

Program EVALPLOT
(Version 2018-1)

by

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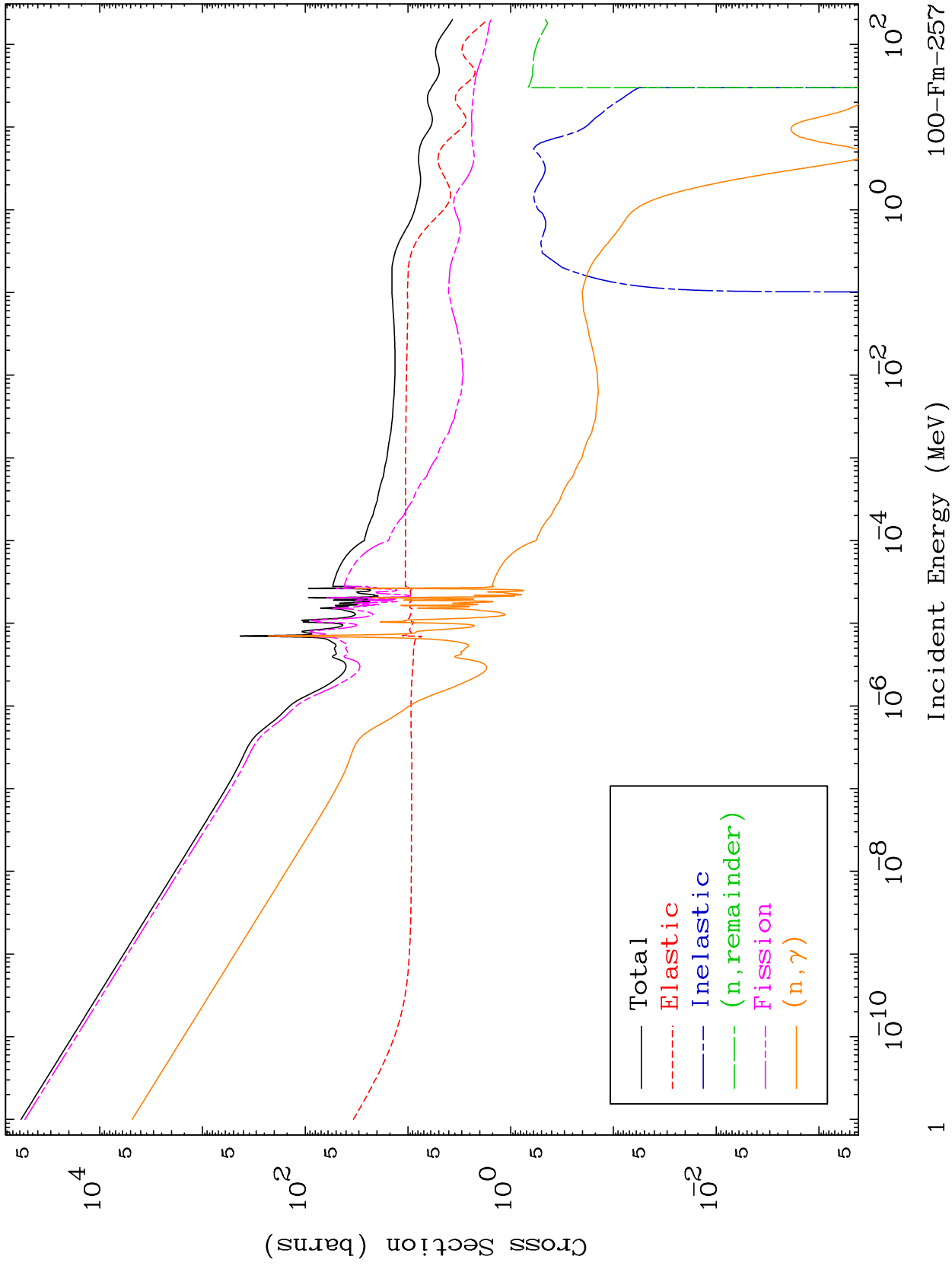
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9938

Major
293 Kelvin Cross Sections

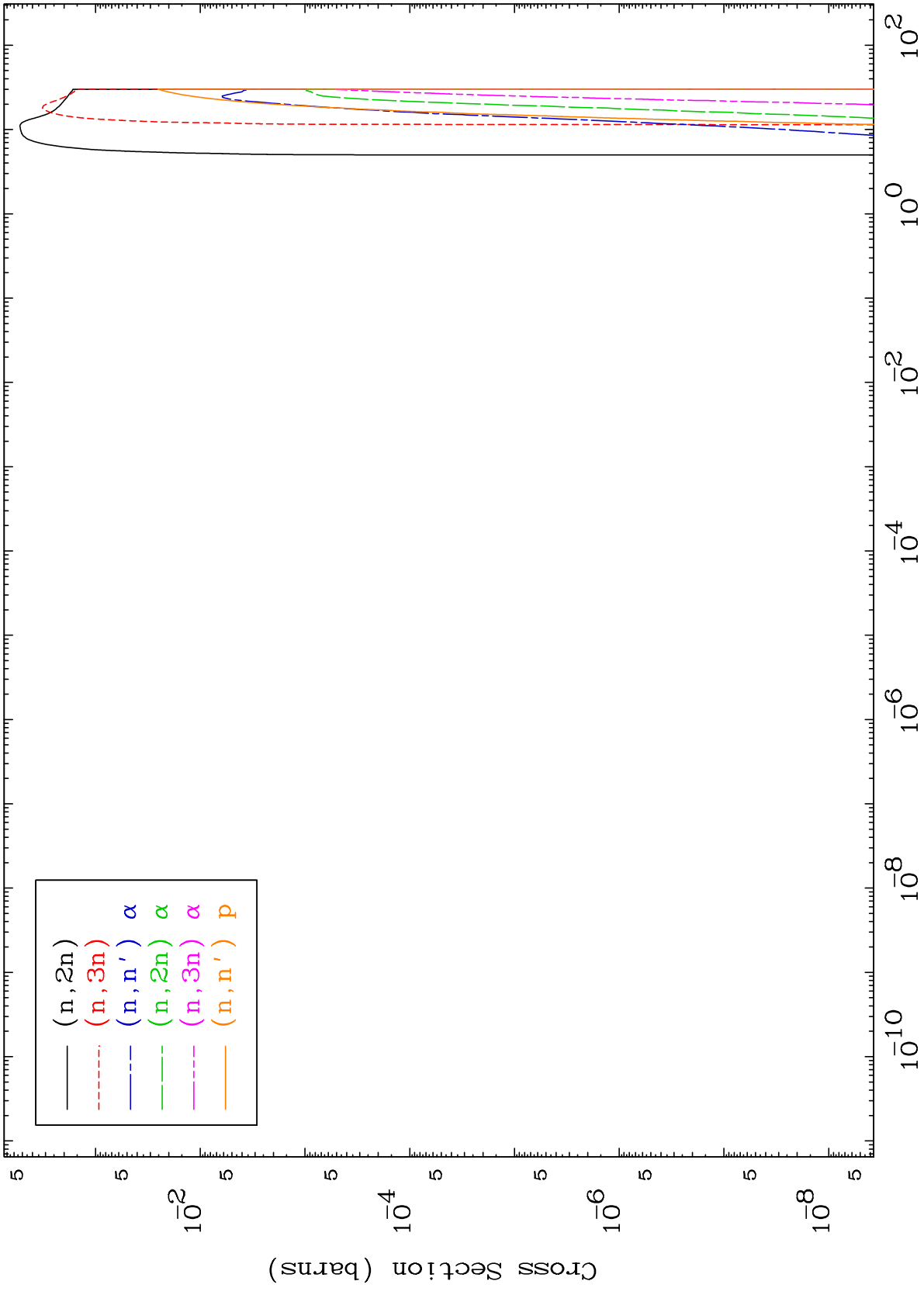
100-Fm-257



MAT 9938

Neutron Production
293 Kelvin Cross Sections

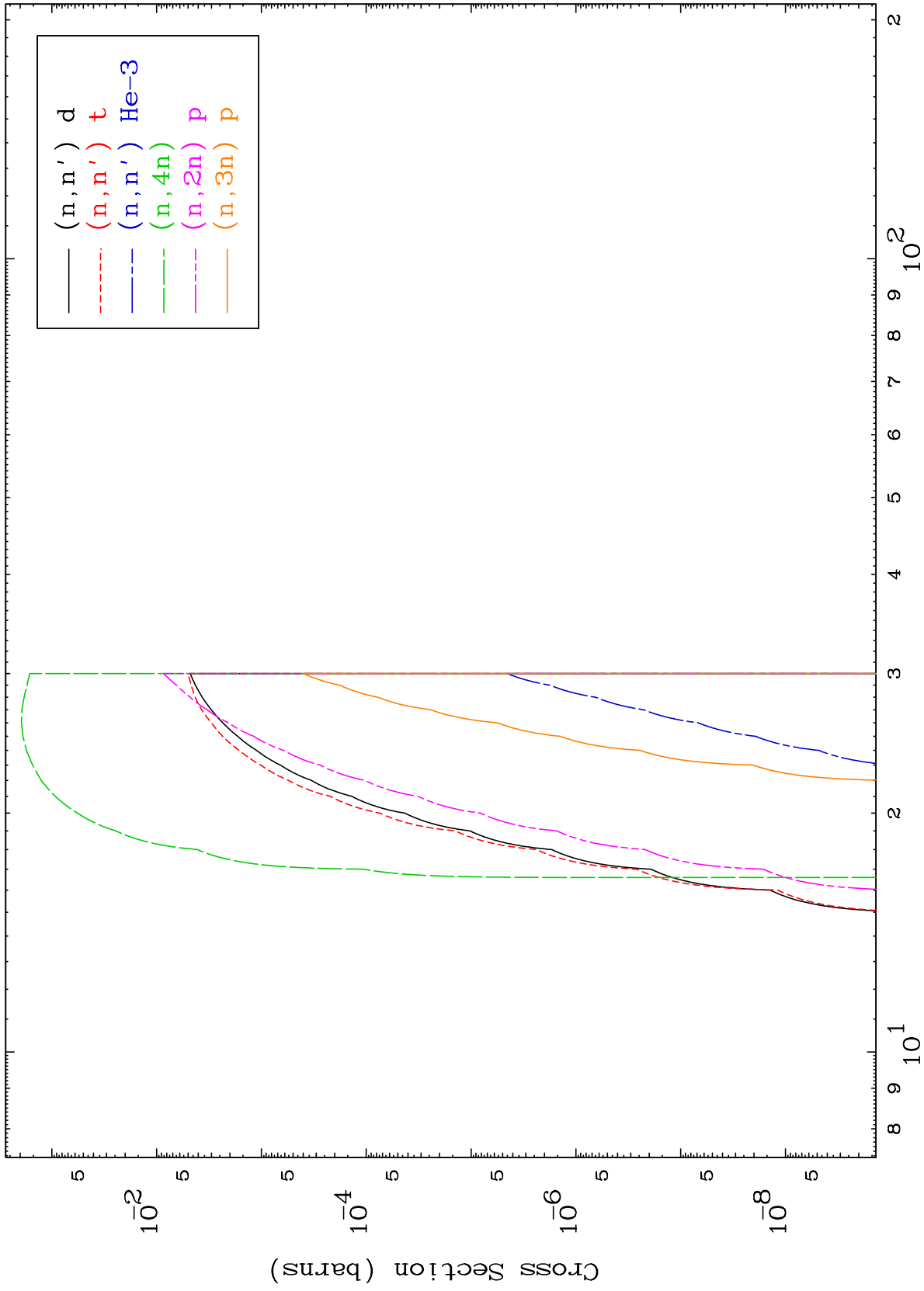
100-Fm-257



MAT 9938

Neutron Production
293 Kelvin Cross Sections

100-Fm-257



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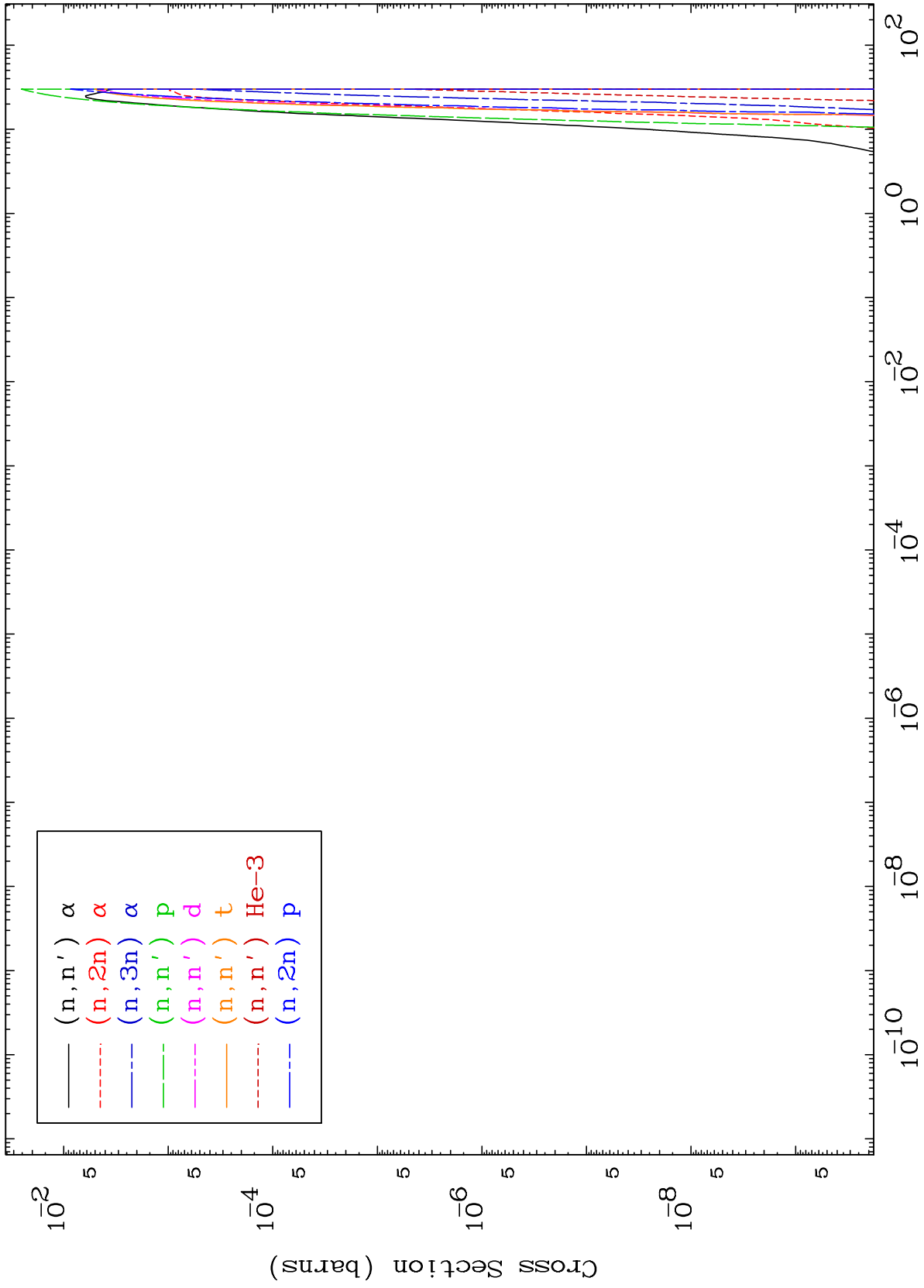
Incident Energy (MeV)

100-Fm-257

MAT 9938

Charged Particle
293 Kelvin Cross Sections

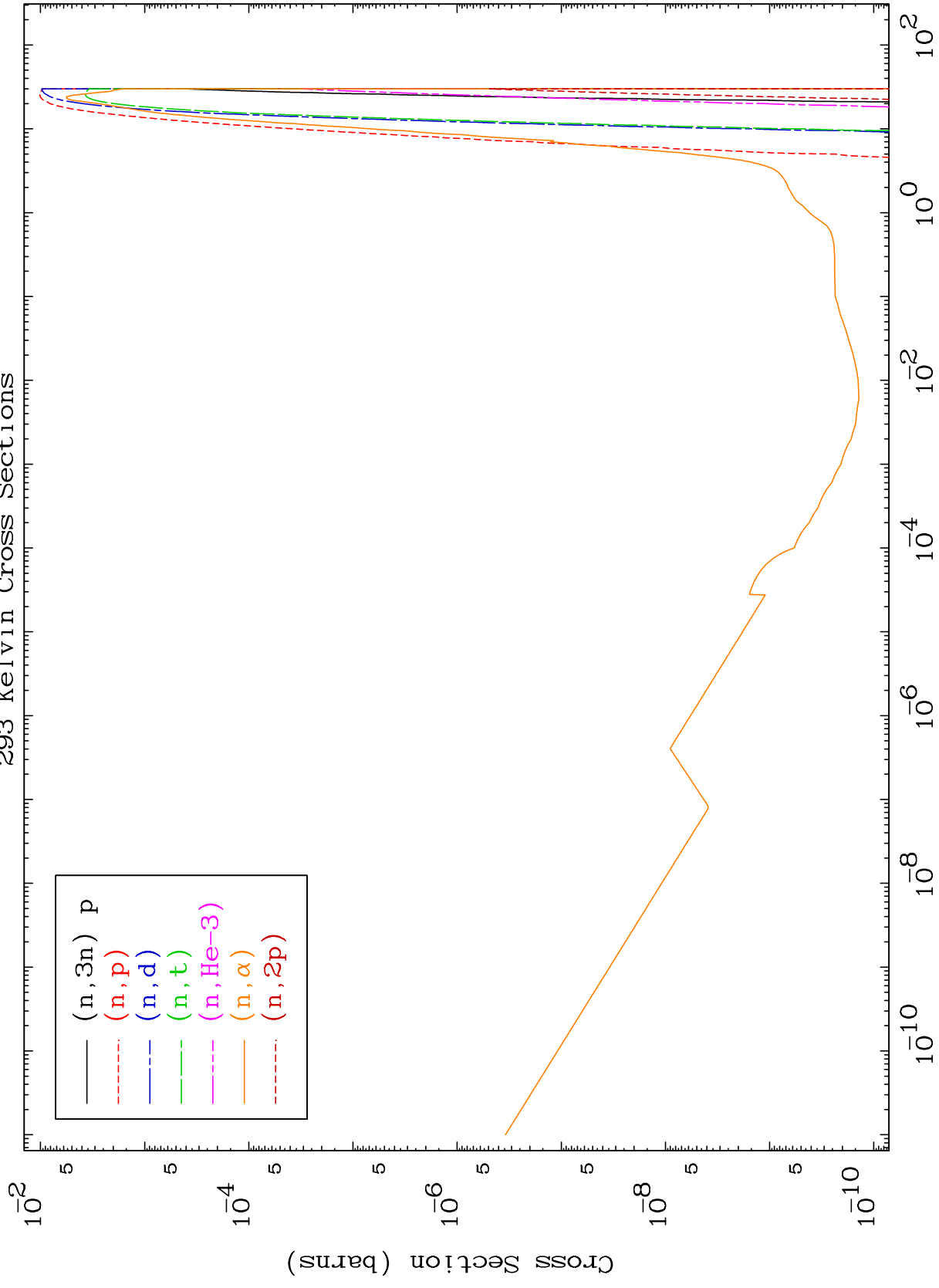
100-Fm-257



MAT 9938

Charged Particle
293 Kelvin Cross Sections

100-Fm-257



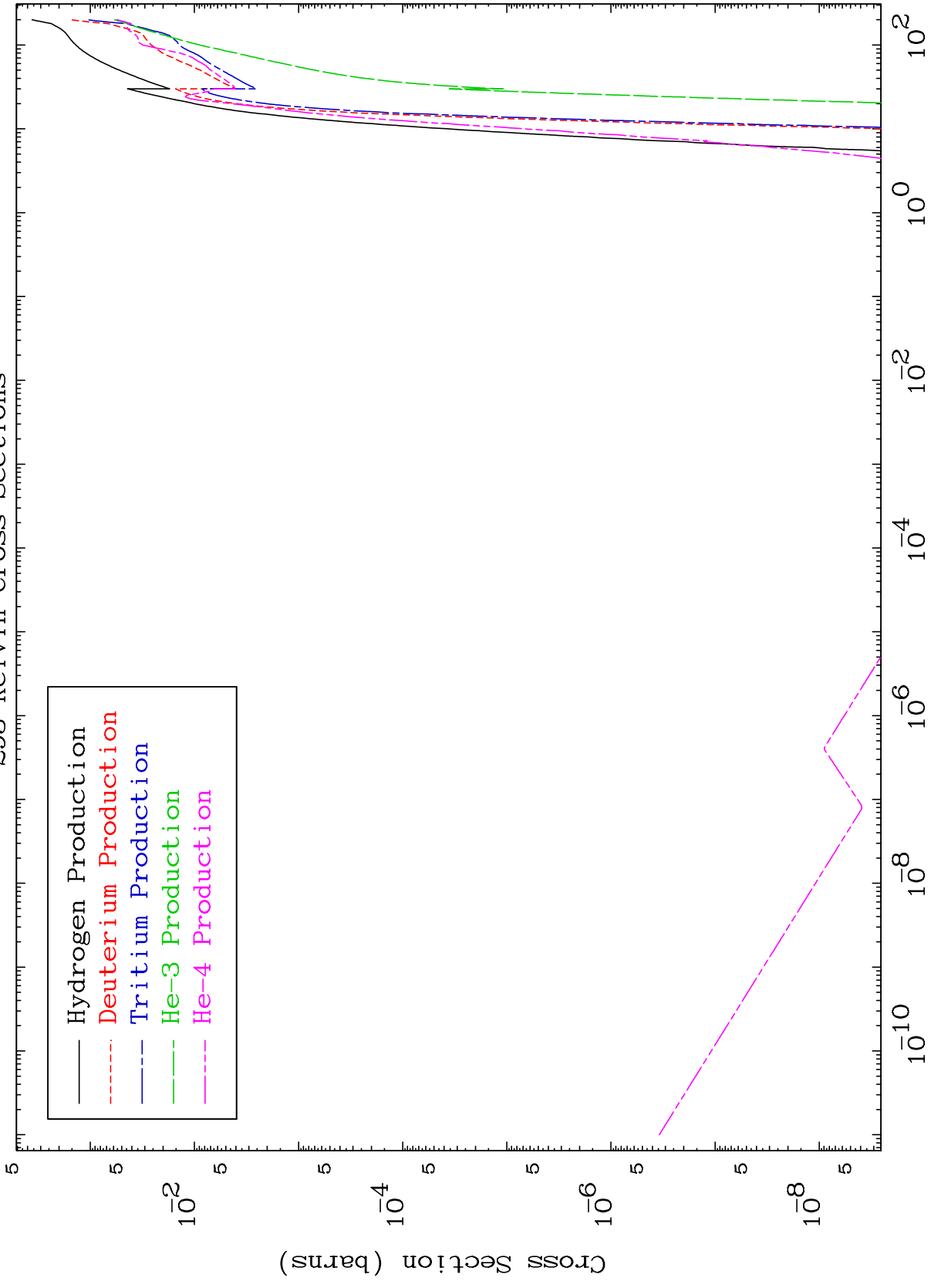
5

100-Fm-257

MAT 9938

Particle Production
293 Kelvin Cross Sections

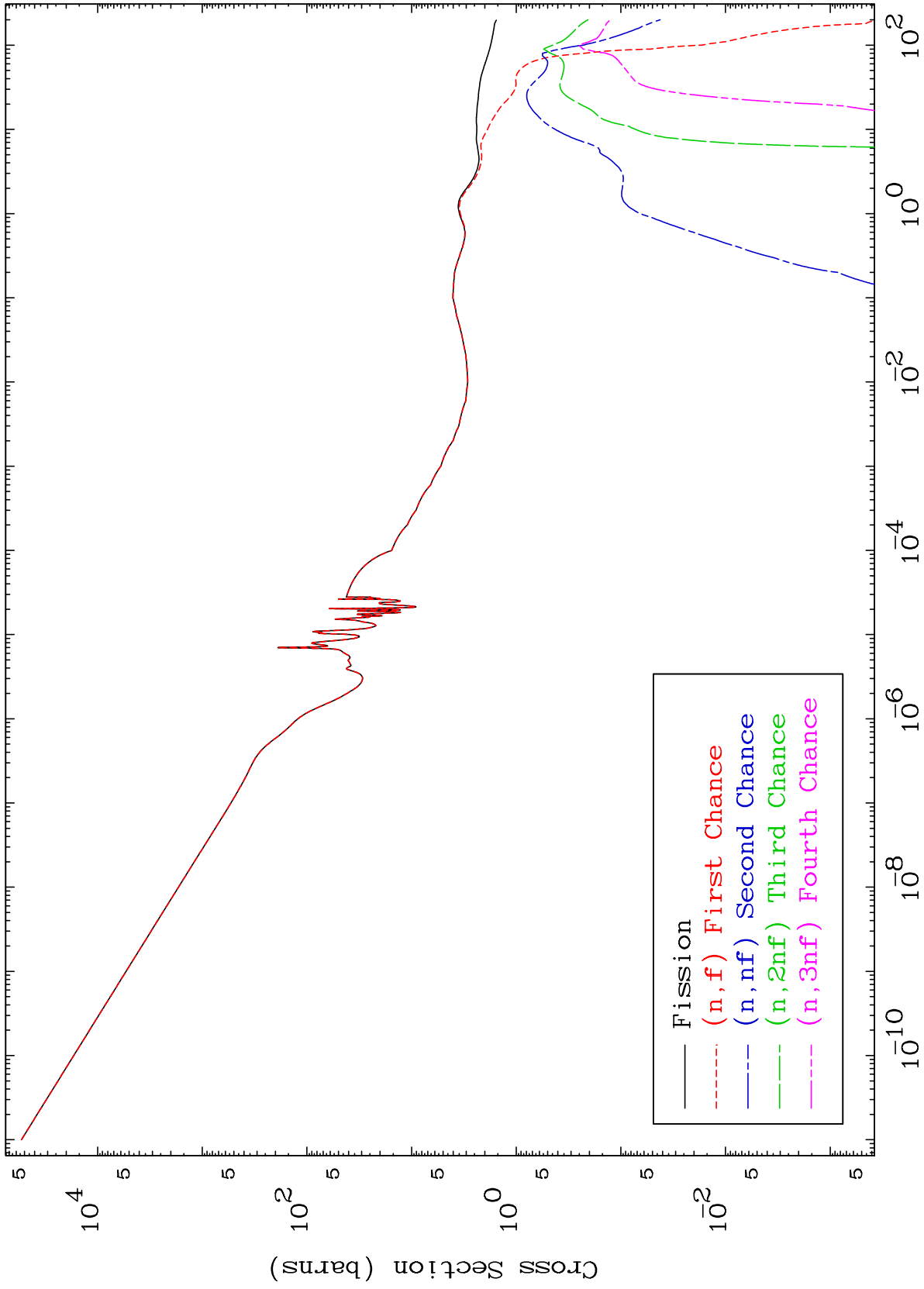
100-Fm-257



MAT 9938

Fission
293 Kelvin Cross Sections

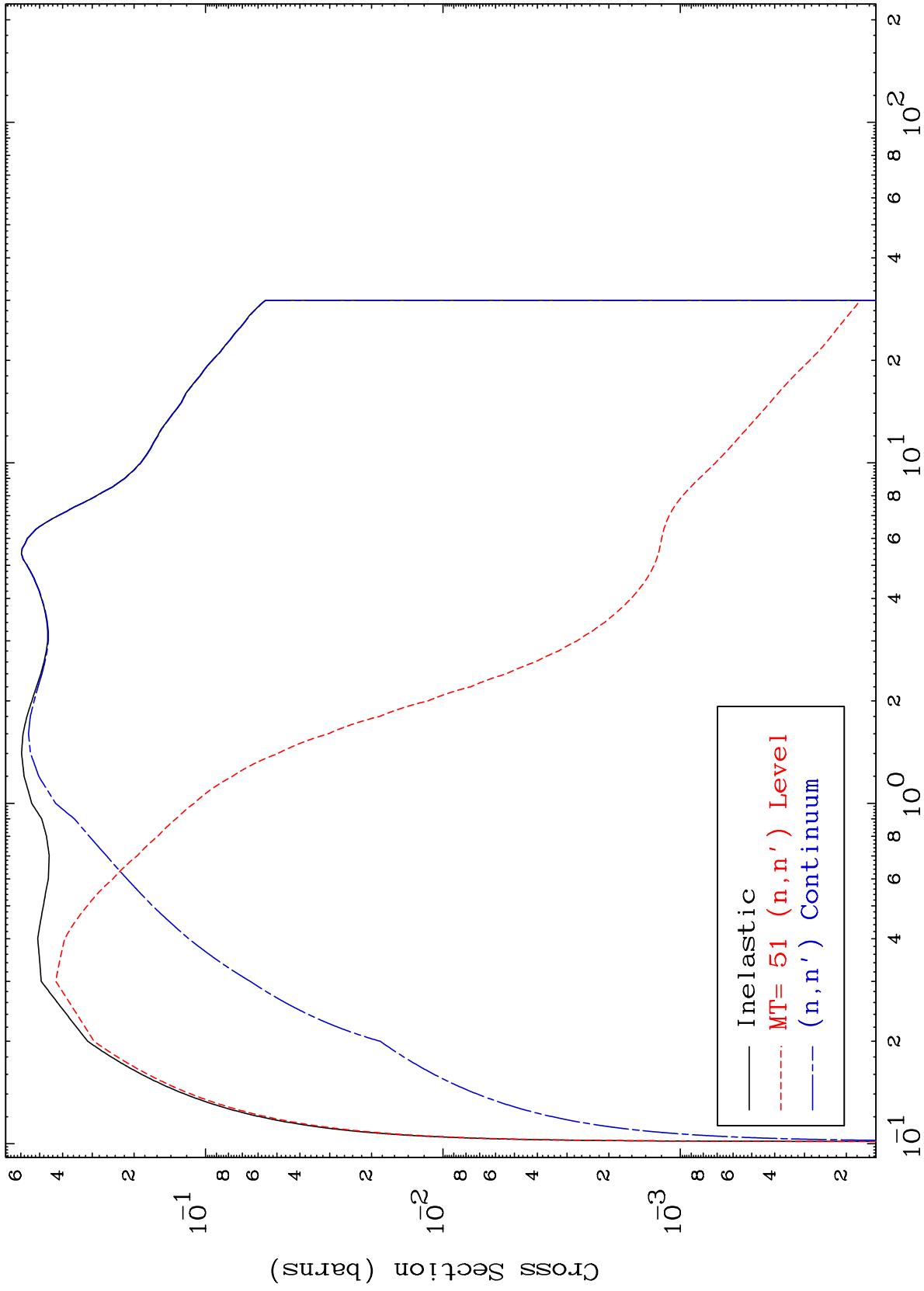
100-Fm-257



MAT 9938

(n,n') Level
293 Kelvin Cross Sections

100-Fm-257

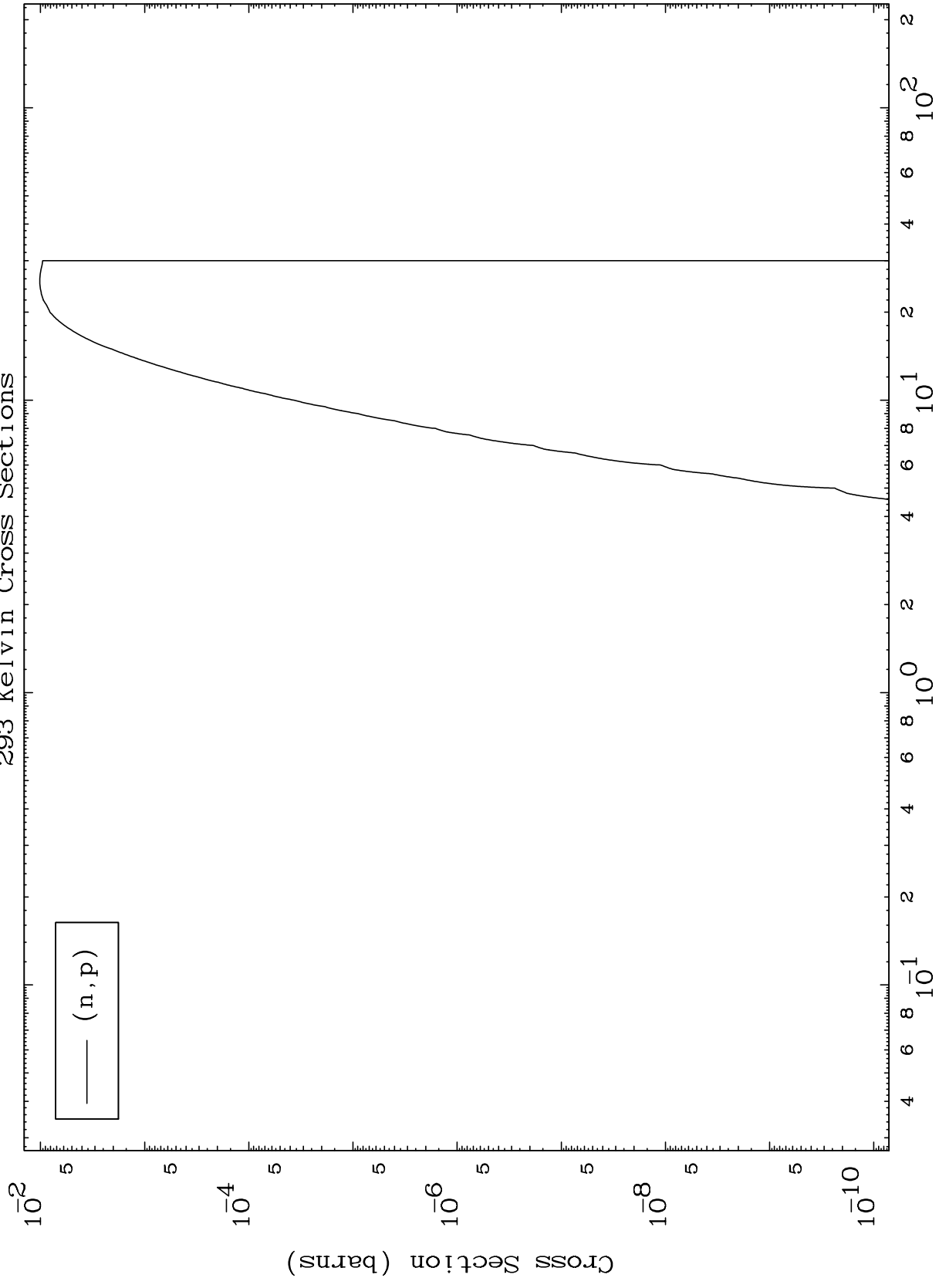


8

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(n,p) Levels
293 Kelvin Cross Sections

100-Fm-257

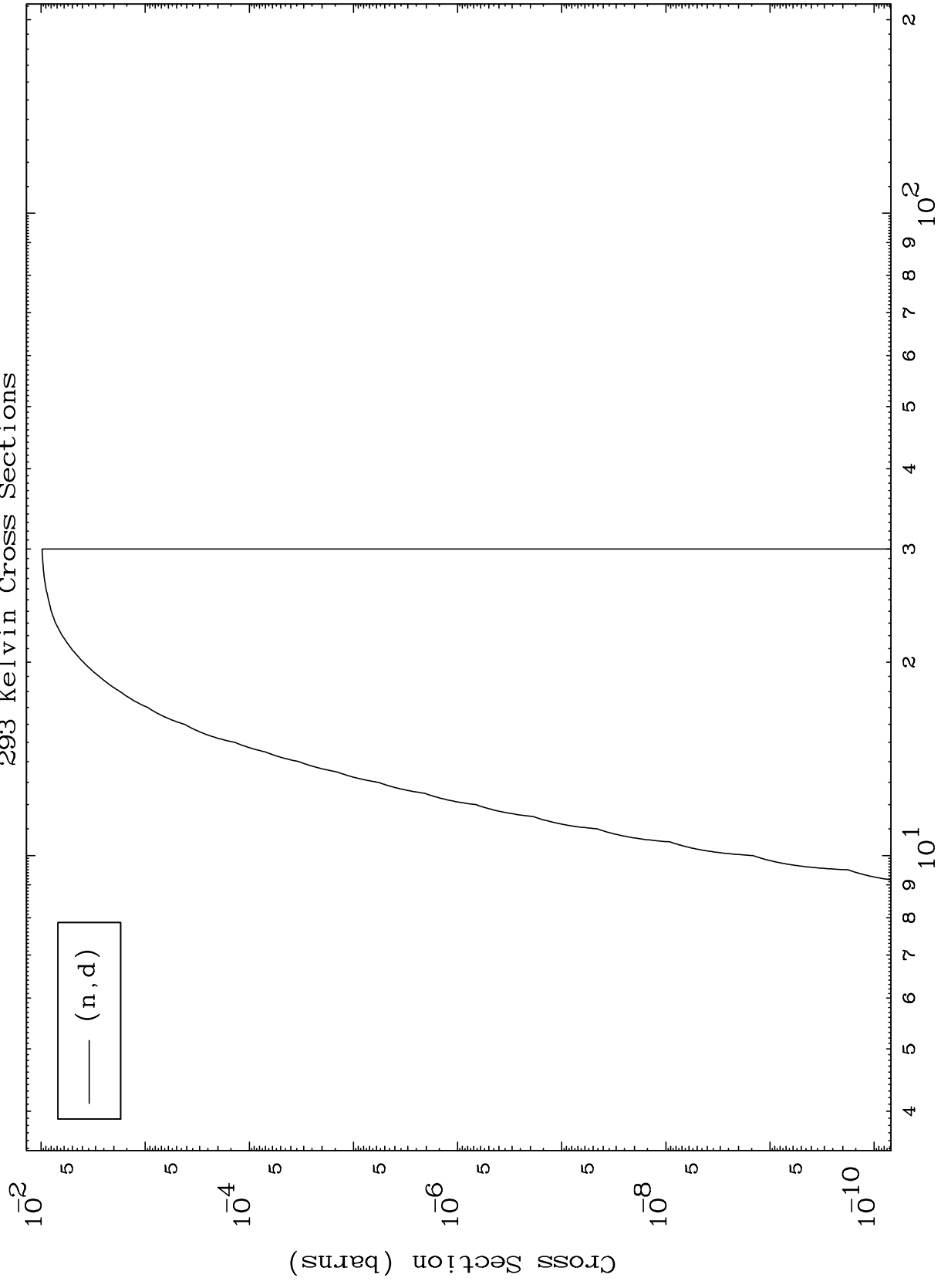


100-Fm-257

MAT 9938

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-257



10

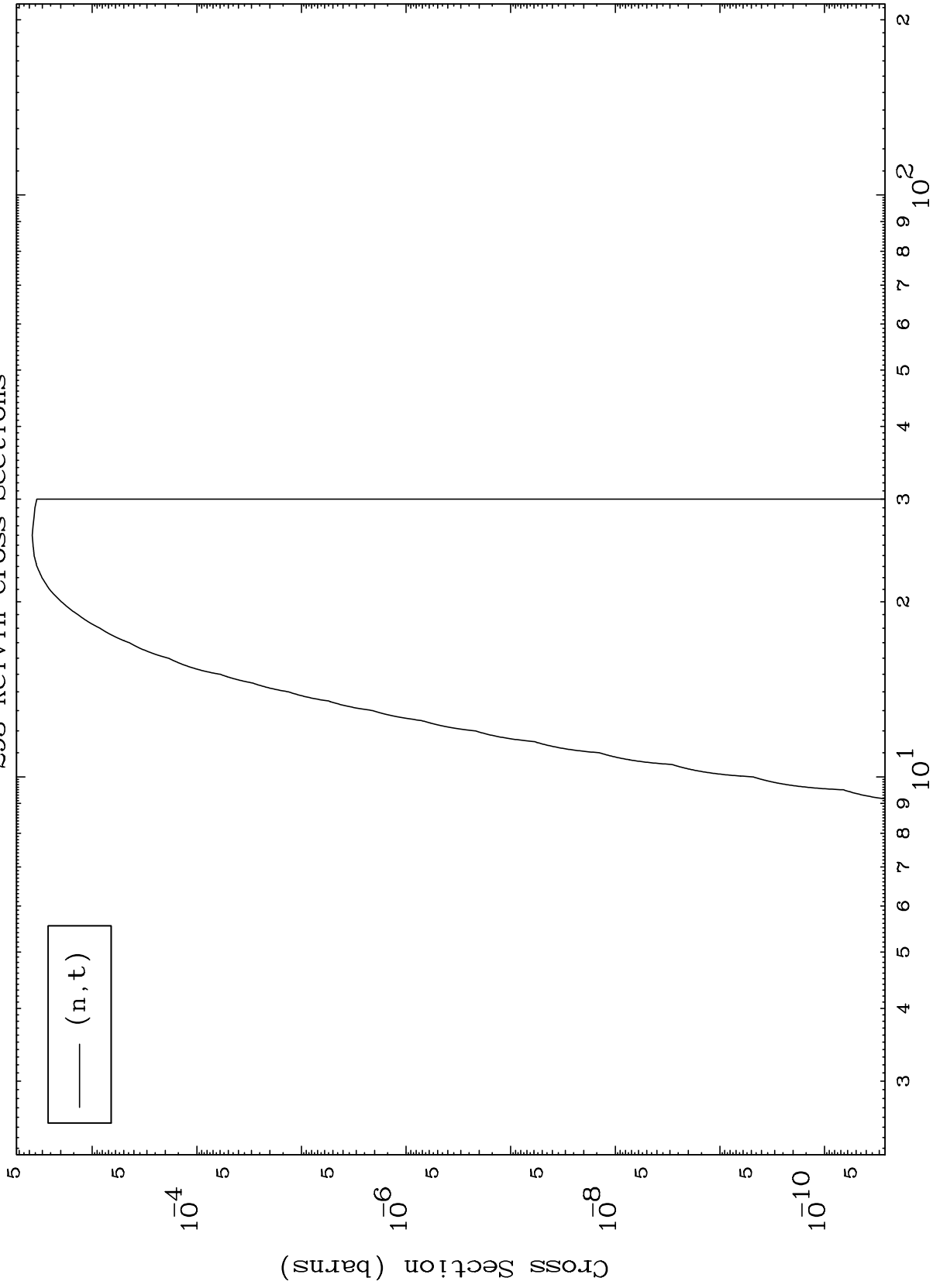
Incident Energy (MeV)

100-Fm-257

MAT 9938

(n,t) Levels
293 Kelvin Cross Sections

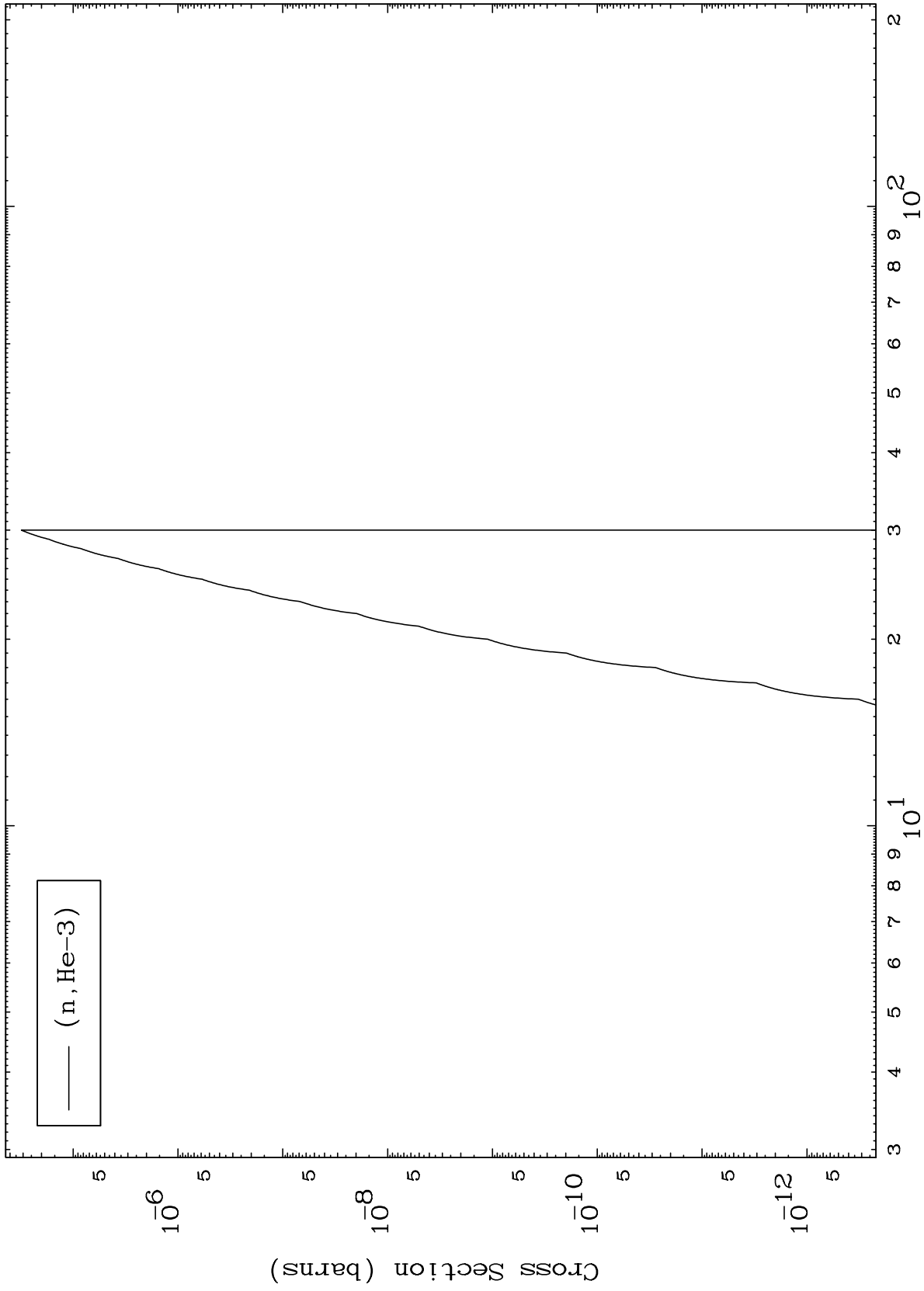
100-Fm-257



MAT 9938

(n,He3) Levels
293 Kelvin Cross Sections

100-Fm-257



12

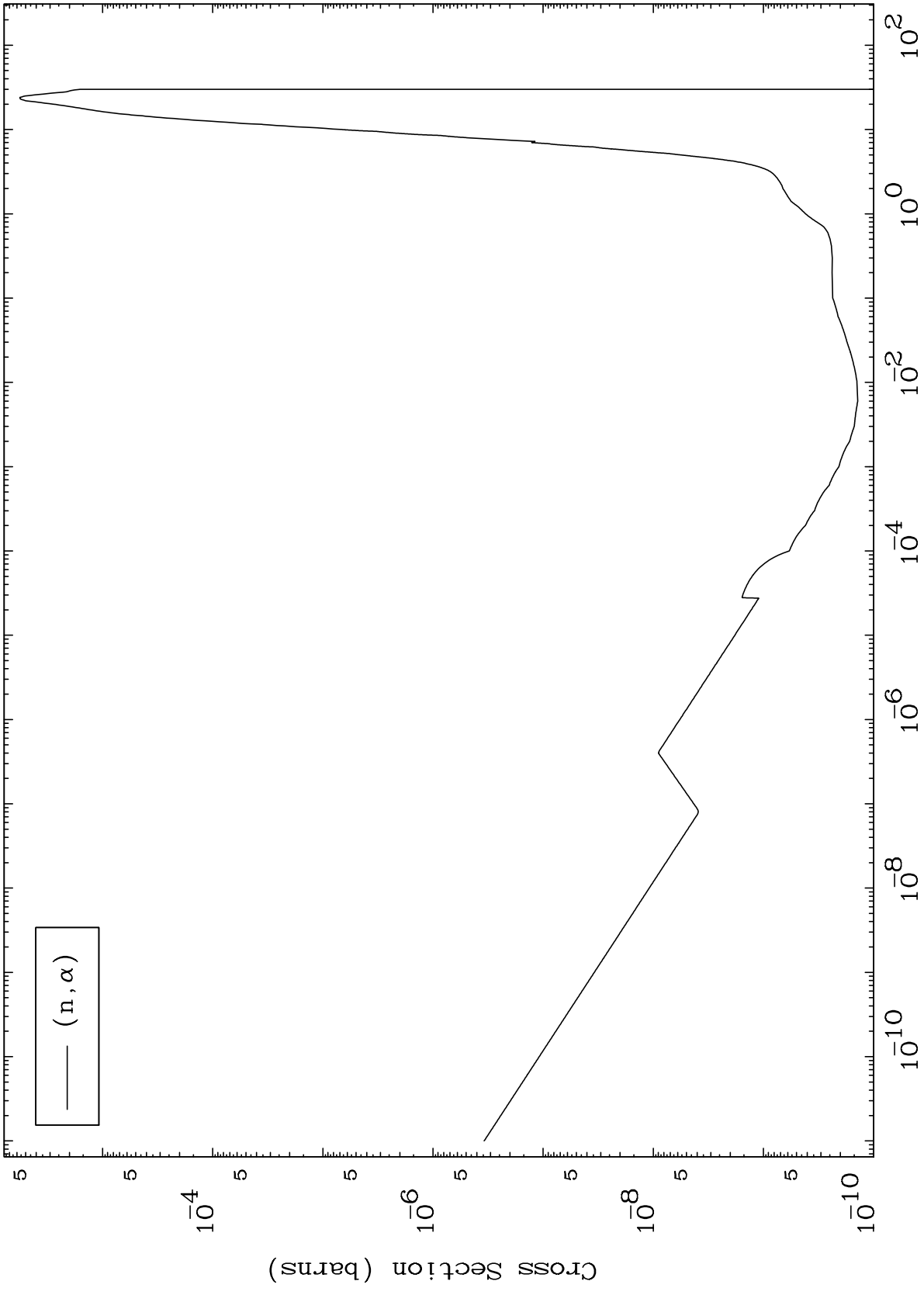
Incident Energy (MeV)

100-Fm-257

MAT 9938

(n,α) Levels
293 Kelvin Cross Sections

100-Fm-257

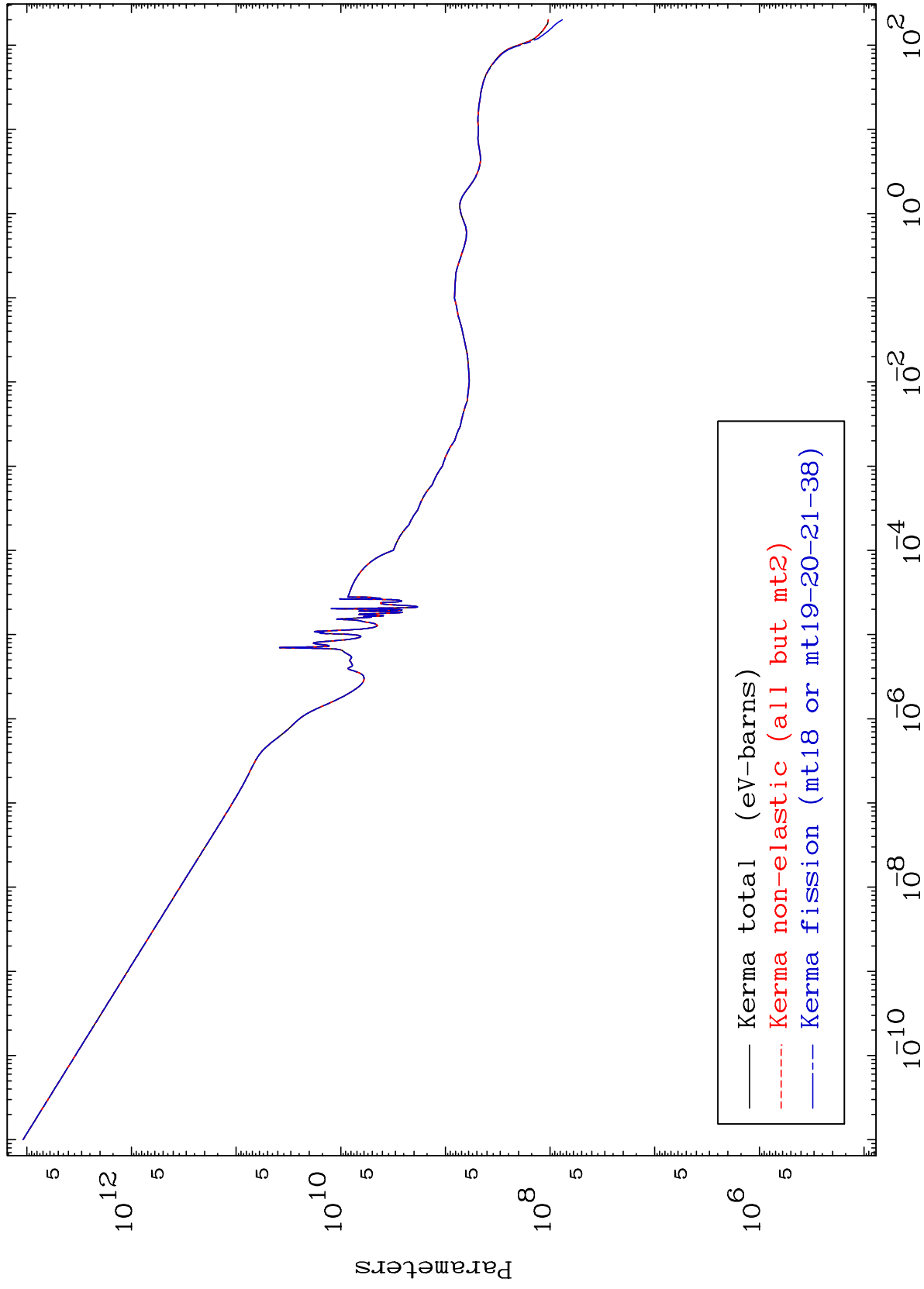


(n,α)

MAT 9938

Energy Release
Parameters

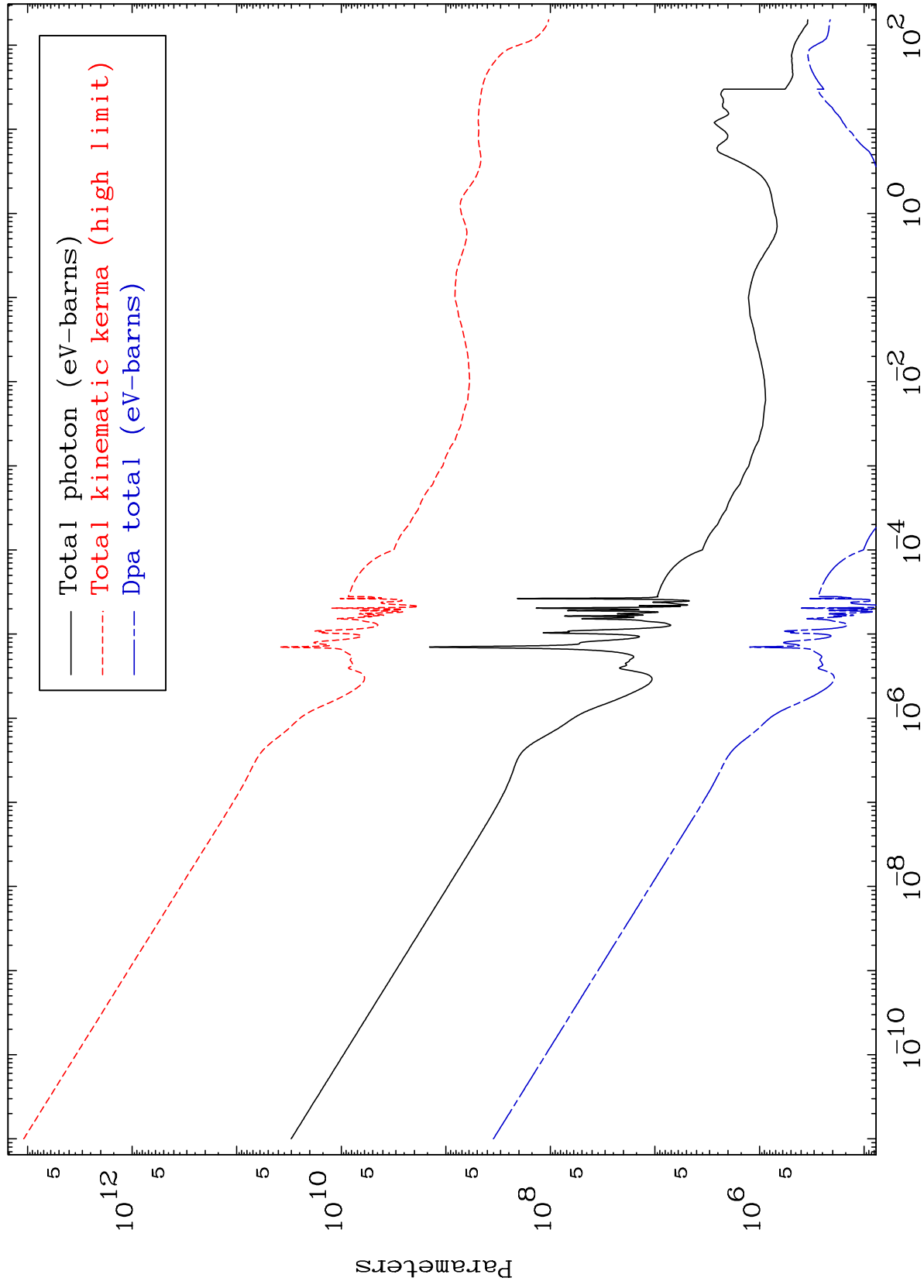
100-Fm-257



MAT 9938

Energy Release
Parameters

100-Fm-257



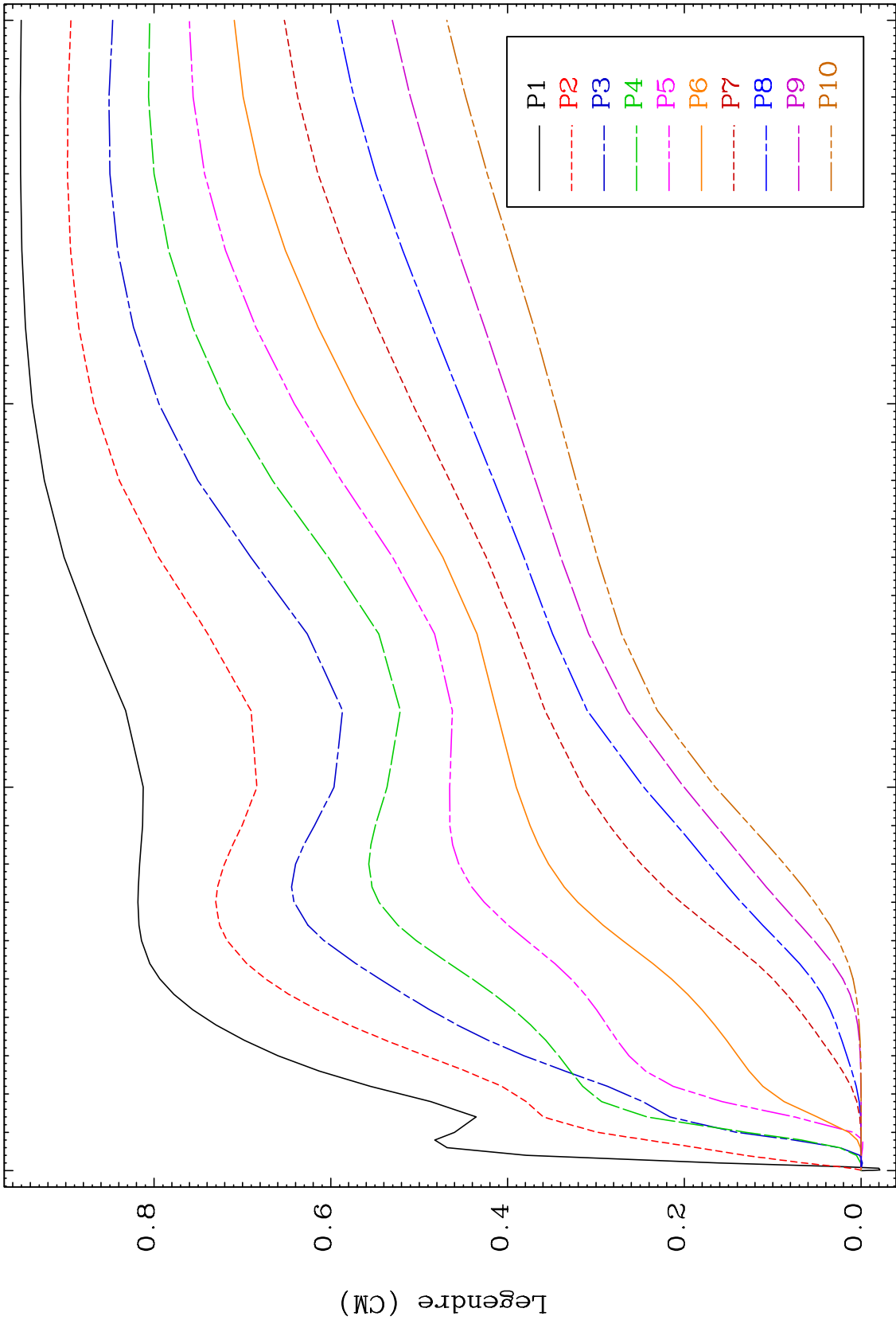
15

100-Fm-257

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Elastic Legendre Coefficients

100-Fm-257



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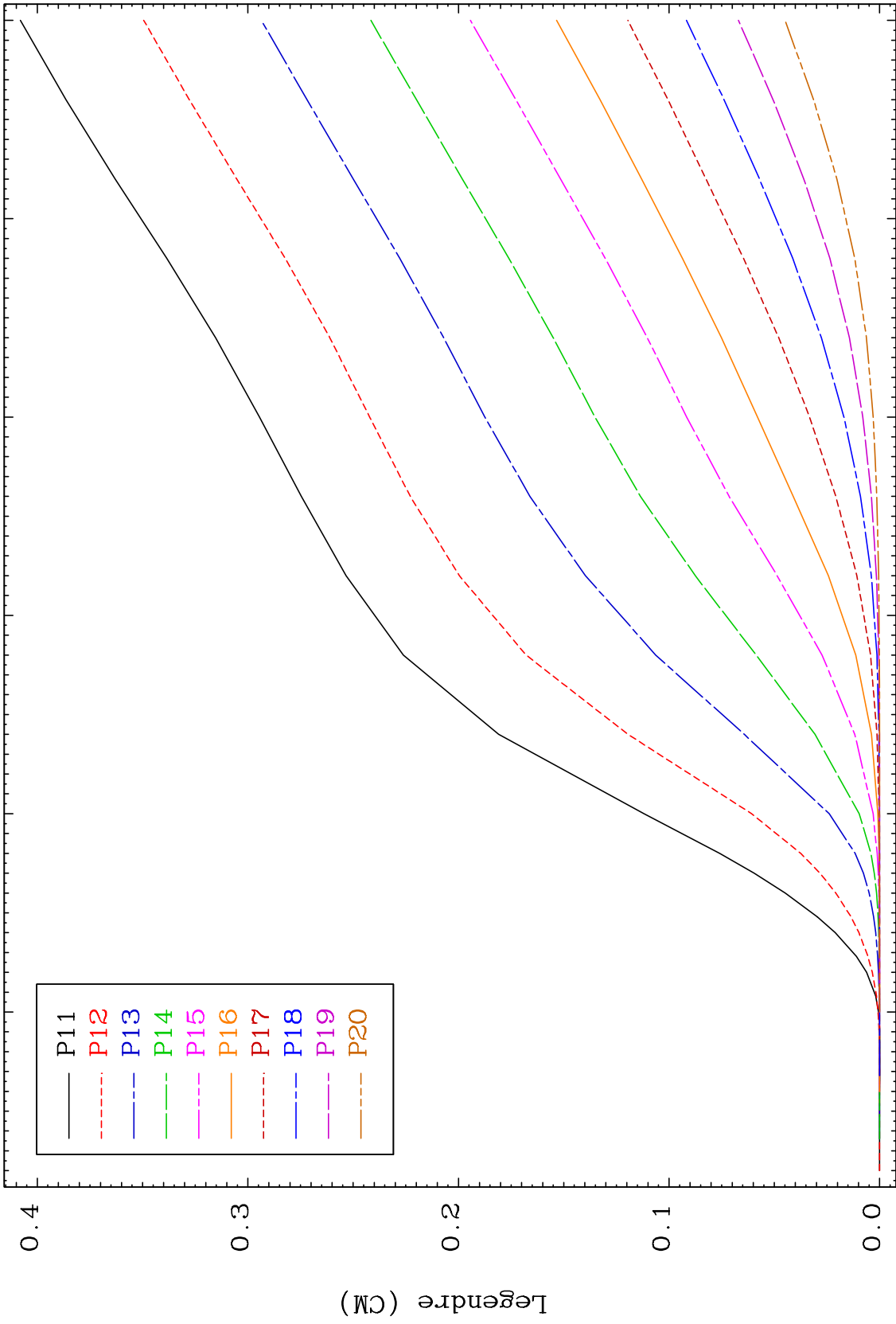
Incident Energy (MeV)

100-Fm-257

MAT 9938

Elastic Legendre Coefficients

100-Fm-257



17

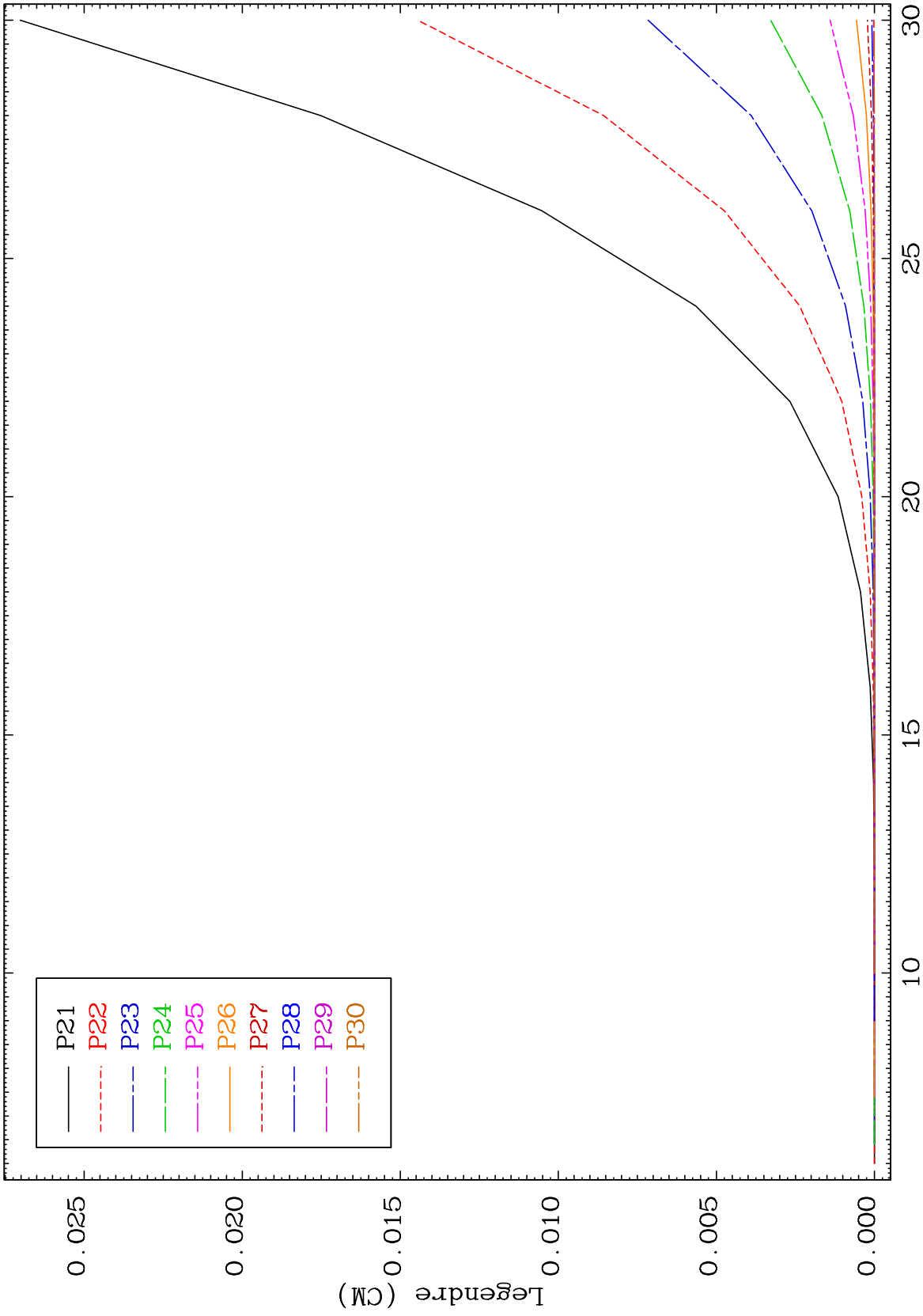
Incident Energy (MeV)

100-Fm-257

MAT 9938

Elastic
Legendre Coefficients

100-Fm-257



18

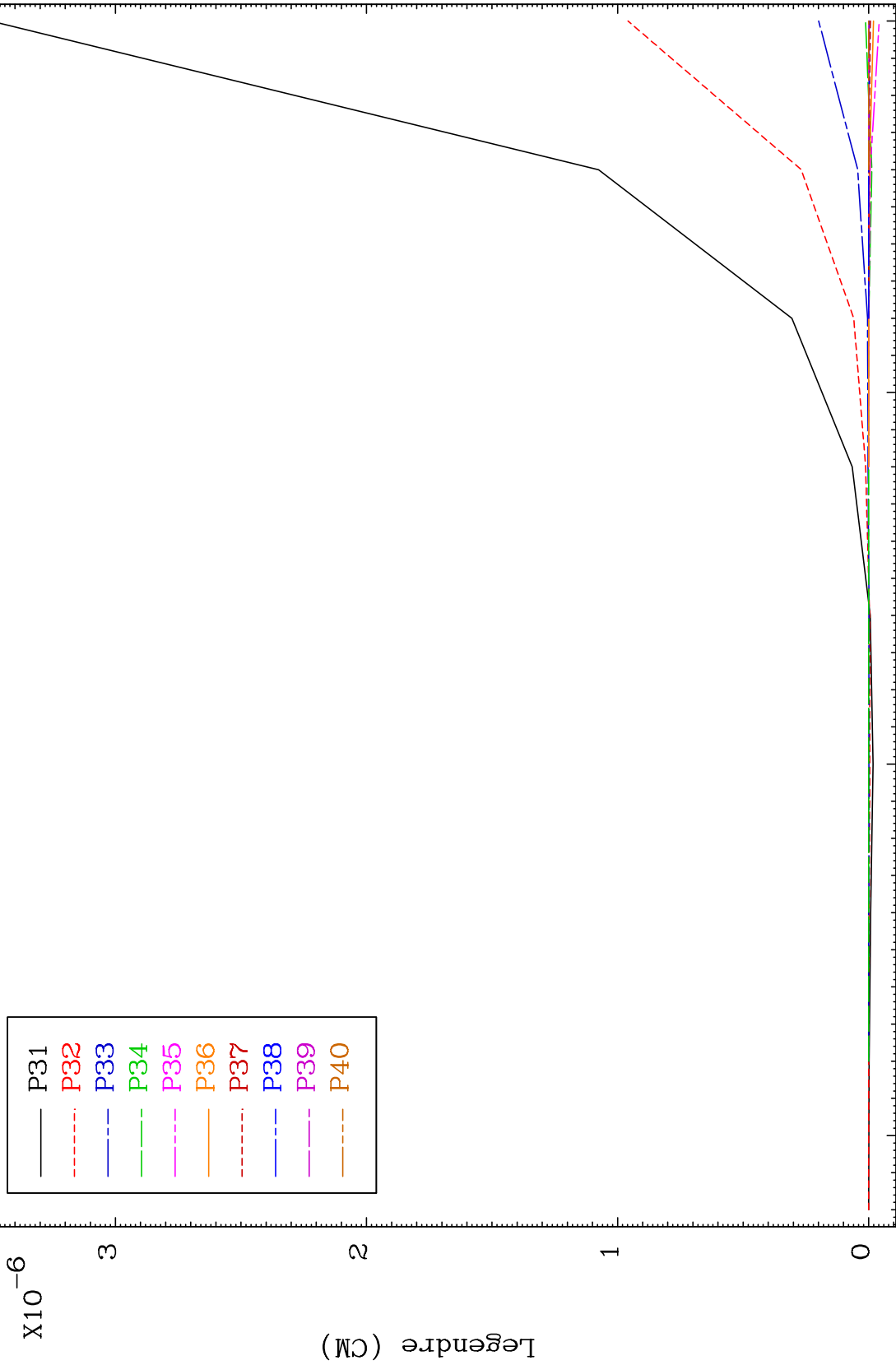
Incident Energy (MeV)

100-Fm-257

MAT 9938

Elastic Legendre Coefficients

100-Fm-257



19

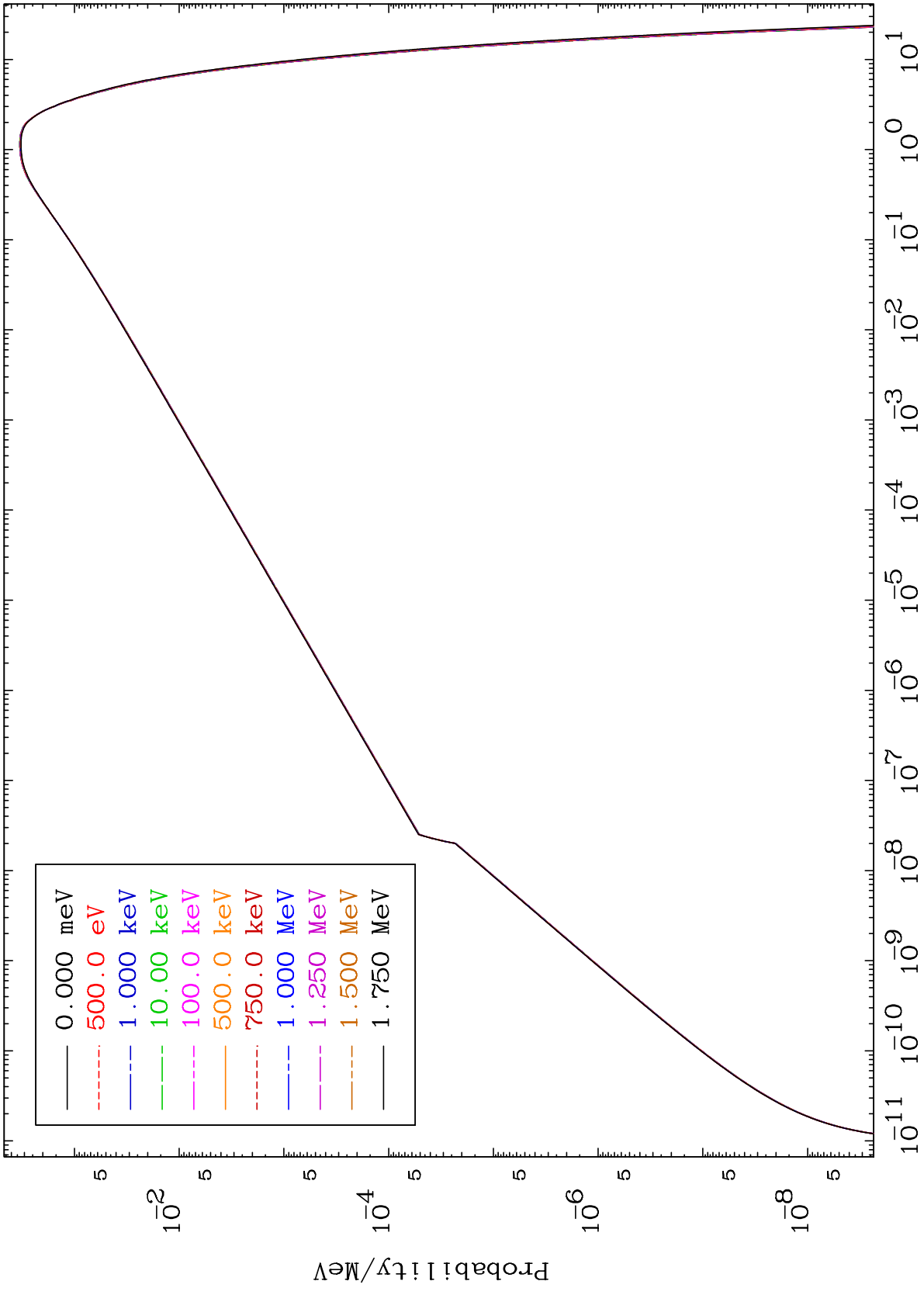
Incident Energy (MeV)

100-Fm-257

MAT 9938

Fission Energy Distribution

100-Fm-257

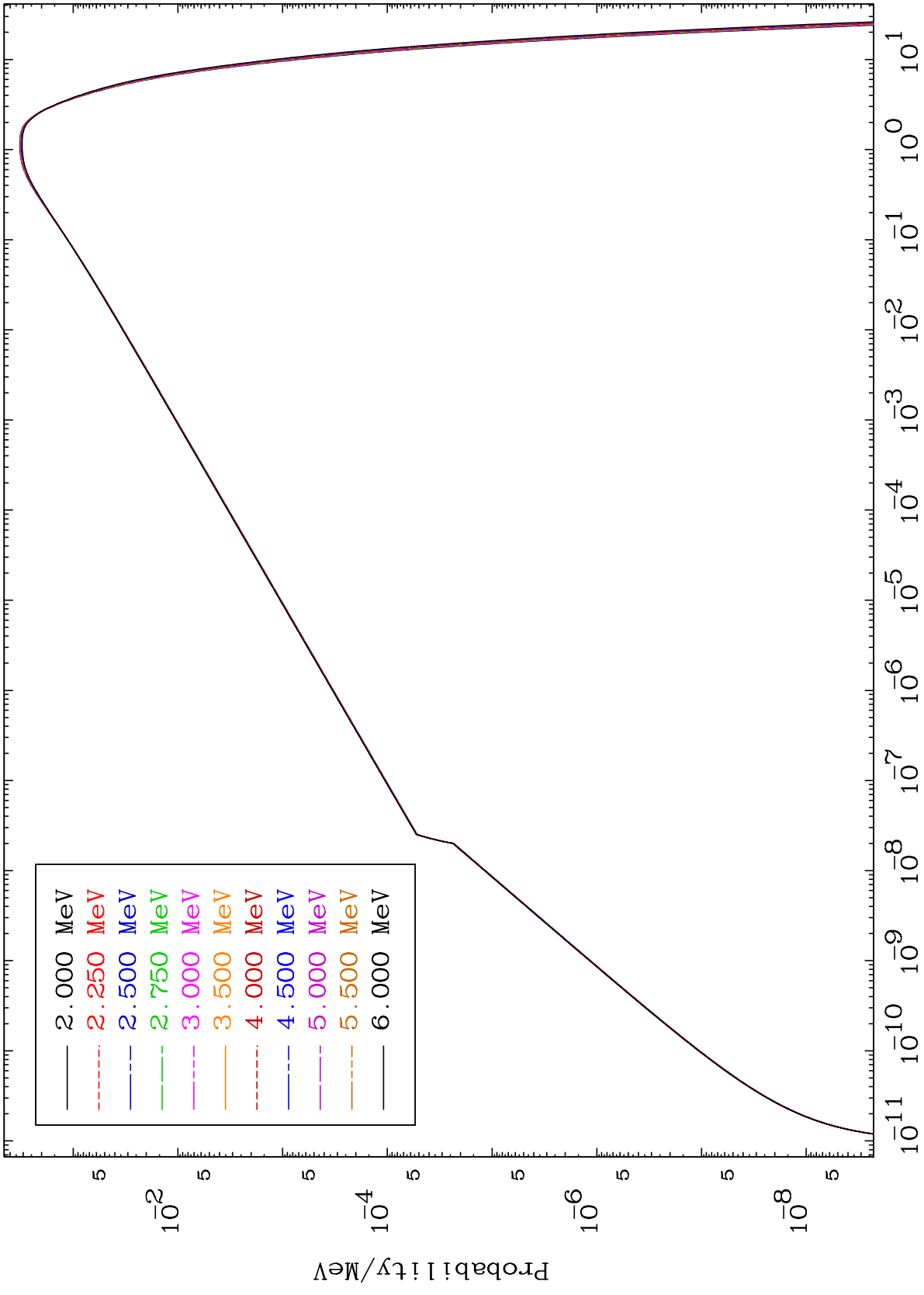


100-Fm-257

MAT 9938

Fission Energy Distribution

100-Fm-257

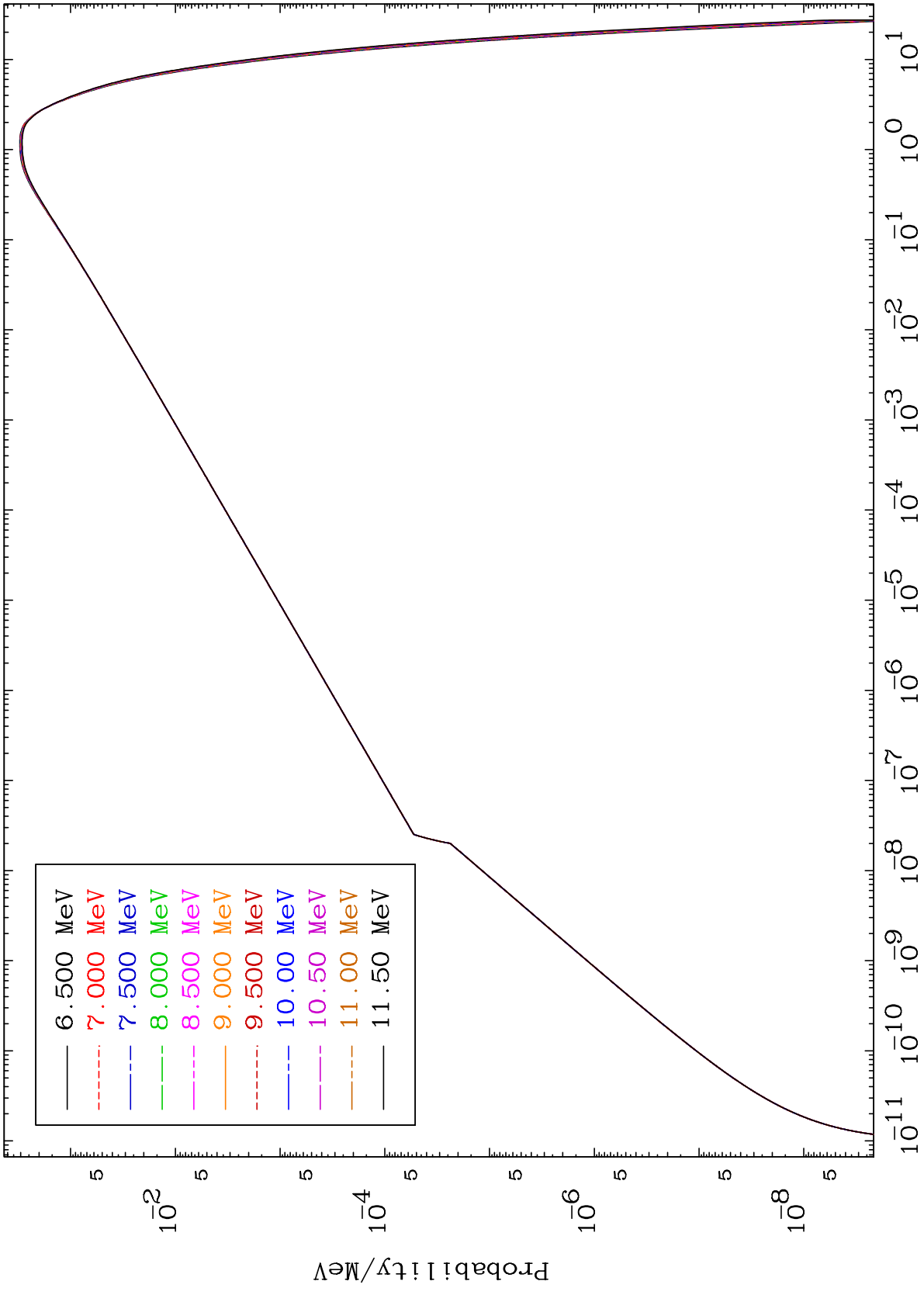


100-Fm-257

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Fission Energy Distribution

100-Fm-257

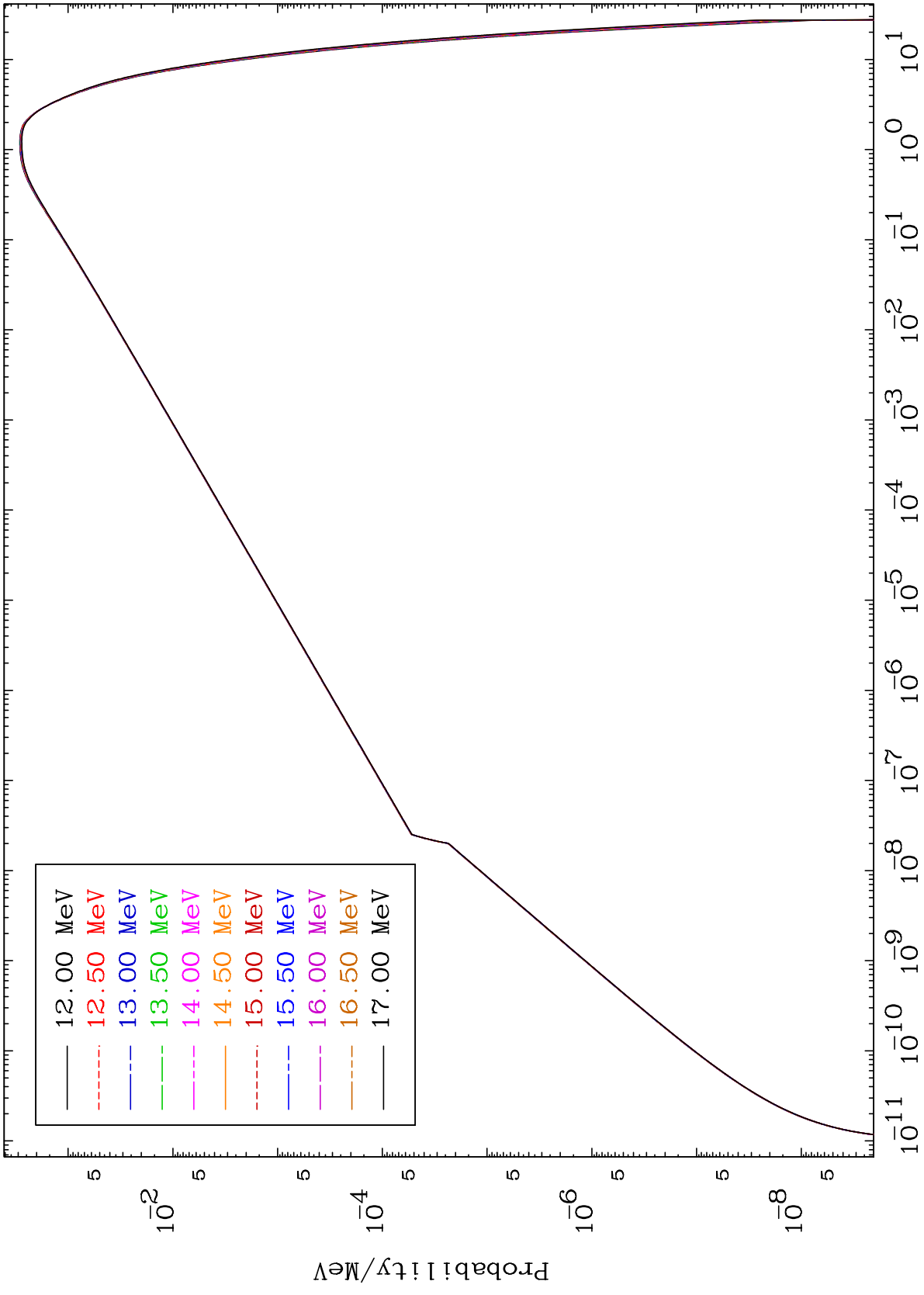


100-Fm-257

MAT 9938

Fission Energy Distribution

100-Fm-257

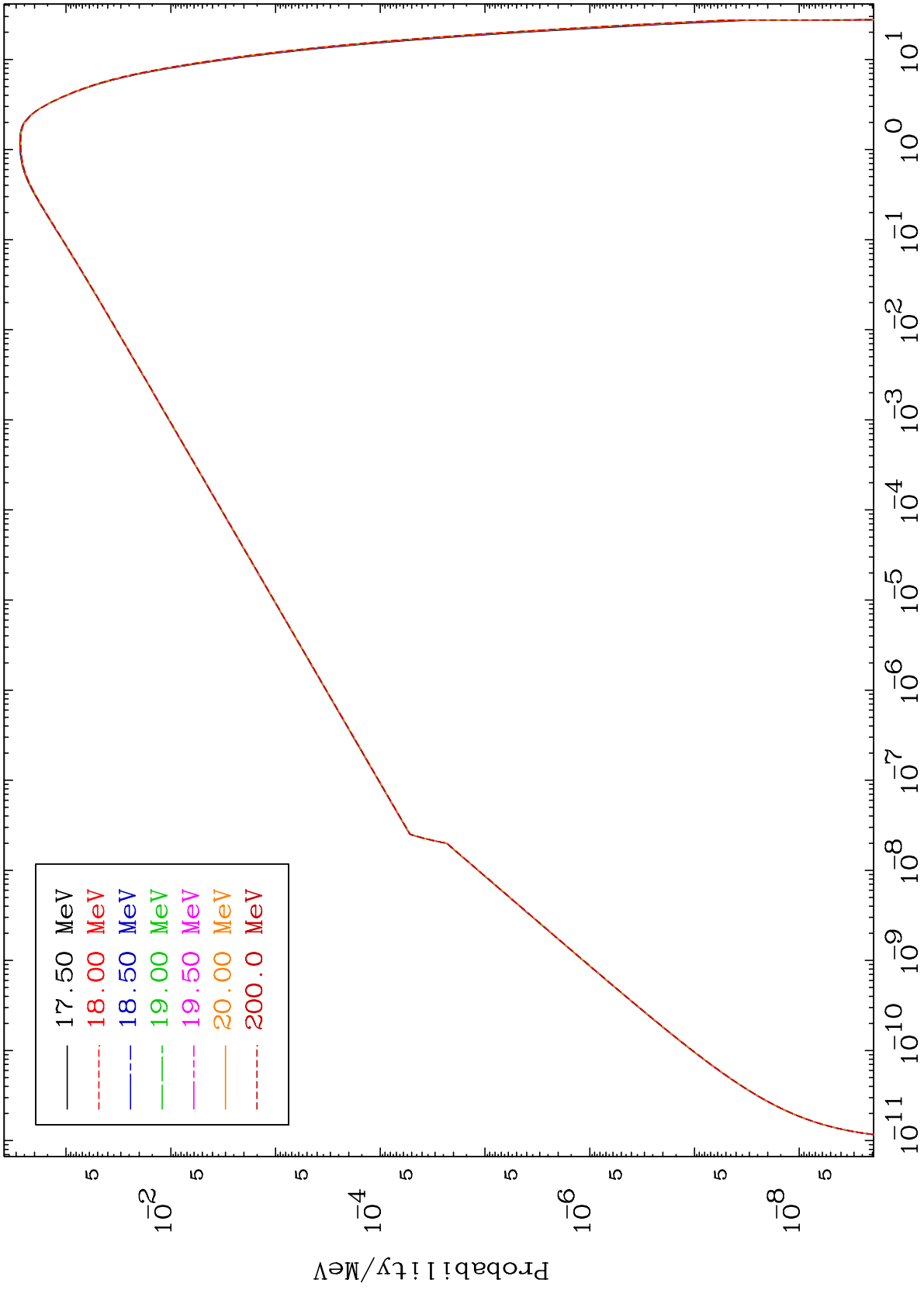


100-Fm-257

MAT 9938

Fission Energy Distribution

100-Fm-257

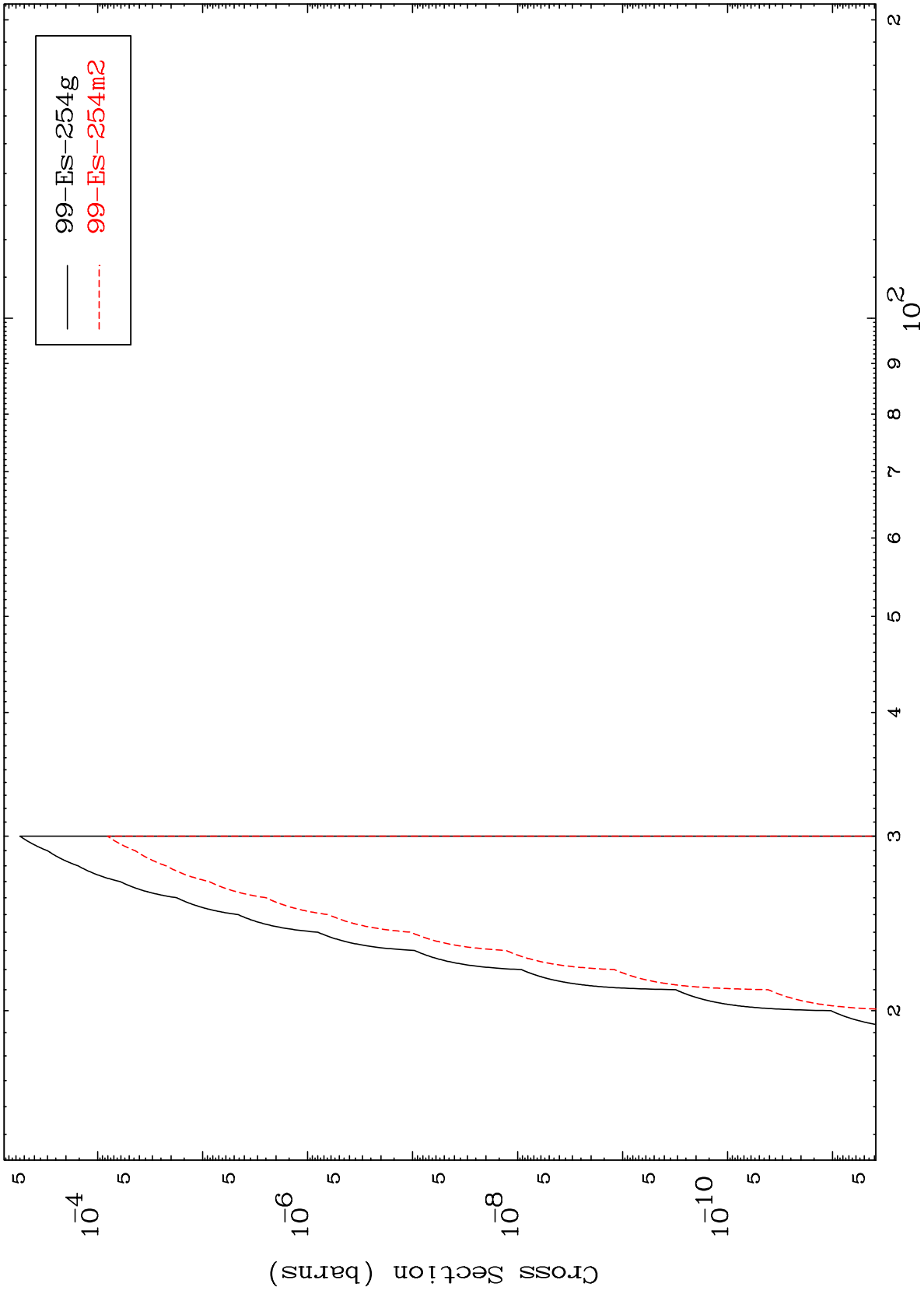


MAT 9938

(n,2n) d

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Radionuclide Production Cross Section



25

Incident Energy (MeV)

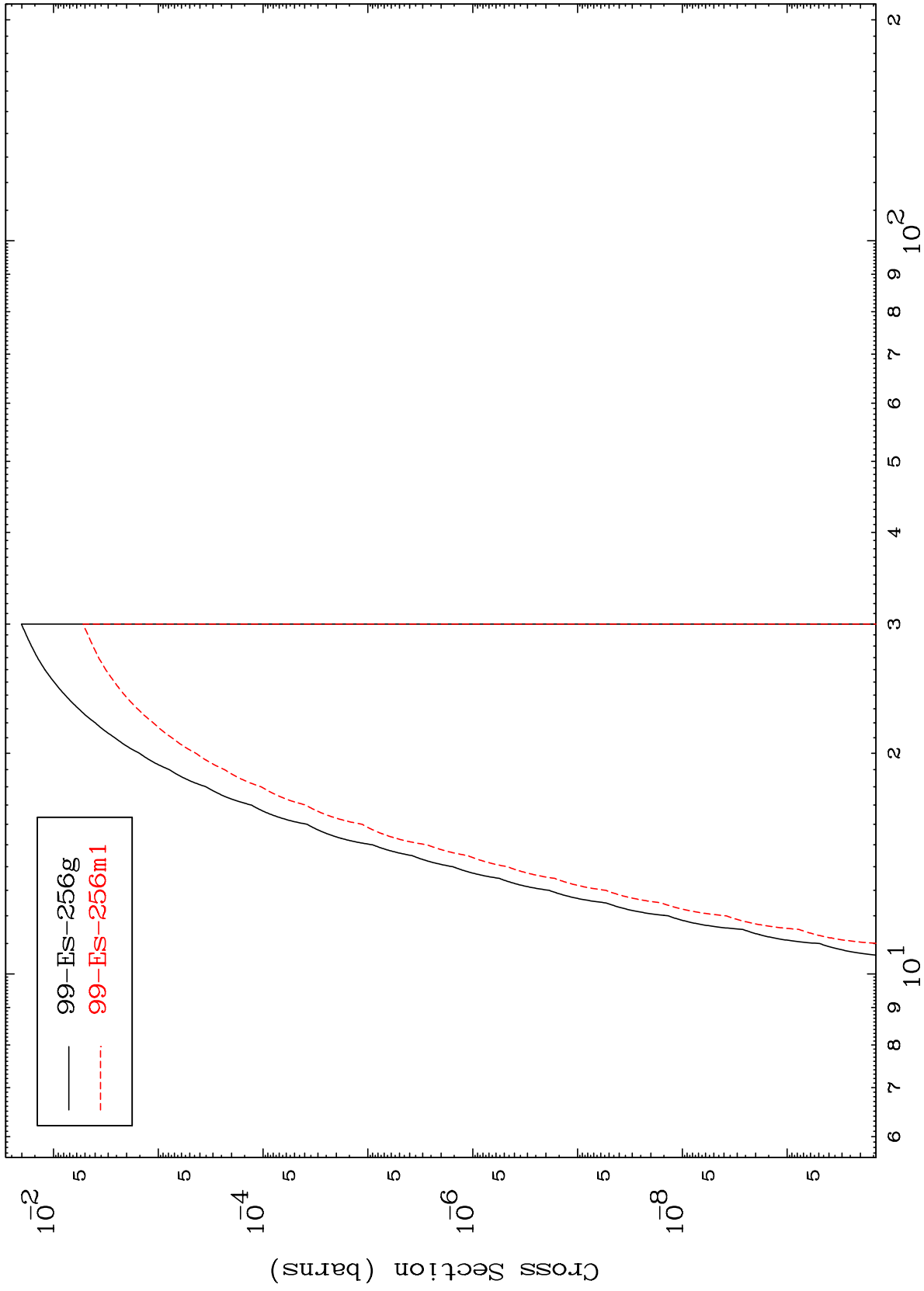
100-Fm-257

MAT 9938

(n,n') p

100-Fm-257

Radionuclide Production Cross Section



26

Incident Energy (MeV)

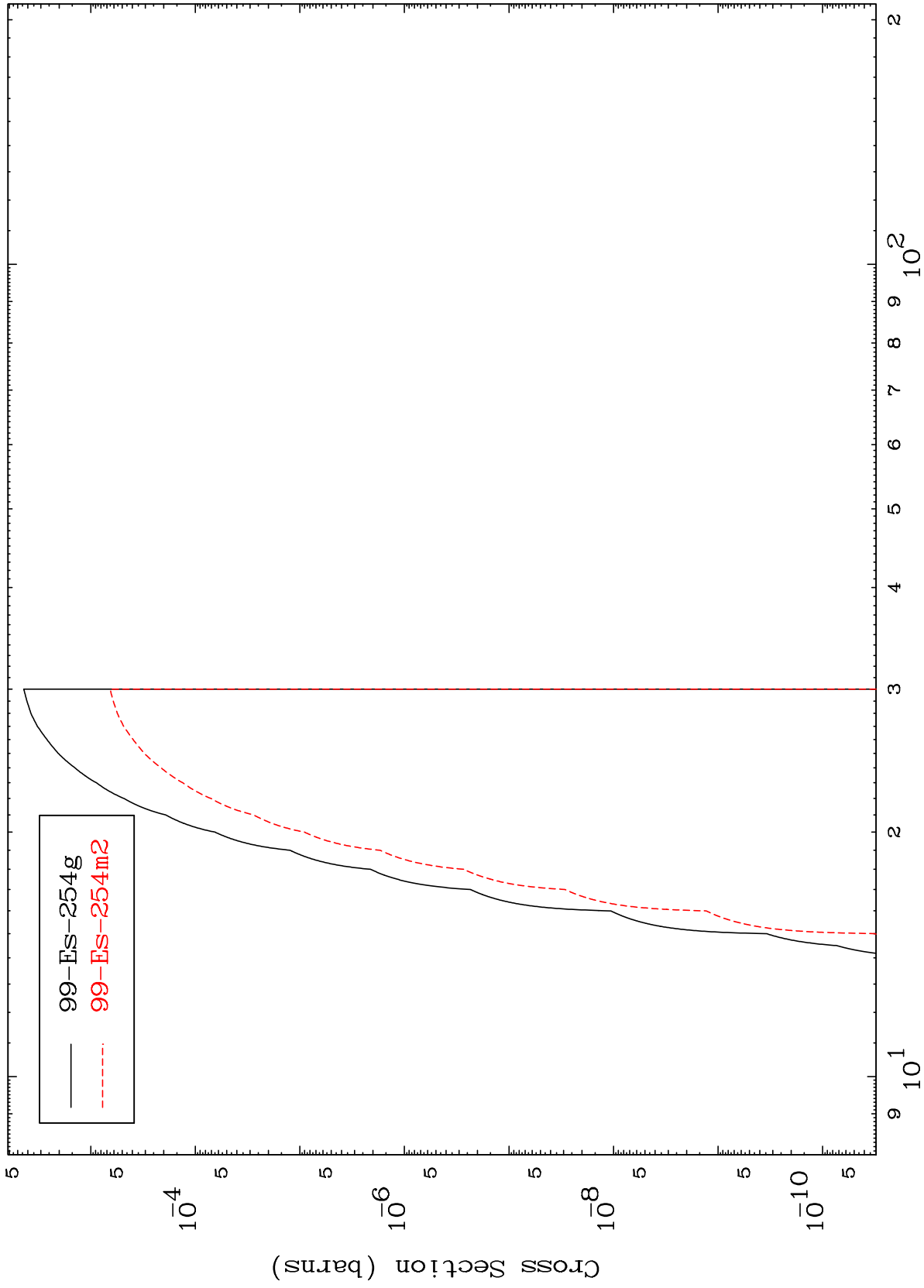
100-Fm-257

MAT 9938

(n,n') t

100-Fm-257

Radionuclide Production Cross Section



27

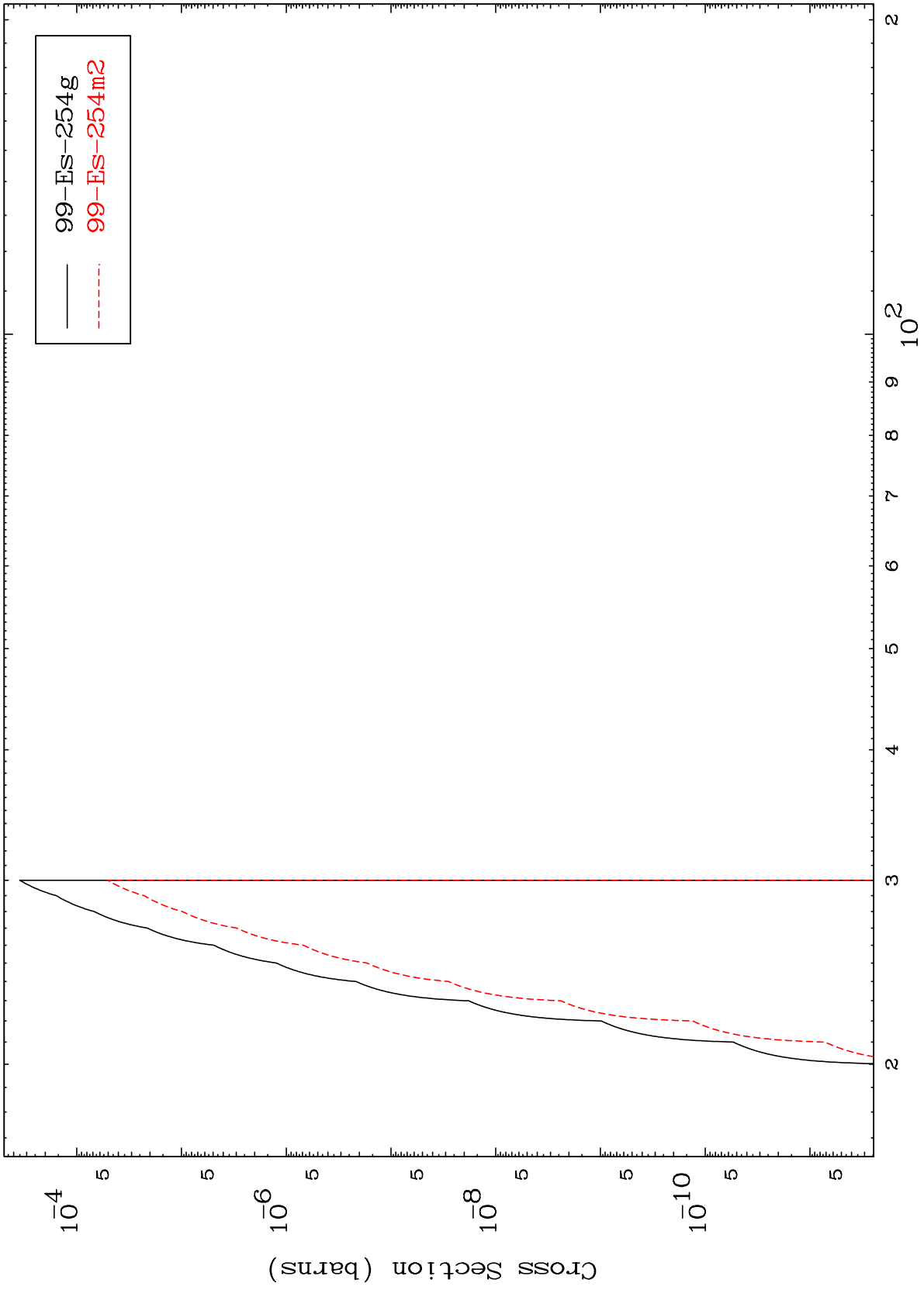
100-Fm-257

MAT 9938

(n,3n) p

100-Fm-257

Radionuclide Production Cross Section



28

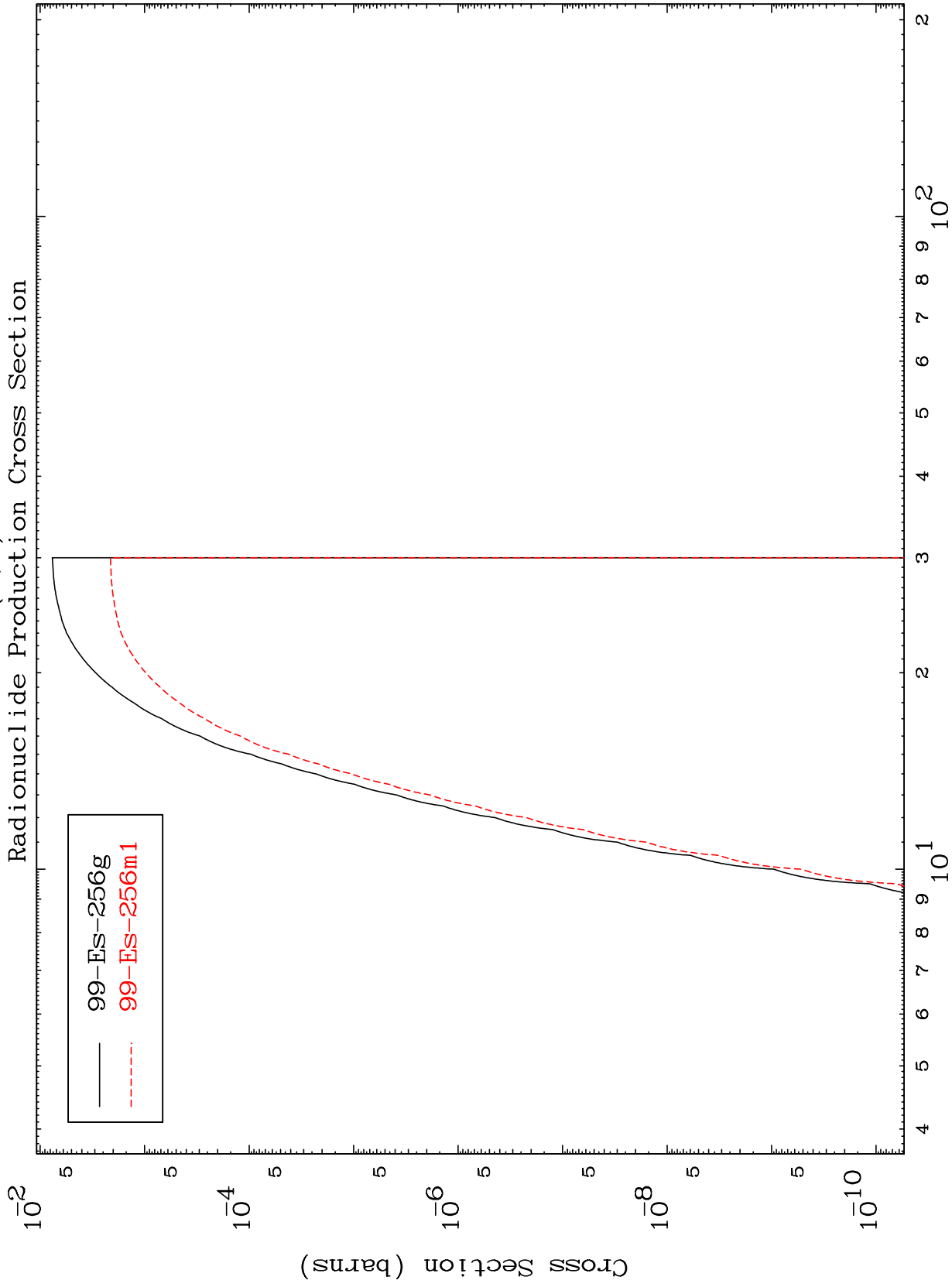
Incident Energy (MeV)

100-Fm-257

MAT 9938

100-Fm-257

(n,d)
Radionuclide Production Cross Section



29

100-Fm-257