

Program EVALPLOT
(Version 2018-1)

by

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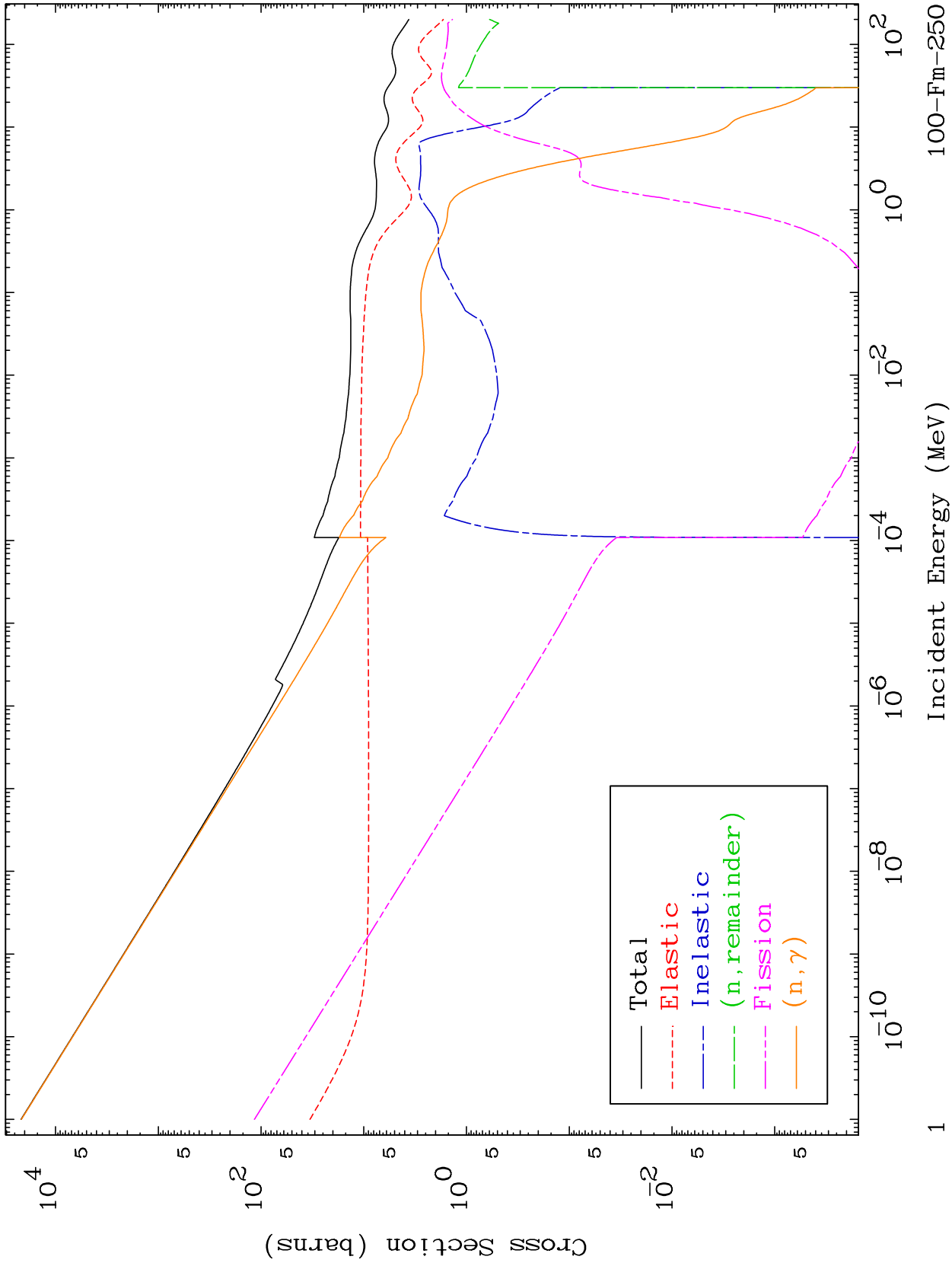
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9936

Major
293 Kelvin Cross Sections

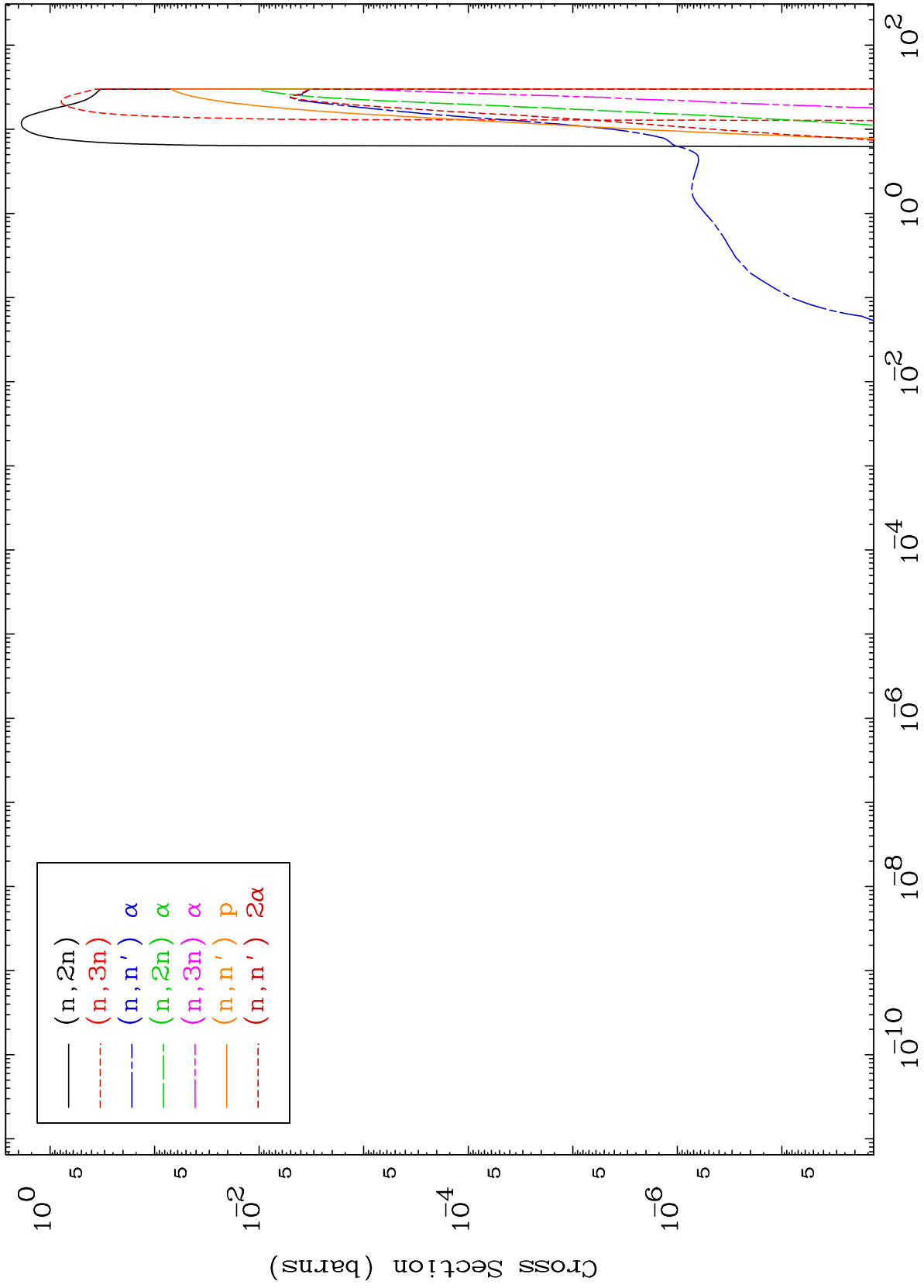
100-Fm-250



MAT 9936

Neutron Production
293 Kelvin Cross Sections

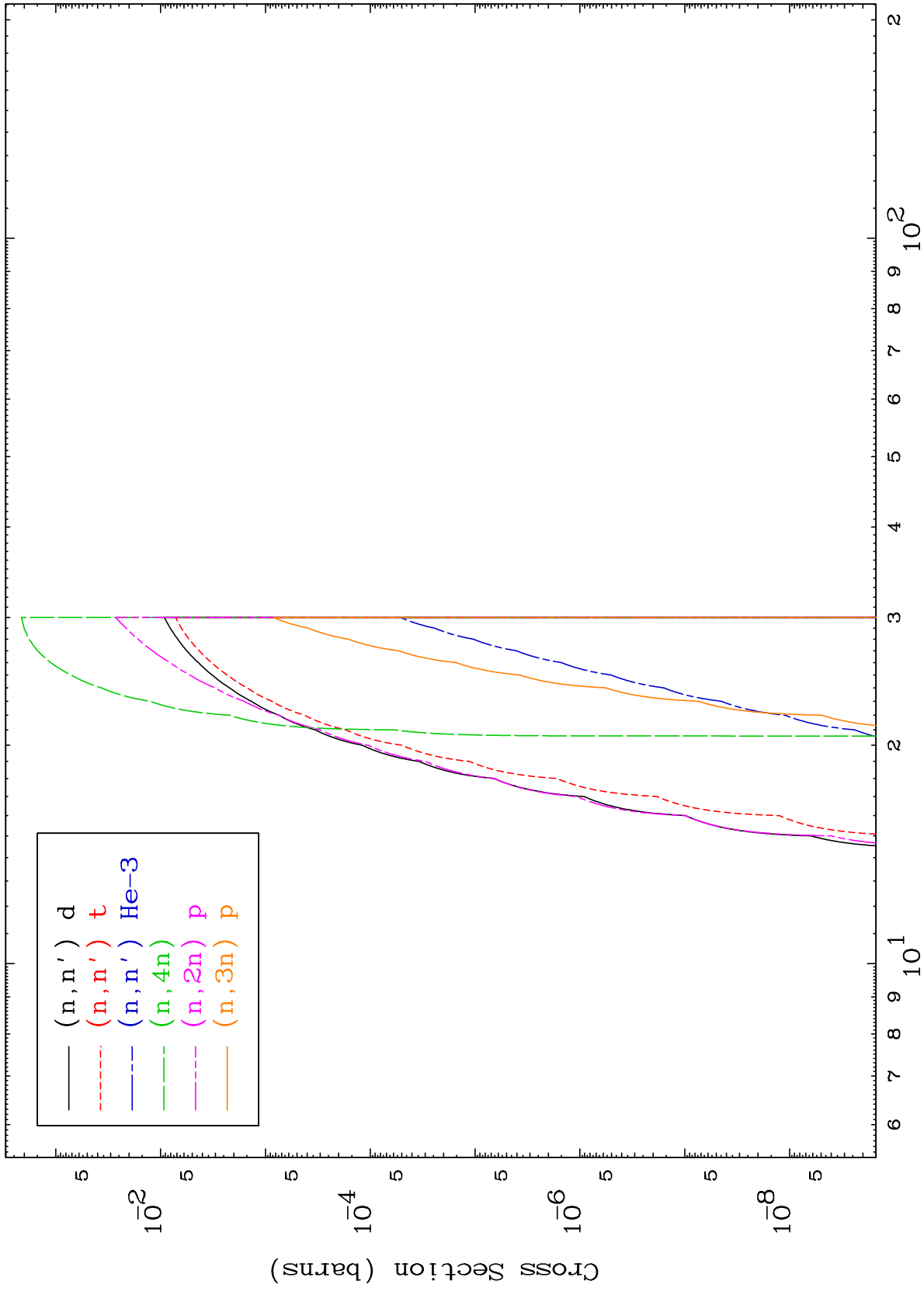
100-Fm-250



MAT 9936

Neutron Production
293 Kelvin Cross Sections

100-Fm-250



3

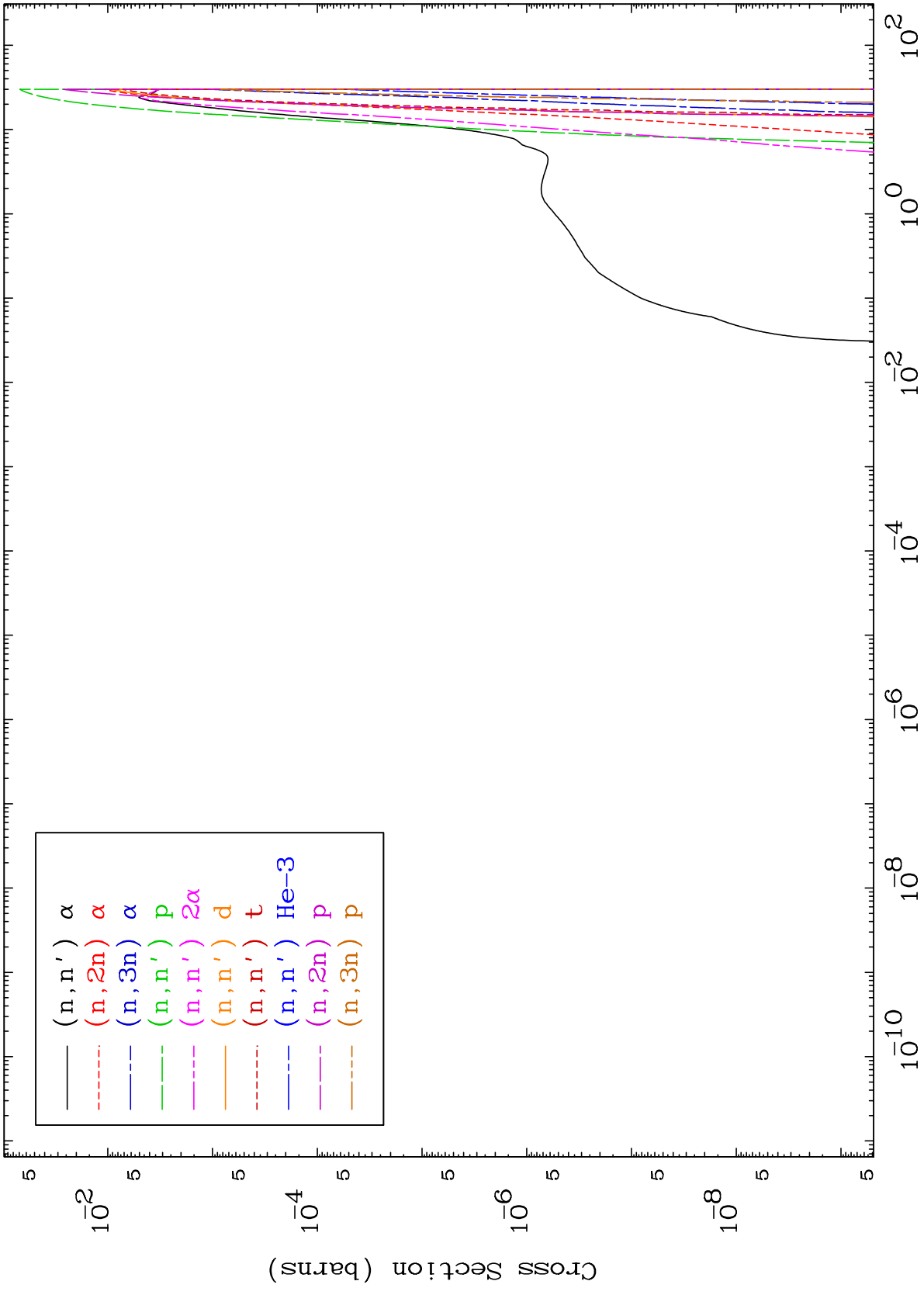
Incident Energy (MeV)

100-Fm-250

MAT 9936

Charged Particle
293 Kelvin Cross Sections

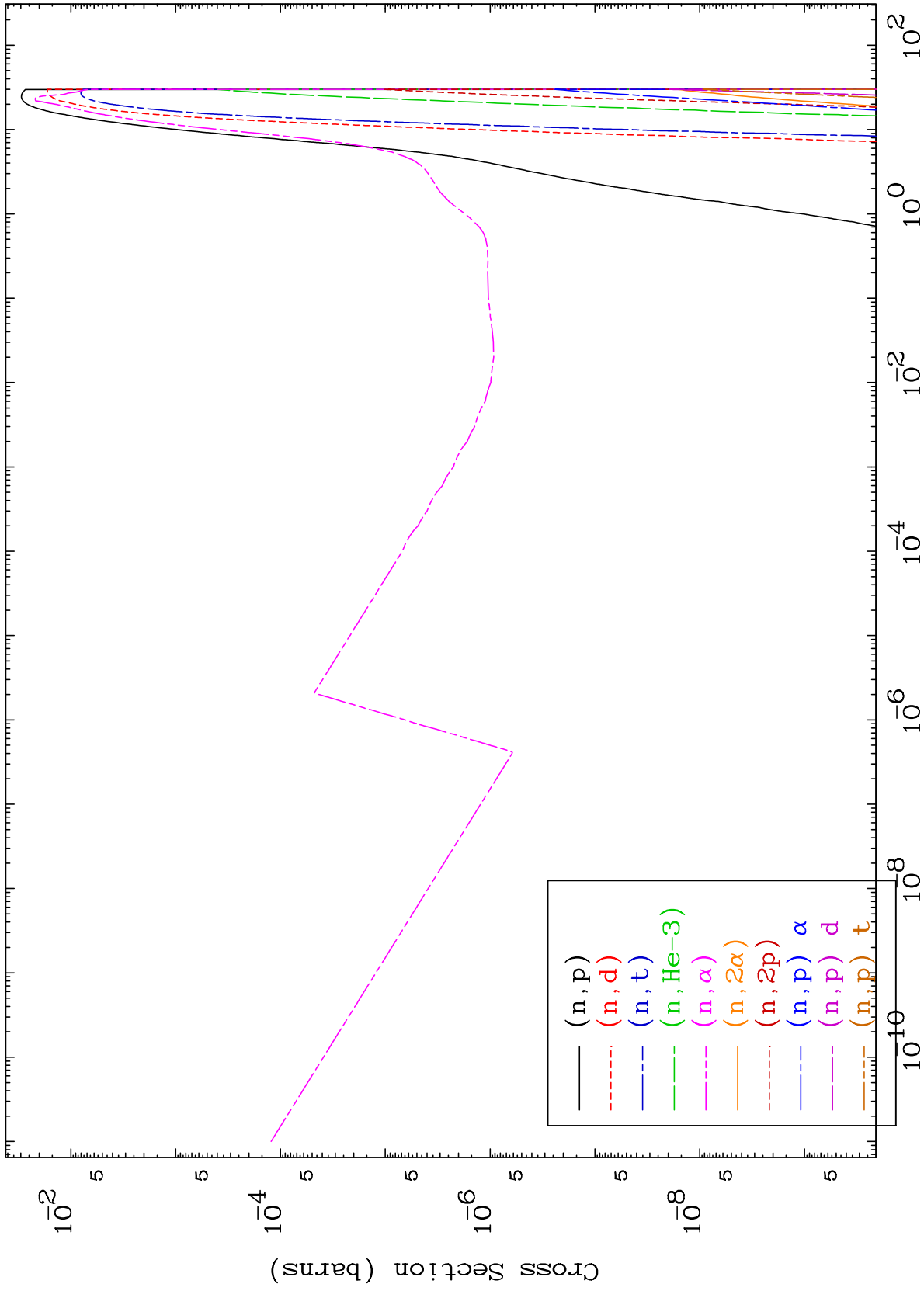
100-Fm-250



MAT 9936

Charged Particle
293 Kelvin Cross Sections

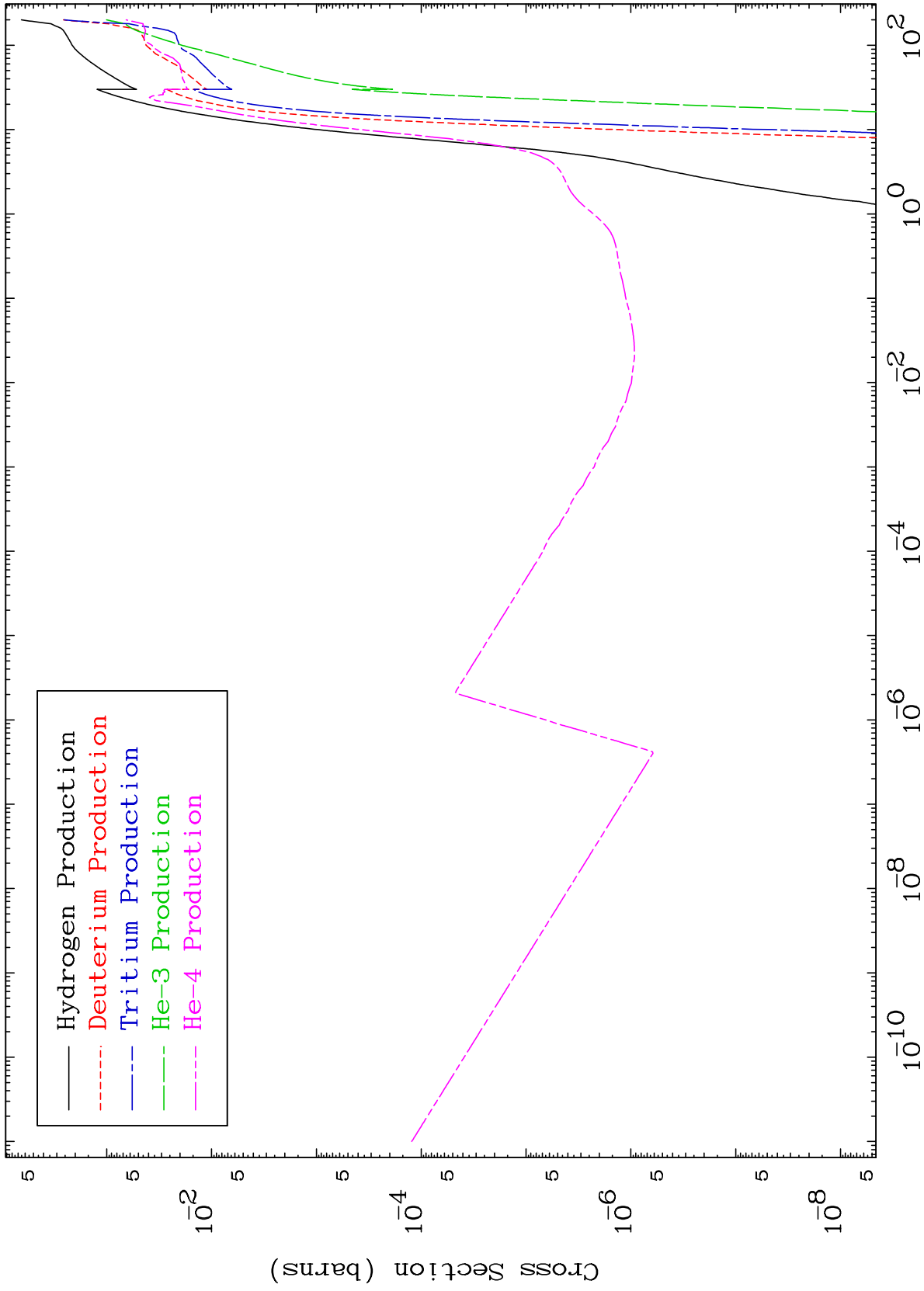
100-Fm-250



MAT 9936

Particle Production
293 Kelvin Cross Sections

100-Fm-250



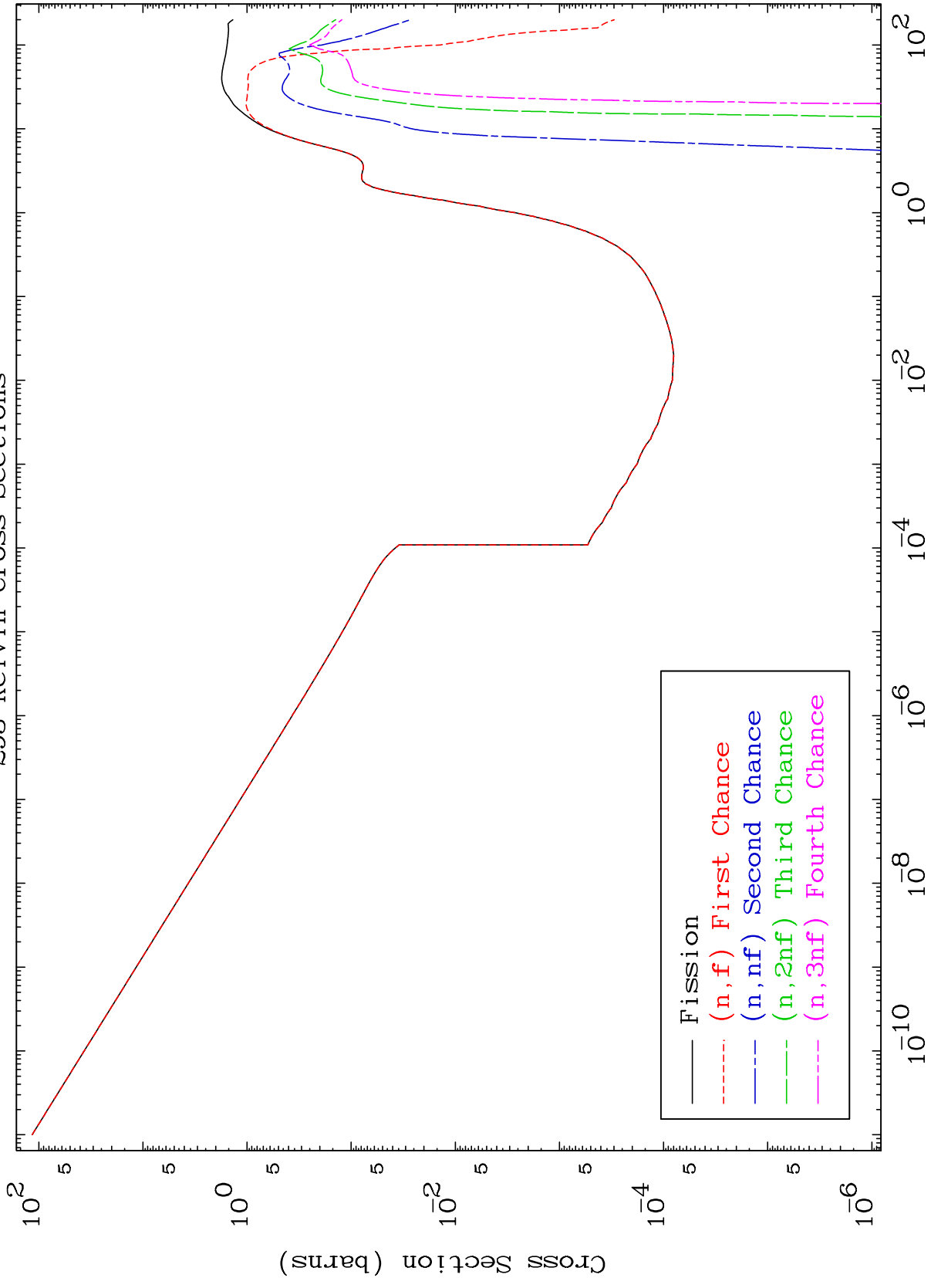
6

100-Fm-250

MAT 9936

Fission
293 Kelvin Cross Sections

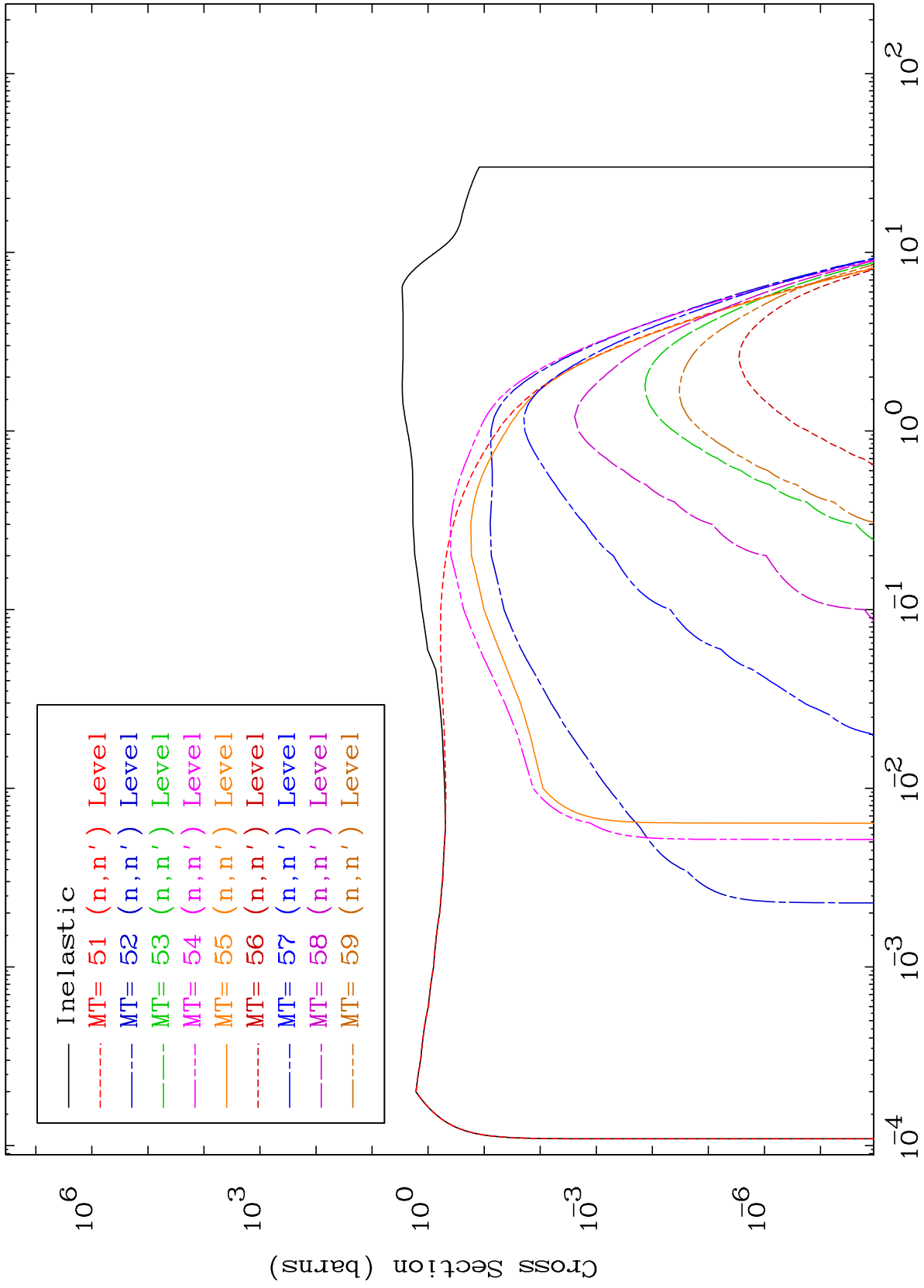
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MAT 9936

(n,n') Level
293 Kelvin Cross Sections

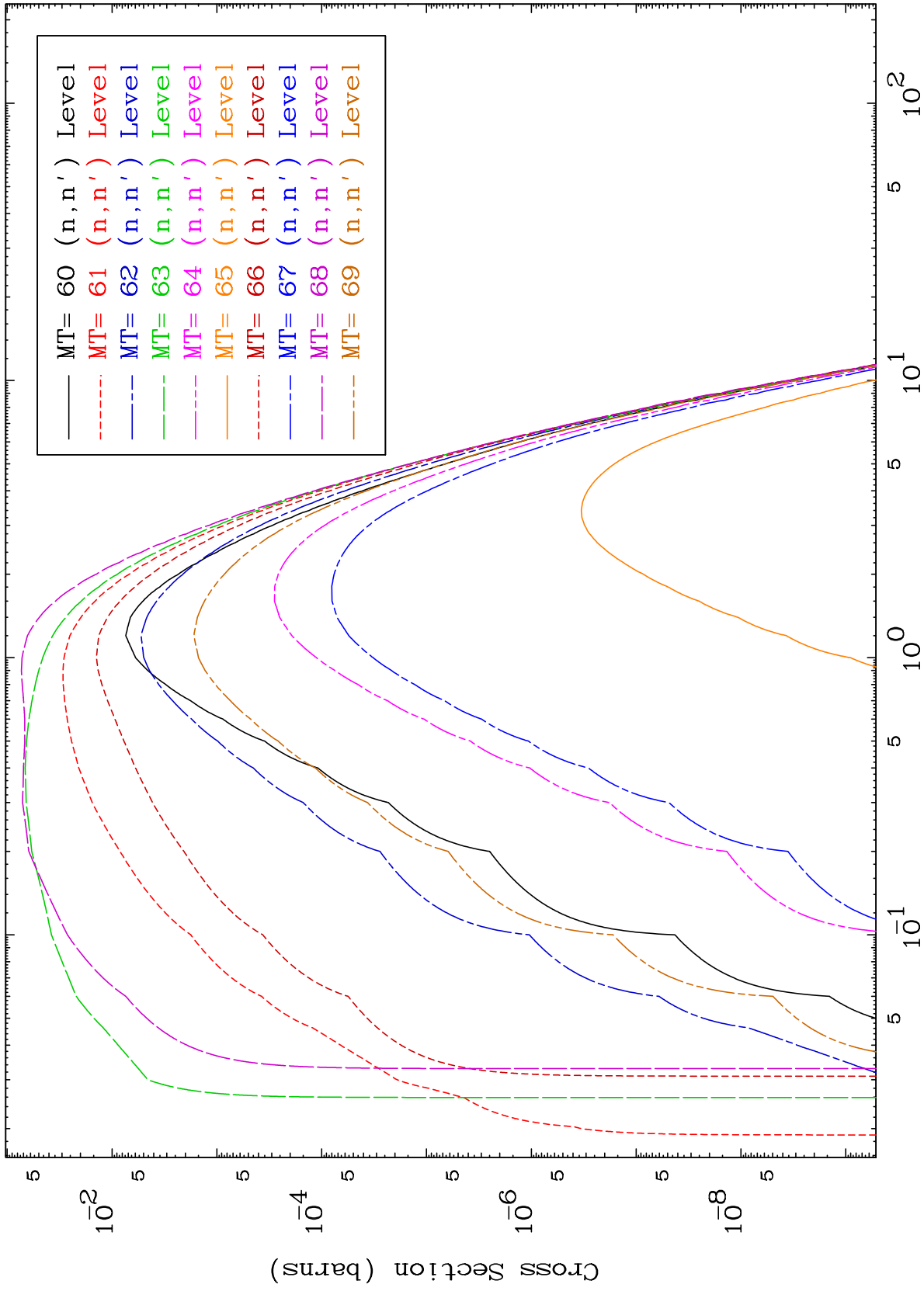
100-Fm-250



MAT 9936

(n,n') Level
293 Kelvin Cross Sections

100-Fm-250



9

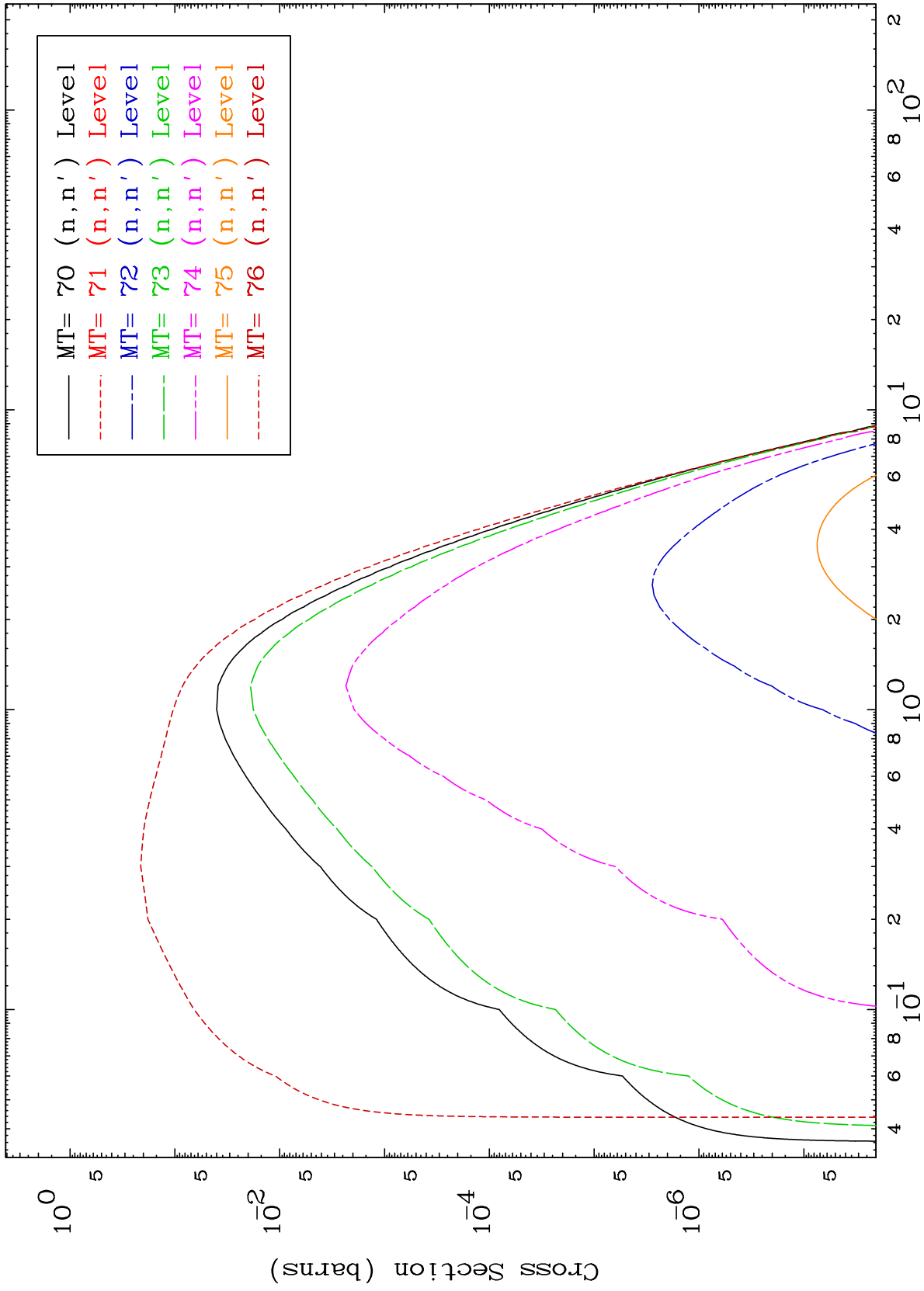
Incident Energy (MeV)

100-Fm-250

MAT 9936

(n,n') Level
293 Kelvin Cross Sections

100-Fm-250



10

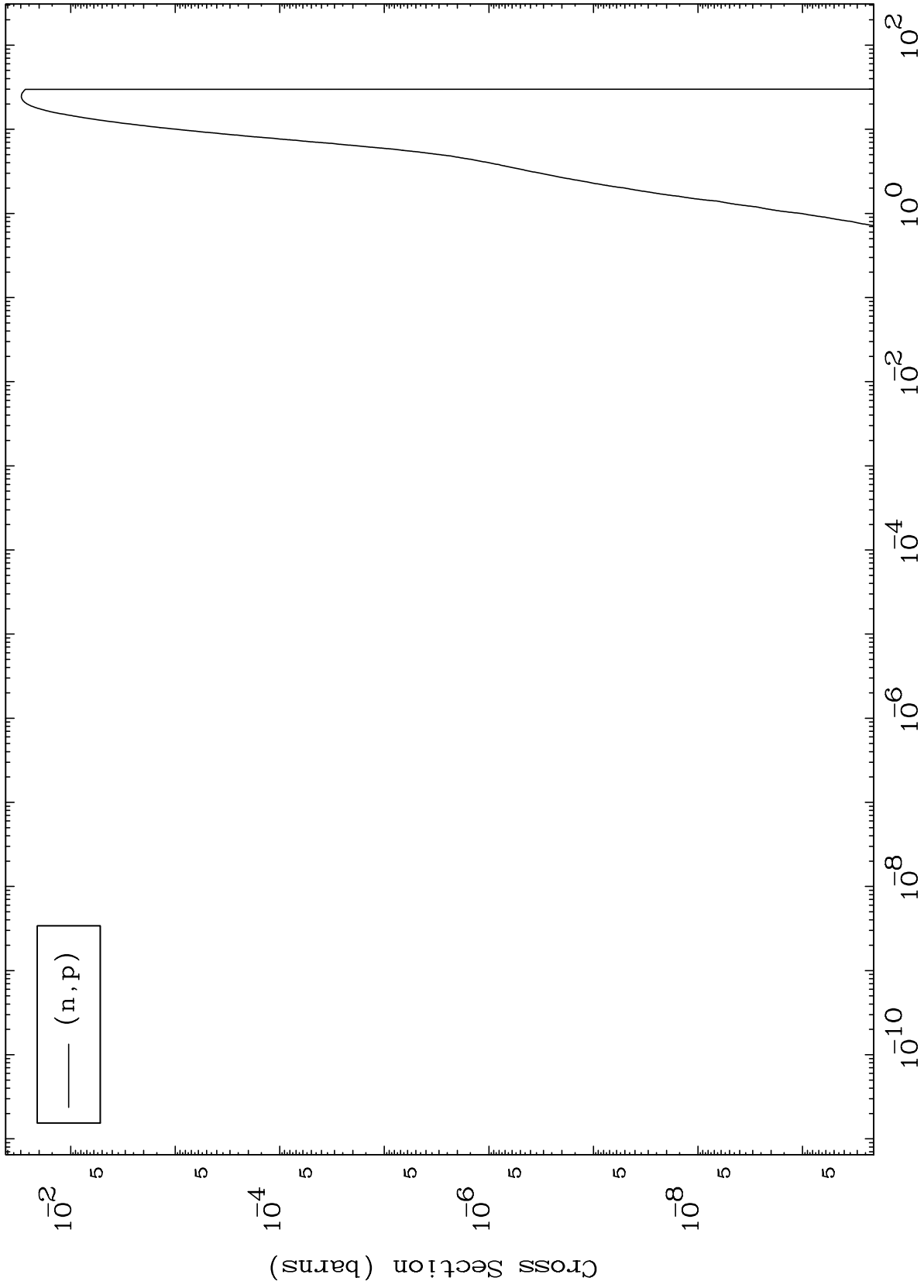
Incident Energy (MeV)

100-Fm-250

MAT 9936

(n,p) Levels
293 Kelvin Cross Sections

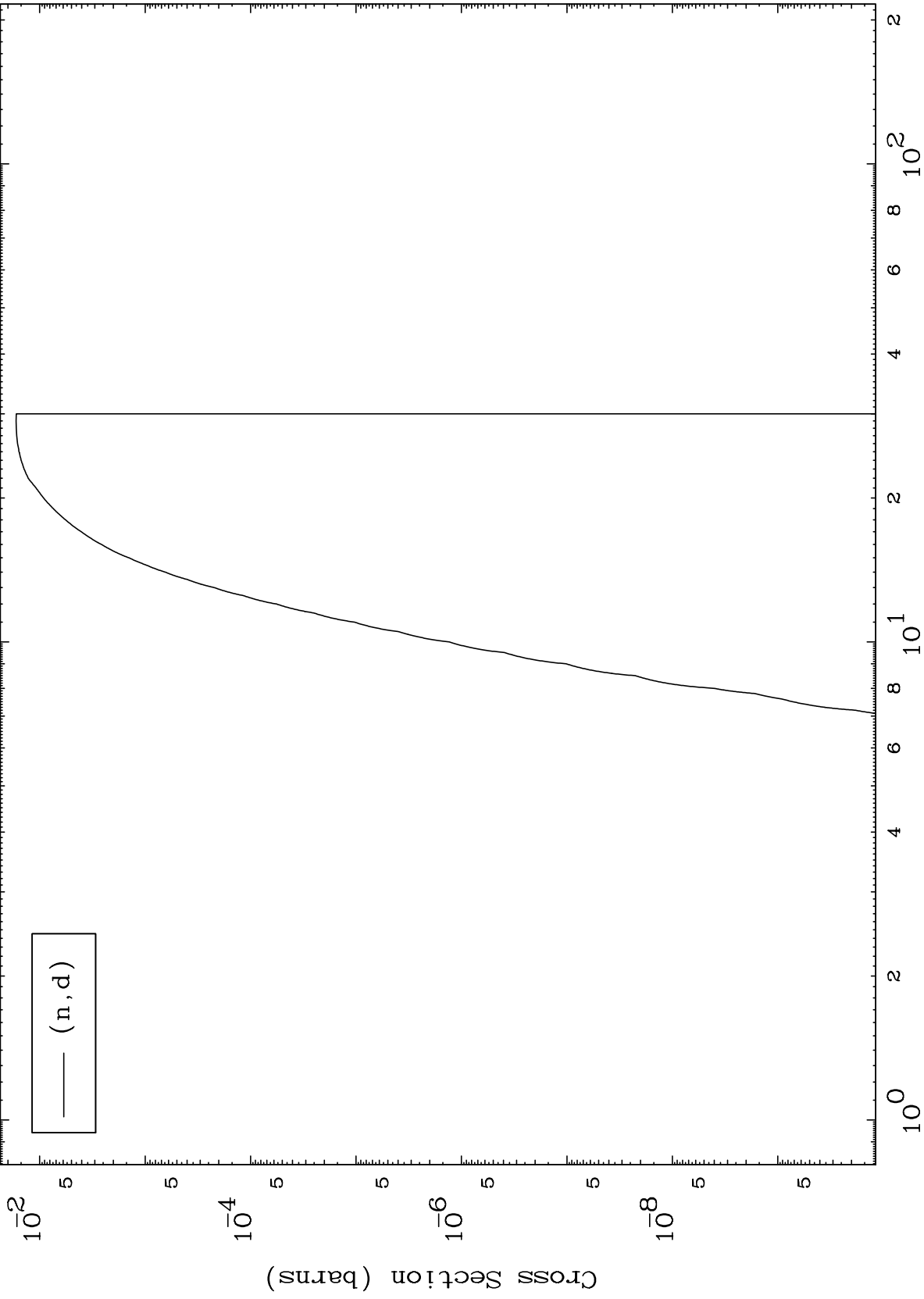
100-Fm-250



MAT 9936

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-250



12

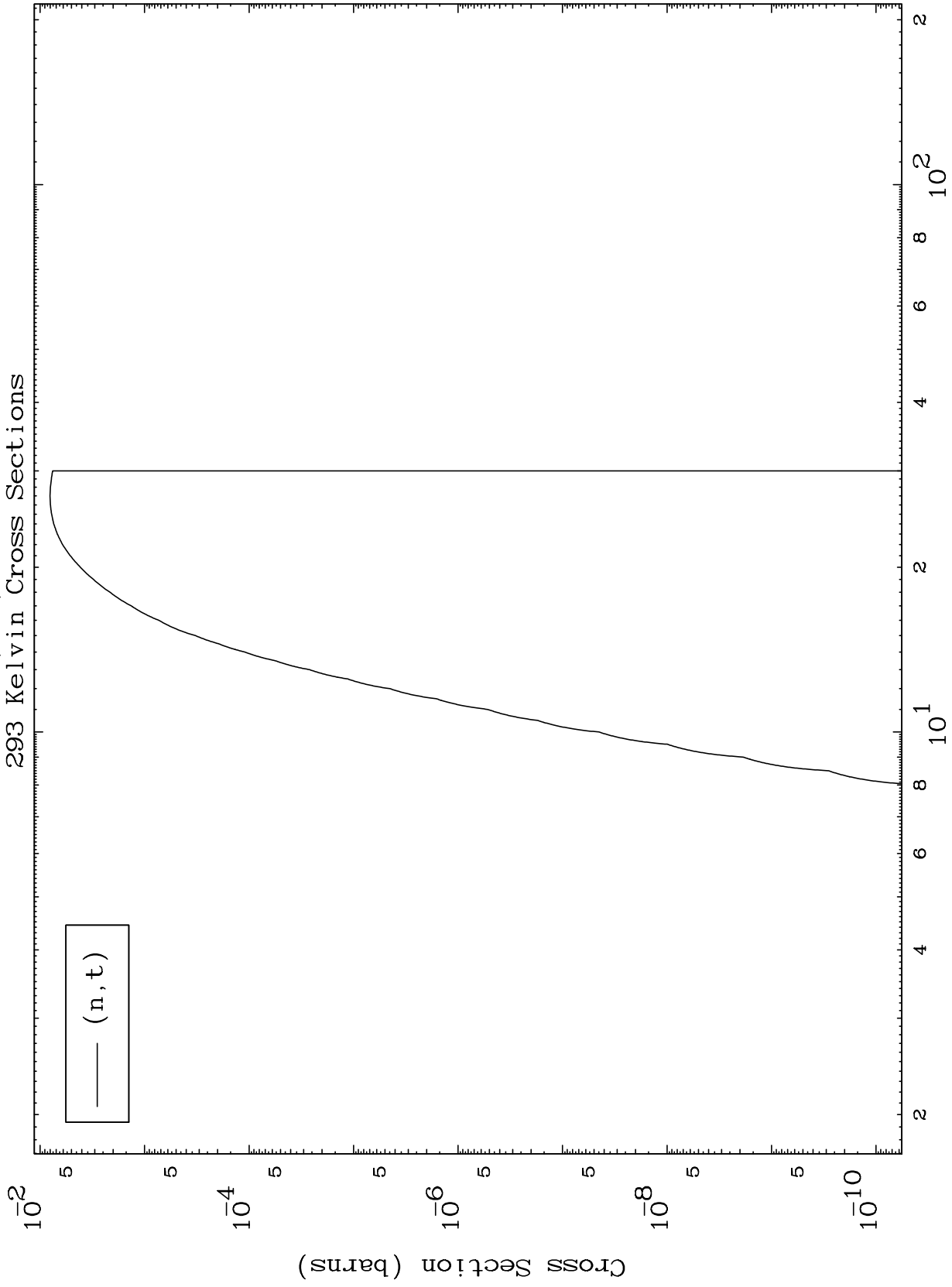
Incident Energy (MeV)

100-Fm-250

MAT 9936

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-250



13

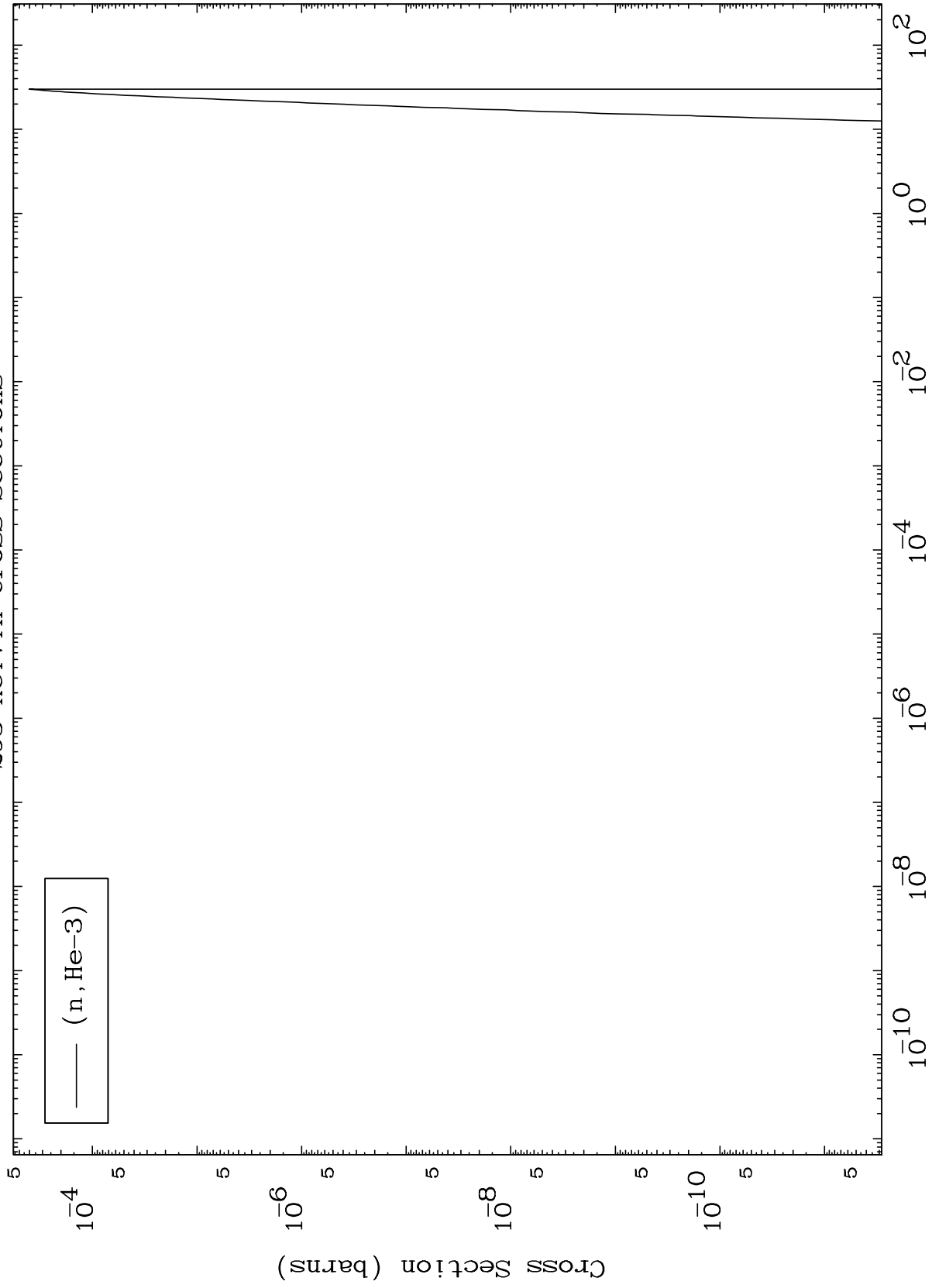
Incident Energy (MeV)

100-Fm-250

MAT 9936

(n,He3) Levels
293 Kelvin Cross Sections

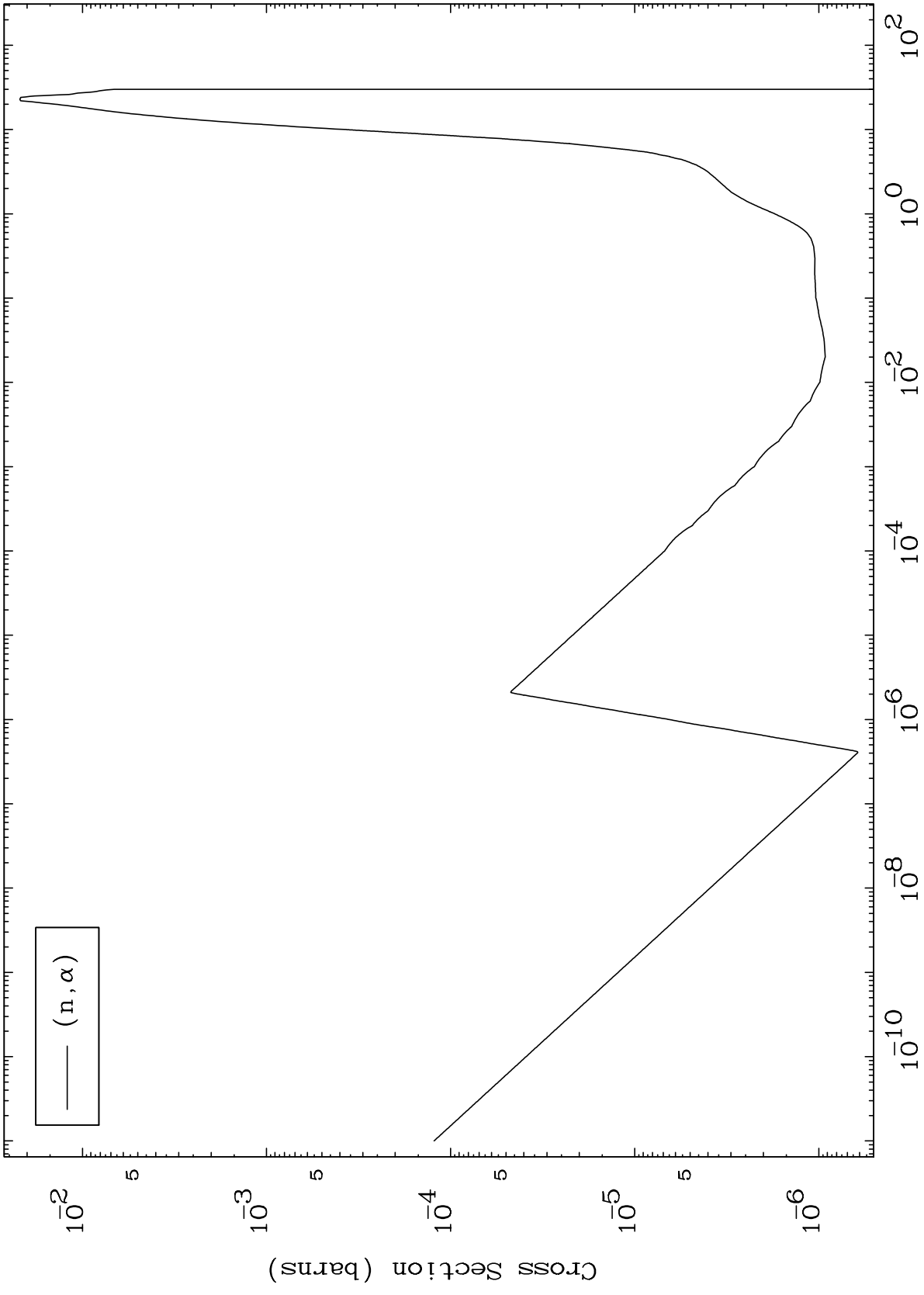
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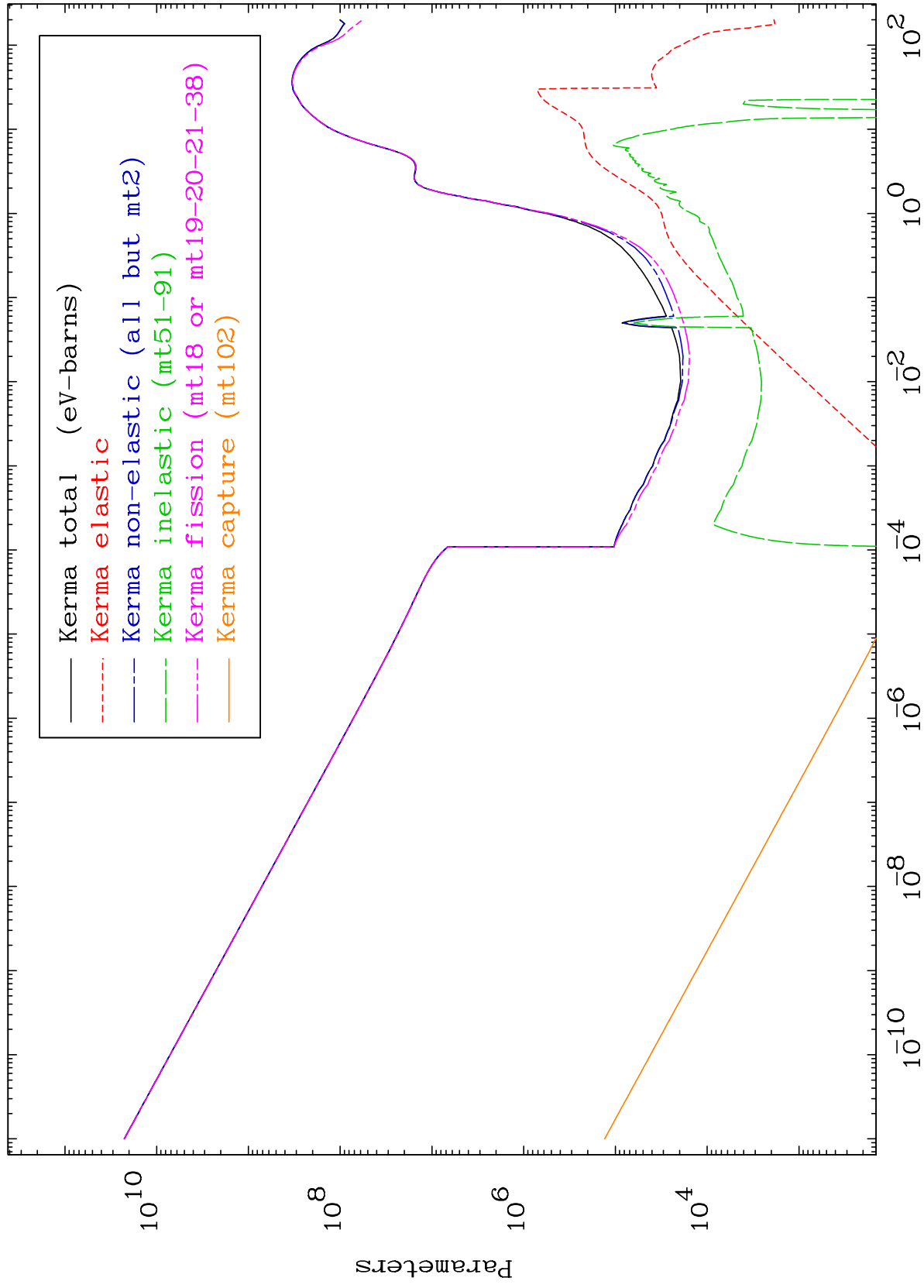
MAT 9936

(n,α) Levels
293 Kelvin Cross Sections

100-Fm-250



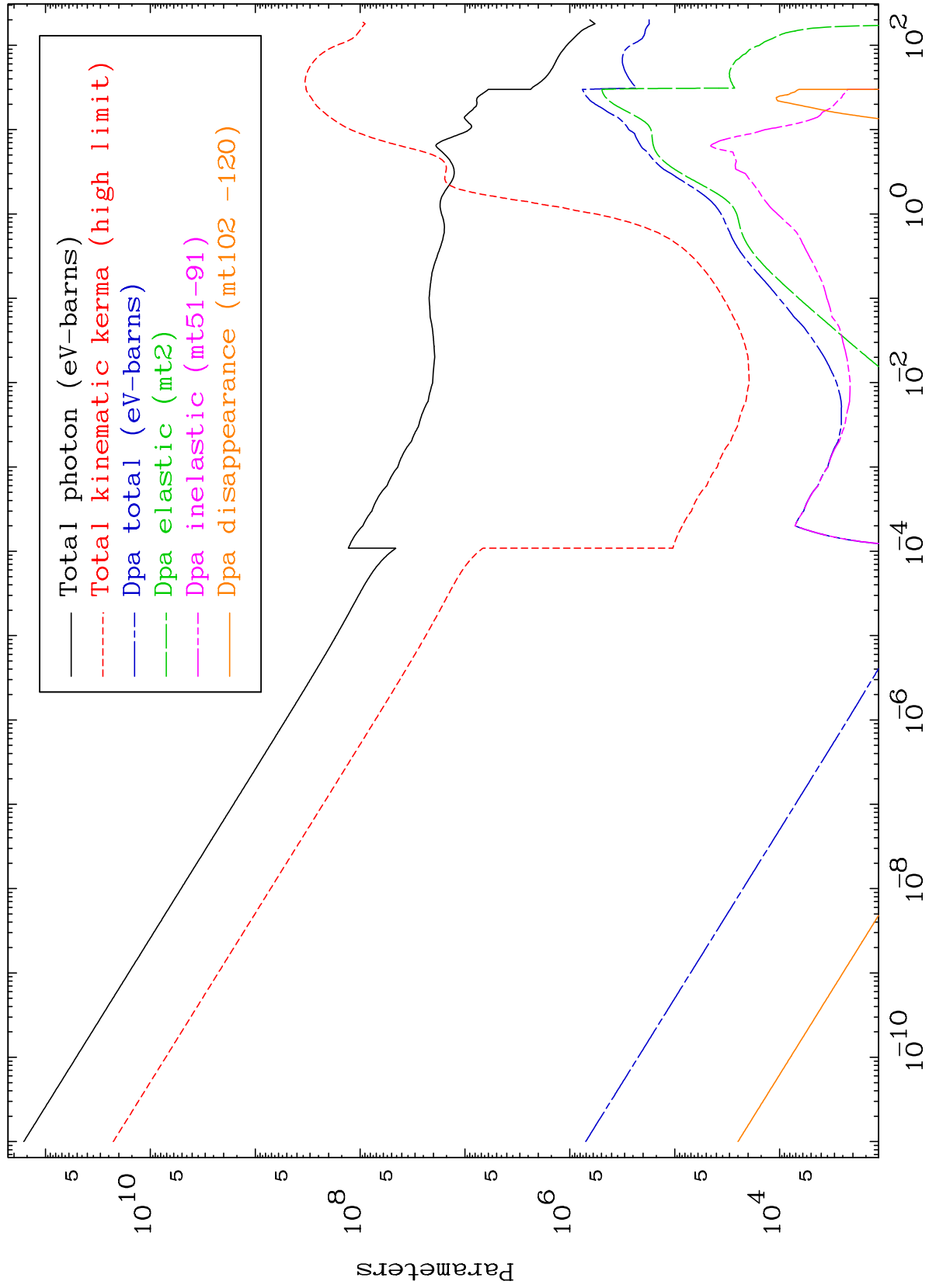
(n,α)



MAT 9936

Energy Release
Parameters

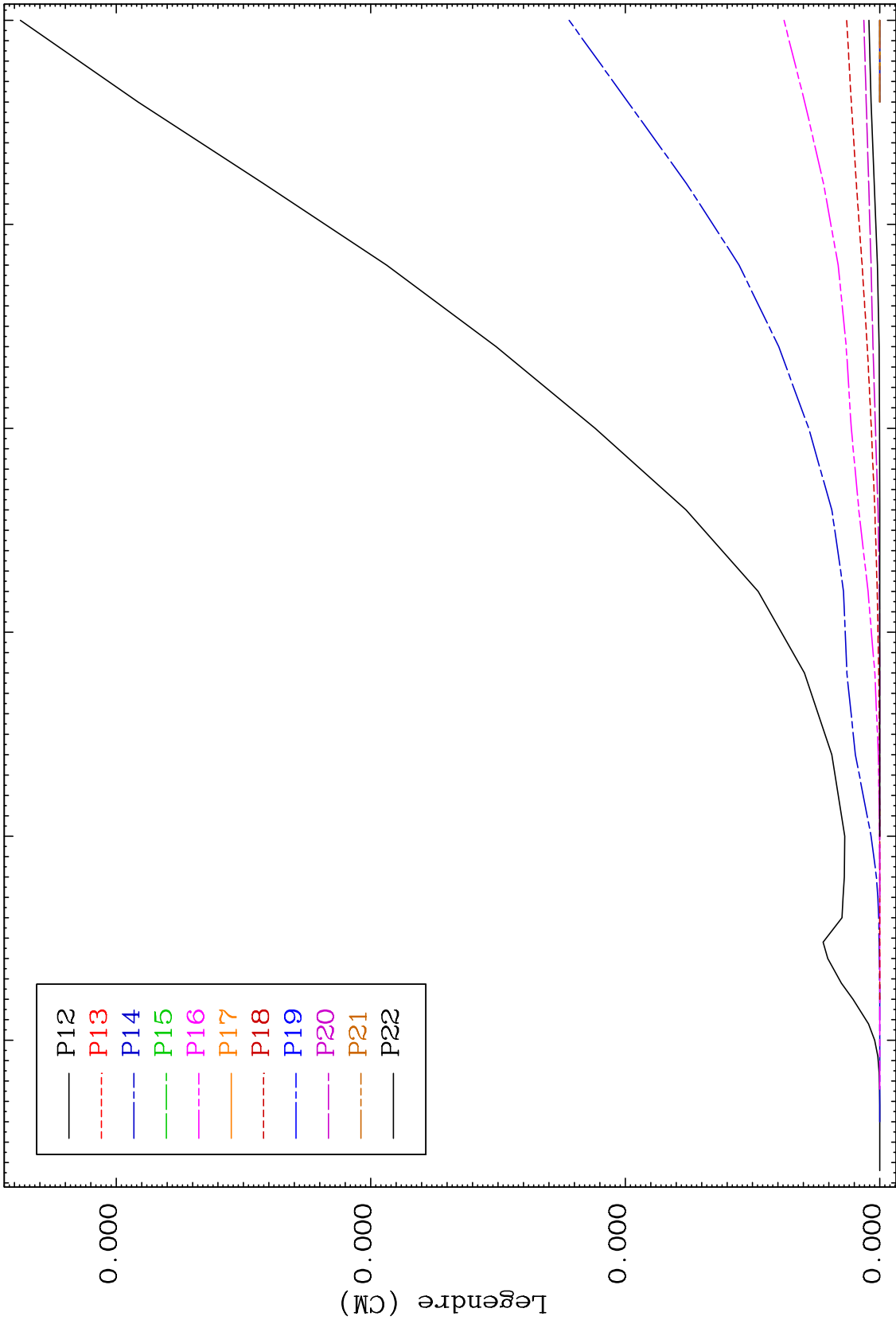
100-Fm-250



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Elastic Legendre Coefficients

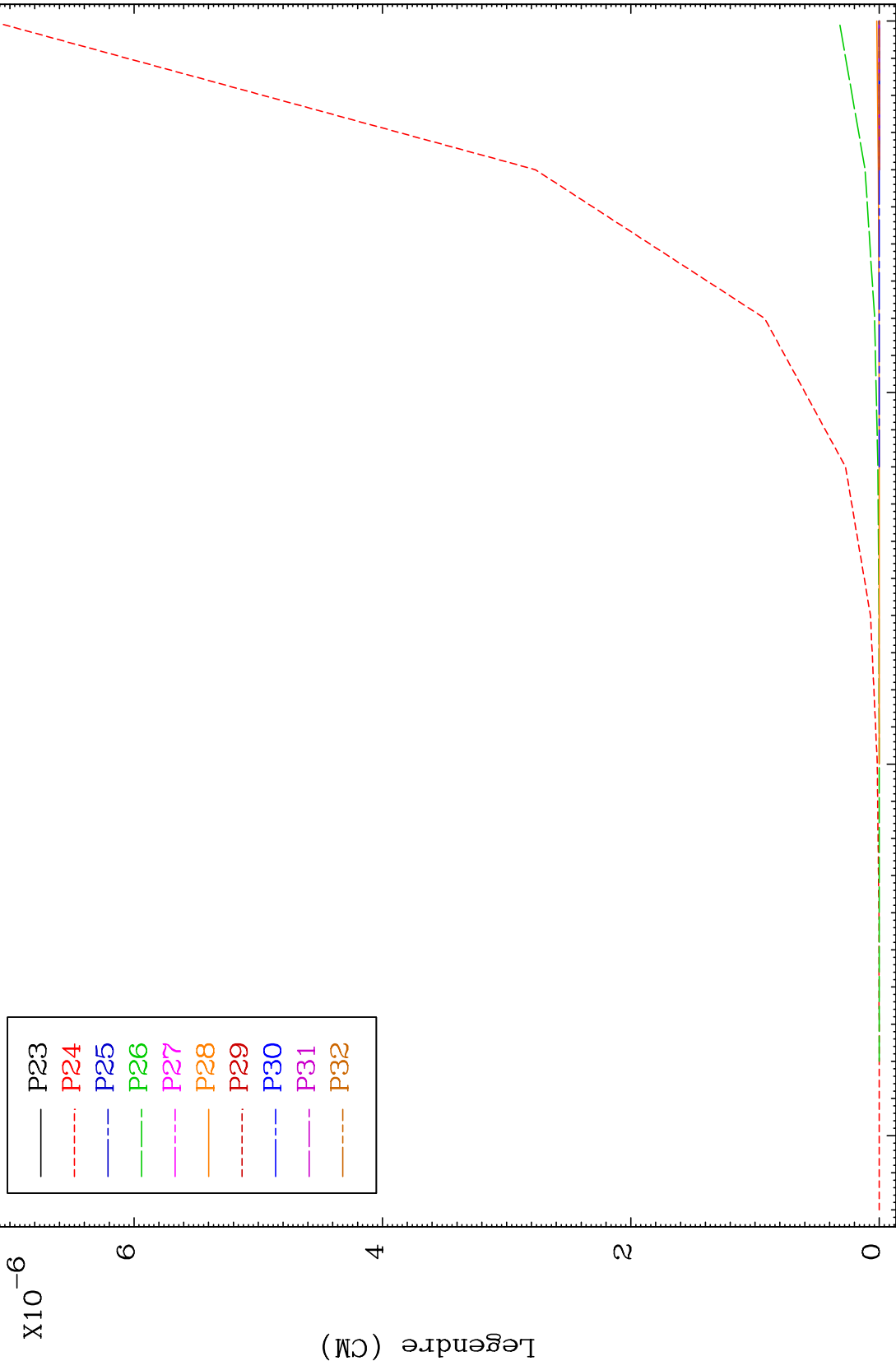
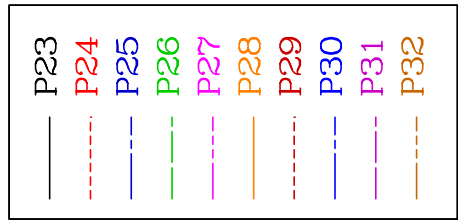
100-Fm-250



MAT 9936

Elastic Legendre Coefficients

100-Fm-250



20

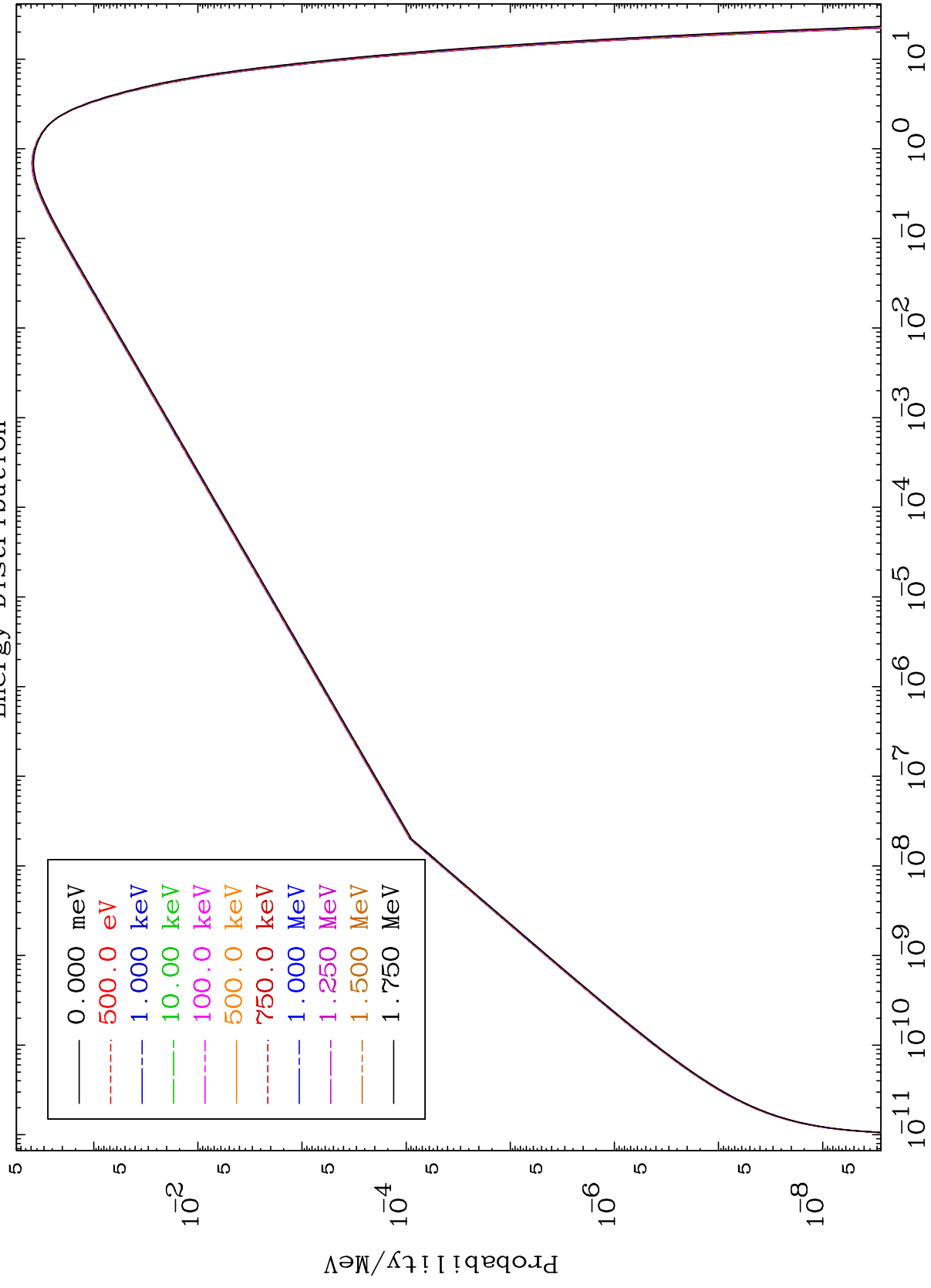
Incident Energy (MeV)

100-Fm-250

MAT 9936

Fission Energy Distribution

100-Fm-250

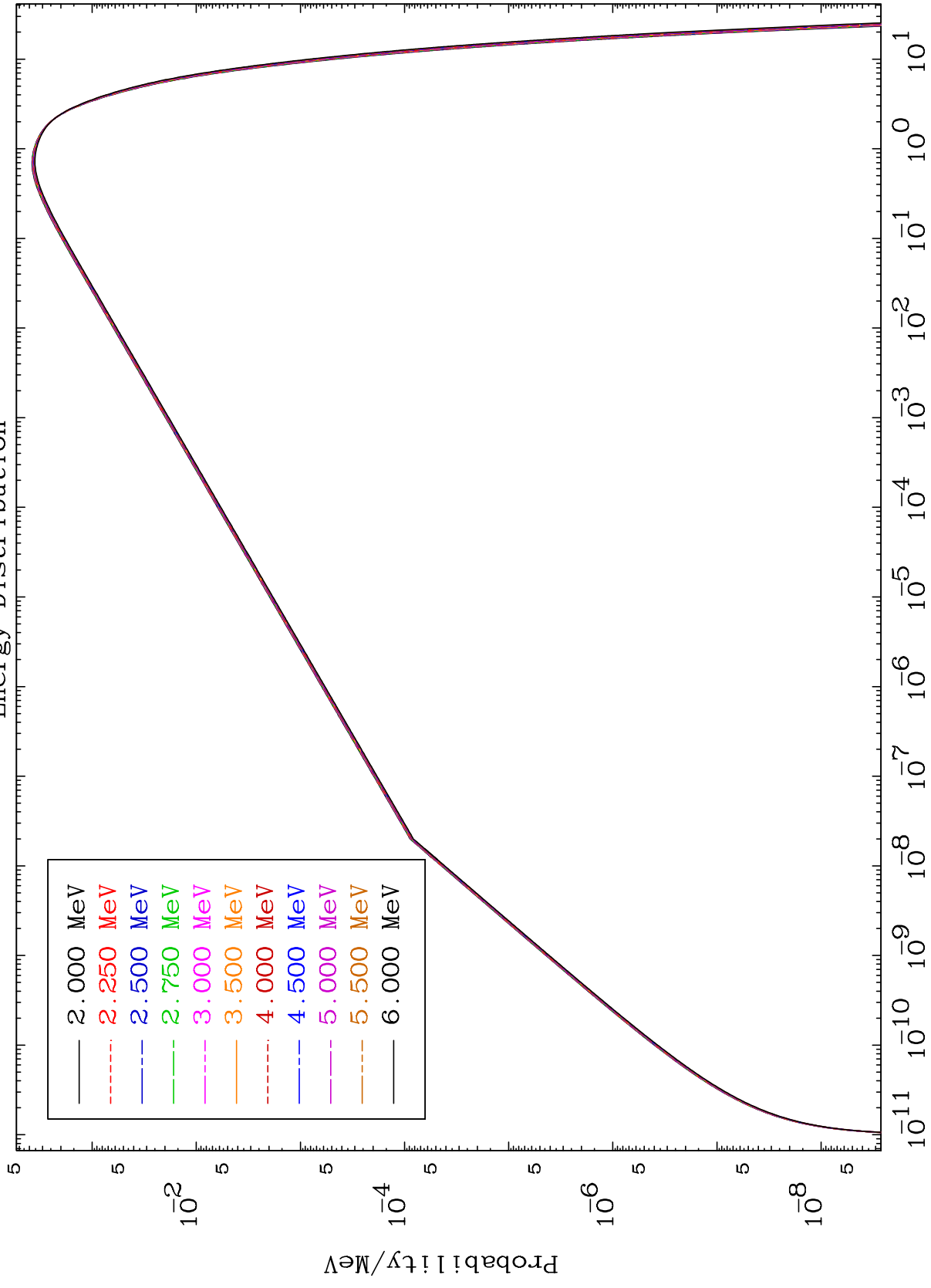


100-Fm-250

MAT 9936

Fission Energy Distribution

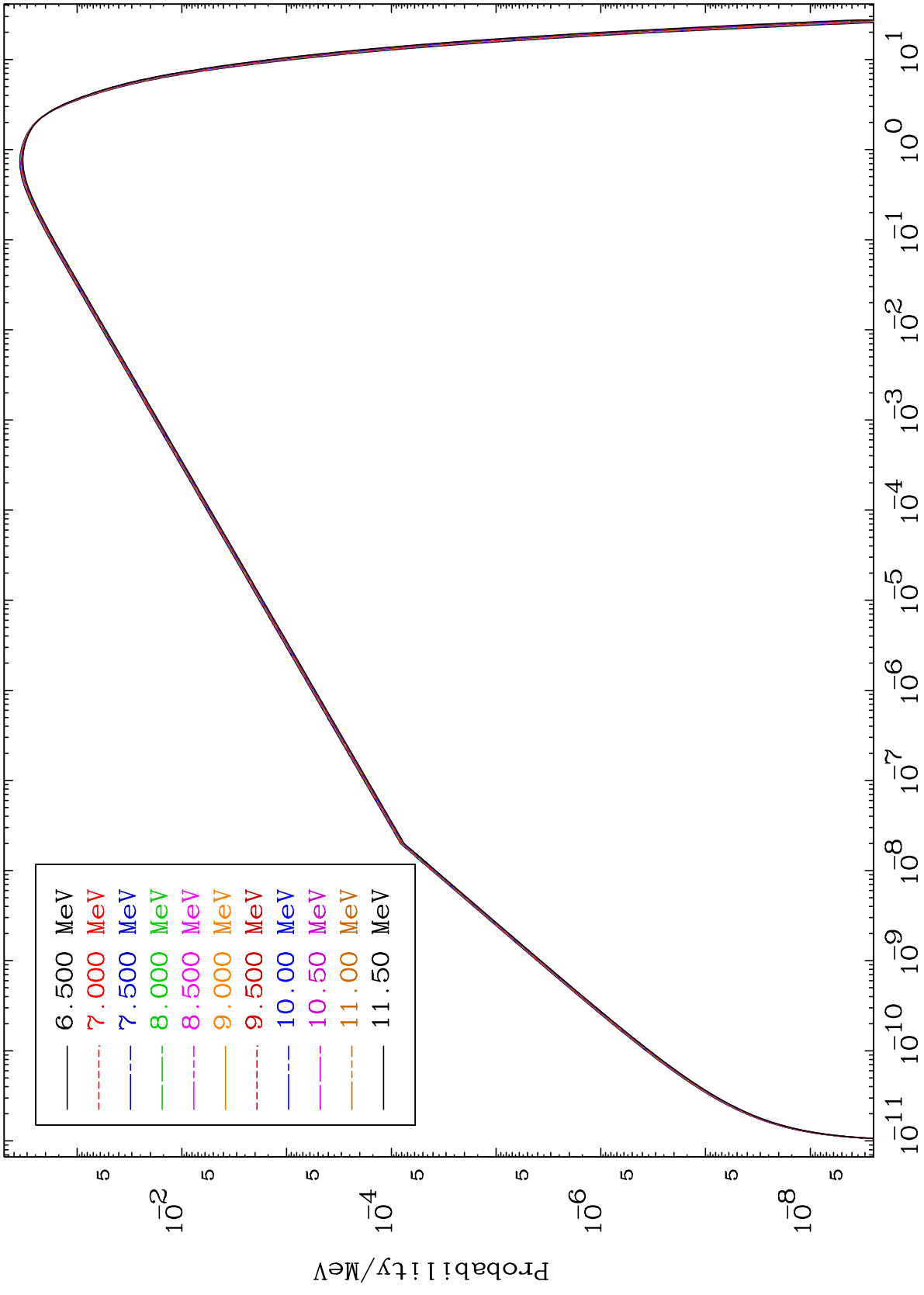
100-Fm-250



MAT 9936

Fission Energy Distribution

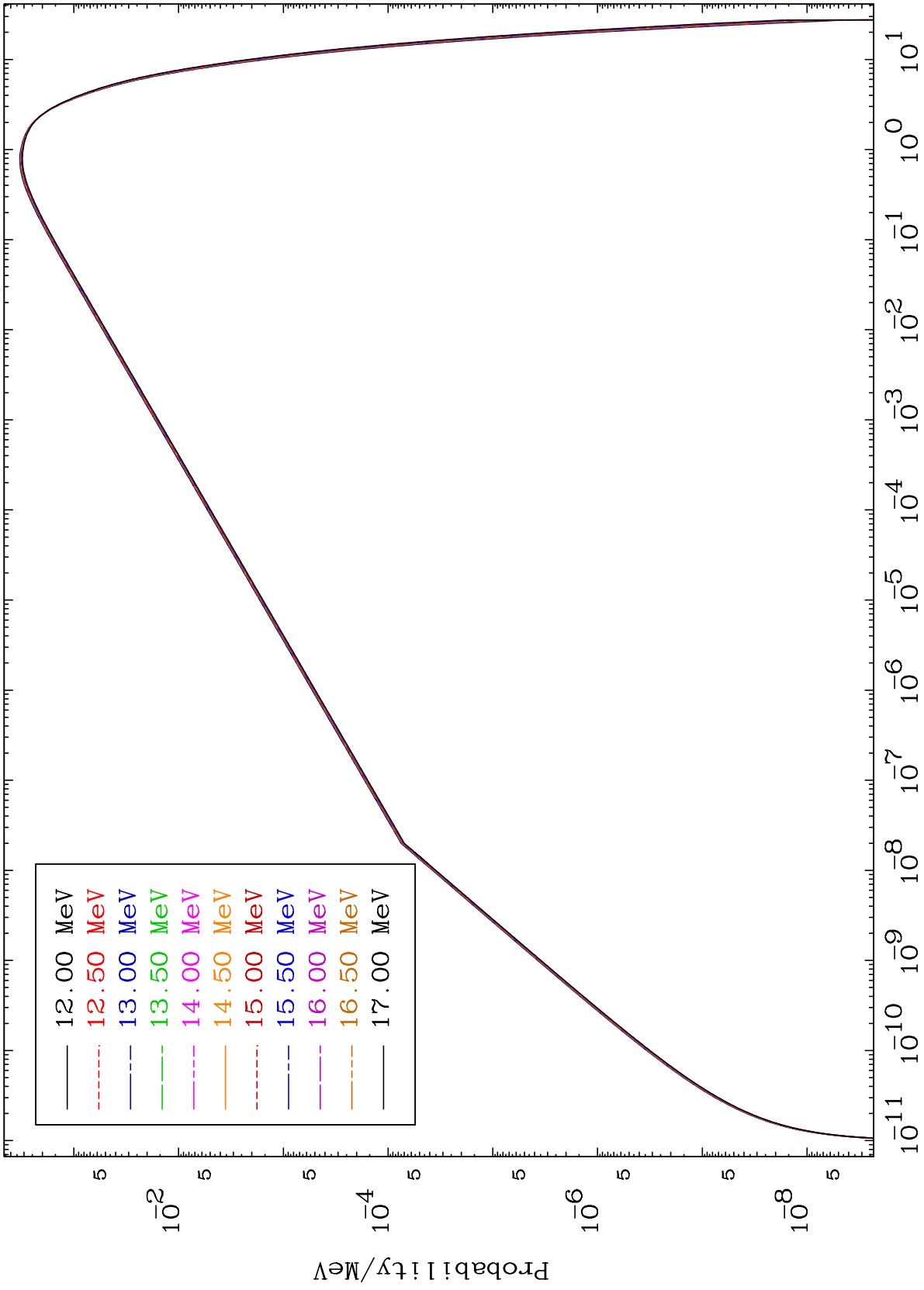
100-Fm-250



MAT 9936

Fission Energy Distribution

100-Fm-250

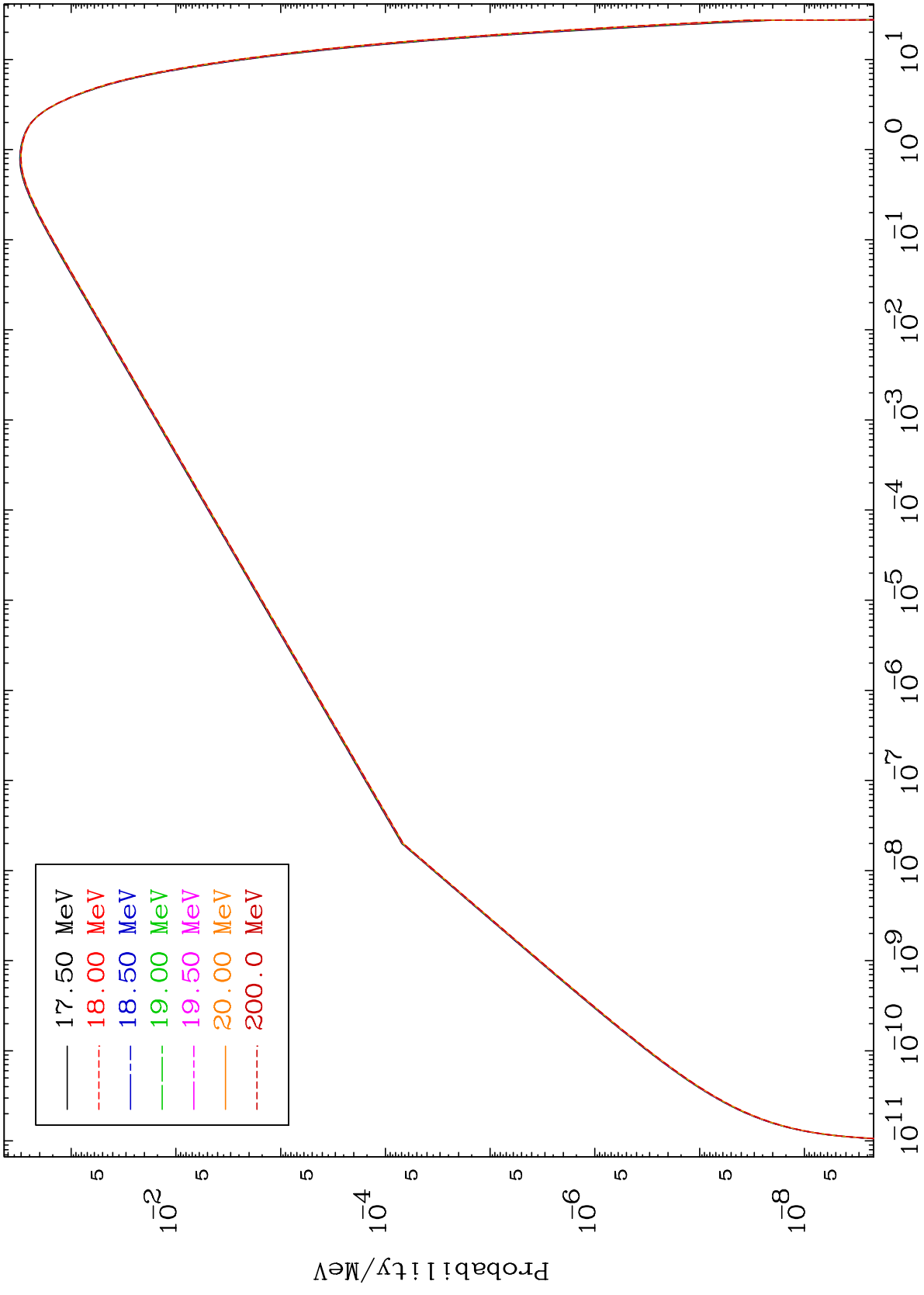


100-Fm-250

MAT 9936

Fission Energy Distribution

100-Fm-250

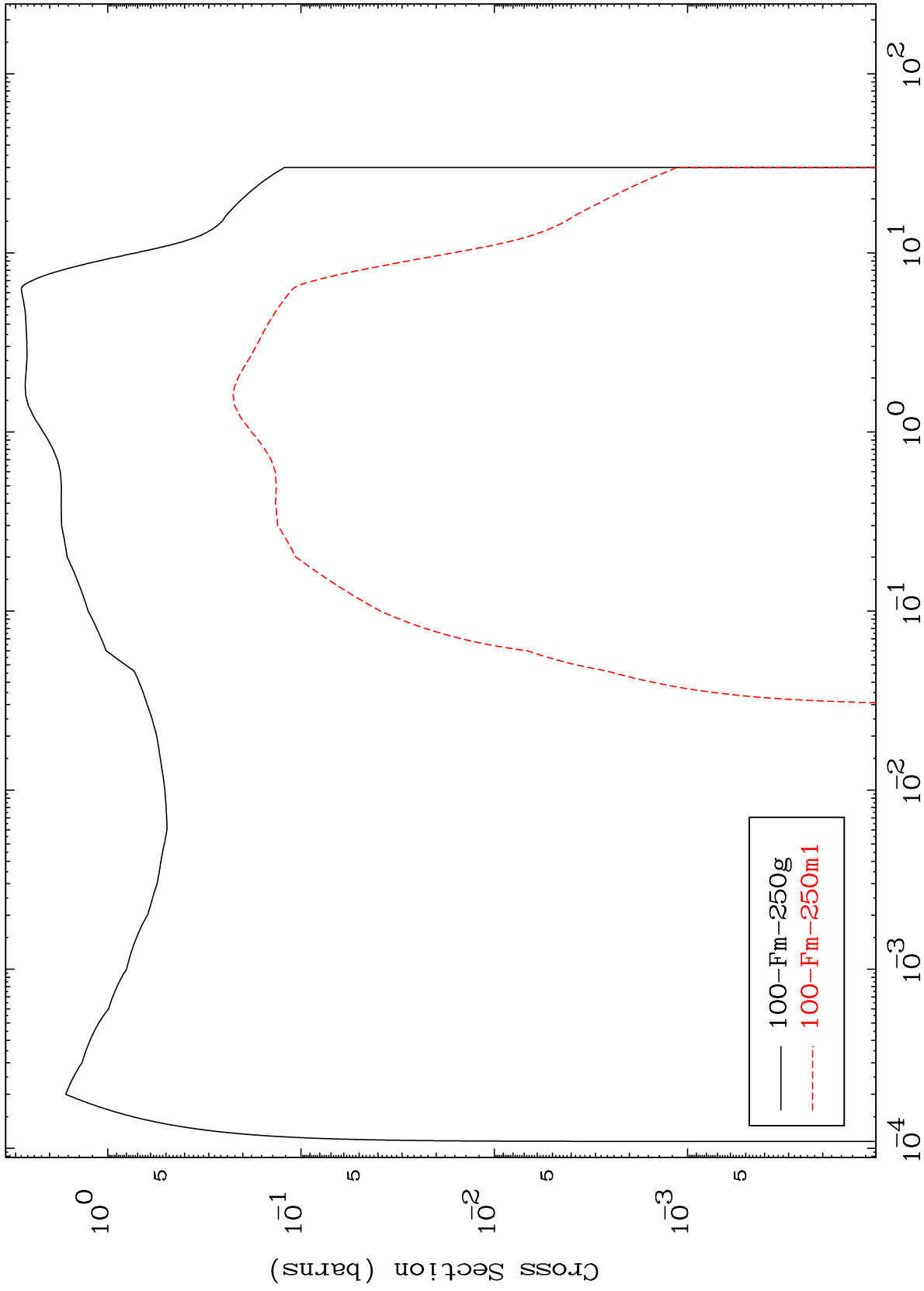


100-Fm-250

MAT 9936

100-Fm-250

Inelastic
Radionuclide Production Cross Section



26

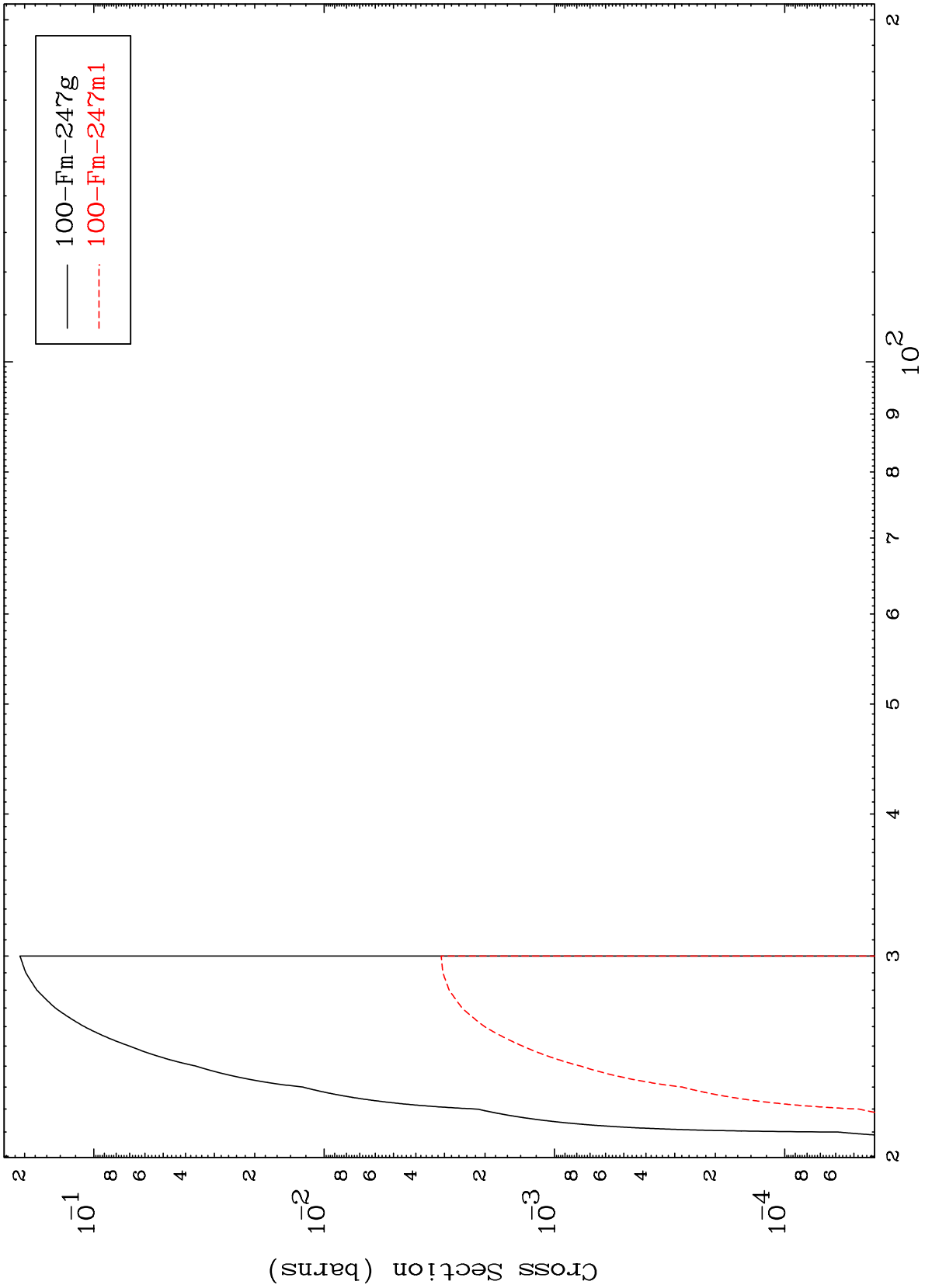
100-Fm-250

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(n,4n)

100-Fm-250

Radionuclide Production Cross Section



27

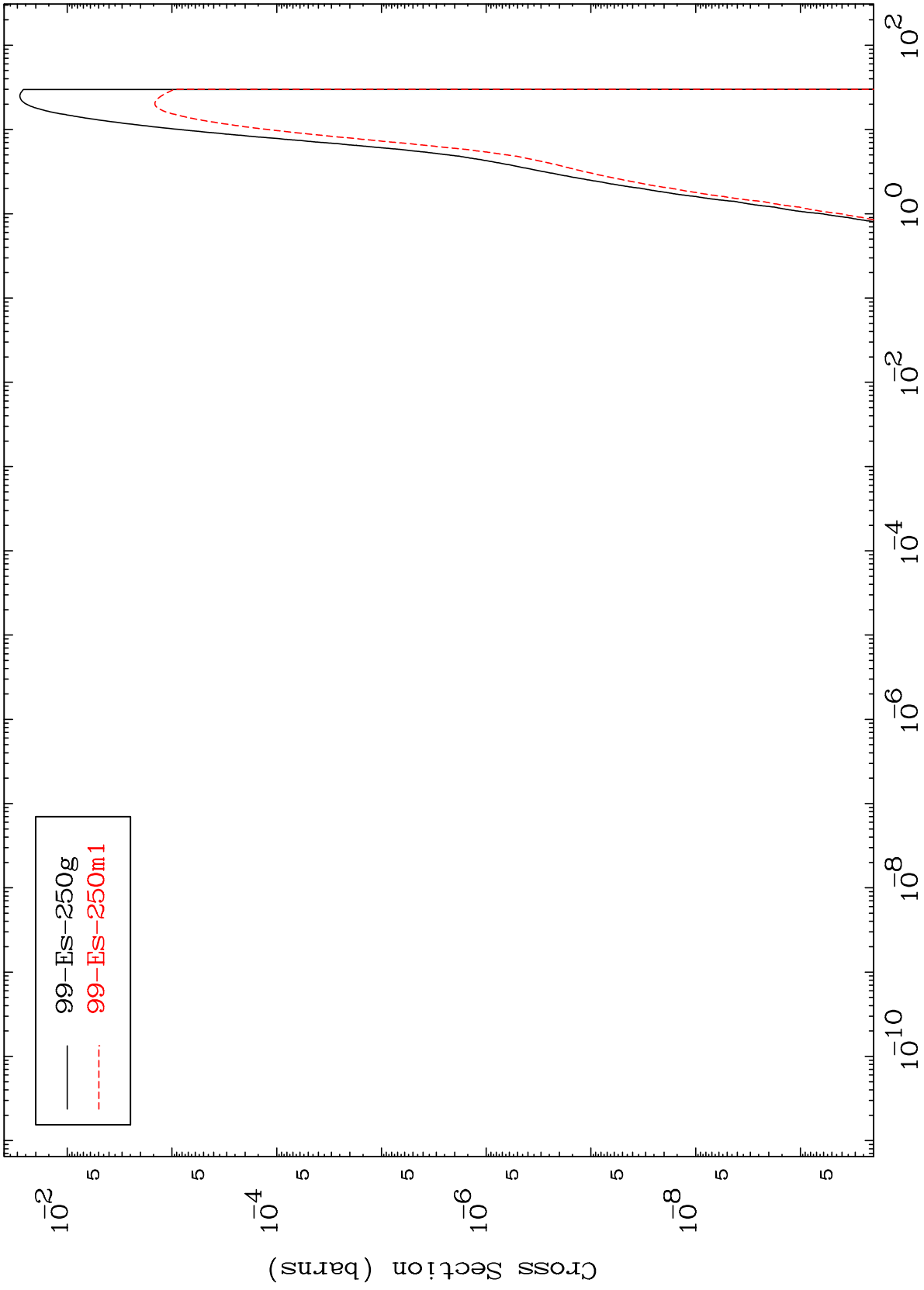
Incident Energy (MeV)

100-Fm-250

MAT 9936

(n,p)
Radionuclide Production Cross Section

100-Fm-250



28

100-Fm-250