

Program EVALPLOT
(Version 2018-1)

by

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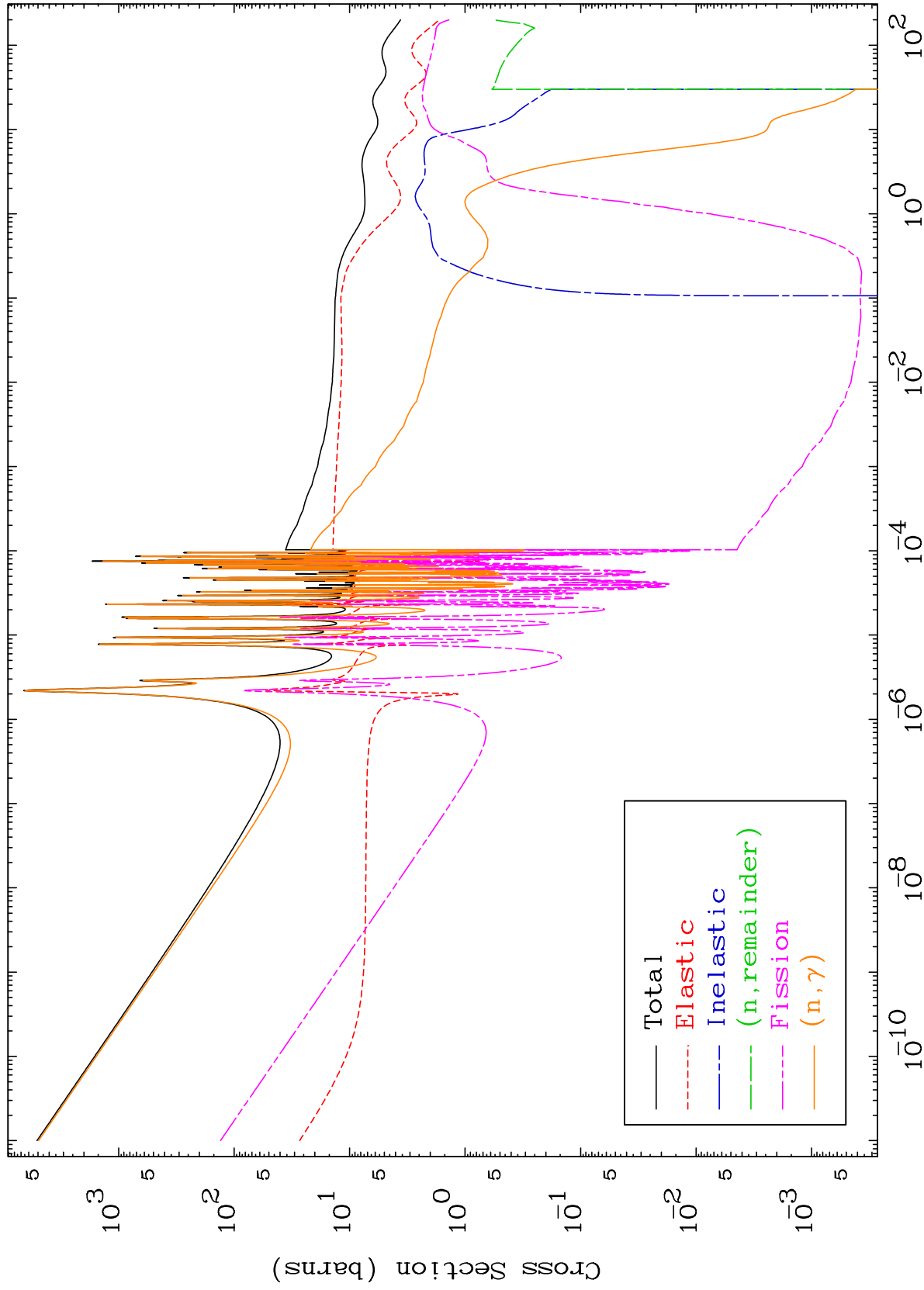
Press Mouse Button to Start

MAT 9927

Major

293 Kelvin Cross Sections

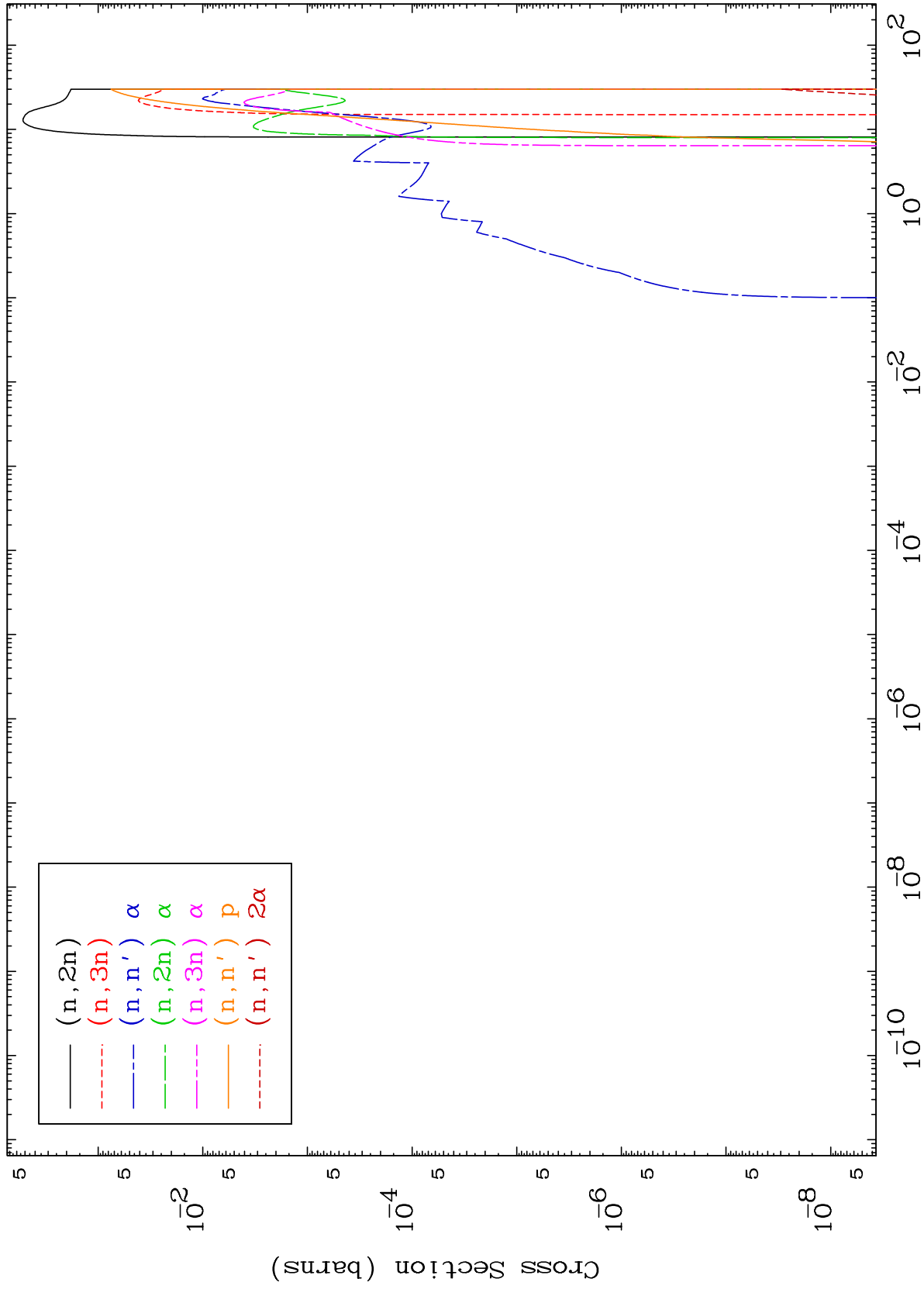
100-Fm-246

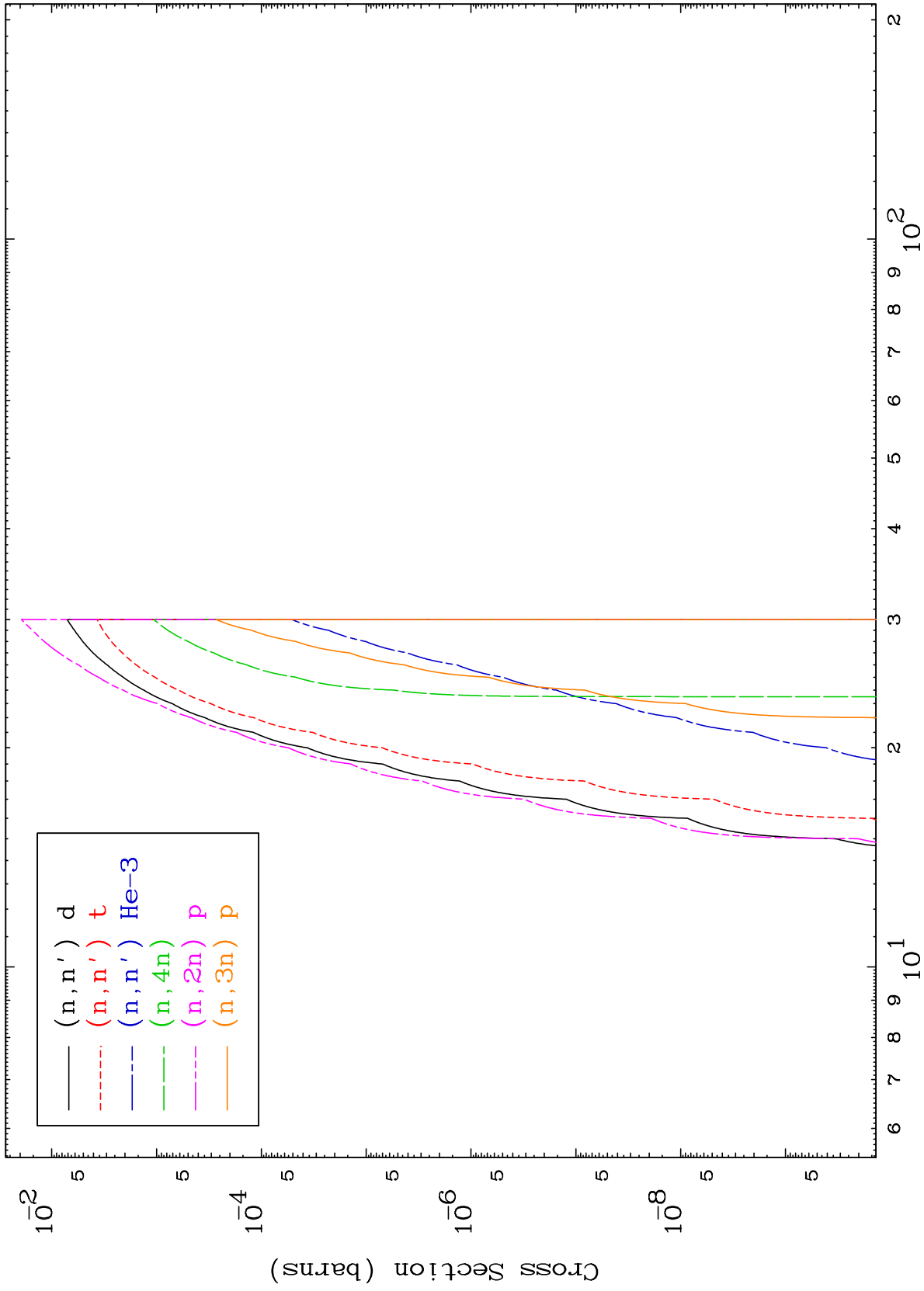


MAT 9927

Neutron Production
293 Kelvin Cross Sections

100-Fm-246

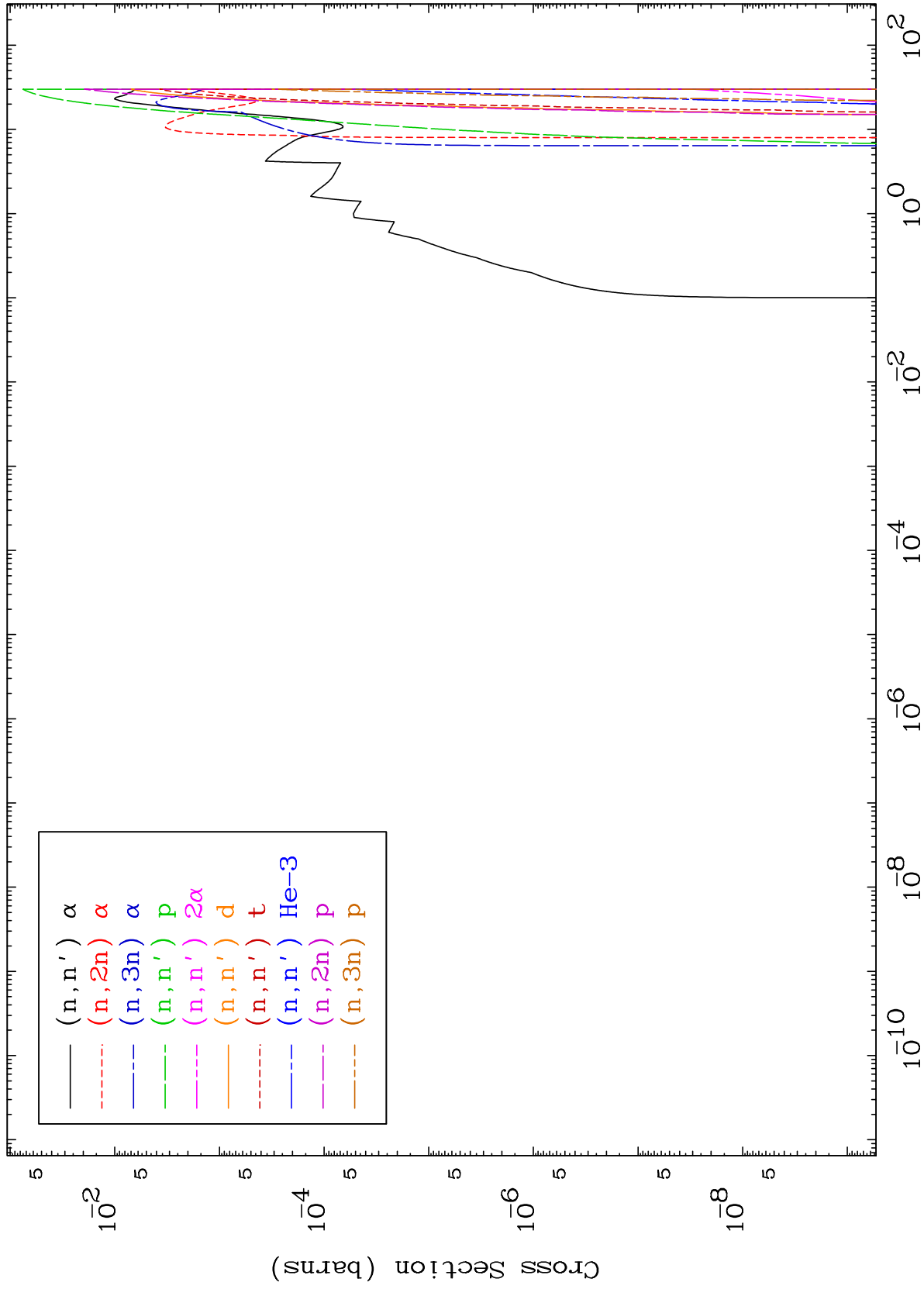




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Charged Particle
293 Kelvin Cross Sections

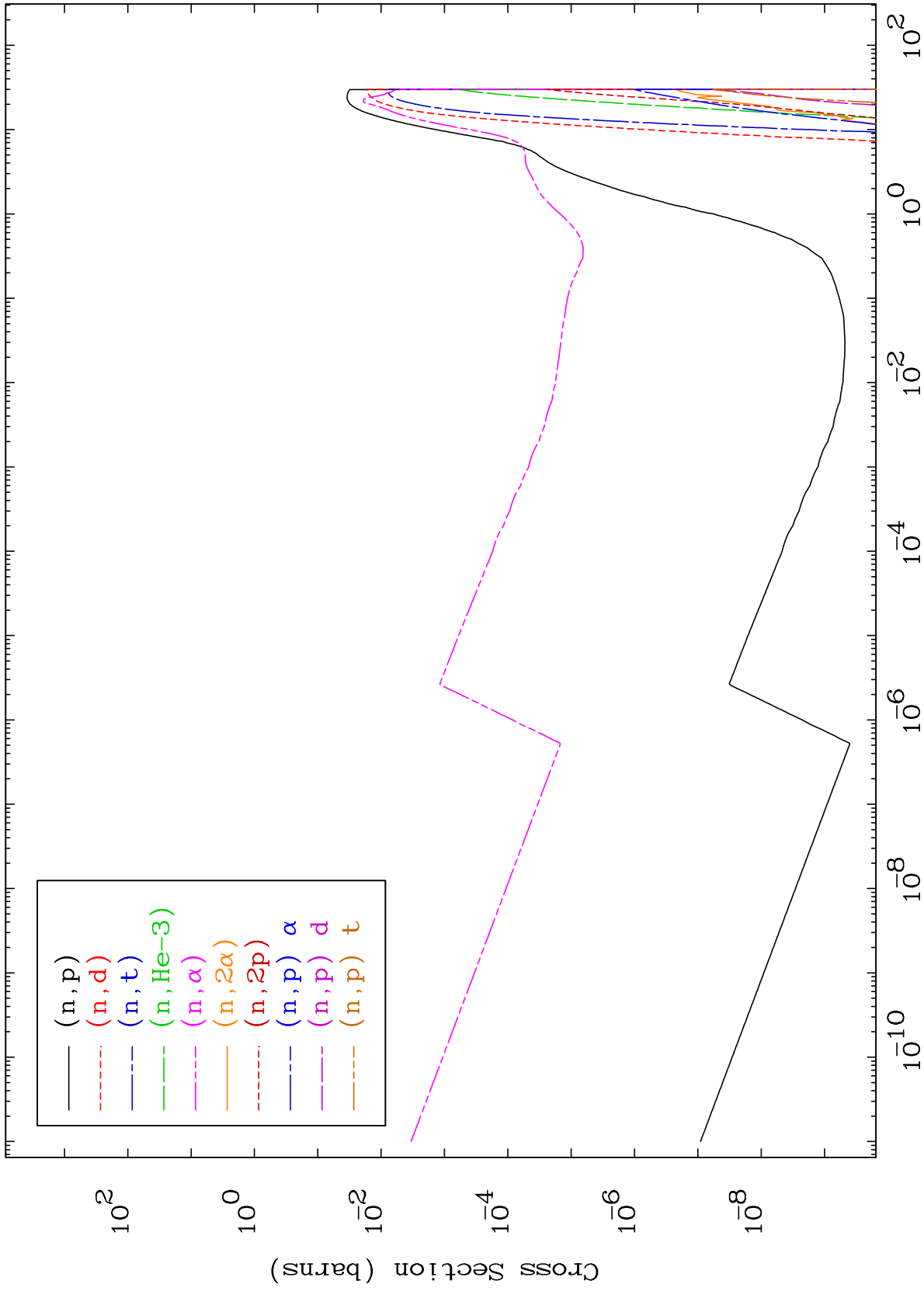
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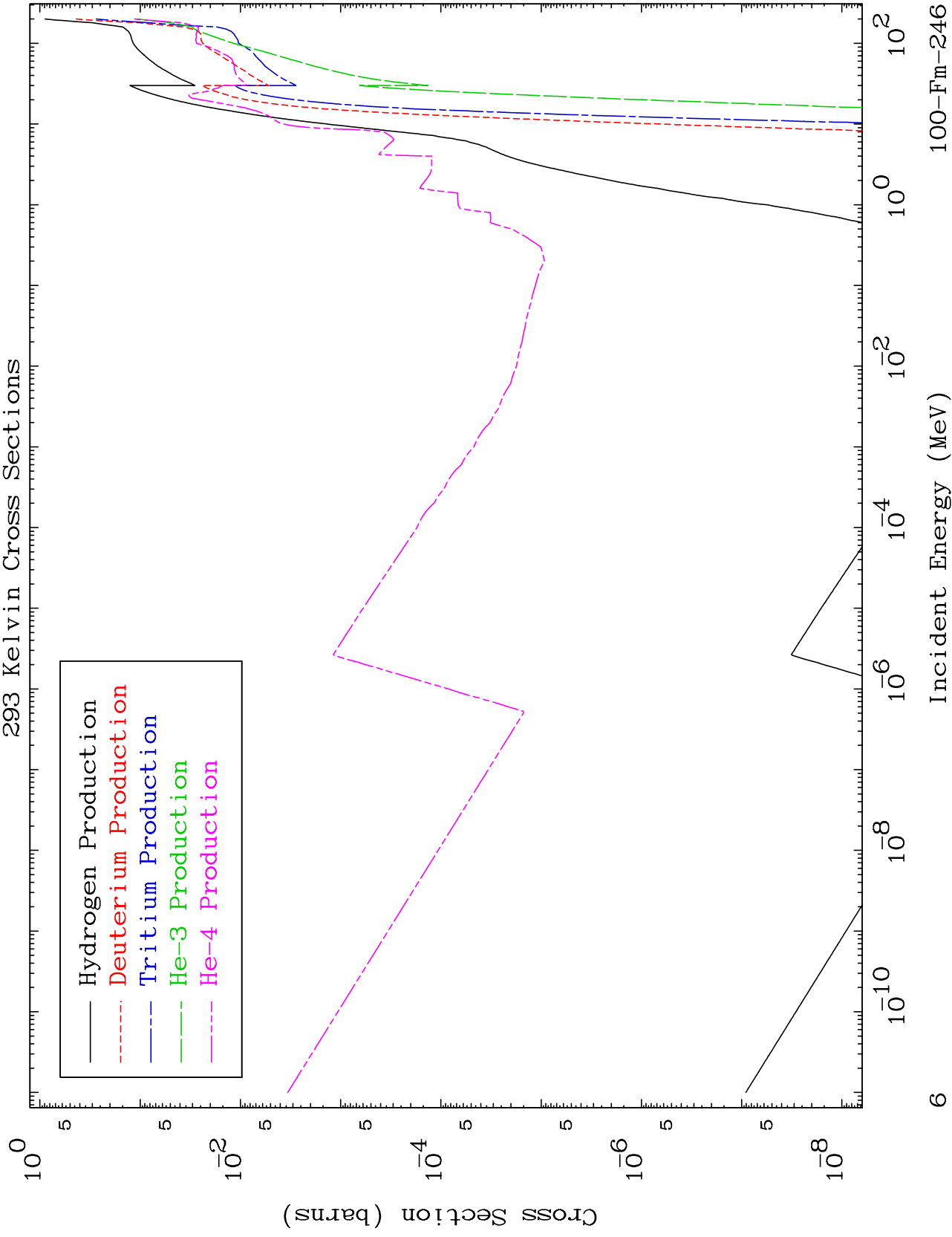


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Charged Particle
293 Kelvin Cross Sections

100-Fm-246

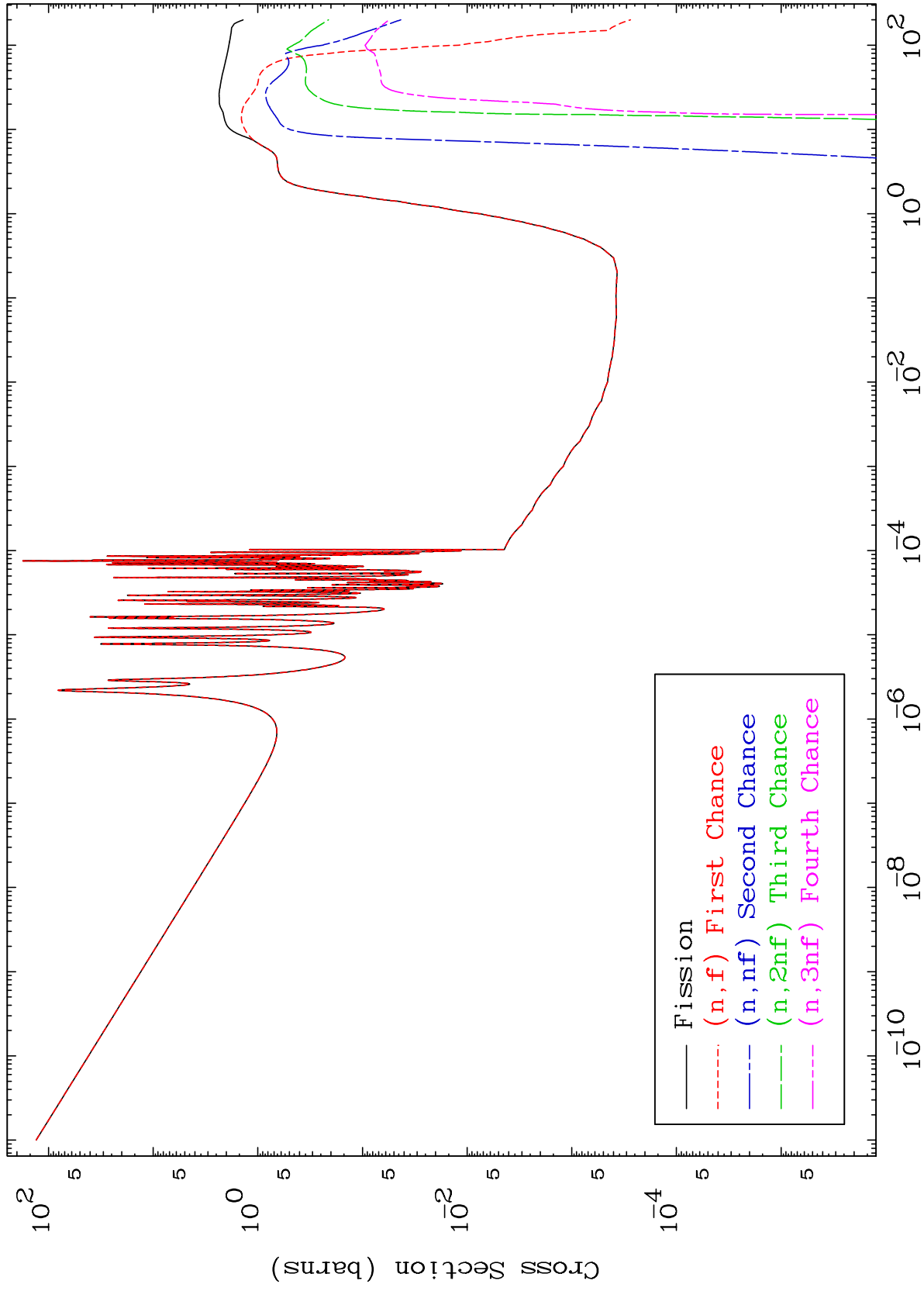




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Fission
293 Kelvin Cross Sections

100-Fm-246

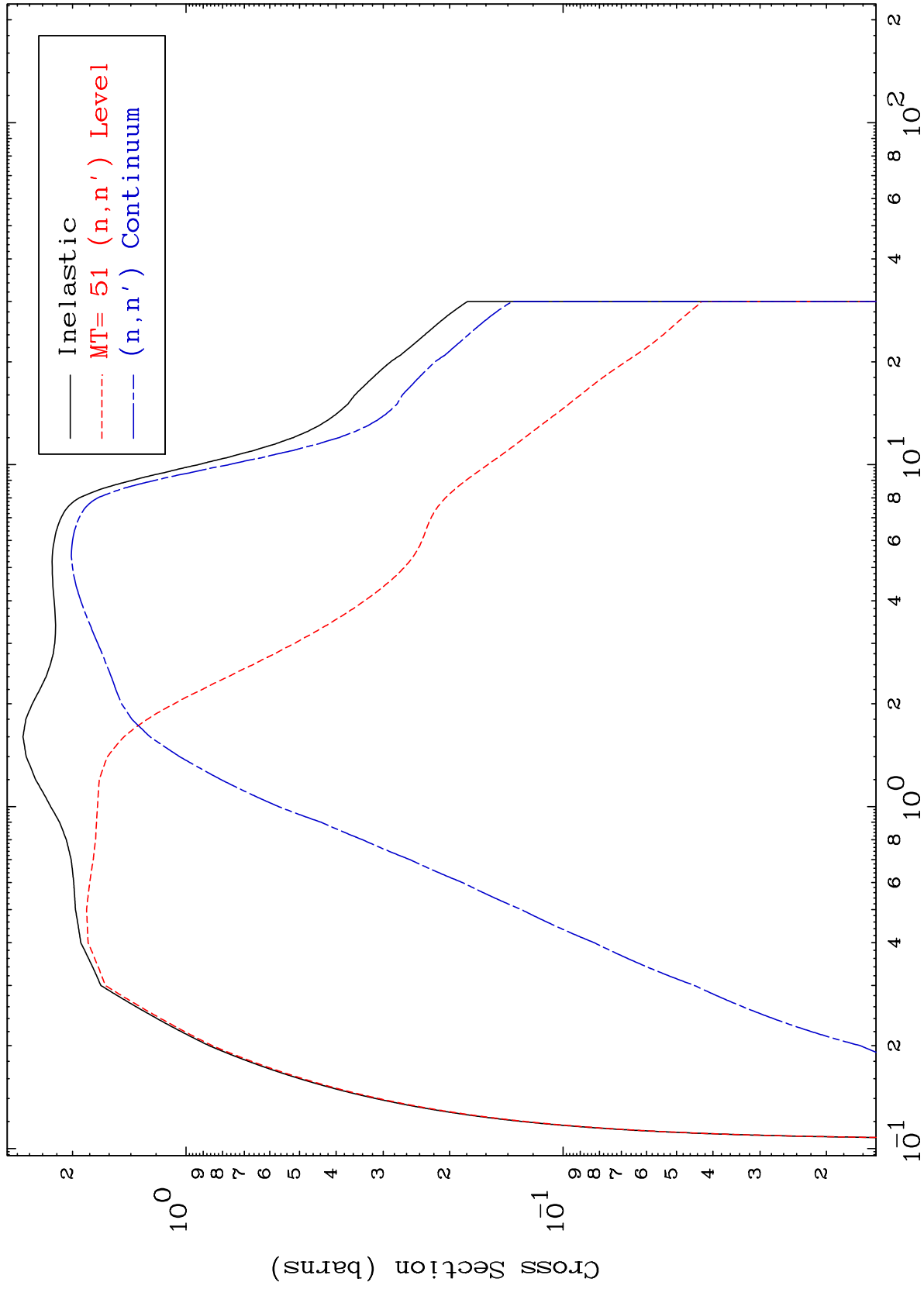


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(n,n') Level

100-Fm-246

293 Kelvin Cross Sections



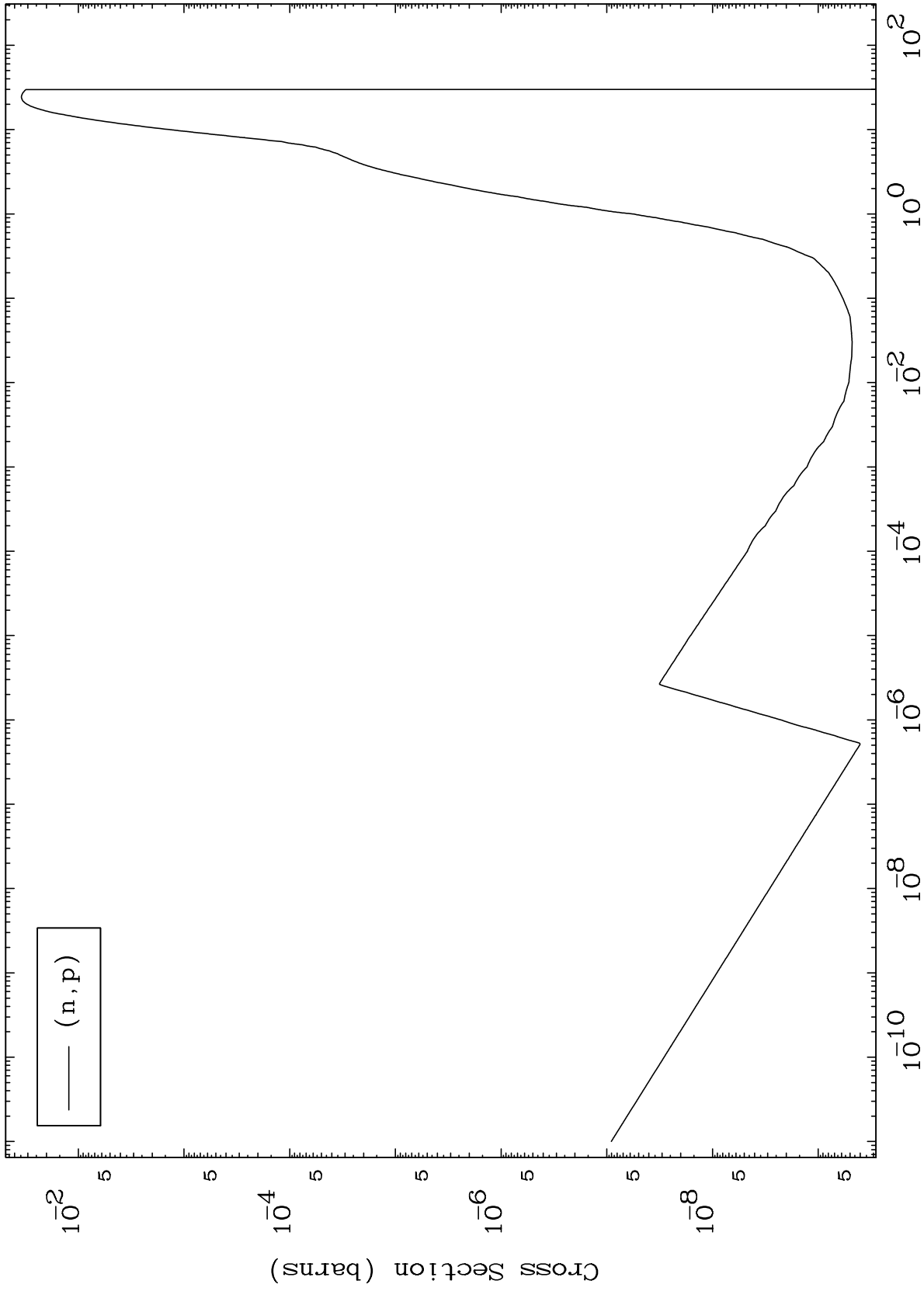
Incident Energy (MeV)

100-Fm-246

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(n,p) Levels
293 Kelvin Cross Sections

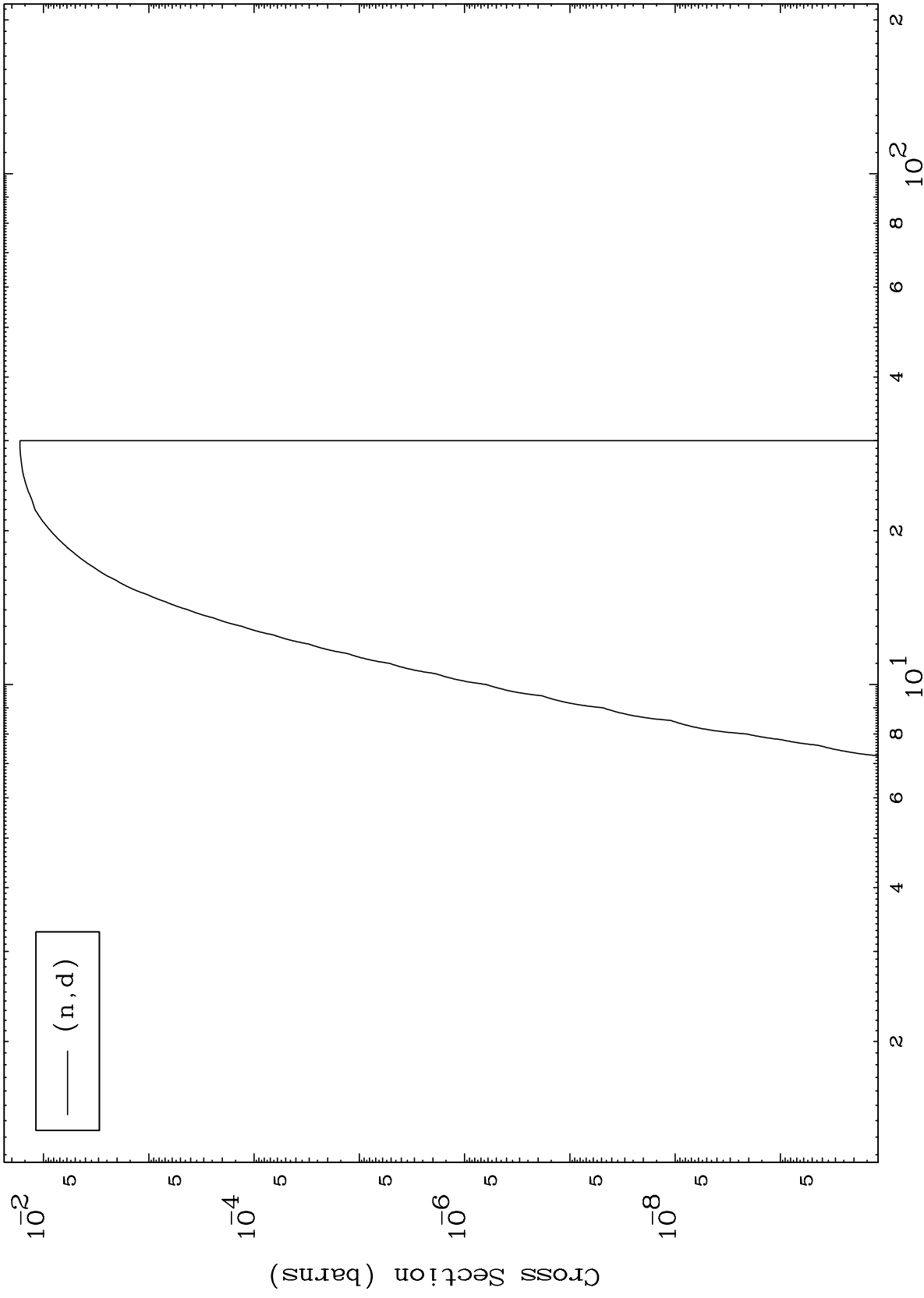
100-Fm-246



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(n,d) Levels
293 Kelvin Cross Sections

100-Fm-246



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Incident Energy (MeV)

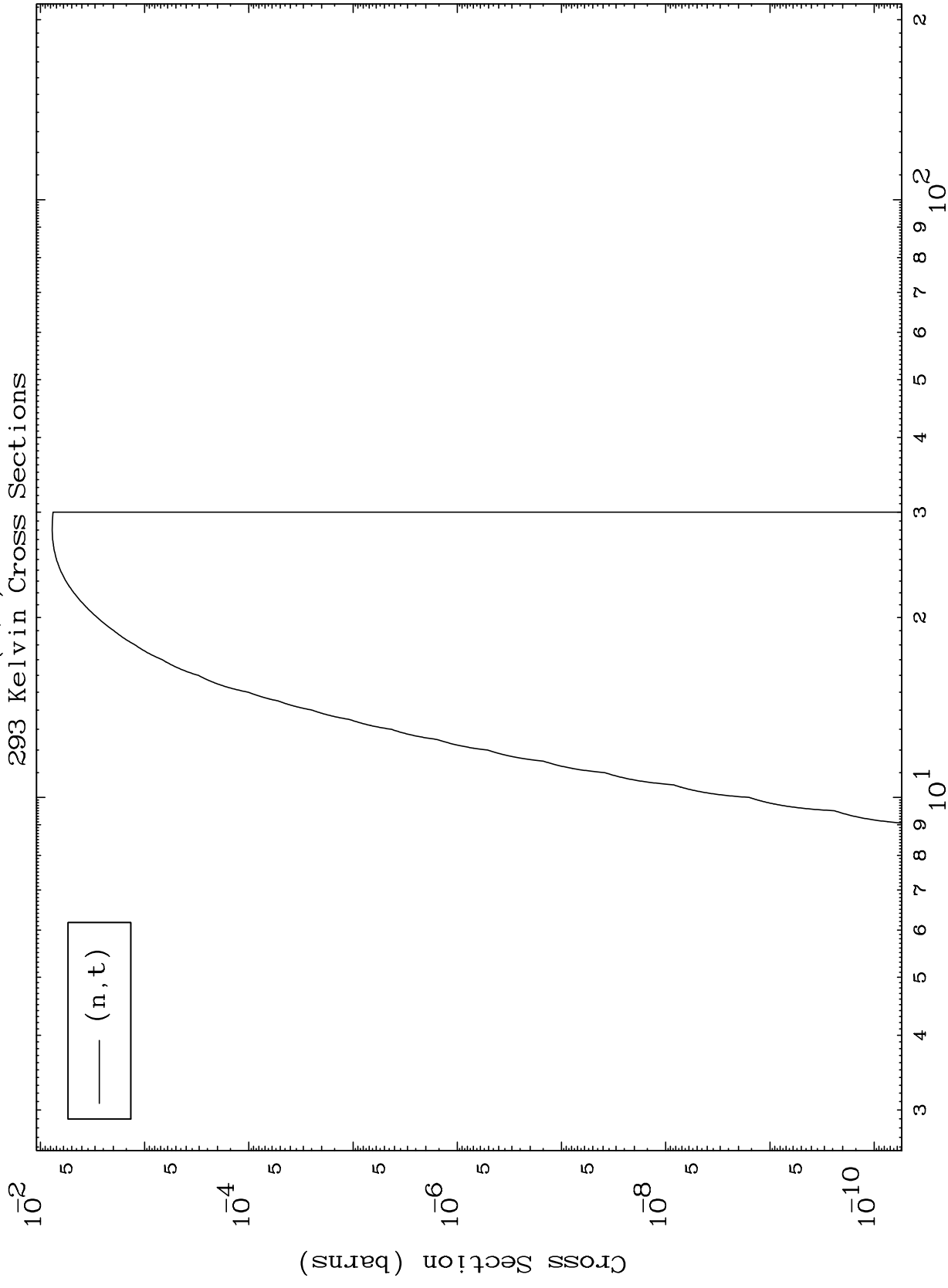
100-Fm-246

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(n, t) Levels

100-Fm-246

293 Kelvin Cross Sections



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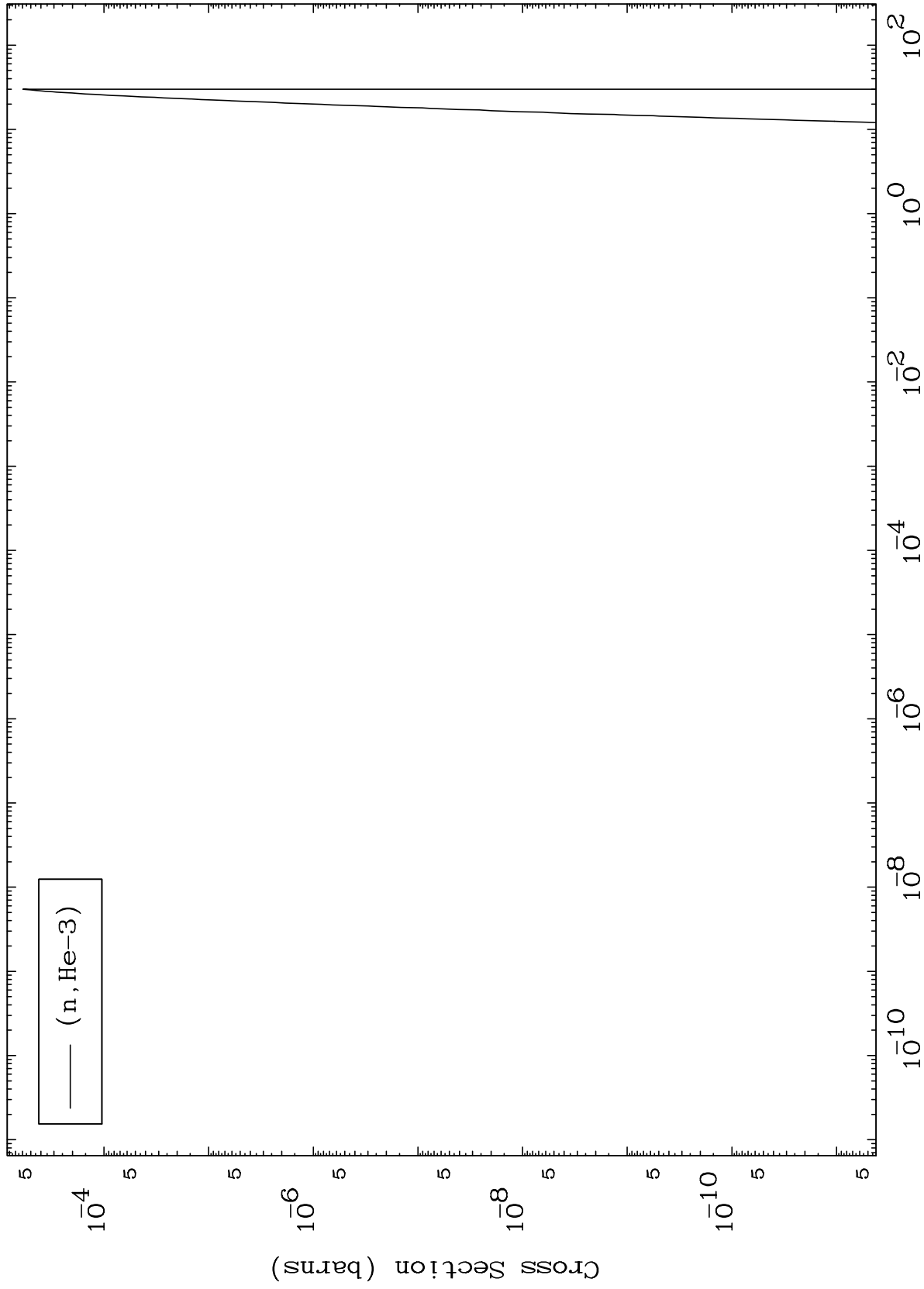
Incident Energy (MeV)

100-Fm-246

MAT 9927

(n,He3) Levels
293 Kelvin Cross Sections

100-Fm-246



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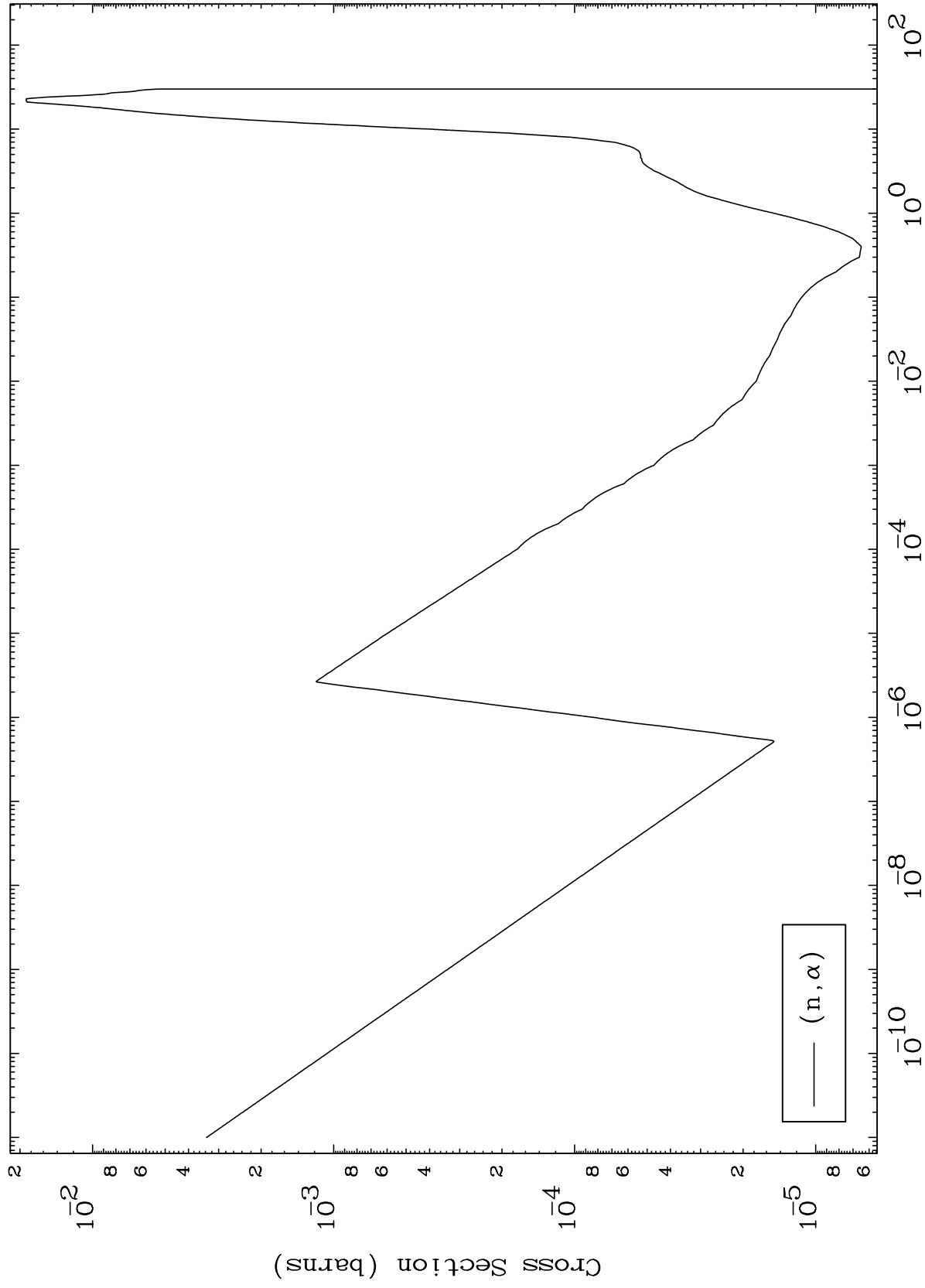
Incident Energy (MeV)

100-Fm-246

MAT 9927

(n, α) Levels
293 Kelvin Cross Sections

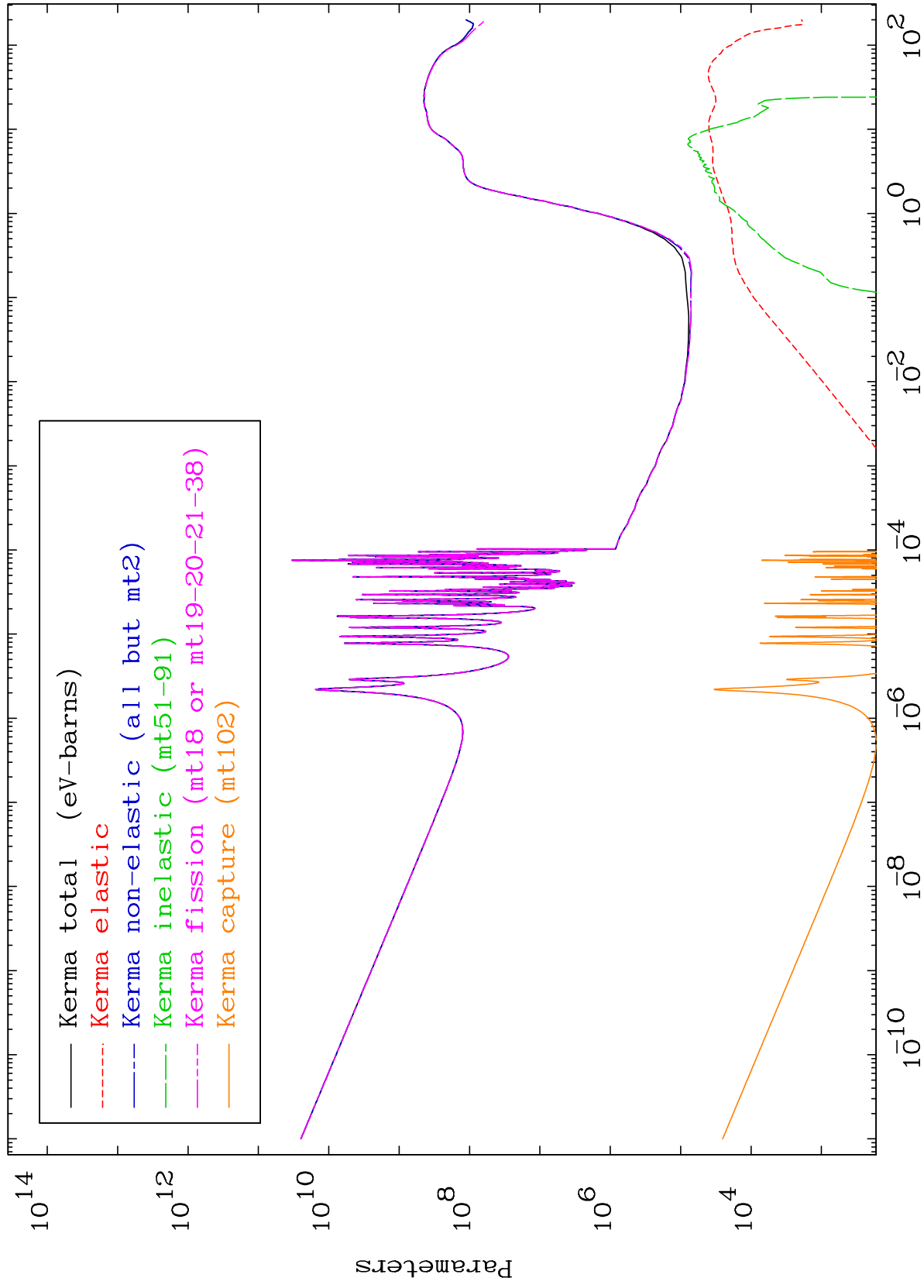
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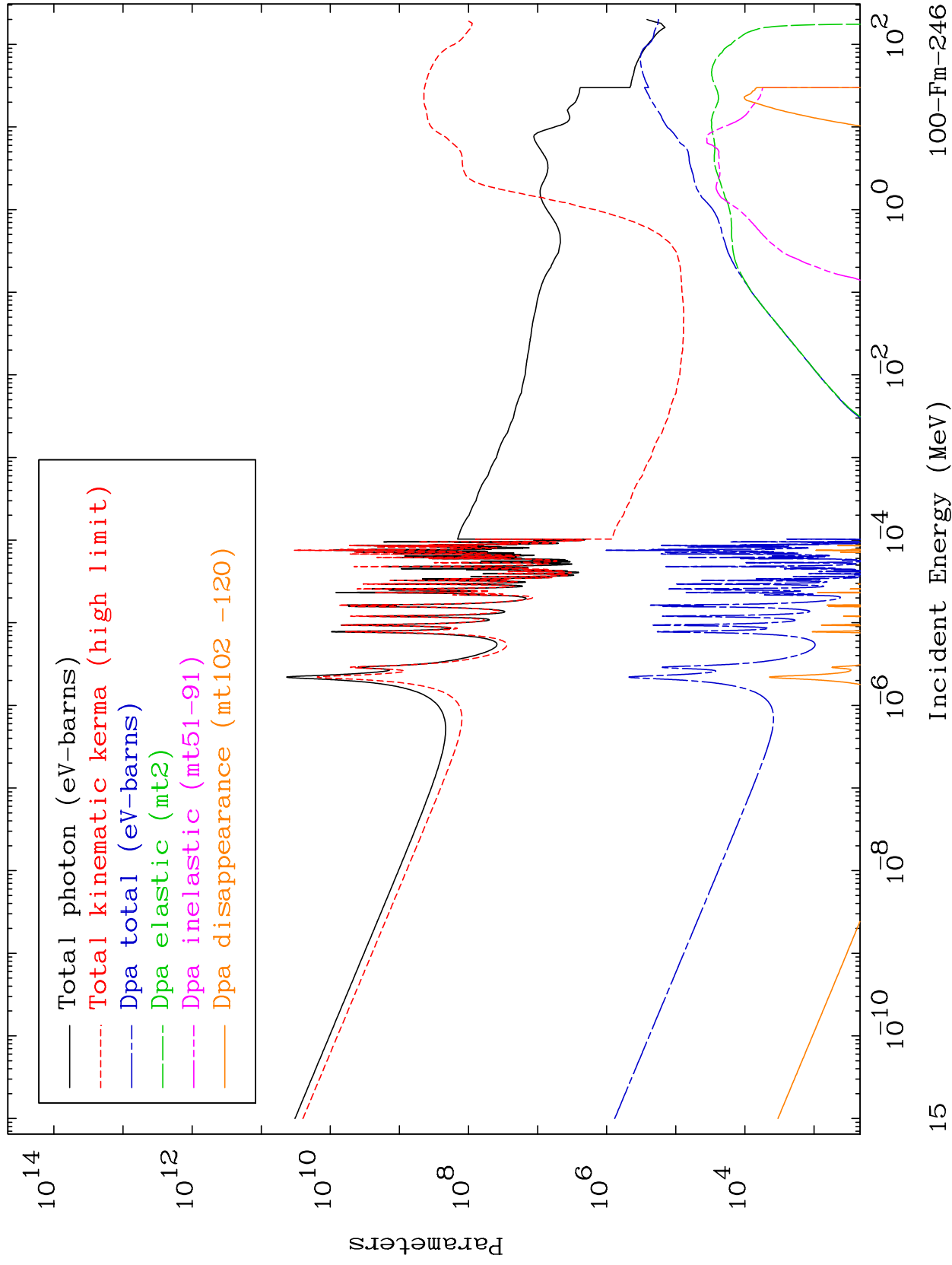


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Incident Energy (MeV)

100-Fm-246

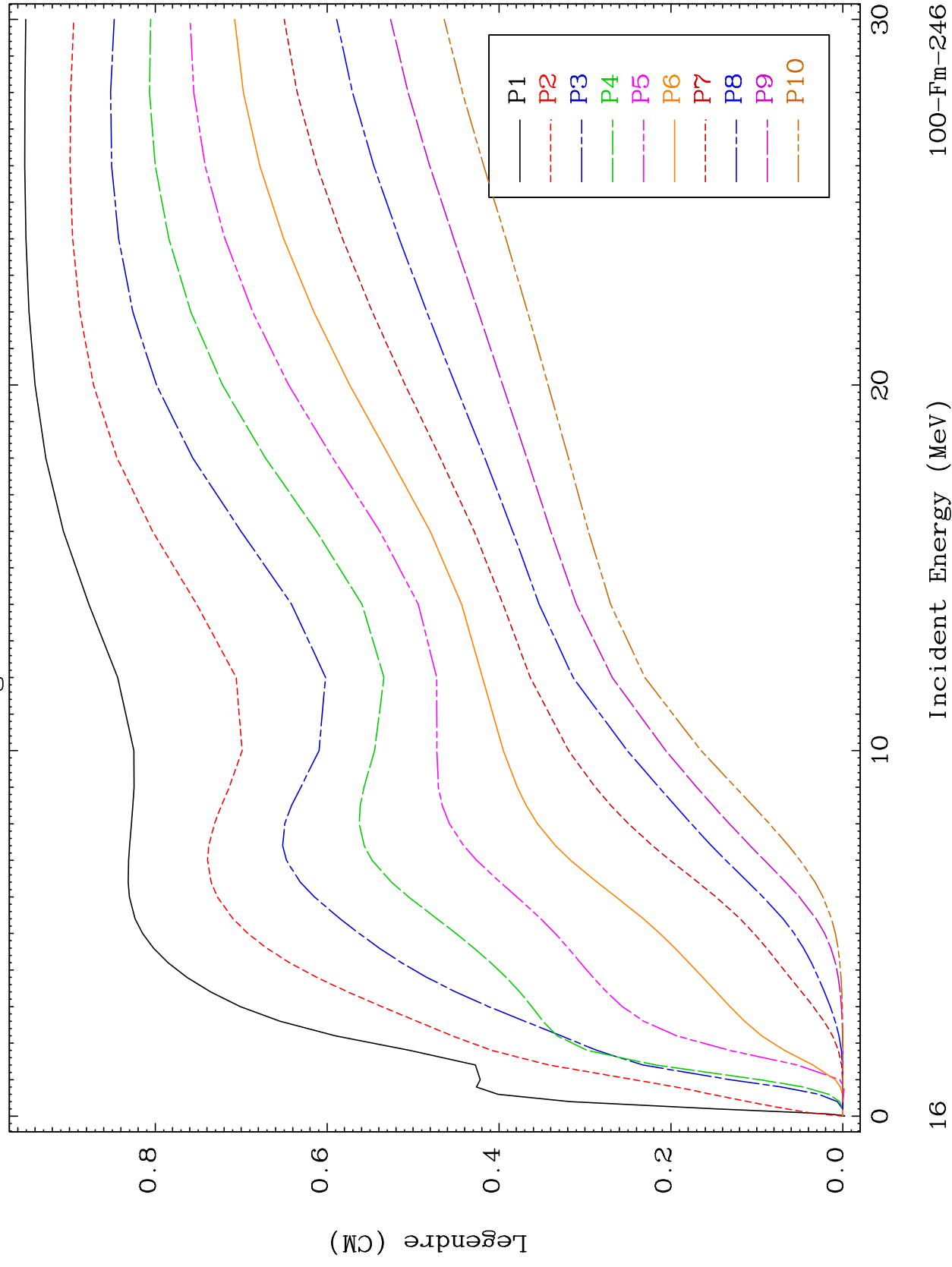


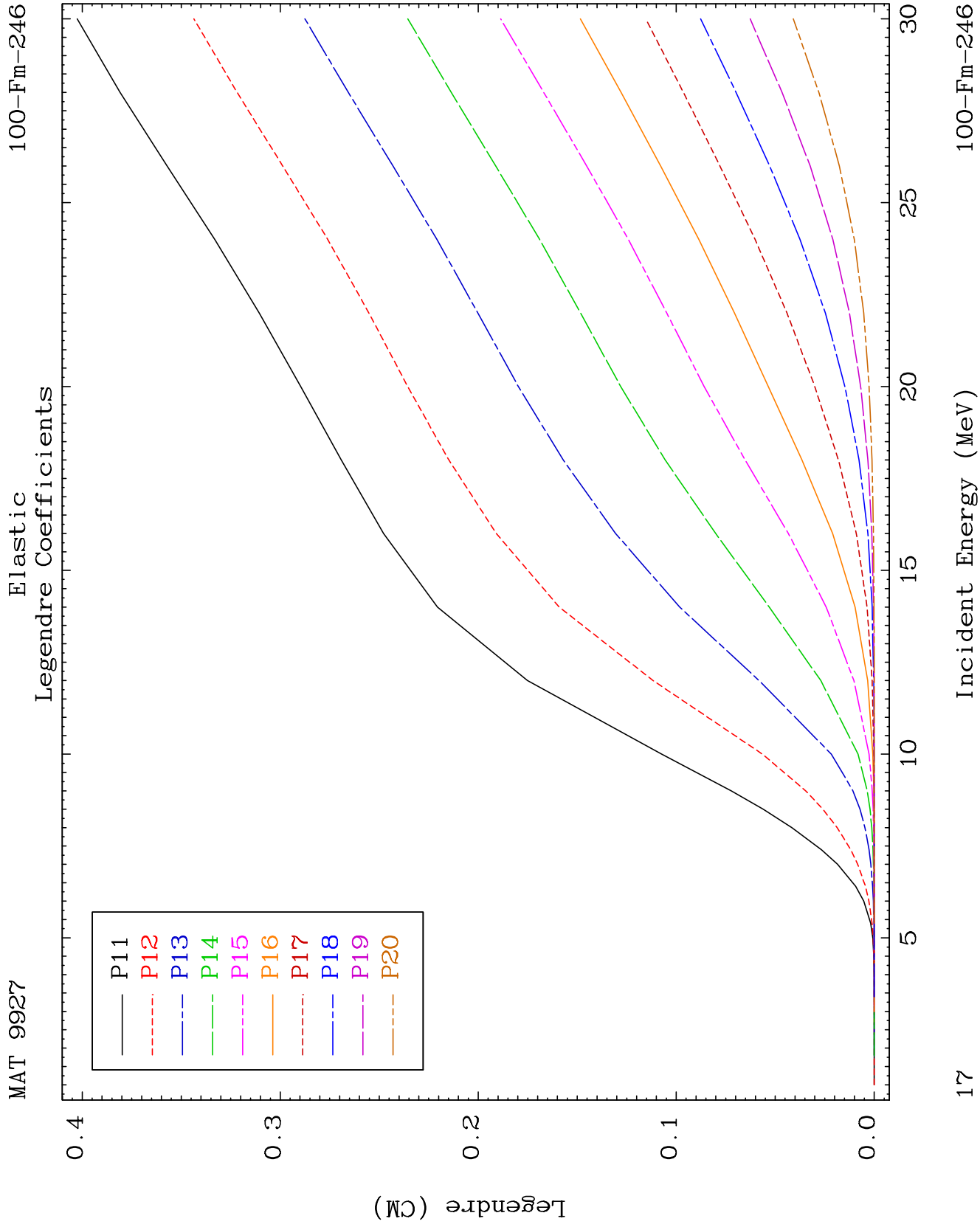


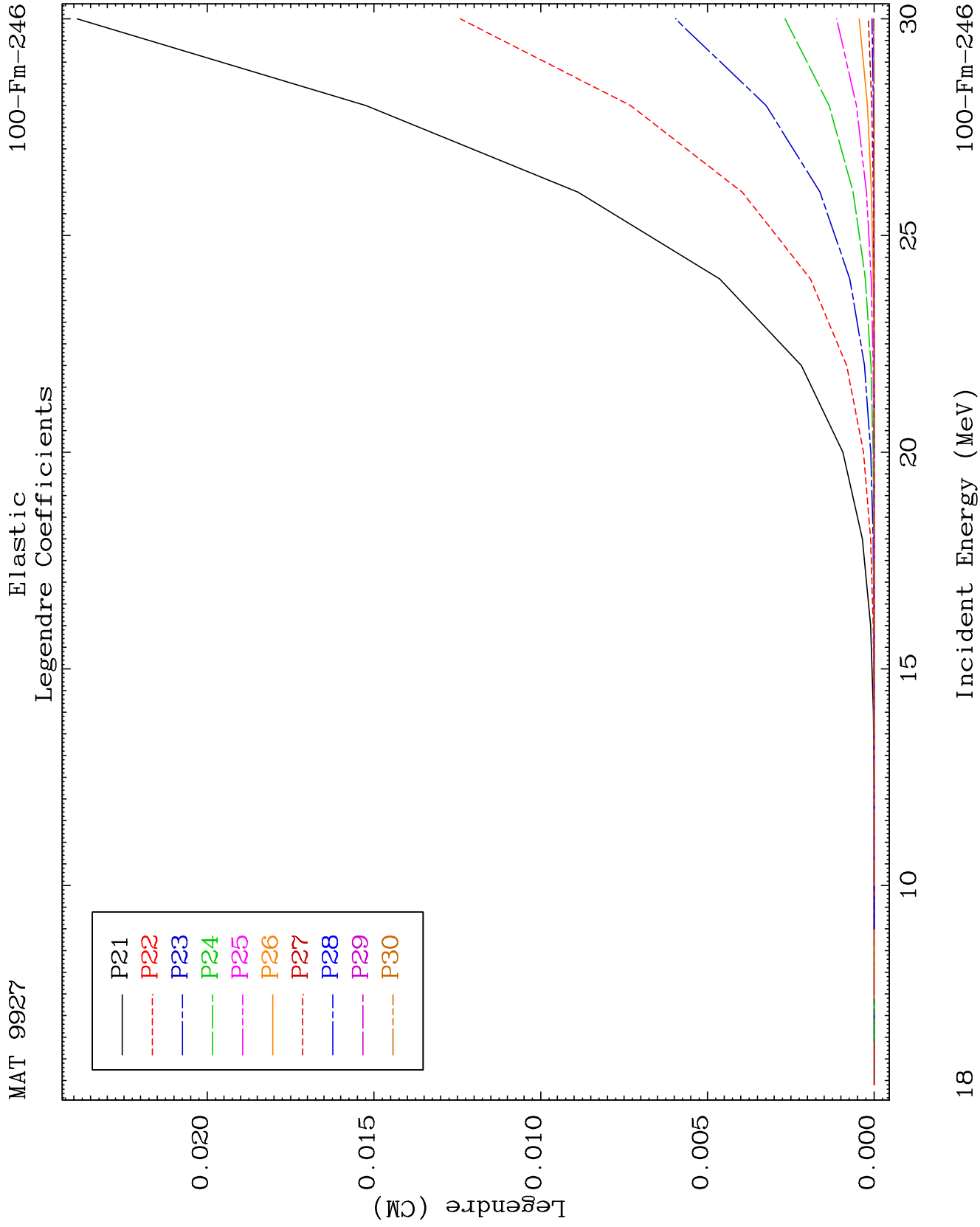
MAT 9927

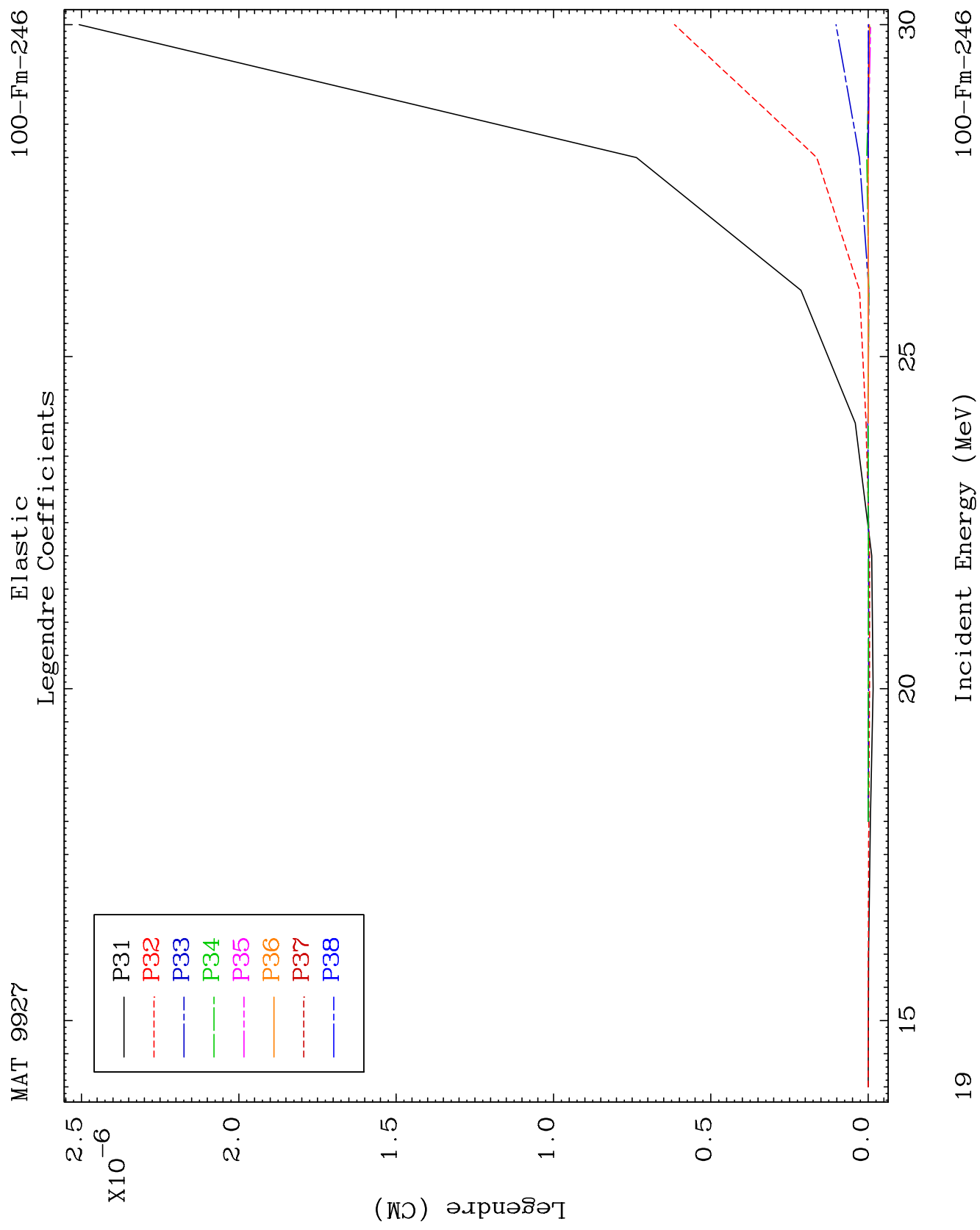
Elastic Legendre Coefficients

100-Fm-246





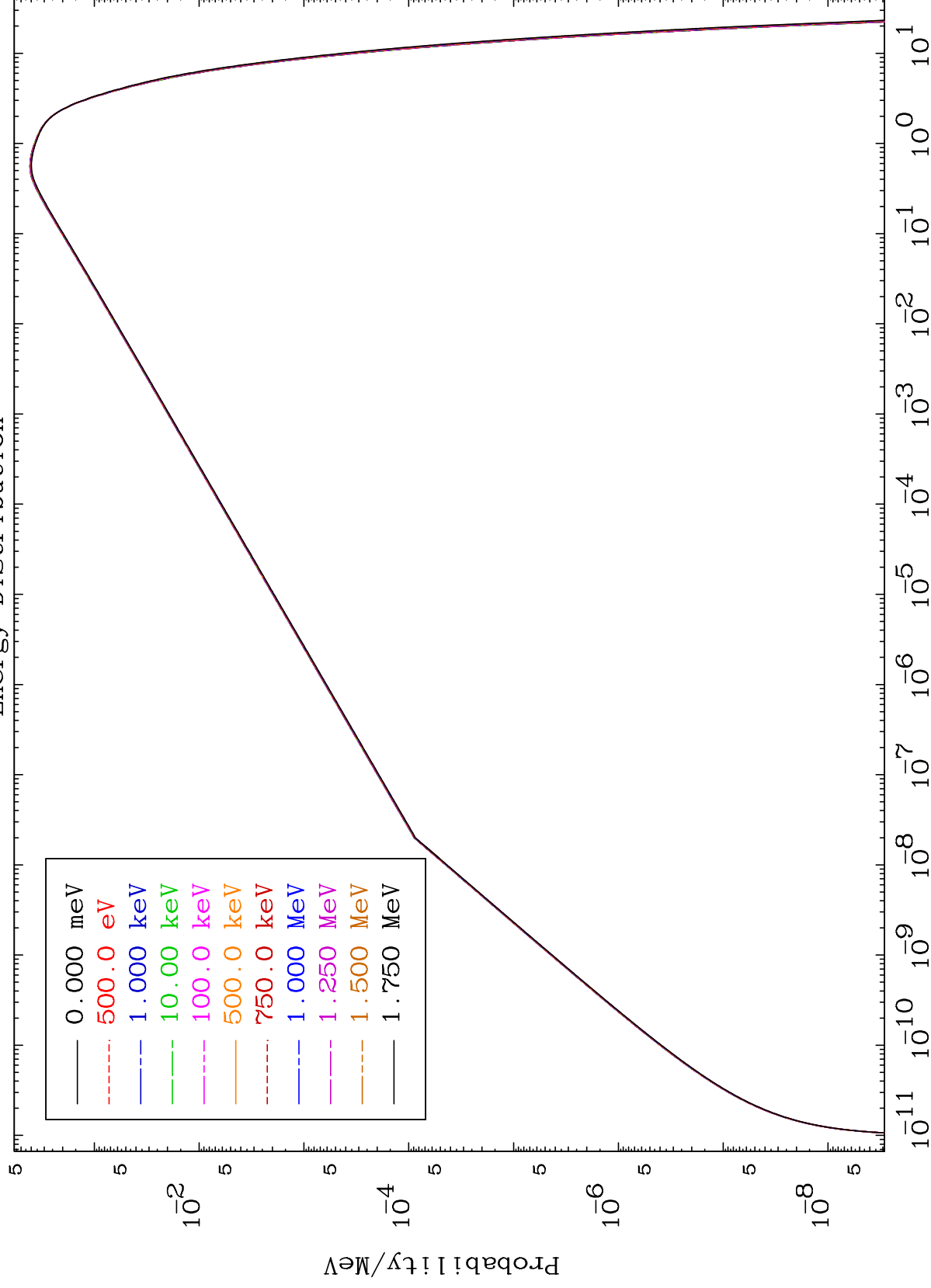




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Fission
Energy Distribution

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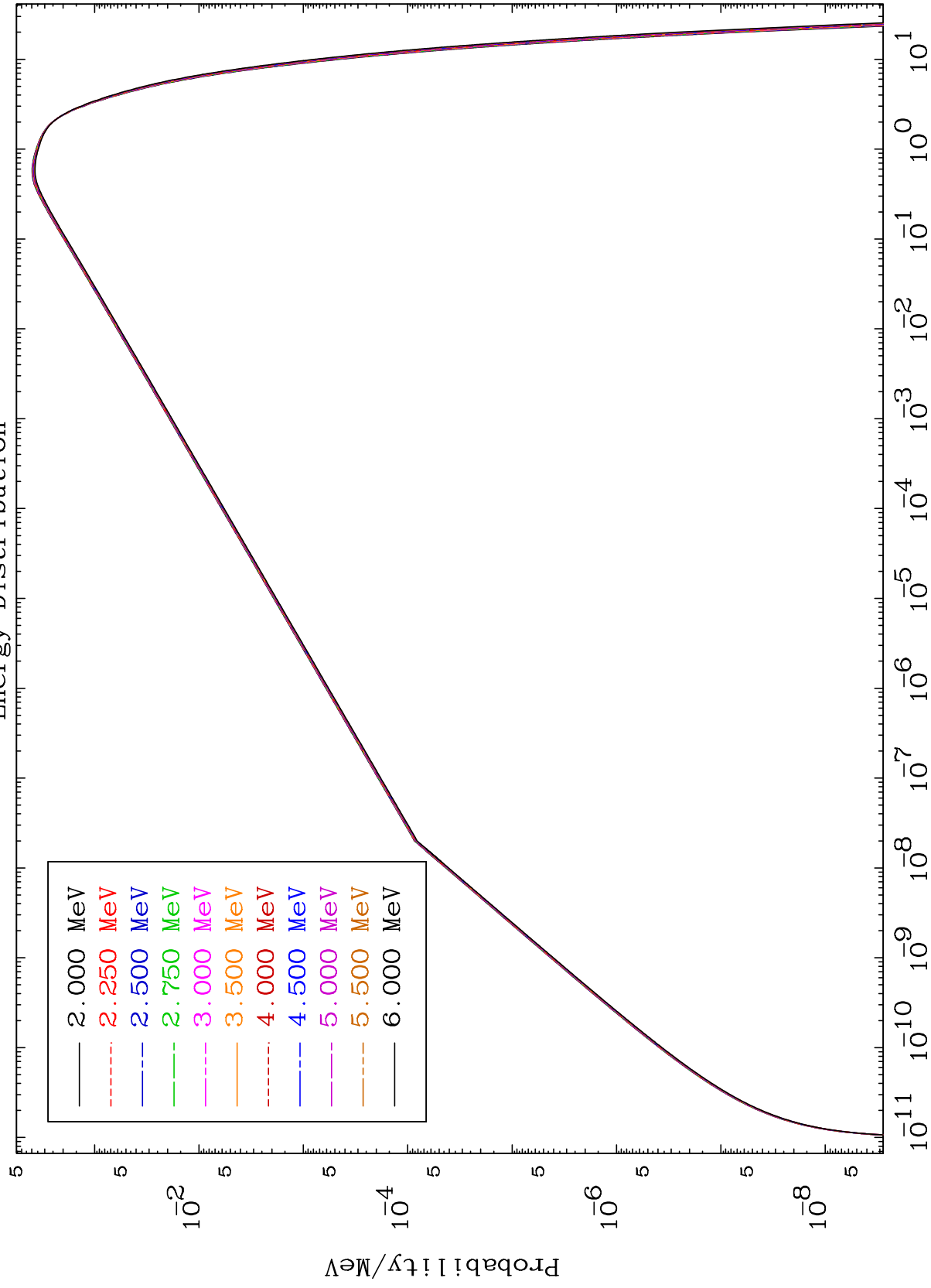
Secondary Energy (MeV)

100-Fm-246

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Fission
Energy Distribution

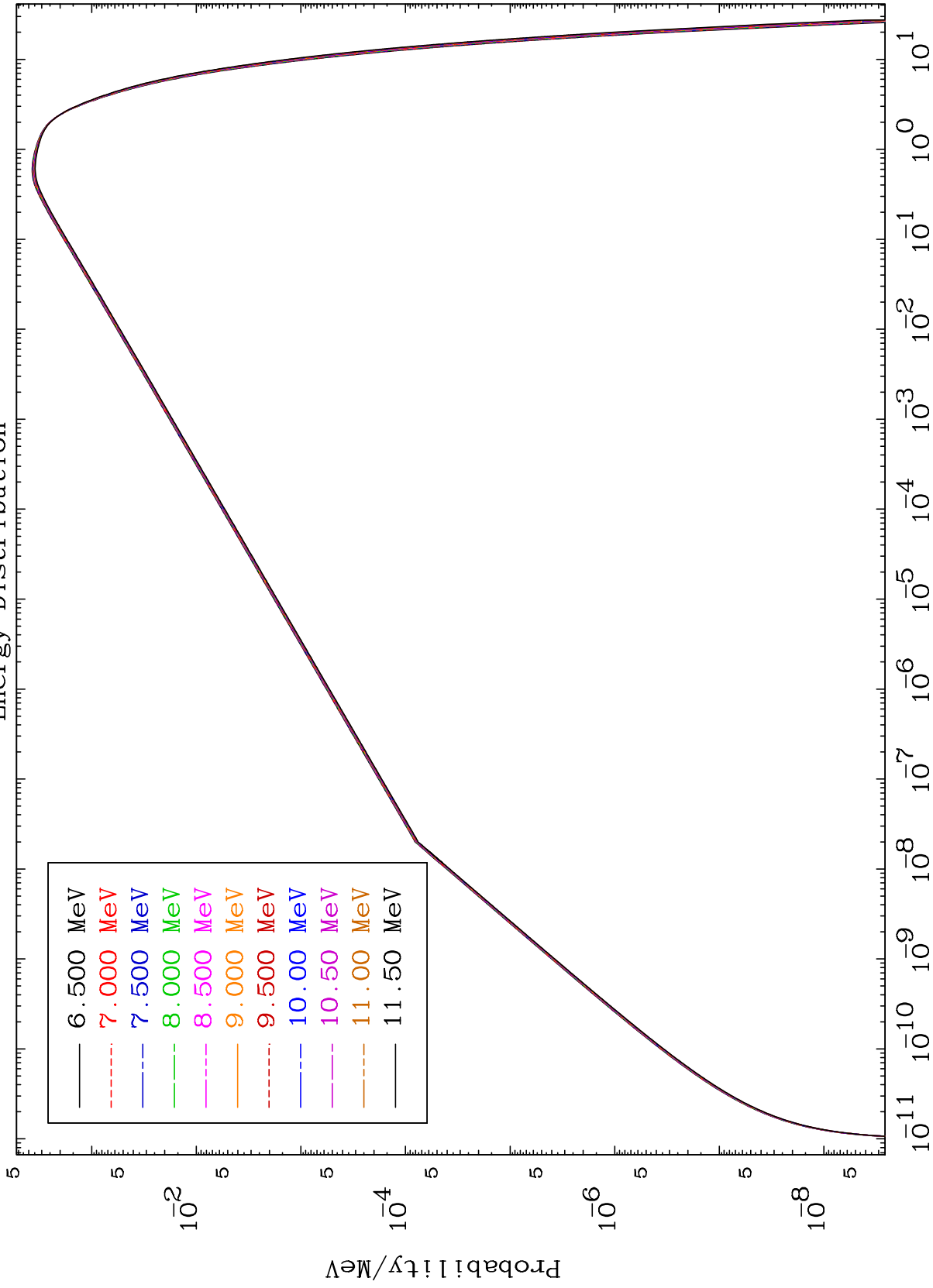
100-Fm-246



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Fission
Energy Distribution

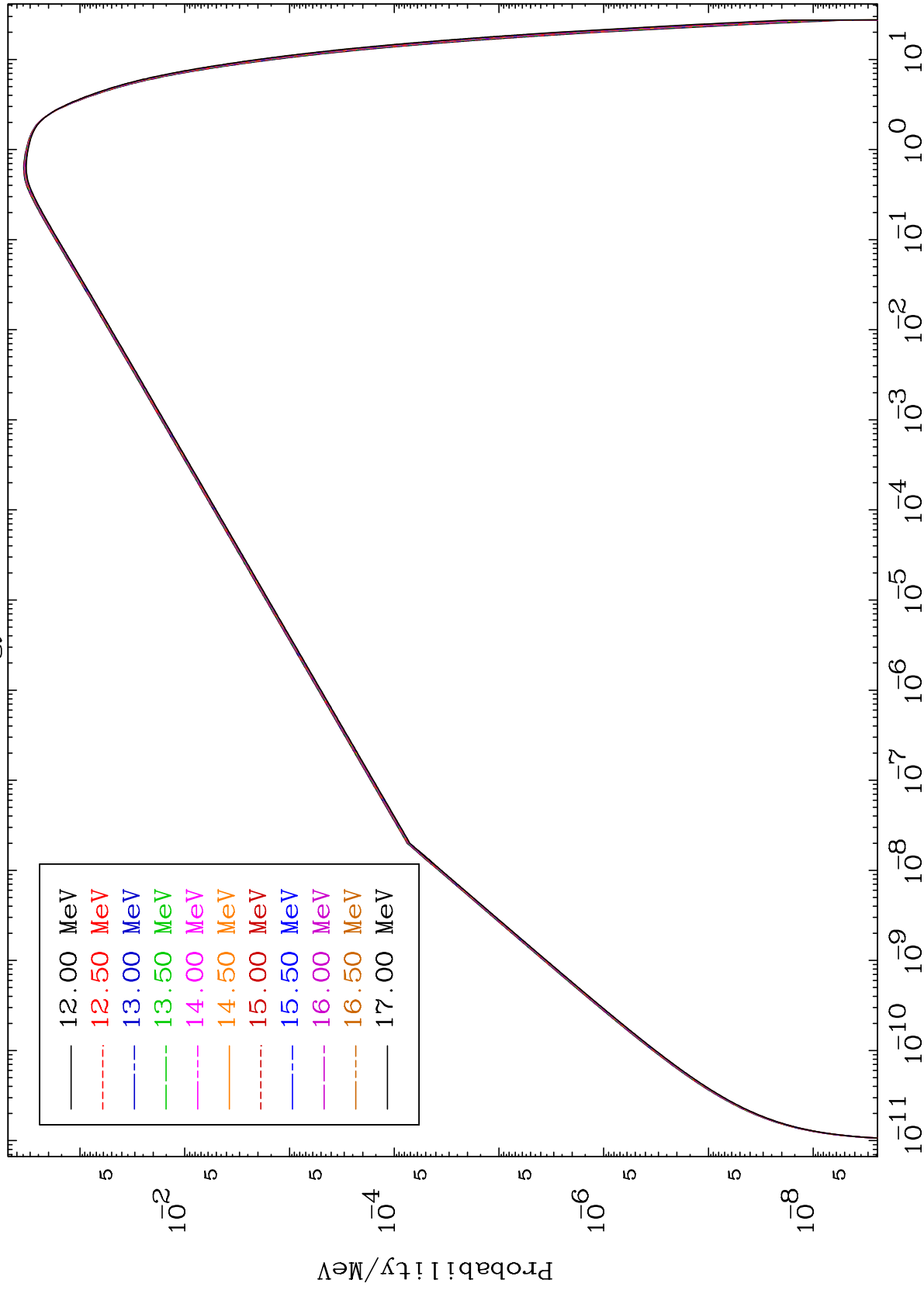
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Fission
Energy Distribution

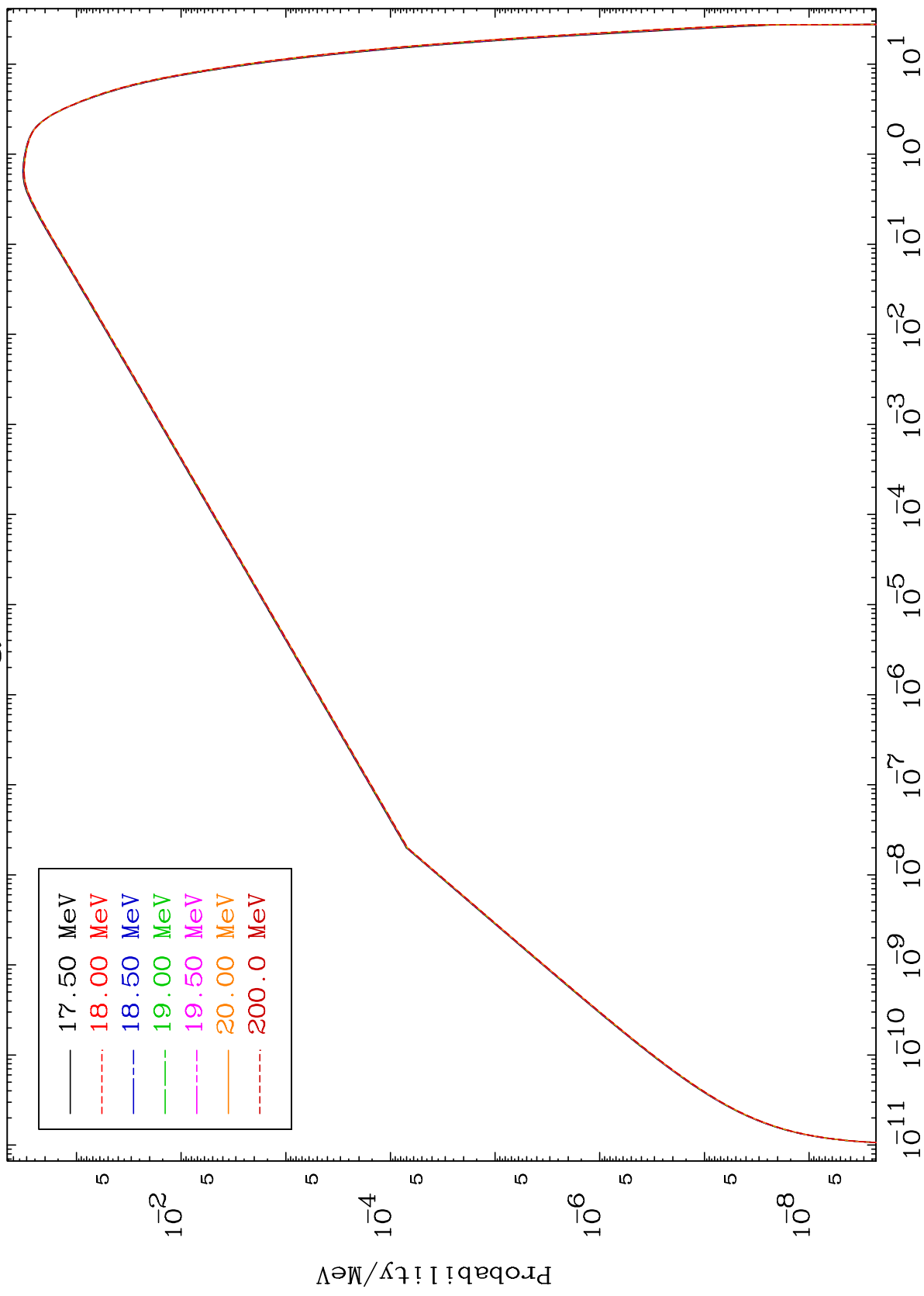
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Fission
Energy Distribution

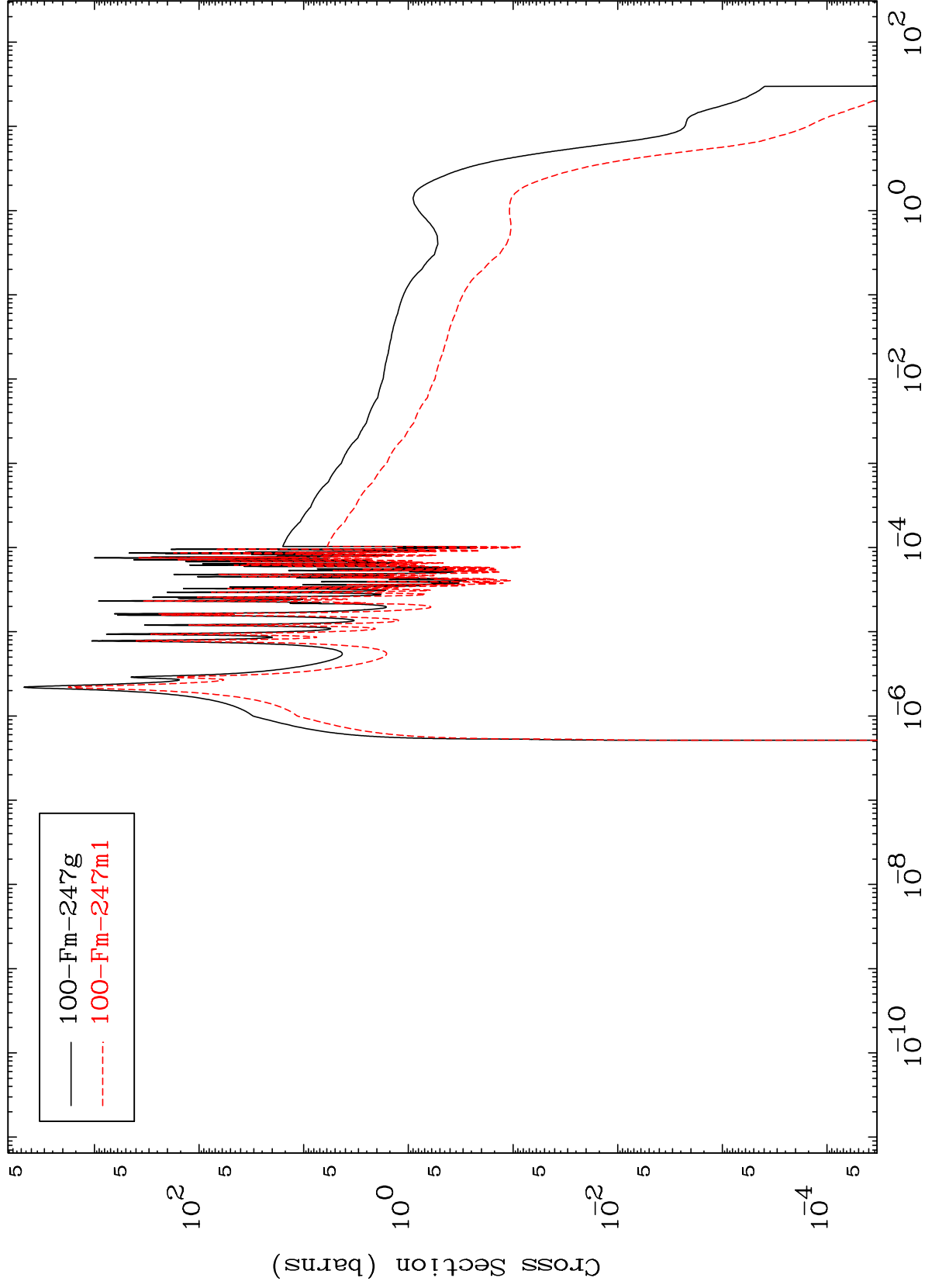
100-Fm-246



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Radionuclide Production Cross Section
(n, γ)

100-Fm-246



25

Incident Energy (MeV)

100-Fm-246