

Program EVALPLOT
(Version 2018-1)

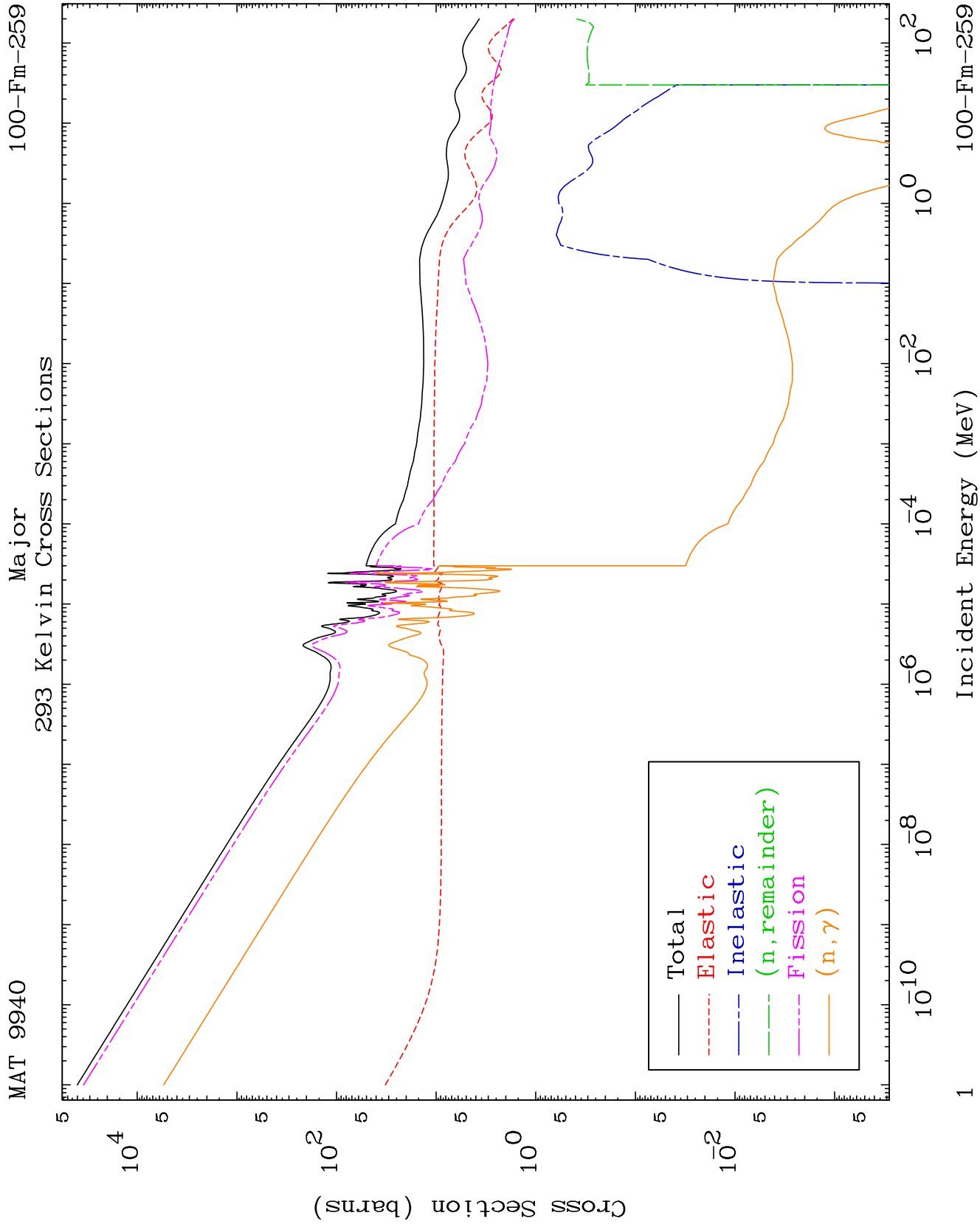
by

Dermott E. Cullen
(Present Contact Information)
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

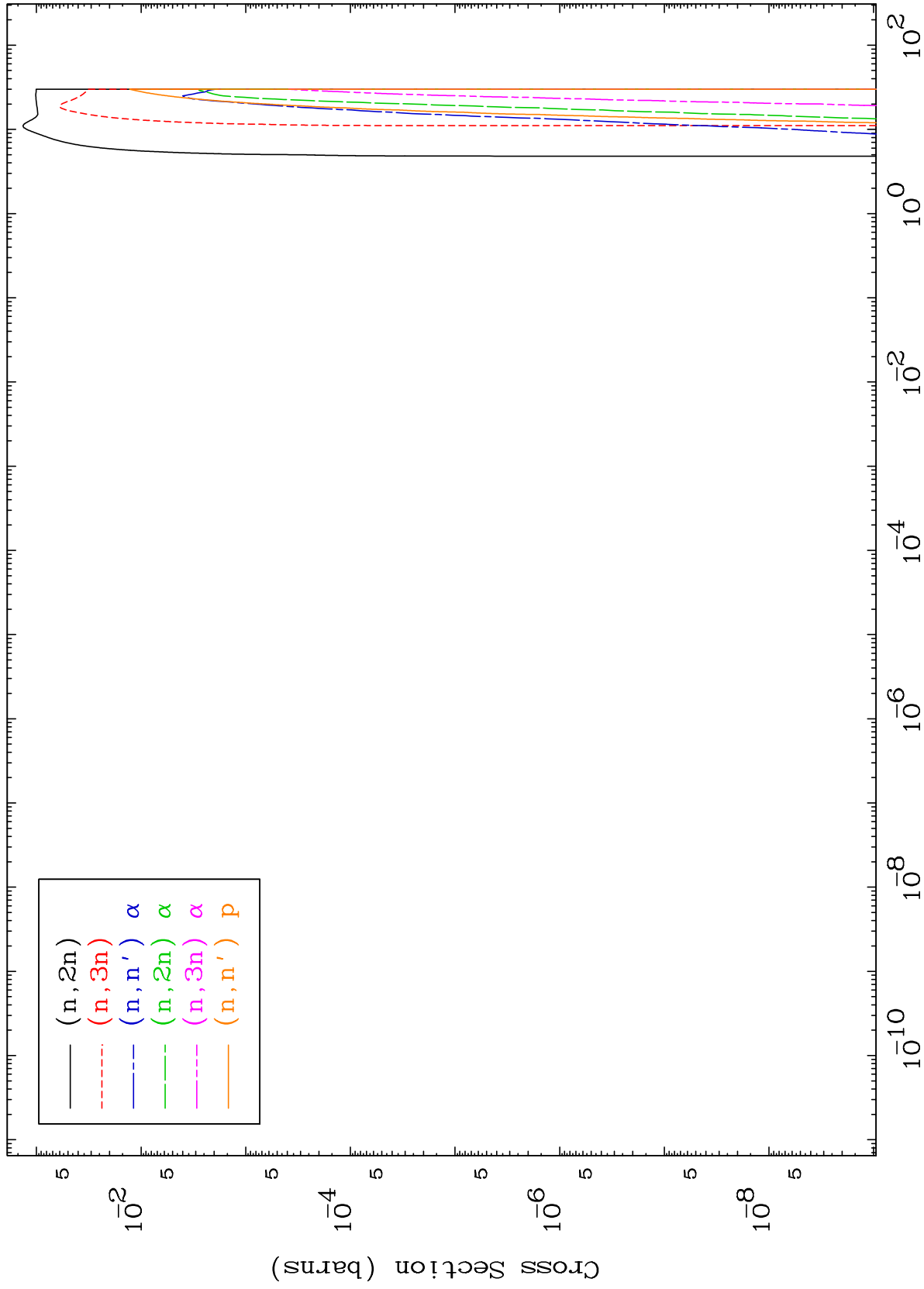
Press Mouse Button to Start



MAT 9940

Neutron Production
293 Kelvin Cross Sections

100-Fm-259



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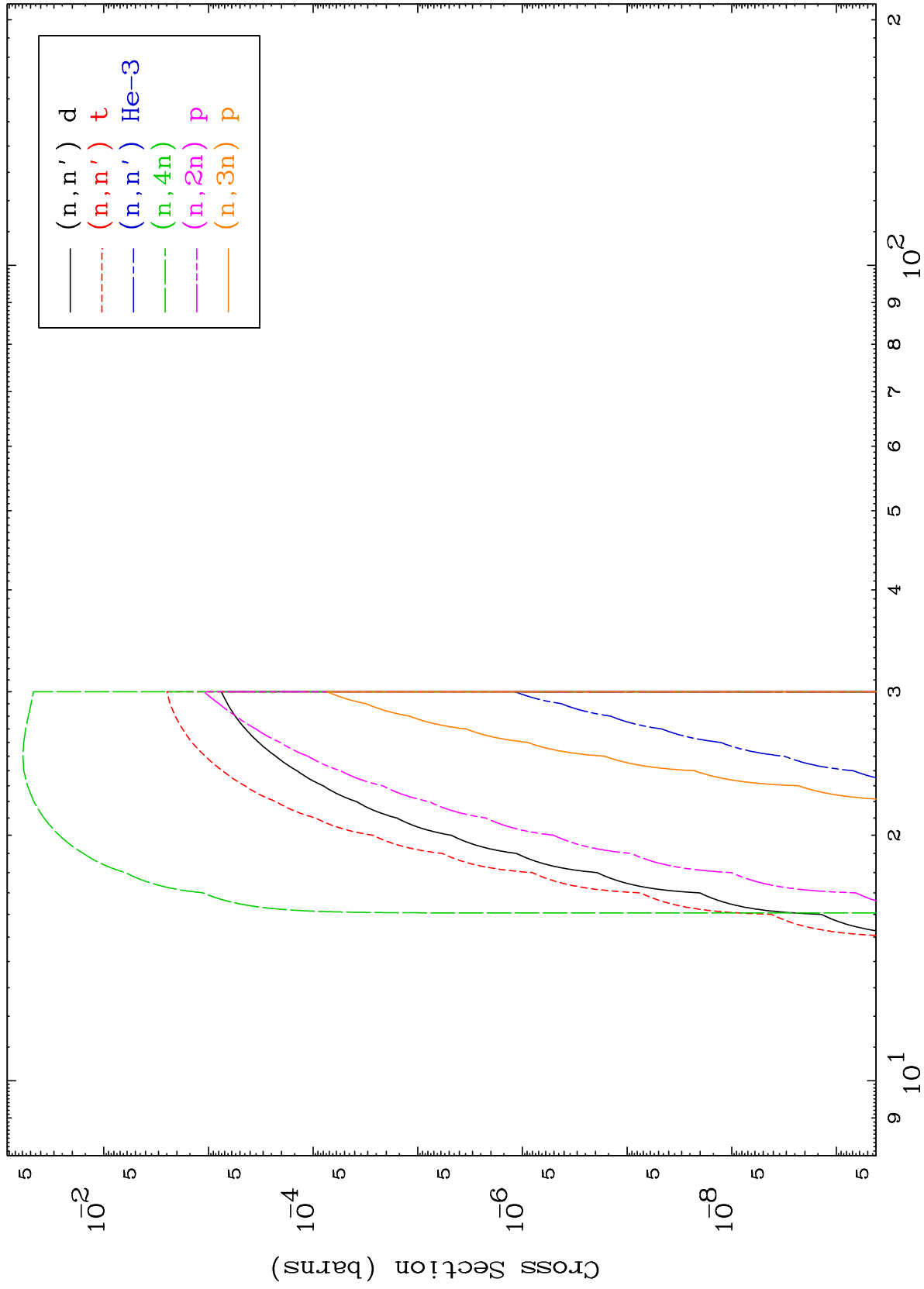
Incident Energy (MeV)

100-Fm-259

MAT 9940

Neutron Production
293 Kelvin Cross Sections

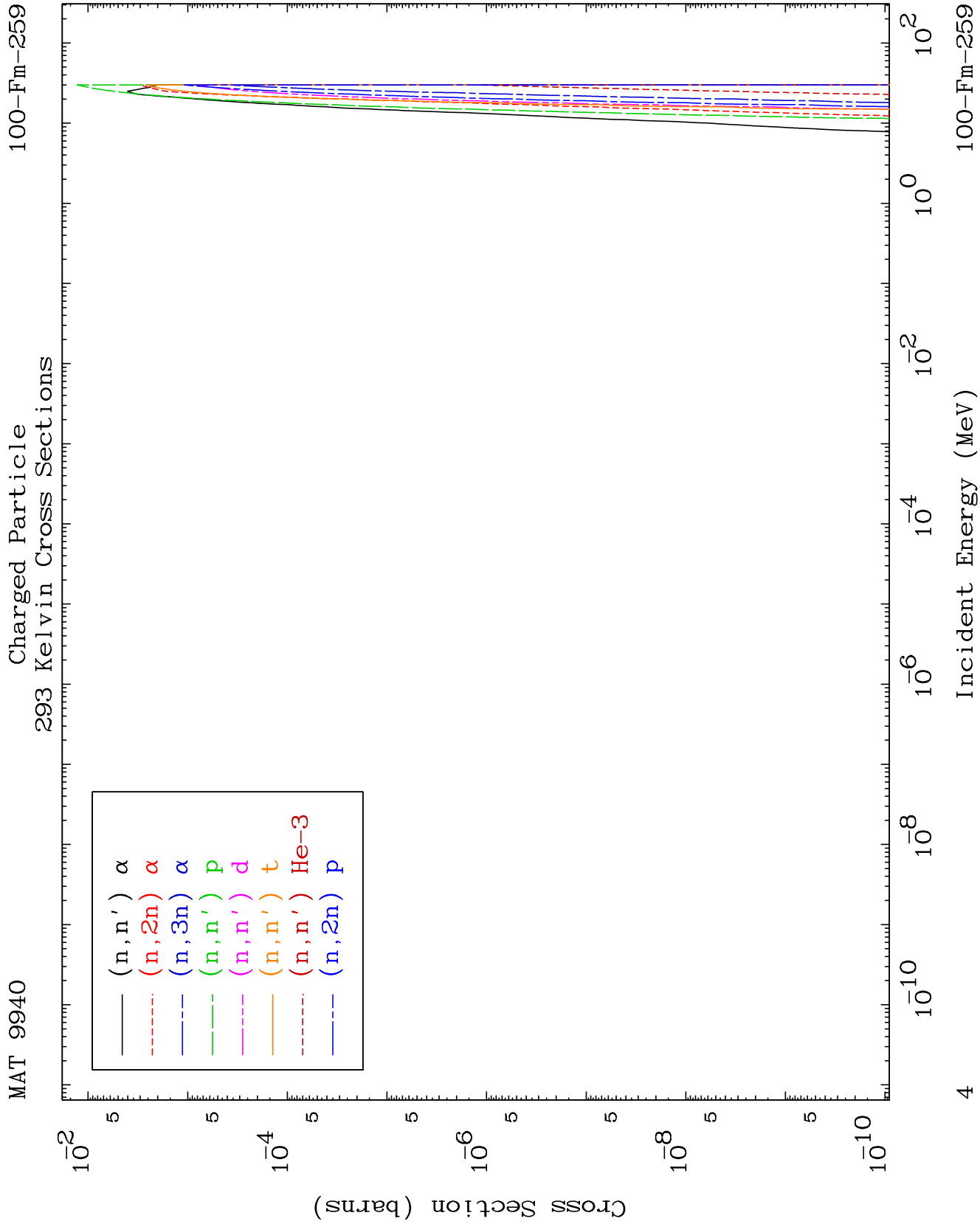
100-Fm-259

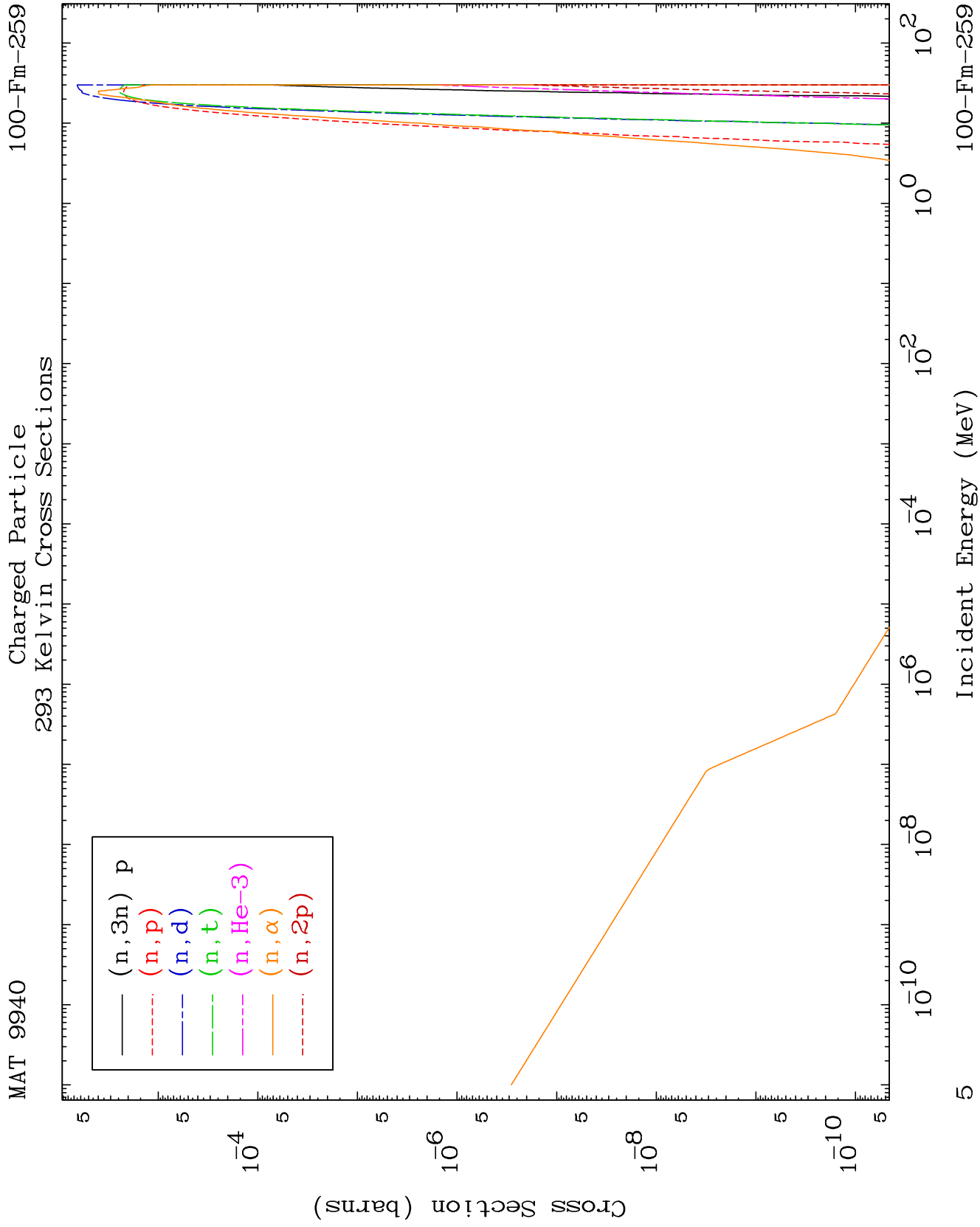


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Incident Energy (MeV)

100-Fm-259

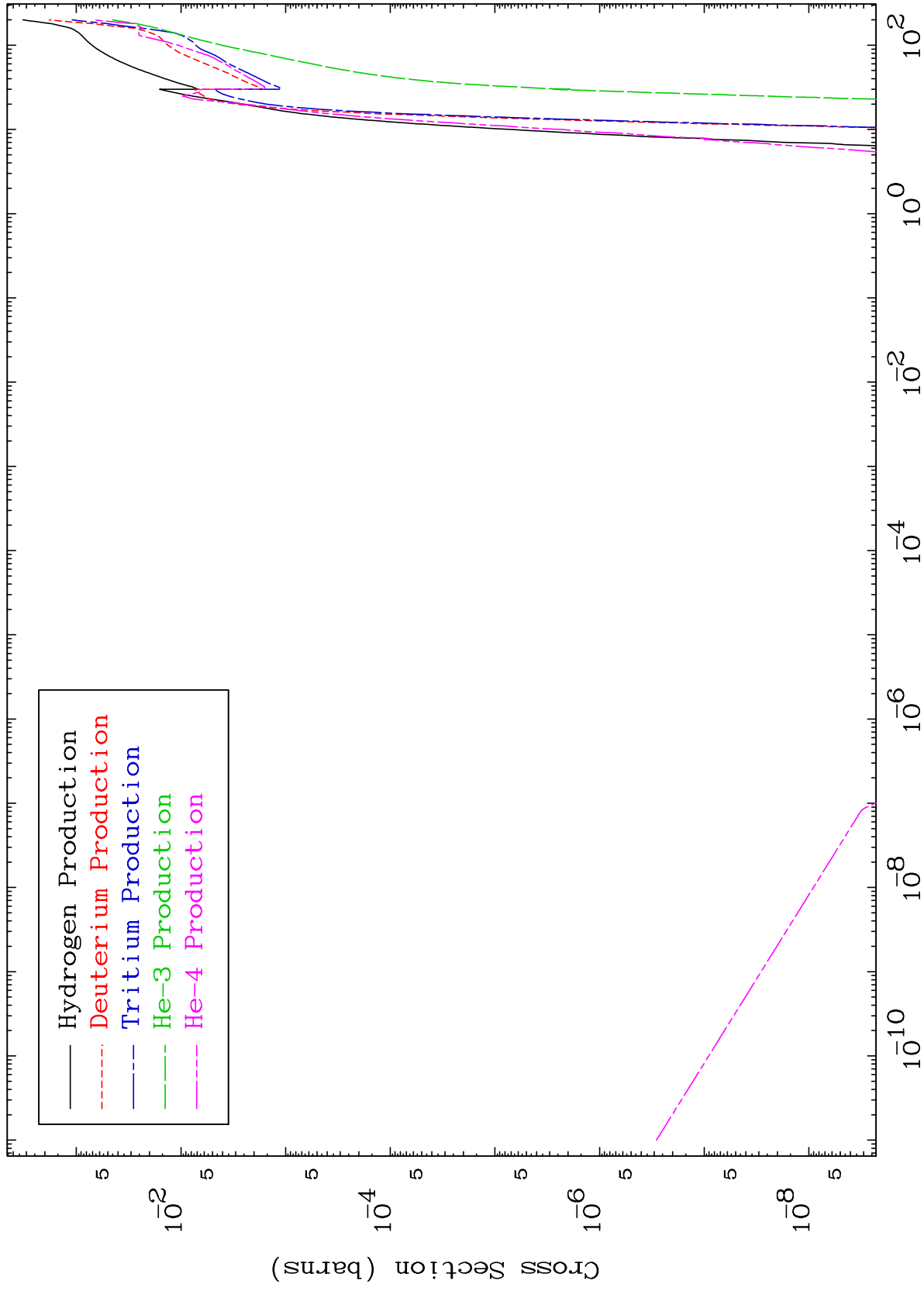




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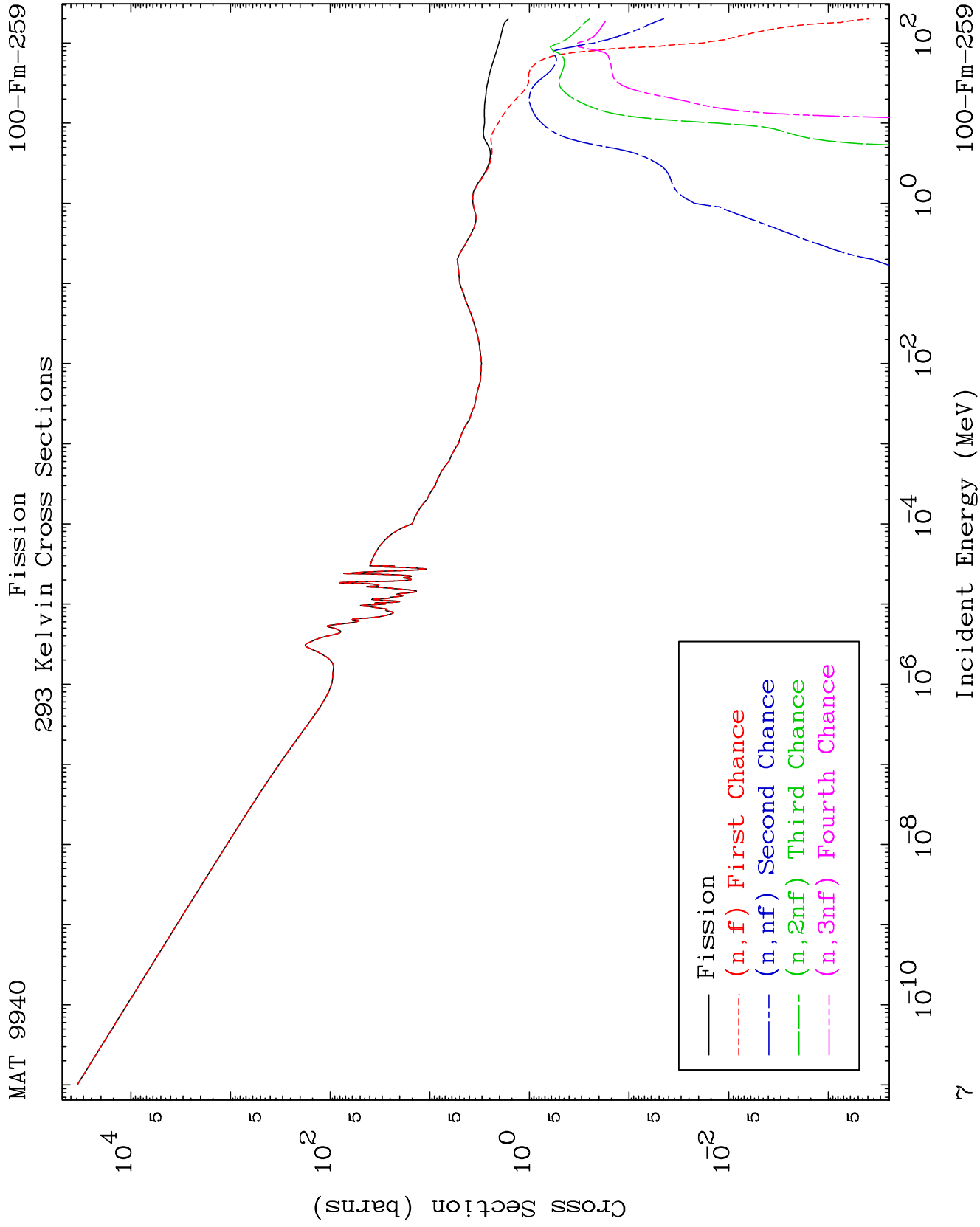
Particle Production
293 Kelvin Cross Sections

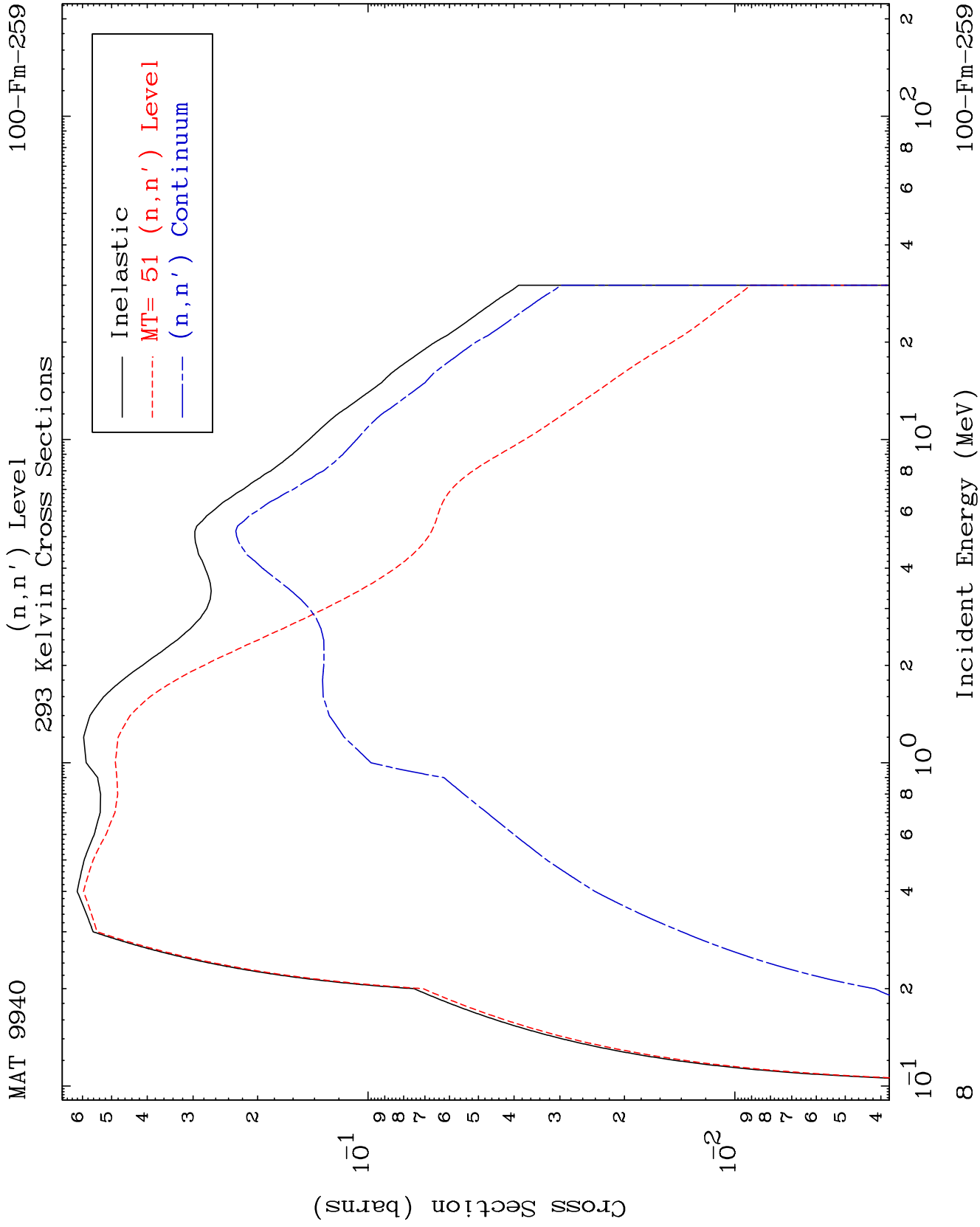
100-Fm-259



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100-Fm-259

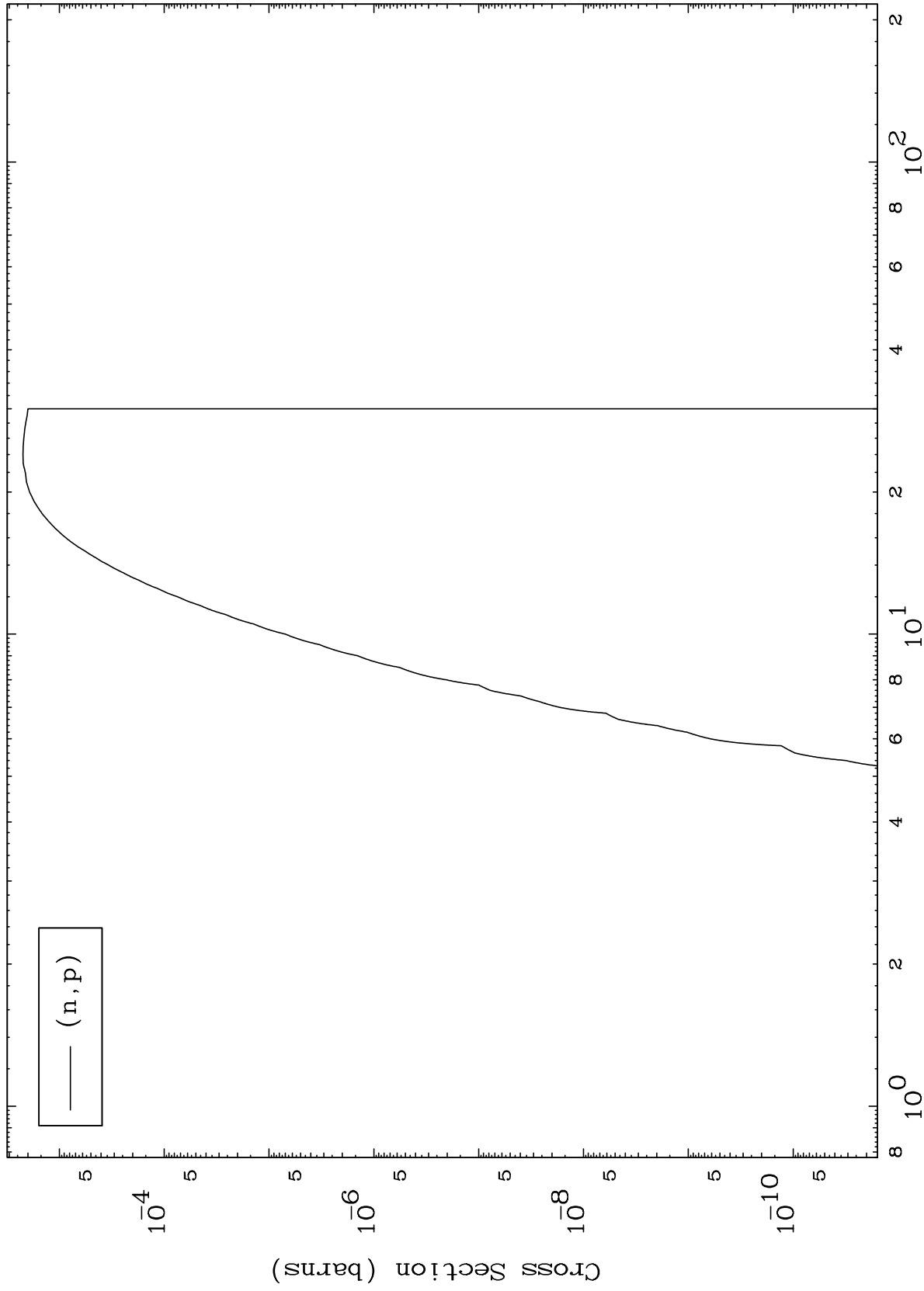




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(n,p) Levels
293 Kelvin Cross Sections

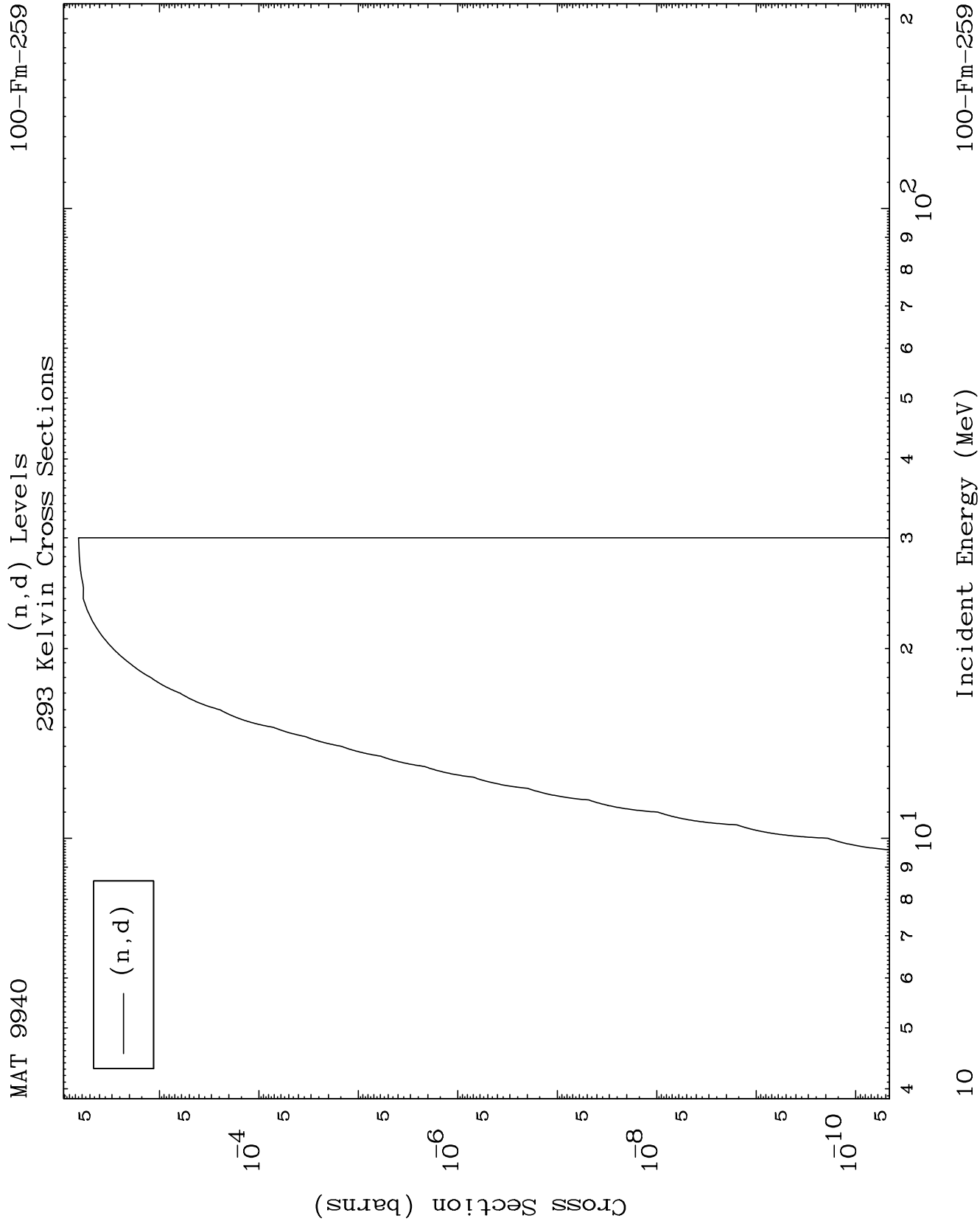
100-Fm-259



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Incident Energy (MeV)

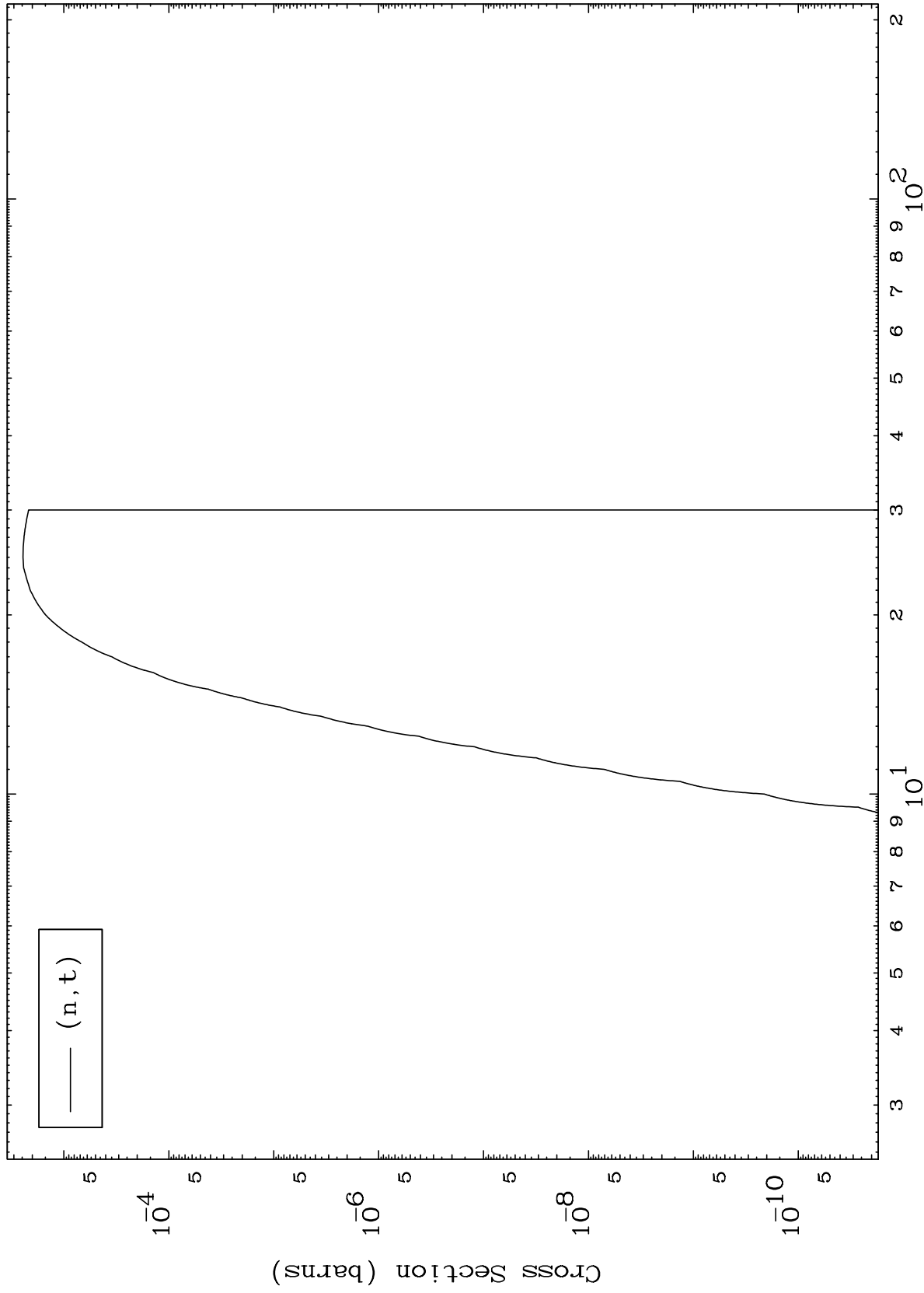
100-Fm-259



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(n,t) Levels
293 Kelvin Cross Sections

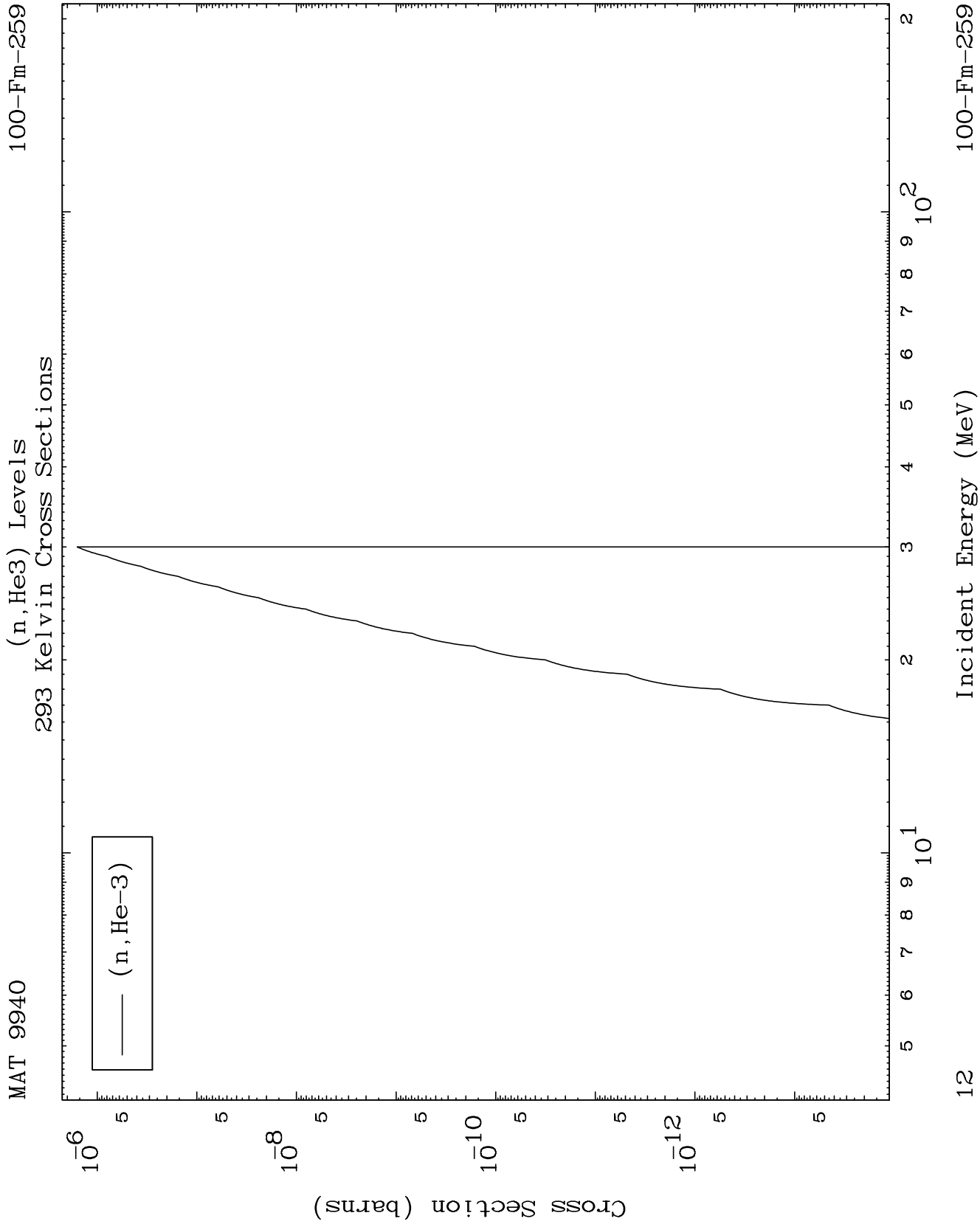
100-Fm-259

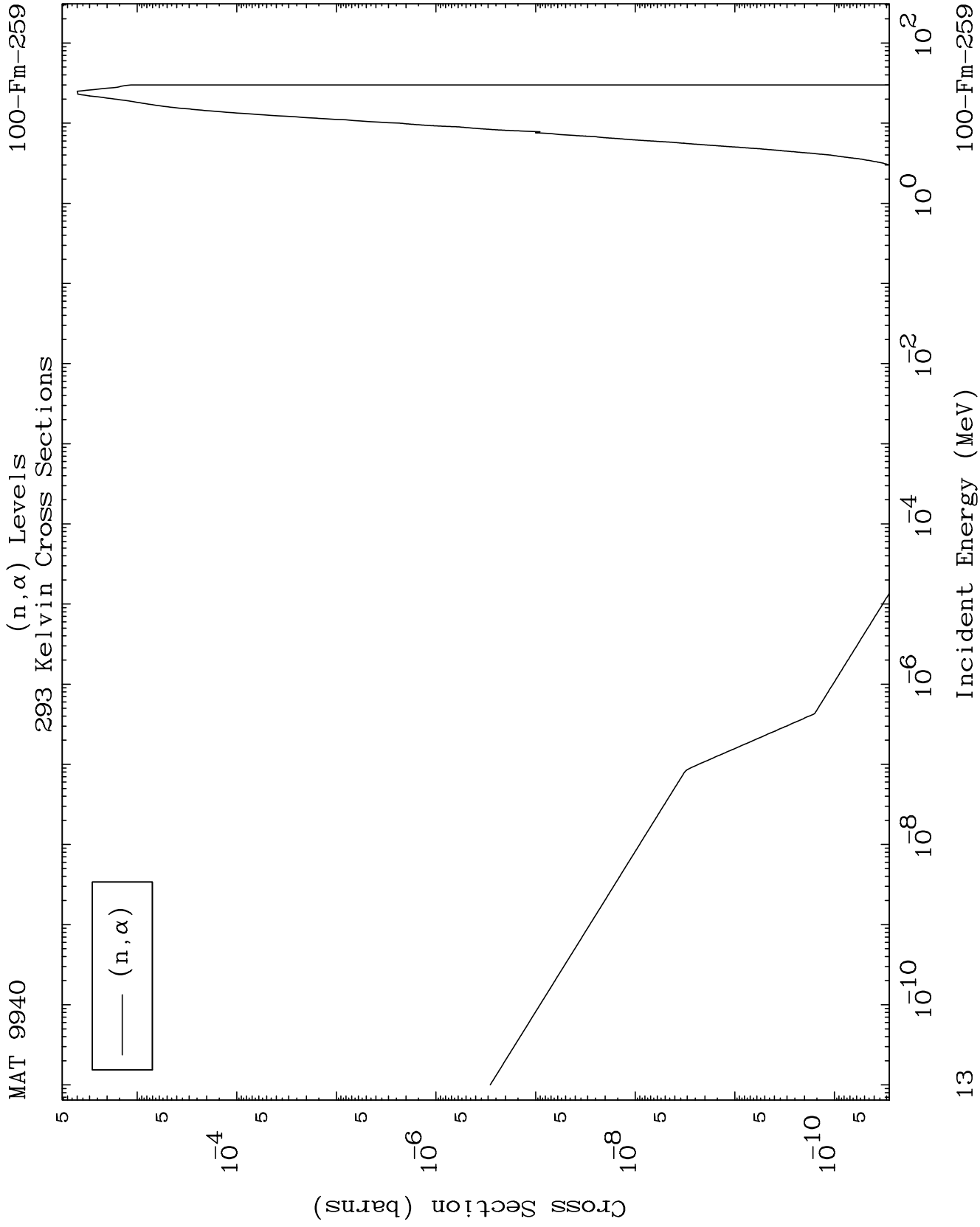


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Incident Energy (MeV)

100-Fm-259

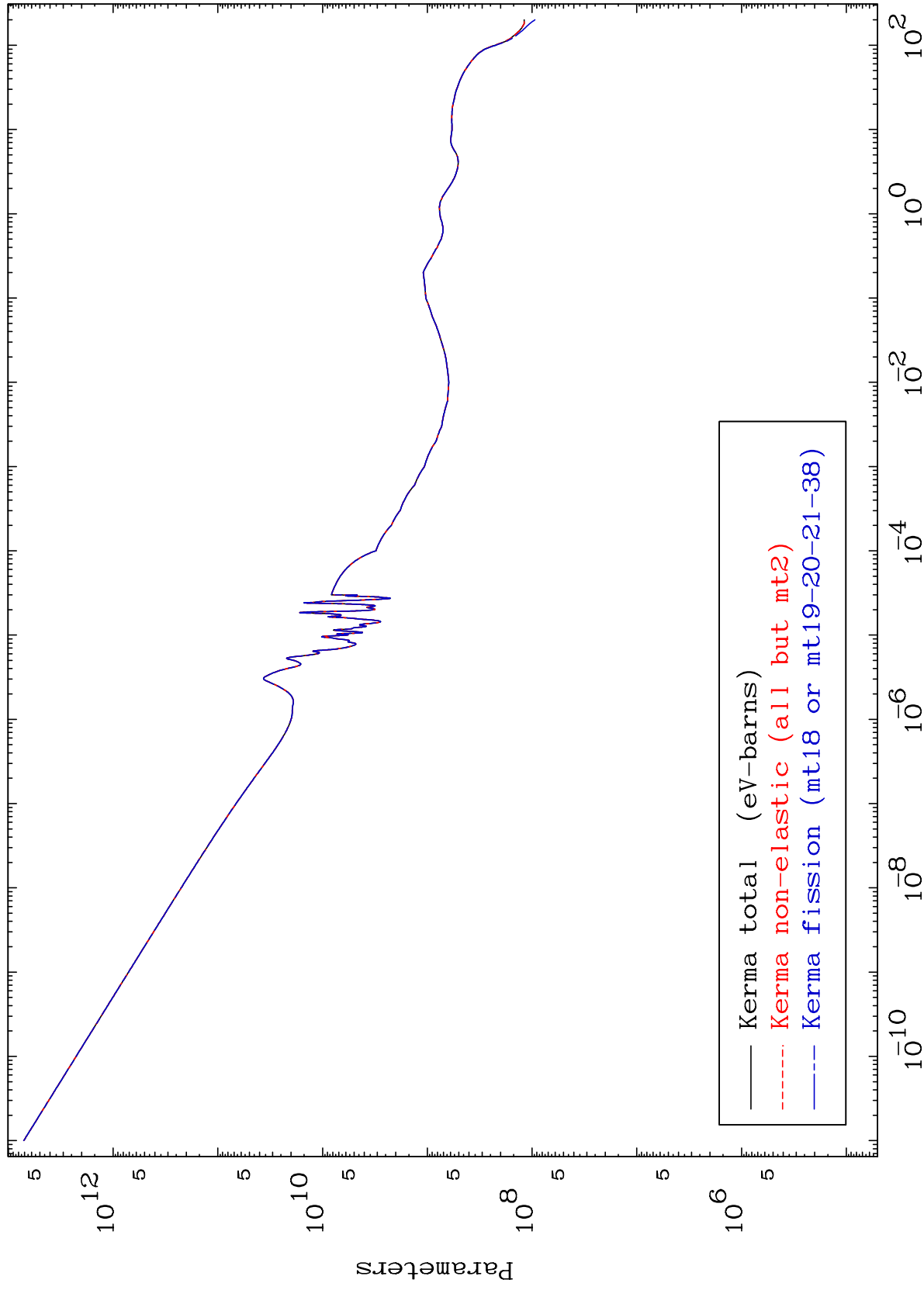




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Energy Release
Parameters

100-Fm-259

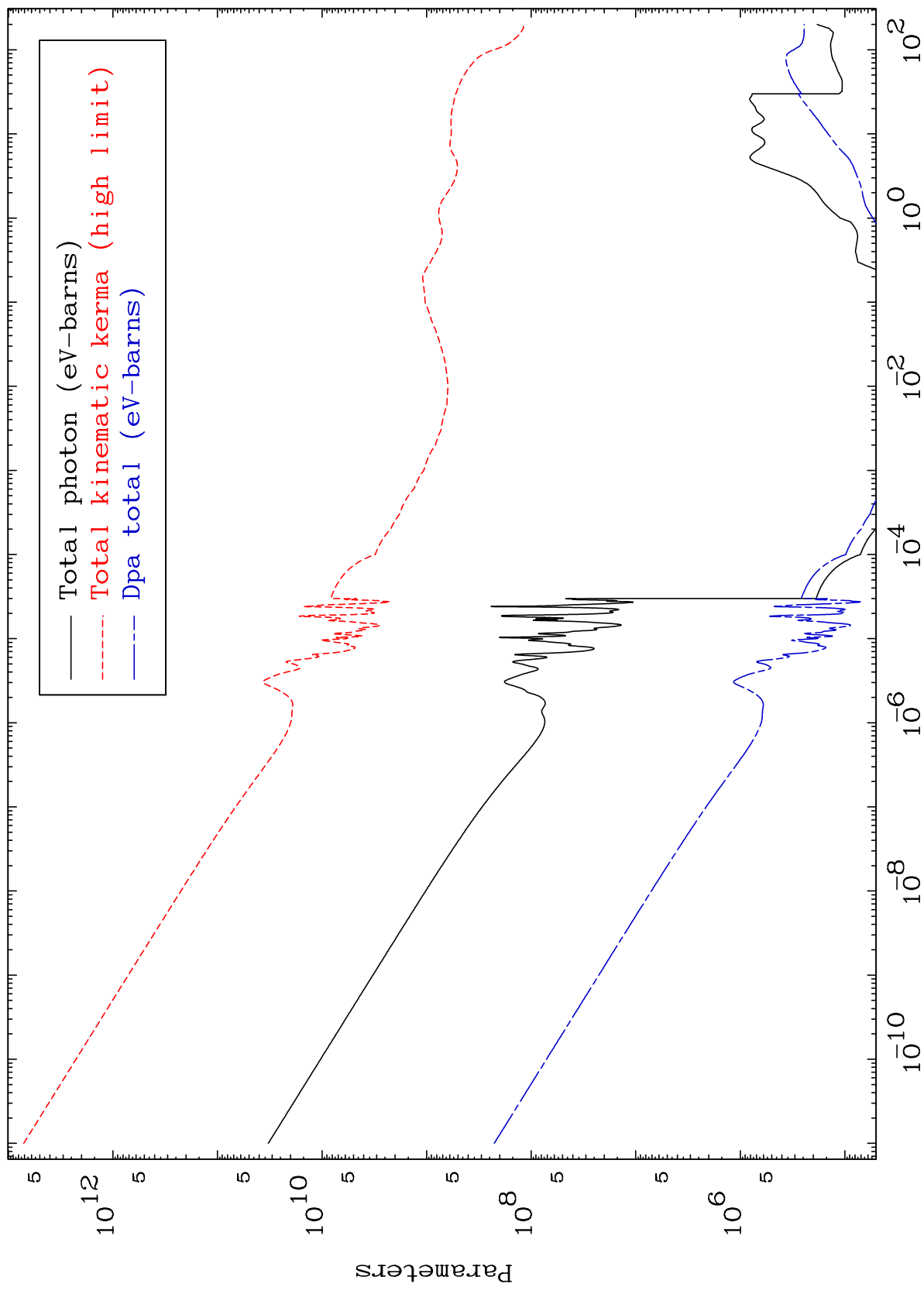


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Energy Release Parameters

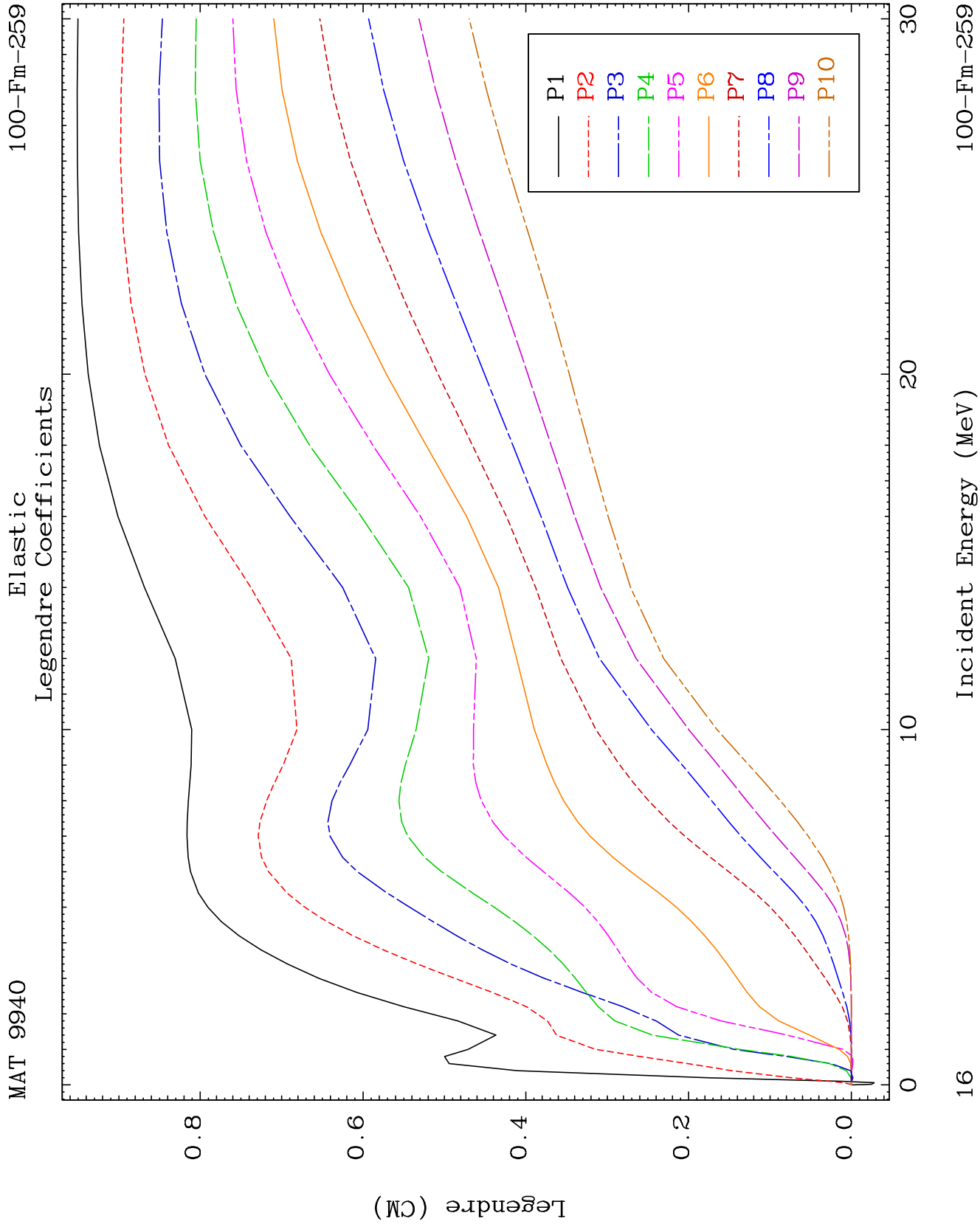
100-Fm-259

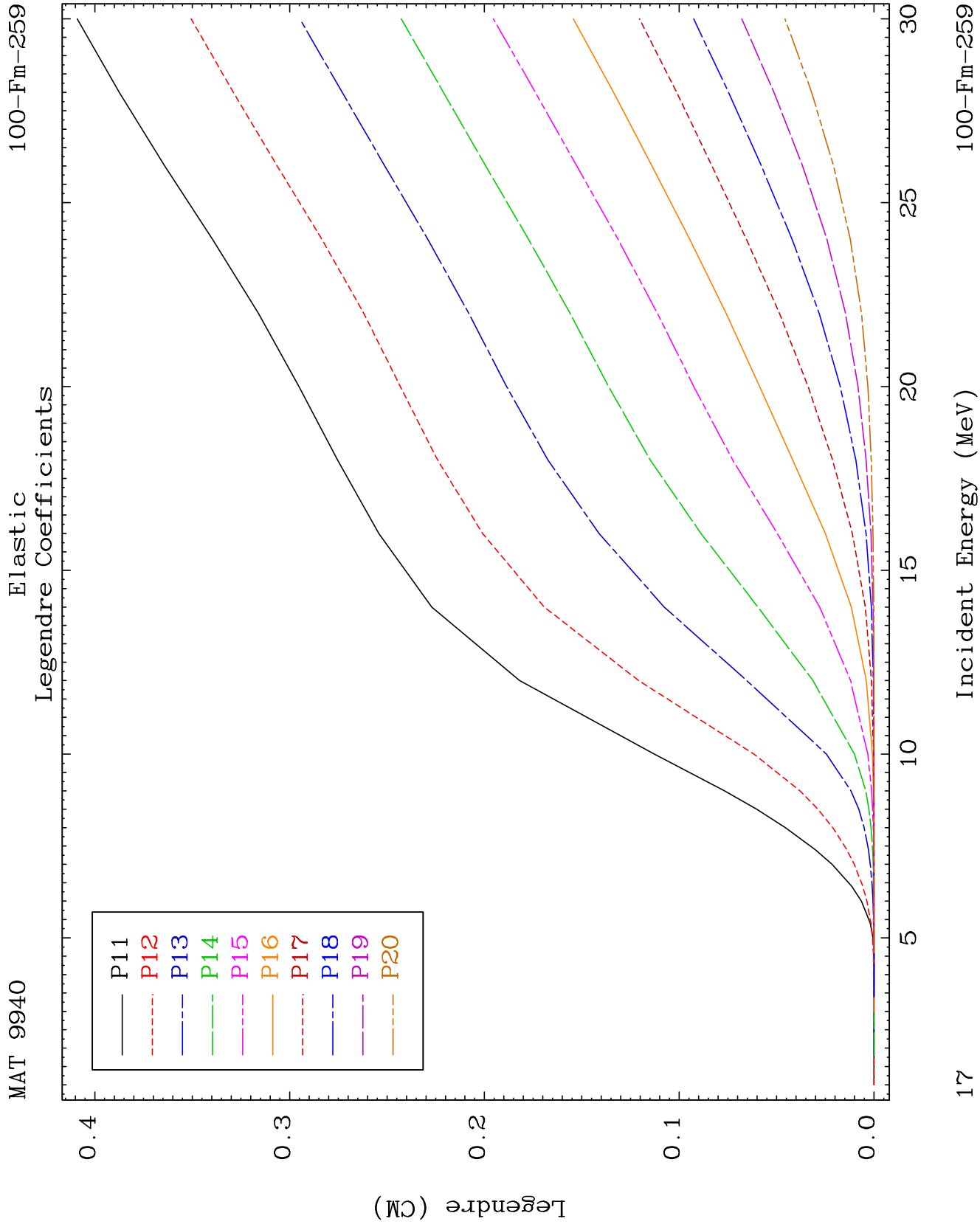


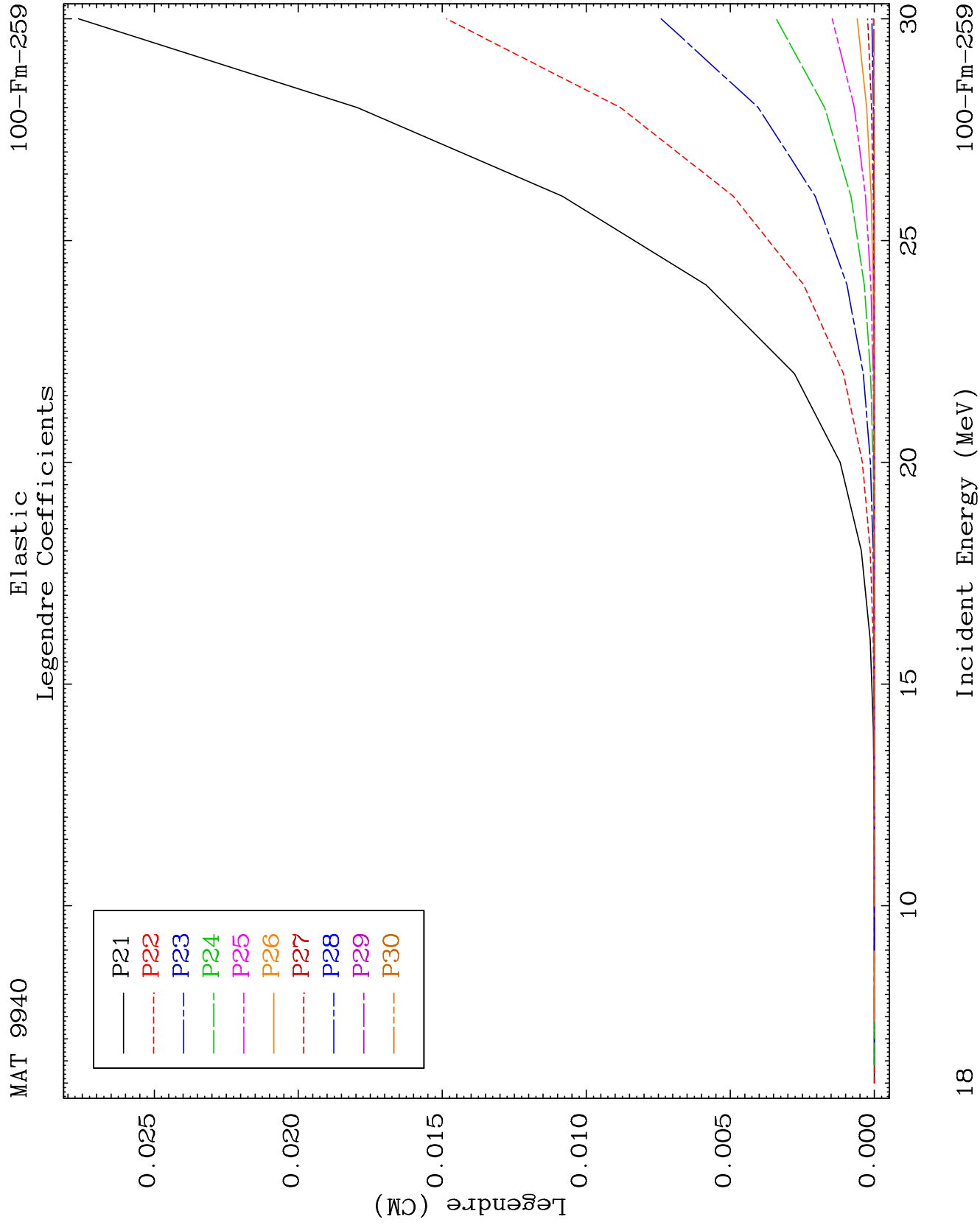
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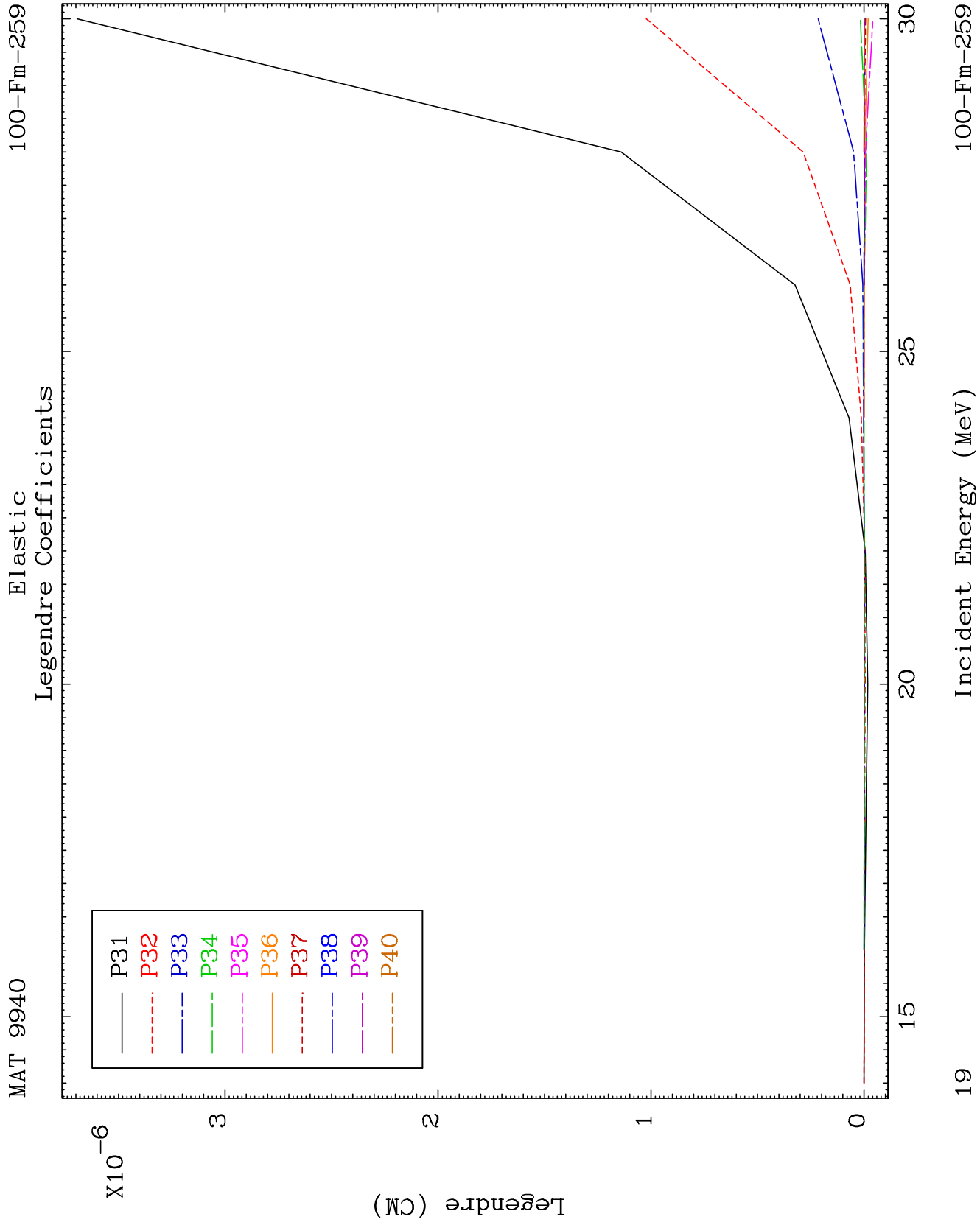
Incident Energy (MeV)

100-Fm-259





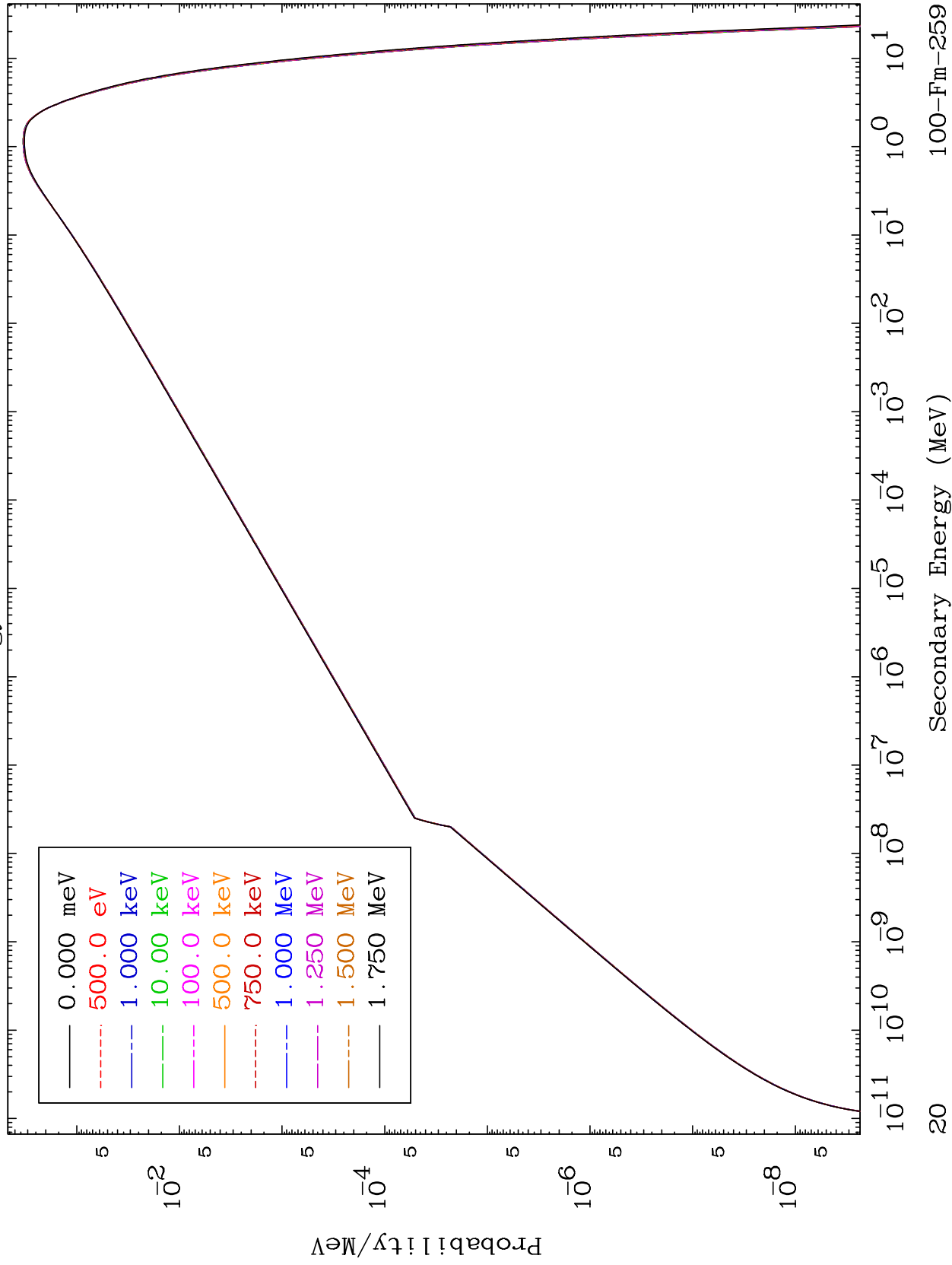




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Fission
Energy Distribution

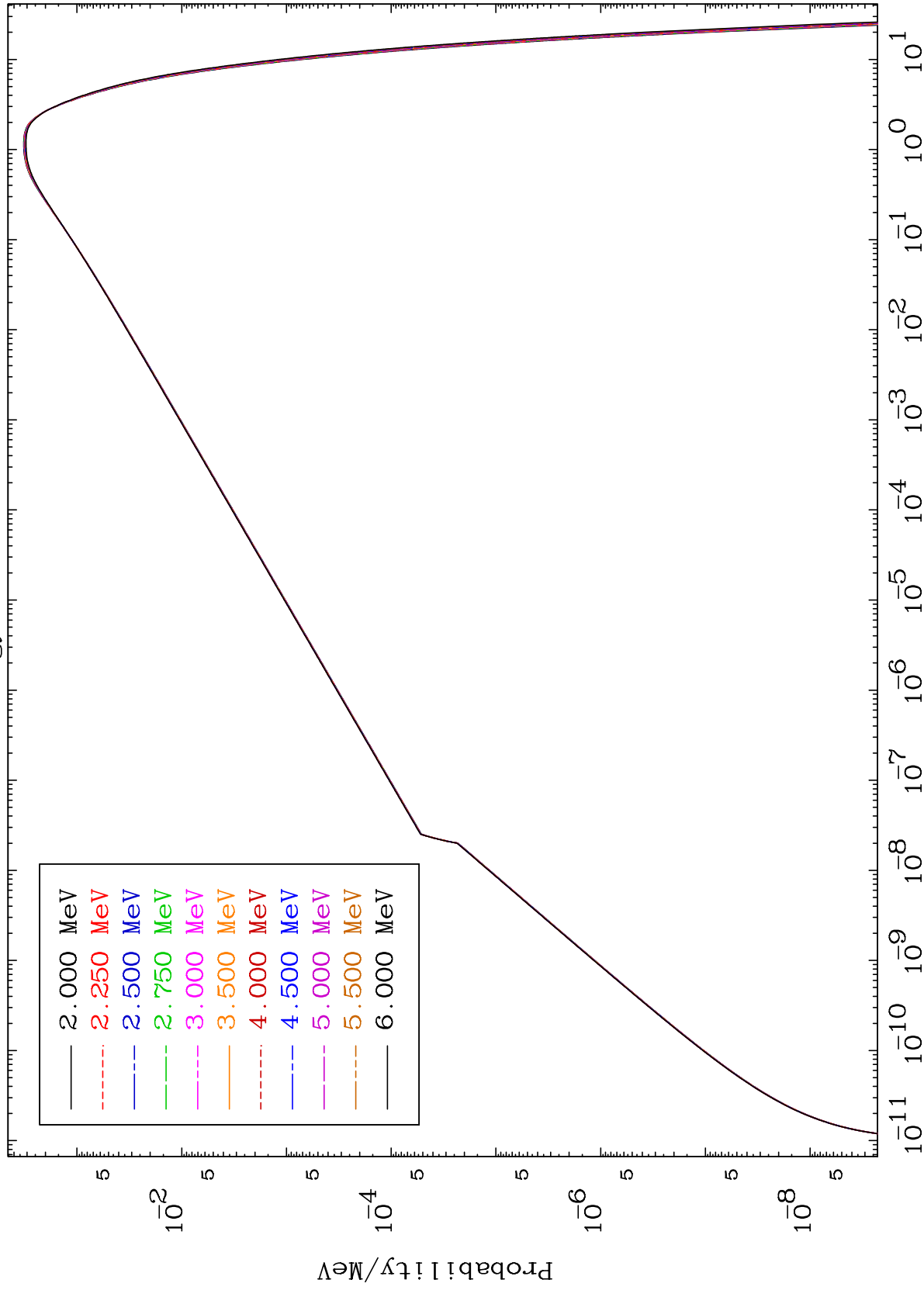
100-Fm-259



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Fission
Energy Distribution

100-Fm-259

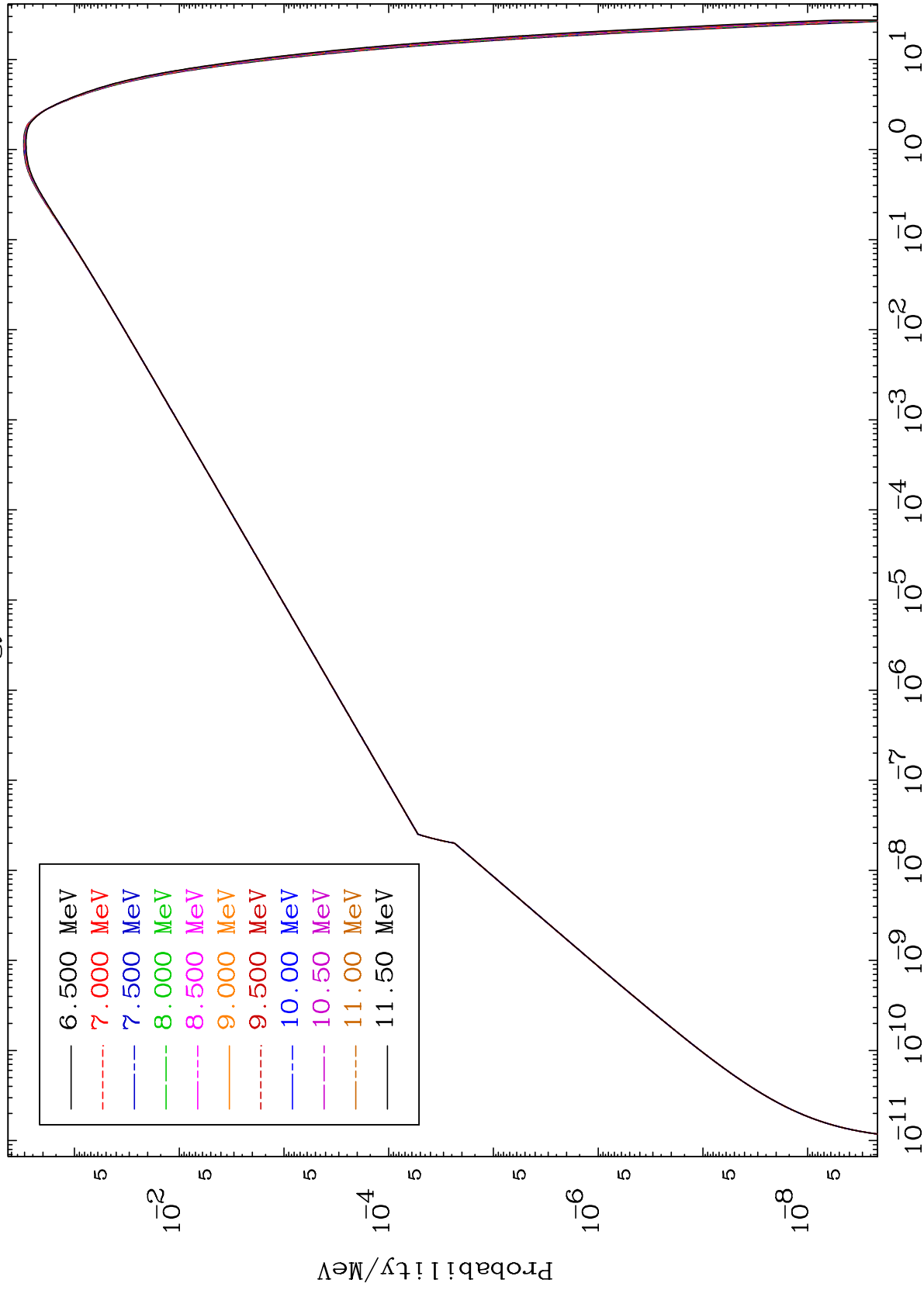


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Fission
Energy Distribution

100-Fm-259



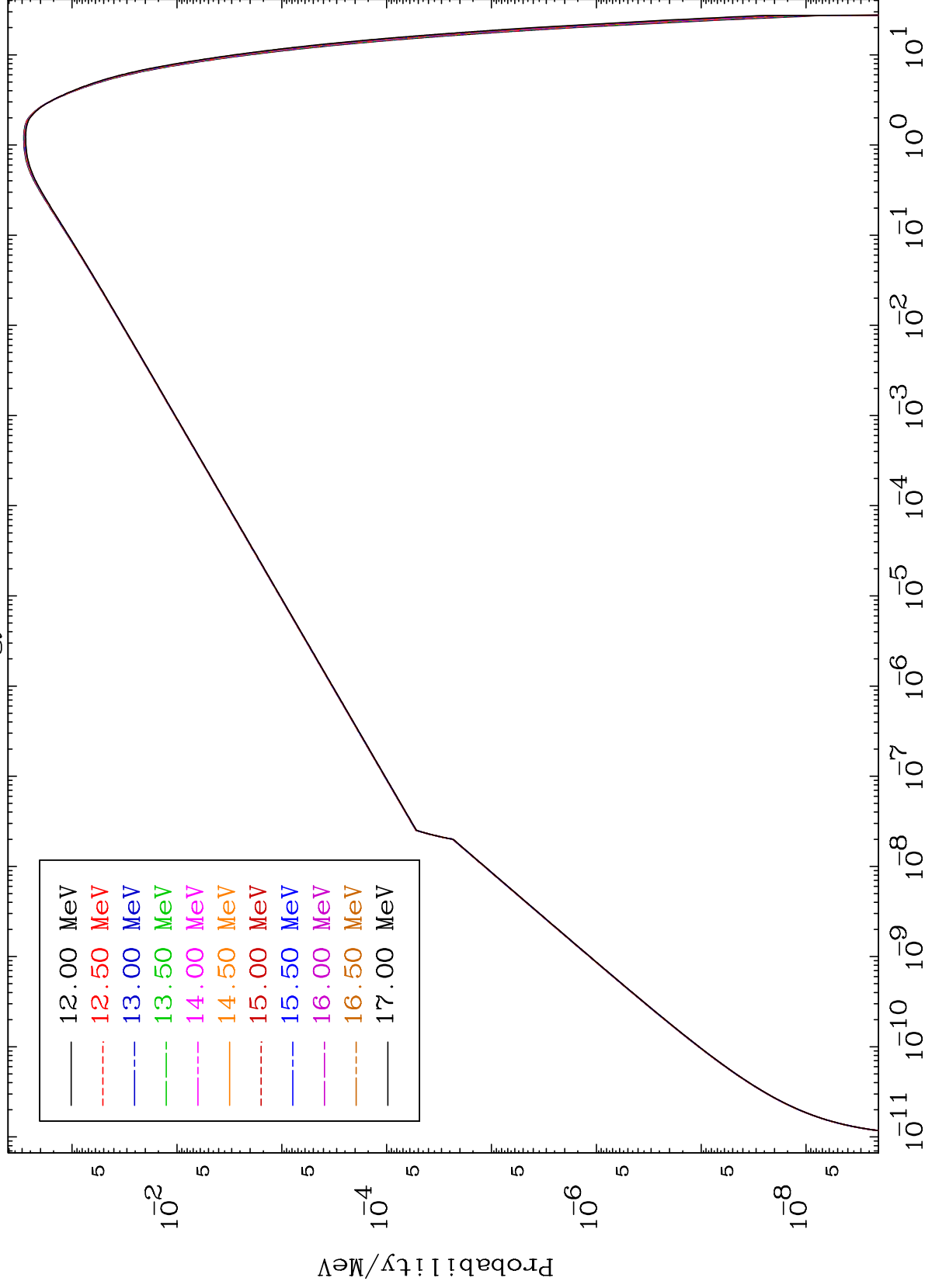
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100-Fm-259

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Fission
Energy Distribution

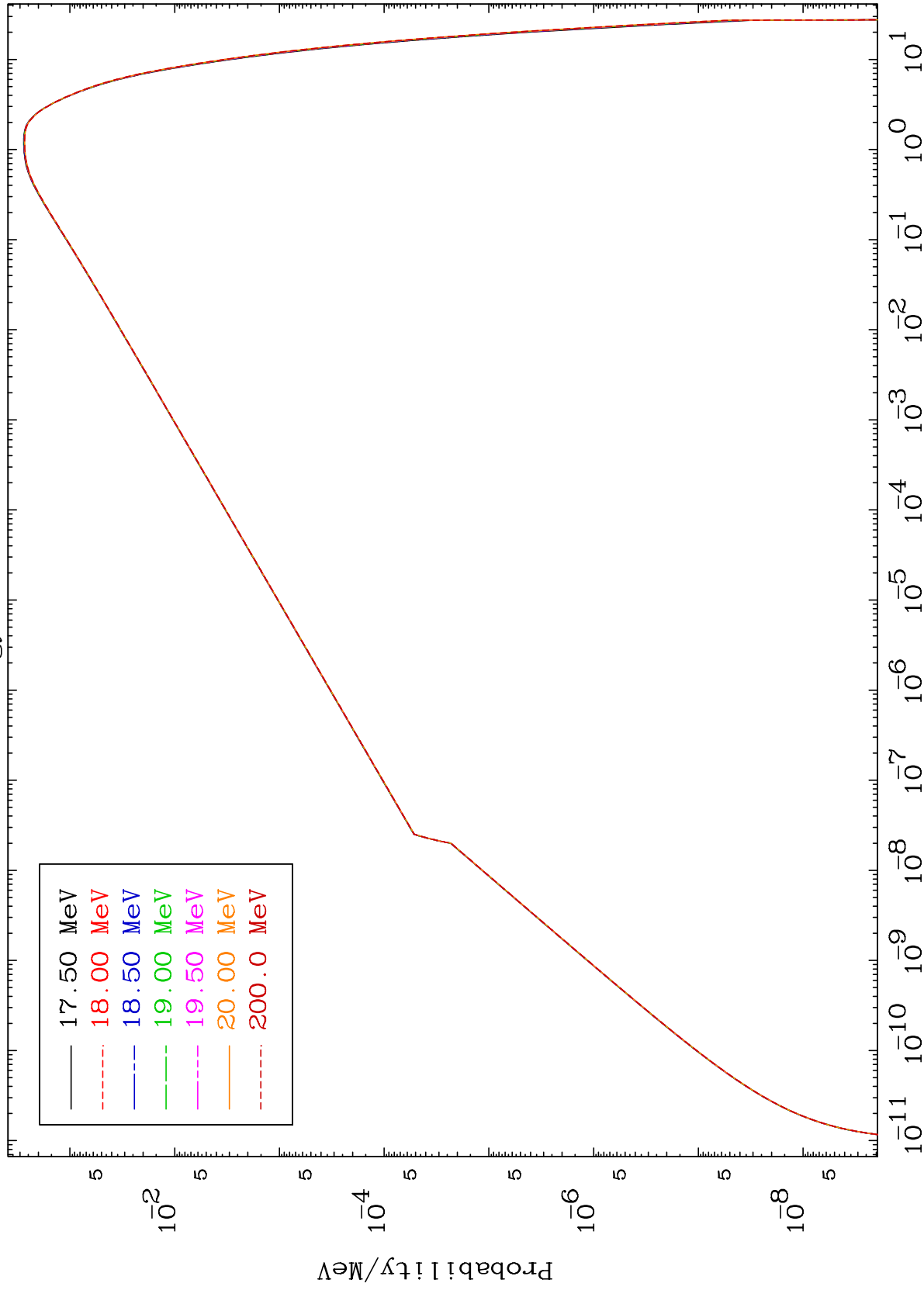
100-Fm-259



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Fission
Energy Distribution

100-Fm-259



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Secondary Energy (MeV)

100-Fm-259

