

Program EVALPLOT
(Version 2018-1)

by

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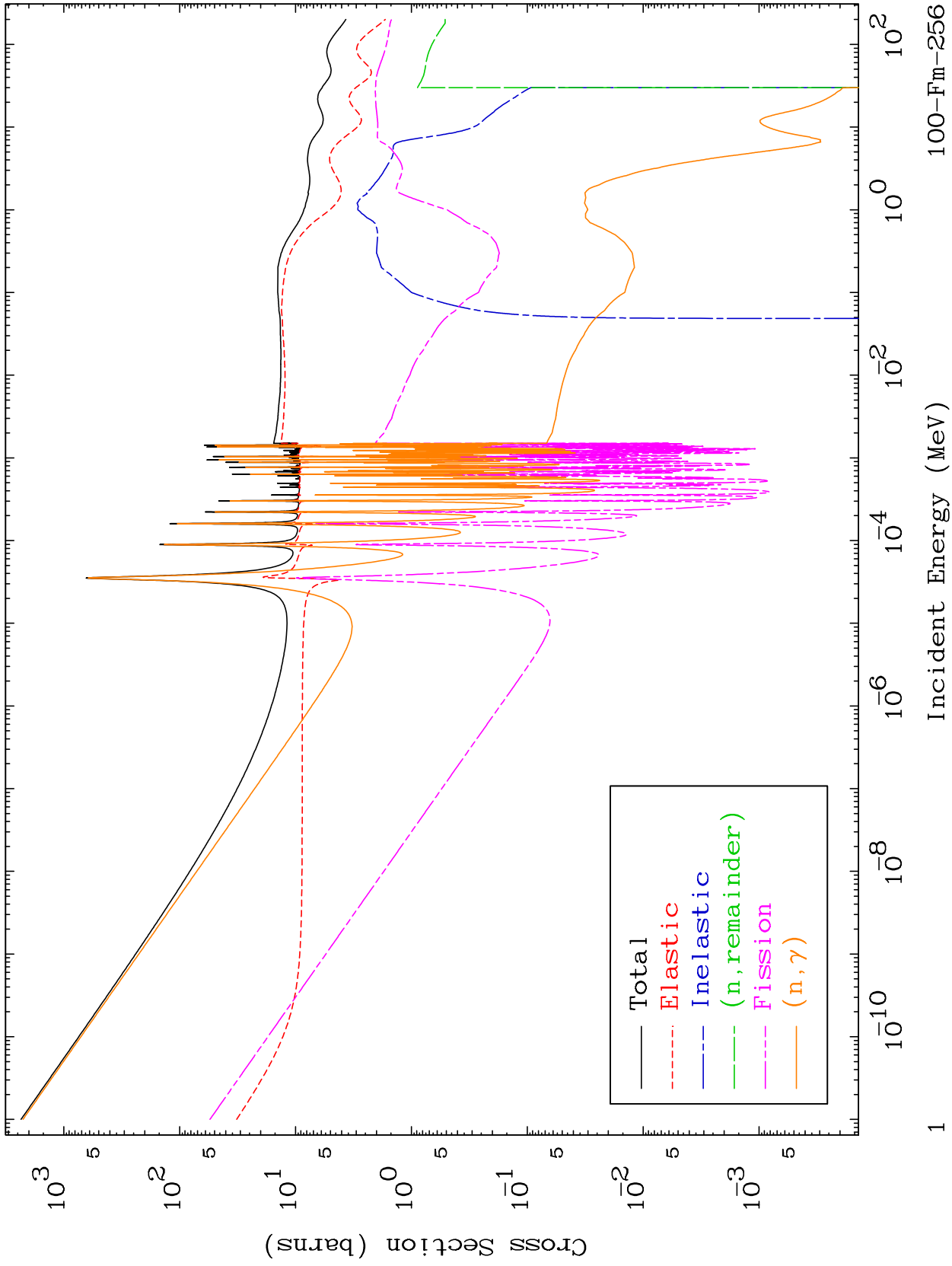
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9937

Major
293 Kelvin Cross Sections

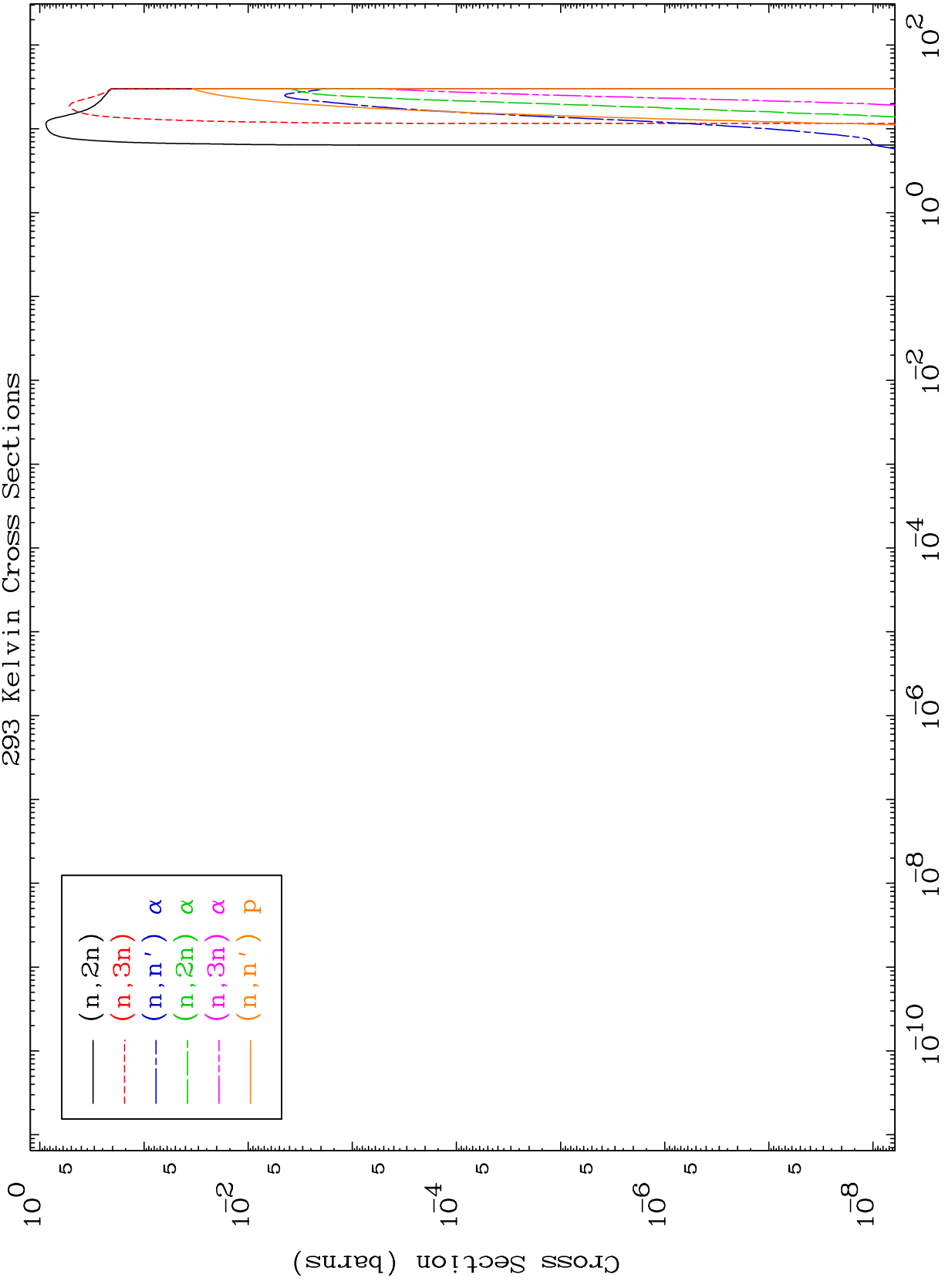
100-Fm-256



MAT 9937

Neutron Production
293 Kelvin Cross Sections

100-Fm-256



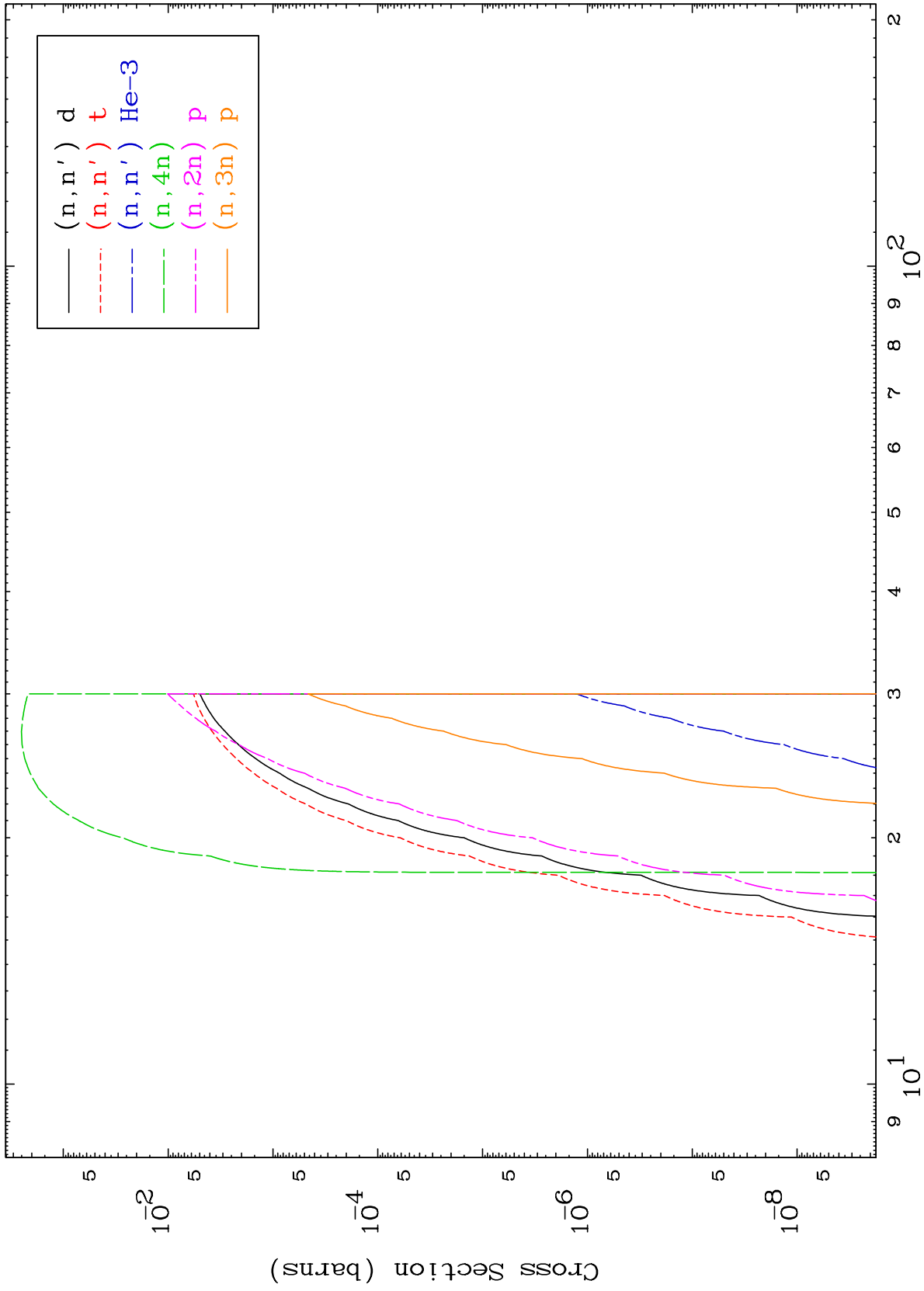
2

100-Fm-256

MAT 9937

Neutron Production
293 Kelvin Cross Sections

100-Fm-256



3

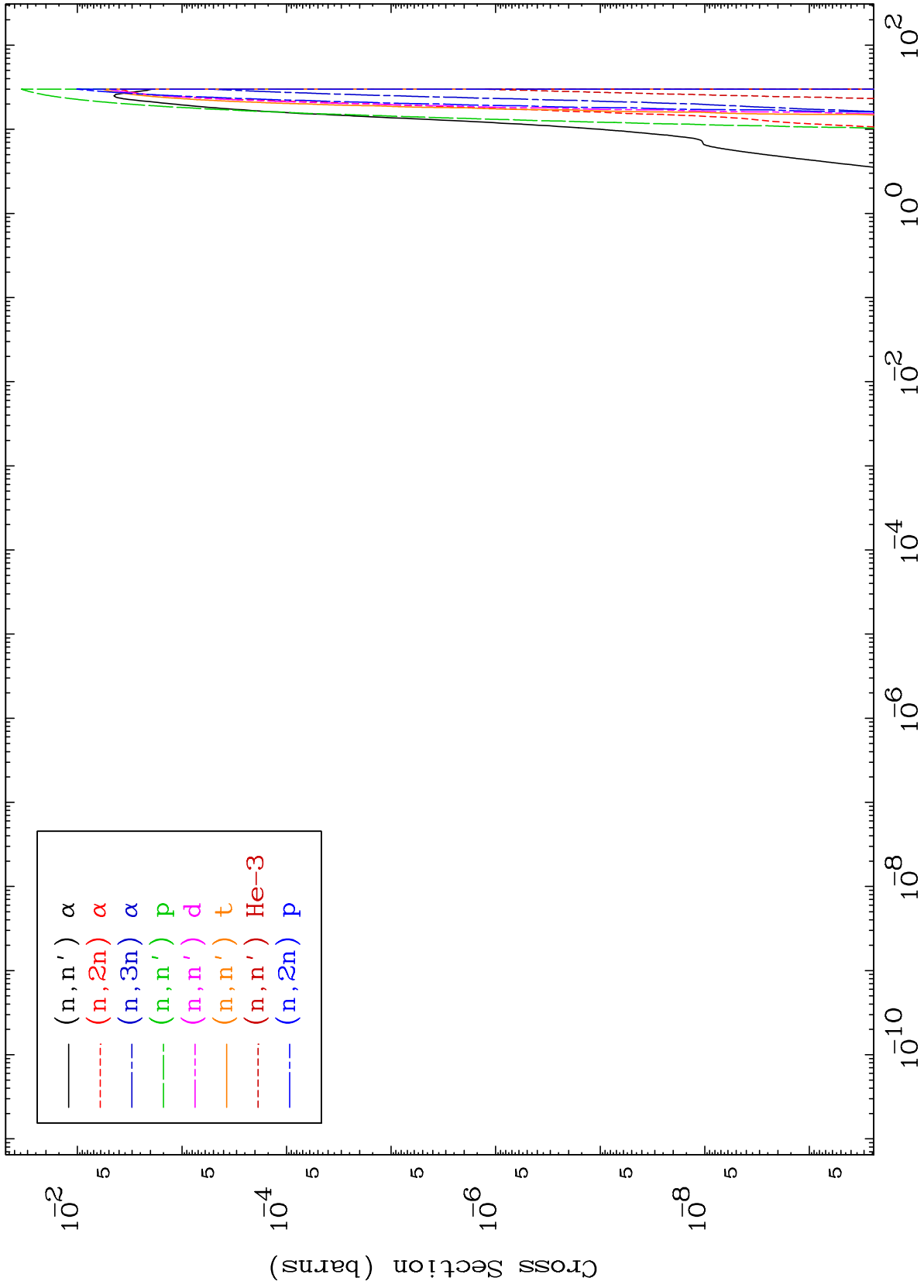
Incident Energy (MeV)

100-Fm-256

MAT 9937

Charged Particle
293 Kelvin Cross Sections

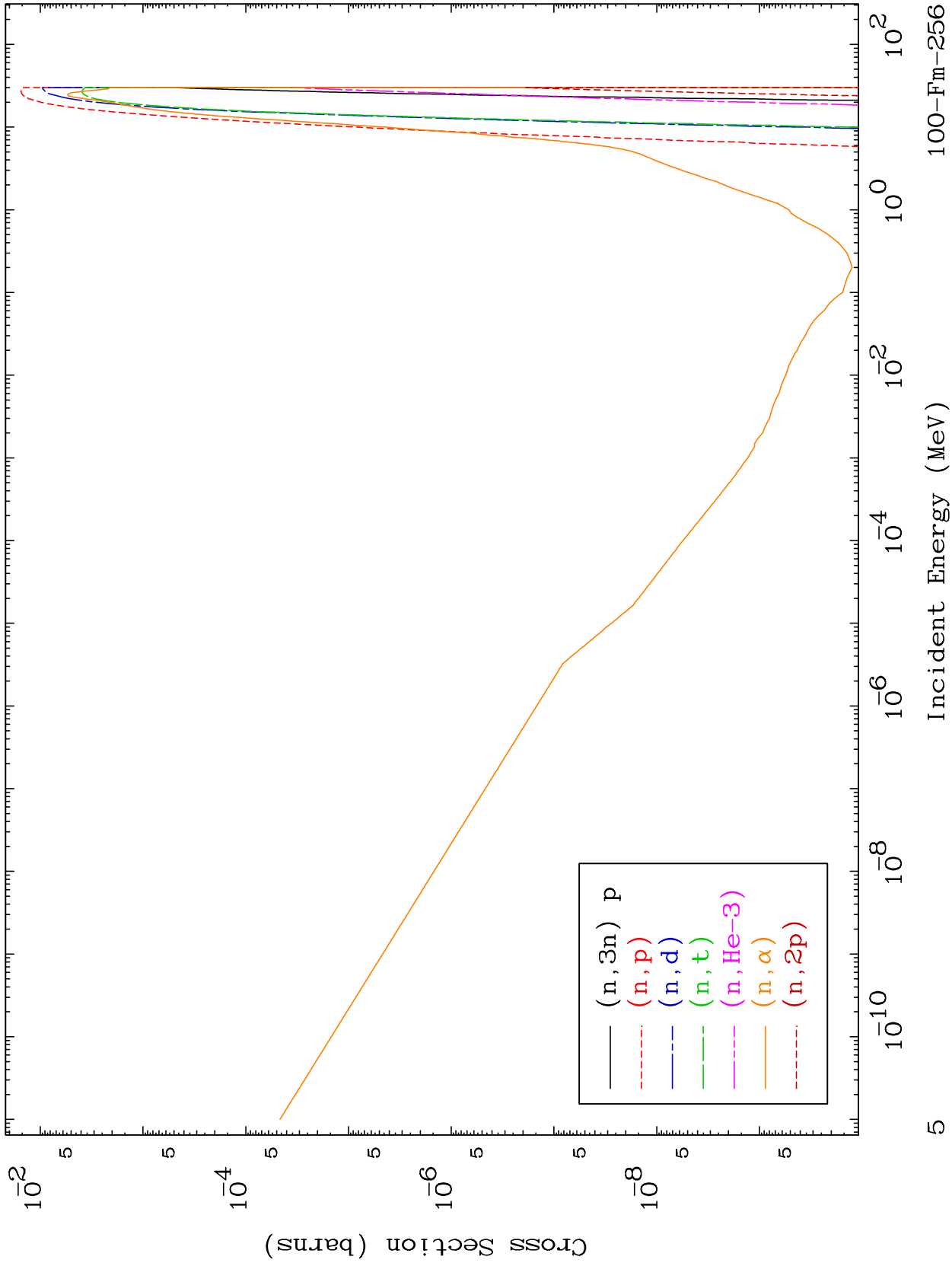
100-Fm-256



MAT 9937

293 Kelvin Cross Sections

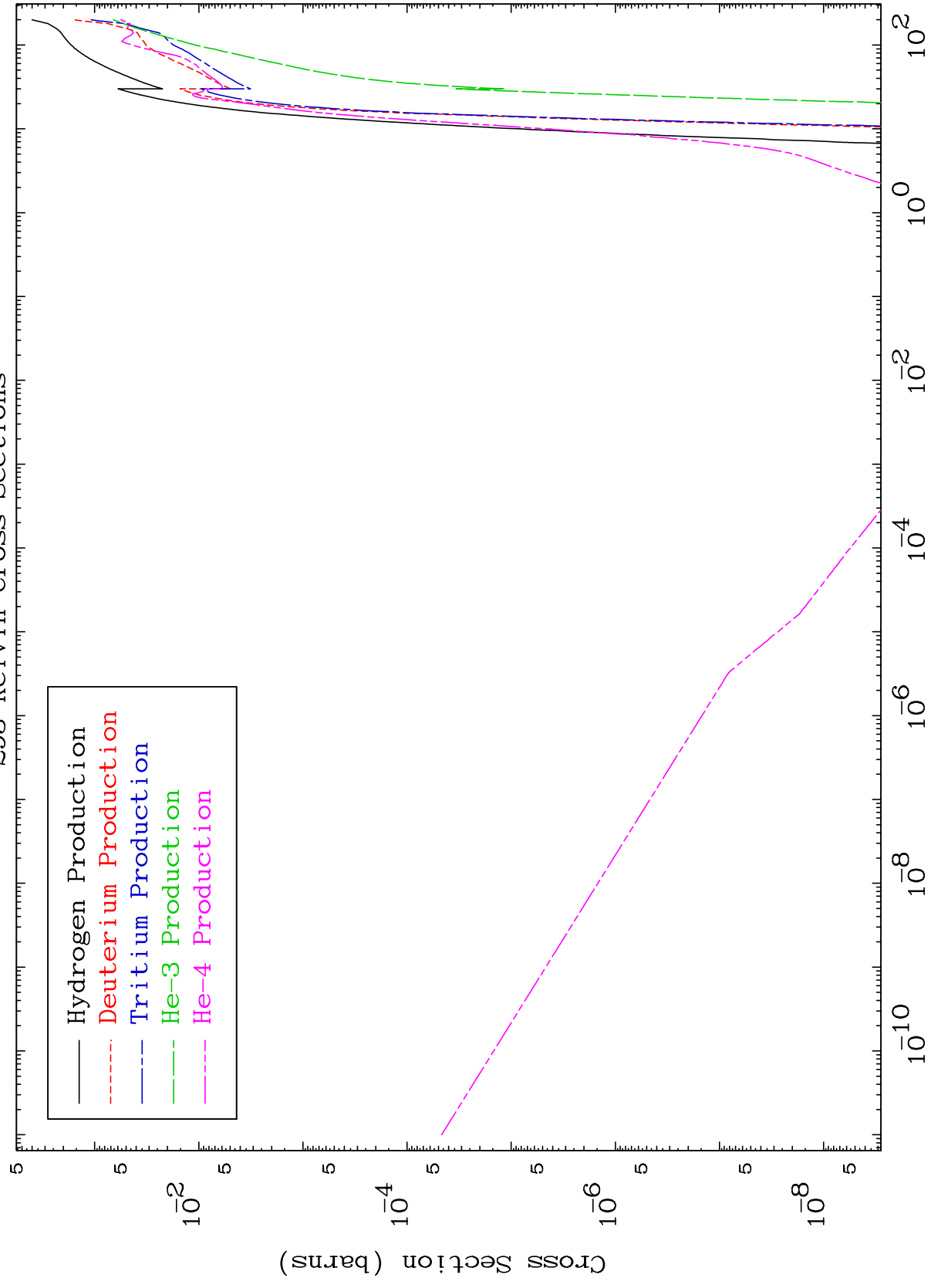
100-Fm-256



MAT 9937

Particle Production
293 Kelvin Cross Sections

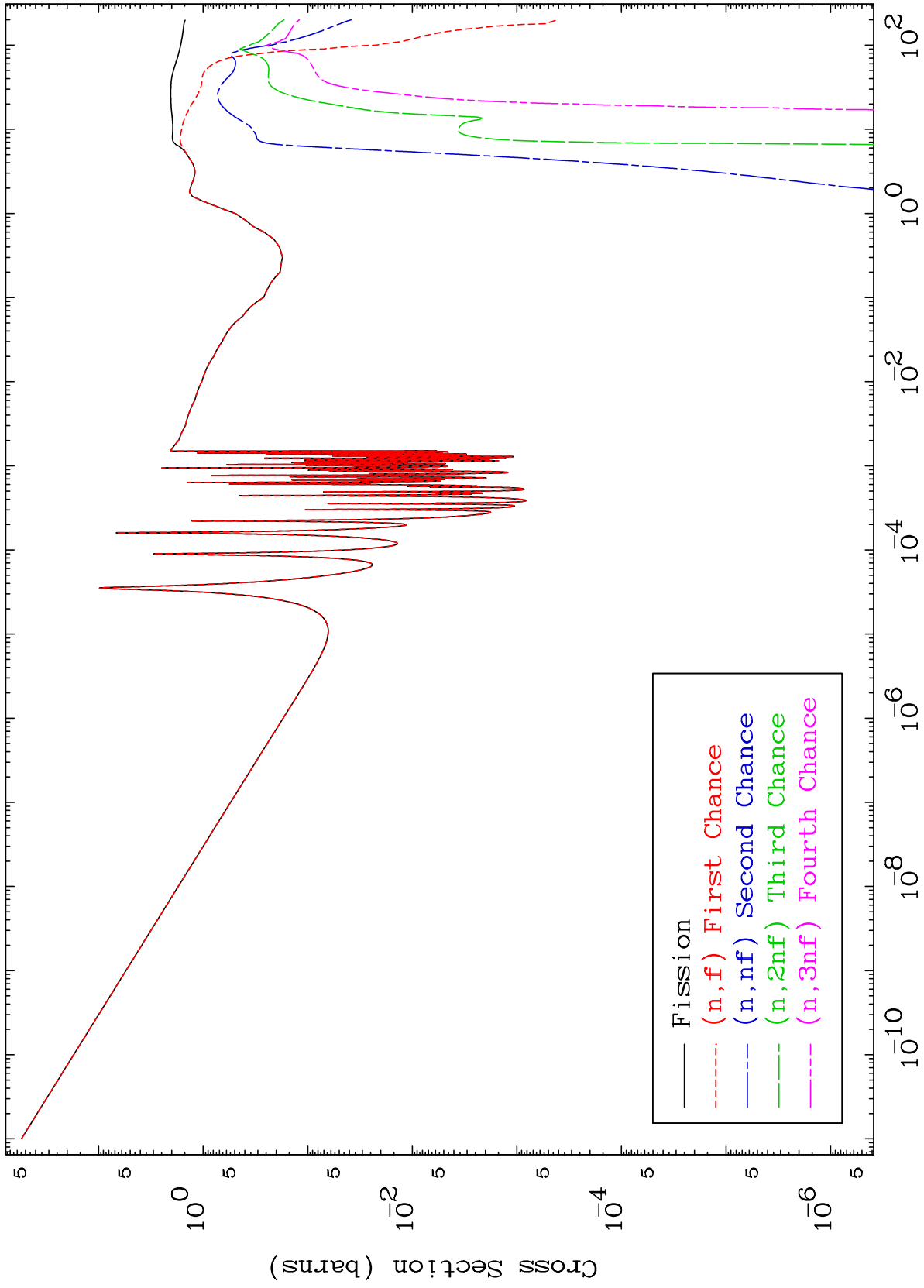
100-Fm-256



MAT 9937

Fission
293 Kelvin Cross Sections

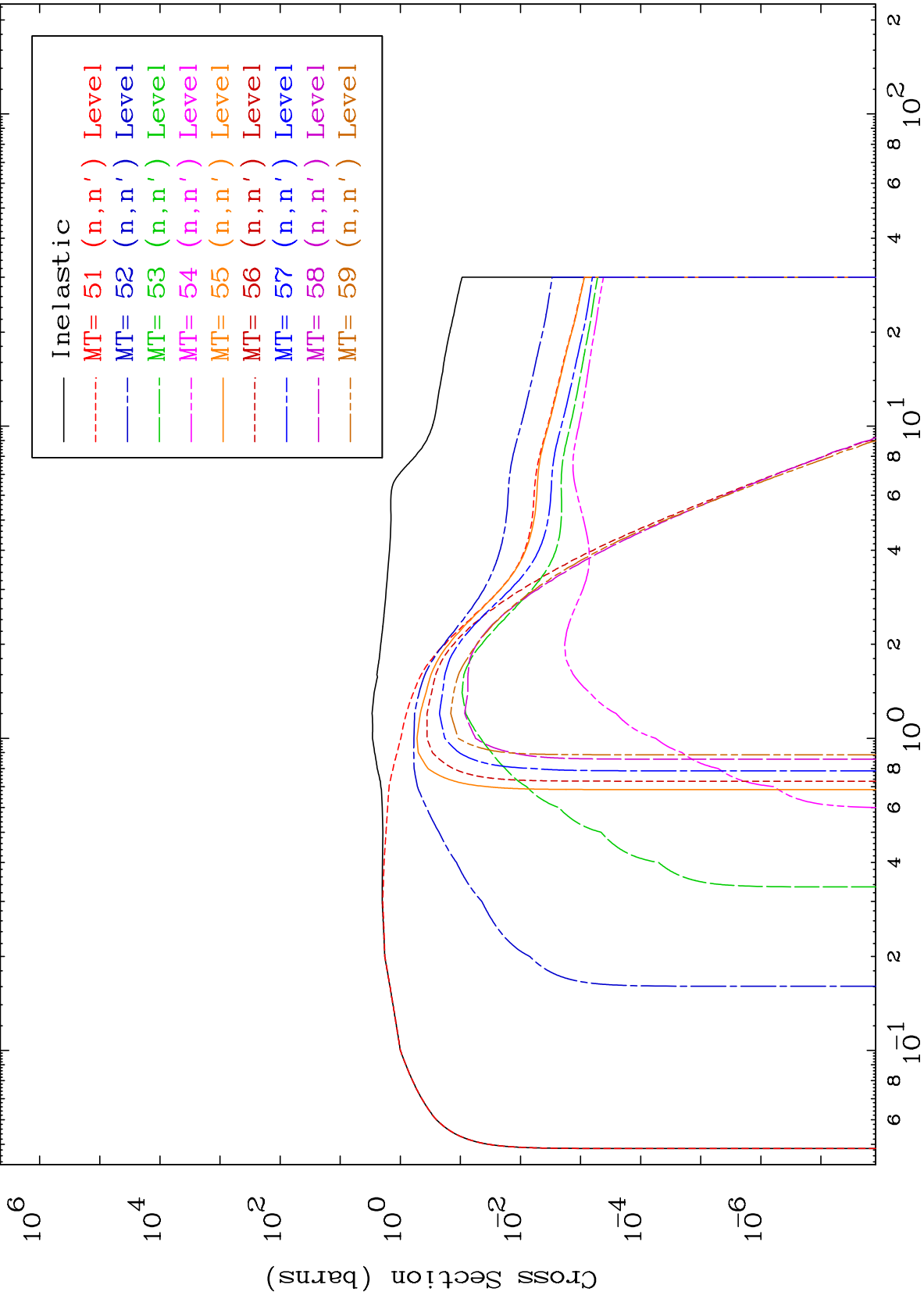
100-Fm-256



MAT 9937

(n,n') Level
293 Kelvin Cross Sections

100-Fm-256

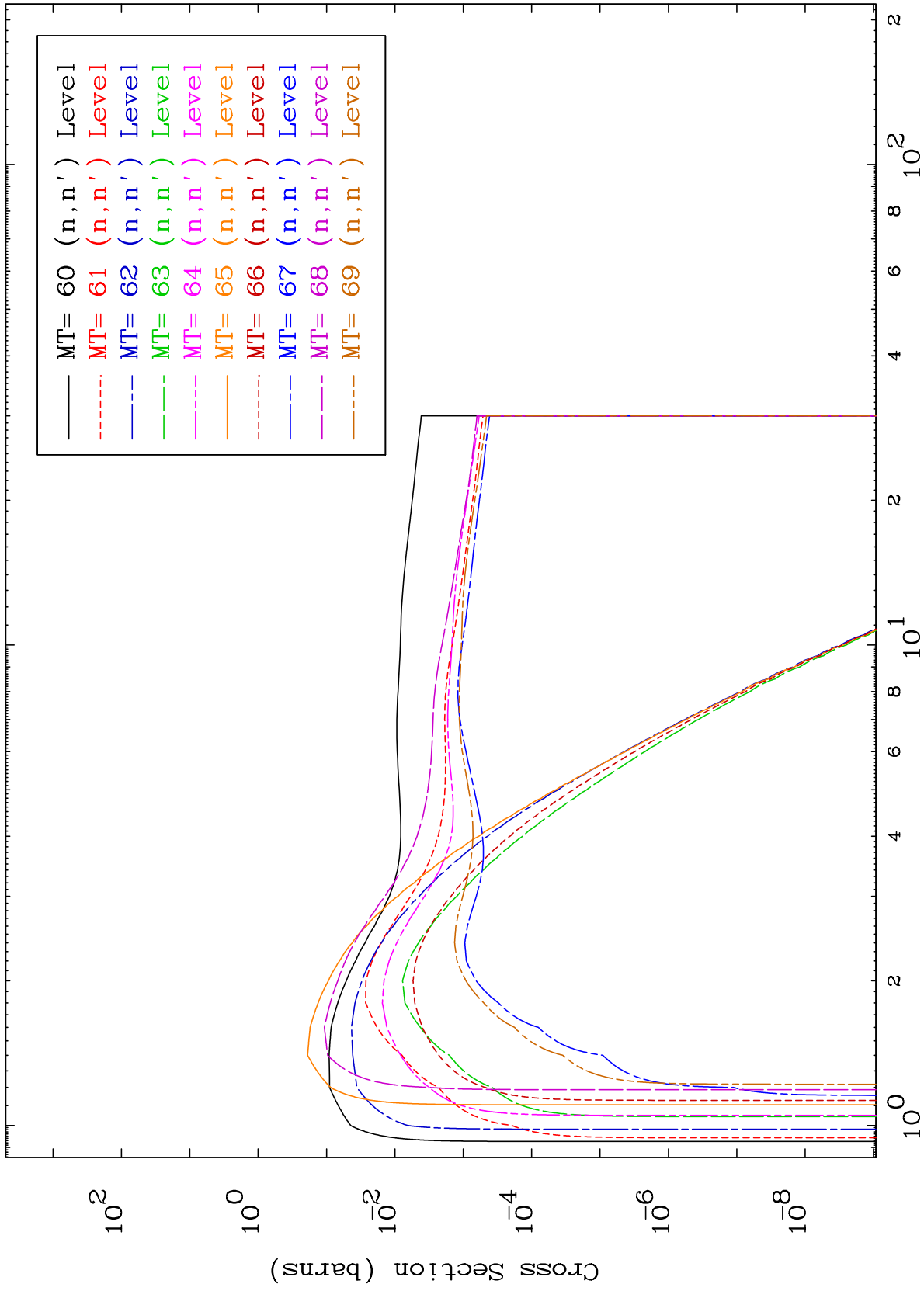


MAT 9937

(n,n') Level

100-Fm-256

293 Kelvin Cross Sections



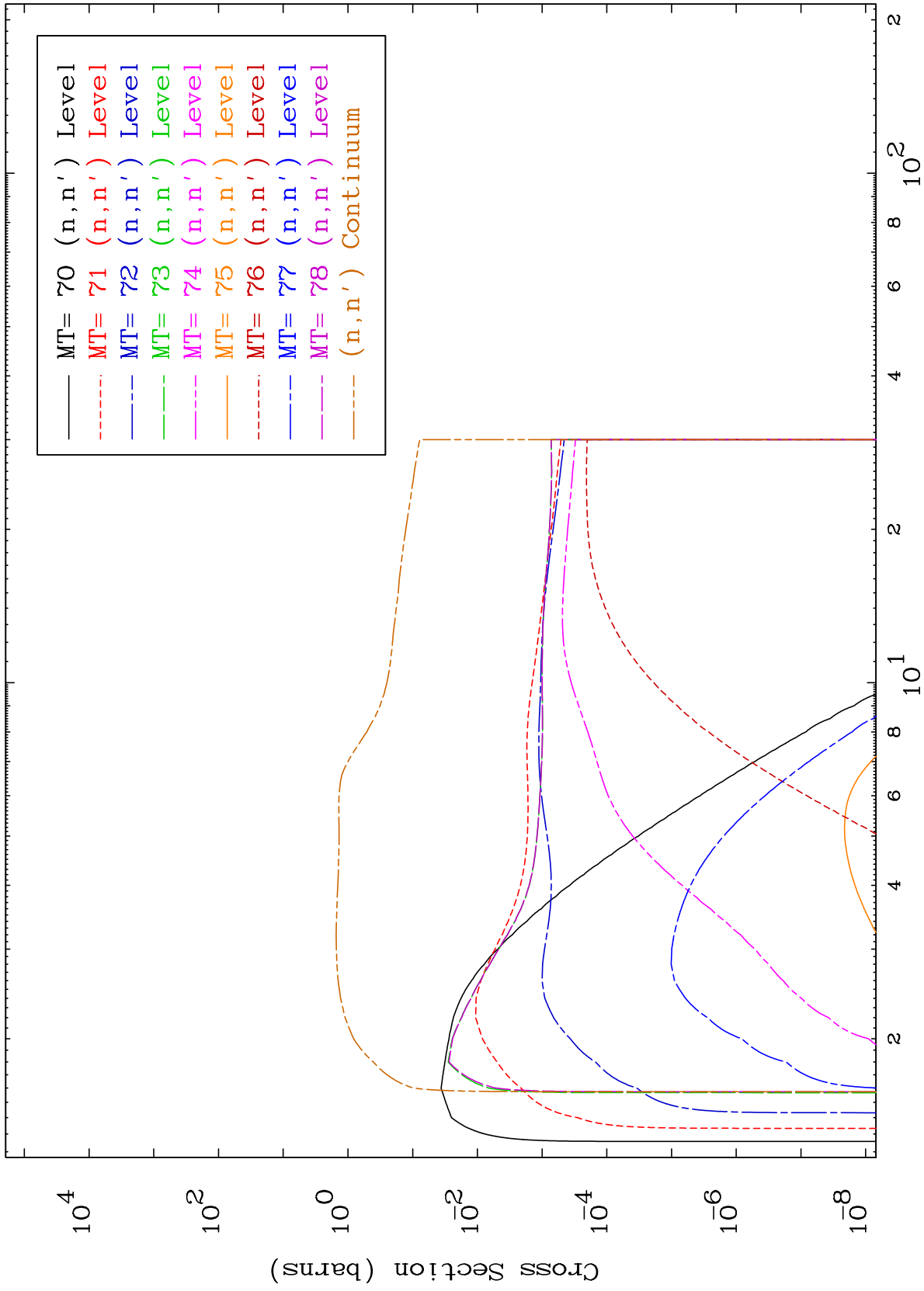
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,n') Level
293 Kelvin Cross Sections

100-Fm-256



10

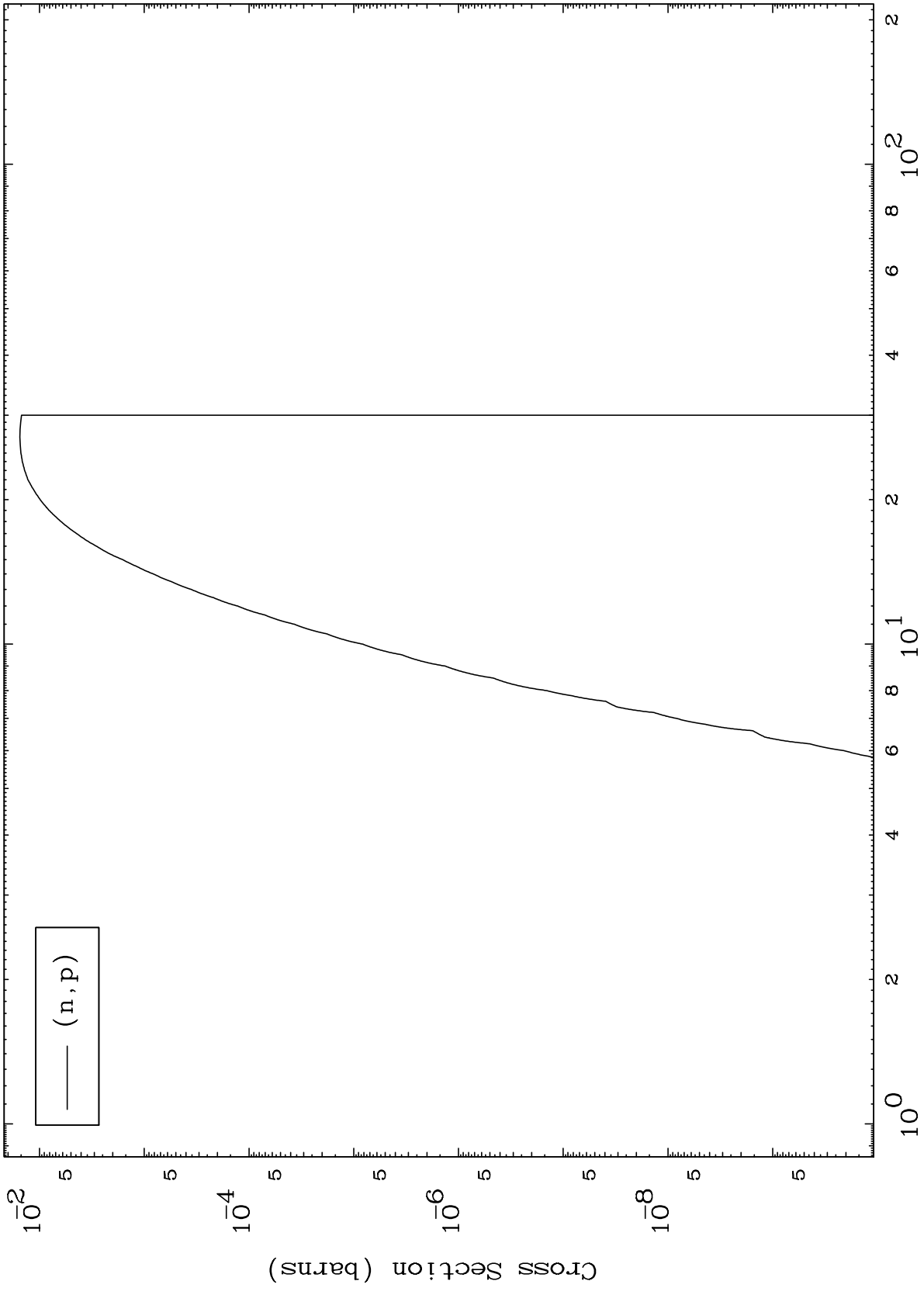
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,p) Levels
293 Kelvin Cross Sections

100-Fm-256

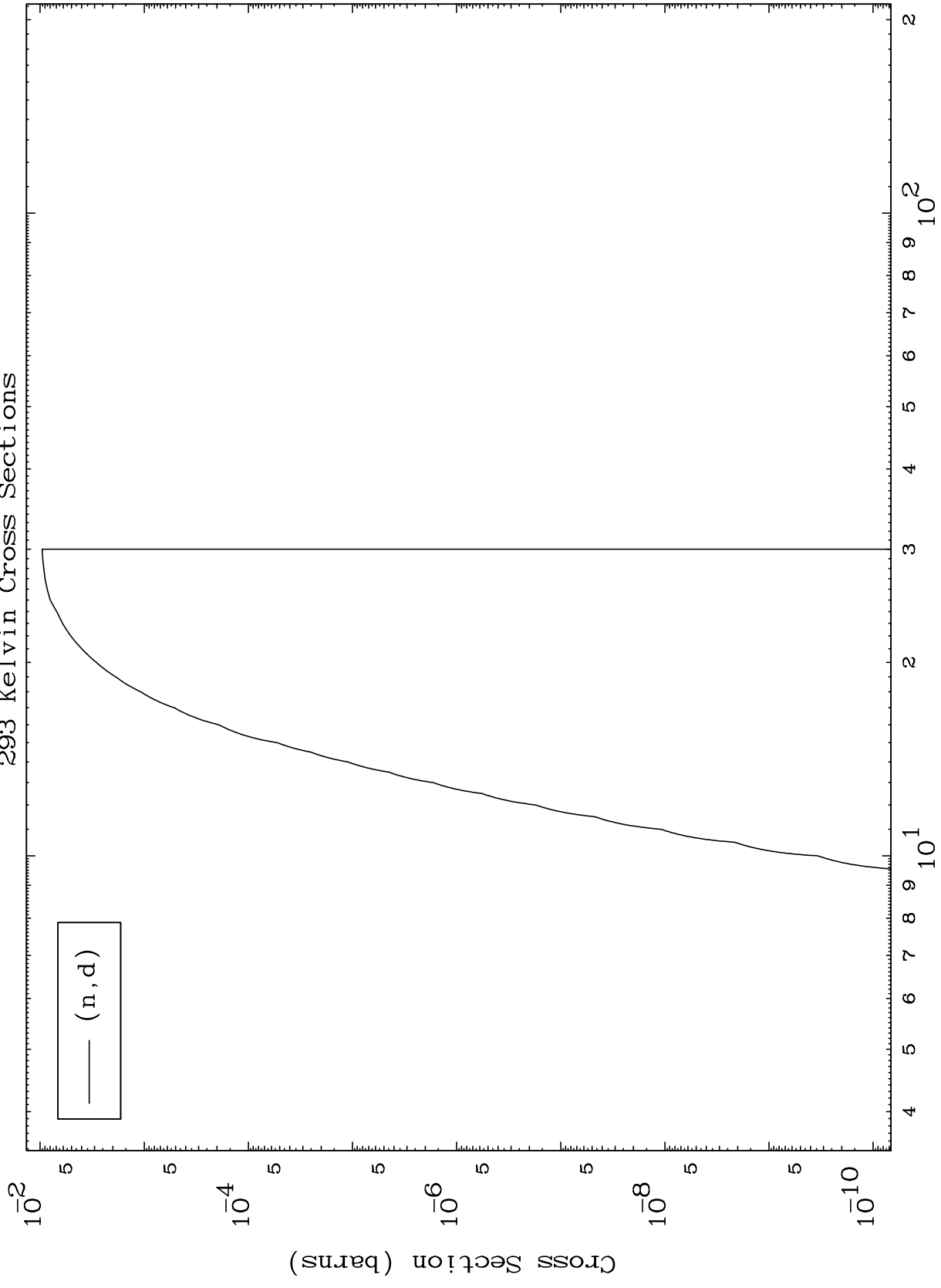


(n,p)

MAT 9937

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-256



12

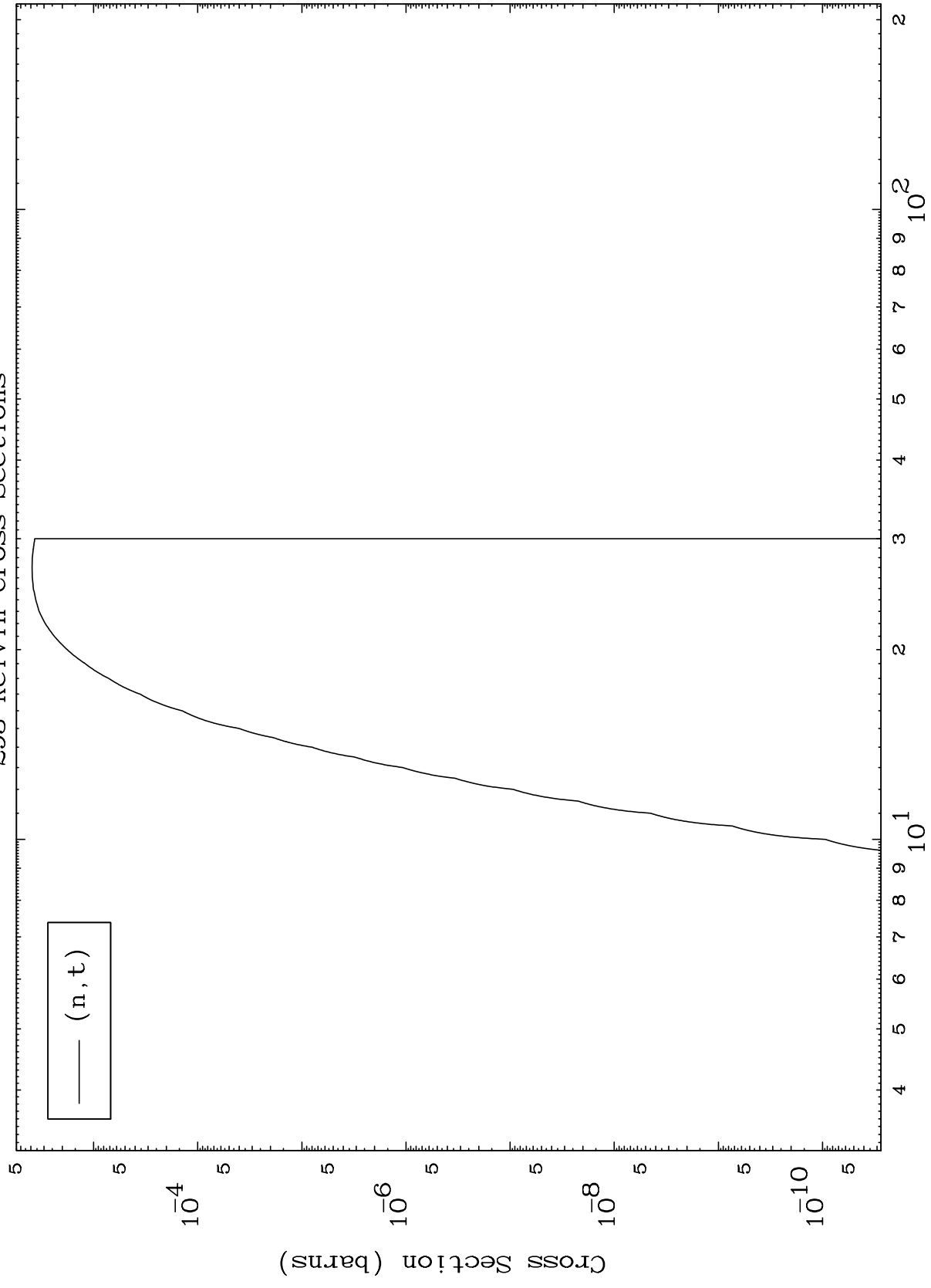
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-256



13

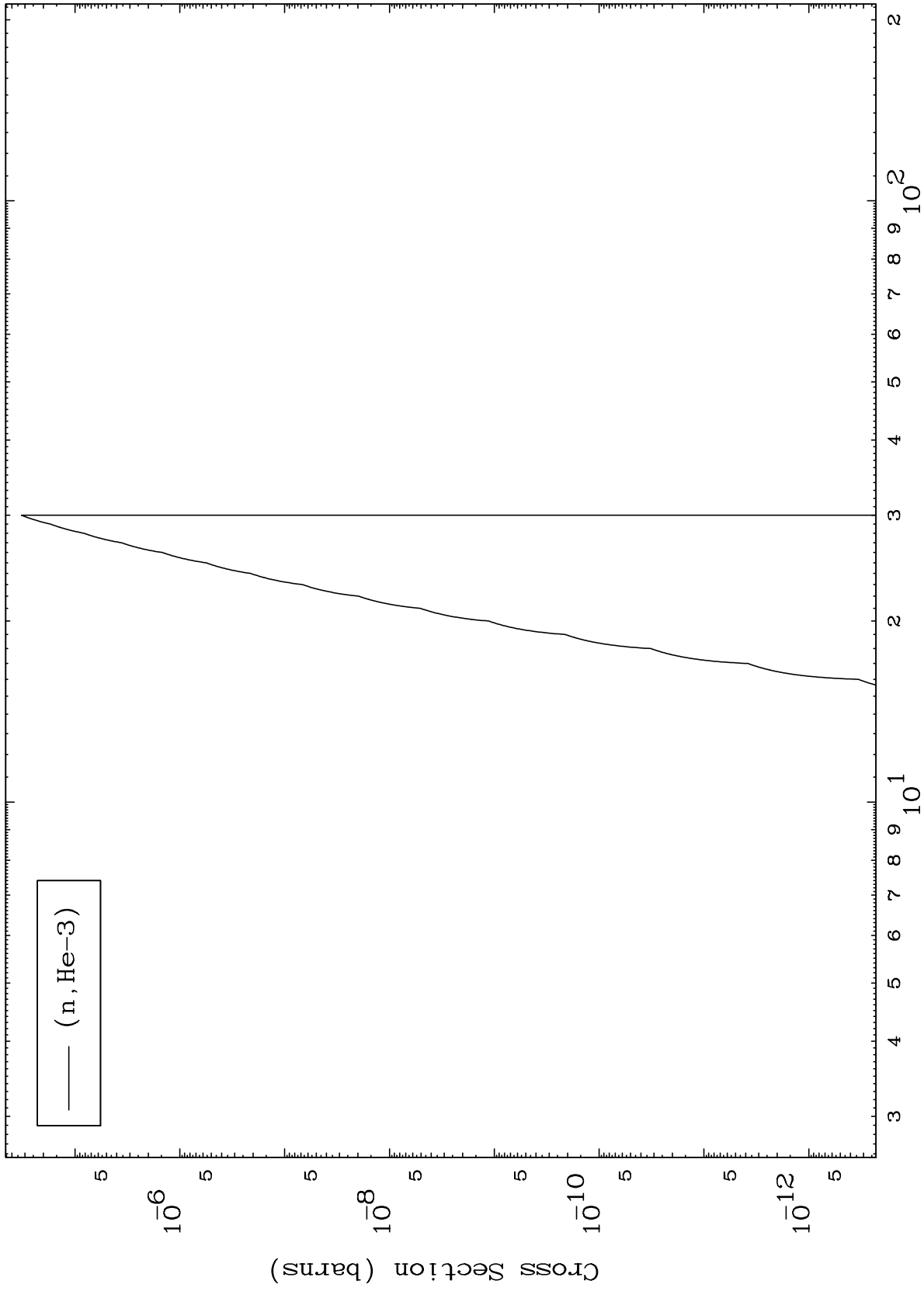
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,He3) Levels
293 Kelvin Cross Sections

100-Fm-256



14

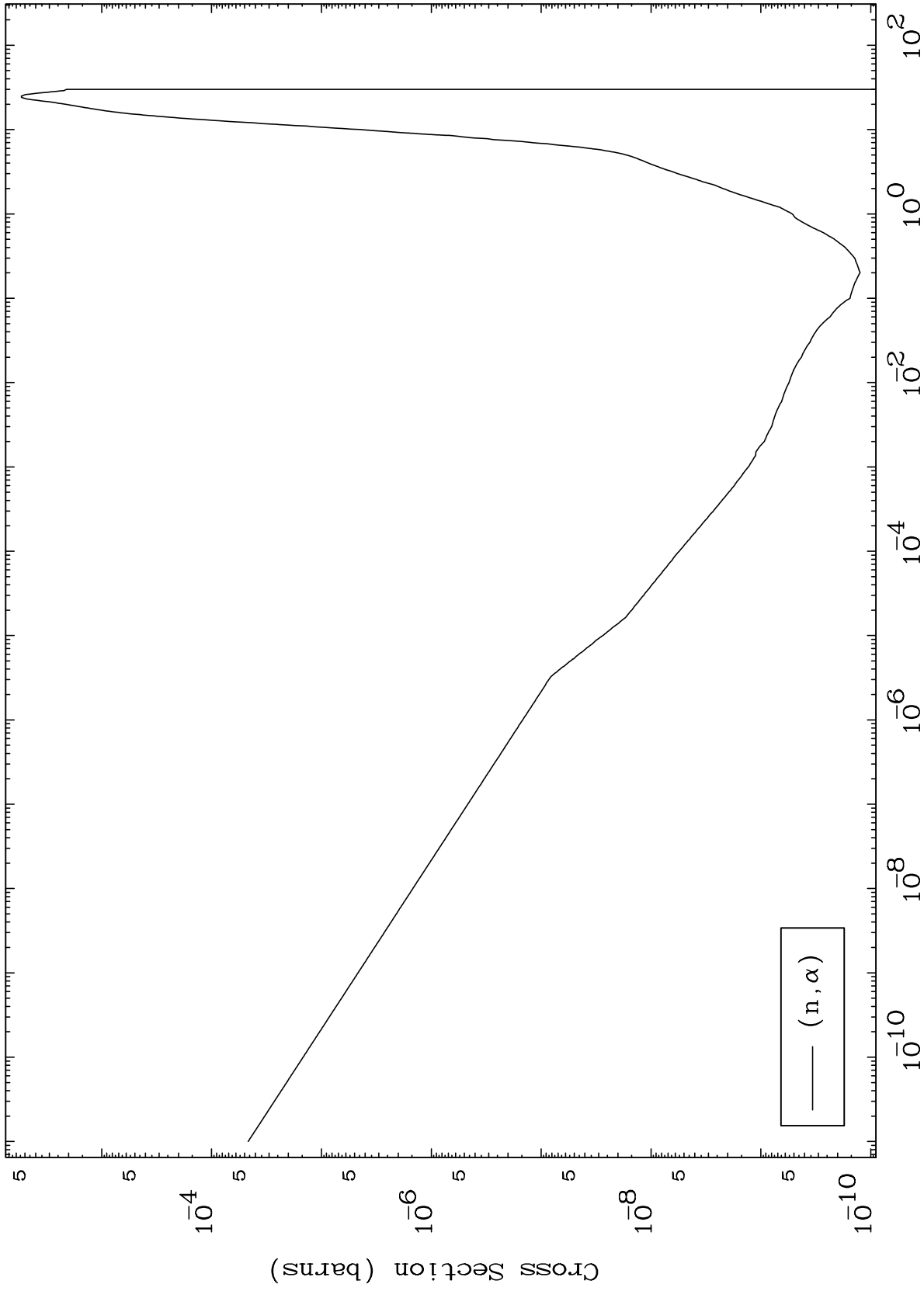
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,α) Levels
293 Kelvin Cross Sections

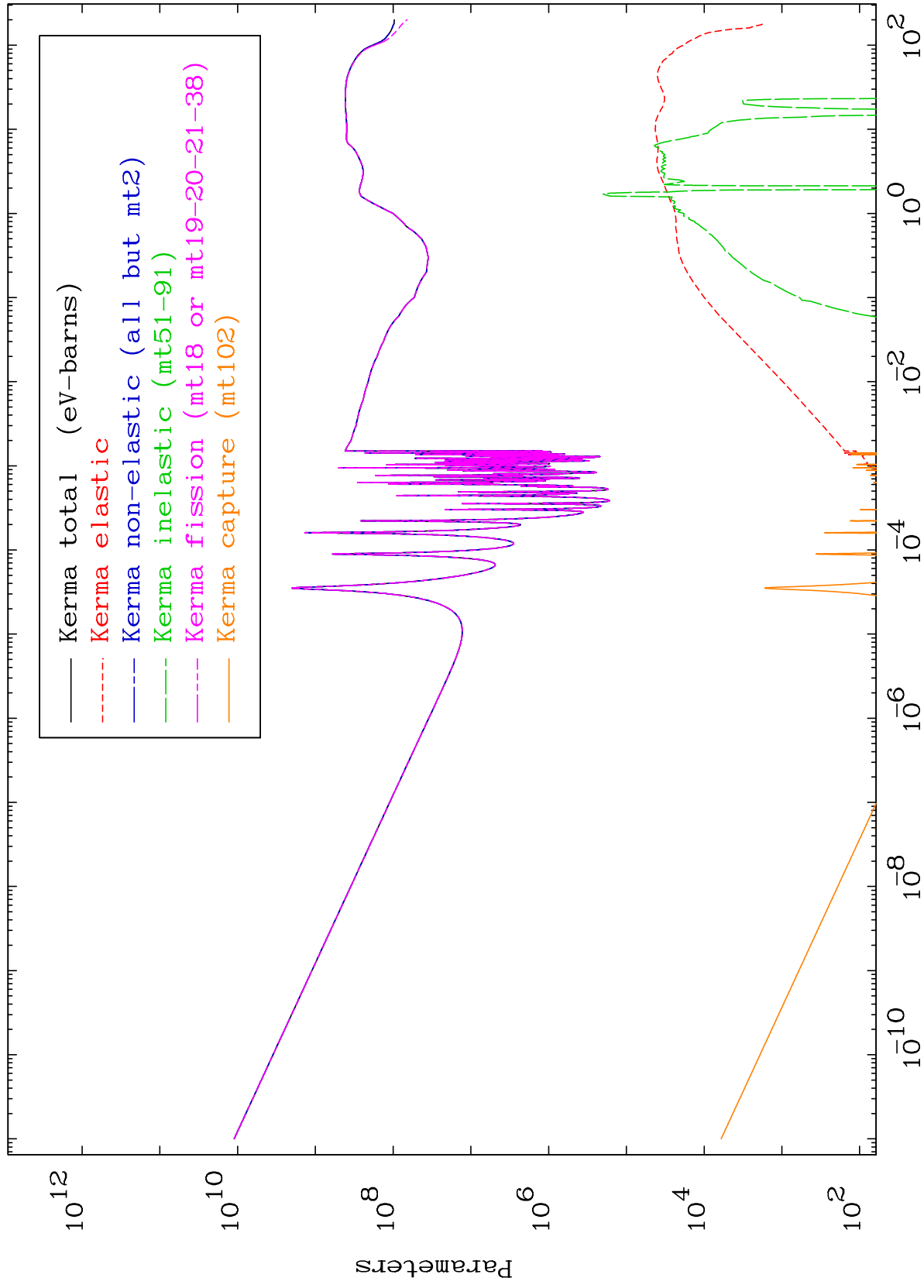
100-Fm-256

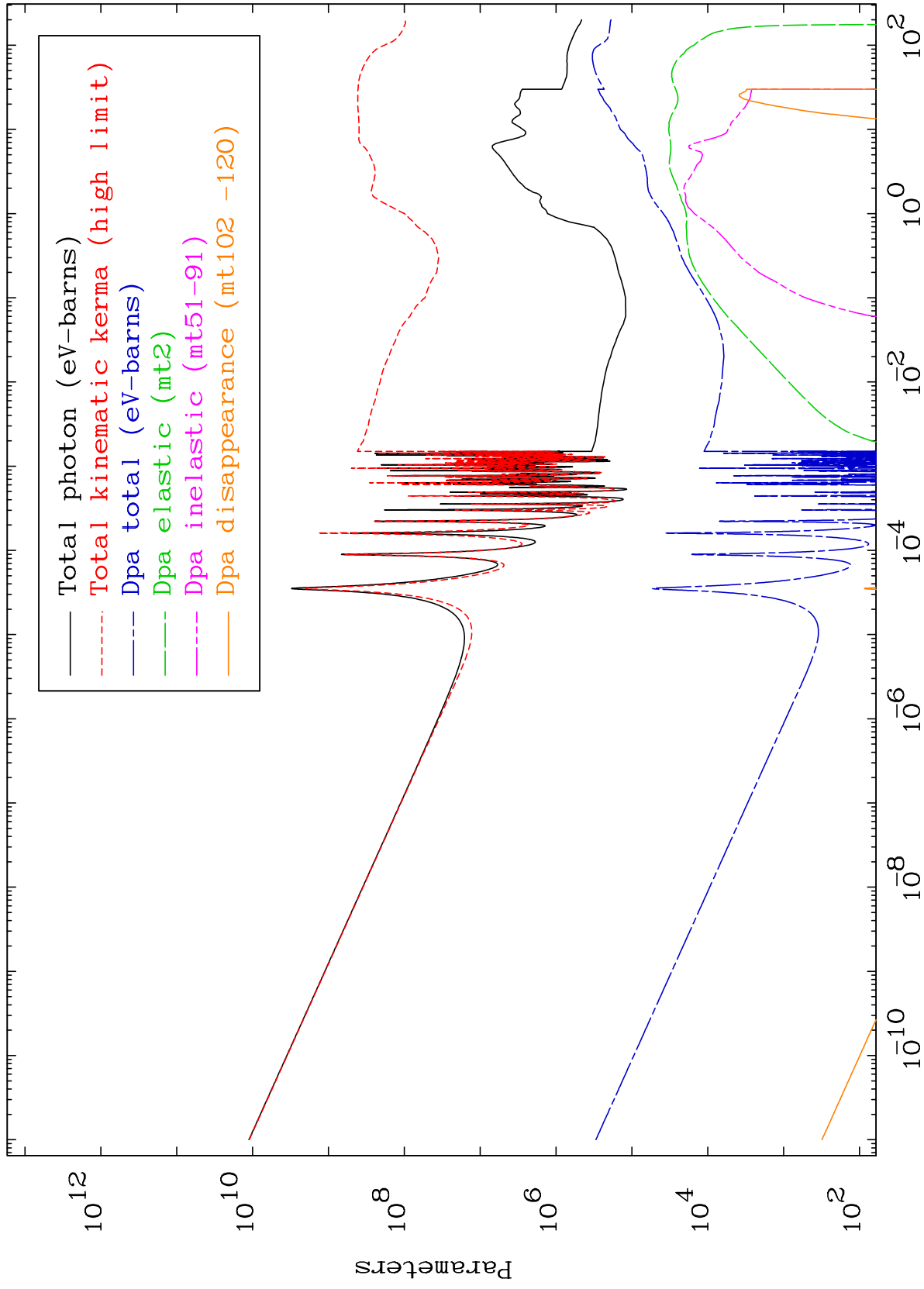


15

Incident Energy (MeV)

100-Fm-256

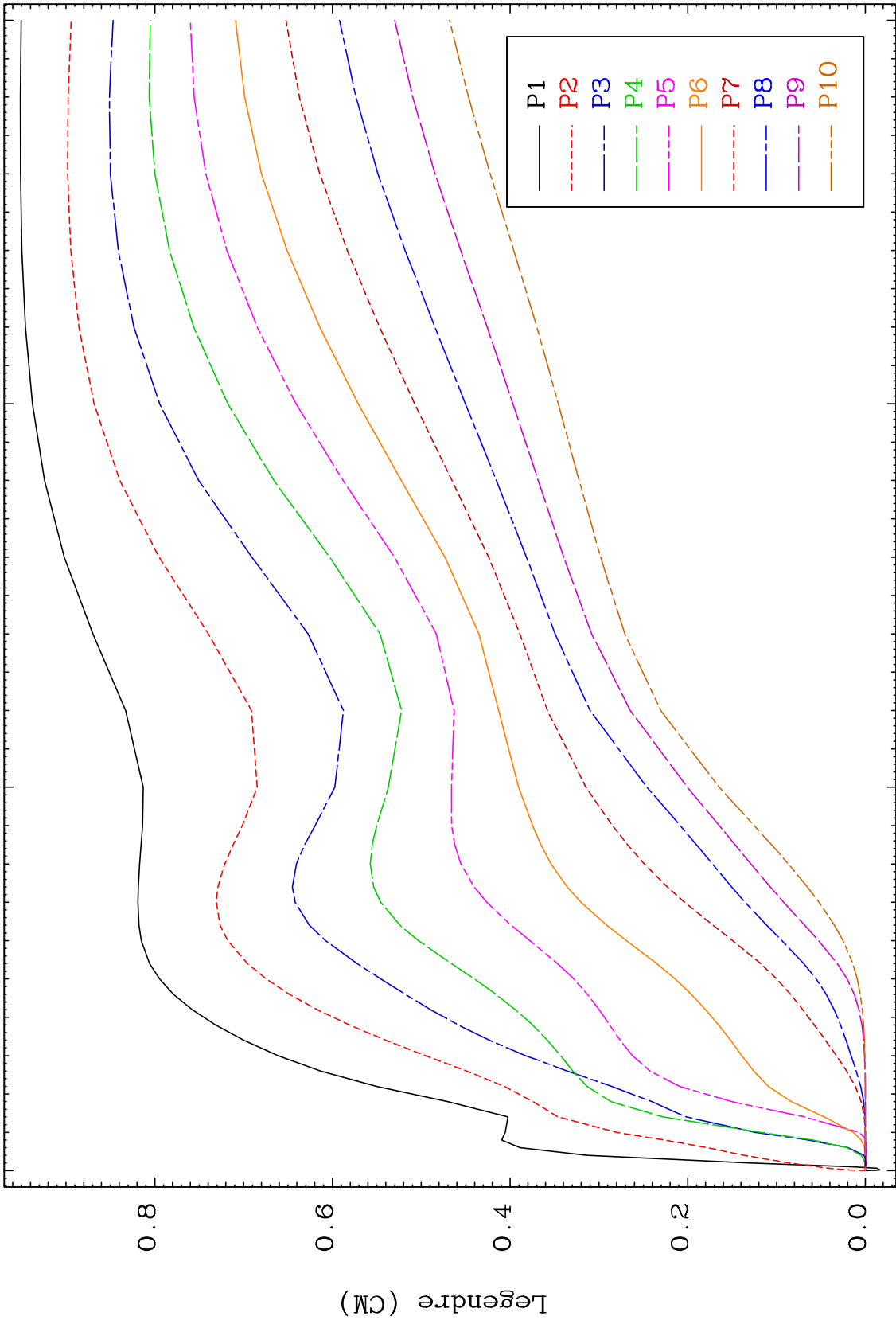




MAT 9937

Elastic Legendre Coefficients

100-Fm-256



18

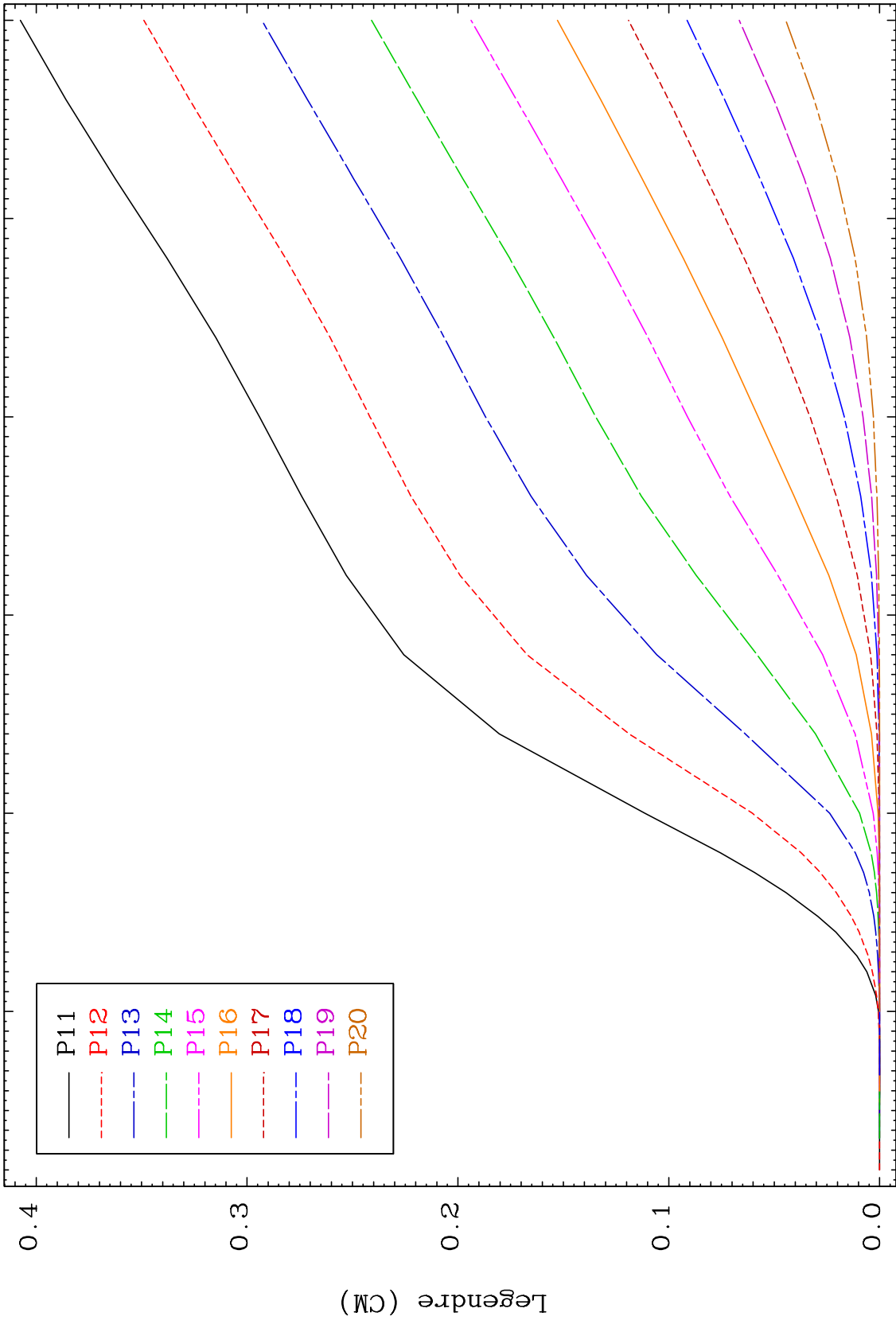
Incident Energy (MeV)

100-Fm-256

MAT 9937

Elastic Legendre Coefficients

100-Fm-256



19

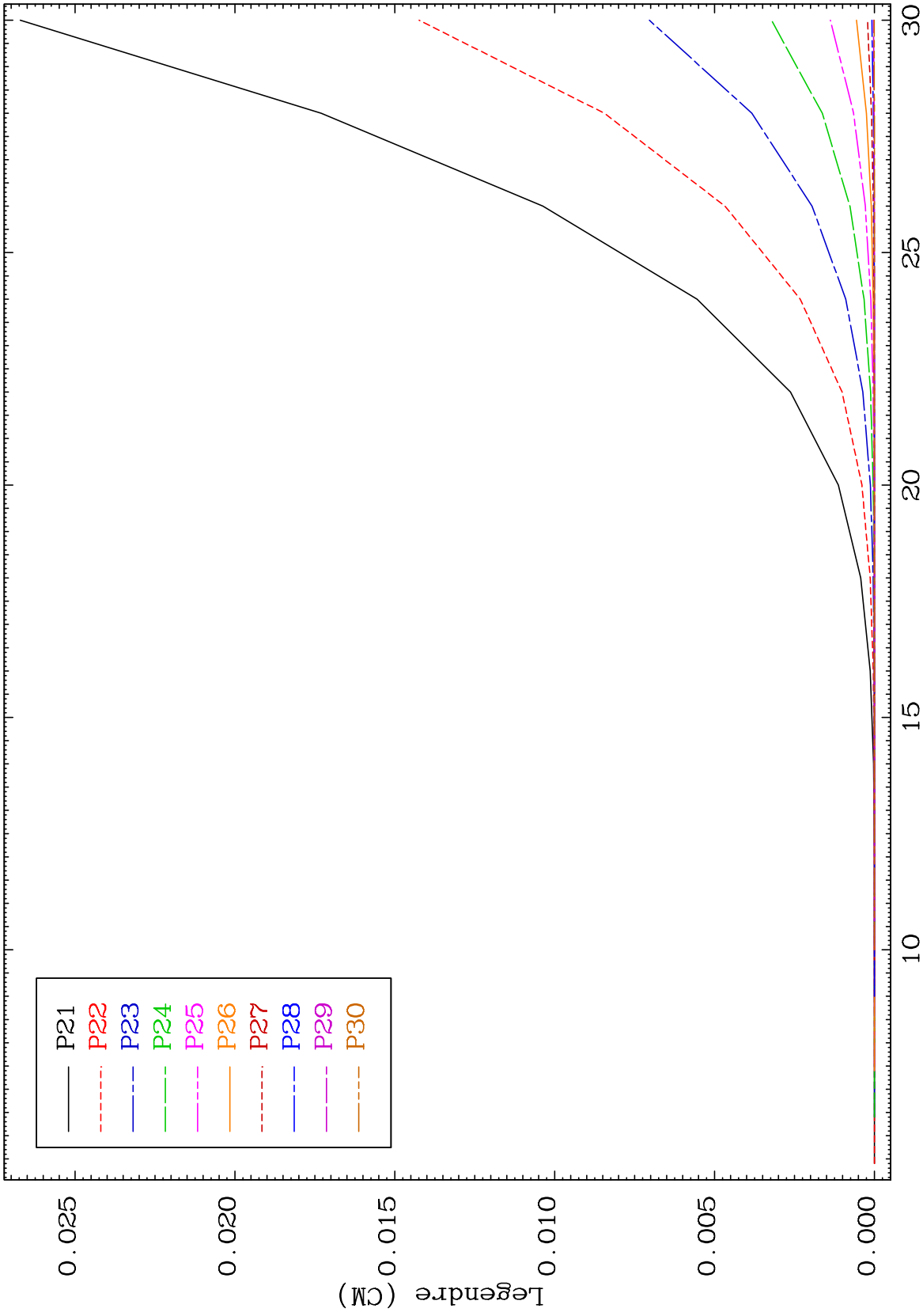
Incident Energy (MeV)

100-Fm-256

MAT 9937

Elastic
Legendre Coefficients

100-Fm-256



20

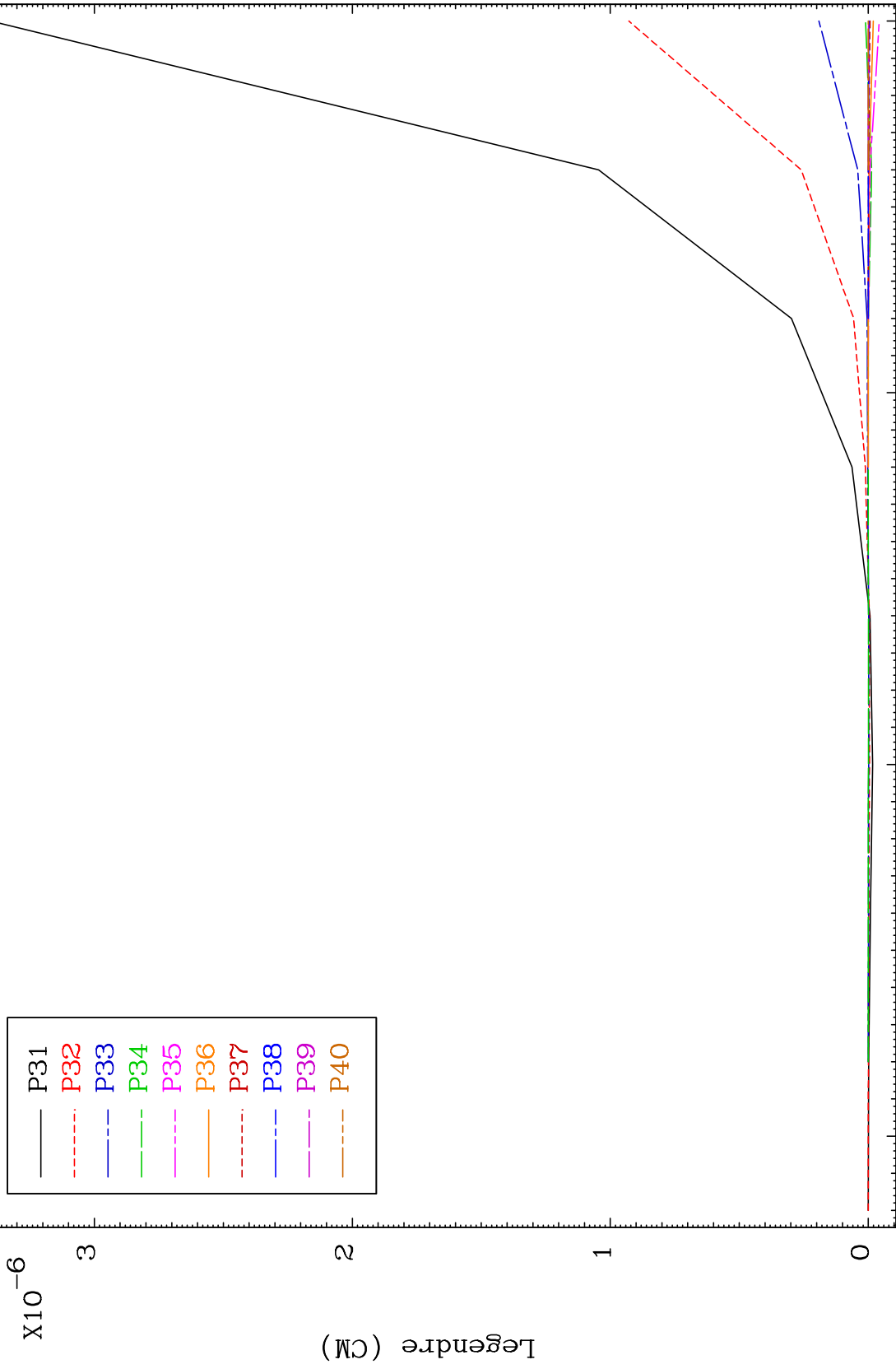
Incident Energy (MeV)

100-Fm-256

MAT 9937

Elastic Legendre Coefficients

100-Fm-256



21

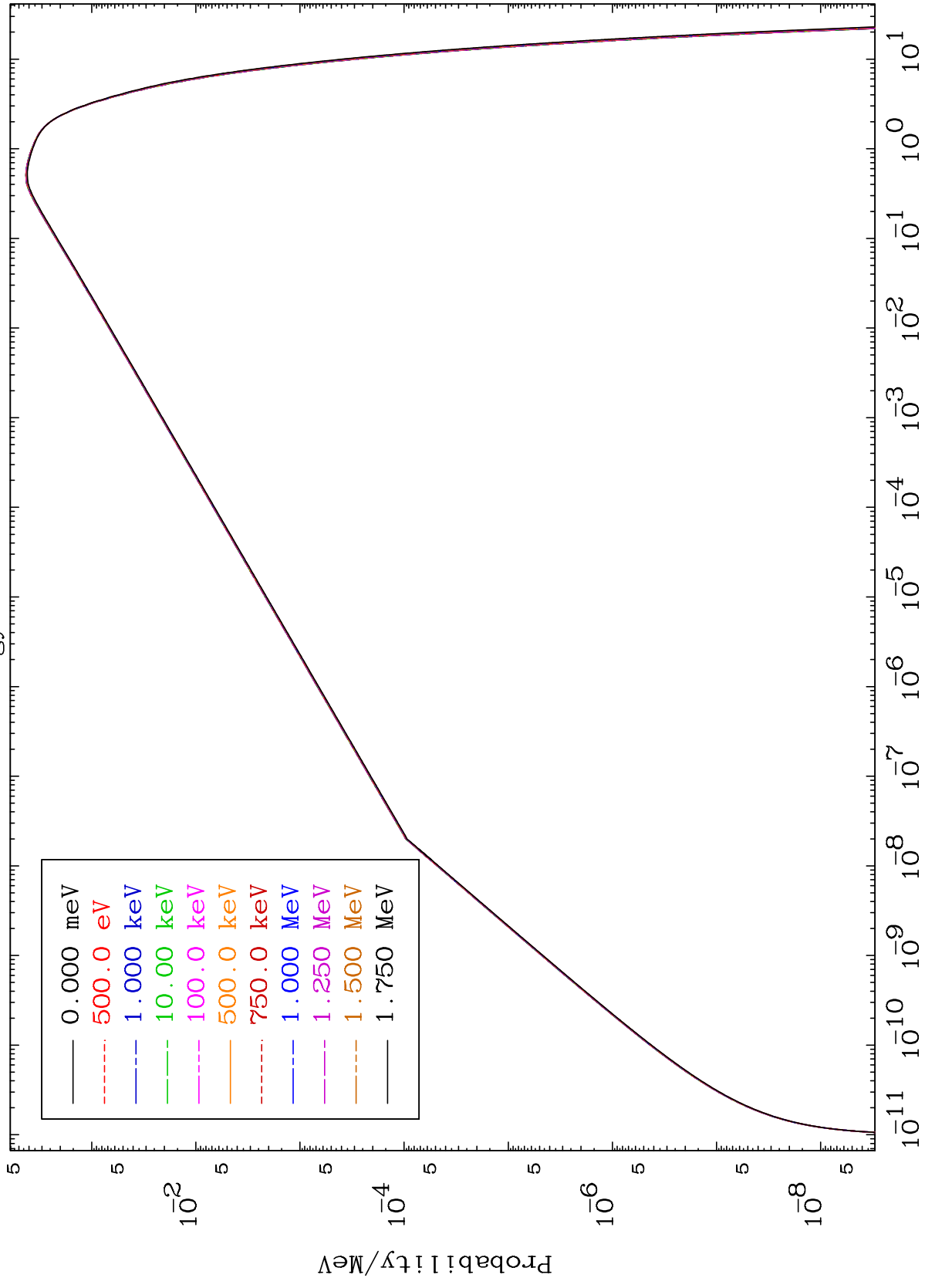
Incident Energy (MeV)

100-Fm-256

MAT 9937

Fission Energy Distribution

100-Fm-256

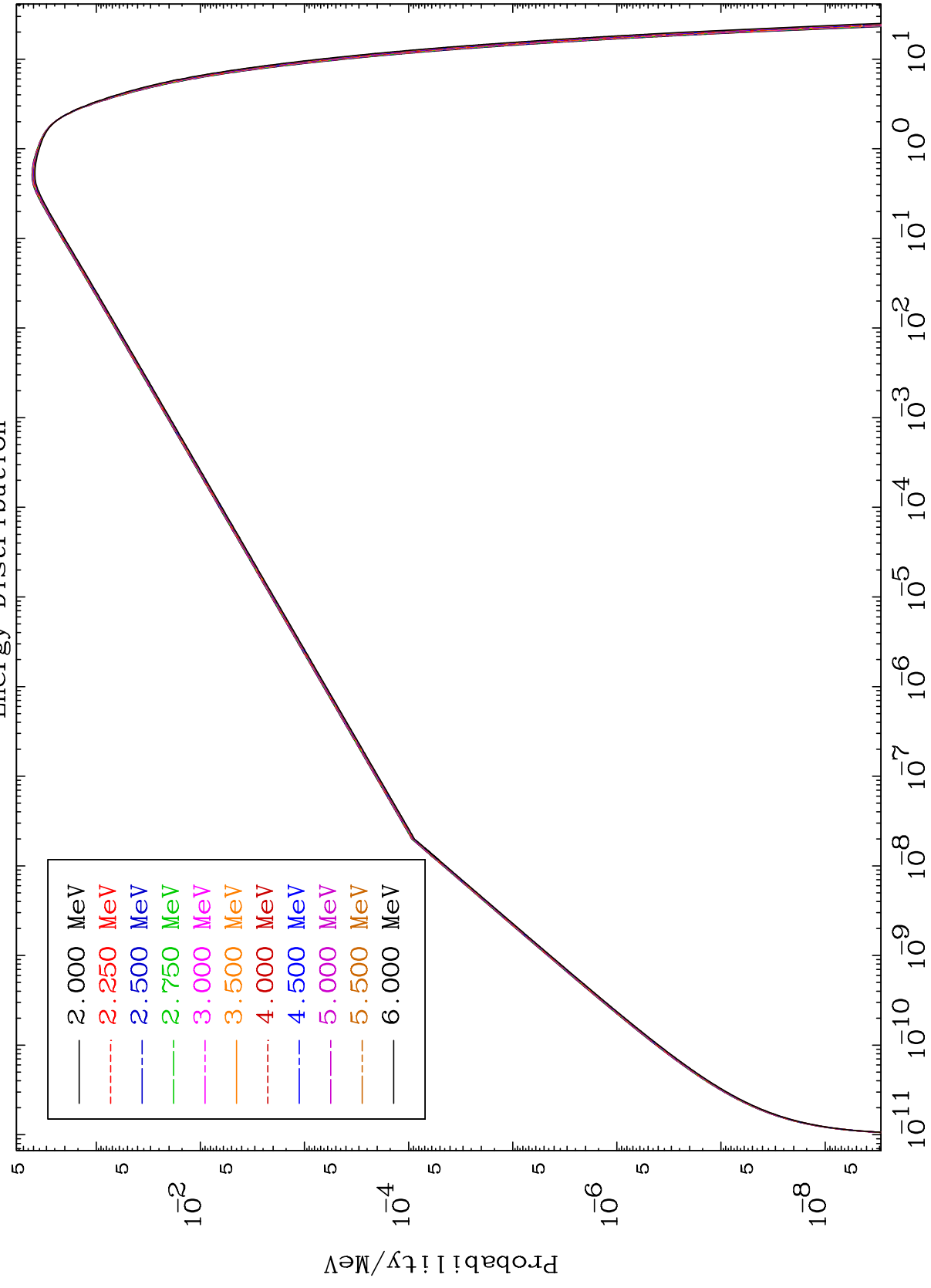


100-Fm-256

MAT 9937

Fission Energy Distribution

100-Fm-256

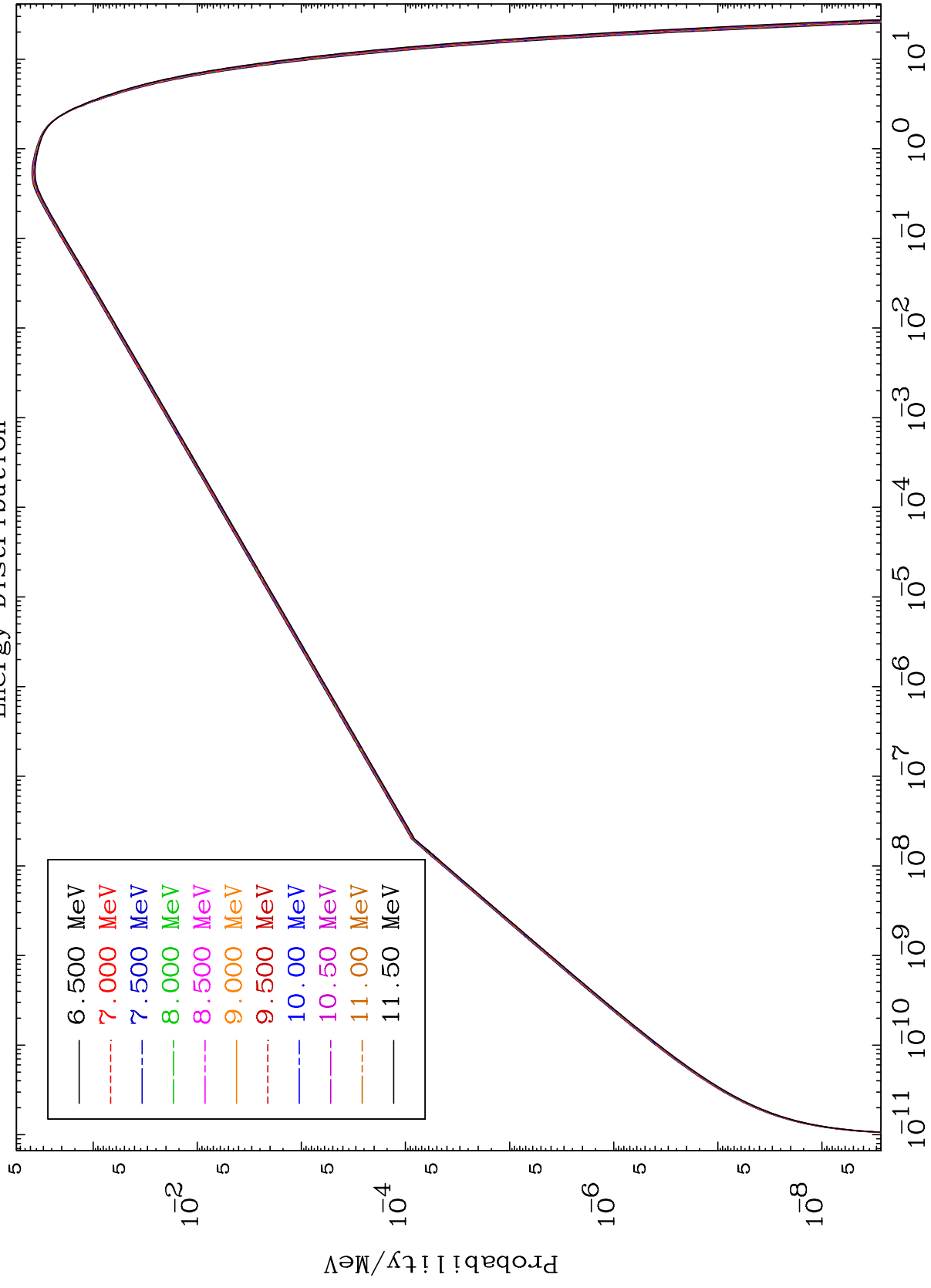


100-Fm-256

MAT 9937

Fission Energy Distribution

100-Fm-256

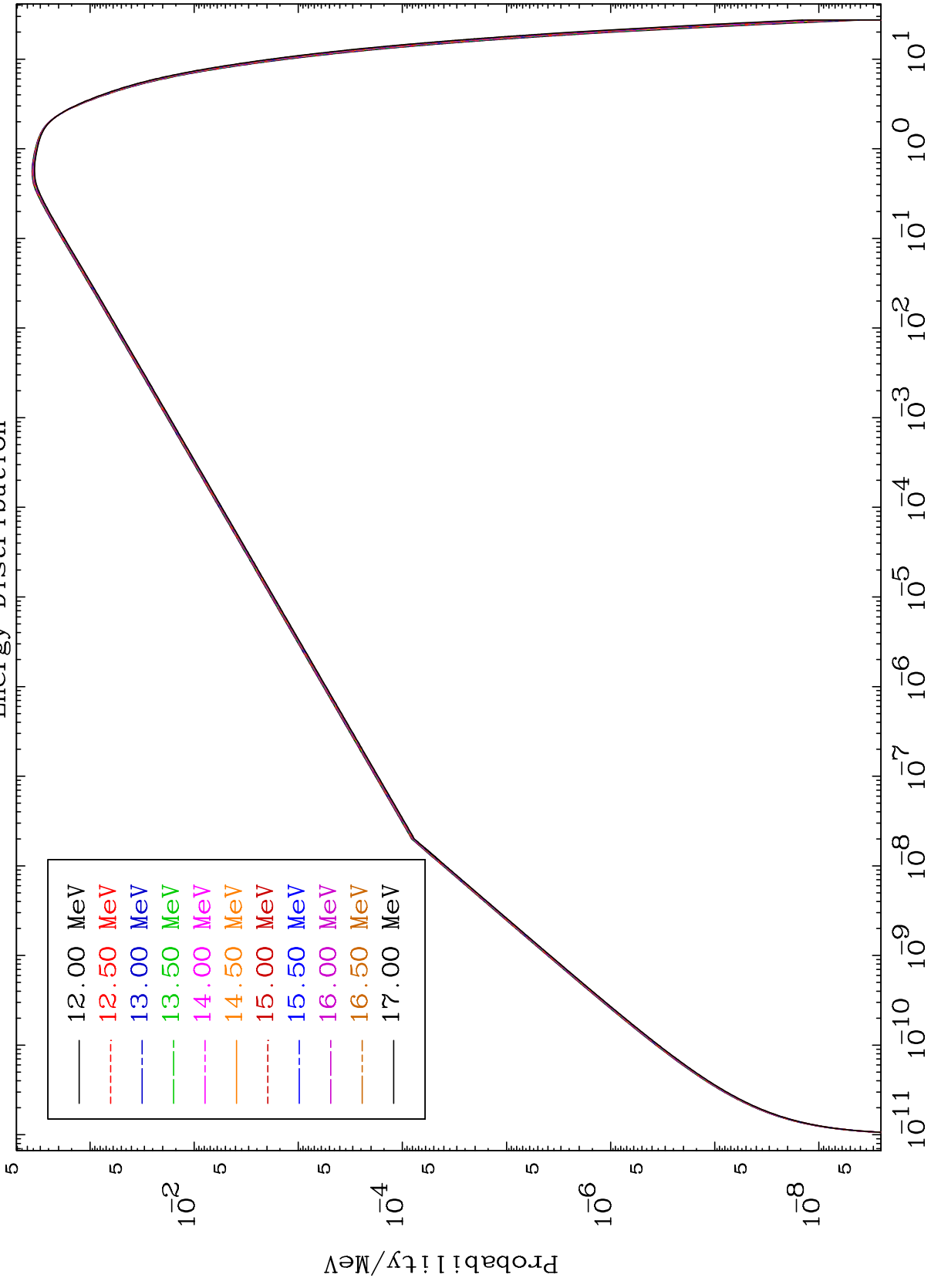


100-Fm-256

MAT 9937

Fission Energy Distribution

100-Fm-256

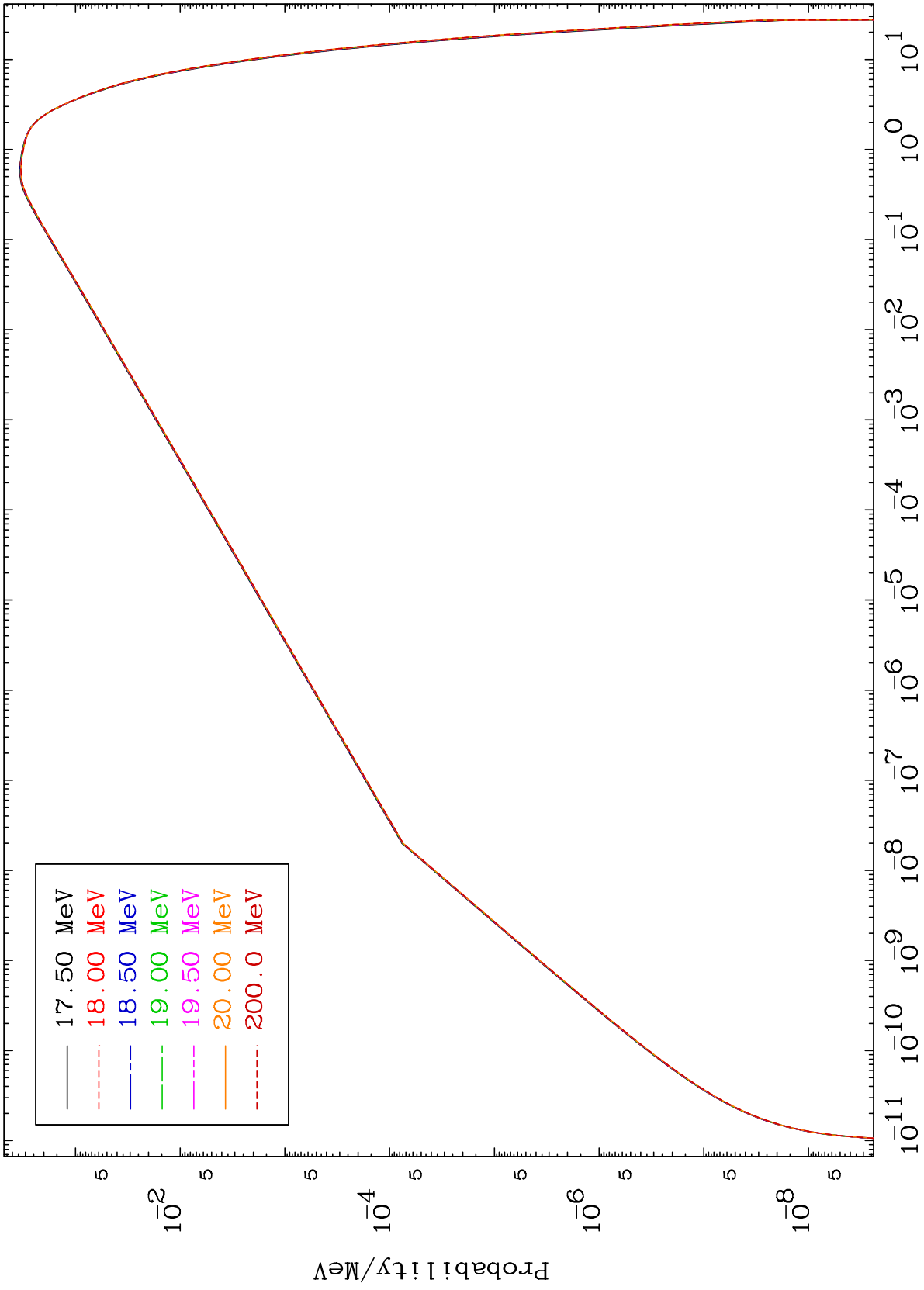


100-Fm-256

MAT 9937

Fission Energy Distribution

100-Fm-256



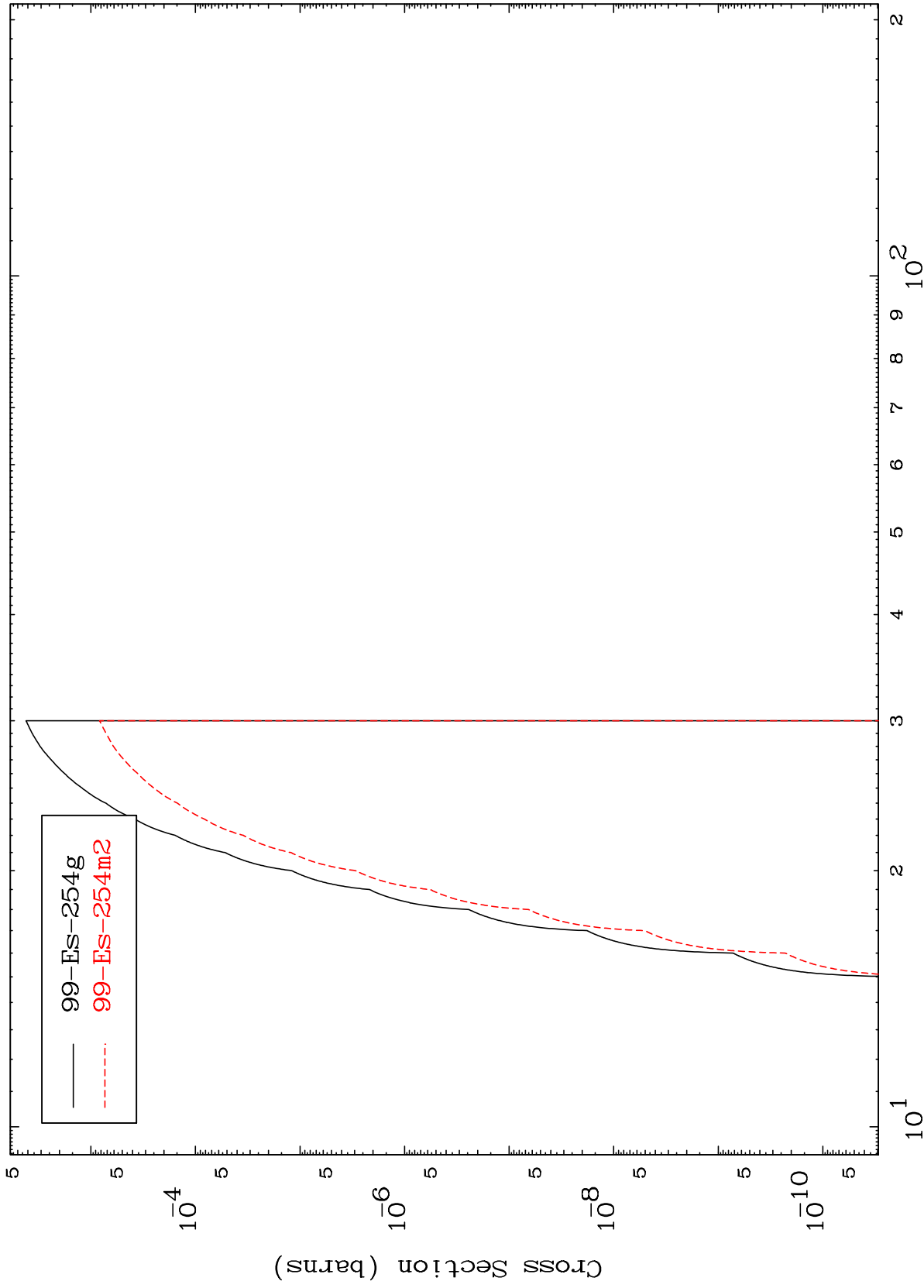
100-Fm-256

MAT 9937

(n,n') d

100-Fm-256

Radionuclide Production Cross Section



27

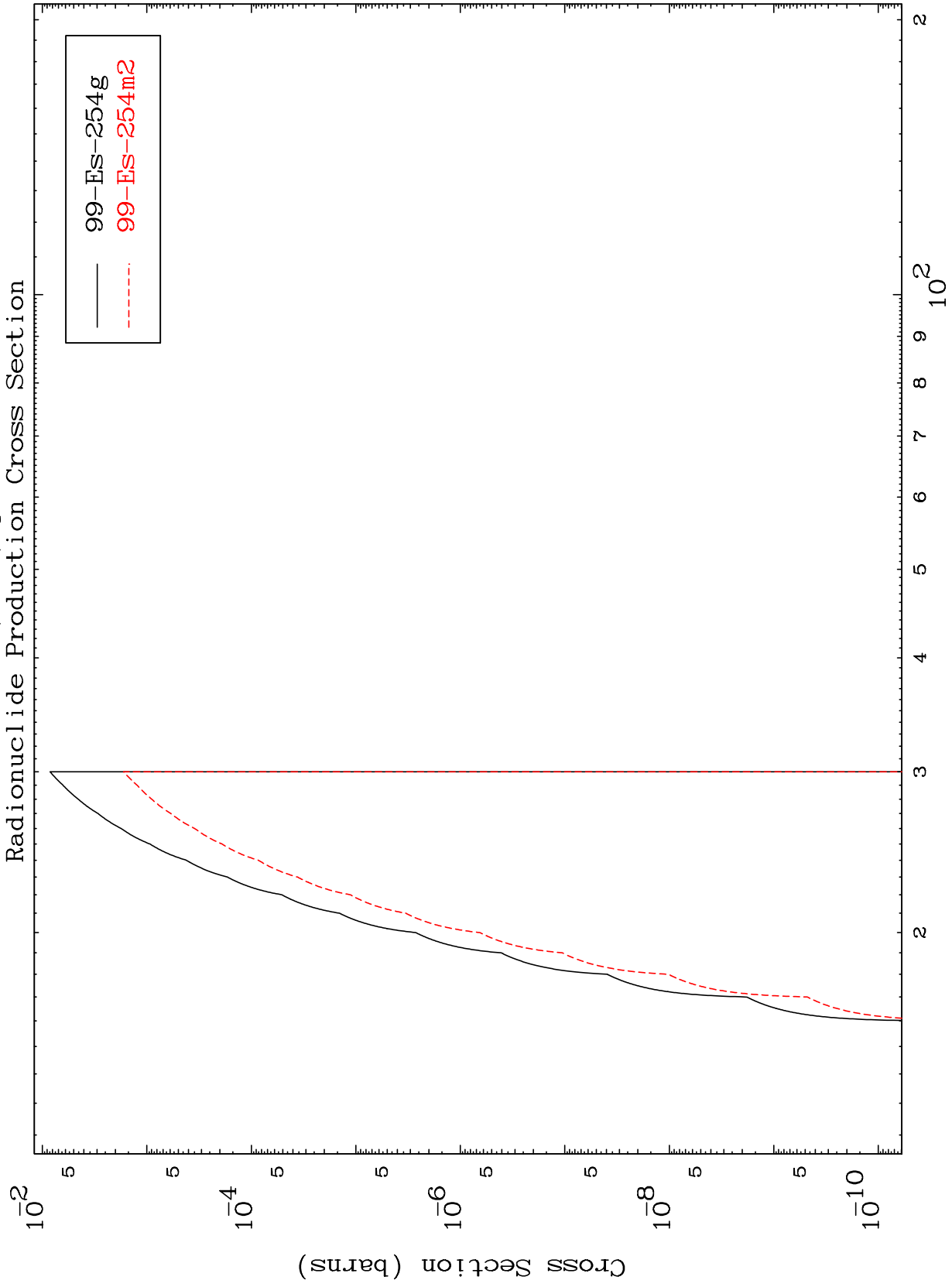
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,2n) p

100-Fm-256



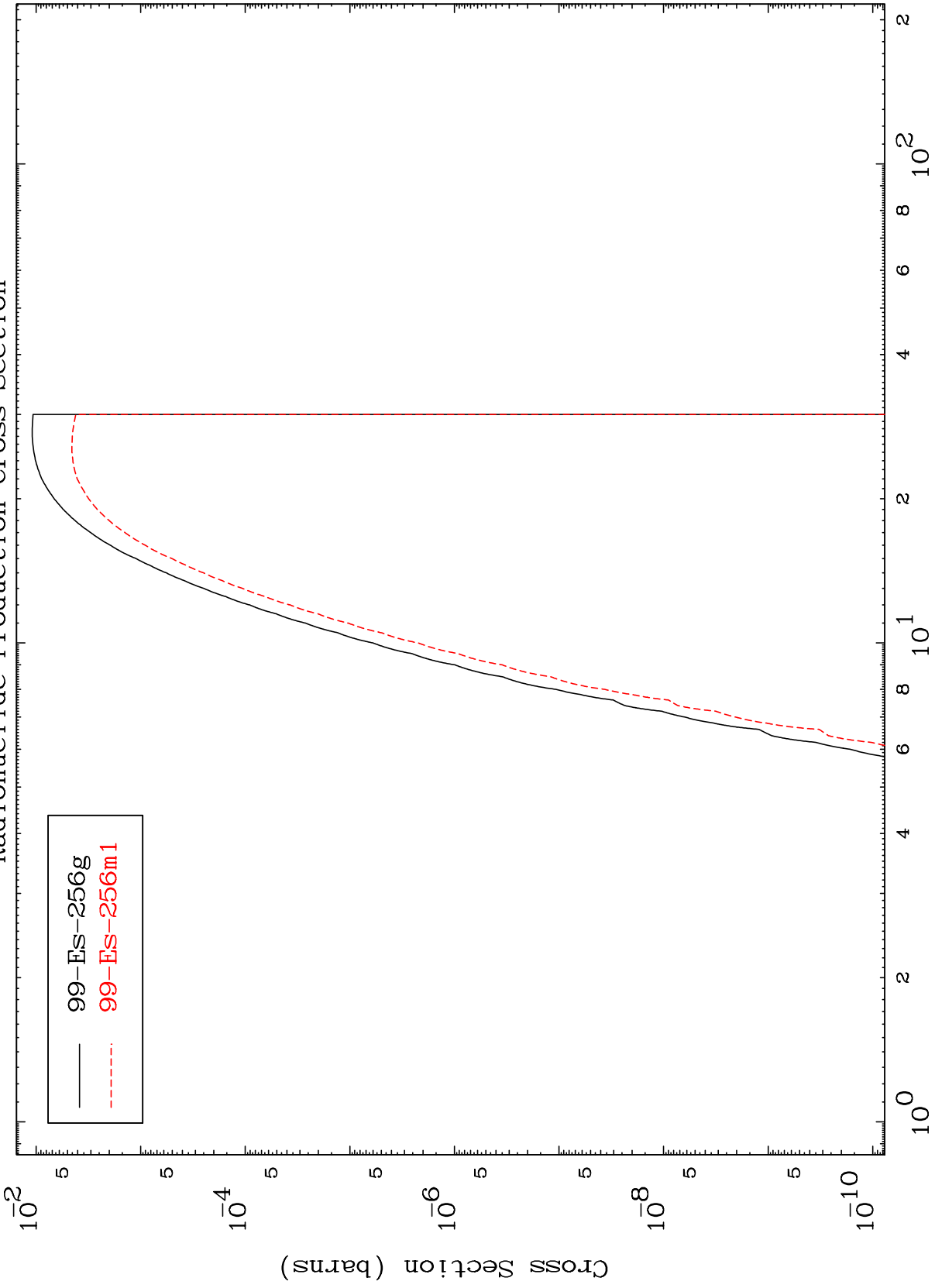
28

100-Fm-256

MAT 9937

(n,p)
Radionuclide Production Cross Section

100-Fm-256



29

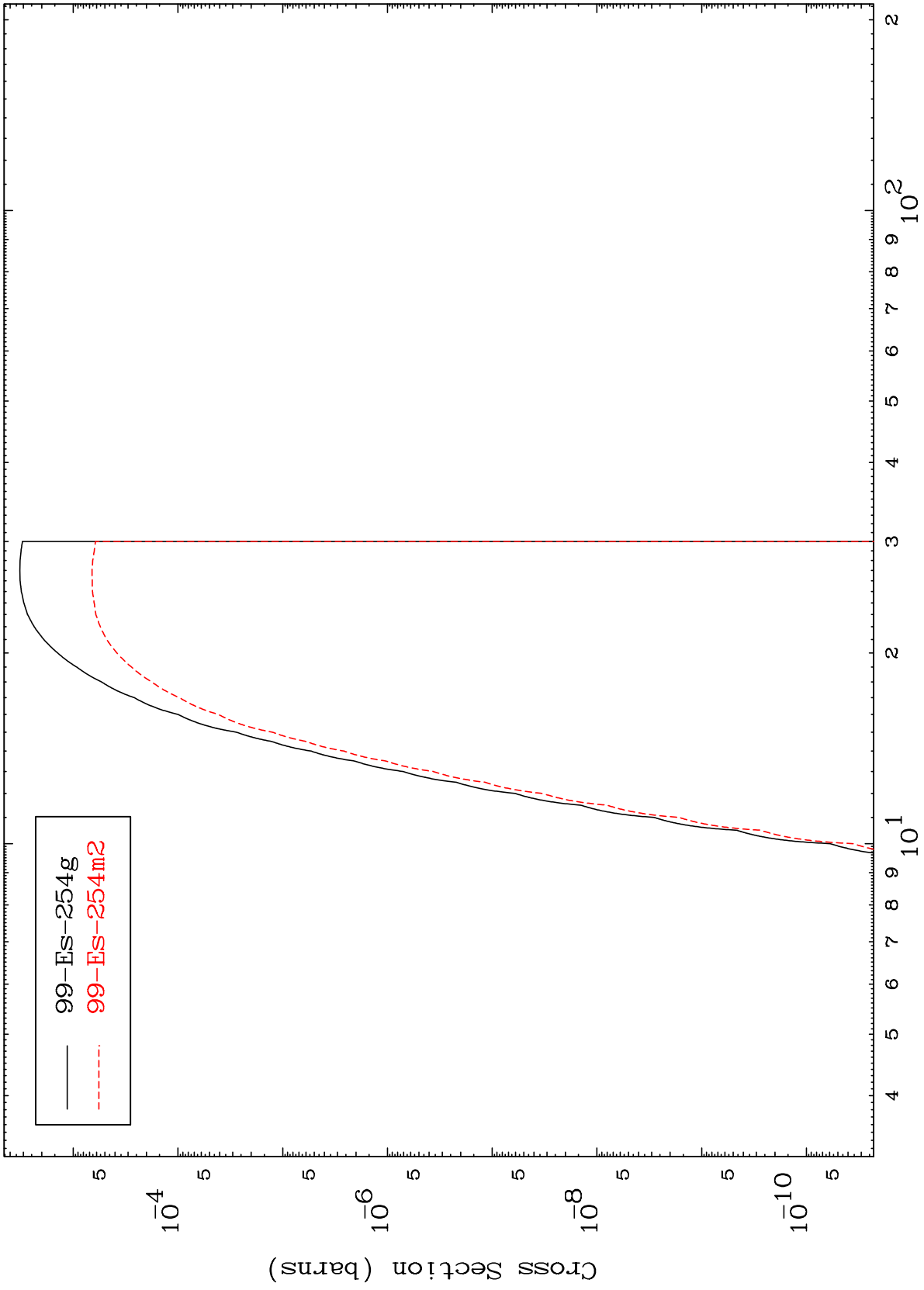
Incident Energy (MeV)

100-Fm-256

MAT 9937

100-Fm-256

(n, t)
Radionuclide Production Cross Section



30

100-Fm-256