

Program EVALPLOT
(Version 2018-1)

by

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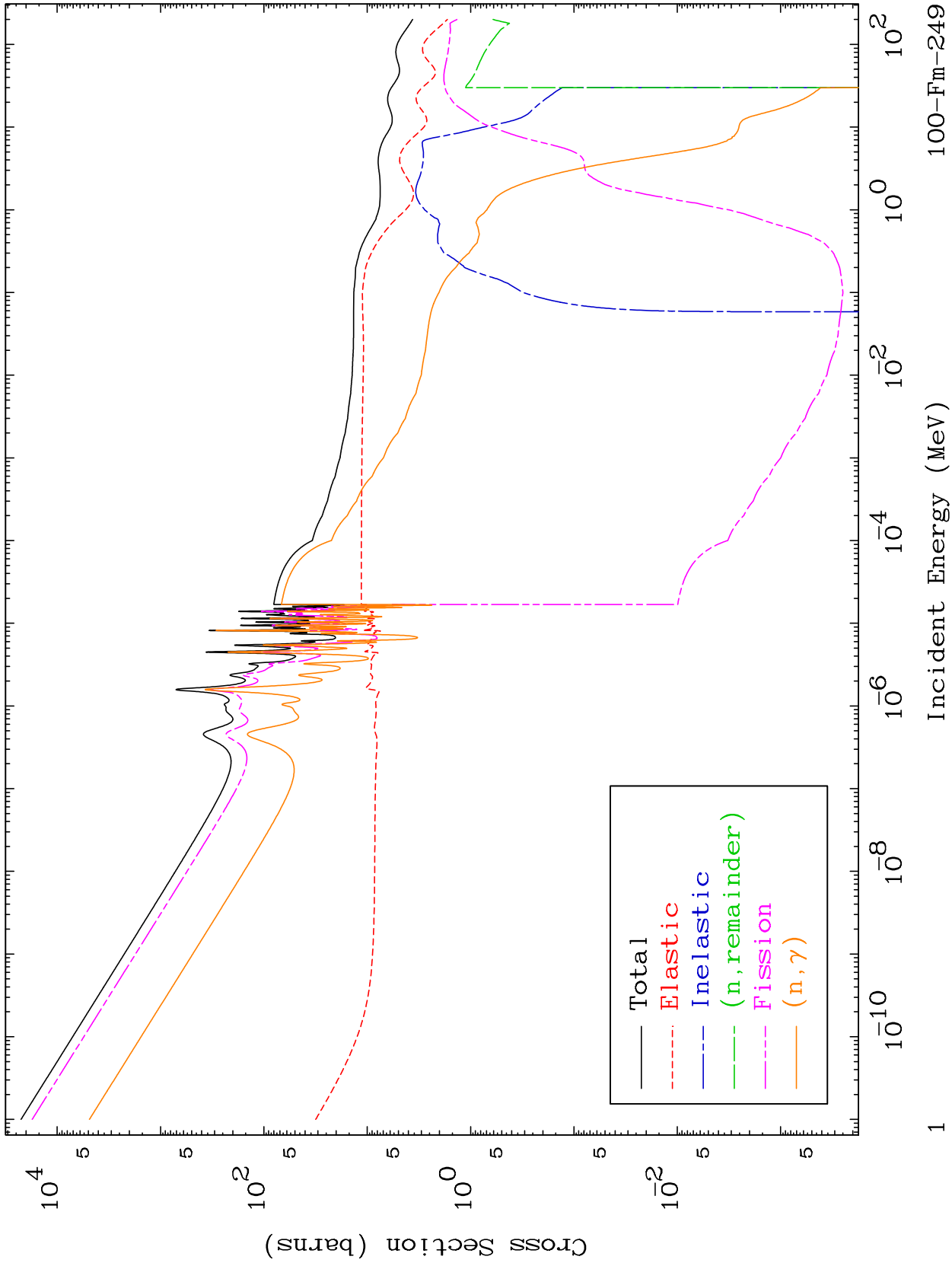
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9930

Major
293 Kelvin Cross Sections

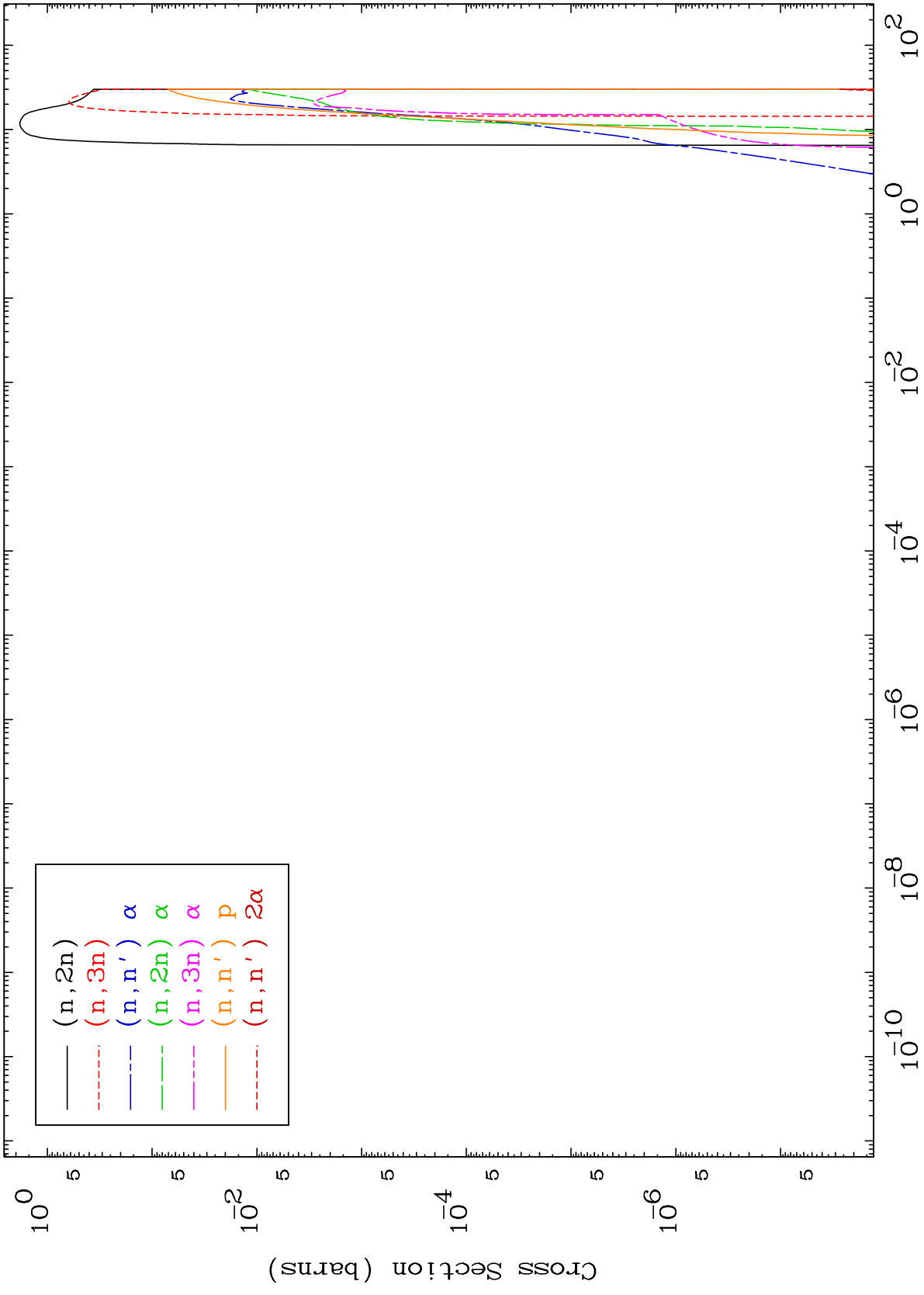
100-Fm-249



MAT 9930

Neutron Production
293 Kelvin Cross Sections

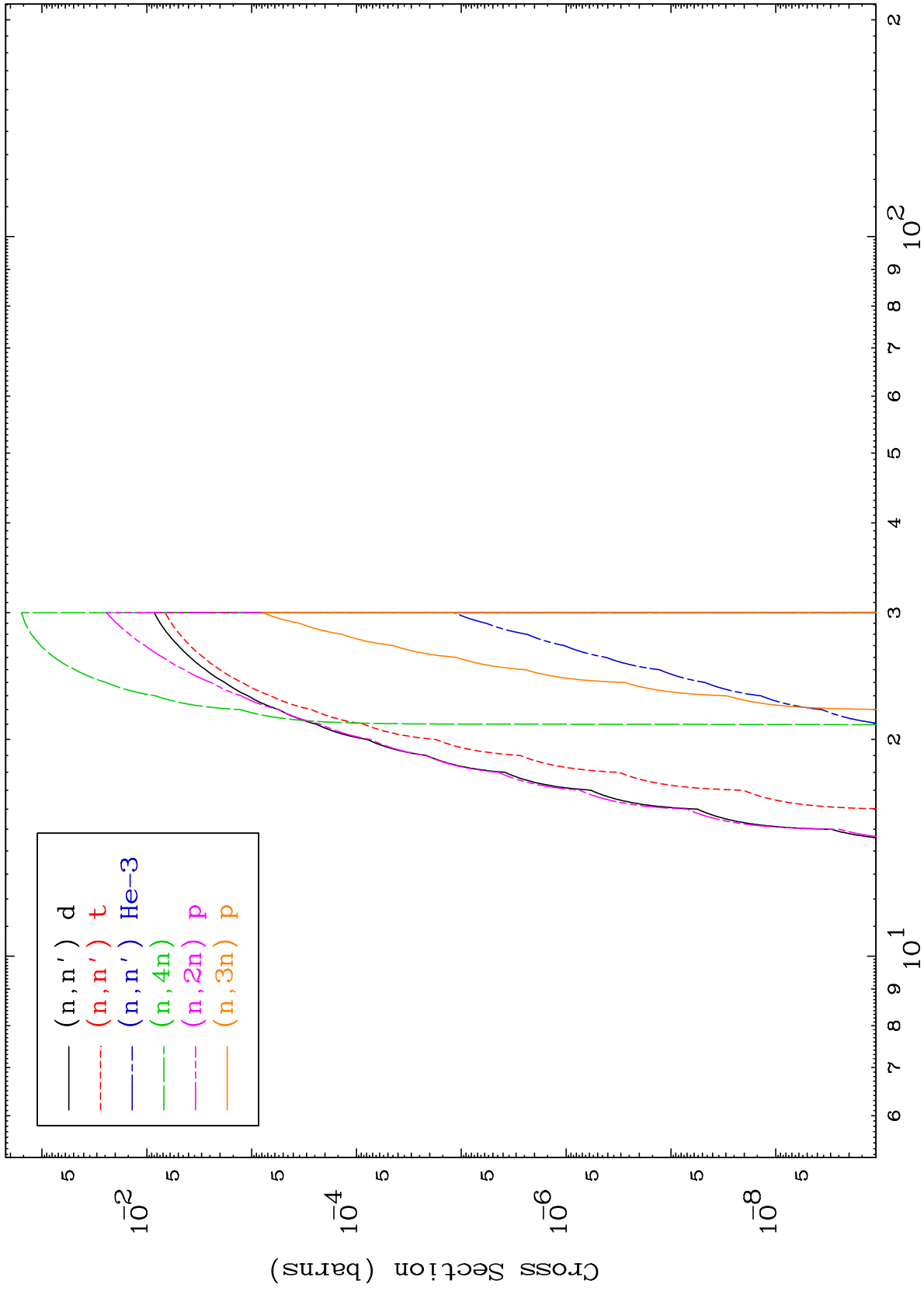
100-Fm-249



MAT 9930

Neutron Production
293 Kelvin Cross Sections

100-Fm-249



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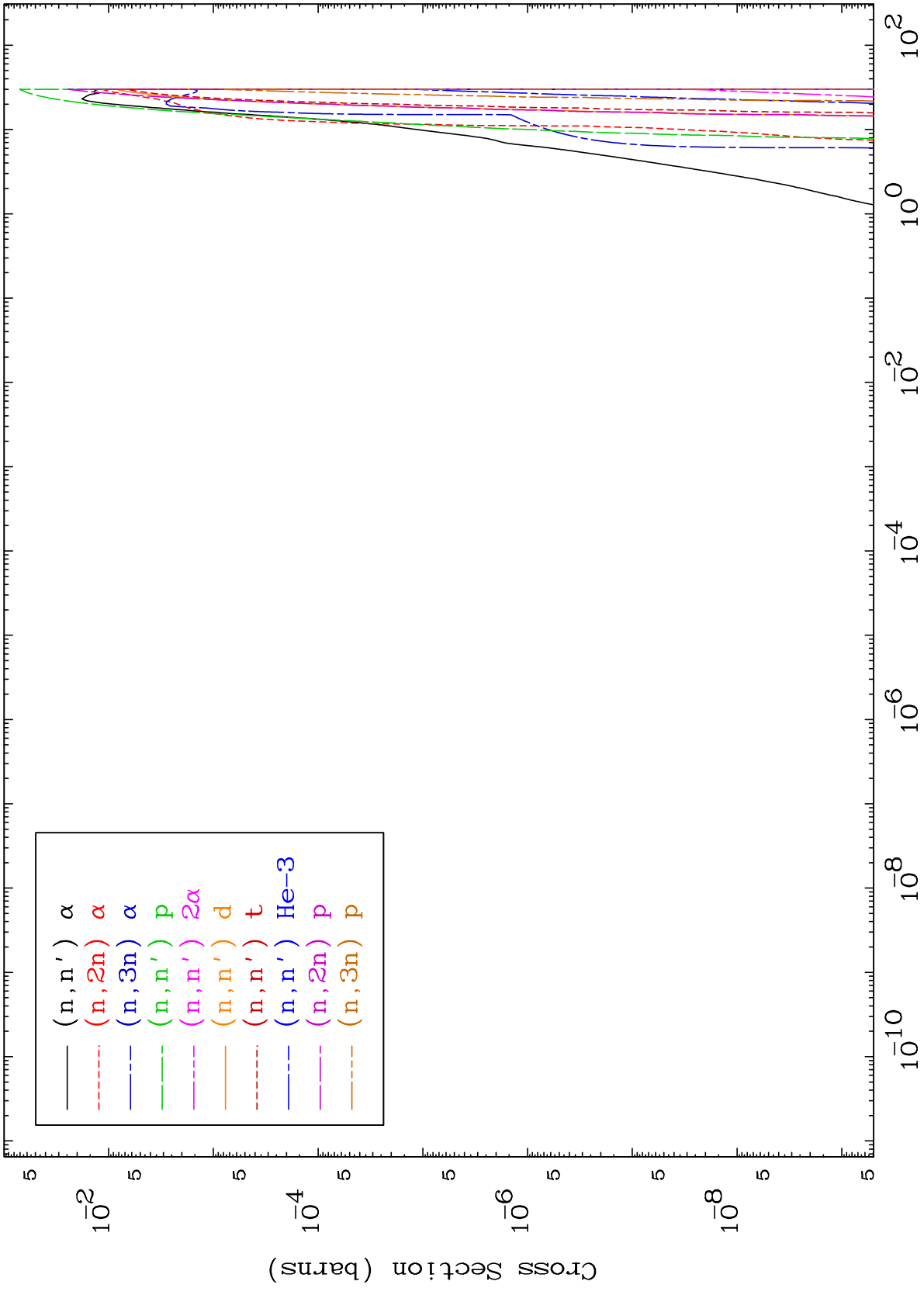
Incident Energy (MeV)

100-Fm-249

MAT 9930

Charged Particle
293 Kelvin Cross Sections

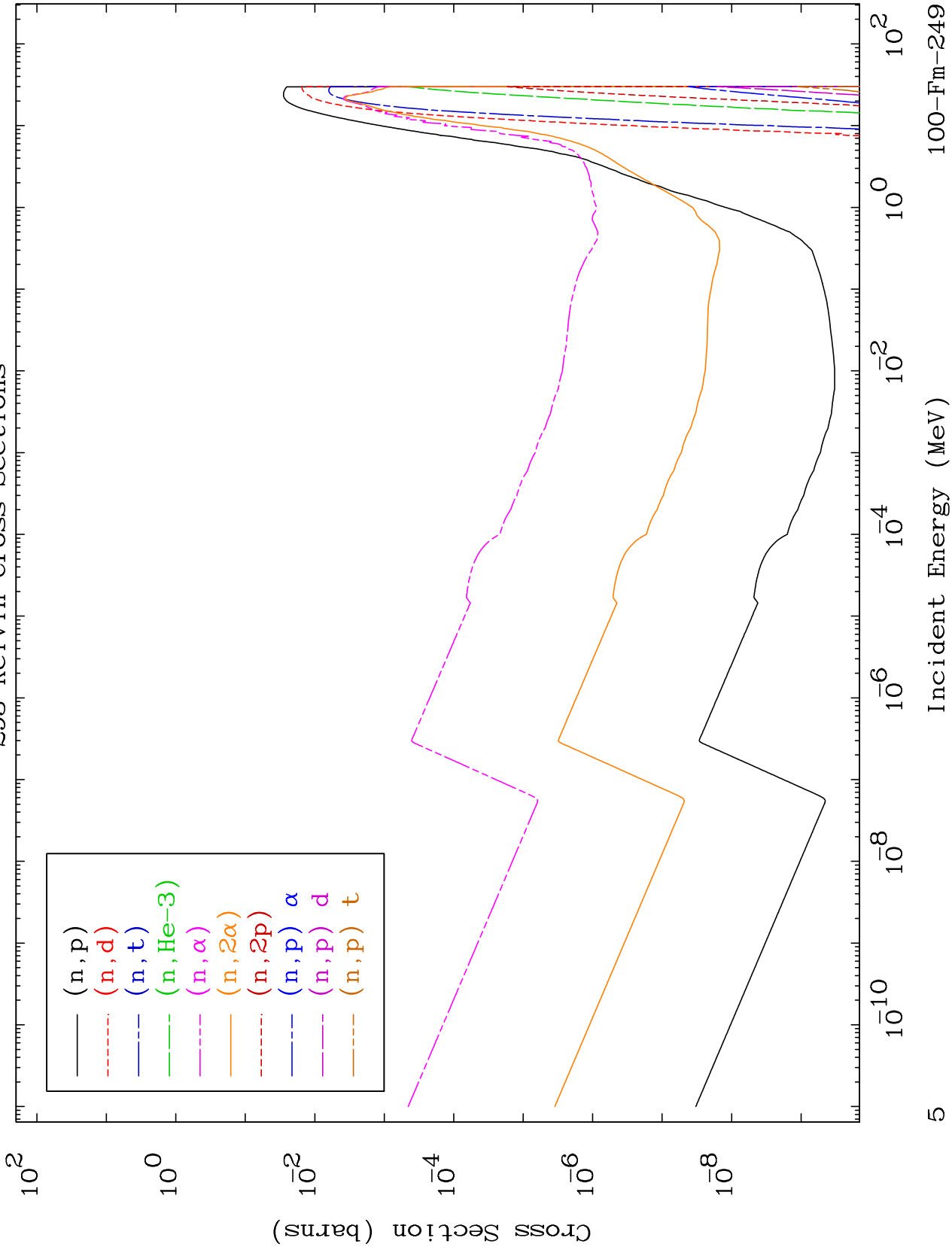
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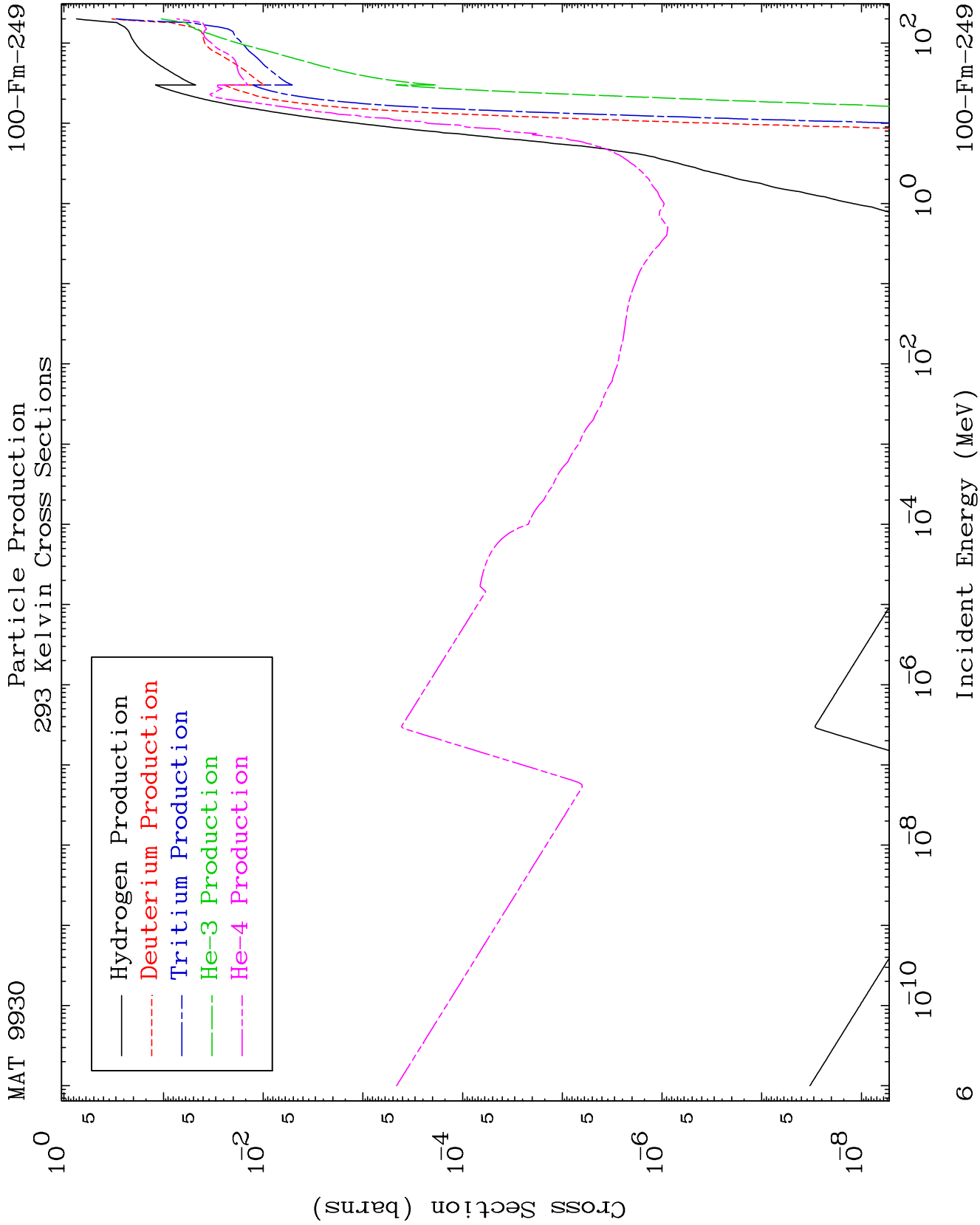


MAT 9930

293 Kelvin Cross Sections

100-Fm-249

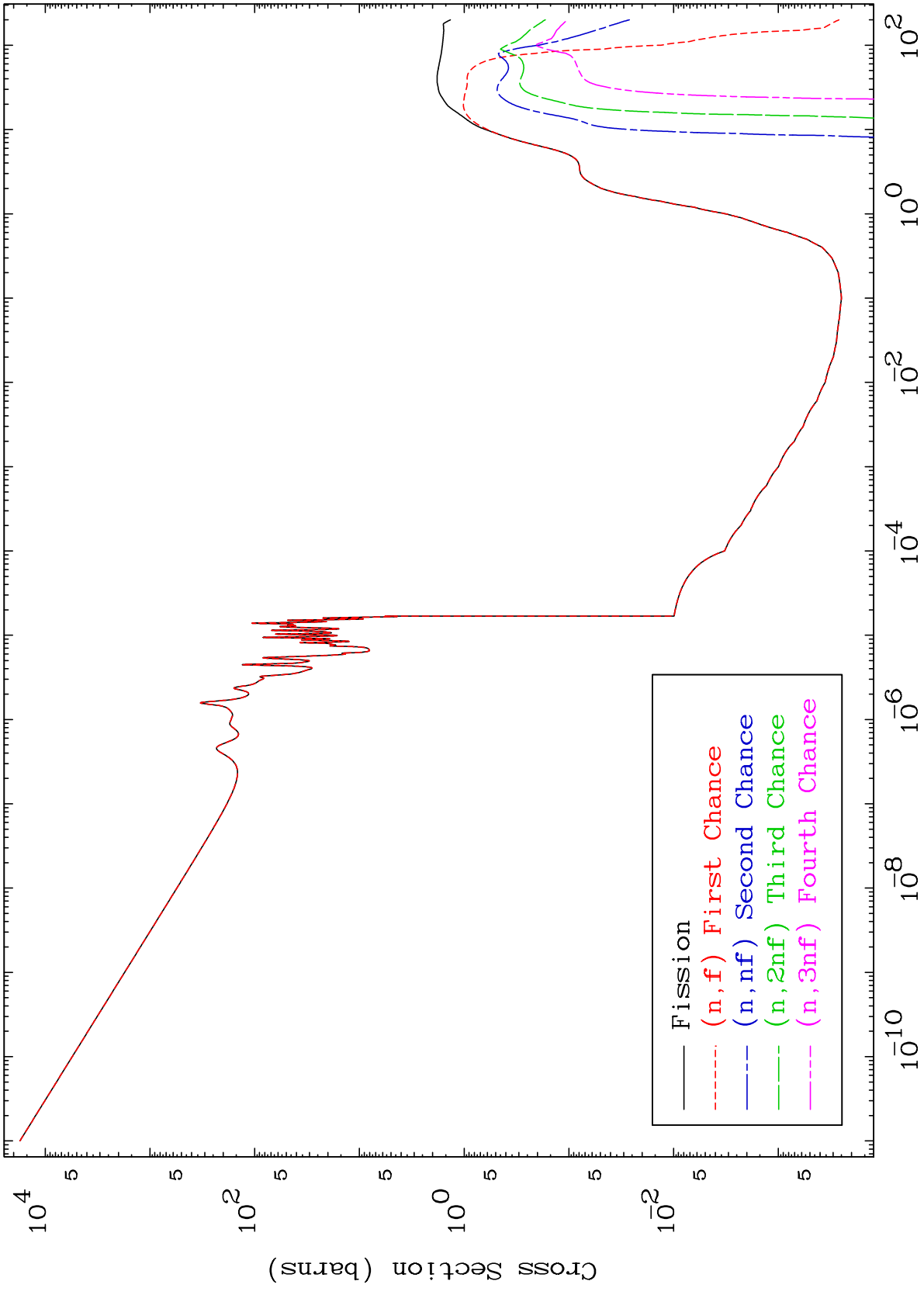




MAT 9930

Fission
293 Kelvin Cross Sections

100-Fm-249

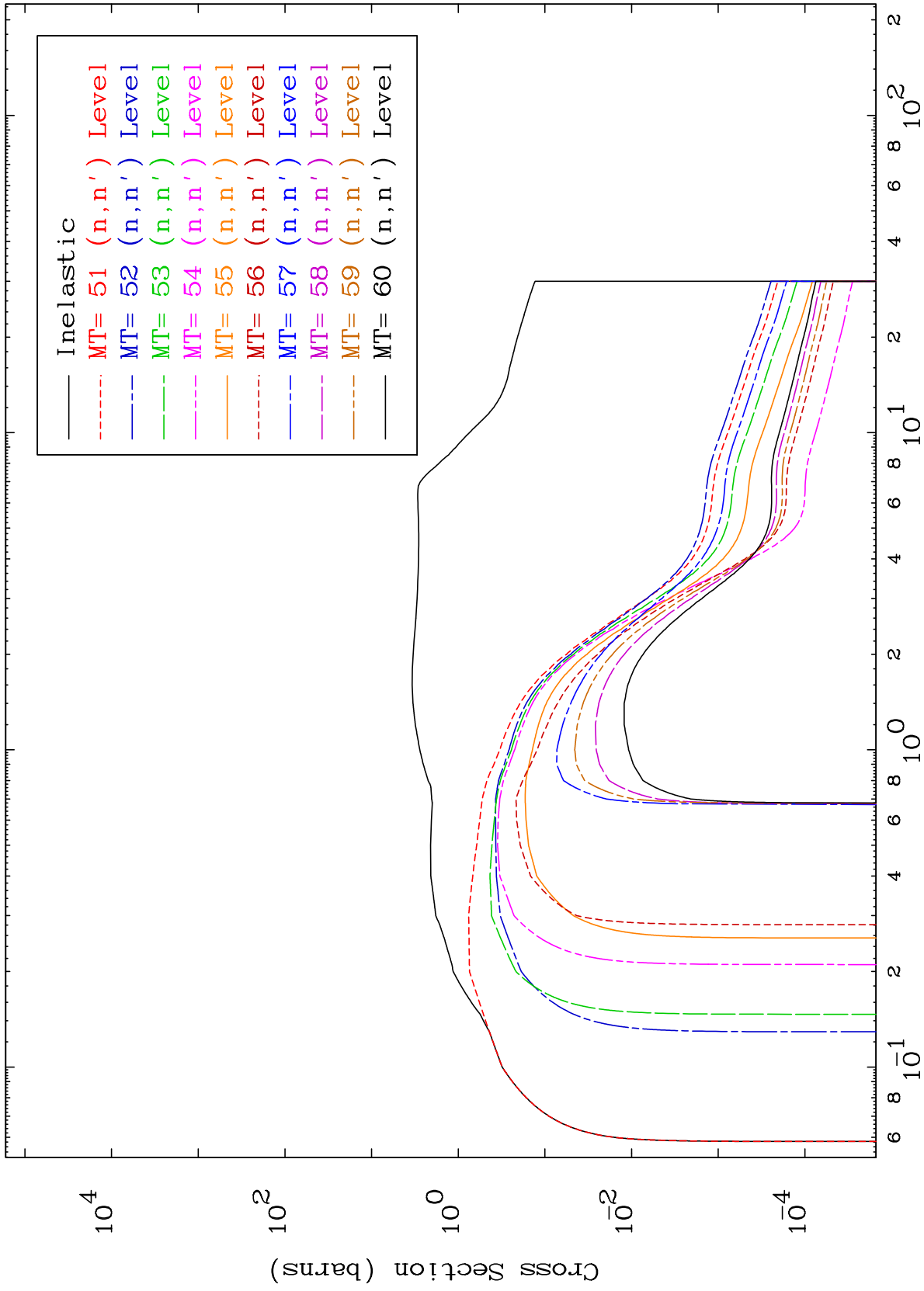


MAT 9930

(n,n') Level

100-Fm-249

293 Kelvin Cross Sections



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Incident Energy (MeV)

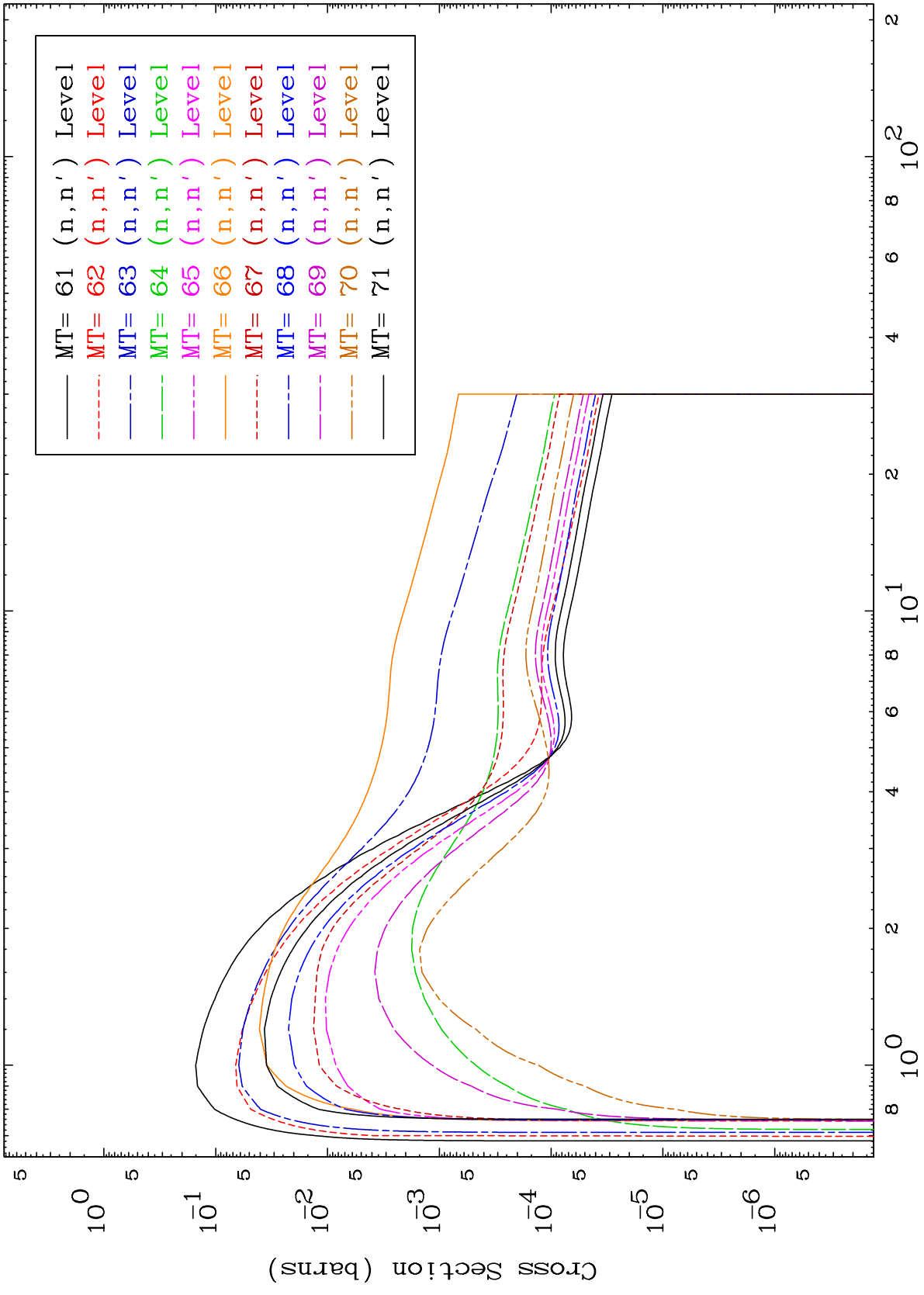
100-Fm-249

MAT 9930

(n,n') Level

100-Fm-249

293 Kelvin Cross Sections



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Incident Energy (MeV)

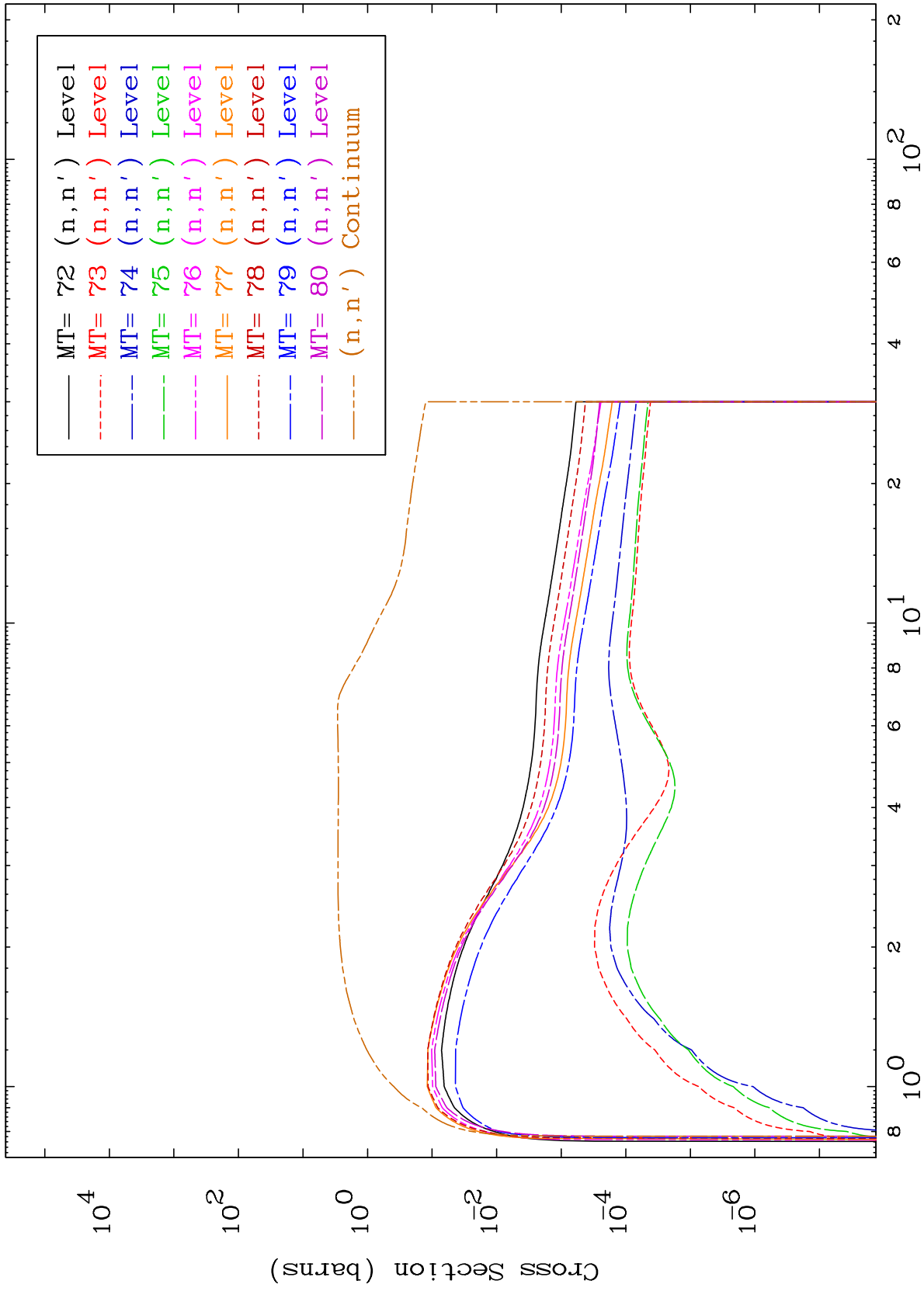
100-Fm-249

MAT 9930

(n,n') Level

100-Fm-249

293 Kelvin Cross Sections



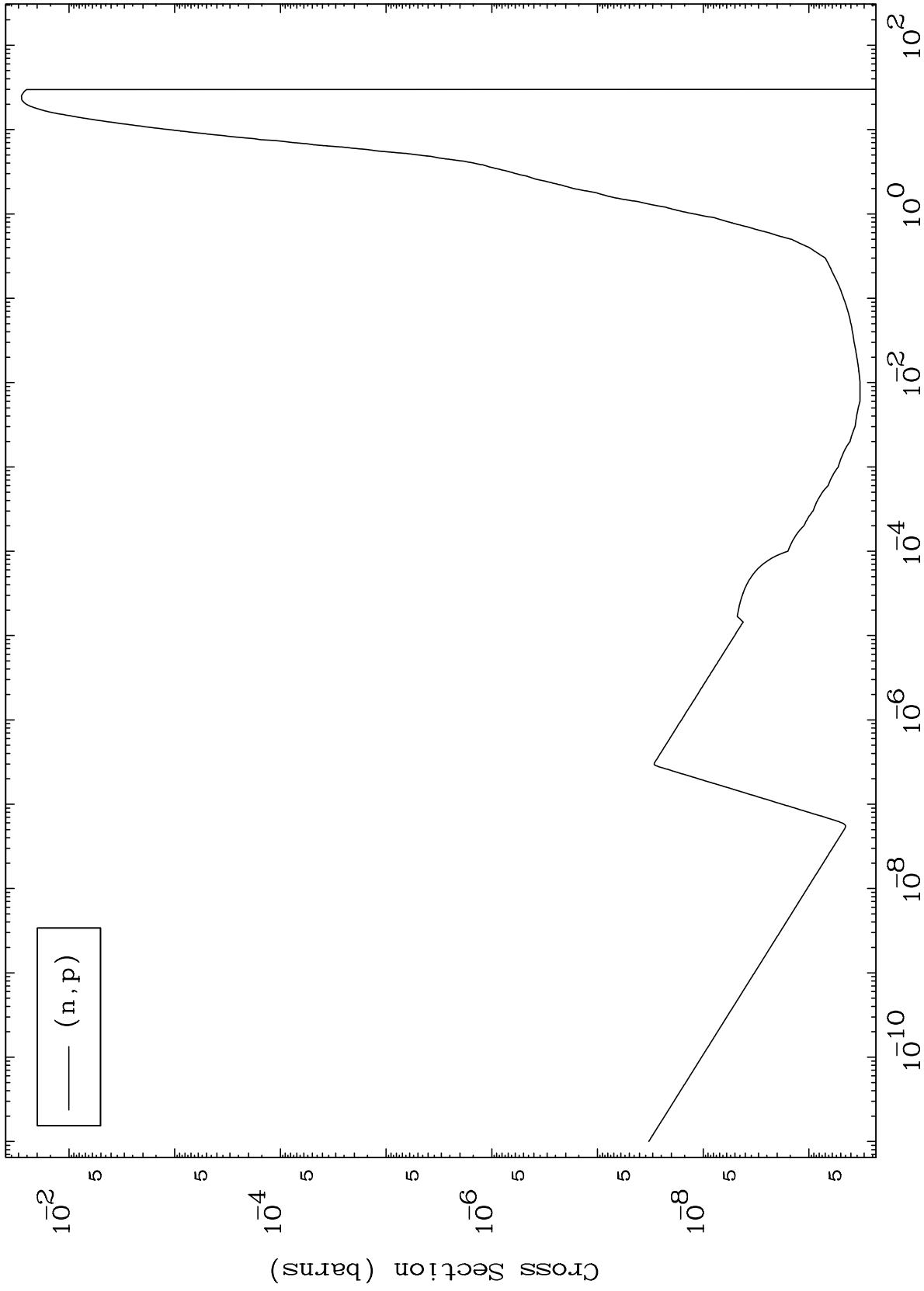
Incident Energy (MeV)

100-Fm-249

MAT 9930

(n,p) Levels
293 Kelvin Cross Sections

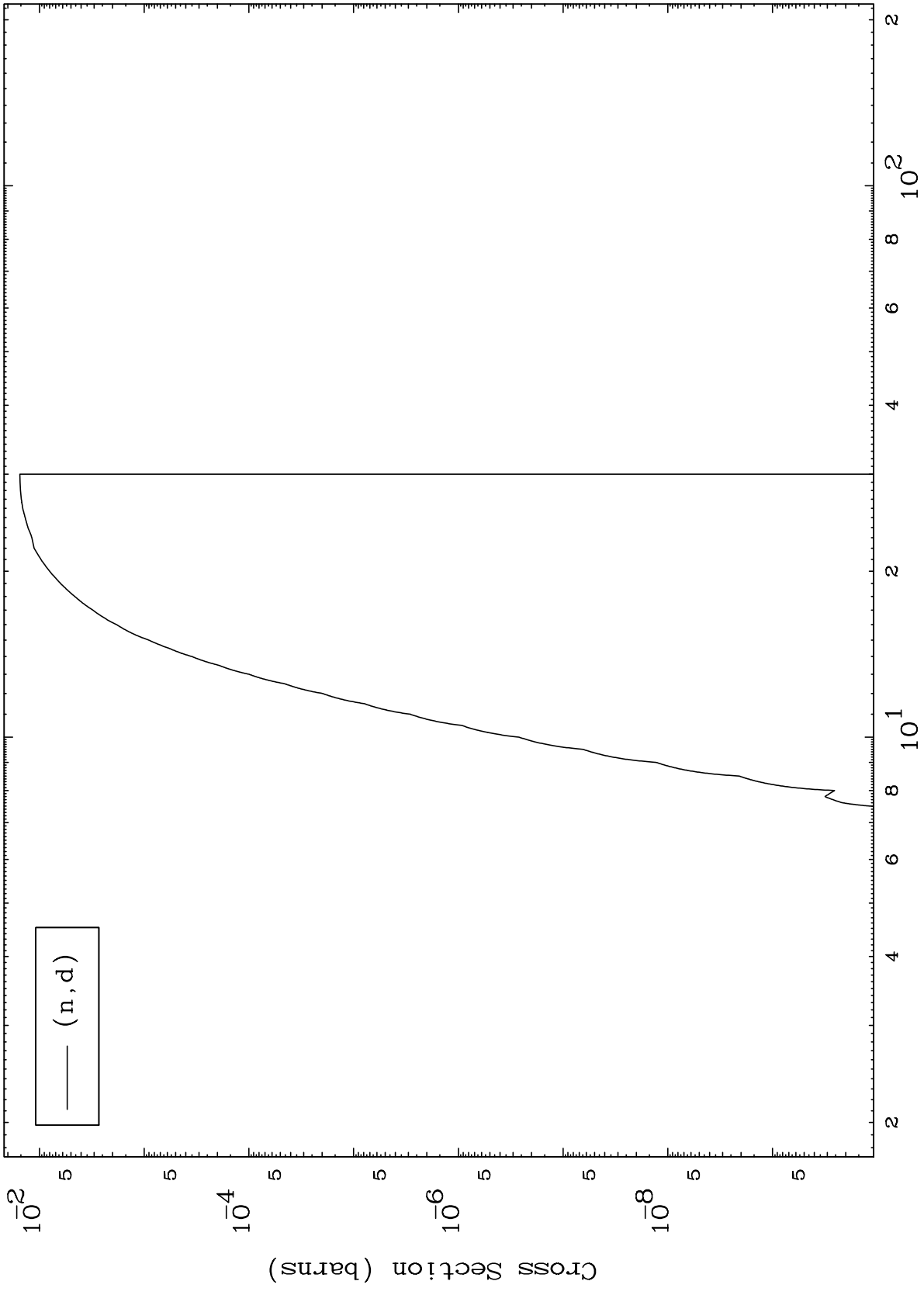
100-Fm-249



MAT 9930

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-249



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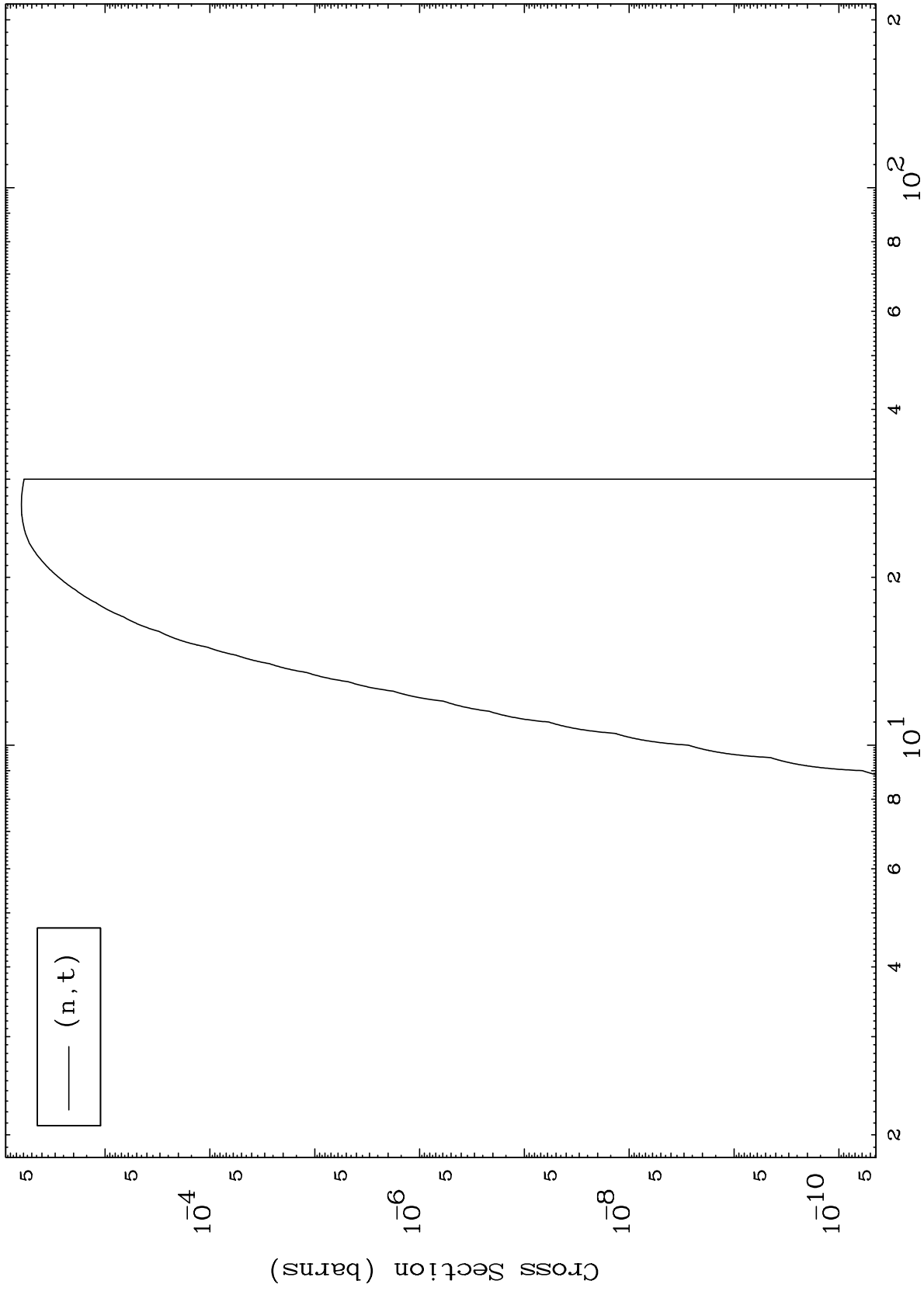
Incident Energy (MeV)

100-Fm-249

MAT 9930

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-249



13

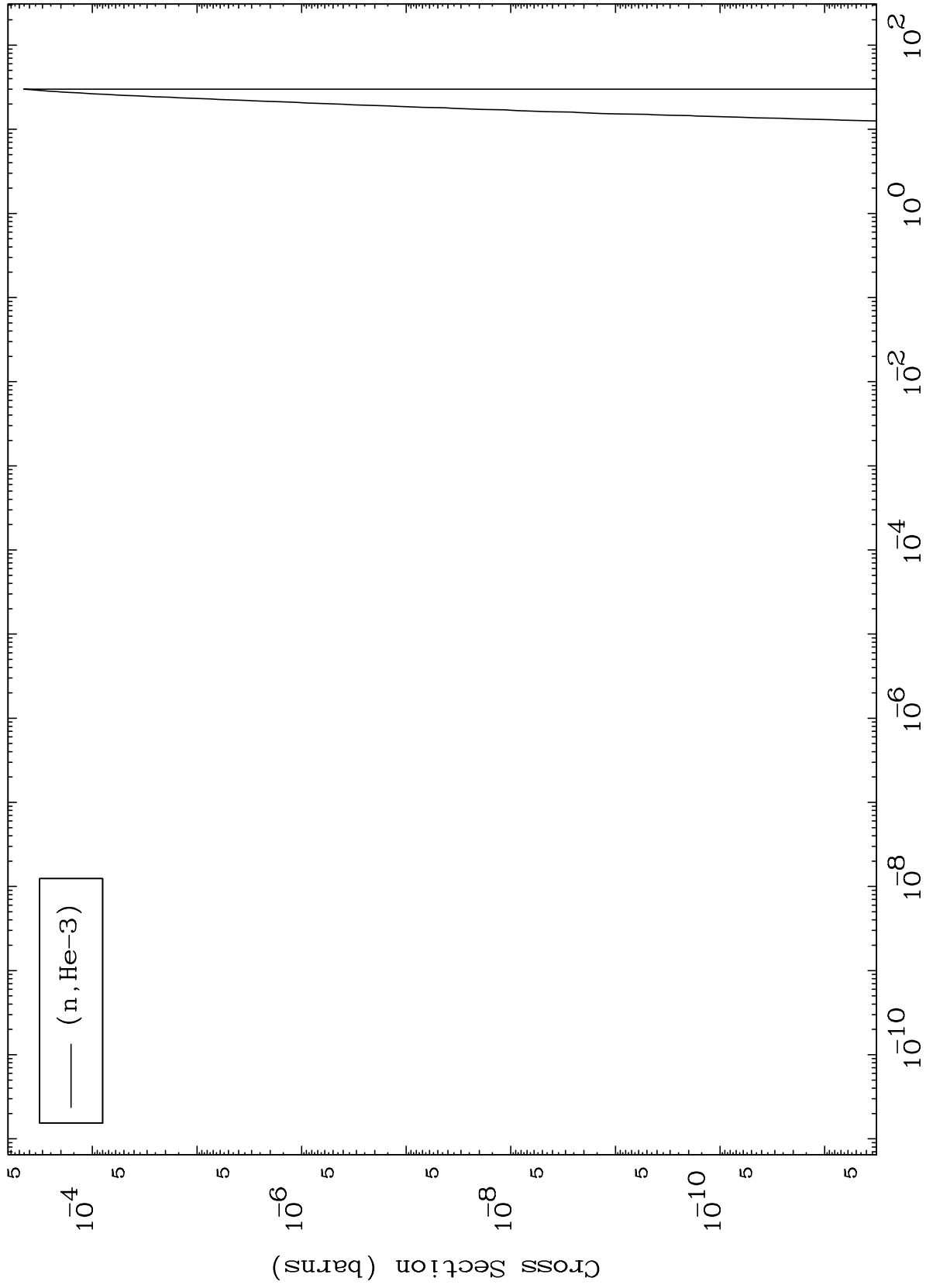
Incident Energy (MeV)

100-Fm-249

MAT 9930

(n,He3) Levels
293 Kelvin Cross Sections

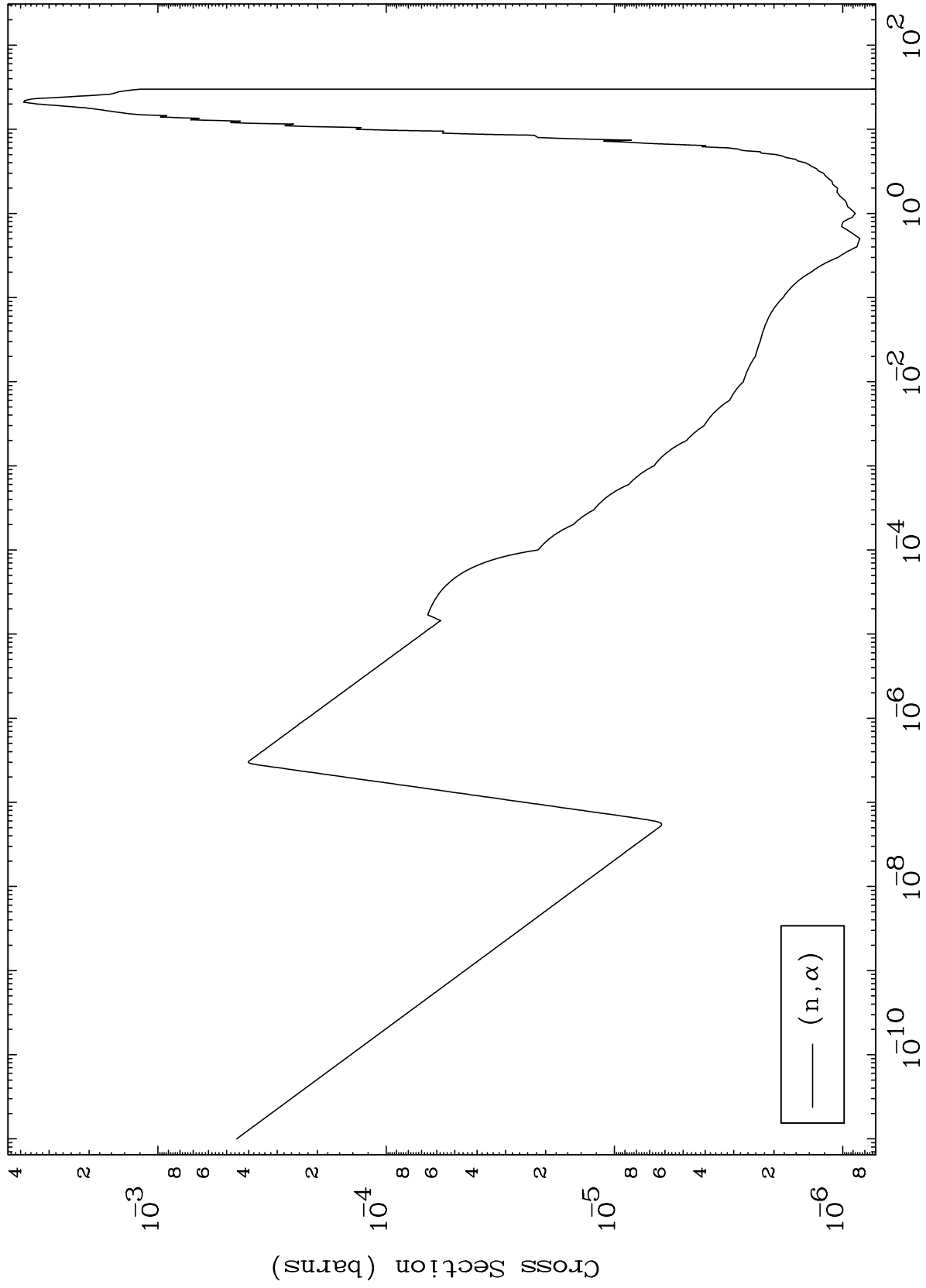
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MAT 9930

(n,α) Levels
293 Kelvin Cross Sections

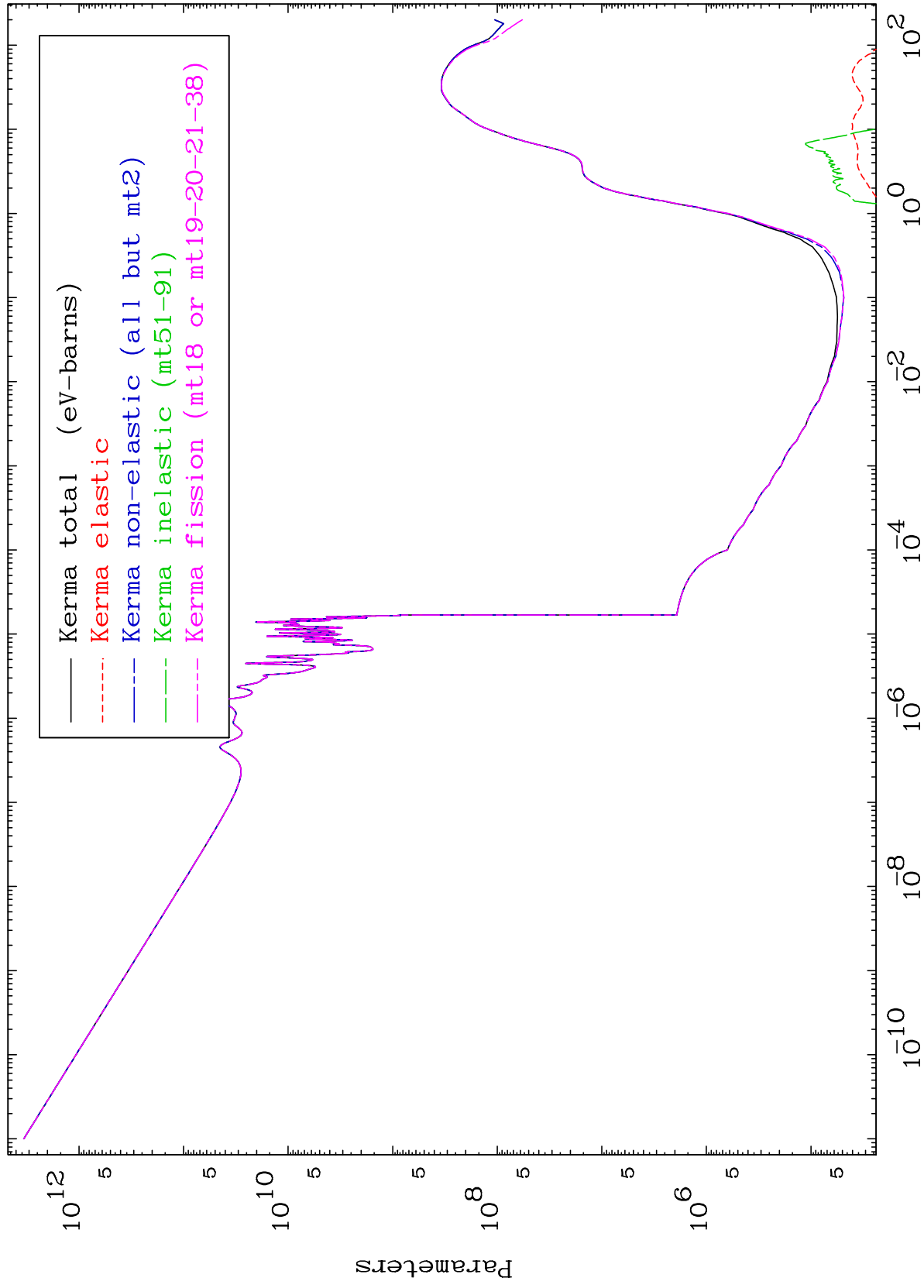
100-Fm-249



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Incident Energy (MeV)

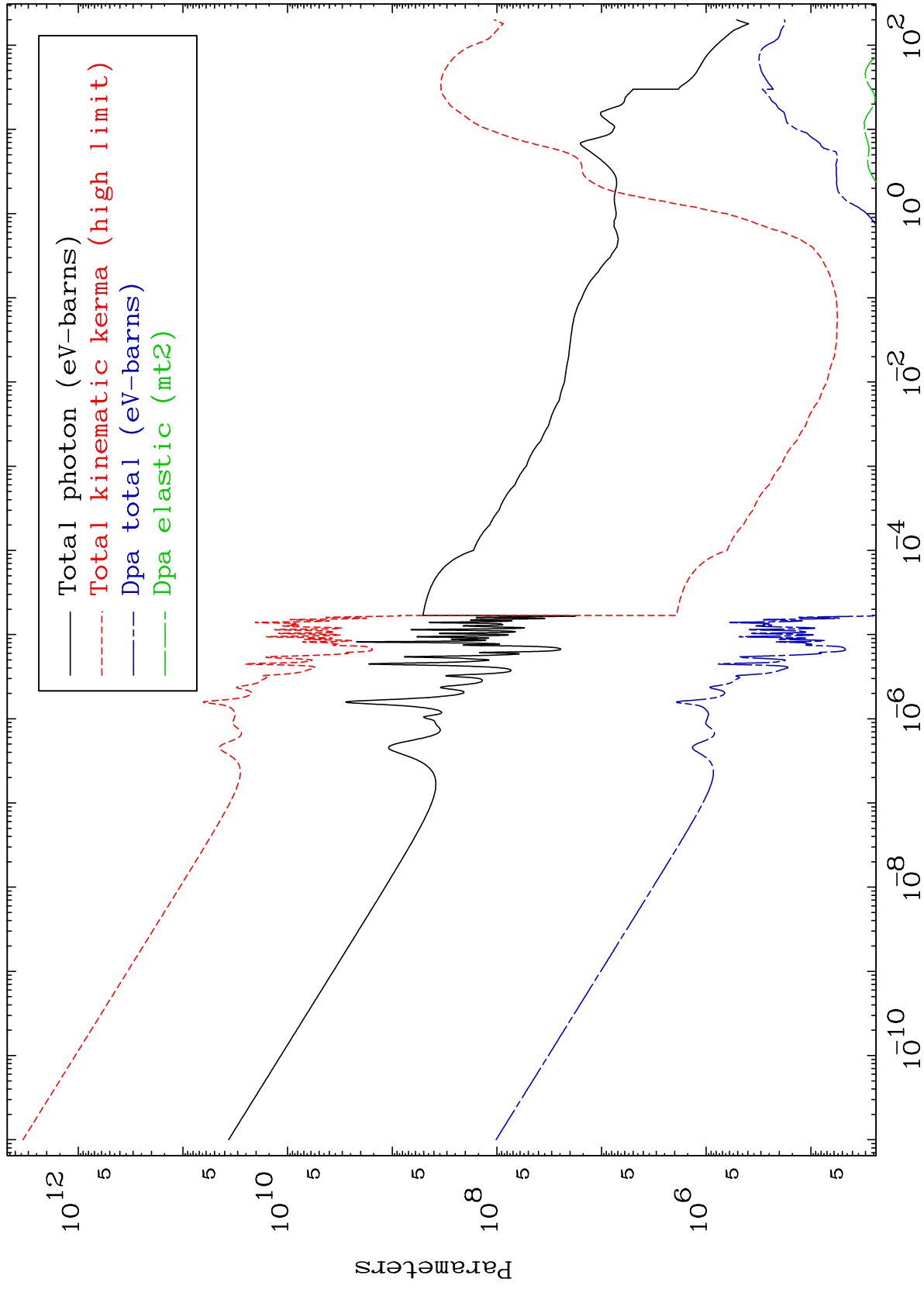
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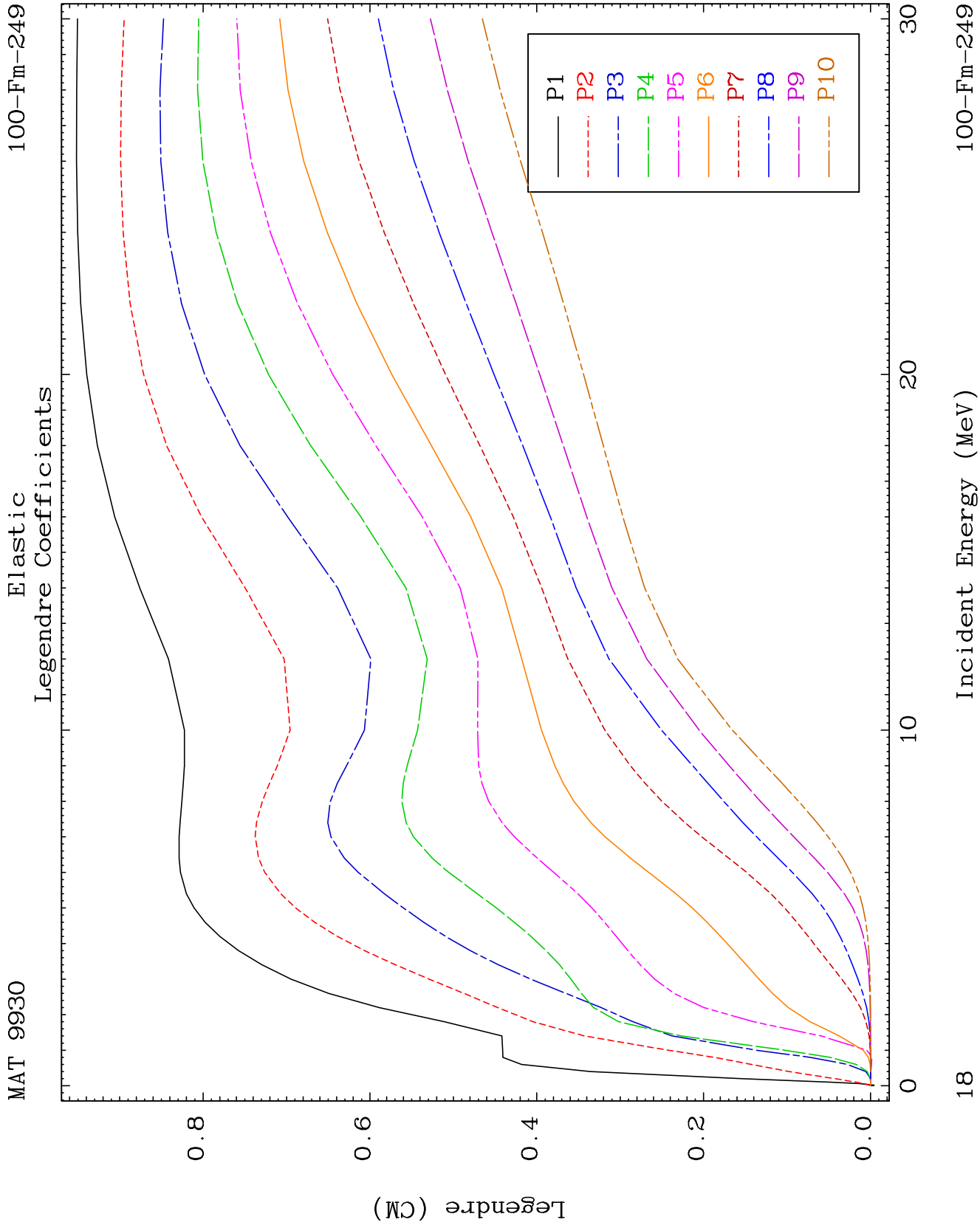


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Energy Release
Parameters

100-Fm-249

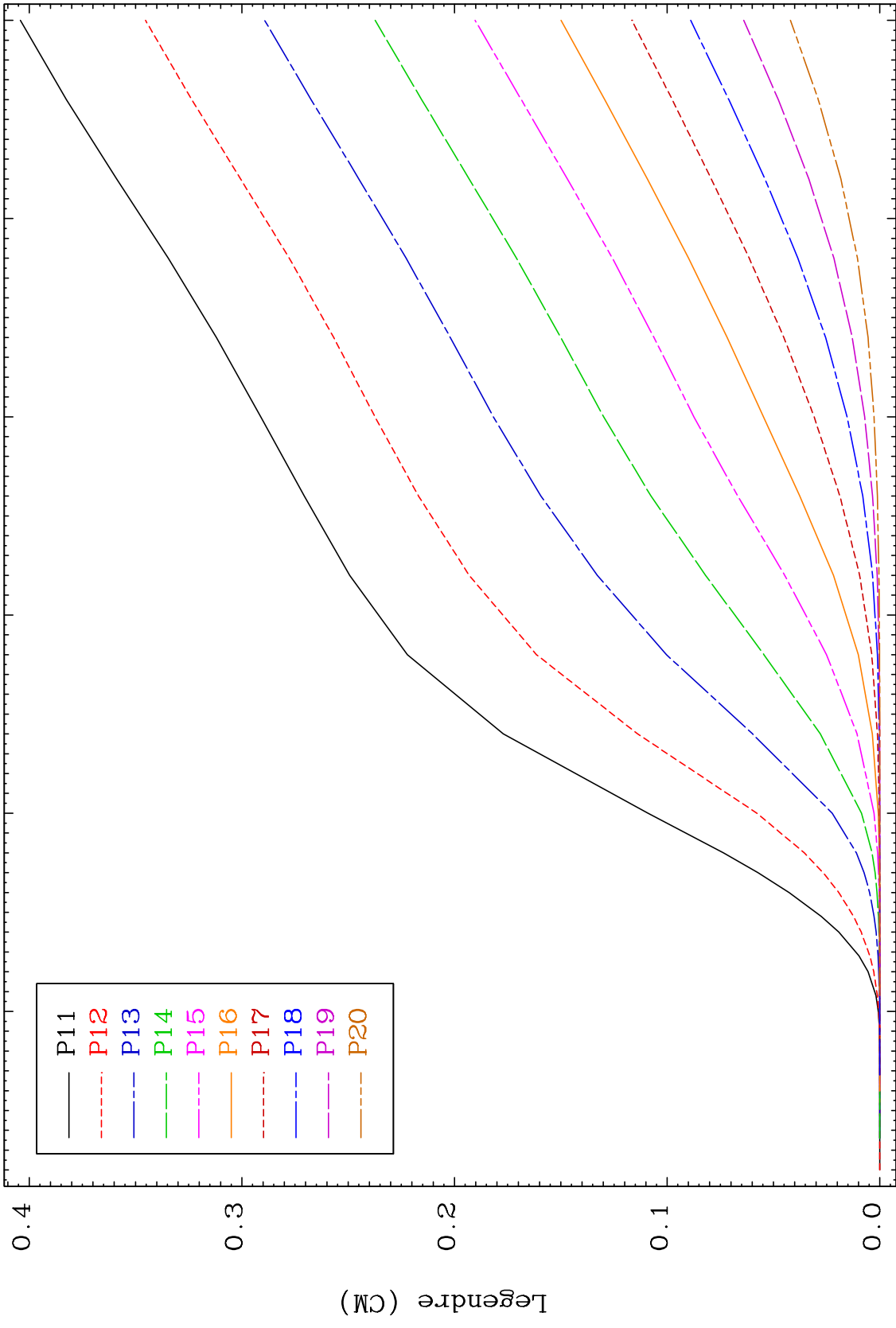




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Elastic Legendre Coefficients

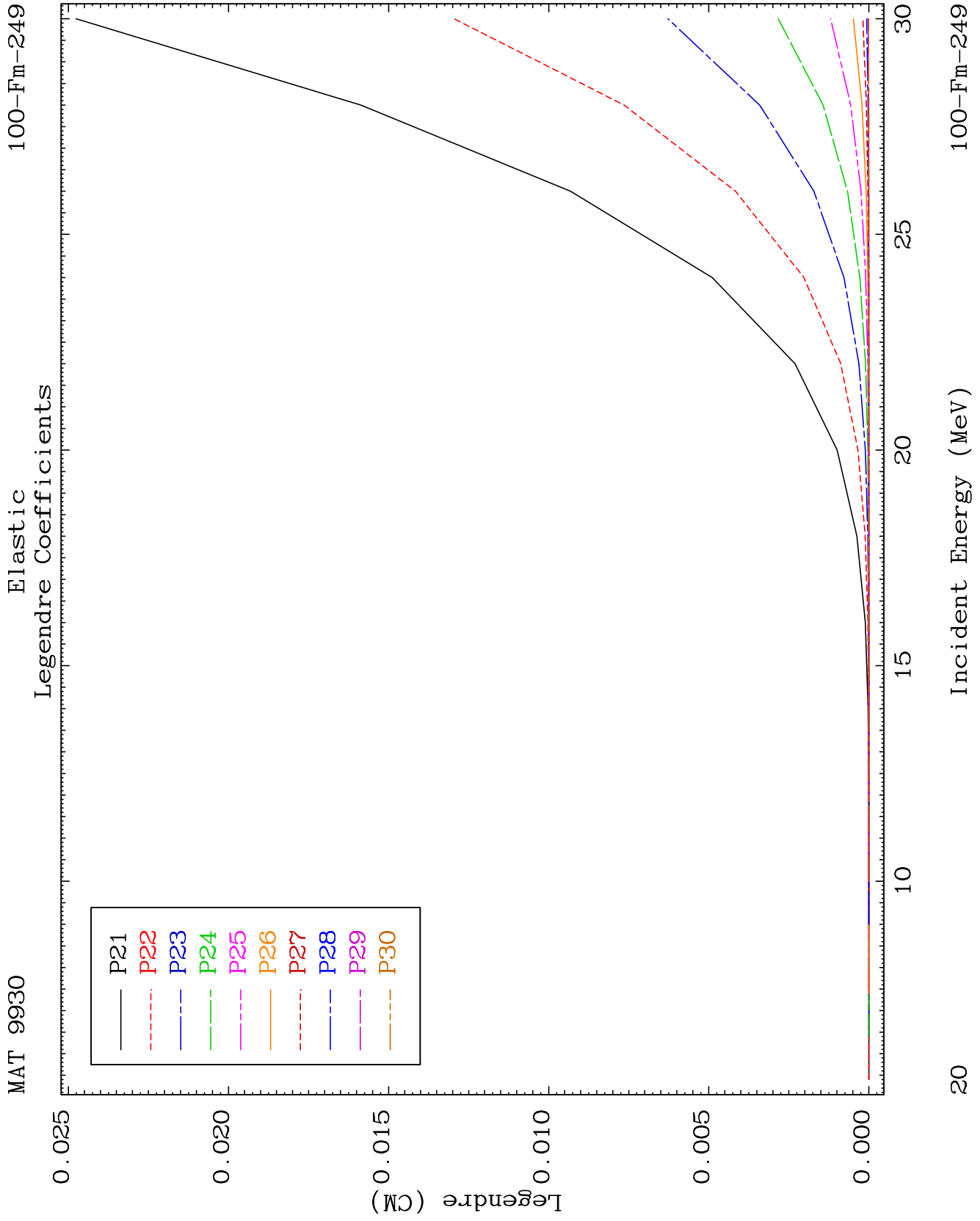
100-Fm-249



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100-Fm-249

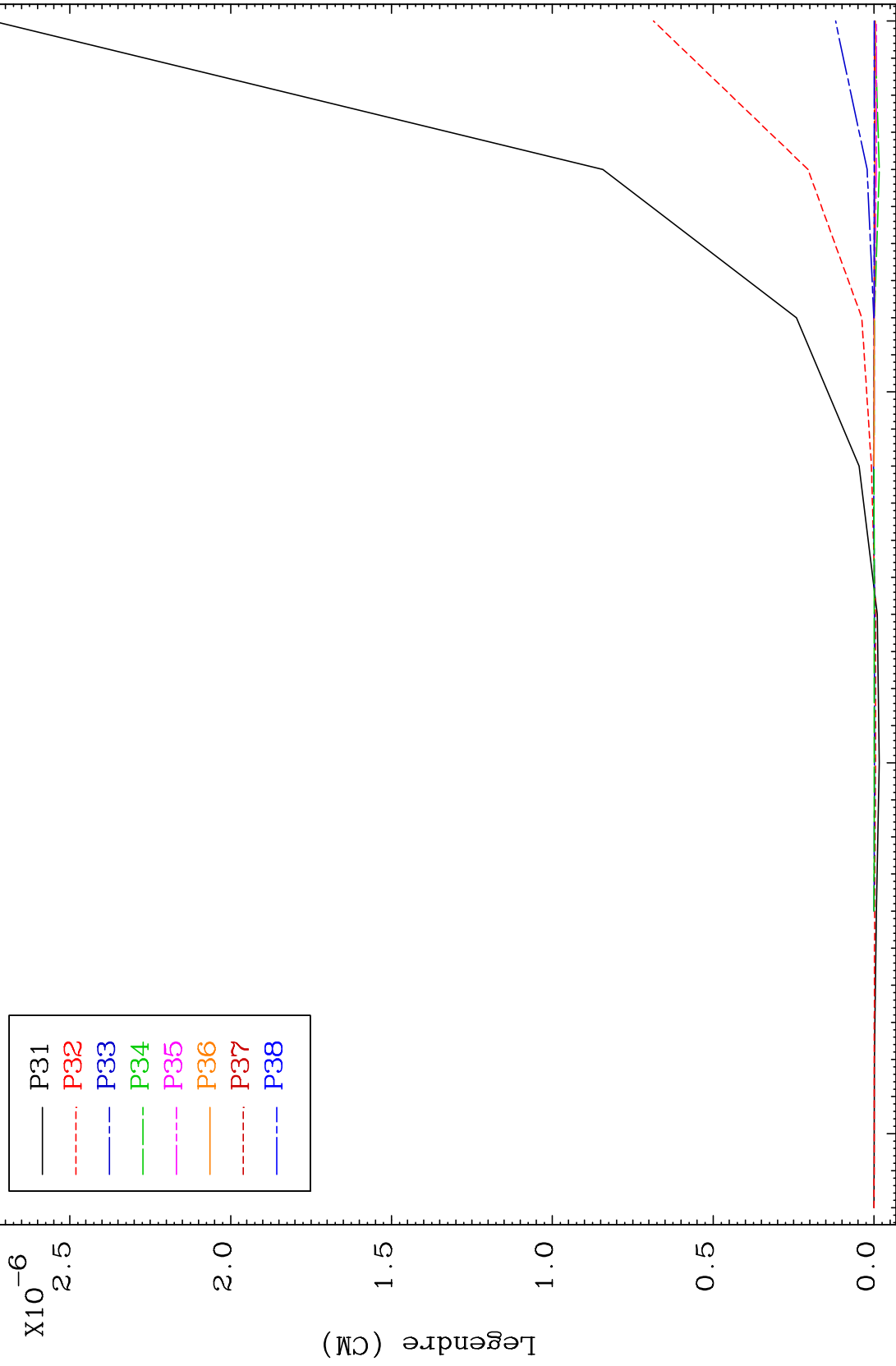
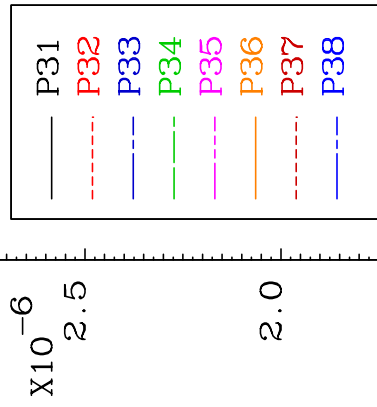
Incident Energy (MeV)



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Elastic Legendre Coefficients

100-Fm-249



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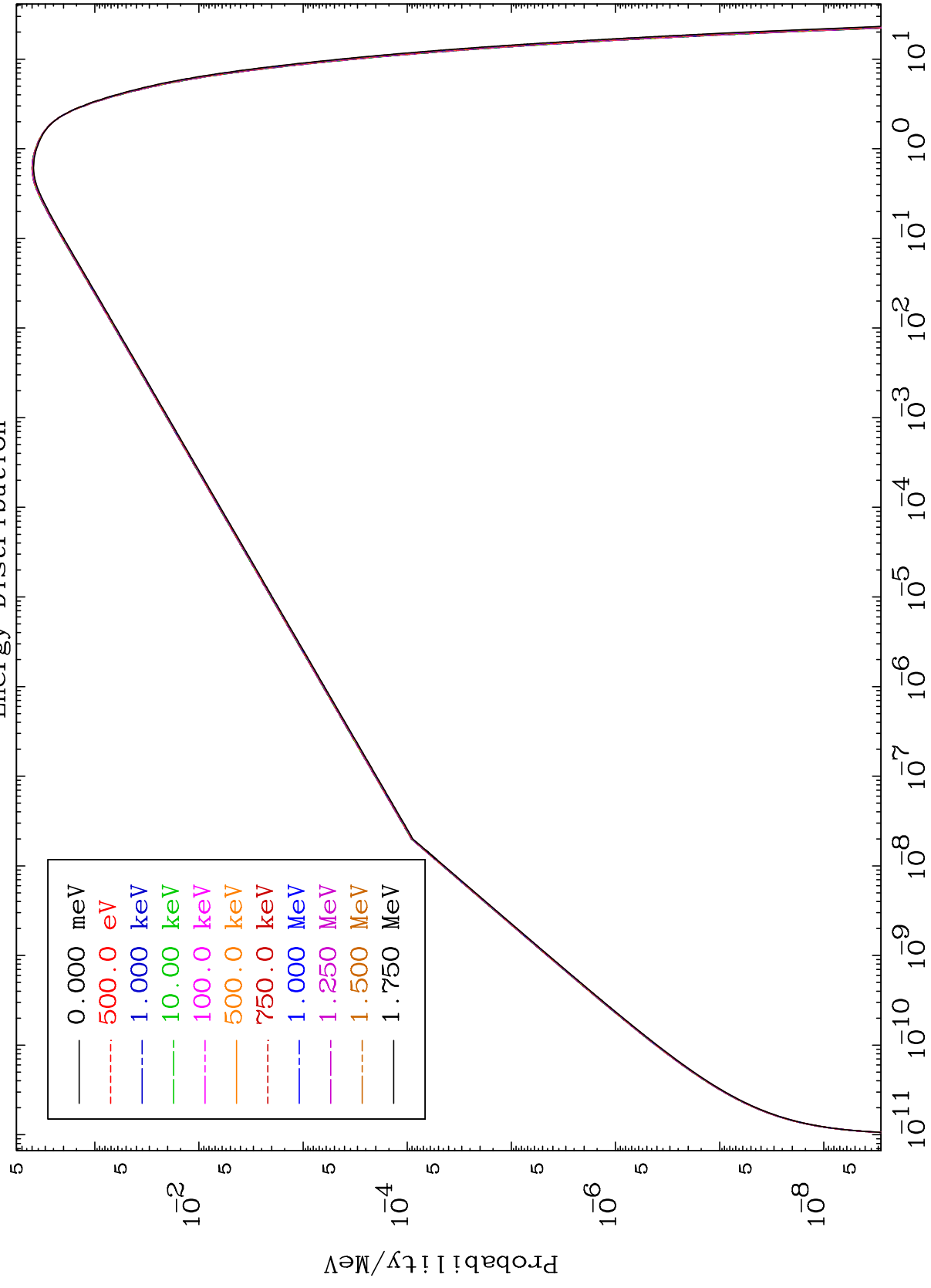
Incident Energy (MeV)

100-Fm-249

MAT 9930

Fission Energy Distribution

100-Fm-249

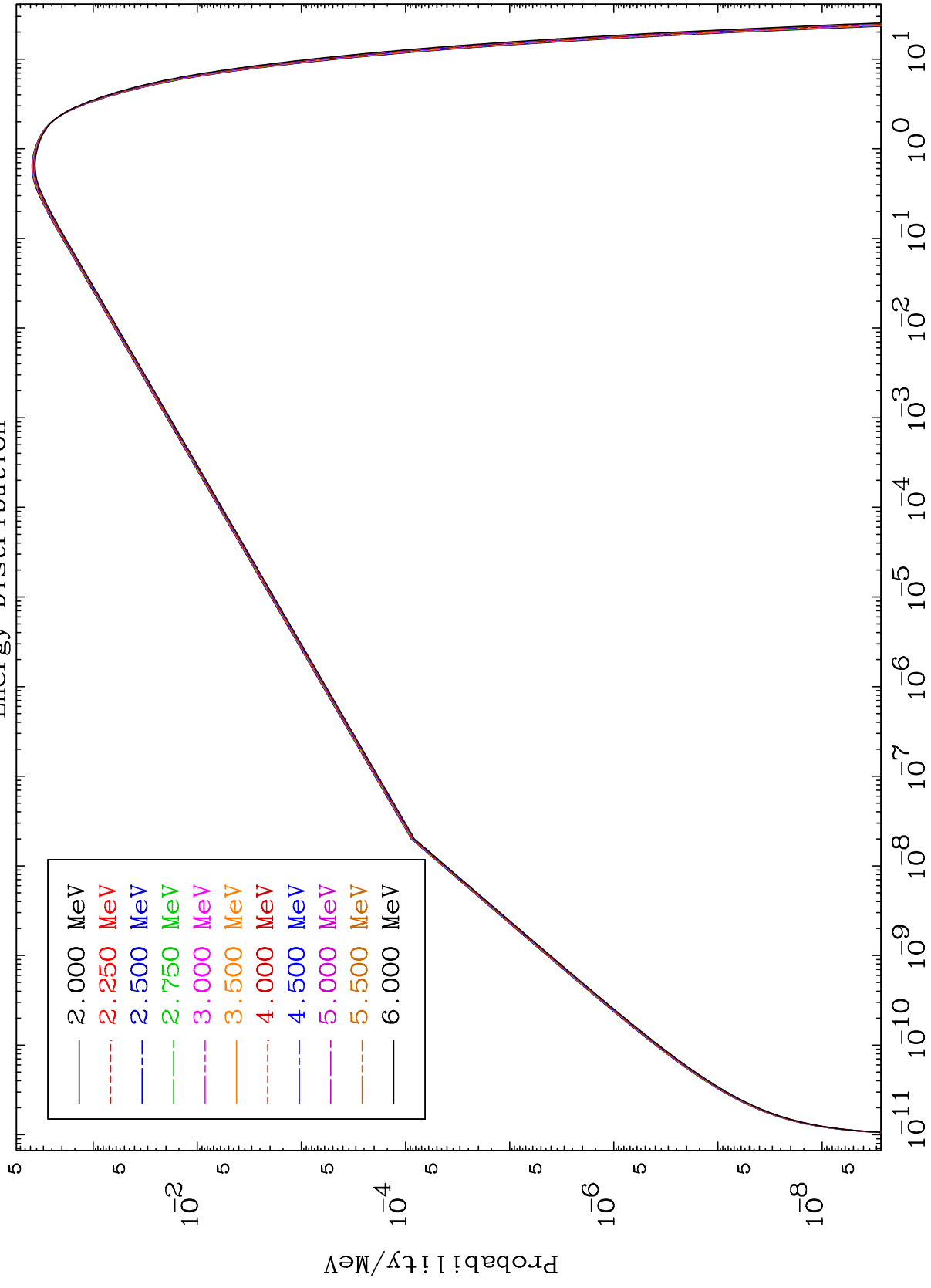


100-Fm-249

MAT 9930

Fission Energy Distribution

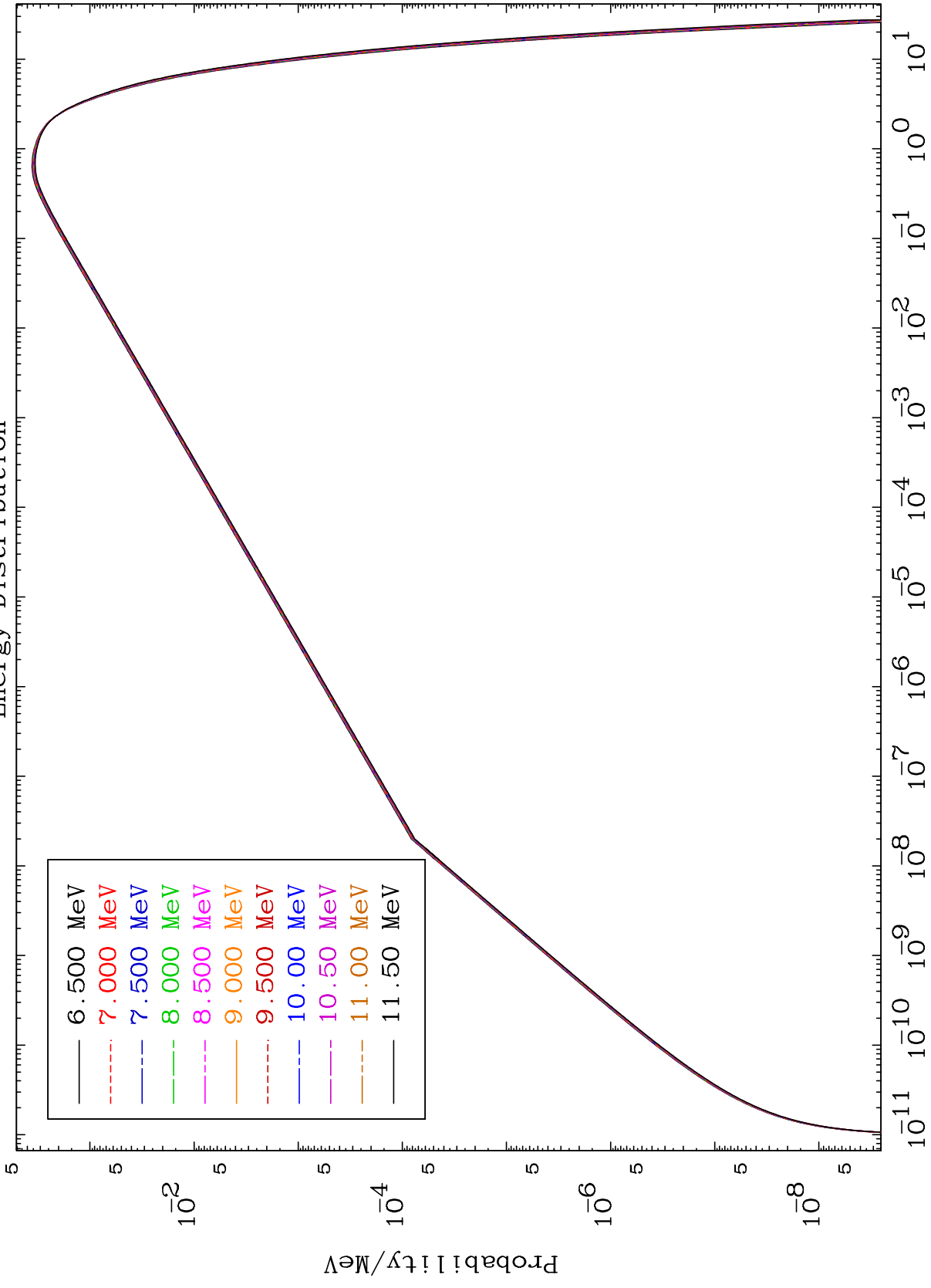
100-Fm-249



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Fission Energy Distribution

100-Fm-249

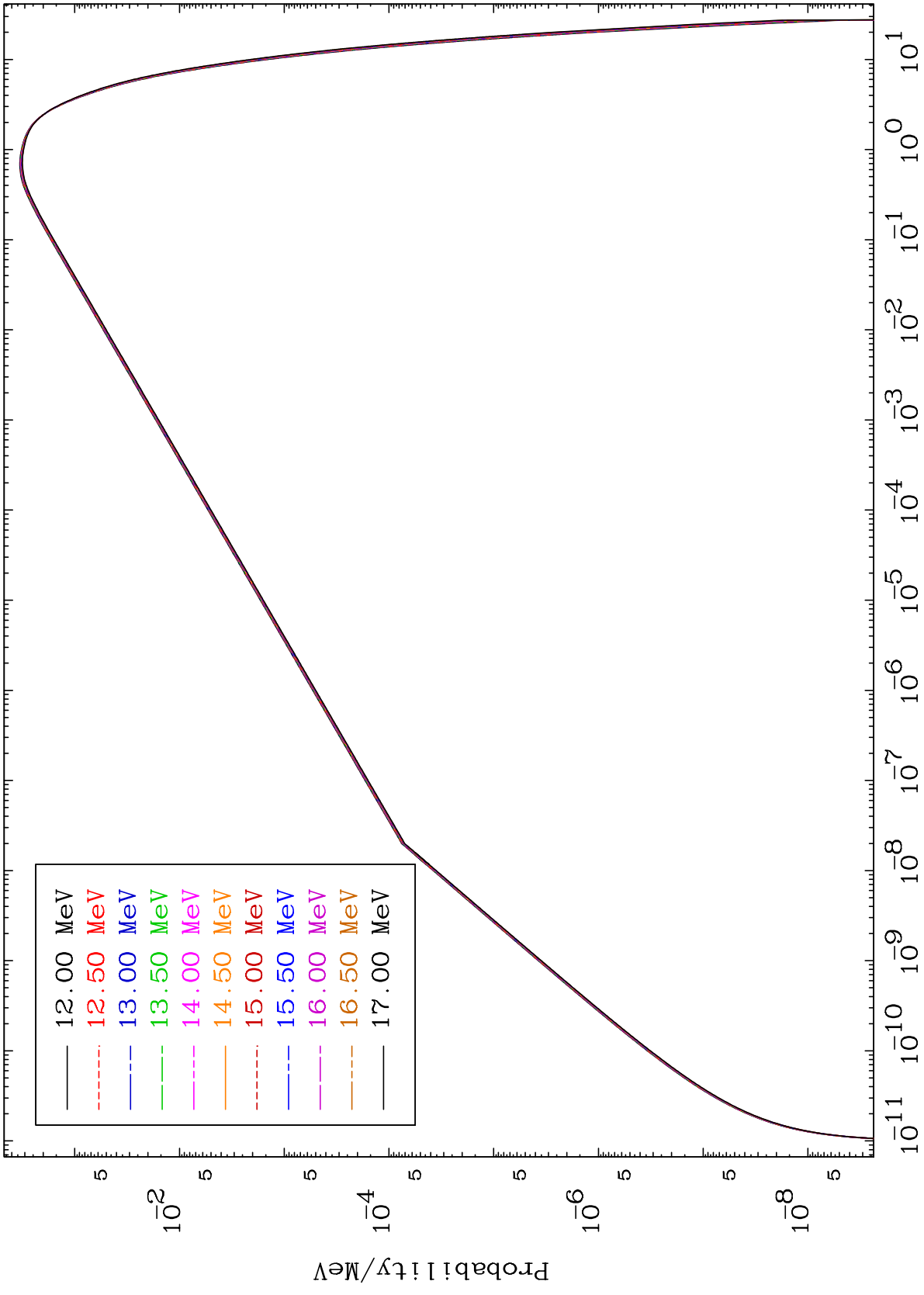


100-Fm-249

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Fission Energy Distribution

100-Fm-249

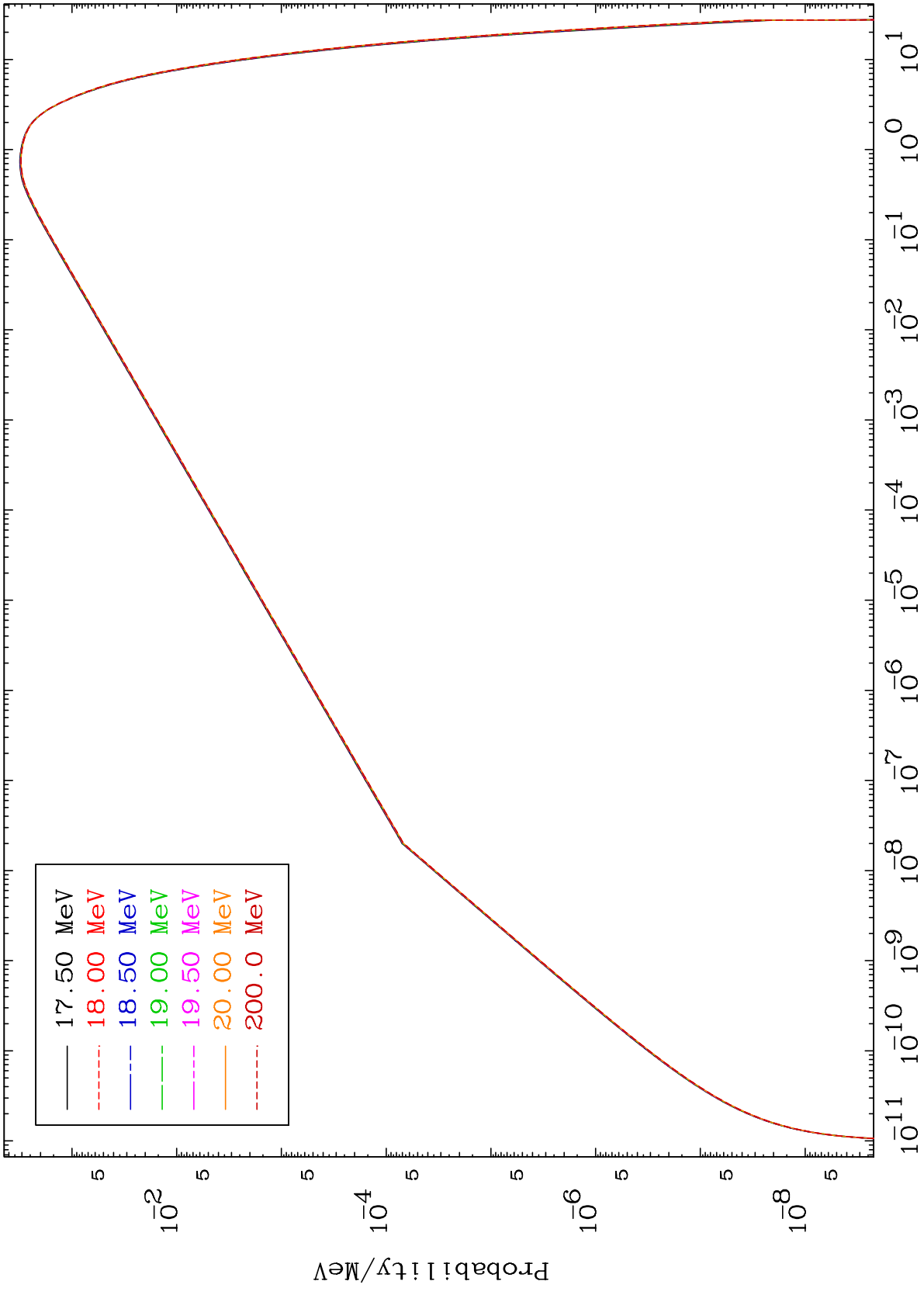


100-Fm-249

MAT 9930

Fission Energy Distribution

100-Fm-249



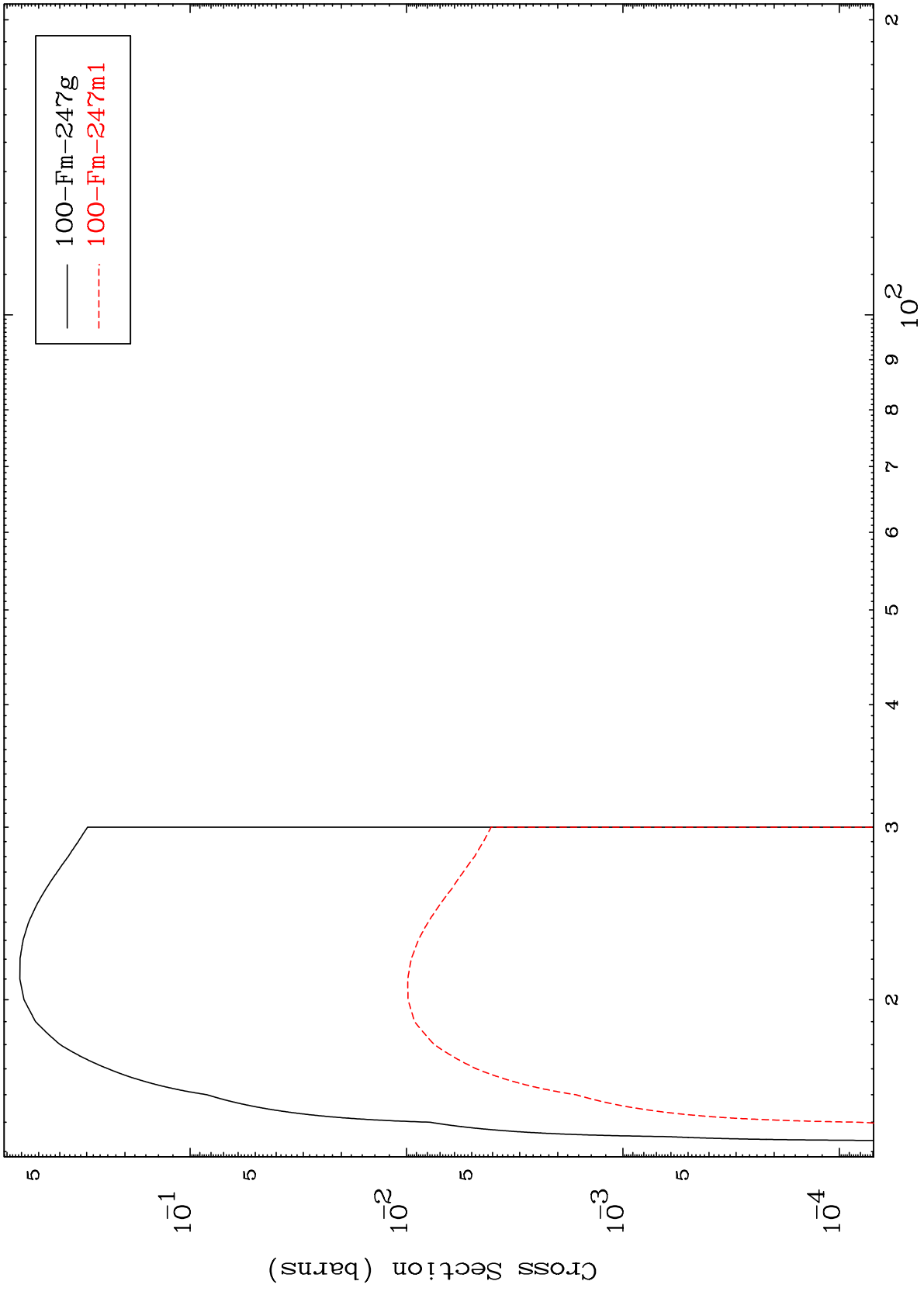
100-Fm-249

MAT 9930

(n,3n)

100-Fm-249

Radionuclide Production Cross Section



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Incident Energy (MeV)

100-Fm-249

MAT 9930

100-Fm-249

Radionuclide Production Cross Section
(n, γ)

