

Program EVALPLOT
(Version 2018-1)

by

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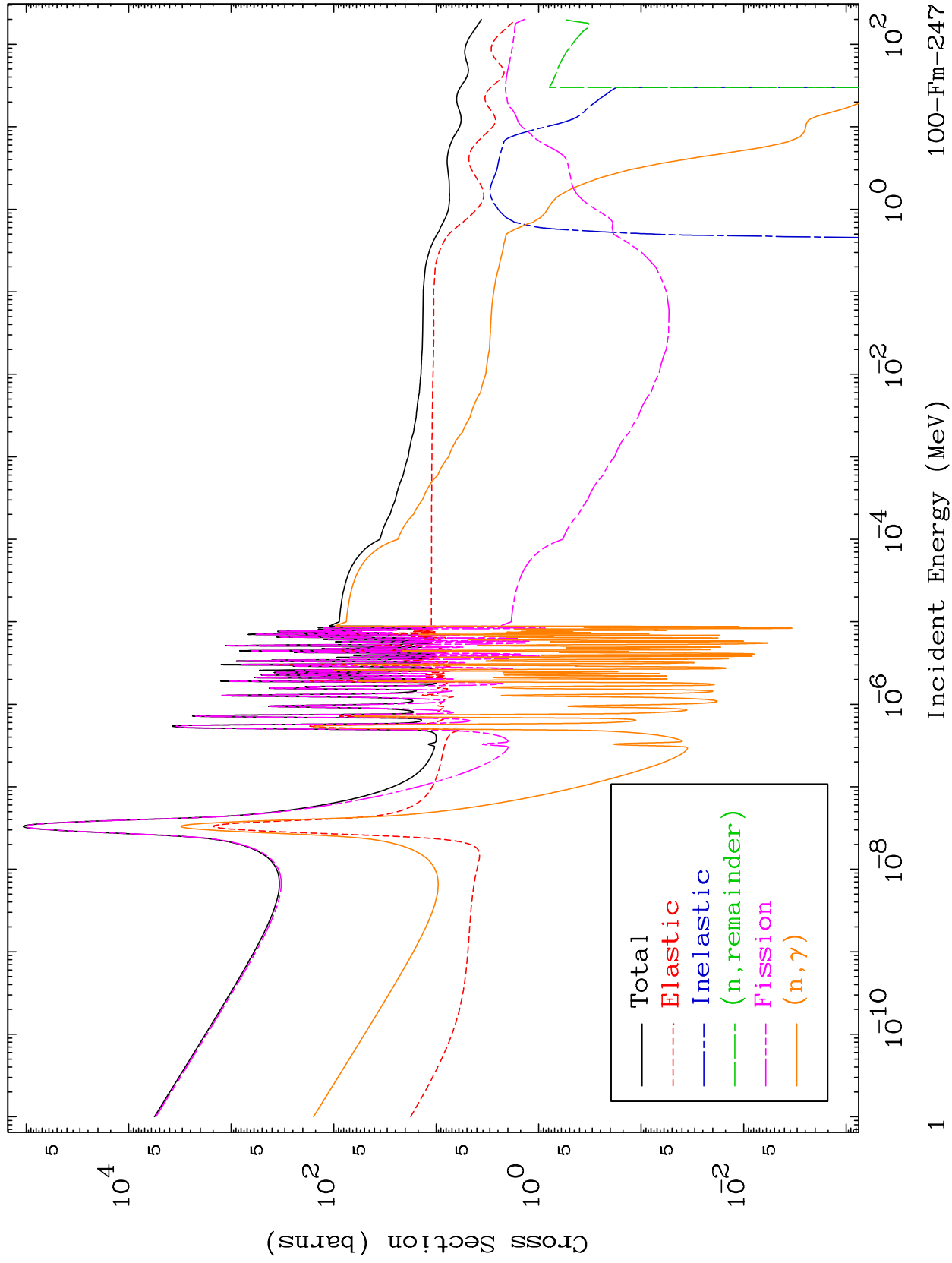
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9928

Major
293 Kelvin Cross Sections

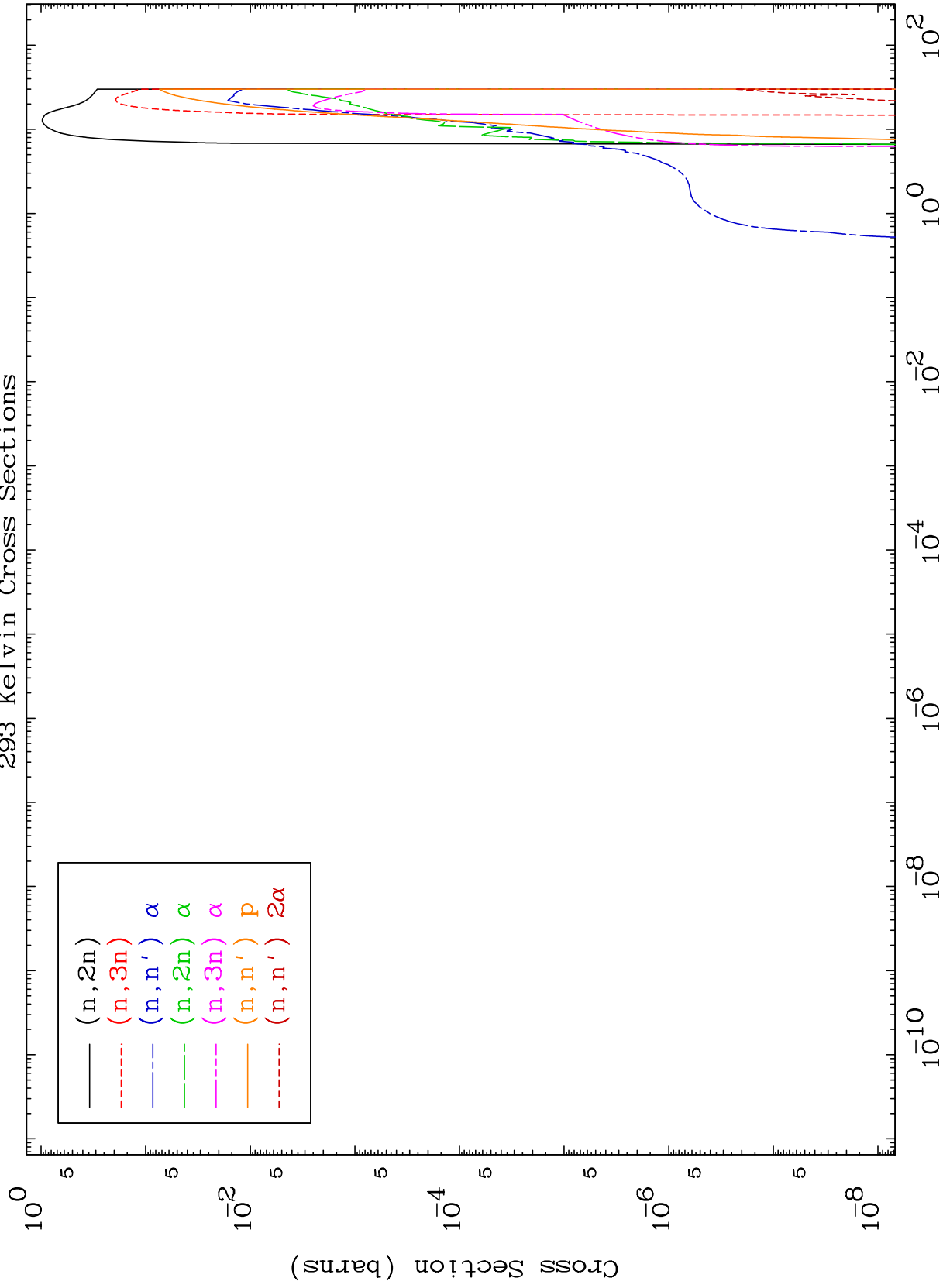
100-Fm-247



MAT 9928

Neutron Production
293 Kelvin Cross Sections

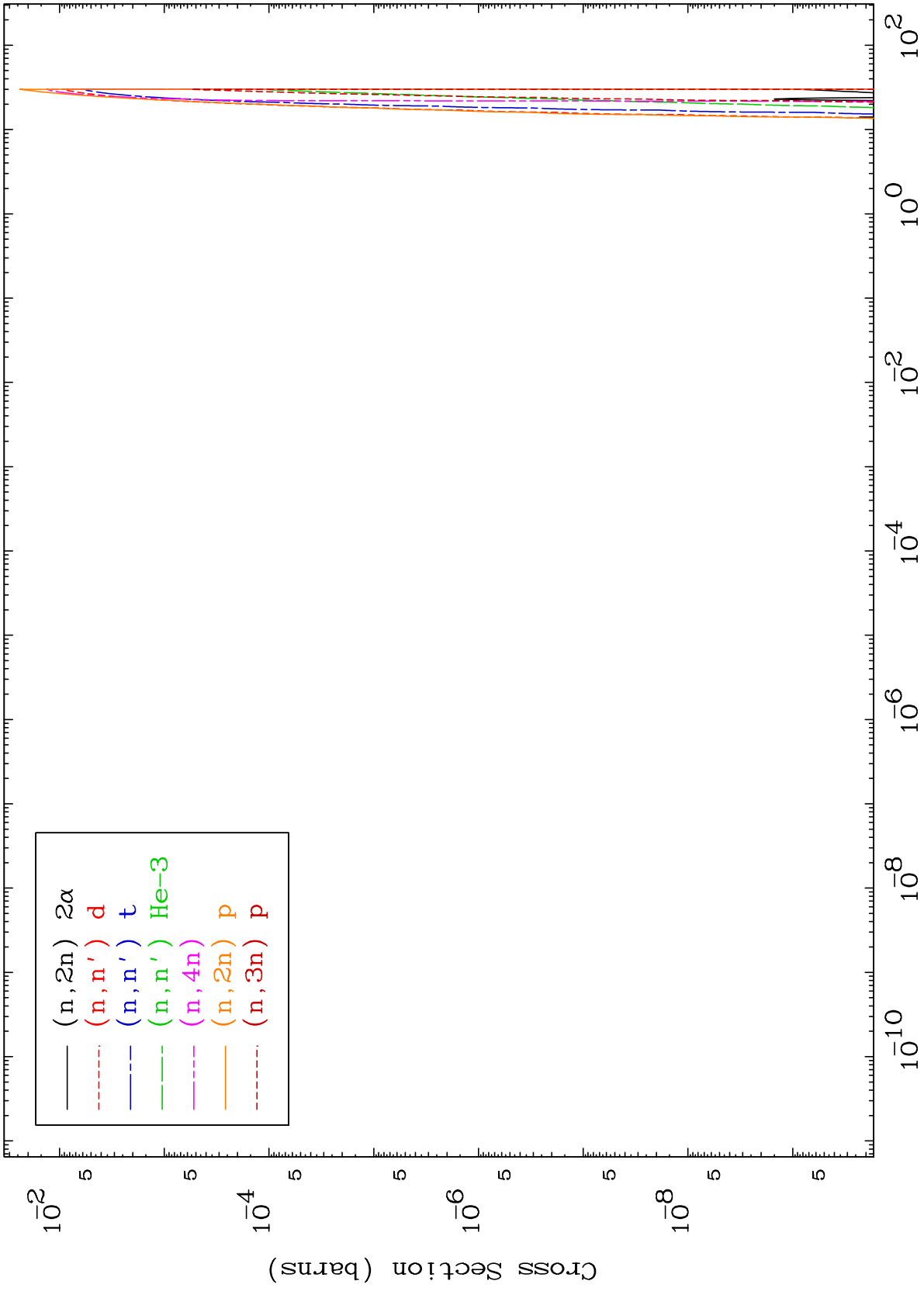
100-Fm-247



MAT 9928

Neutron Production
293 Kelvin Cross Sections

100-Fm-247

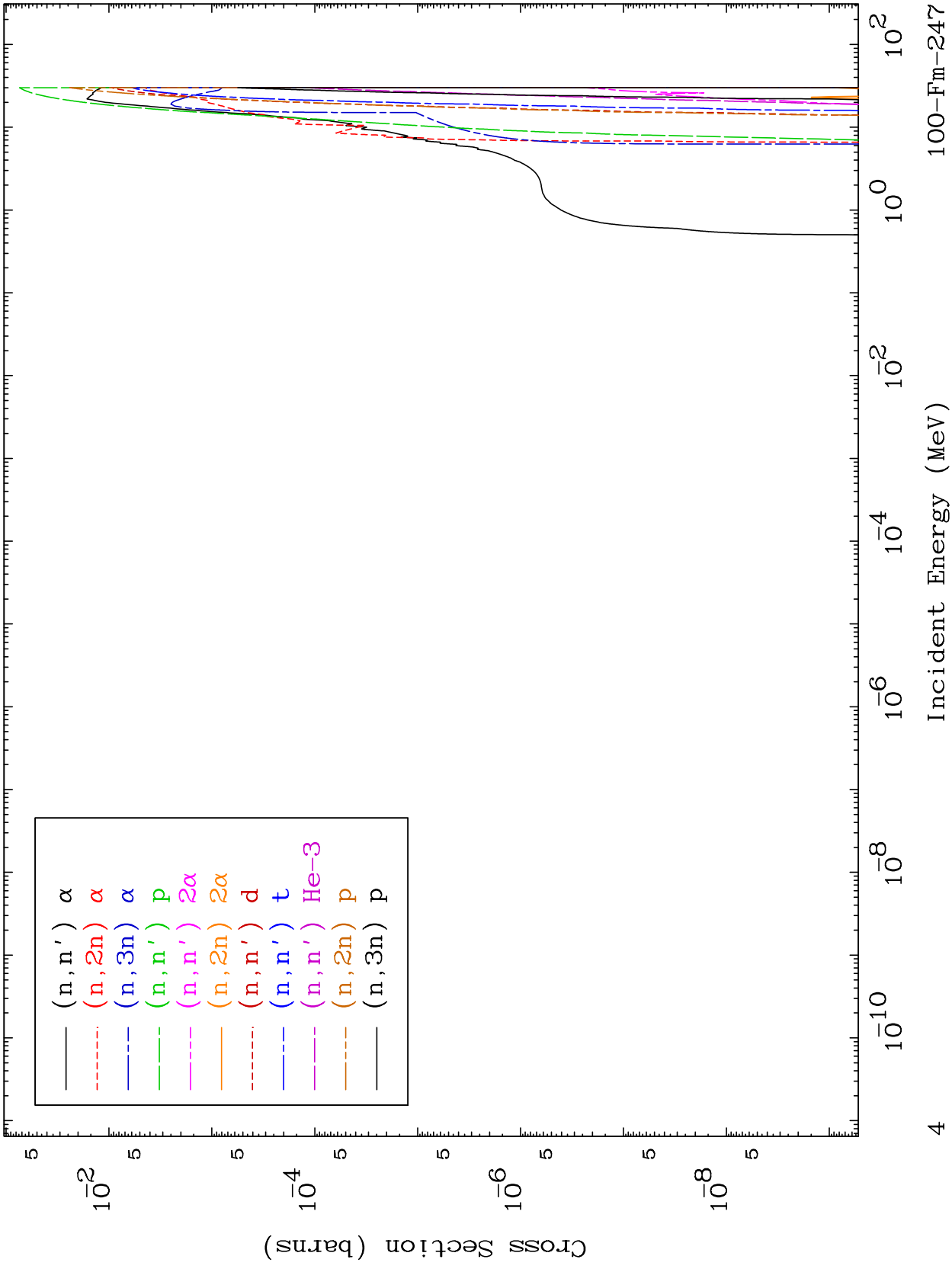


100-Fm-247

MAT 9928

Charged Particle
293 Kelvin Cross Sections

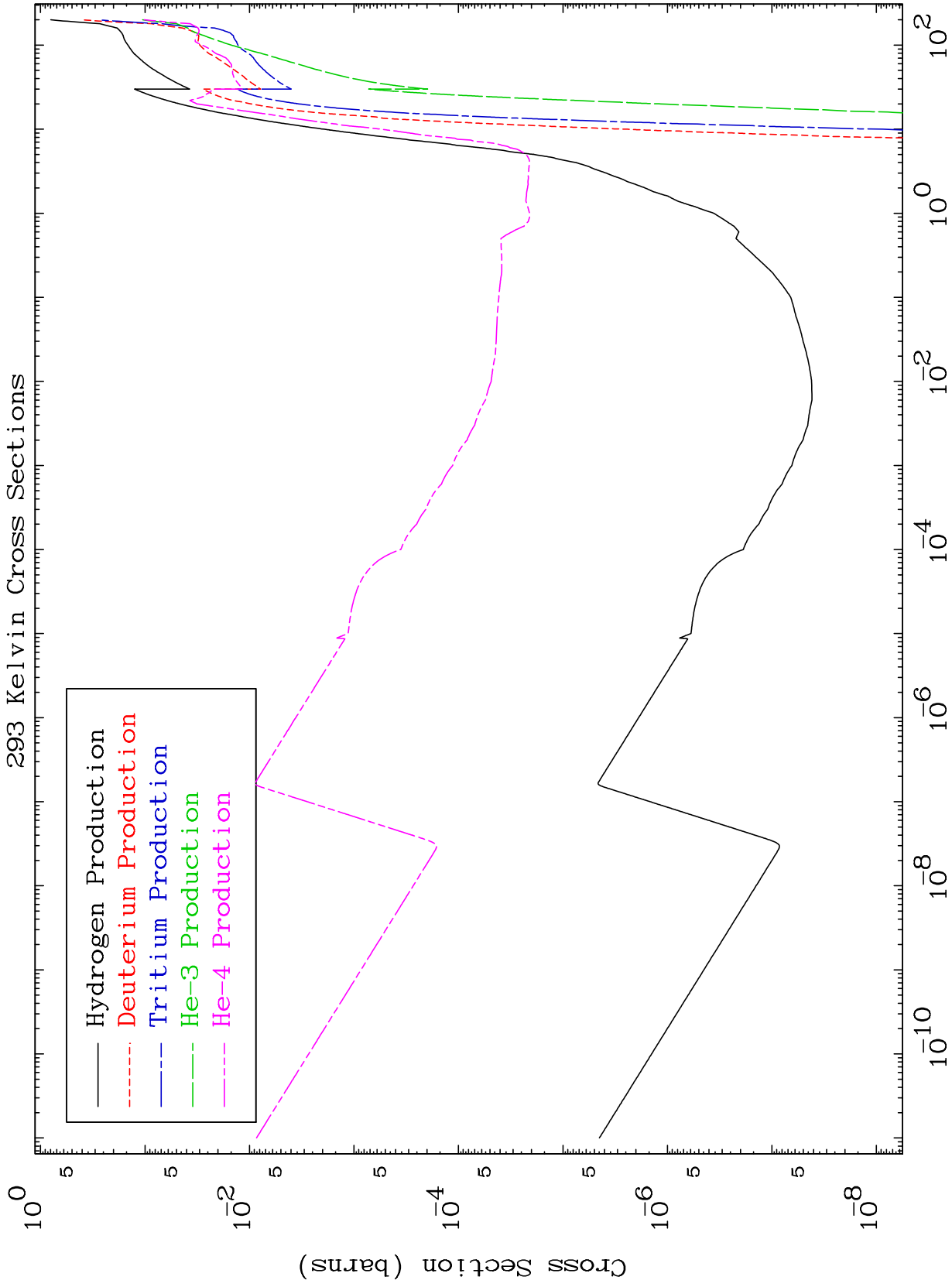
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MAT 9928

Particle Production
293 Kelvin Cross Sections

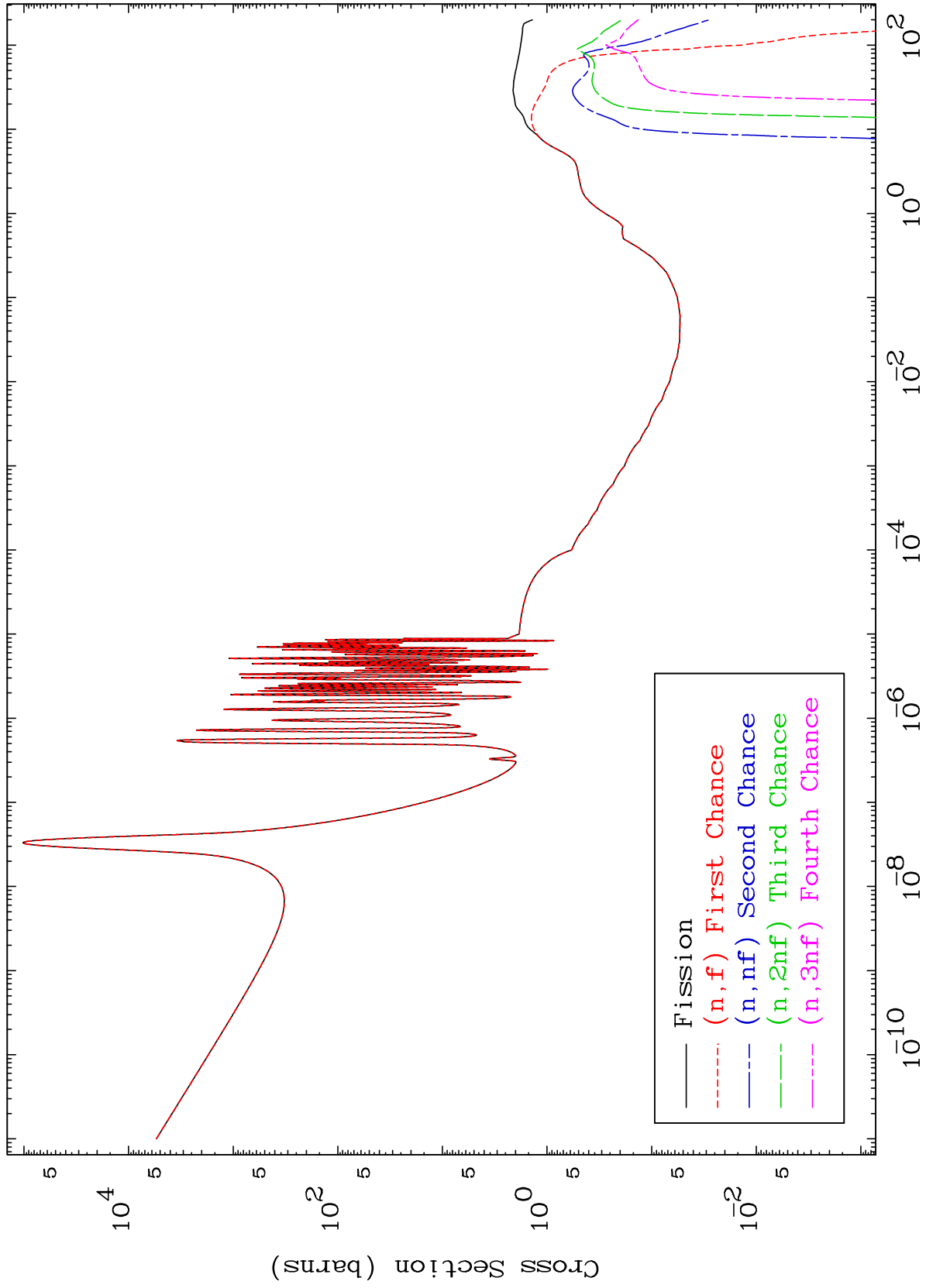
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MAT 9928

Fission
293 Kelvin Cross Sections

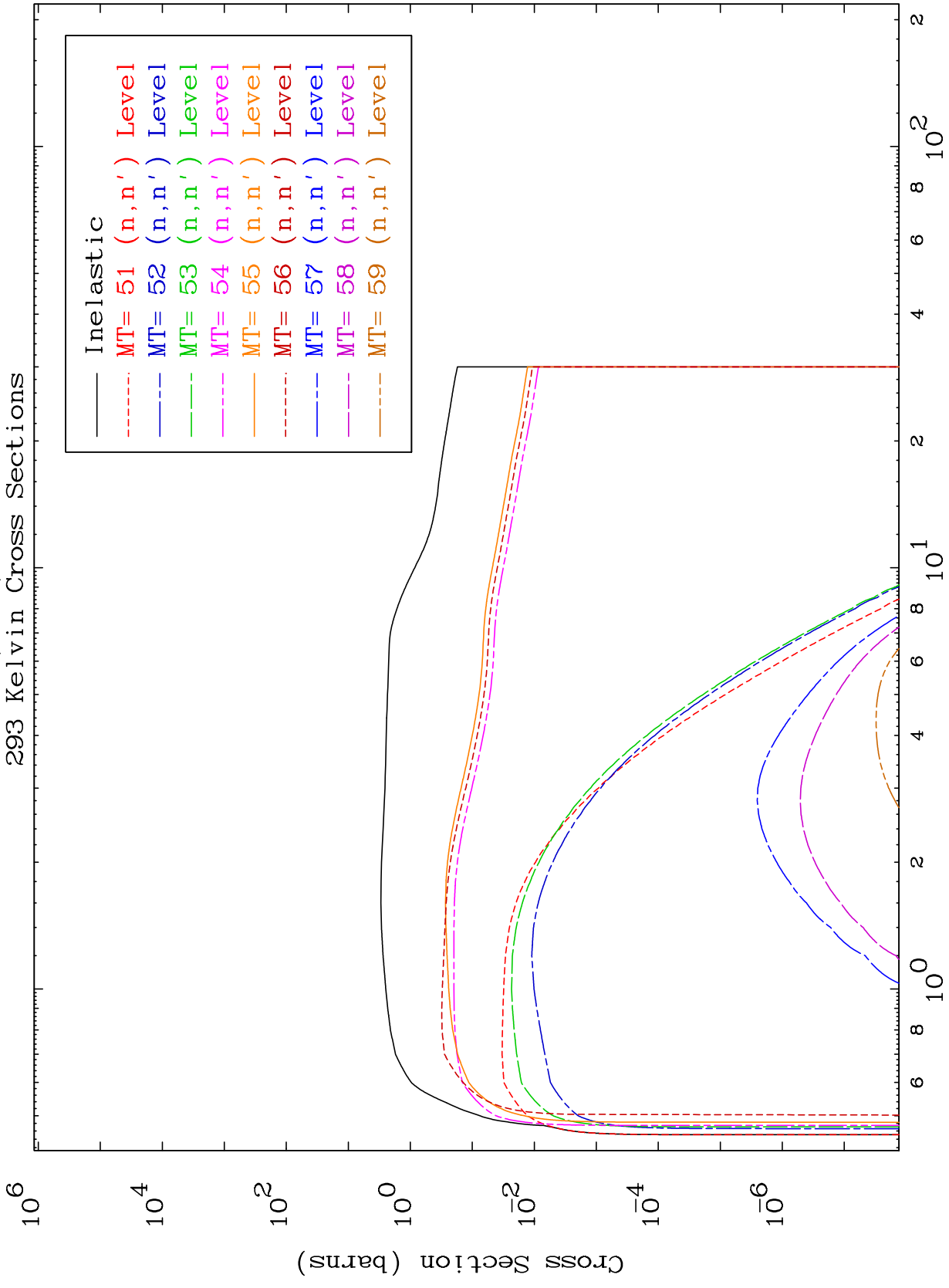
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MAT 9928

(n,n') Level
293 Kelvin Cross Sections

100-Fm-247



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Incident Energy (MeV)

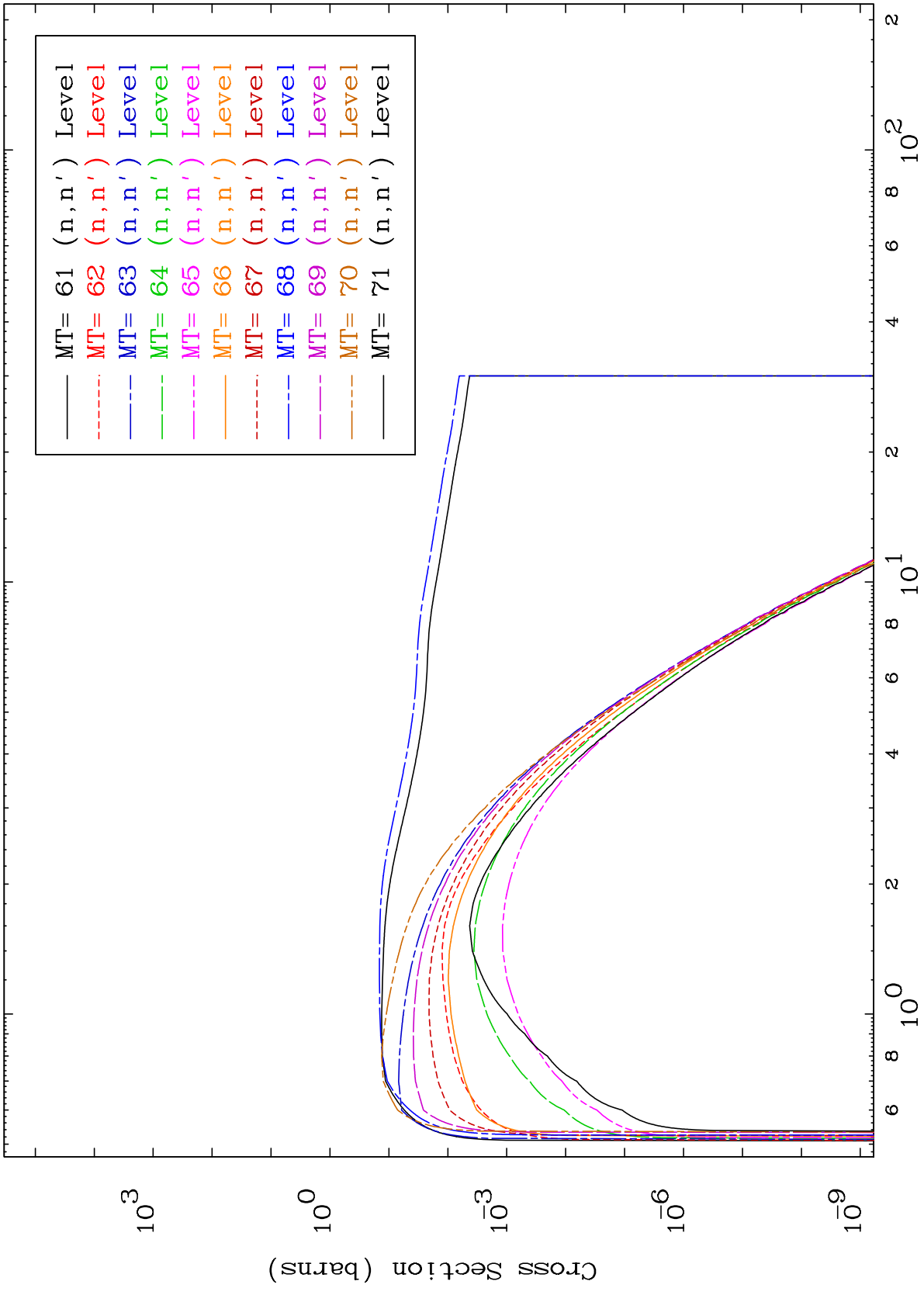
100-Fm-247

MAT 9928

(n,n') Level

100-Fm-247

293 Kelvin Cross Sections



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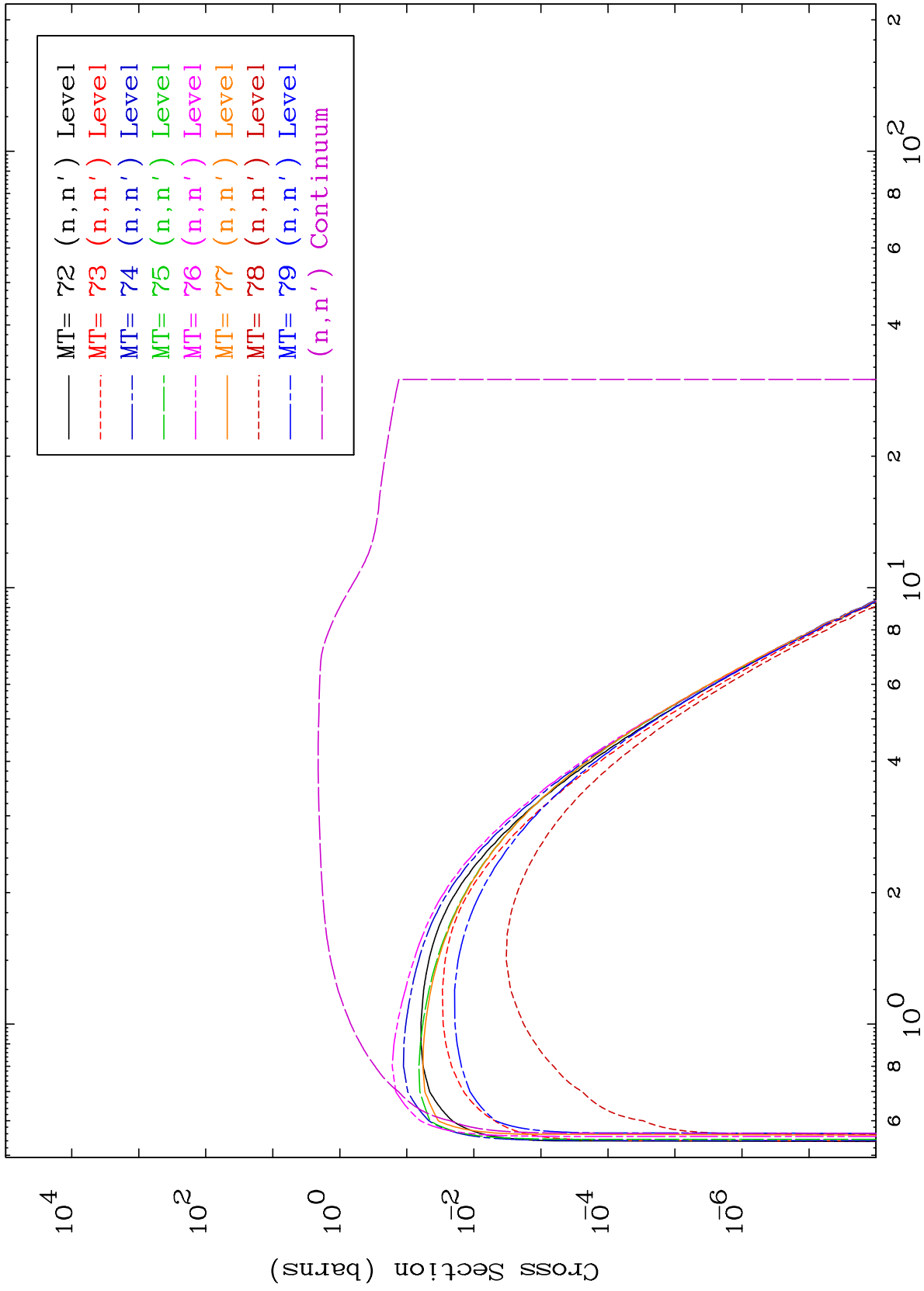
Incident Energy (MeV)

100-Fm-247

MAT 9928

(n,n') Level
293 Kelvin Cross Sections

100-Fm-247



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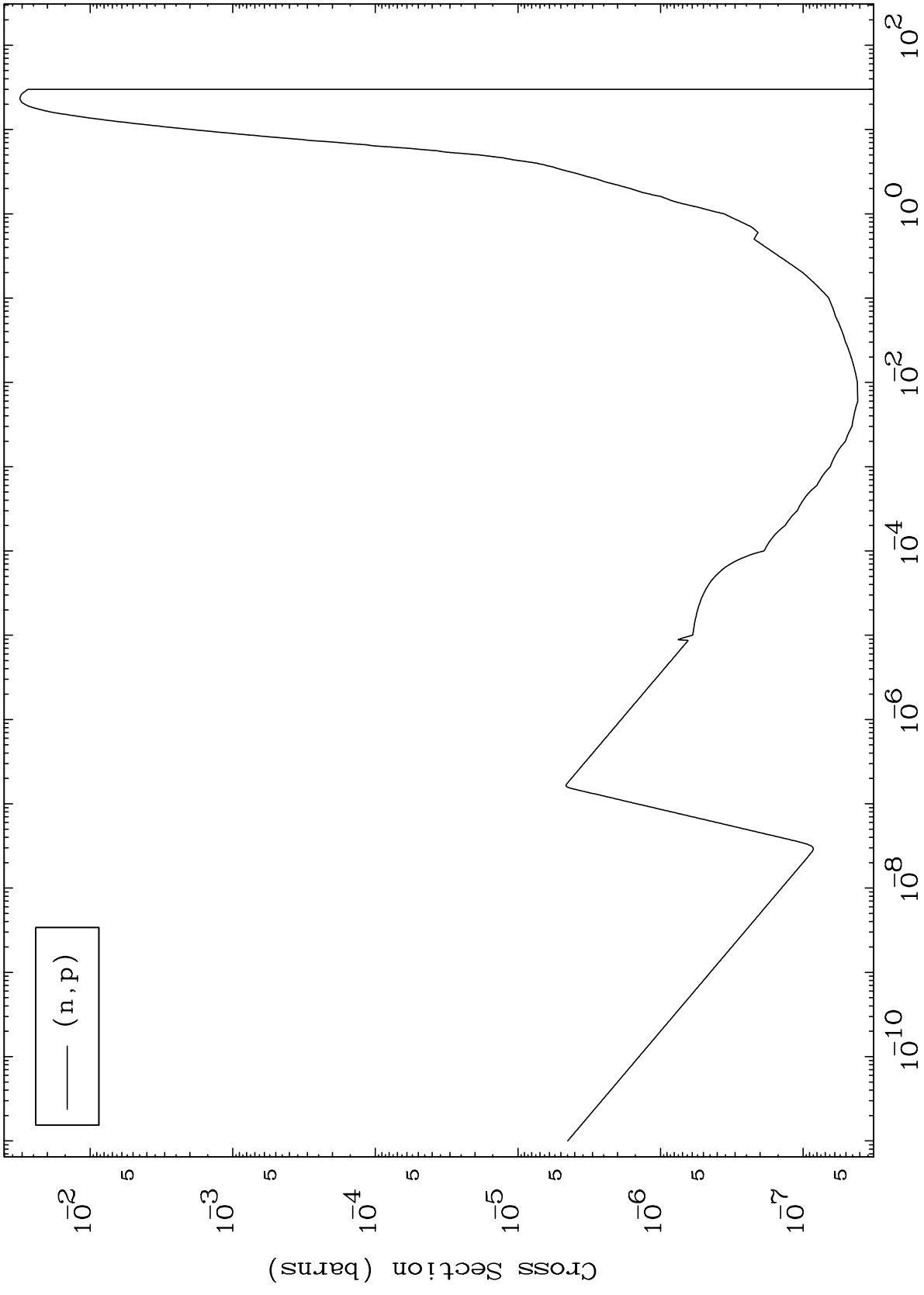
Incident Energy (MeV)

100-Fm-247

MAT 9928

(n,p) Levels
293 Kelvin Cross Sections

100-Fm-247



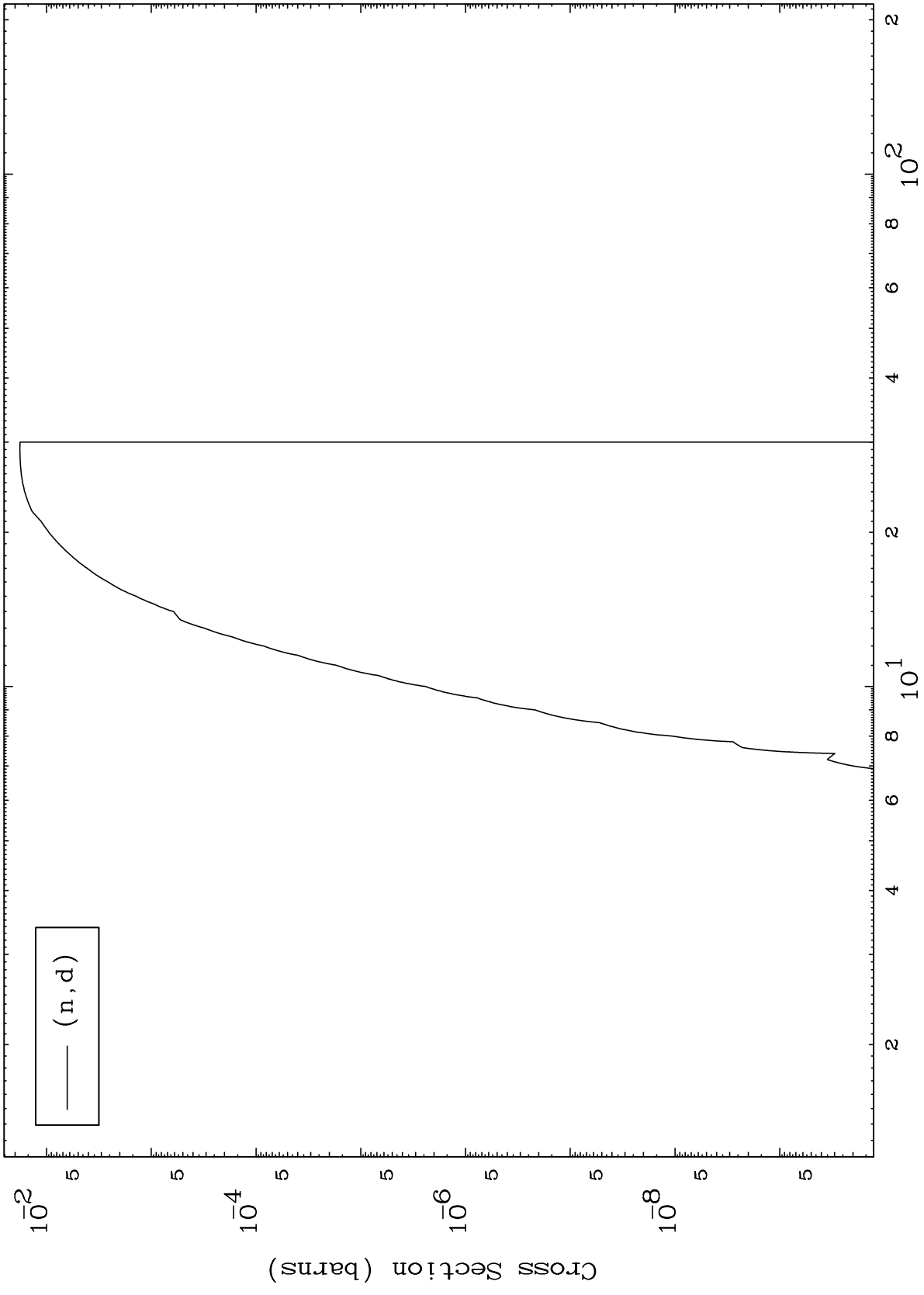
(n,p)

100-Fm-247

MAT 9928

(n,d) Levels
293 Kelvin Cross Sections

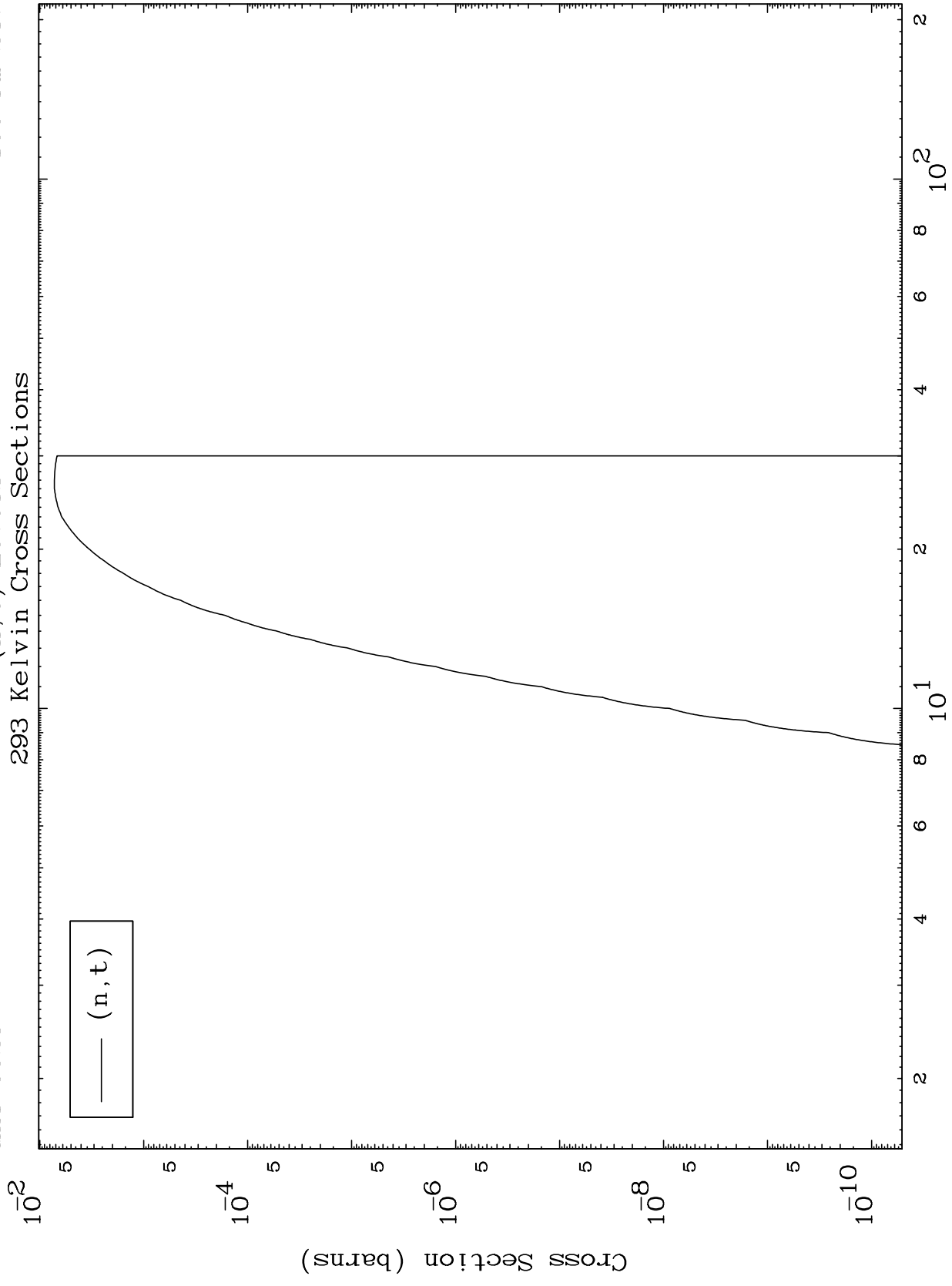
100-Fm-247



MAT 9928

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-247

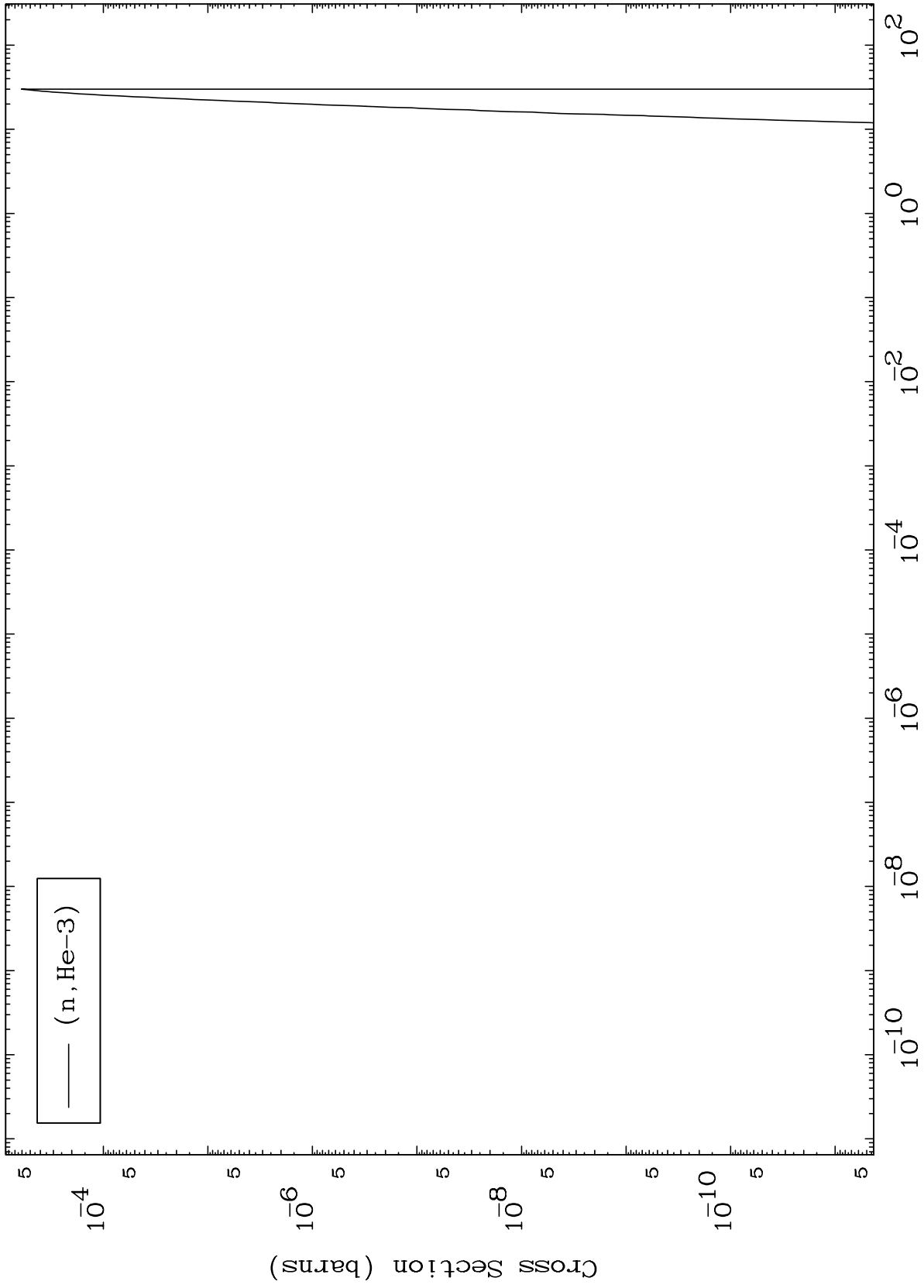


(n,t)

MAT 9928

(n,He3) Levels
293 Kelvin Cross Sections

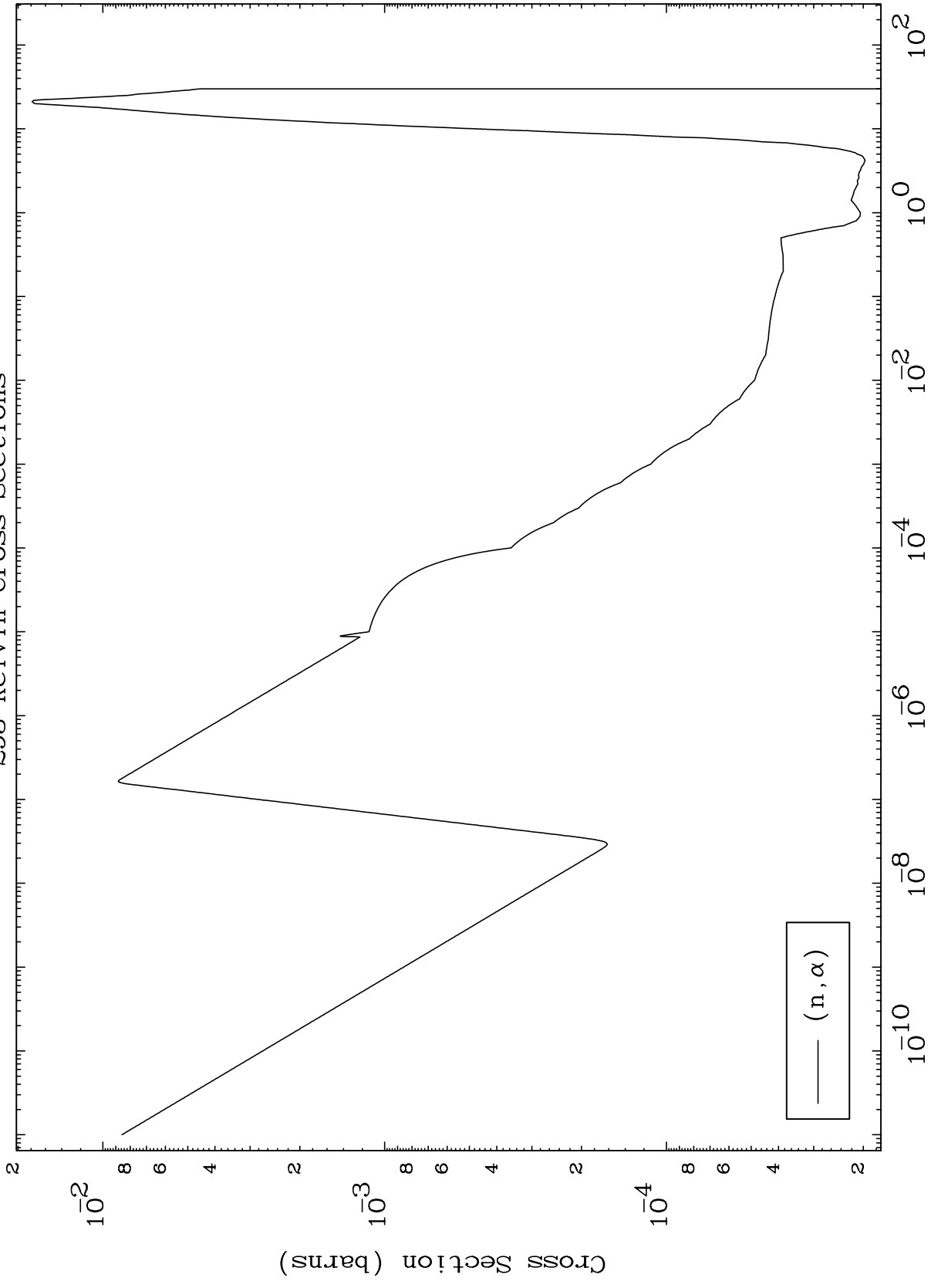
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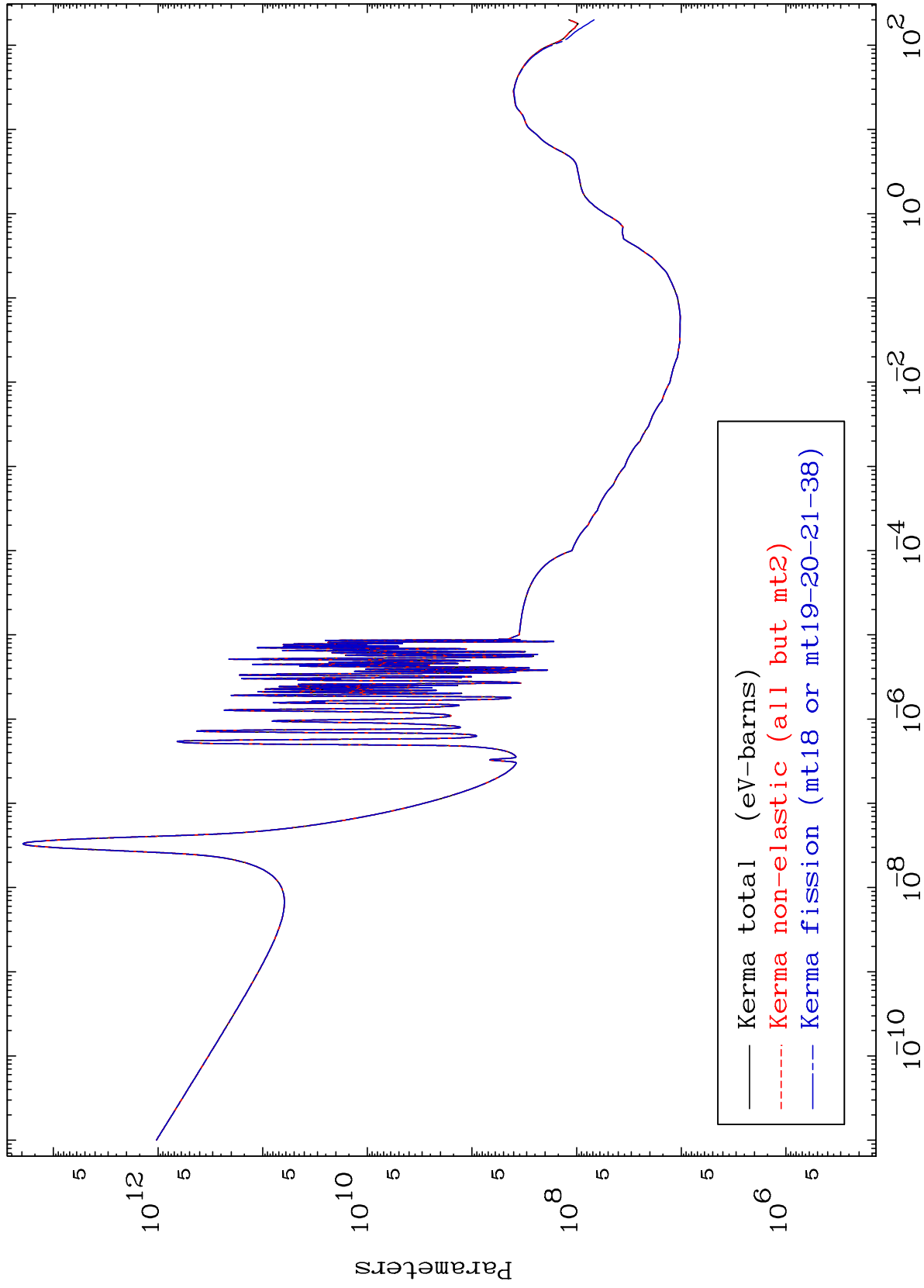
MAT 9928

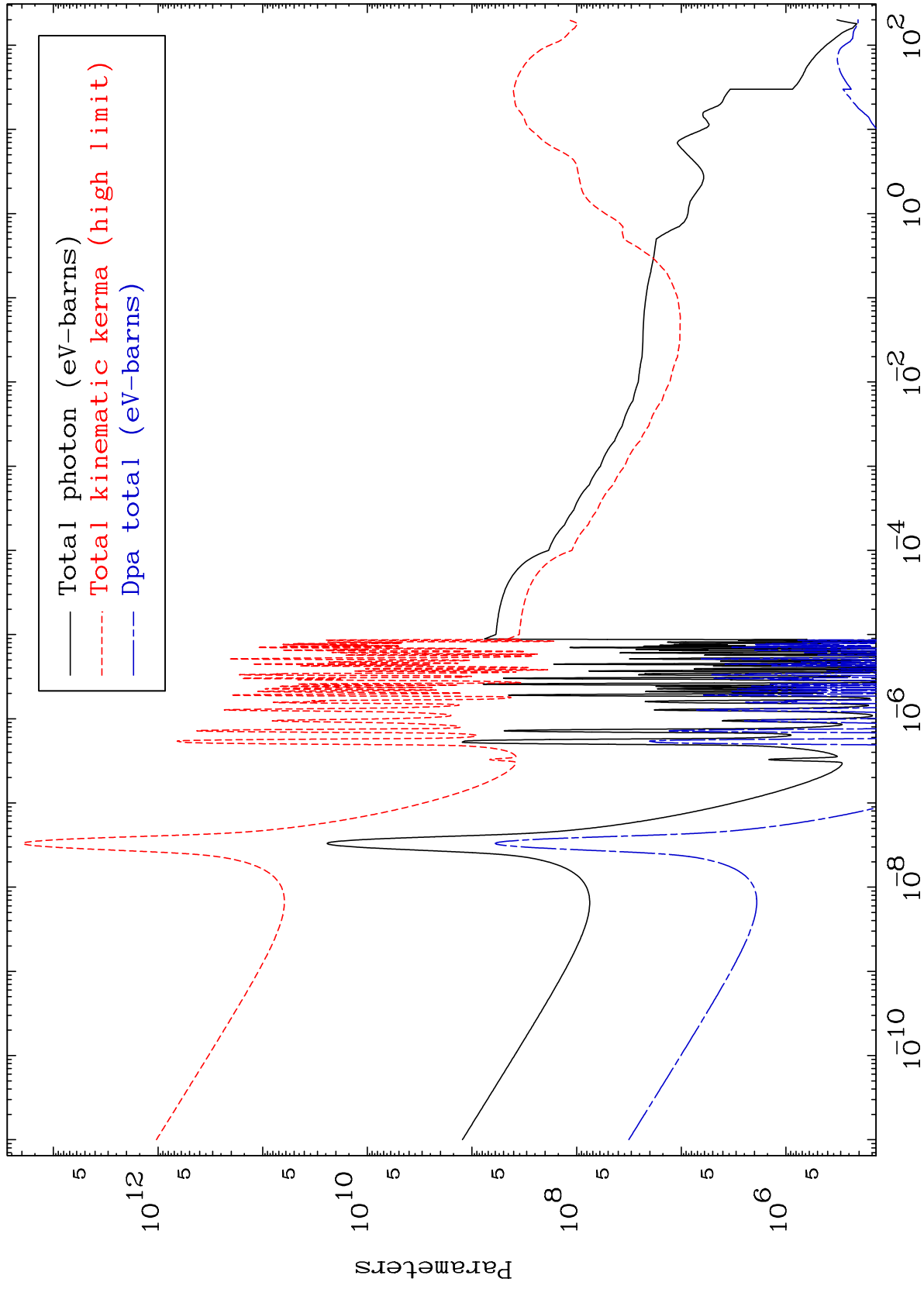
(n,α) Levels
293 Kelvin Cross Sections

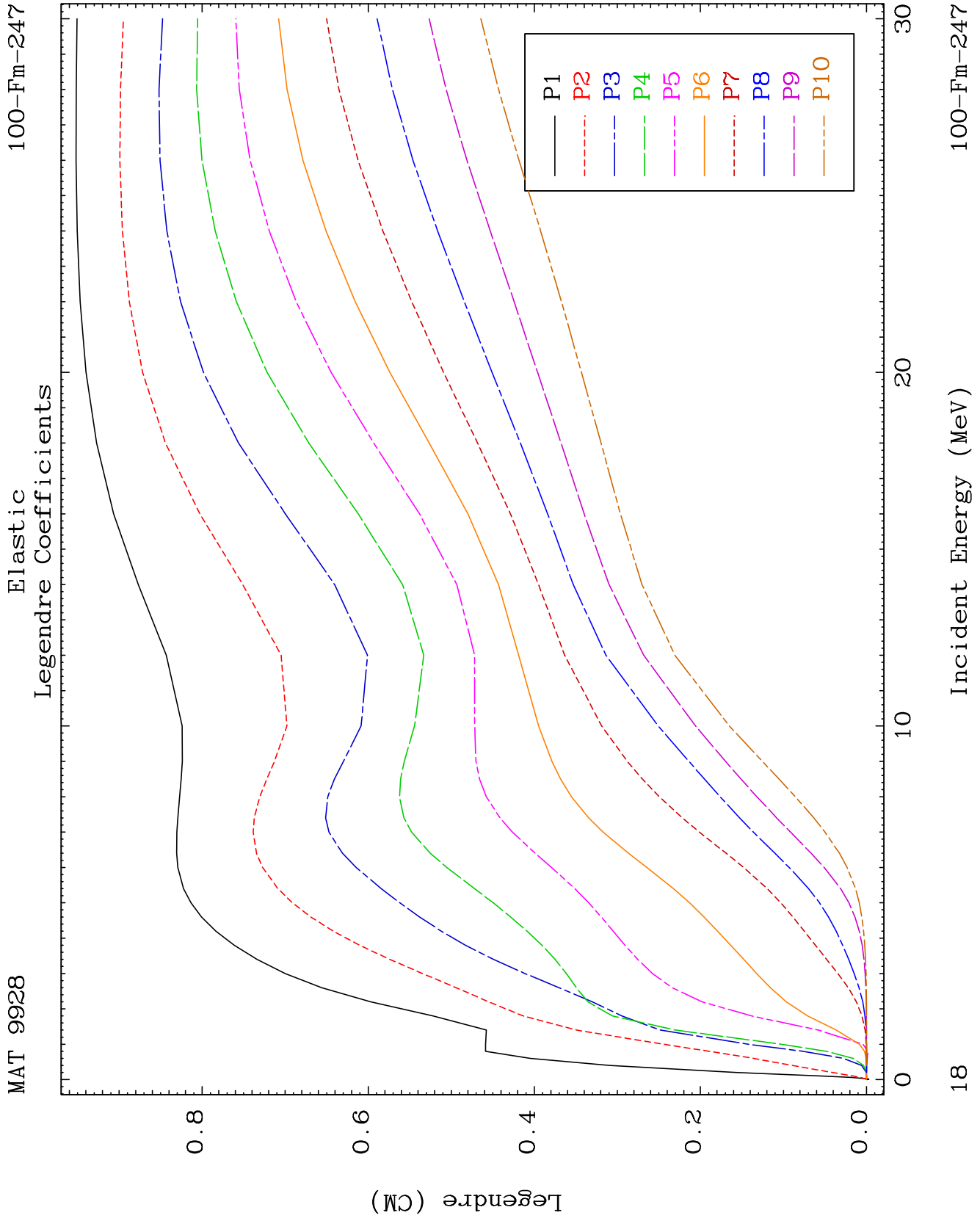
100-Fm-247



— (n,α)



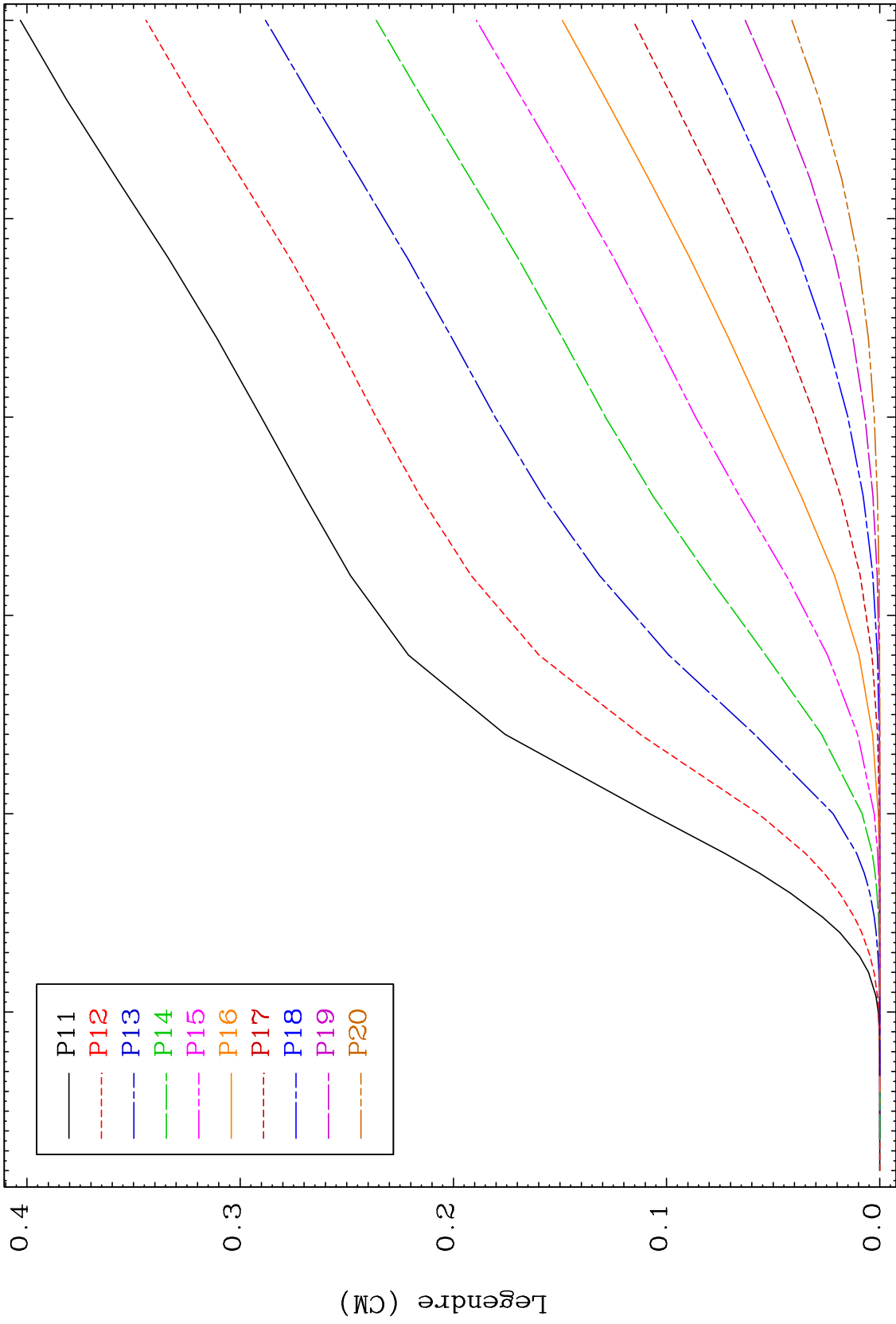




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Elastic Legendre Coefficients

100-Fm-247



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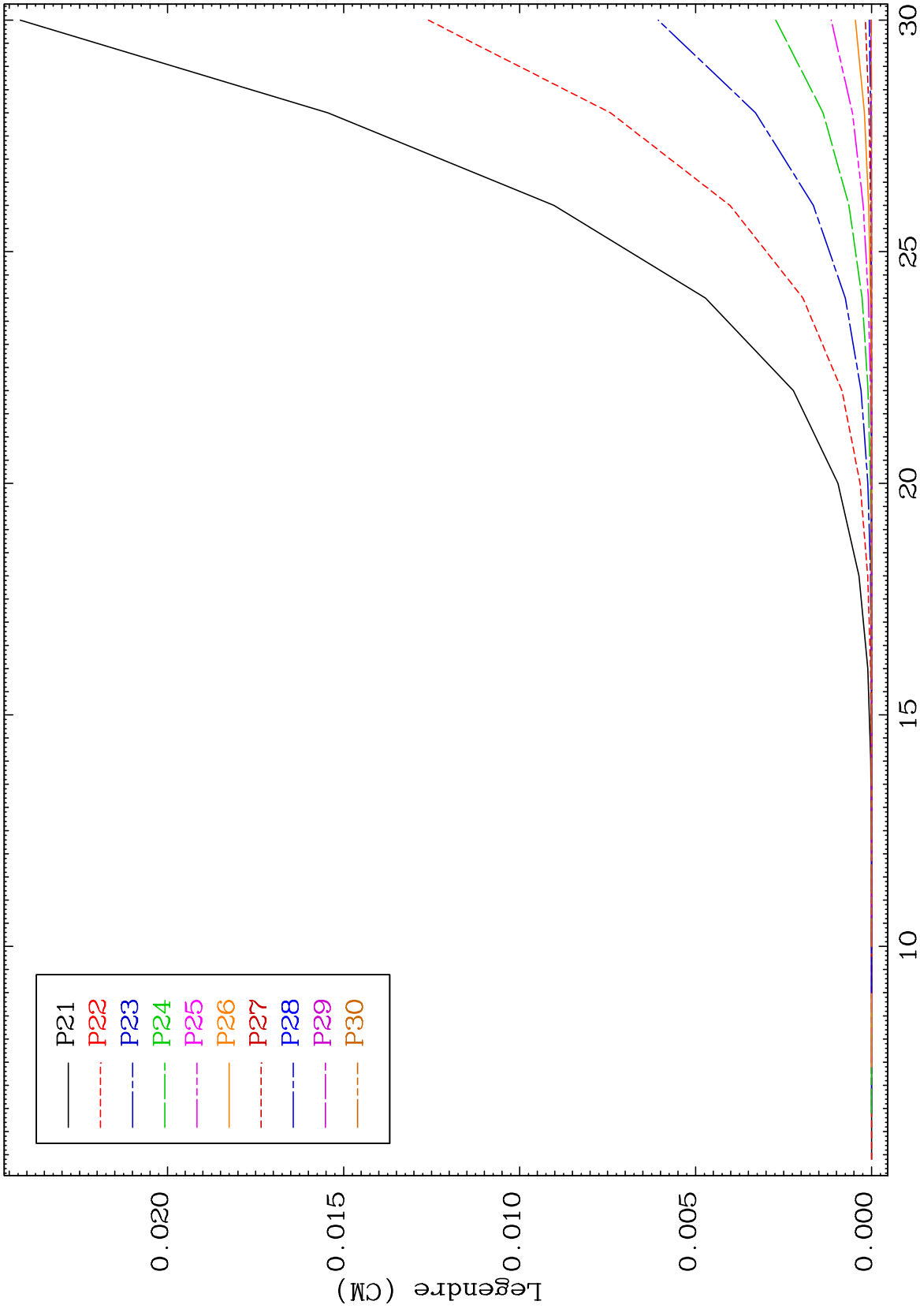
Incident Energy (MeV)

100-Fm-247

MAT 9928

Elastic Legendre Coefficients

100-Fm-247



20

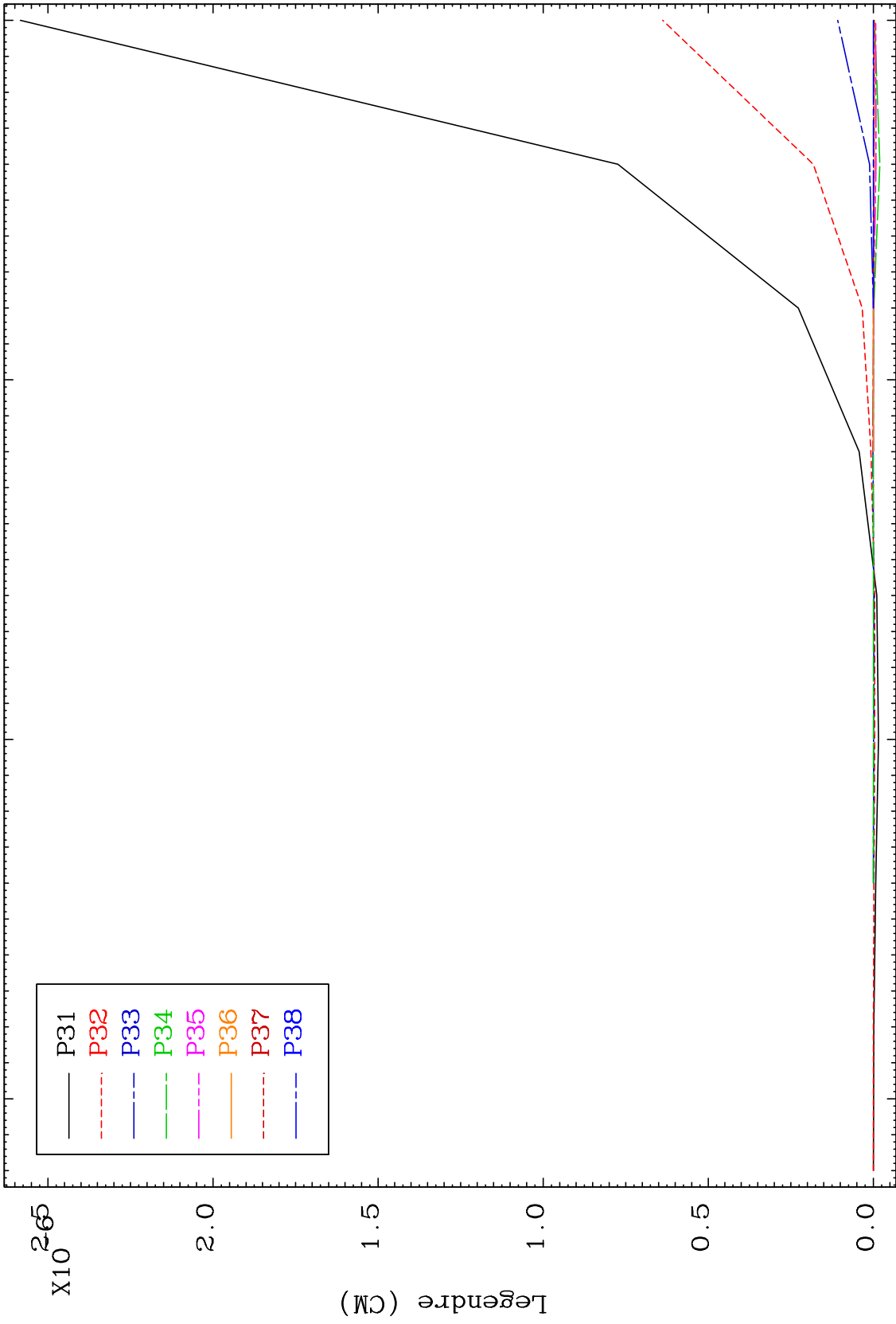
Incident Energy (MeV)

100-Fm-247

MAT 9928

Elastic Legendre Coefficients

100-Fm-247



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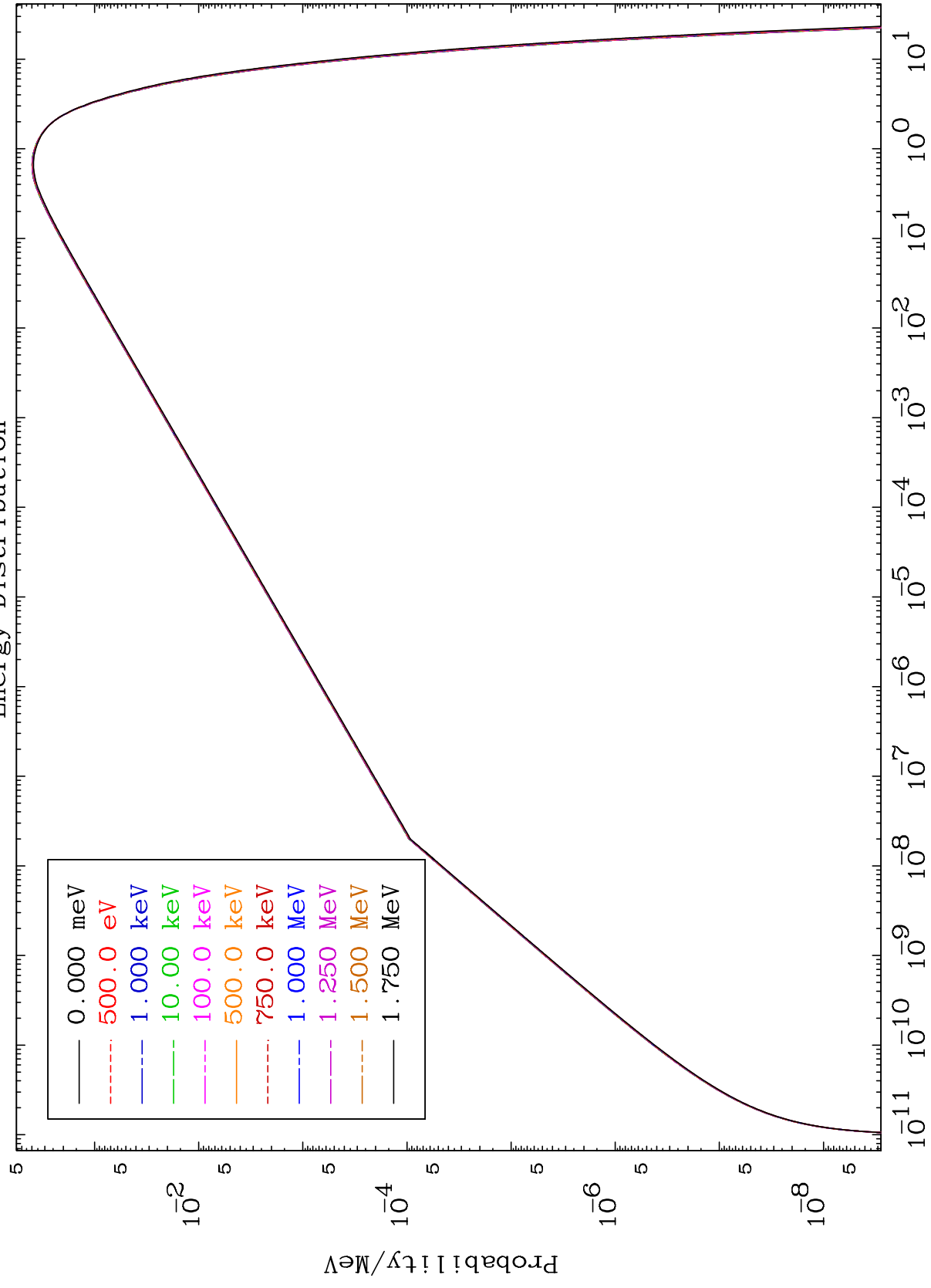
Incident Energy (MeV)

100-Fm-247

MAT 9928

Fission Energy Distribution

100-Fm-247

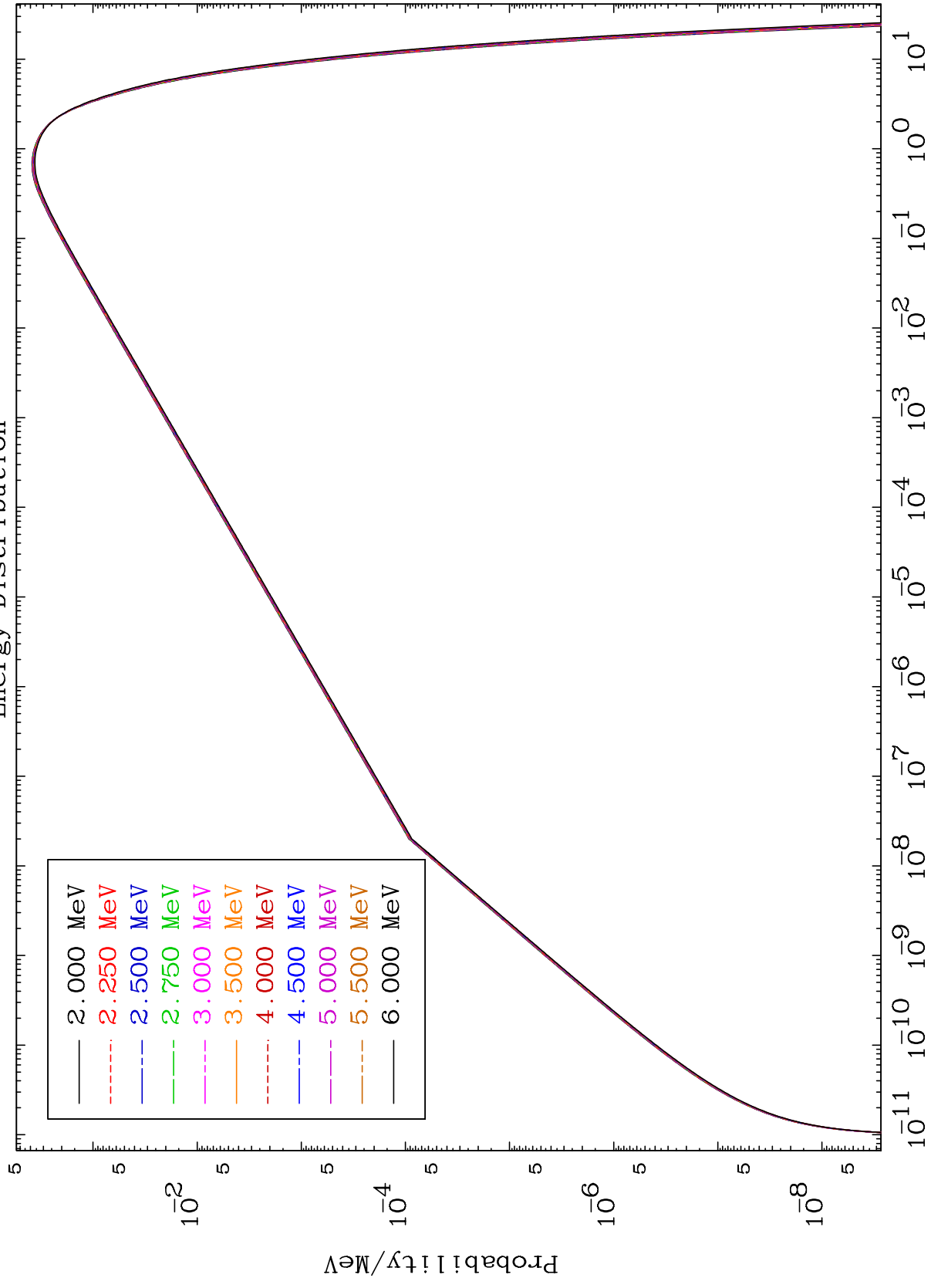


100-Fm-247

MAT 9928

Fission Energy Distribution

100-Fm-247

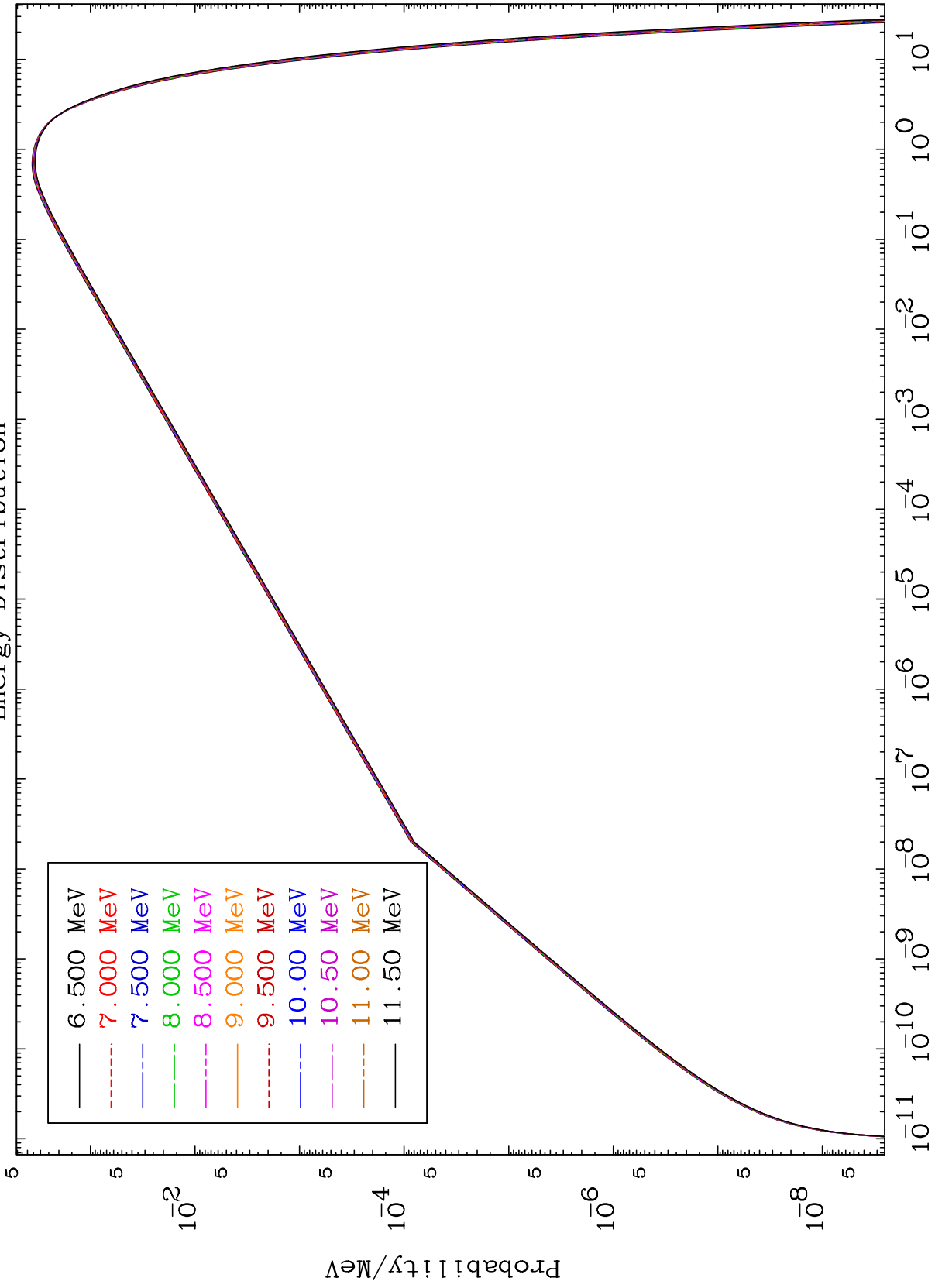


100-Fm-247

MAT 9928

Fission Energy Distribution

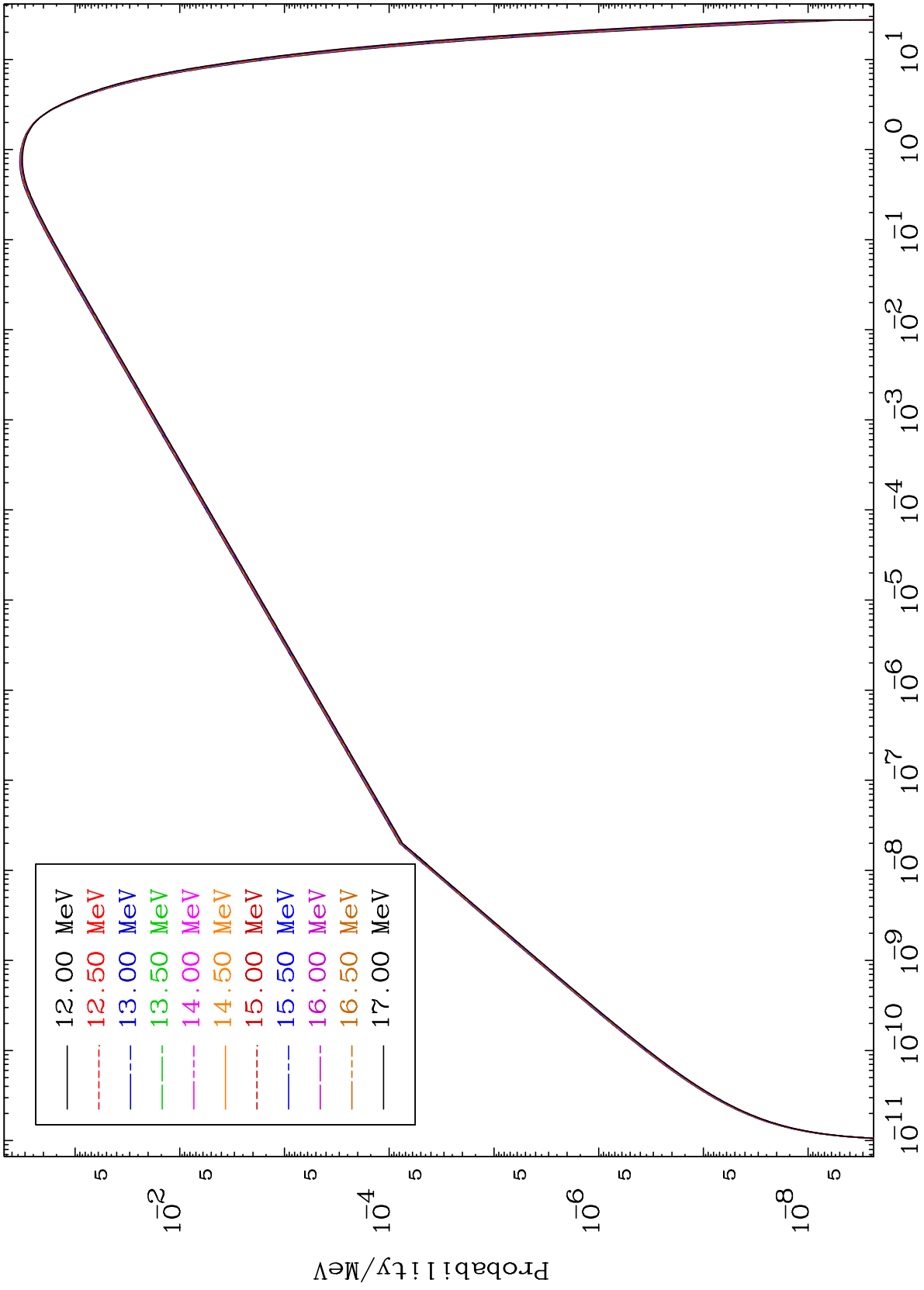
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MAT 9928

Fission Energy Distribution

100-Fm-247

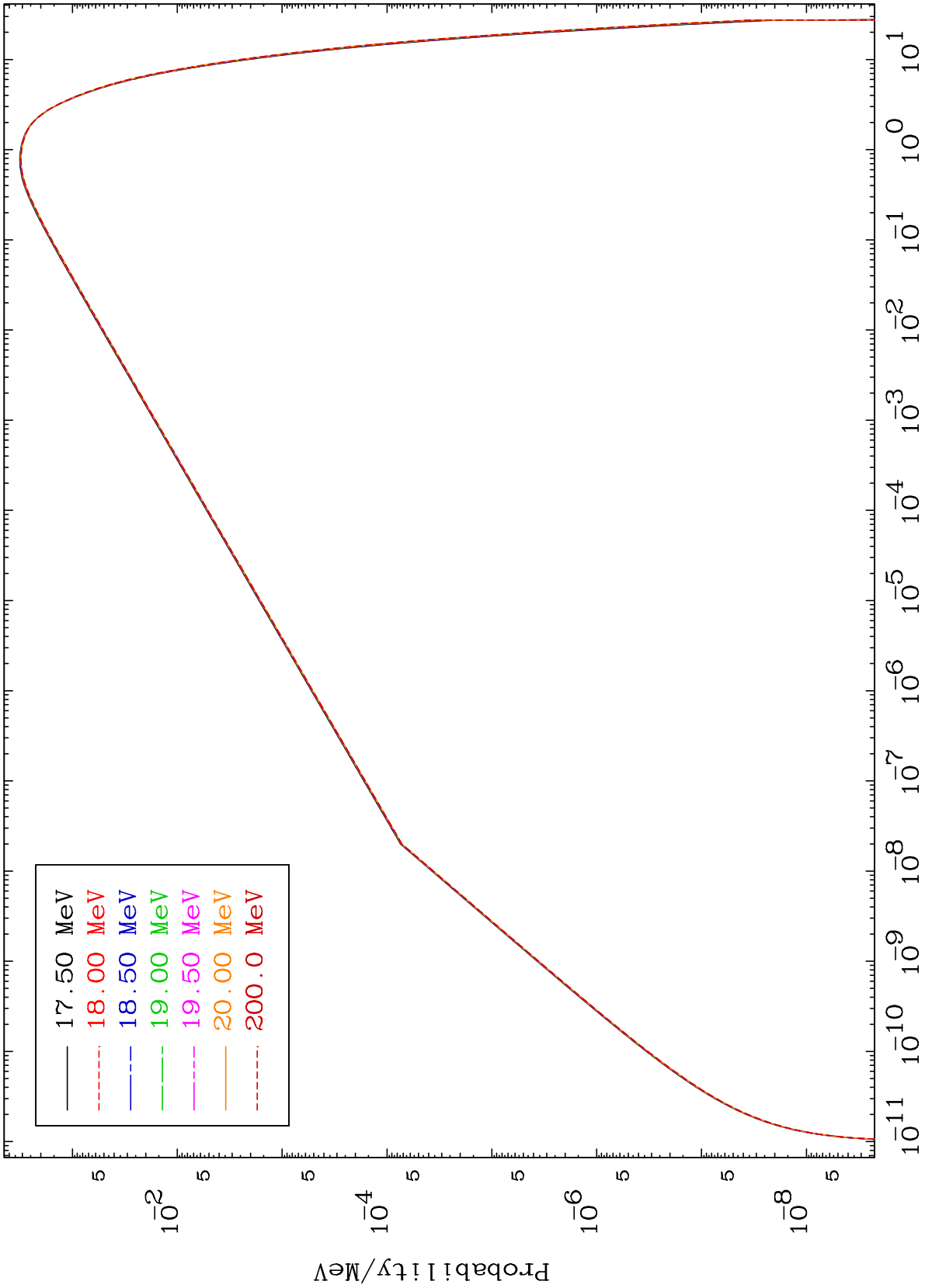


100-Fm-247

MAT 9928

Fission Energy Distribution

100-Fm-247

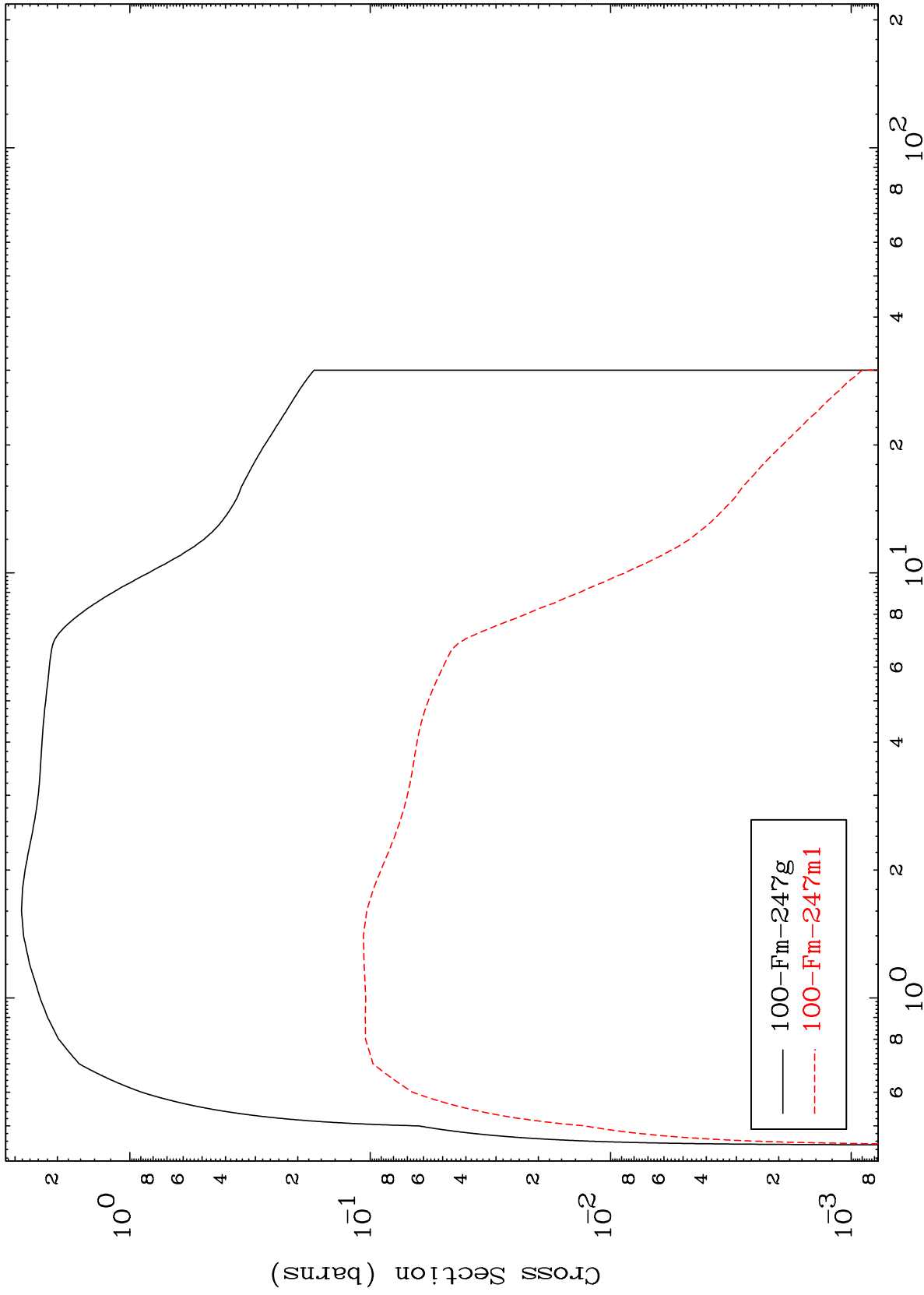


100-Fm-247

MAT 9928

100-Fm-247

Inelastic
Radionuclide Production Cross Section



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100-Fm-247