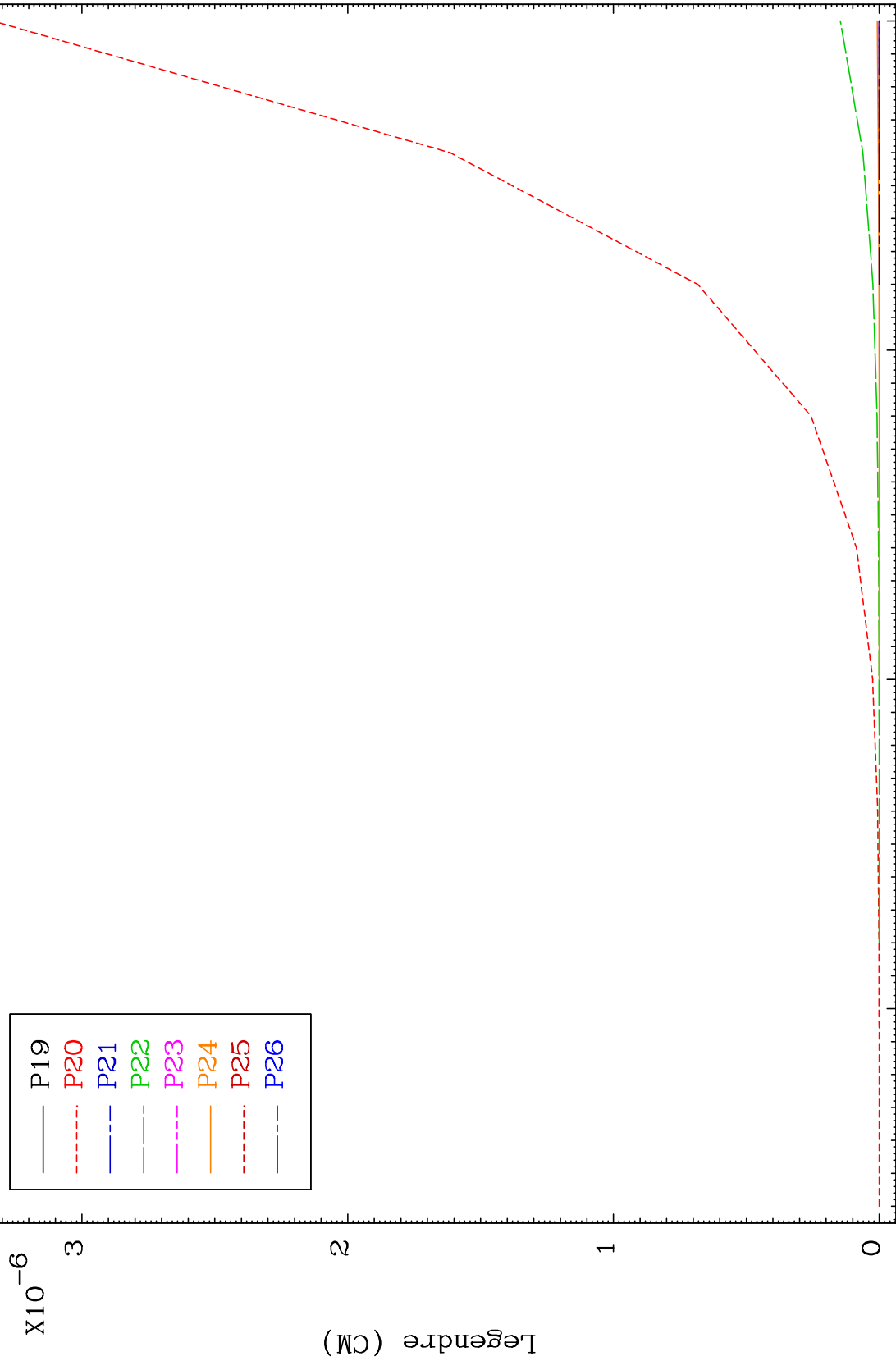
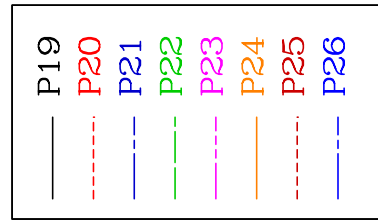
66-Dy-145



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Elastic Legendre Coefficients

66-Dy-145



16

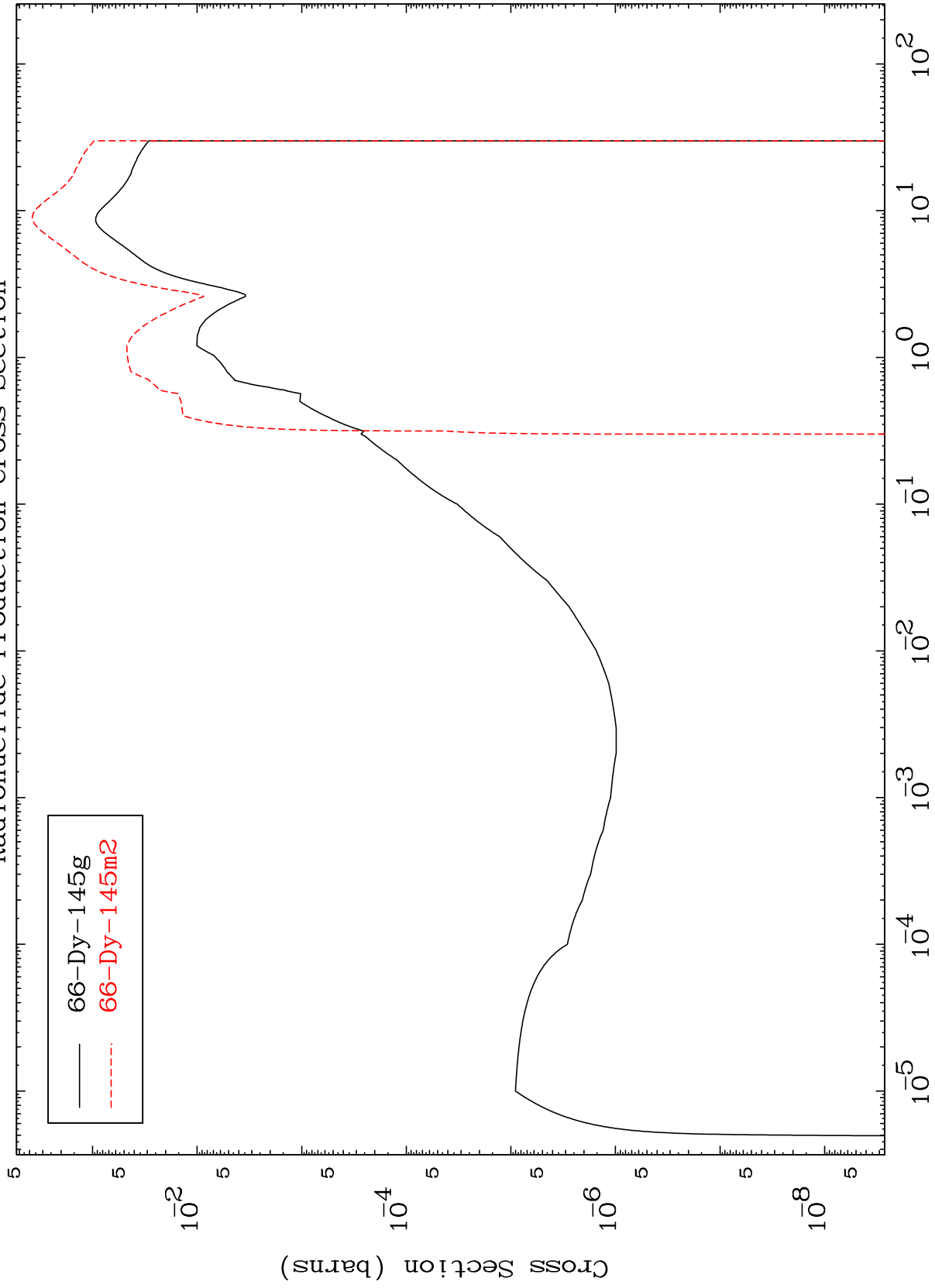
Incident Energy (MeV)

66-Dy-145

MAT 6593

Inelastic  
Radionuclide Production Cross Section

66-Dy-145

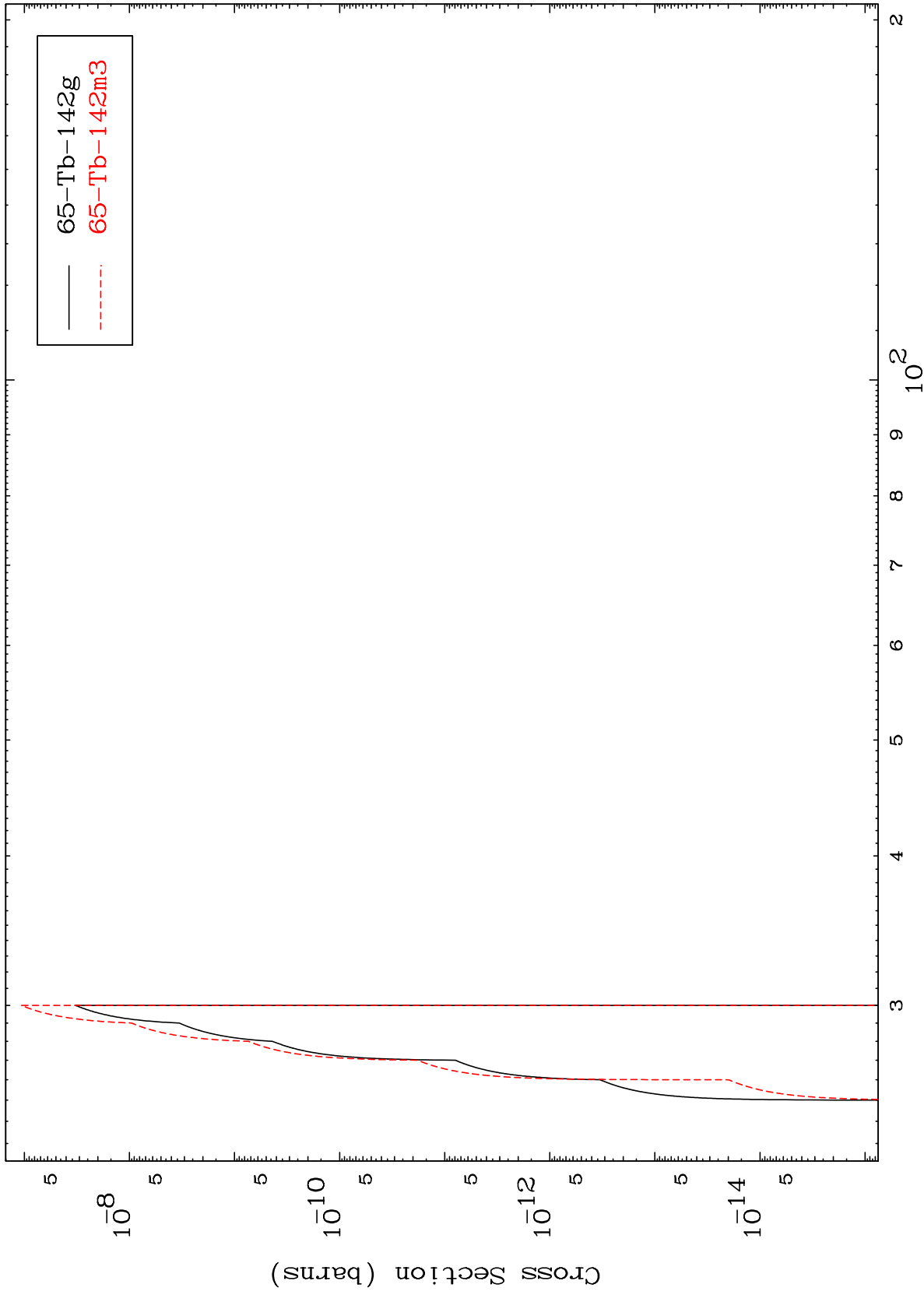


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(n,2n) d

66-Dy-145

Radionuclide Production Cross Section



18

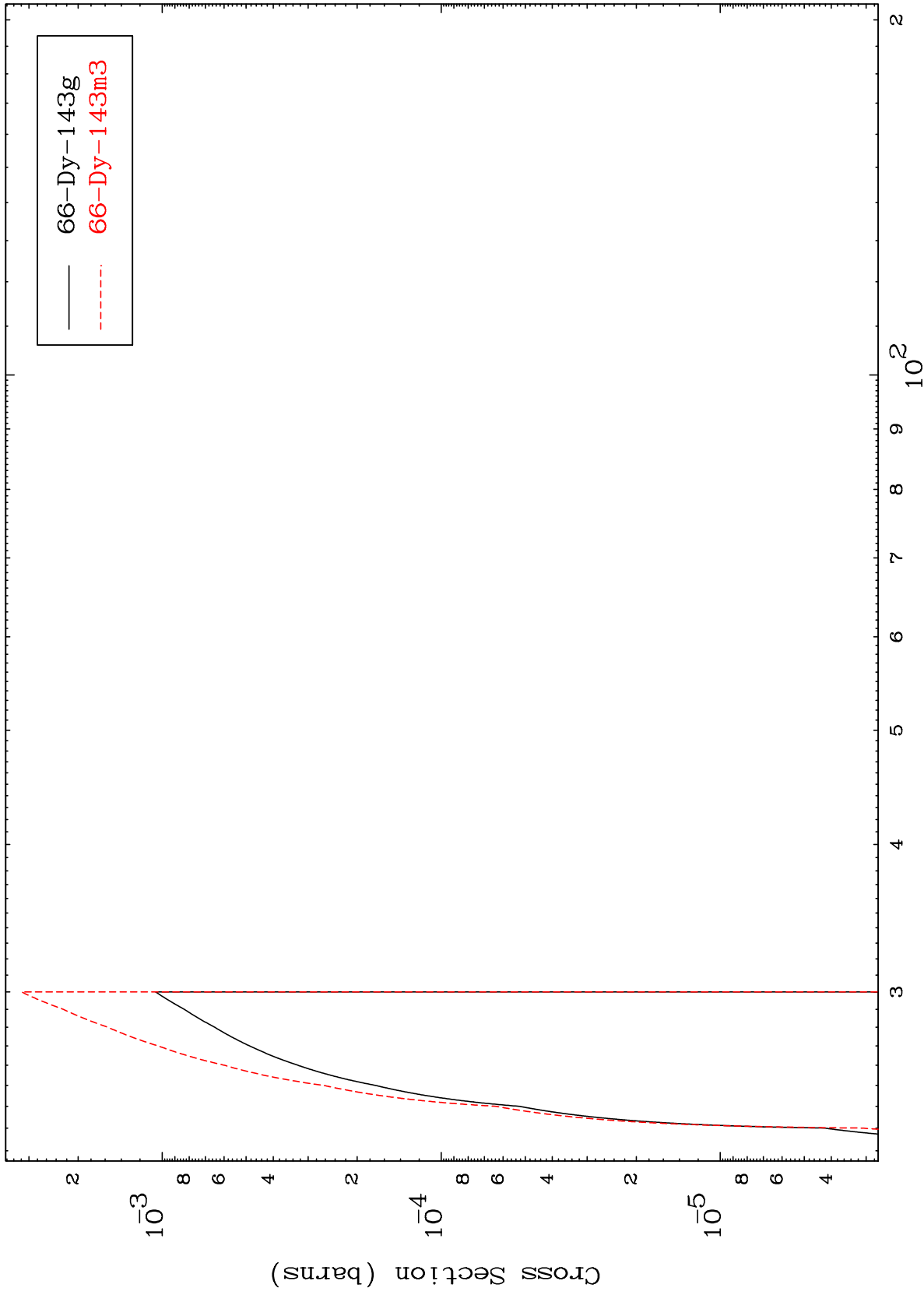
Incident Energy (MeV)

66-Dy-145

MAT 6593

66-Dy-145

(n,3n)  
Radionuclide Production Cross Section



19

Incident Energy (MeV)

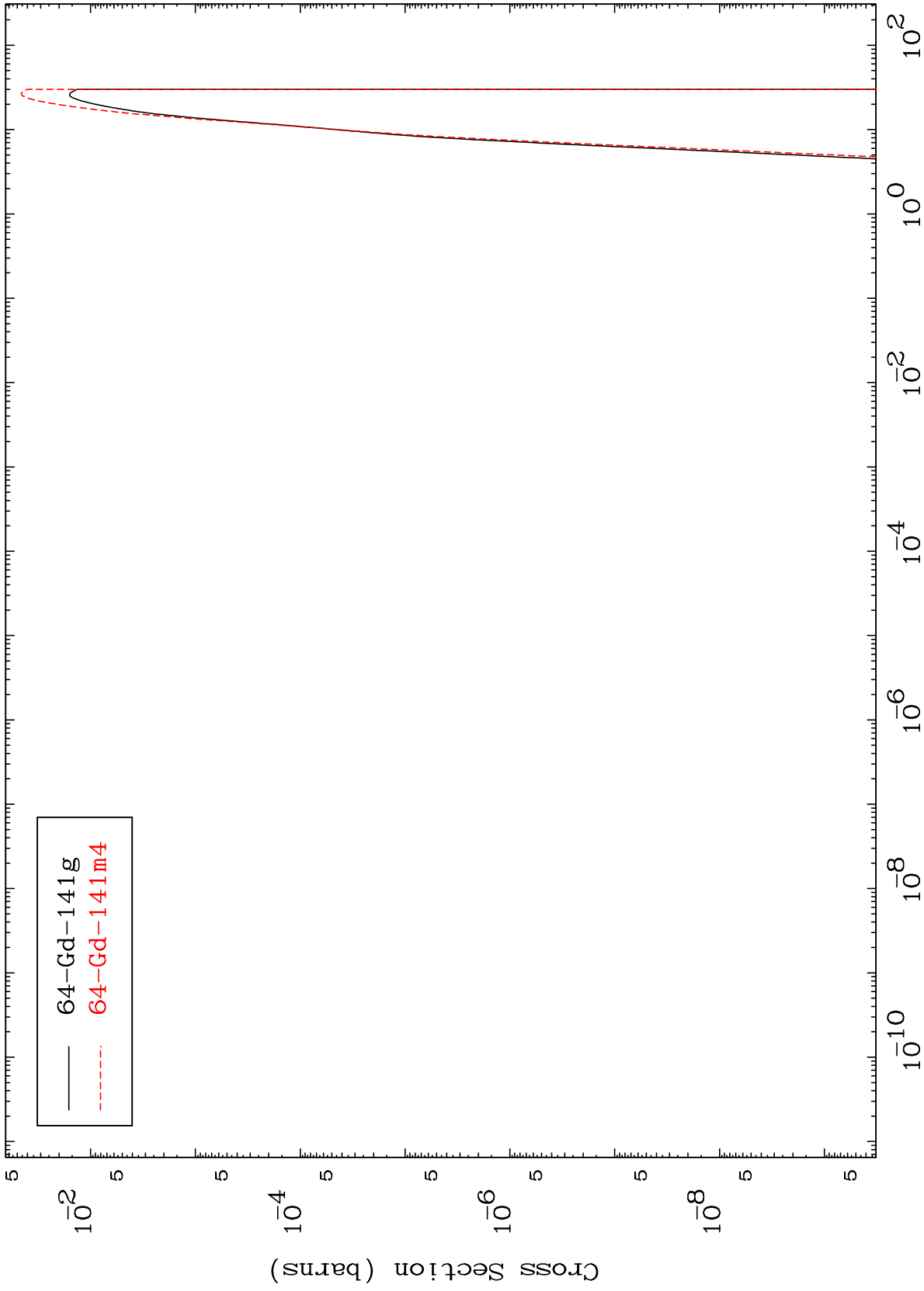
66-Dy-145

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(n,n')  $\alpha$

66-Dy-145

Radionuclide Production Cross Section



20

Incident Energy (MeV)

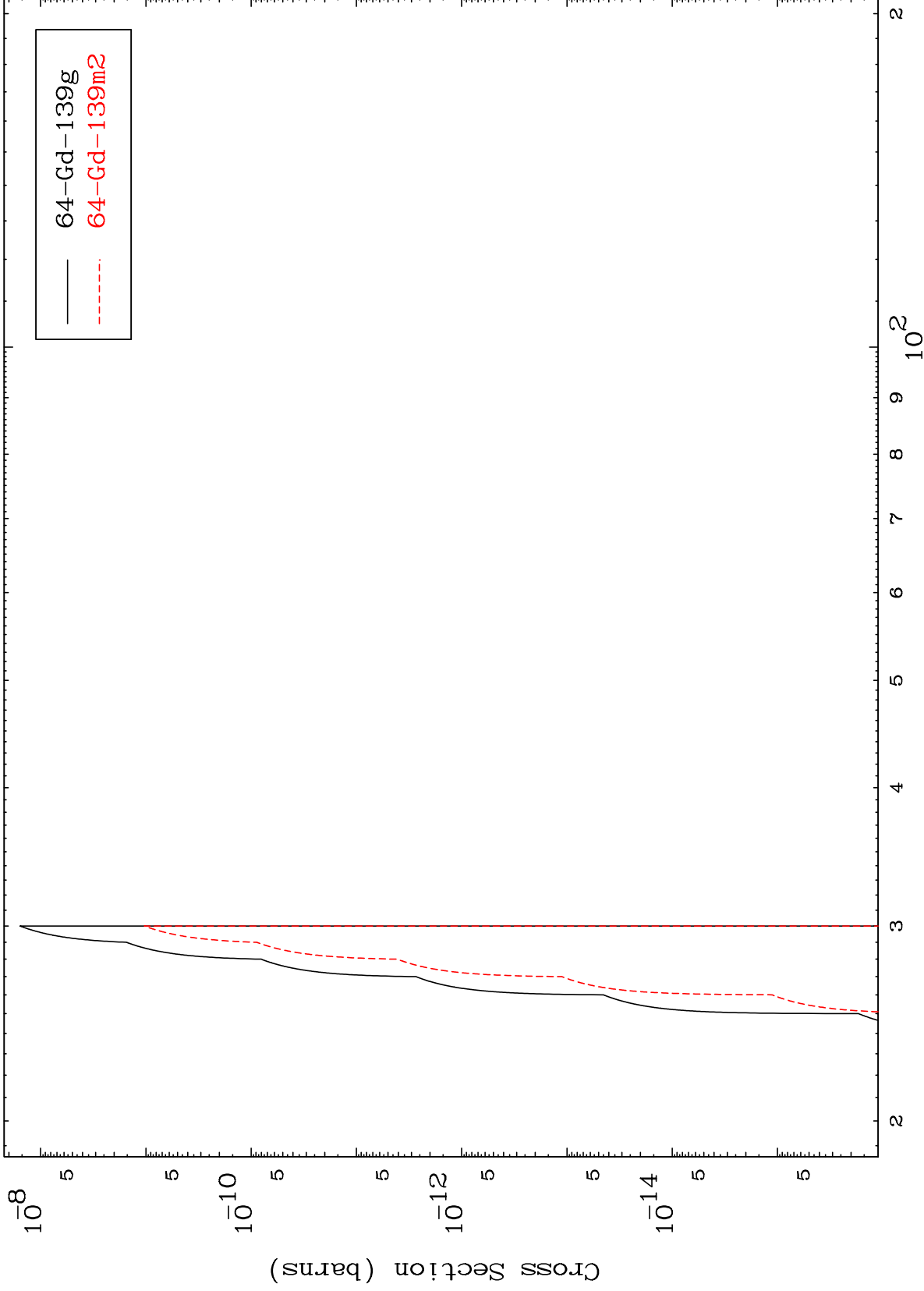
66-Dy-145

MAT 6593

(n,3n)  $\alpha$

66-Dy-145

Radionuclide Production Cross Section



21

Incident Energy (MeV)

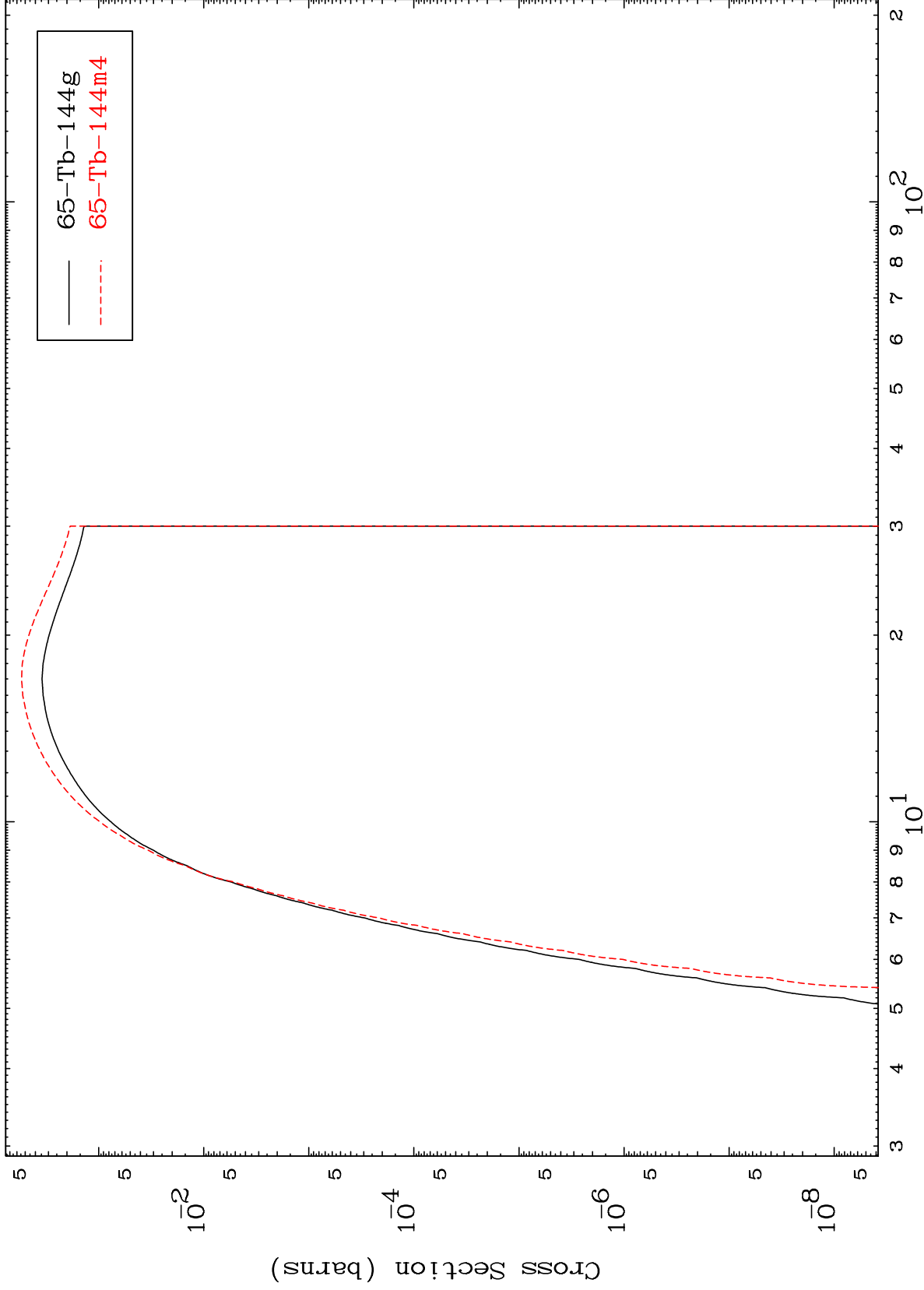
66-Dy-145

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(n,n') p

66-Dy-145

Radionuclide Production Cross Section



22

Incident Energy (MeV)

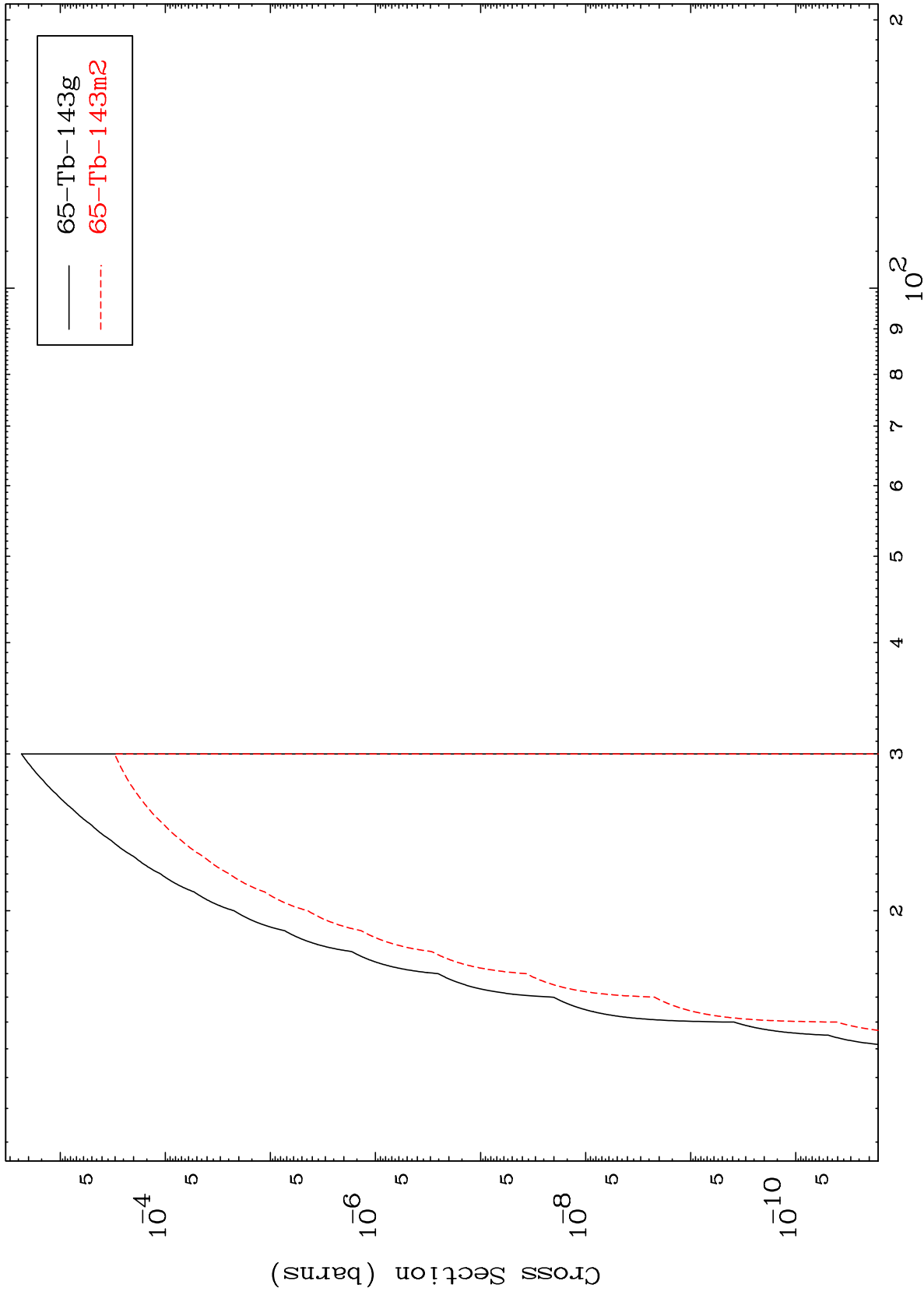
66-Dy-145

MAT 6593

(n,n') d

66-Dy-145

Radionuclide Production Cross Section



23

Incident Energy (MeV)

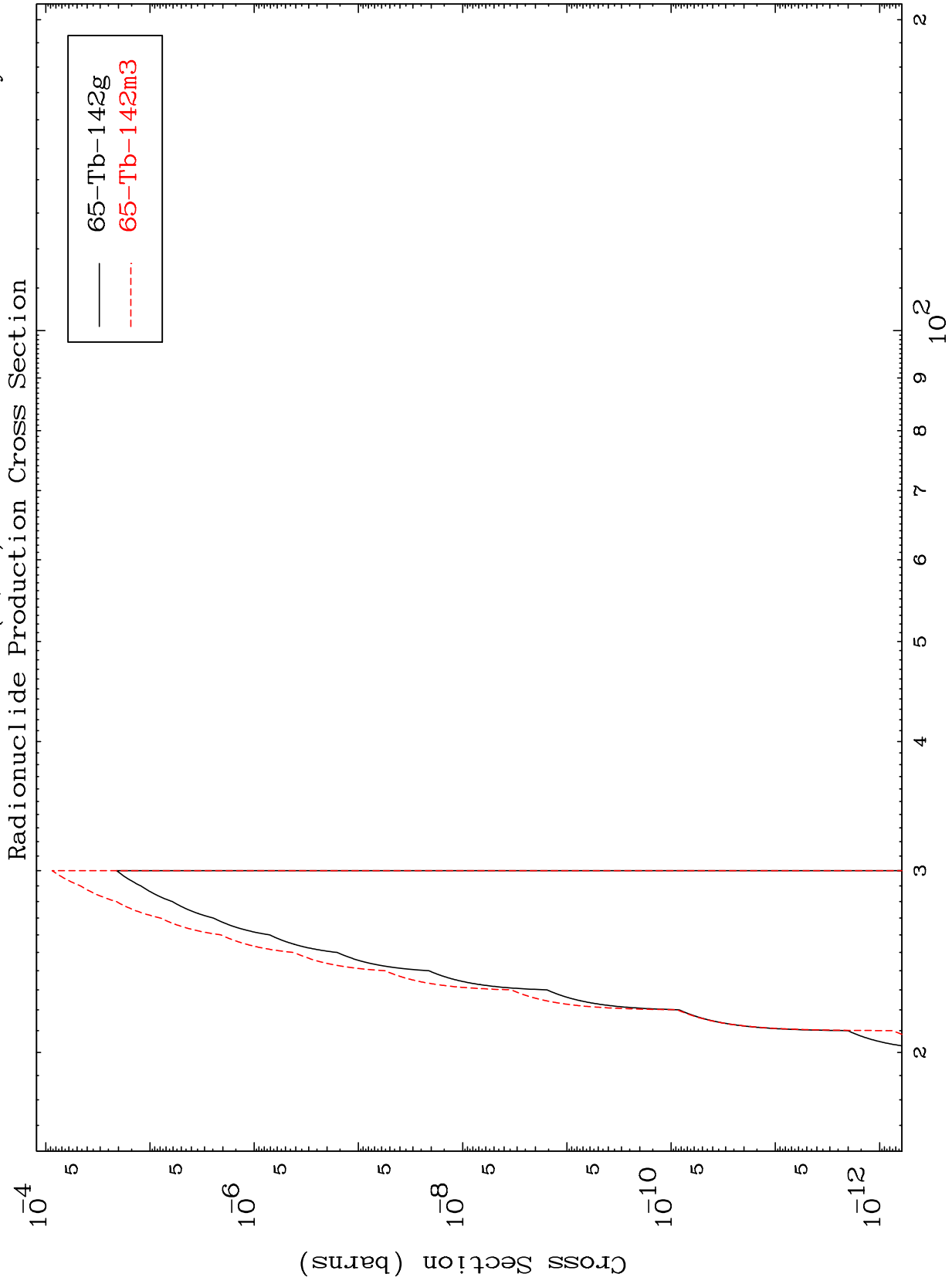
66-Dy-145



MAT 6593

(n,n') t

66-Dy-145



24

Incident Energy (MeV)

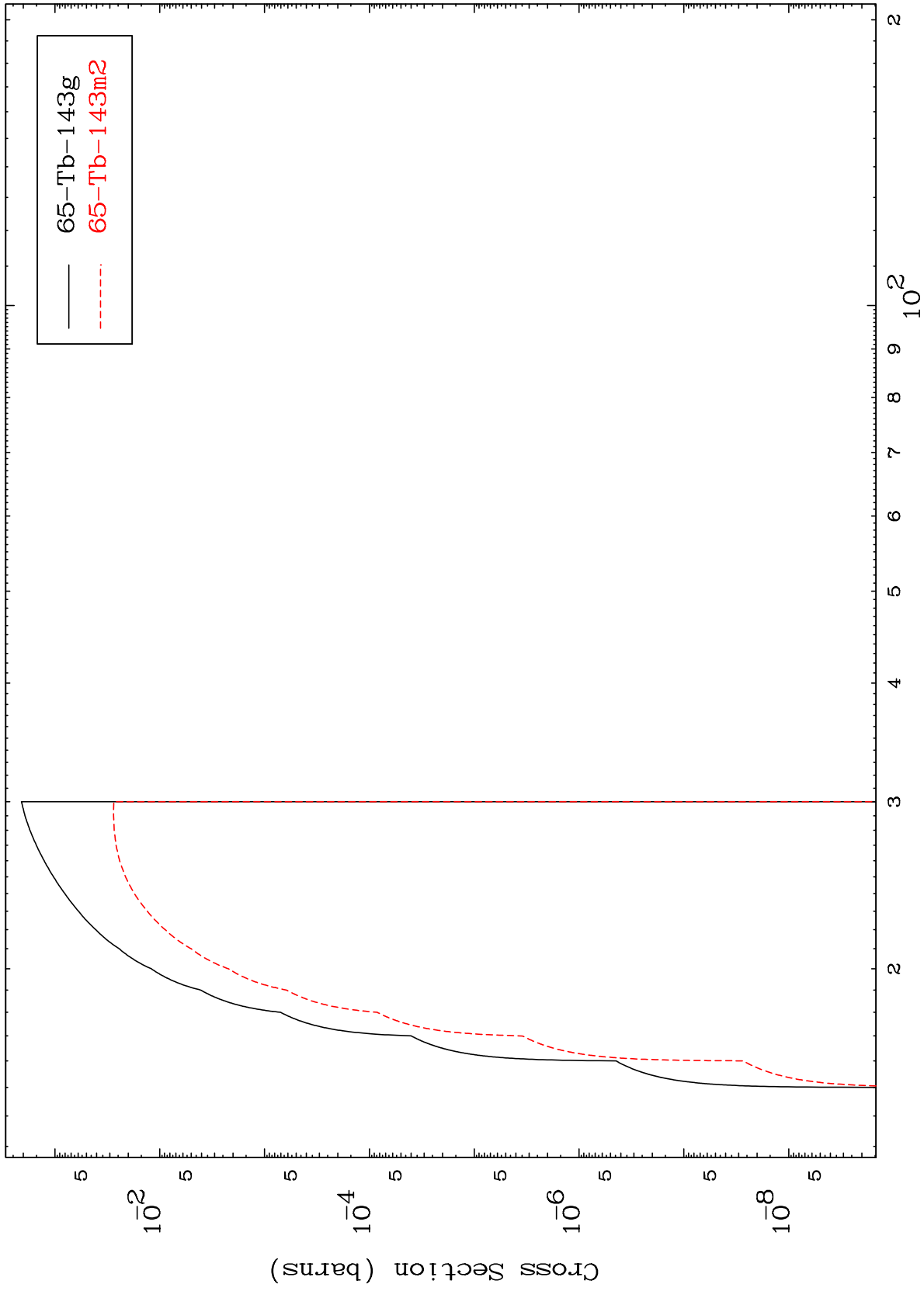
66-Dy-145

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(n,2n) p

66-Dy-145

Radionuclide Production Cross Section



25

Incident Energy (MeV)

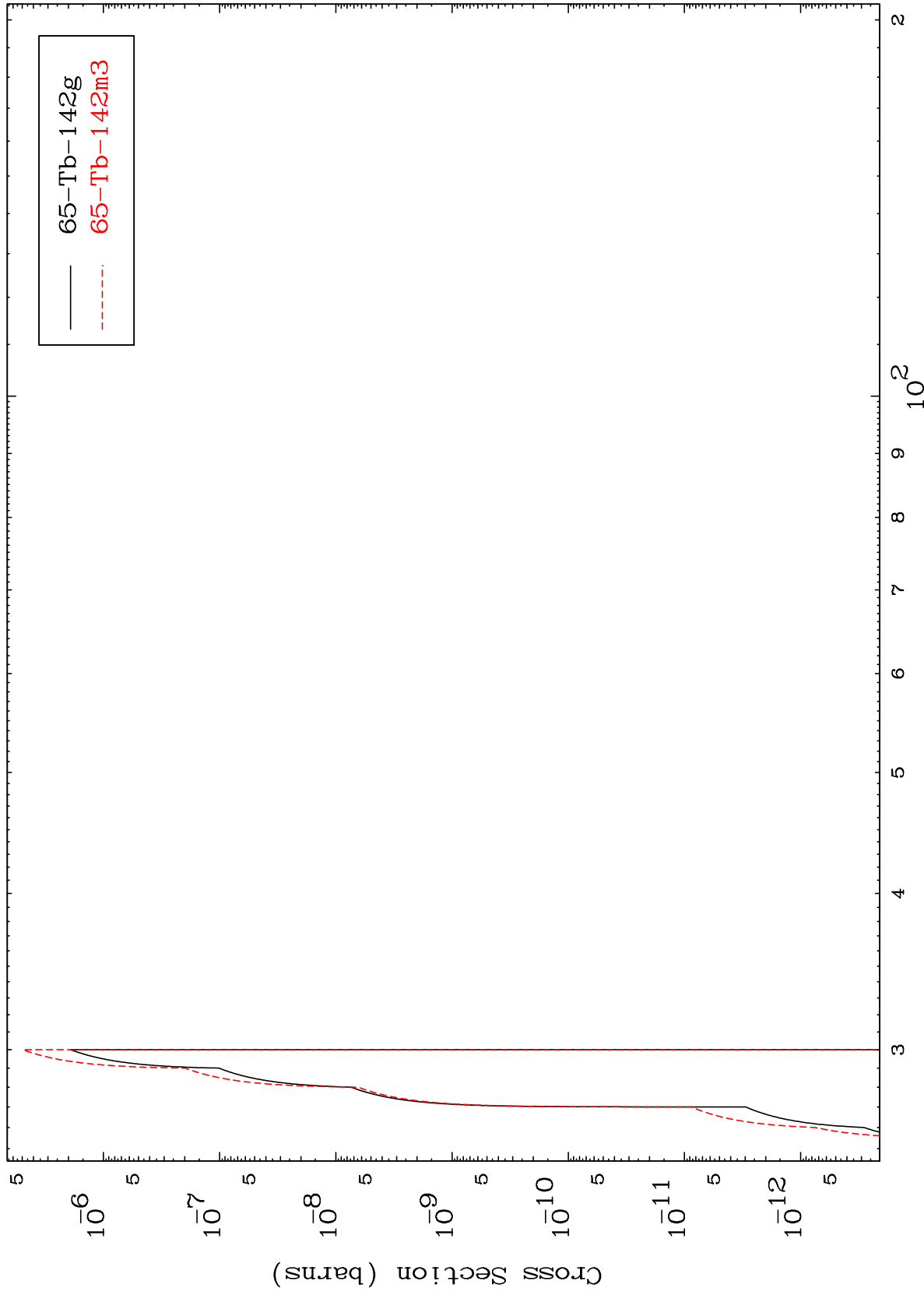
66-Dy-145

MAT 6593

(n,3n) p

66-Dy-145

Radionuclide Production Cross Section



26

Incident Energy (MeV)

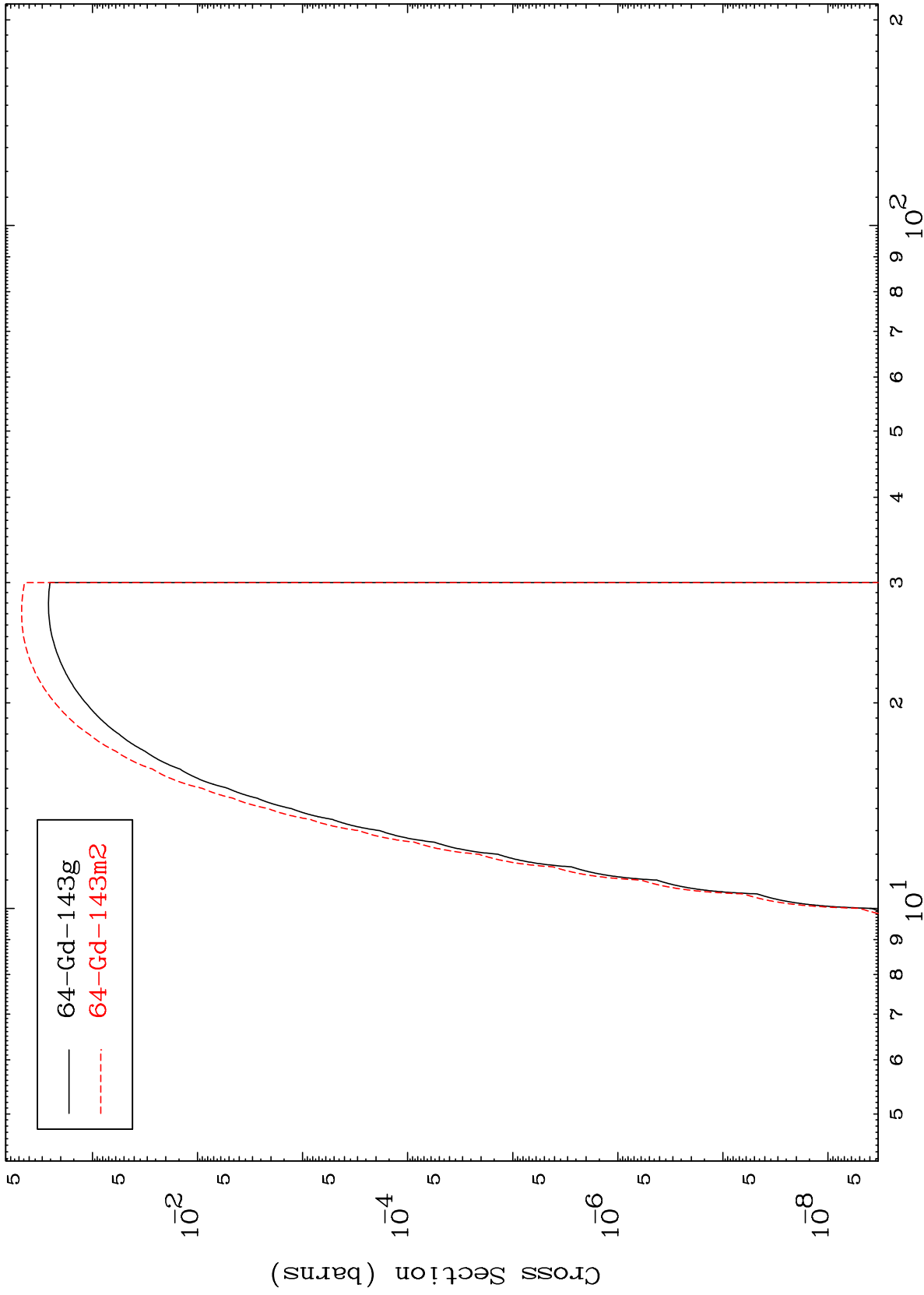
66-Dy-145

MAT 6593

(n,2n) p

66-Dy-145

Radionuclide Production Cross Section



27

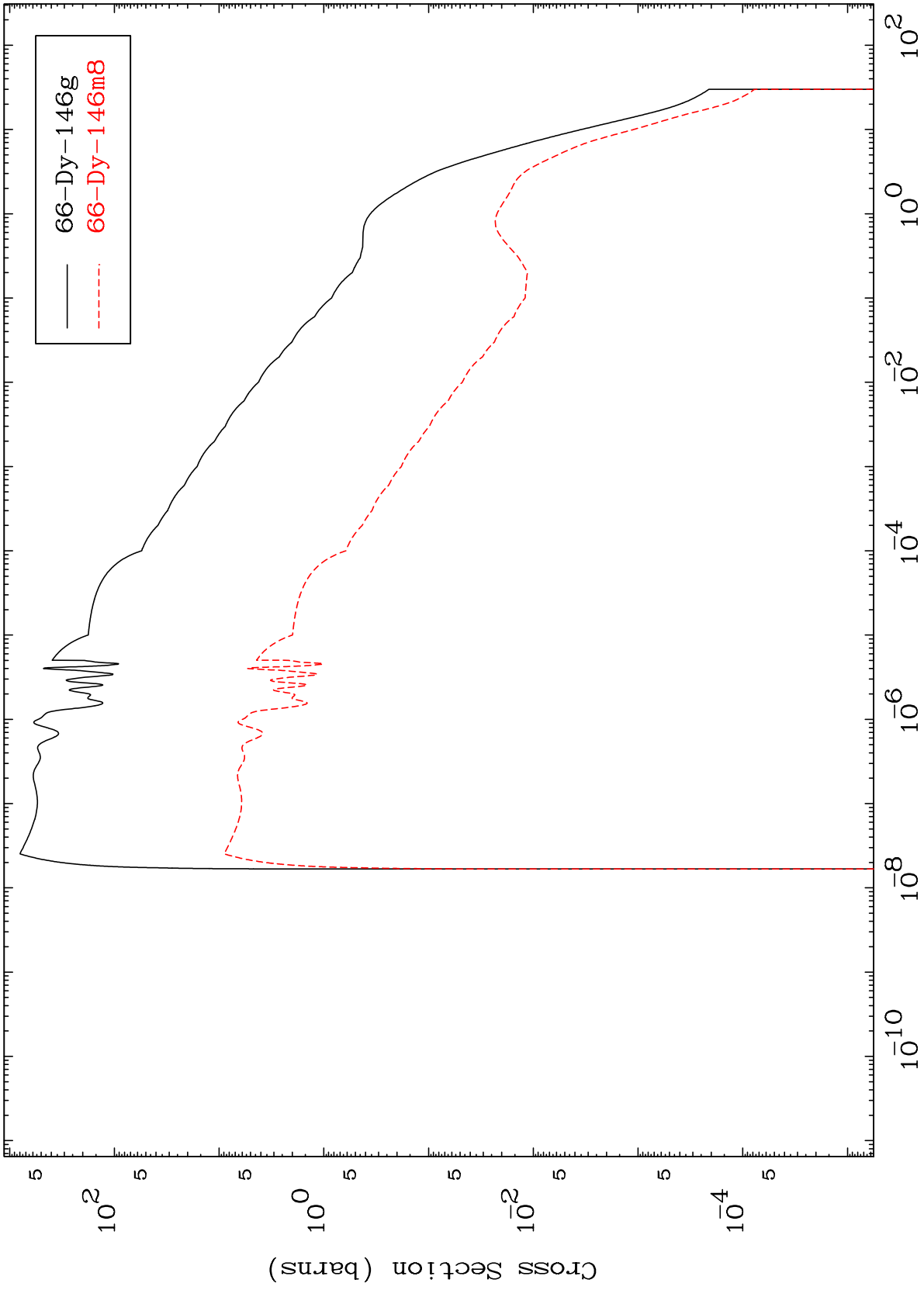
Incident Energy (MeV)

66-Dy-145

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66-Dy-145

(n,  $\gamma$ )  
Radionuclide Production Cross Section



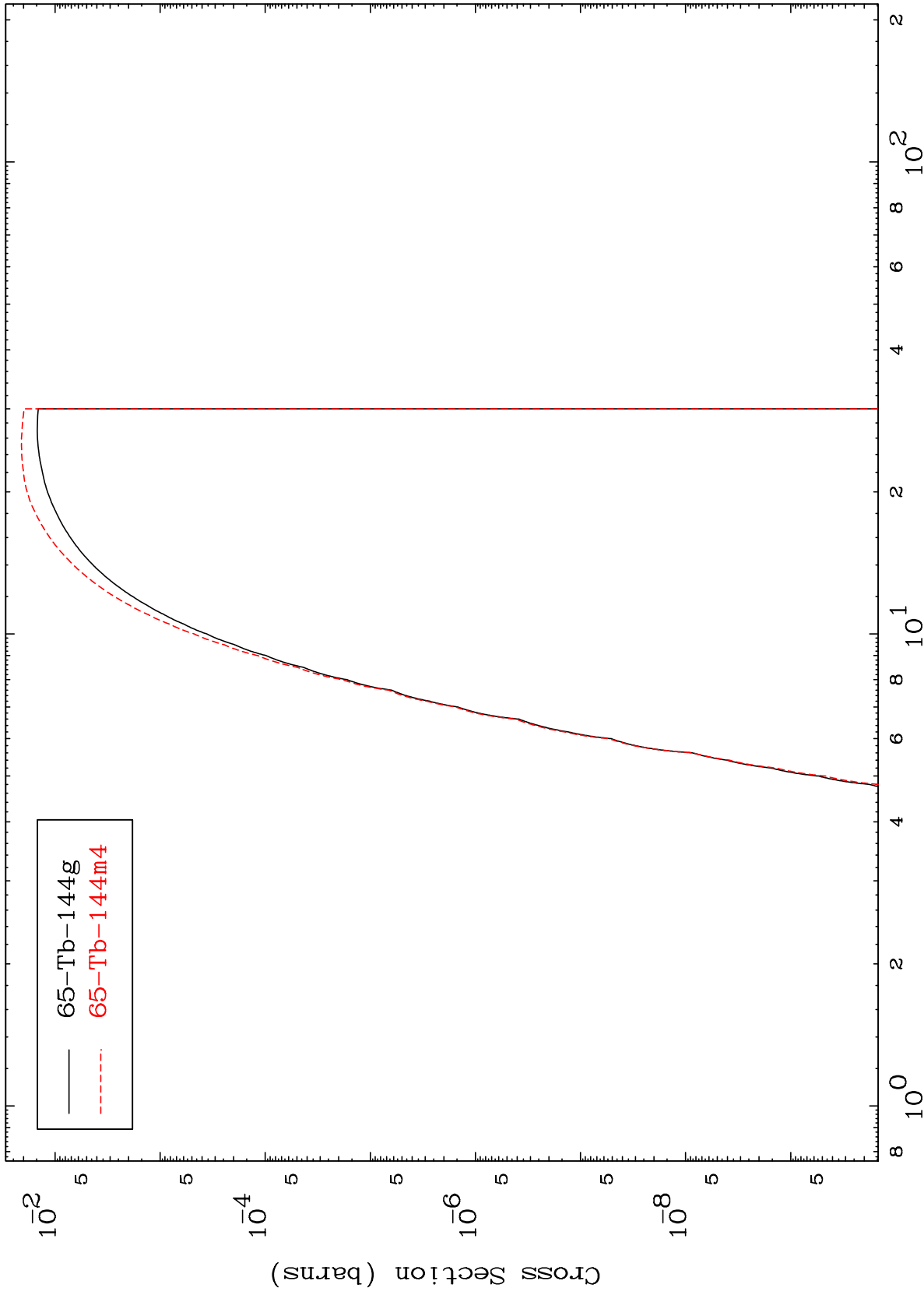
28

66-Dy-145

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66-Dy-145

(n,d)  
Radionuclide Production Cross Section



29

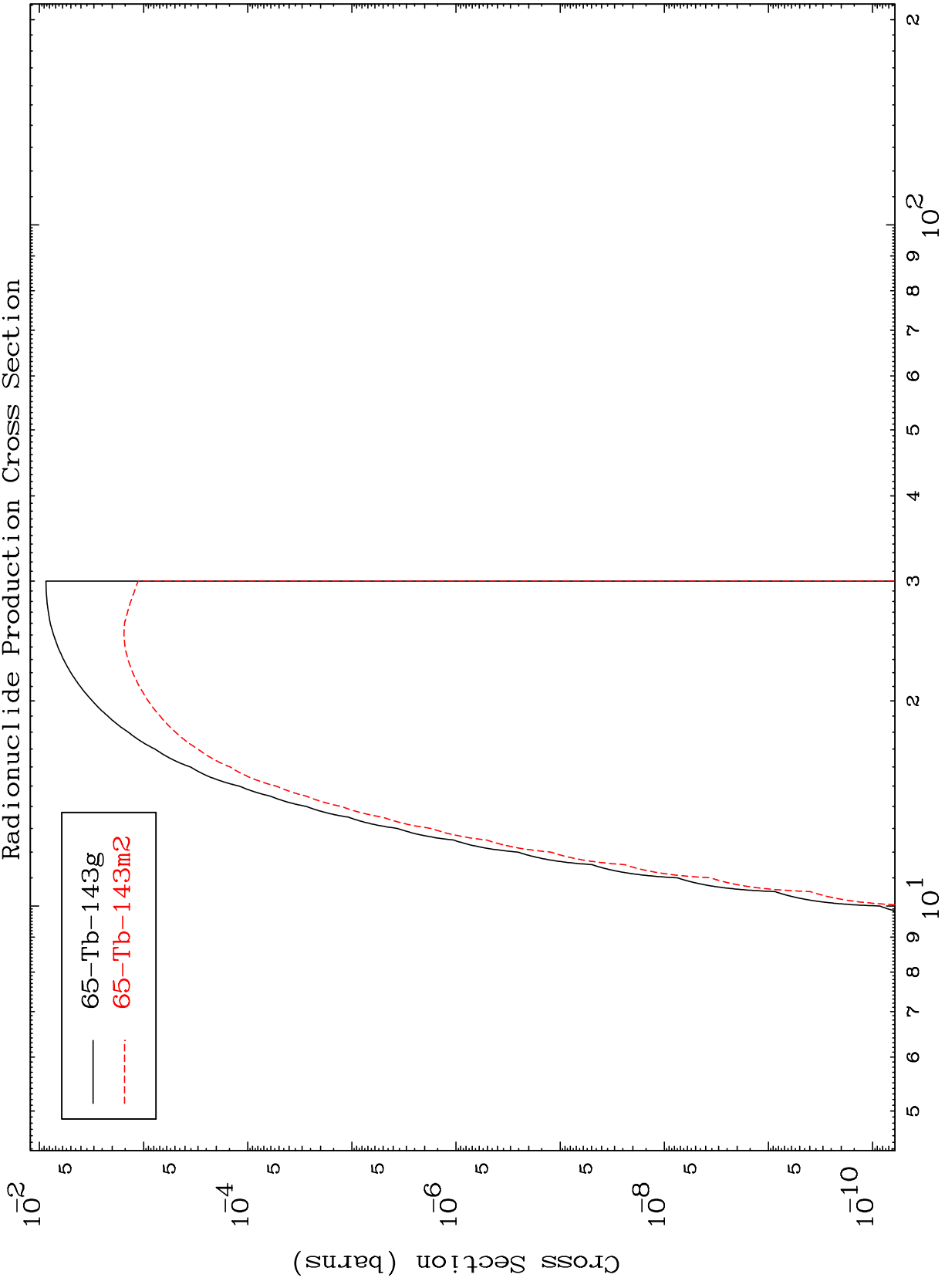
Incident Energy (MeV)

66-Dy-145

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66-Dy-145

Radionuclide Production Cross Section (n, t)



65-Tb-143g  
65-Tb-143m2

30

Incident Energy (MeV)

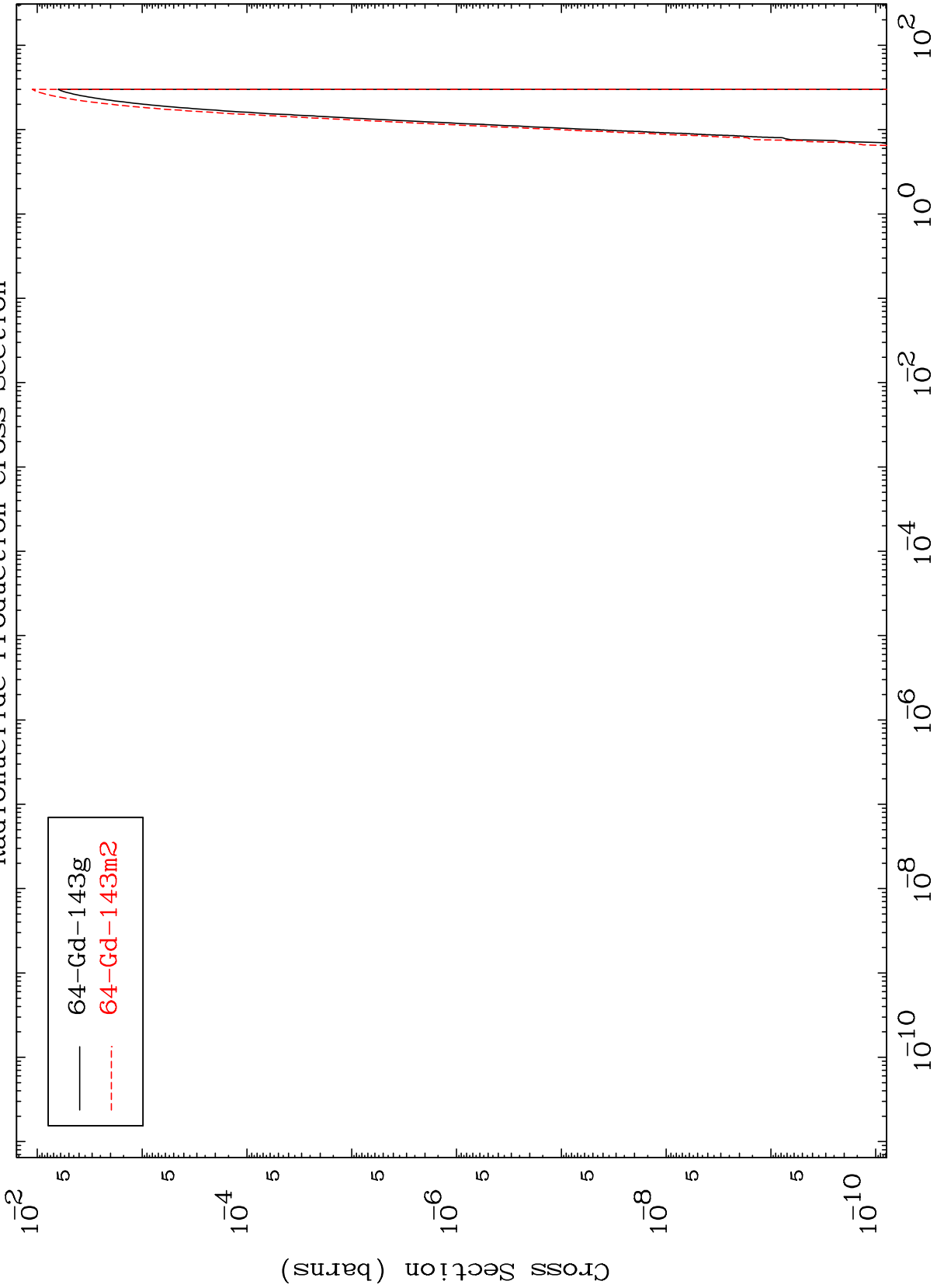
66-Dy-145

MAT 6593

(n, He-3)

66-Dy-145

Radionuclide Production Cross Section



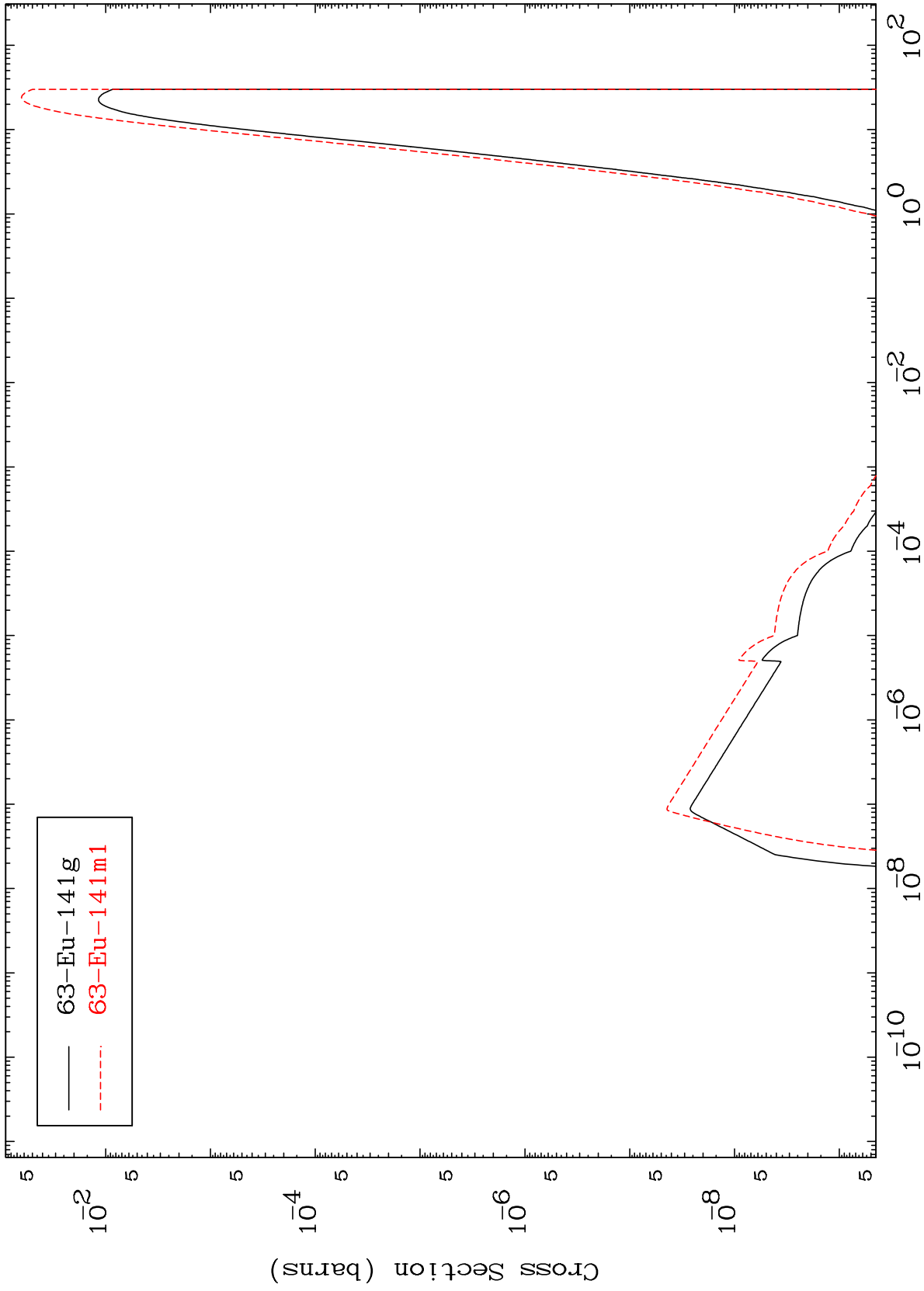


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(n,p)  $\alpha$

66-Dy-145

Radionuclide Production Cross Section



32

Incident Energy (MeV)

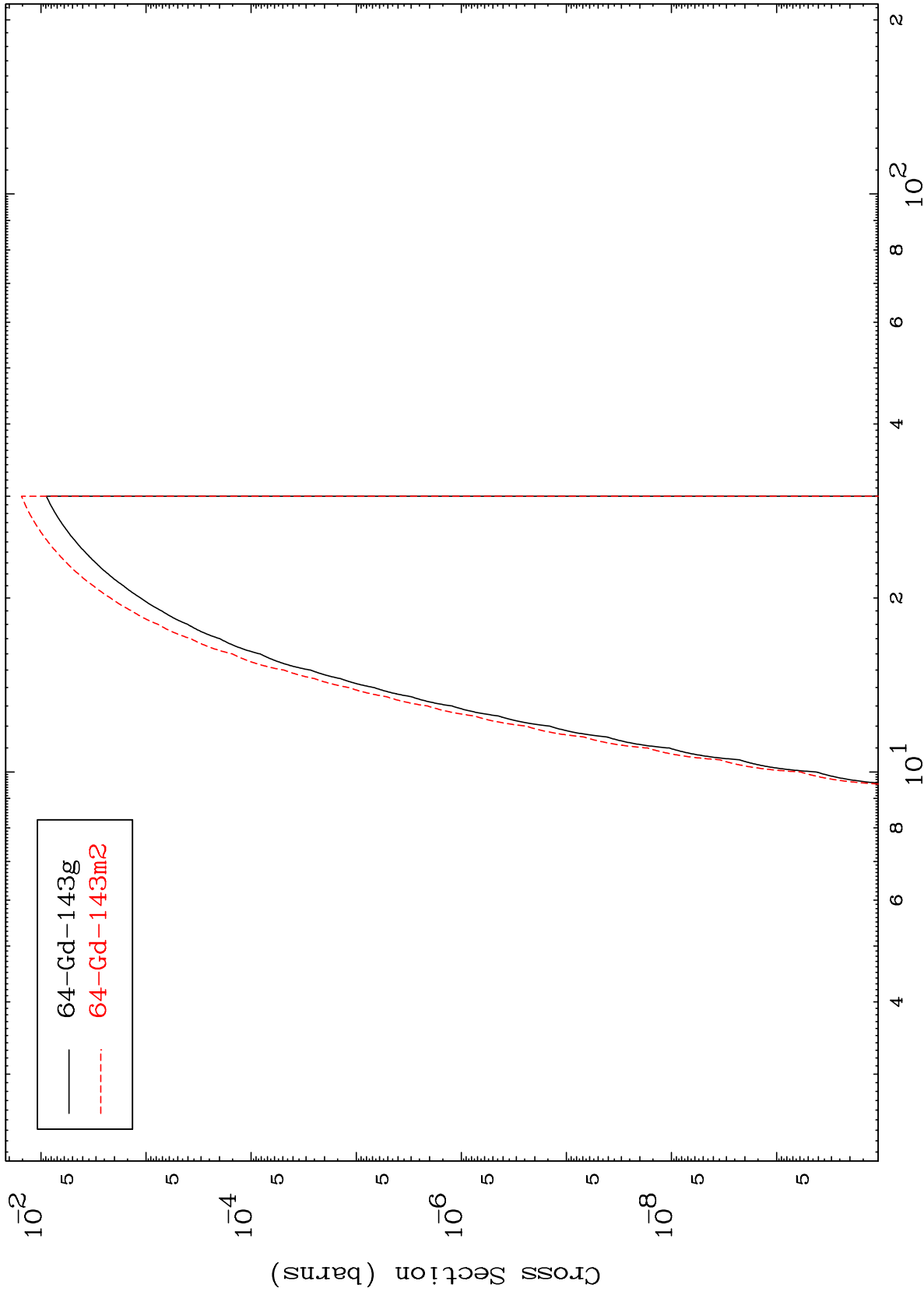
66-Dy-145

MAT 6593

(n,p) d

66-Dy-145

Radionuclide Production Cross Section



64-Gd-143g  
64-Gd-143m2

33

Incident Energy (MeV)

66-Dy-145