

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

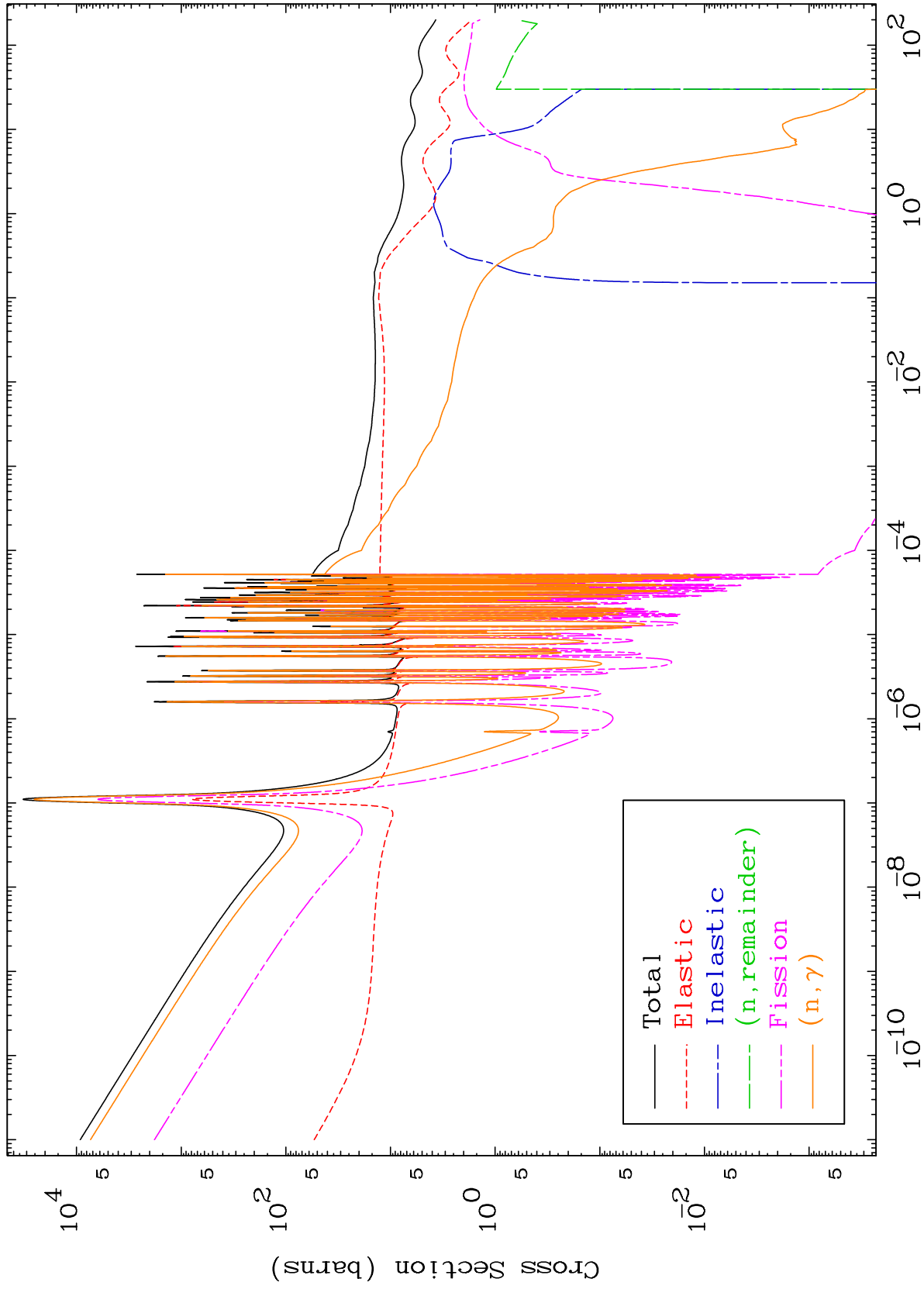
Press Mouse Button to Start

MAT 9986

Major

293 Kelvin Cross Sections

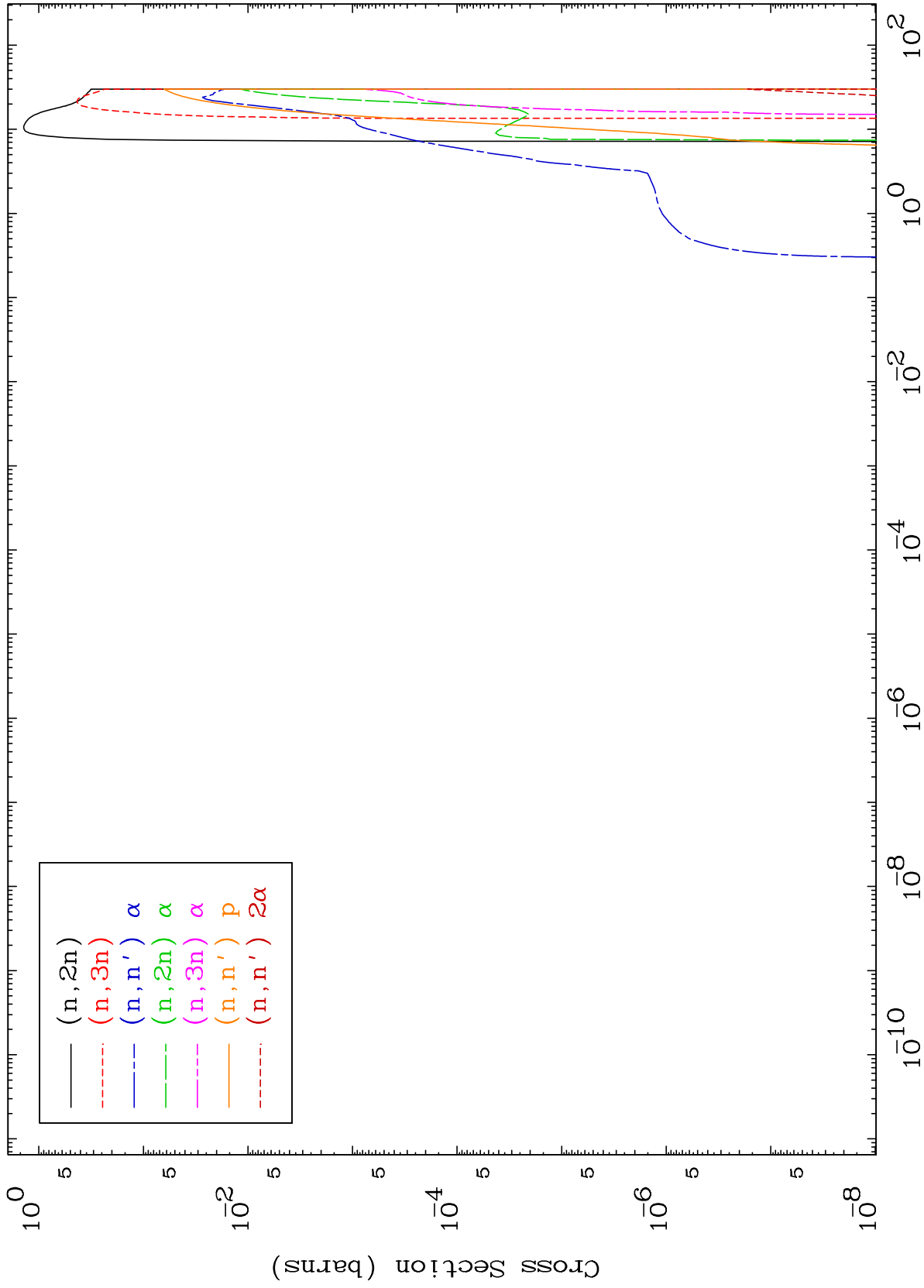
103-Lr-257



MAT 9986

Neutron Production  
293 Kelvin Cross Sections

103-Lr-257

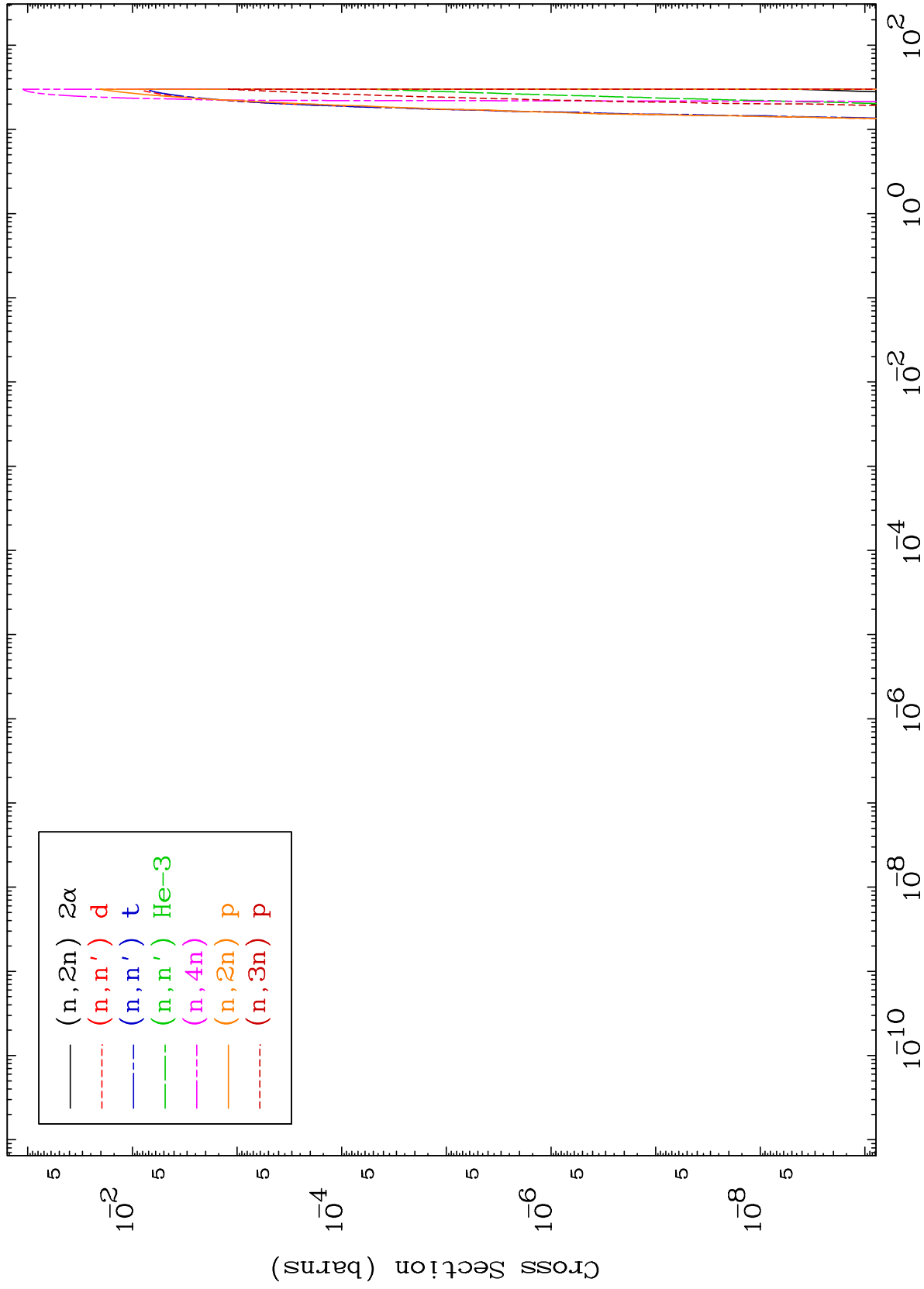


103-Lr-257

MAT 9986

Neutron Production  
293 Kelvin Cross Sections

103-Lr-257

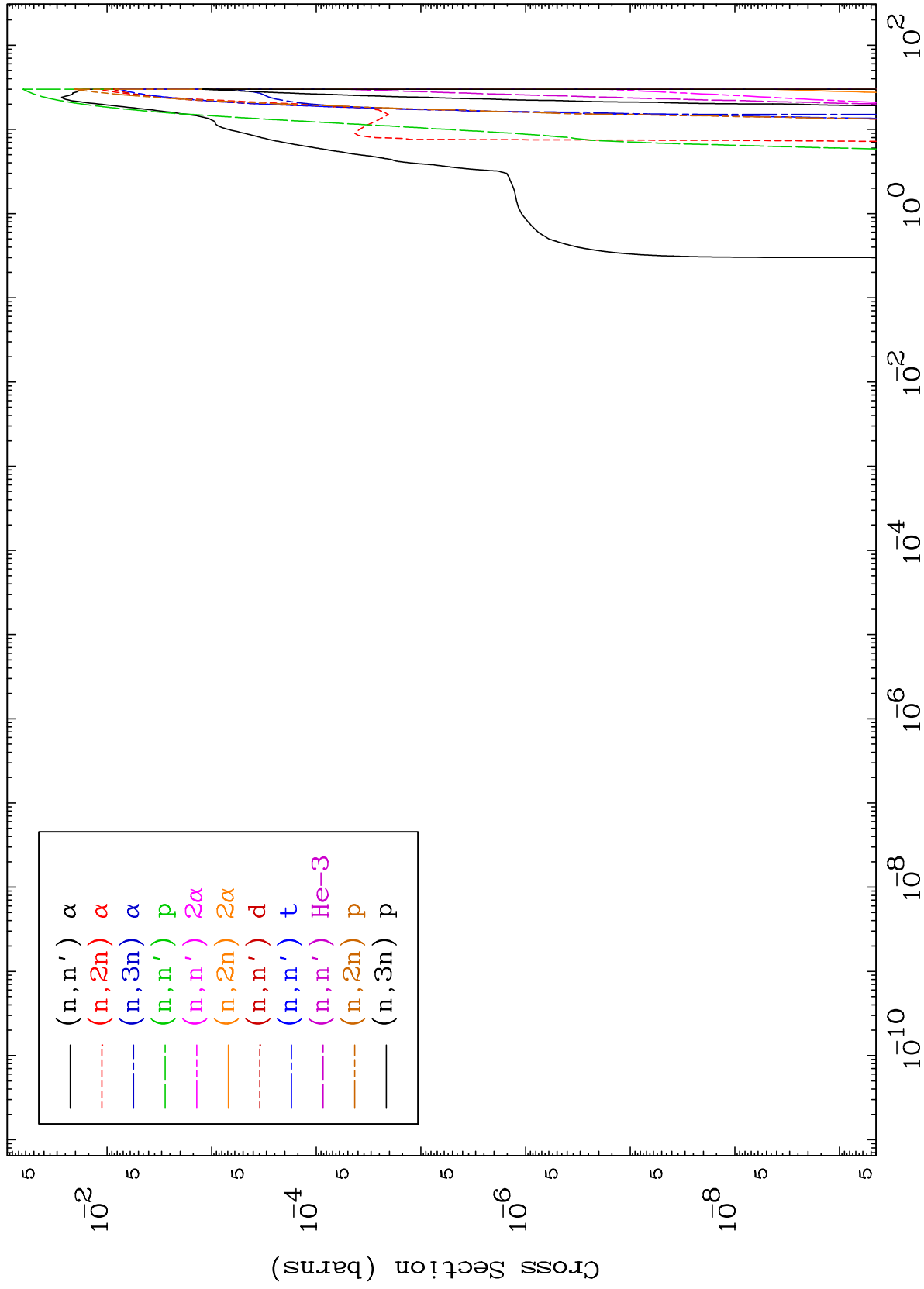


103-Lr-257

MAT 9986

Charged Particle  
293 Kelvin Cross Sections

103-Lr-257

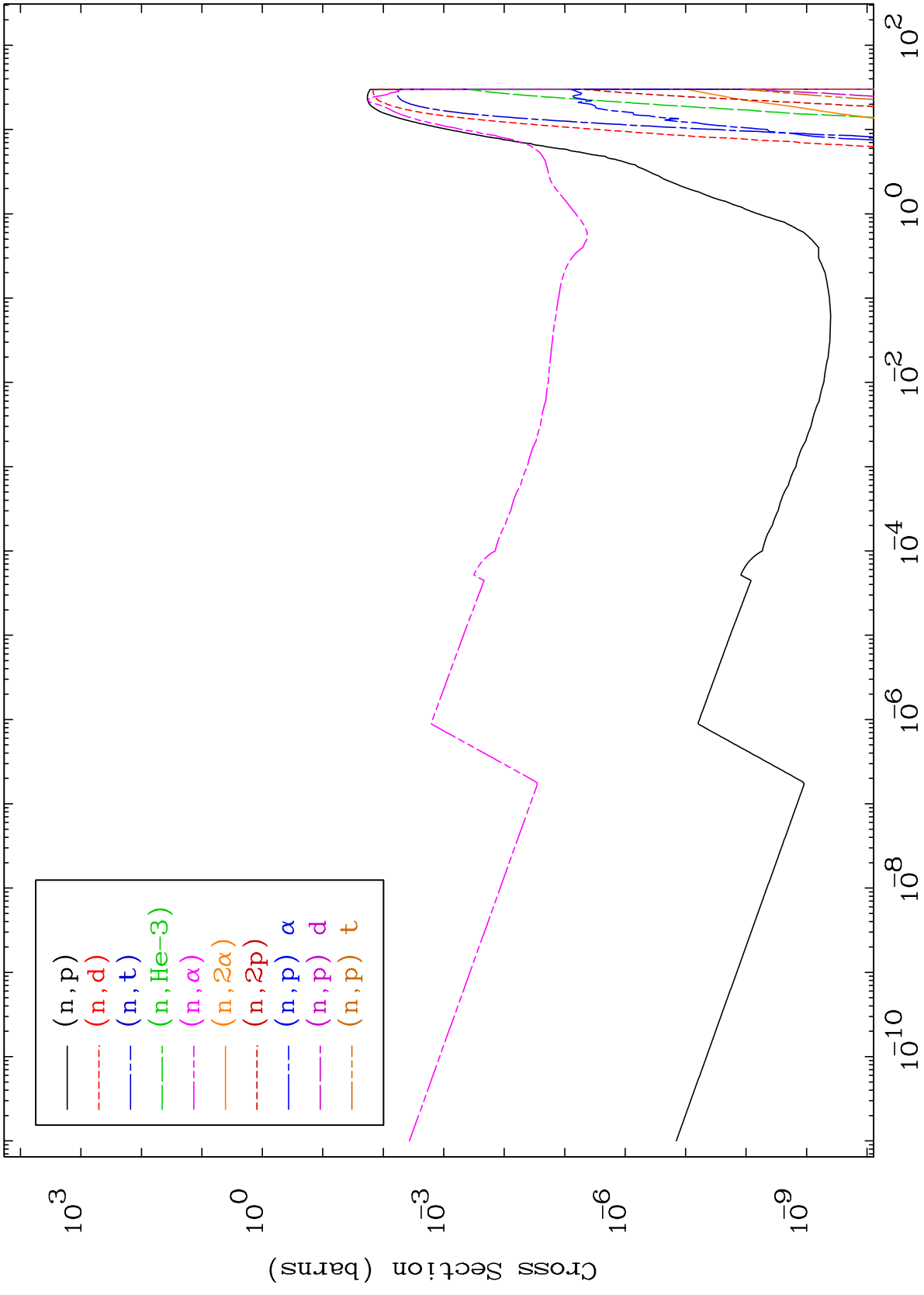


103-Lr-257

MAT 9986

Charged Particle  
293 Kelvin Cross Sections

103-Lr-257

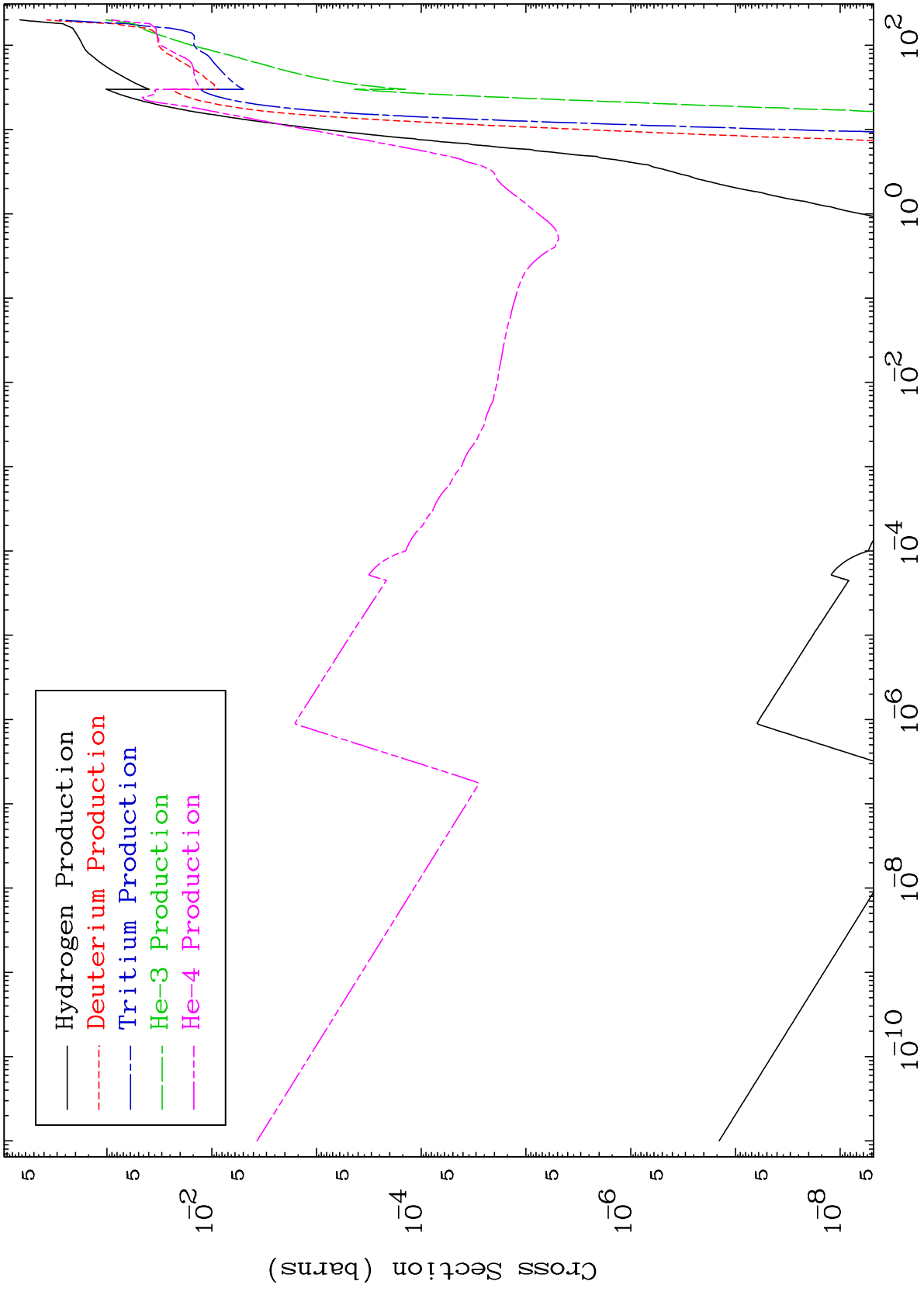


103-Lr-257

MAT 9986

Particle Production  
293 Kelvin Cross Sections

103-Lr-257

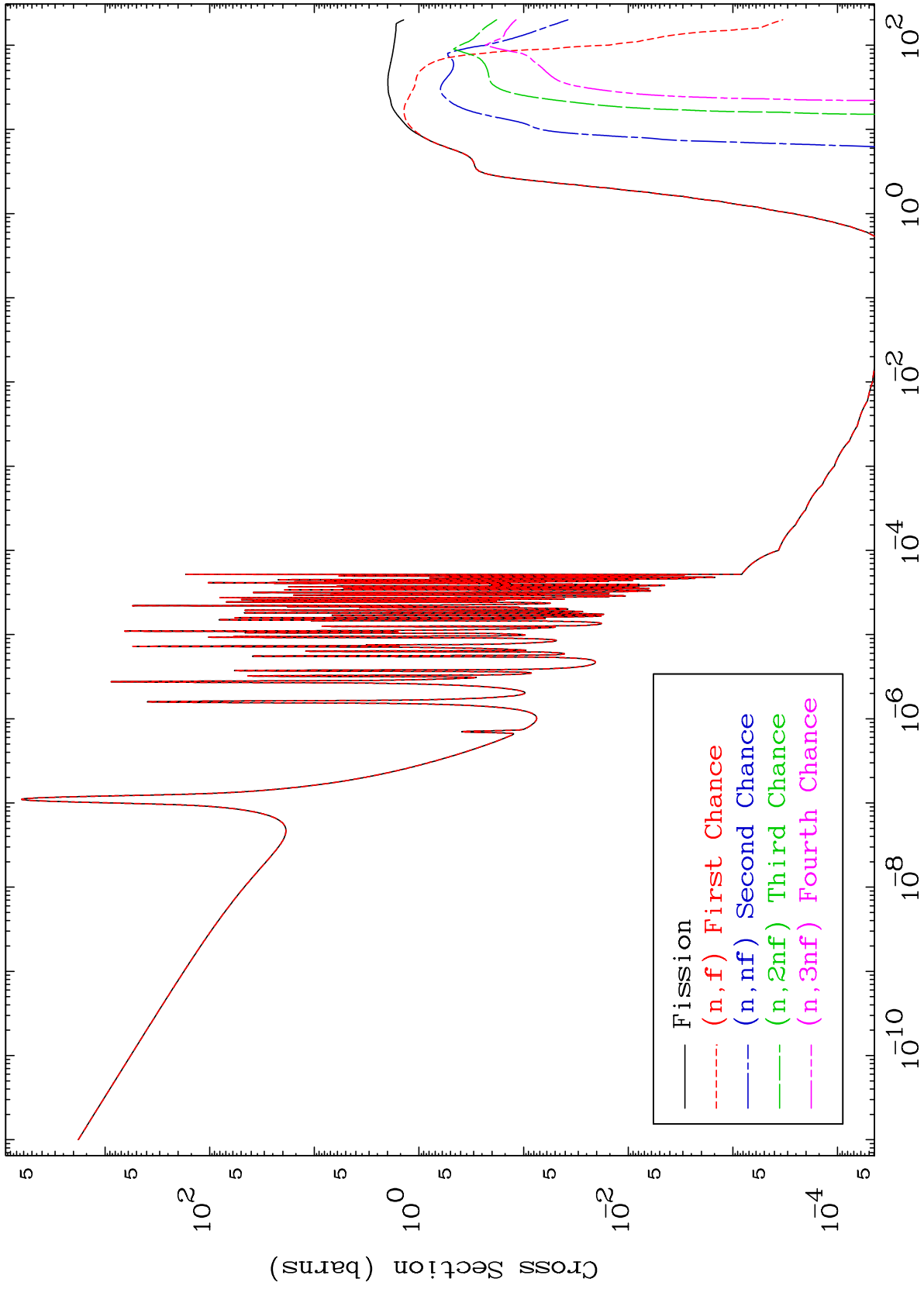


103-Lr-257

MAT 9986

Fission  
293 Kelvin Cross Sections

103-Lr-257



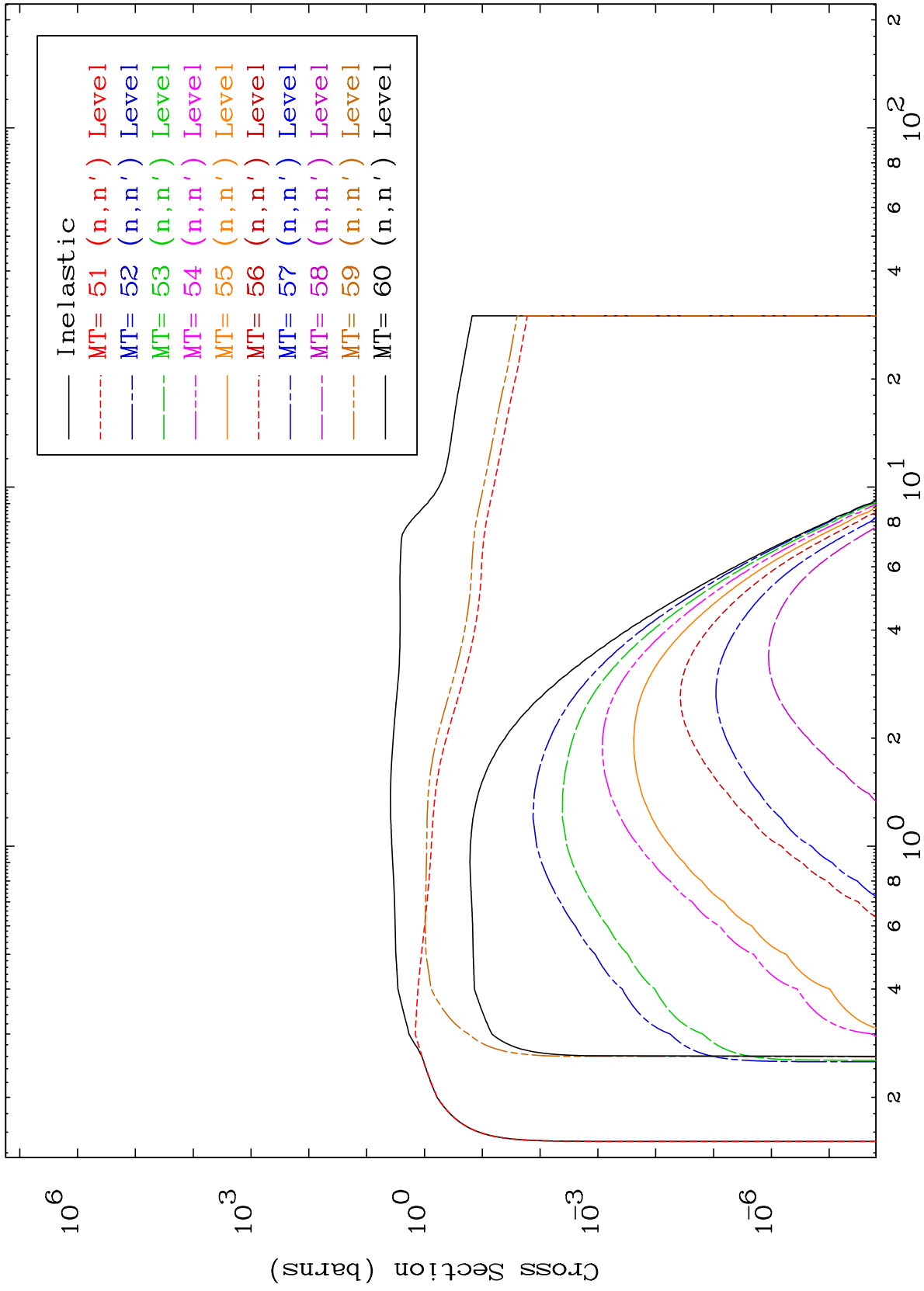


MAT 9986

(n,n') Level

103-Lr-257

293 Kelvin Cross Sections

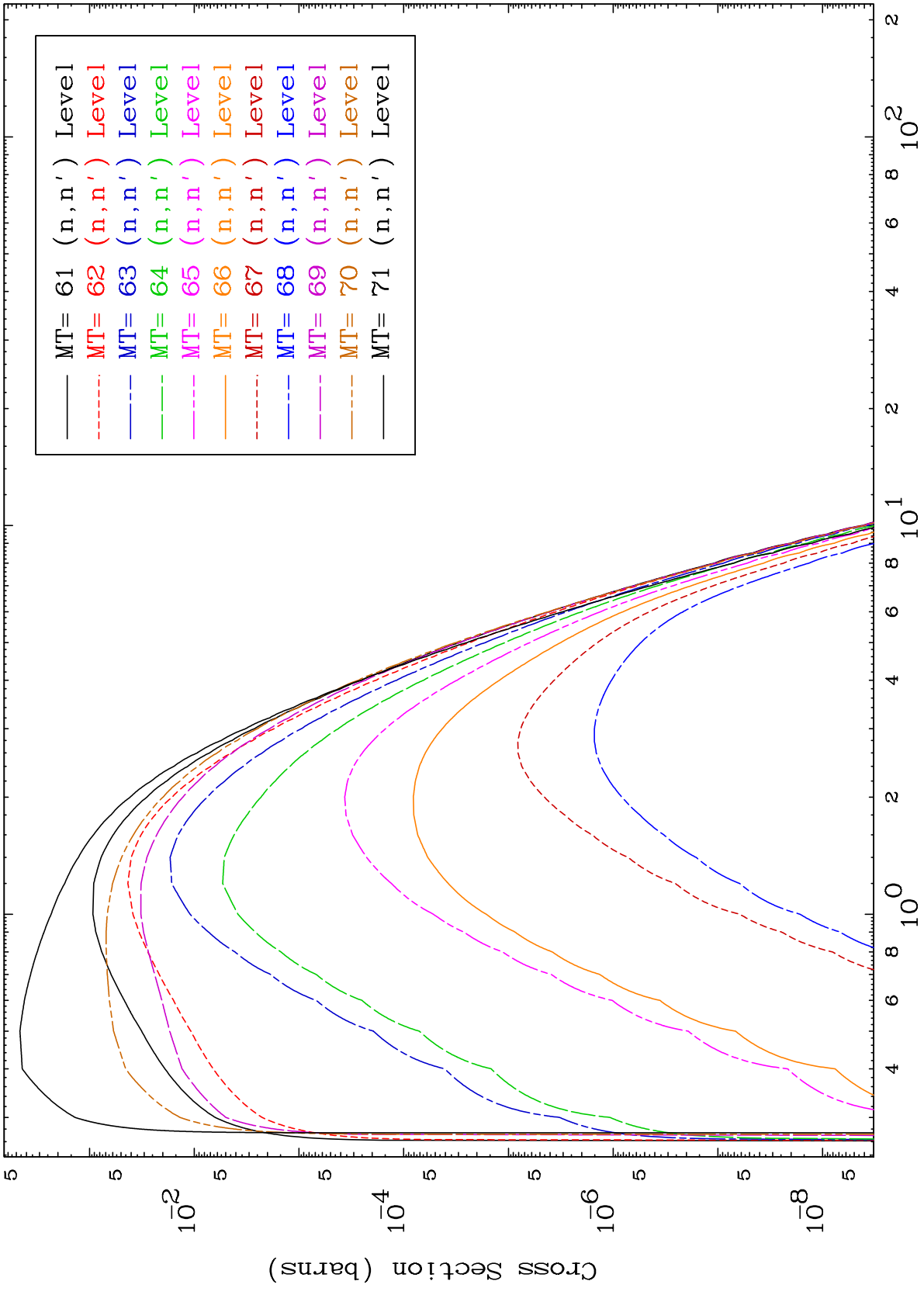


MAT 9986

(n,n') Level

103-Lr-257

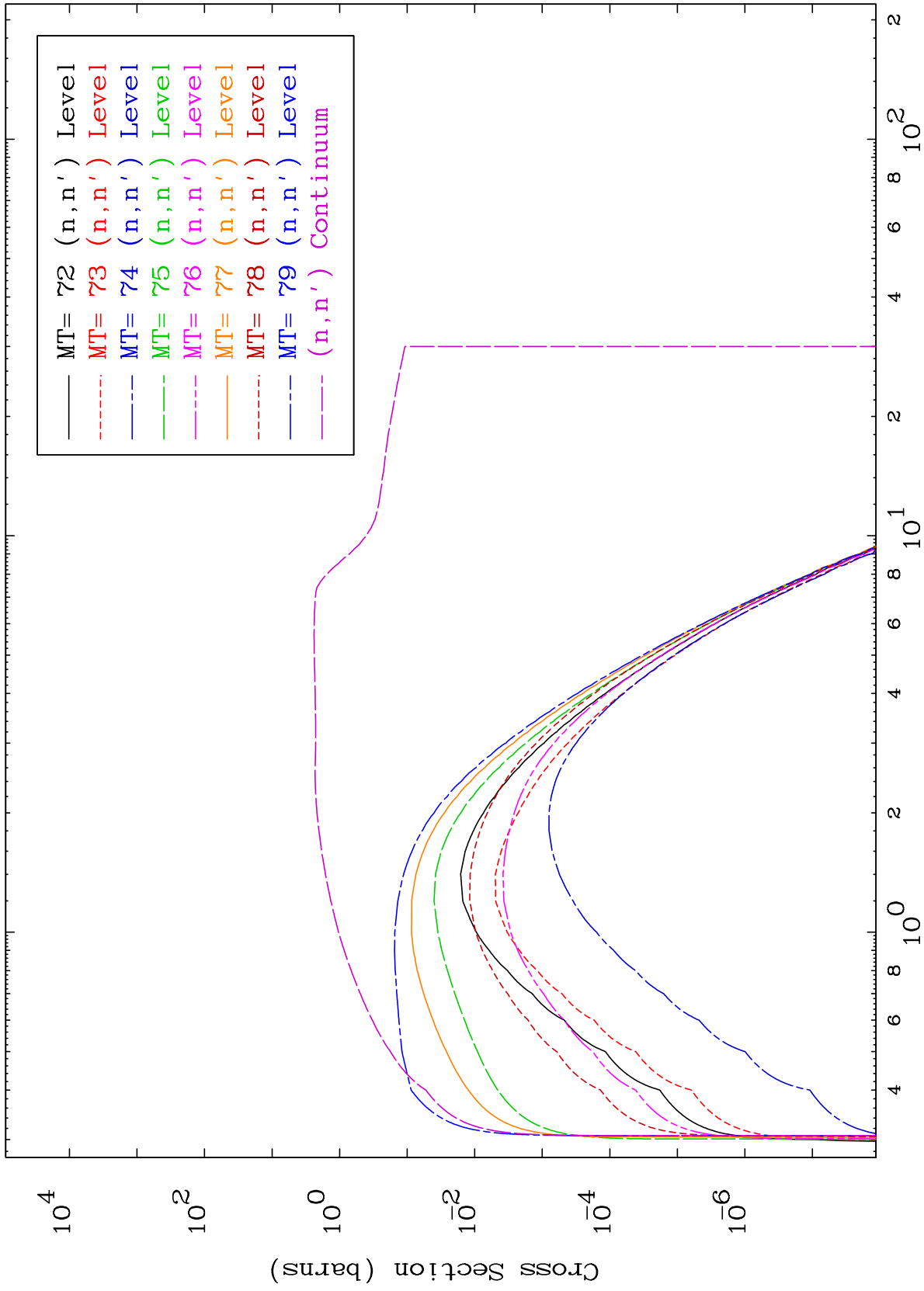
293 Kelvin Cross Sections



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(n,n') Level  
293 Kelvin Cross Sections

103-Lr-257



10

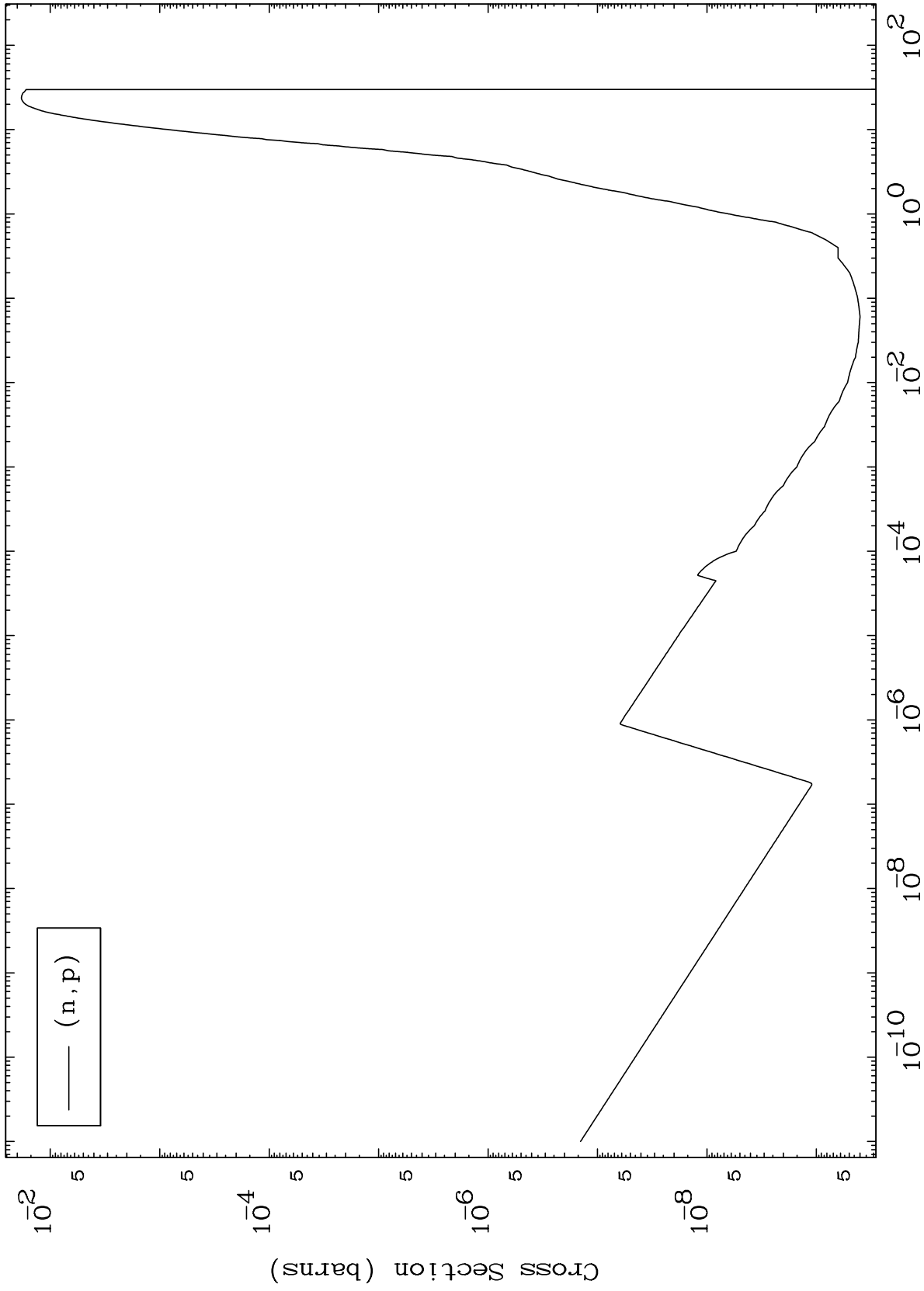
Incident Energy (MeV)

103-Lr-257

MAT 9986

(n,p) Levels  
293 Kelvin Cross Sections

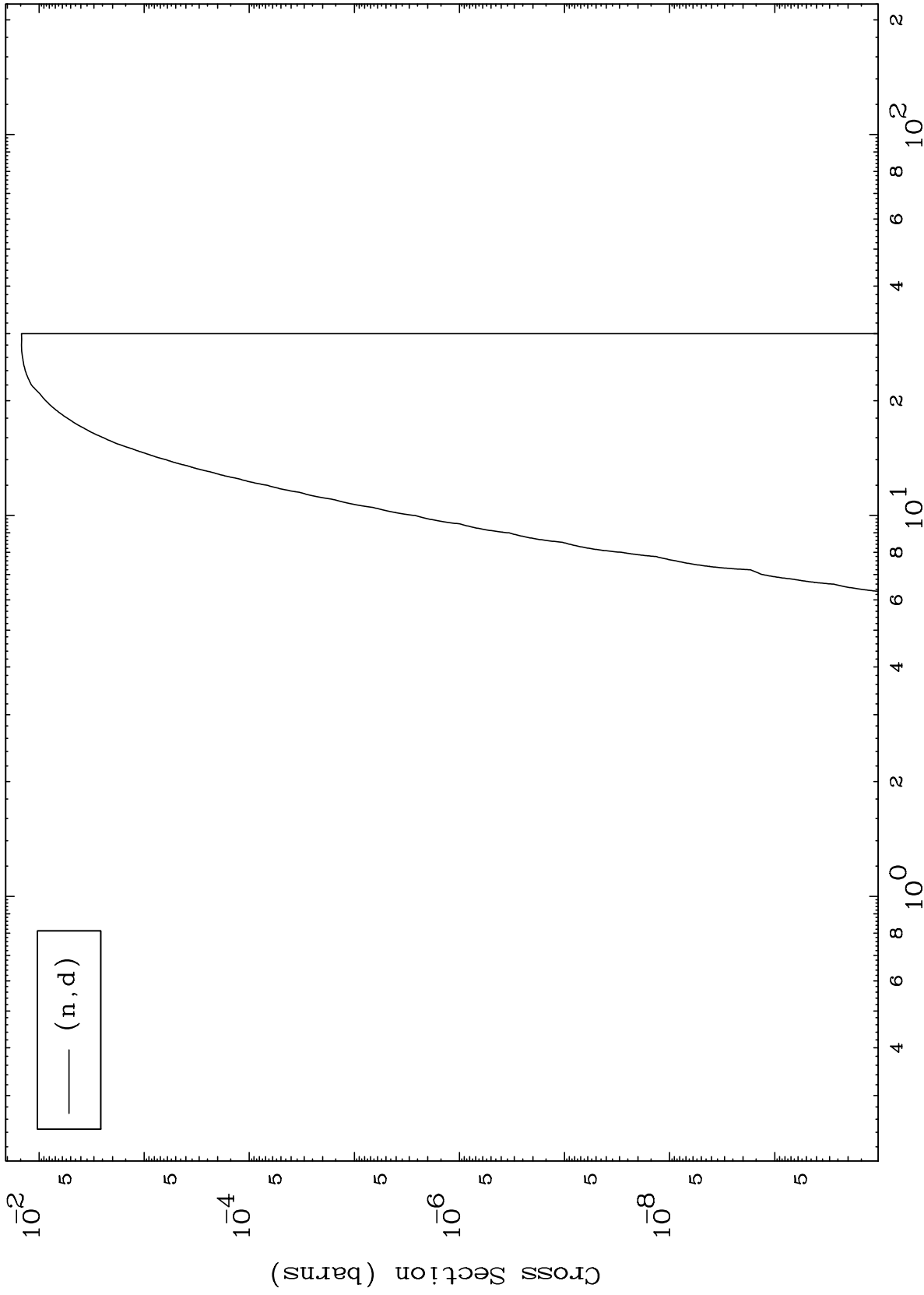
103-Lr-257



MAT 9986

(n,d) Levels  
293 Kelvin Cross Sections

103-Lr-257



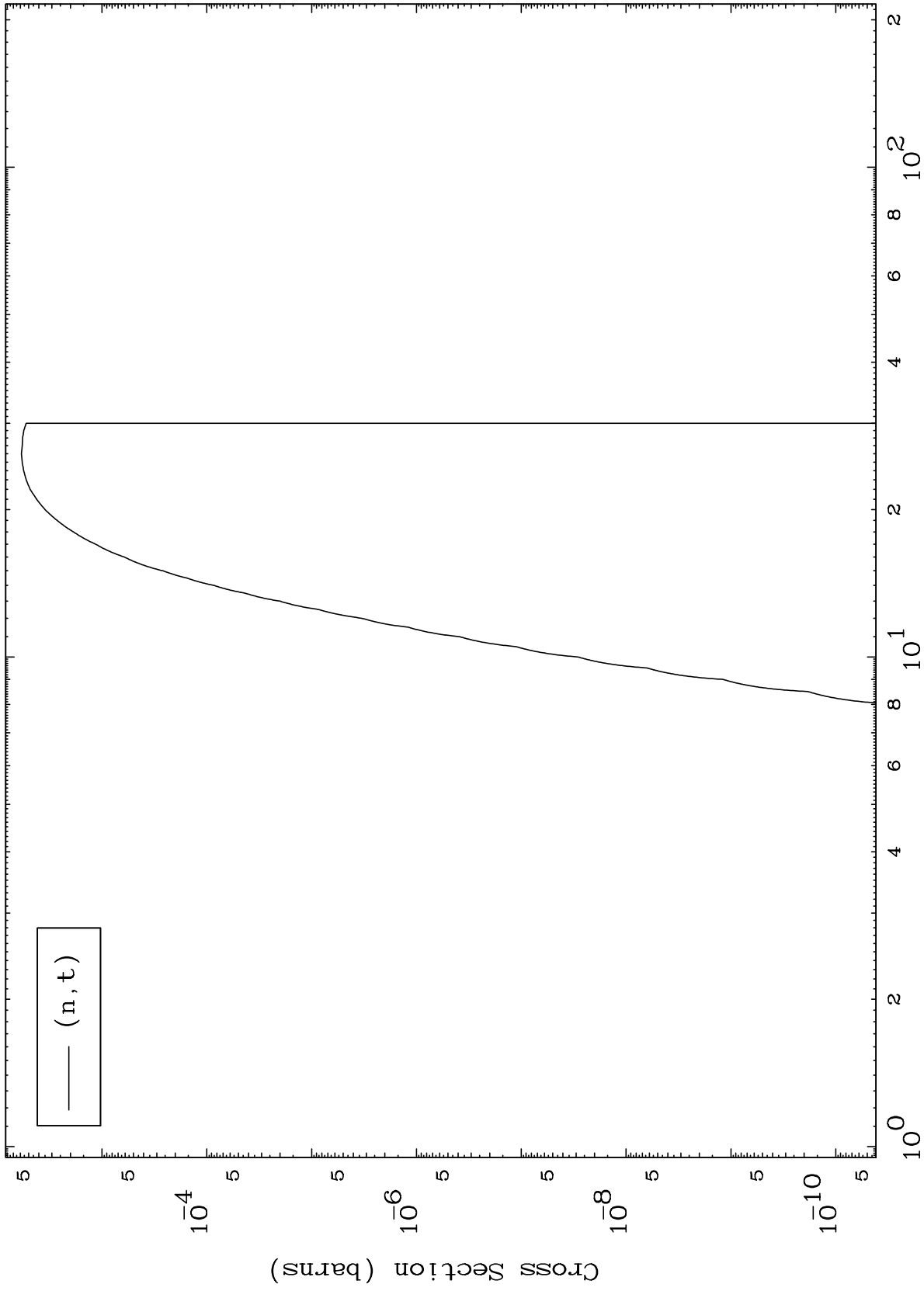
12

103-Lr-257

MAT 9986

(n,t) Levels  
293 Kelvin Cross Sections

103-Lr-257



13

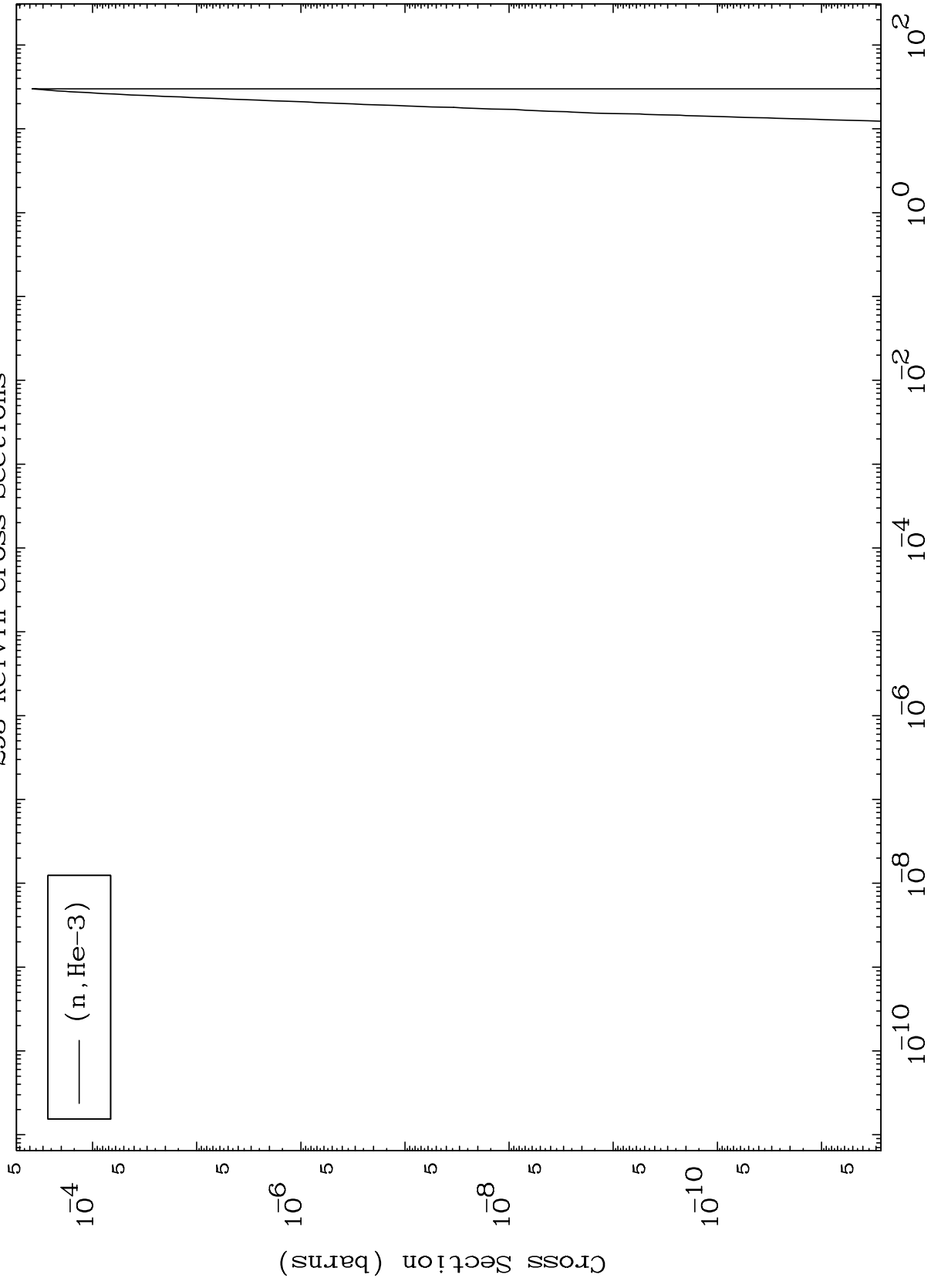
Incident Energy (MeV)

103-Lr-257

MAT 9986

(n,He3) Levels  
293 Kelvin Cross Sections

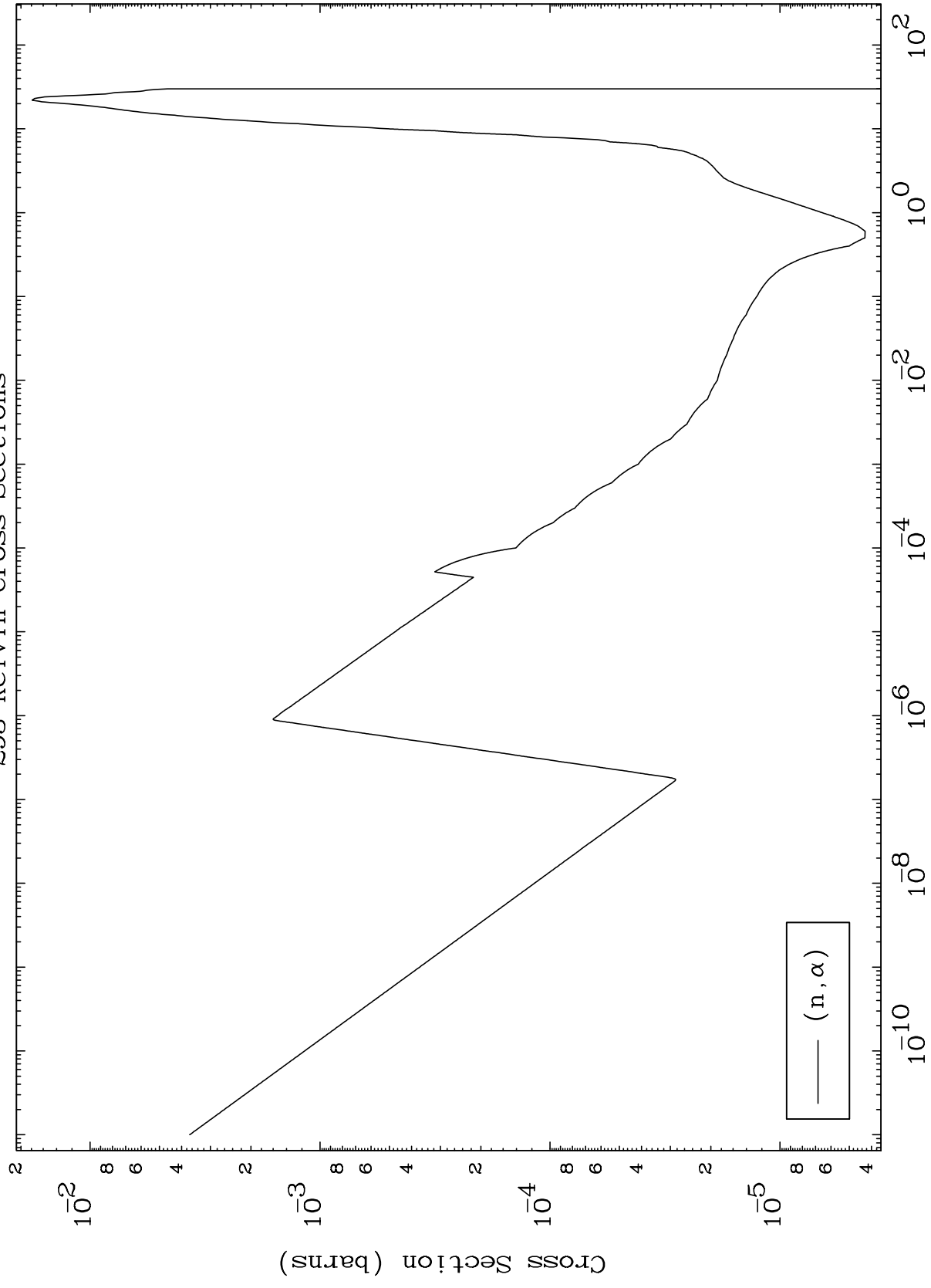
103-Lr-257



MAT 9986

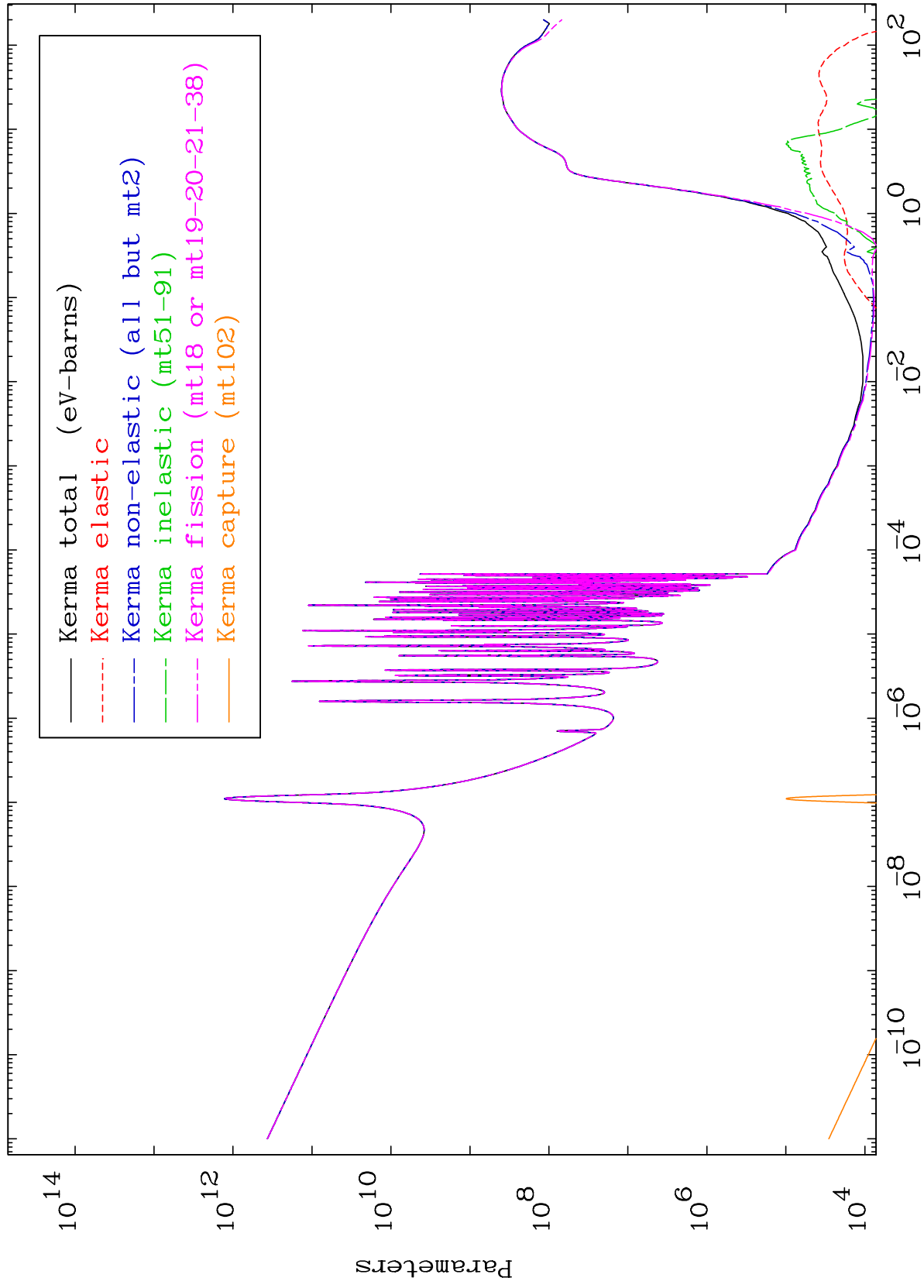
(n,α) Levels  
293 Kelvin Cross Sections

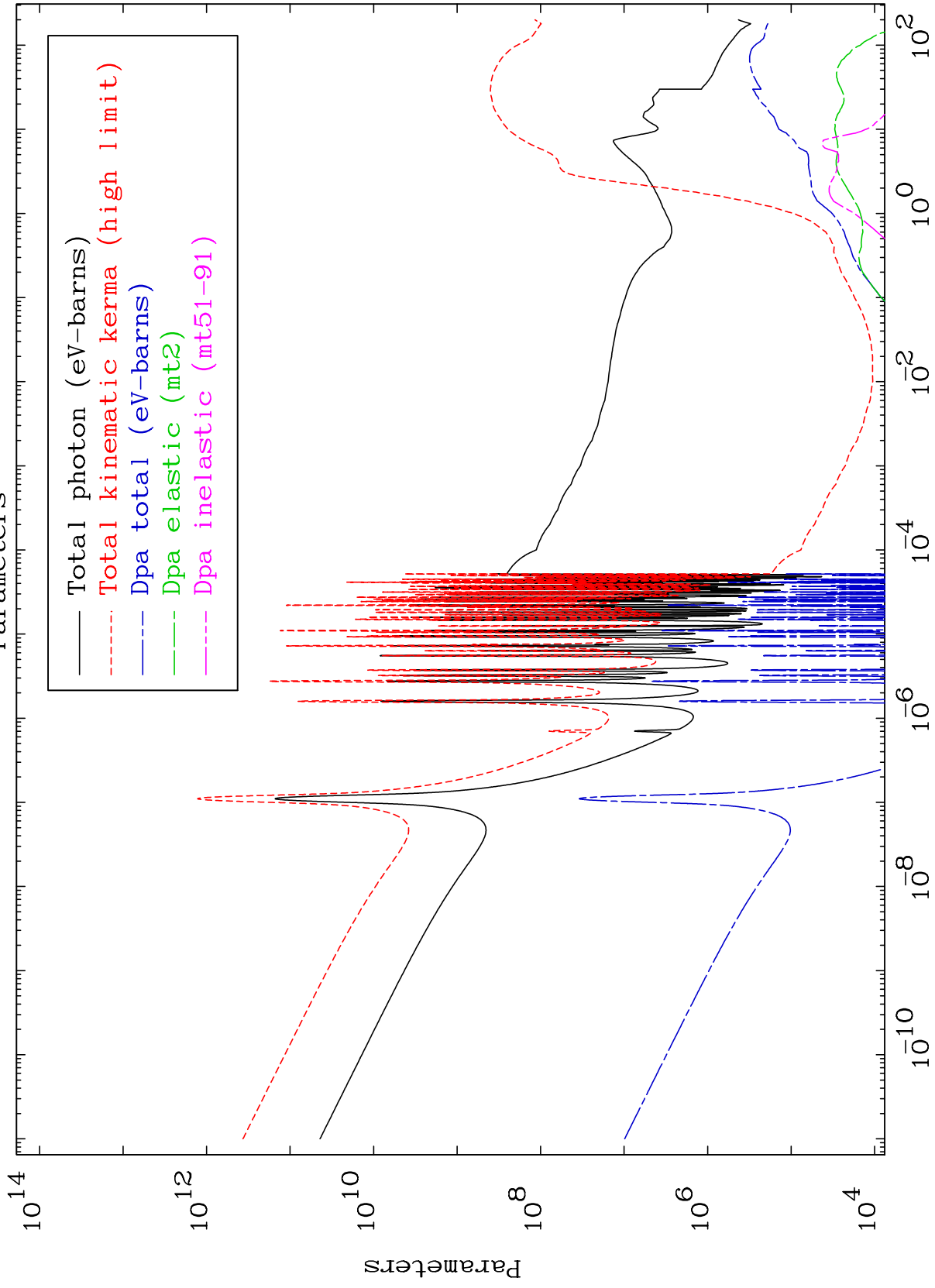
103-Lr-257



103-Lr-257



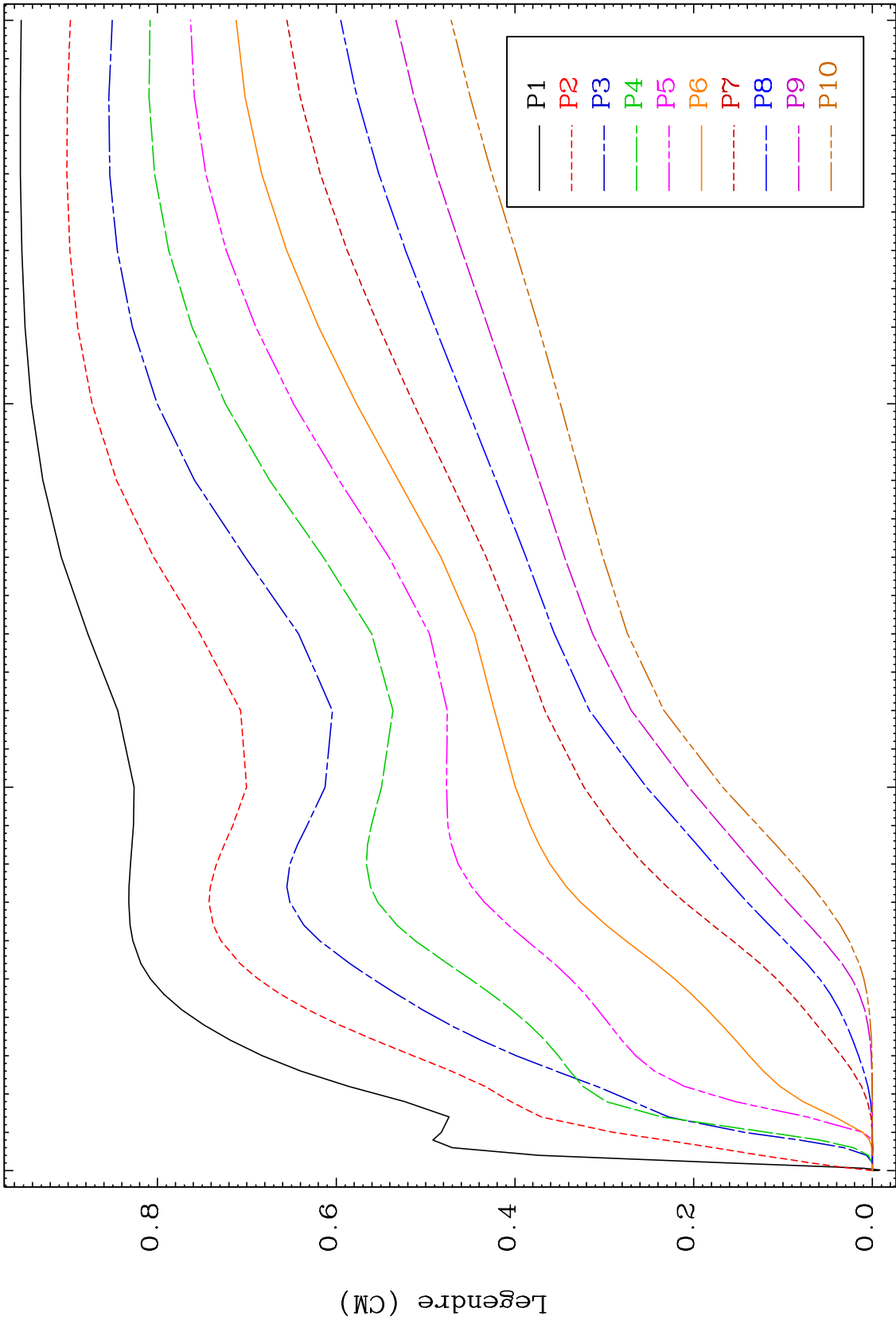




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Elastic Legendre Coefficients

103-Lr-257



18

Incident Energy (MeV)

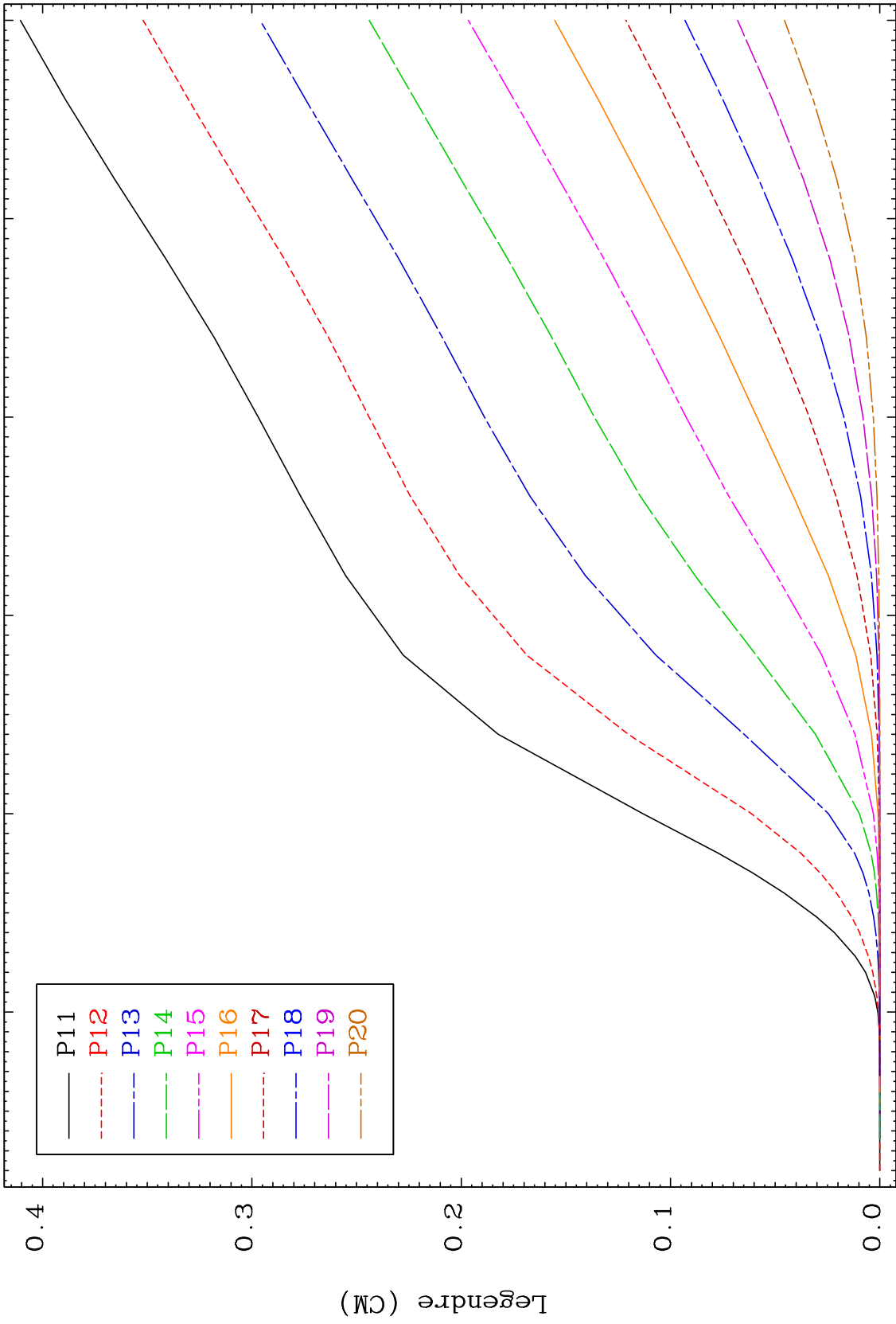
103-Lr-257

30

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Elastic Legendre Coefficients

$^{103}\text{--Lr--257}$



19

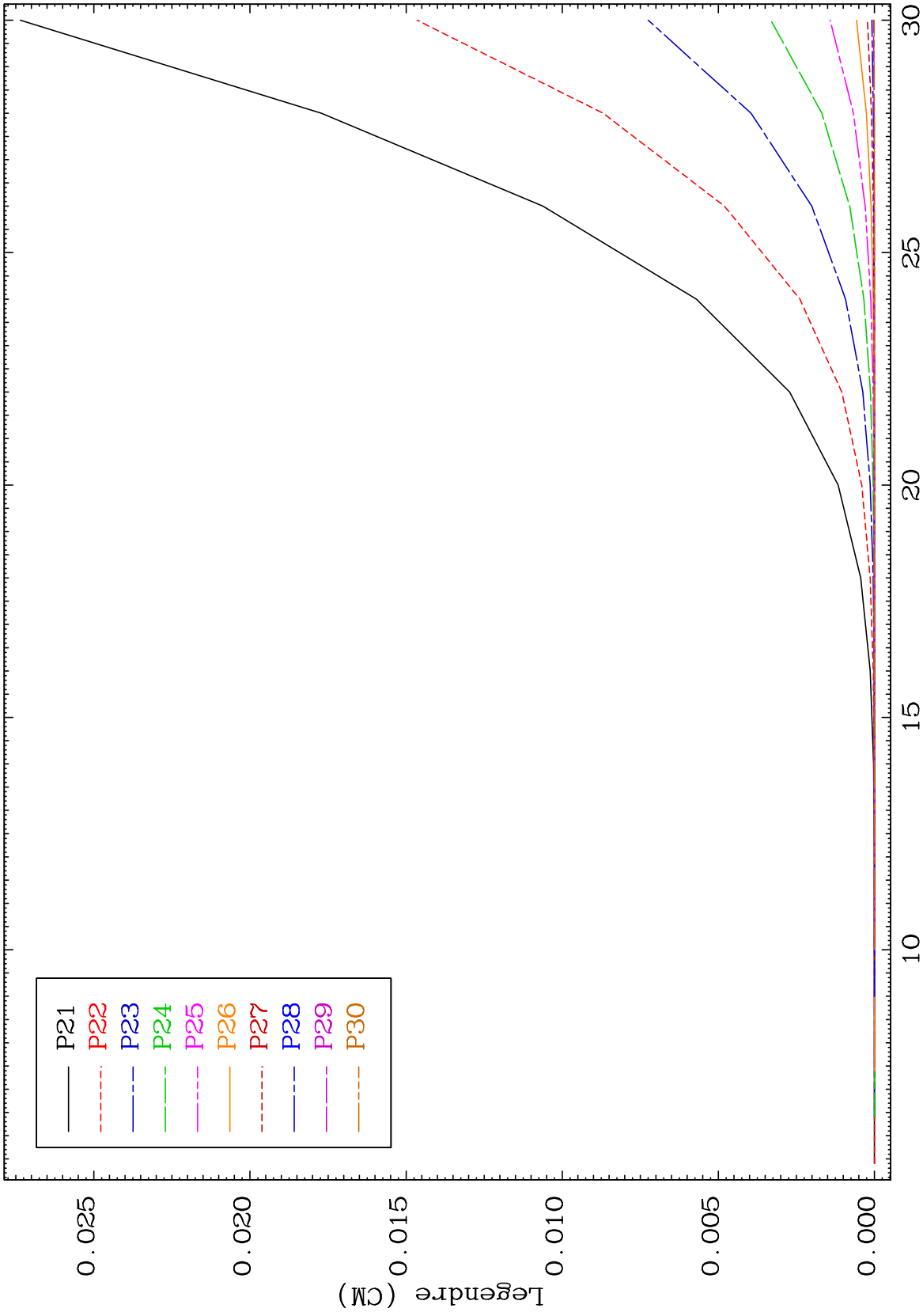
Incident Energy (MeV)

$^{103}\text{--Lr--257}$

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### Elastic Legendre Coefficients

103-Lr-257



20

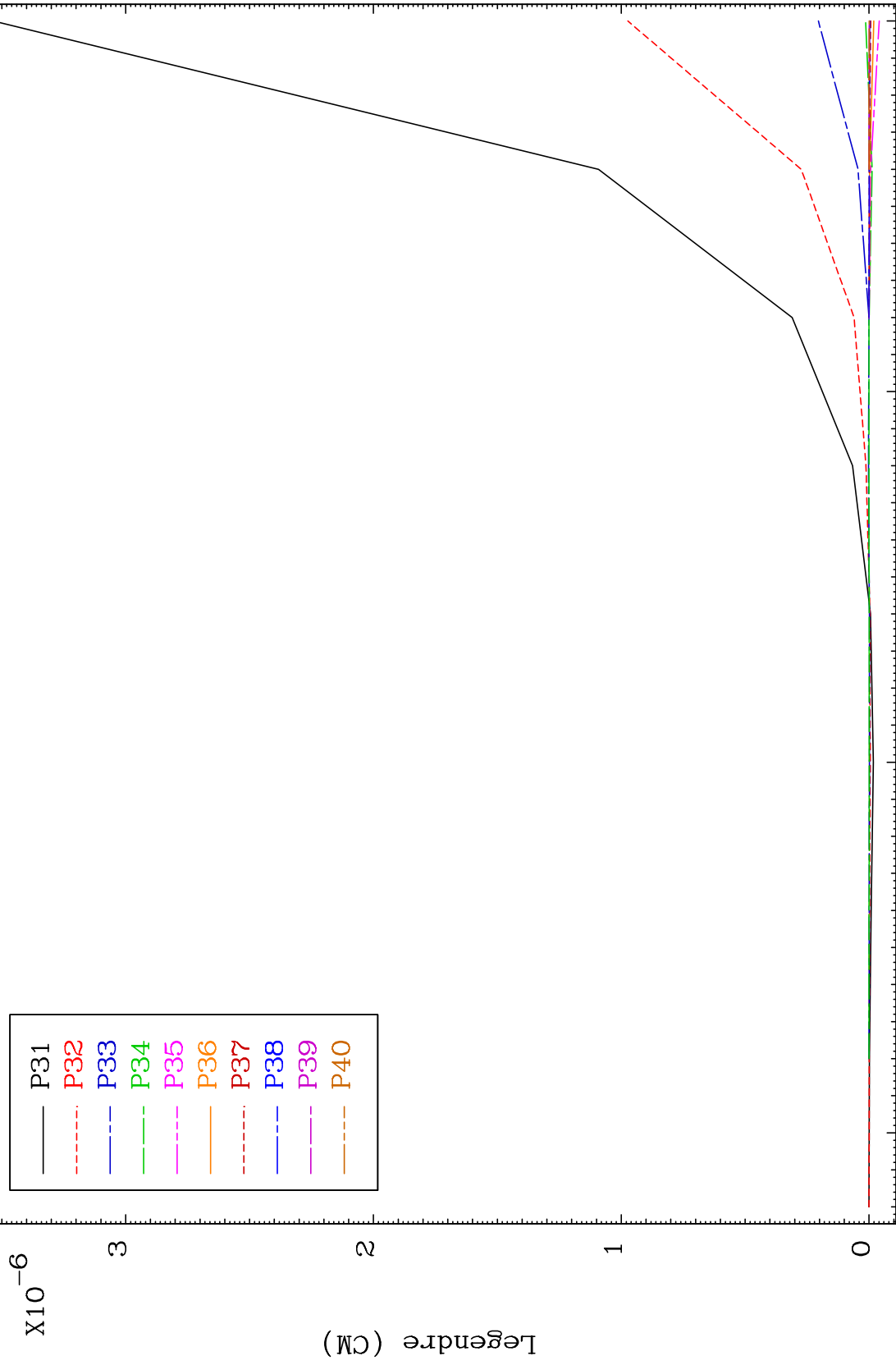
Incident Energy (MeV)

103-Lr-257

MAT 9986

Elastic Legendre Coefficients

103-Lr-257



21

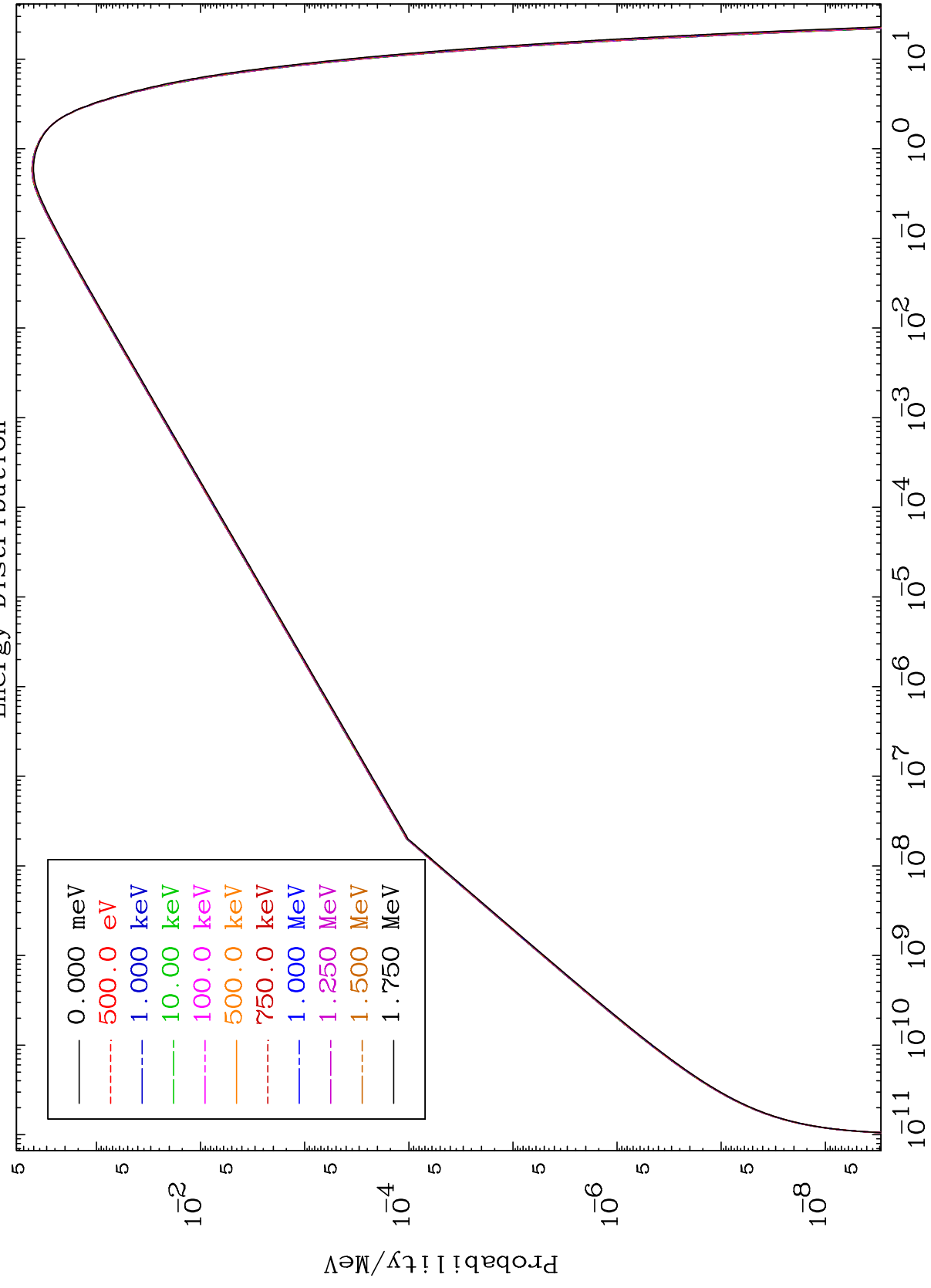
Incident Energy (MeV)

103-Lr-257

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### Fission Energy Distribution

103-Lr-257

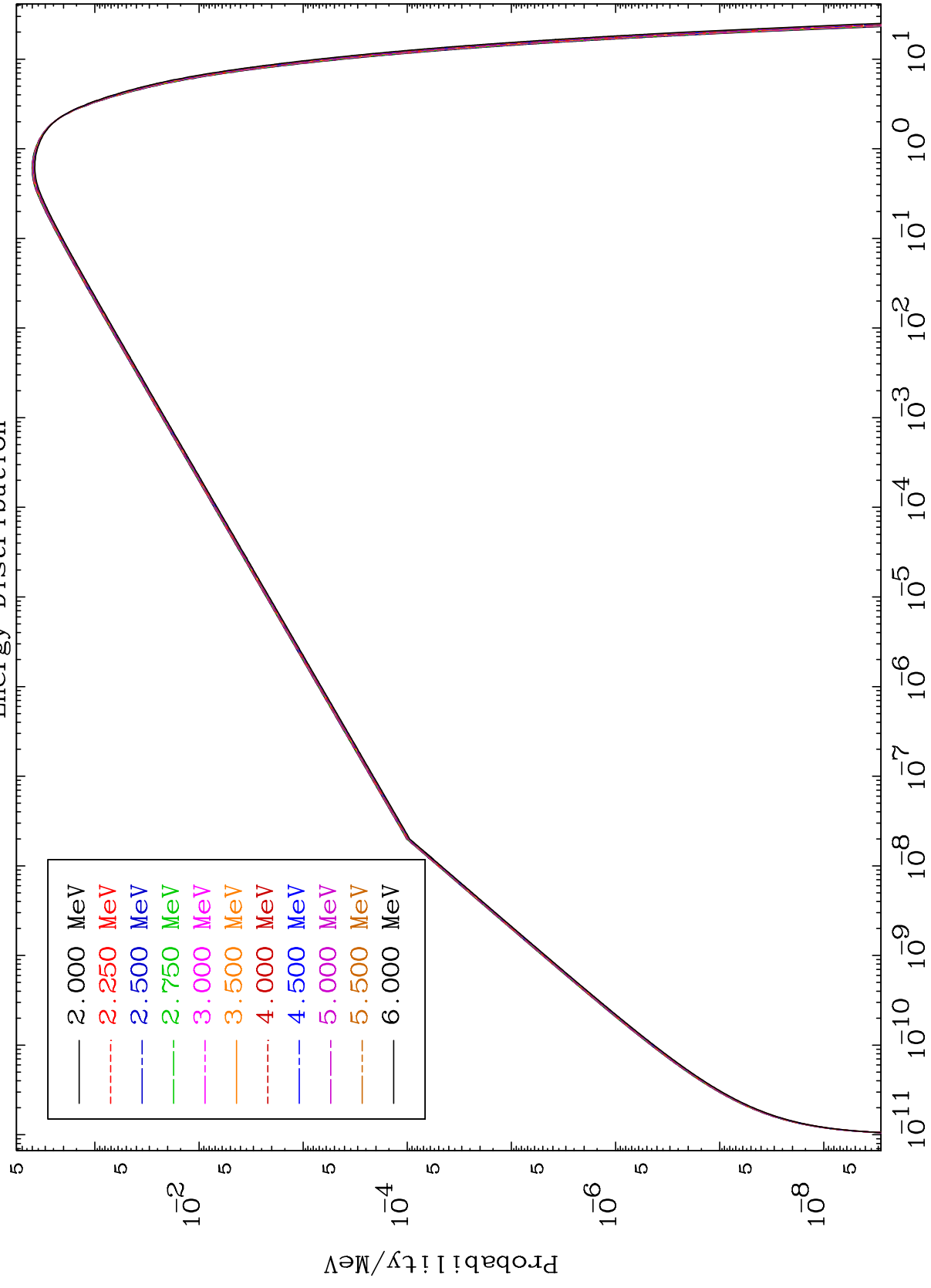


103-Lr-257

MAT 9986

Fission  
Energy Distribution

103-Lr-257



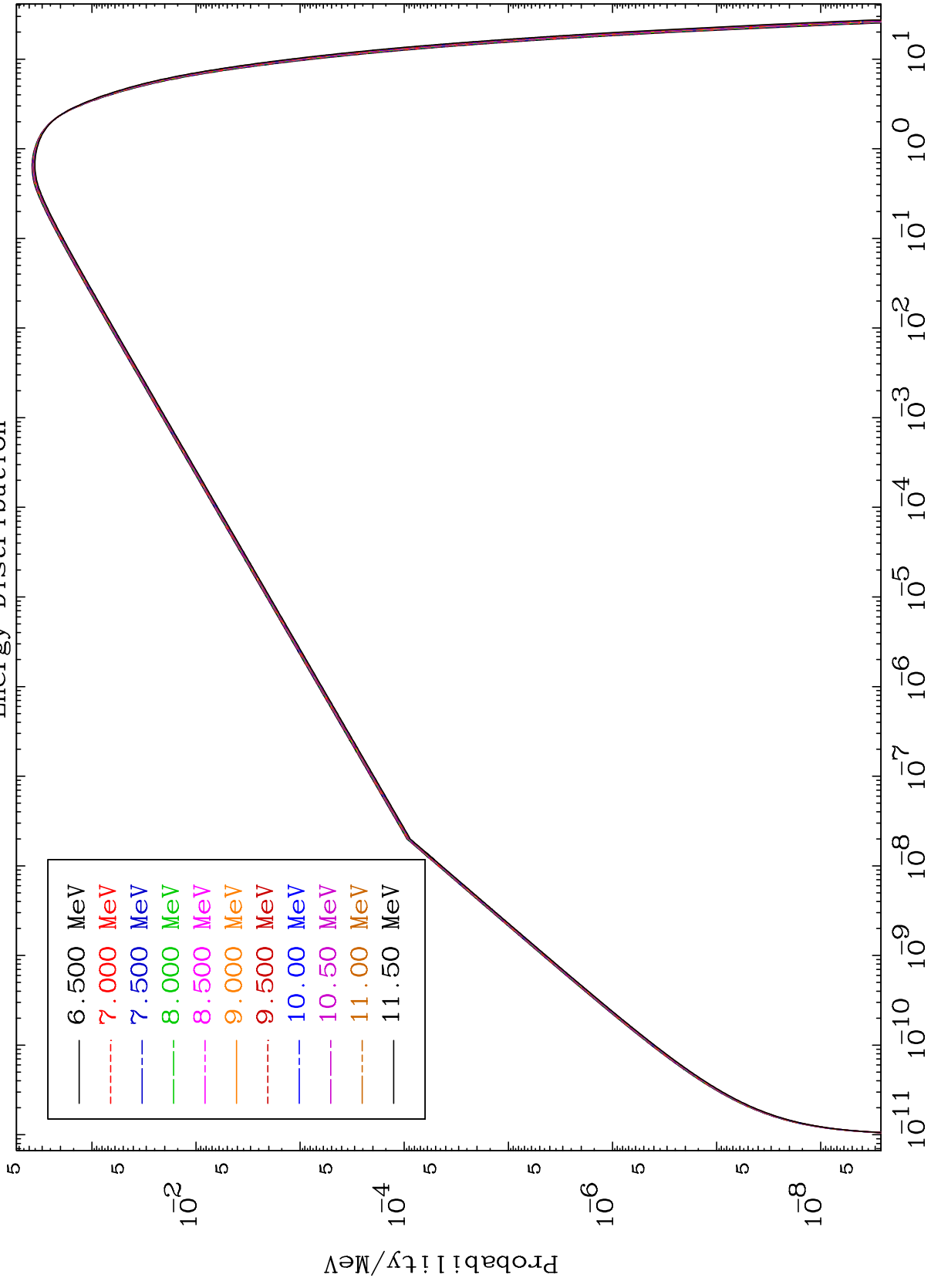
103-Lr-257



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Fission  
Energy Distribution

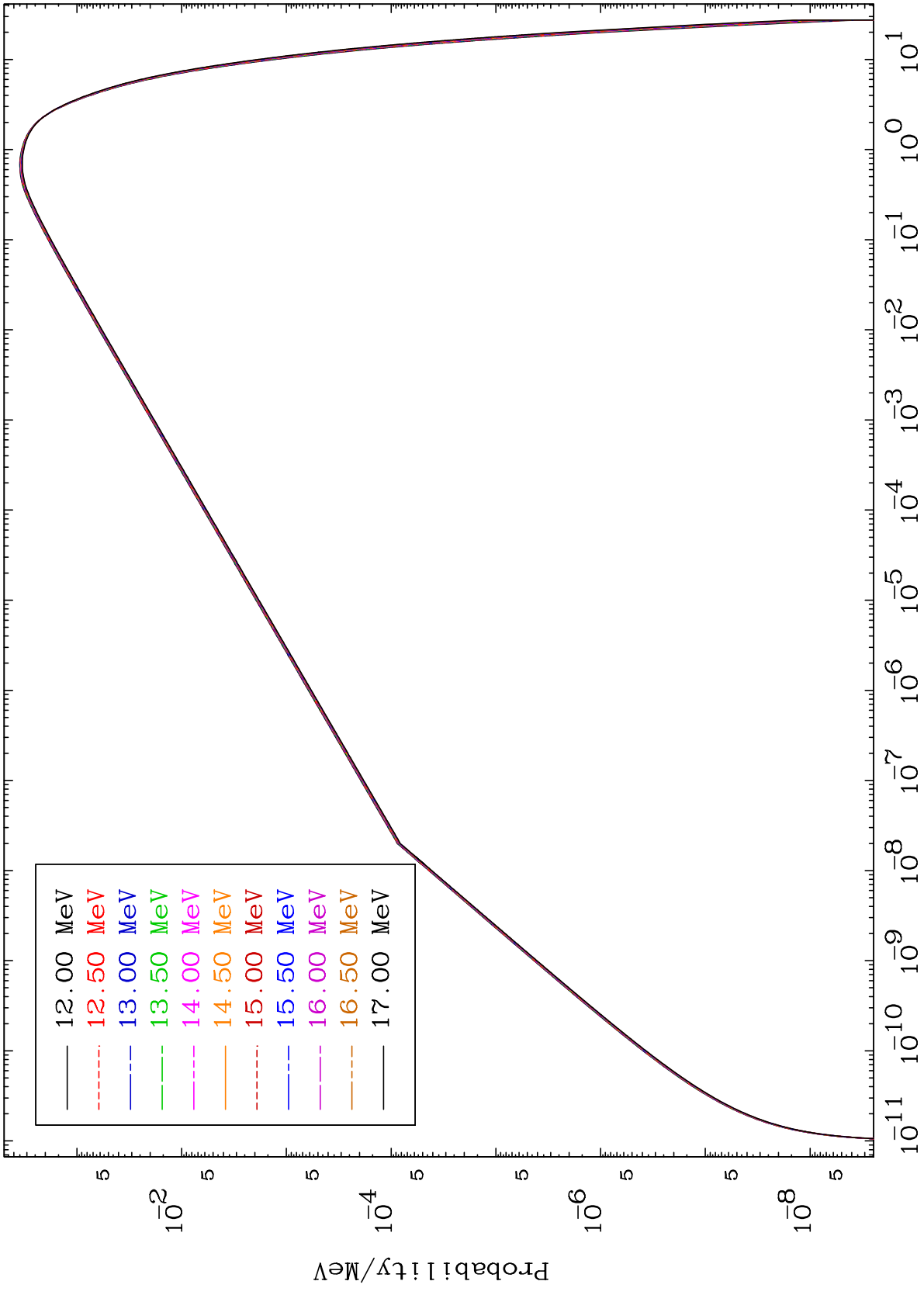
103-Lr-257



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### Fission Energy Distribution

103-Lr-257

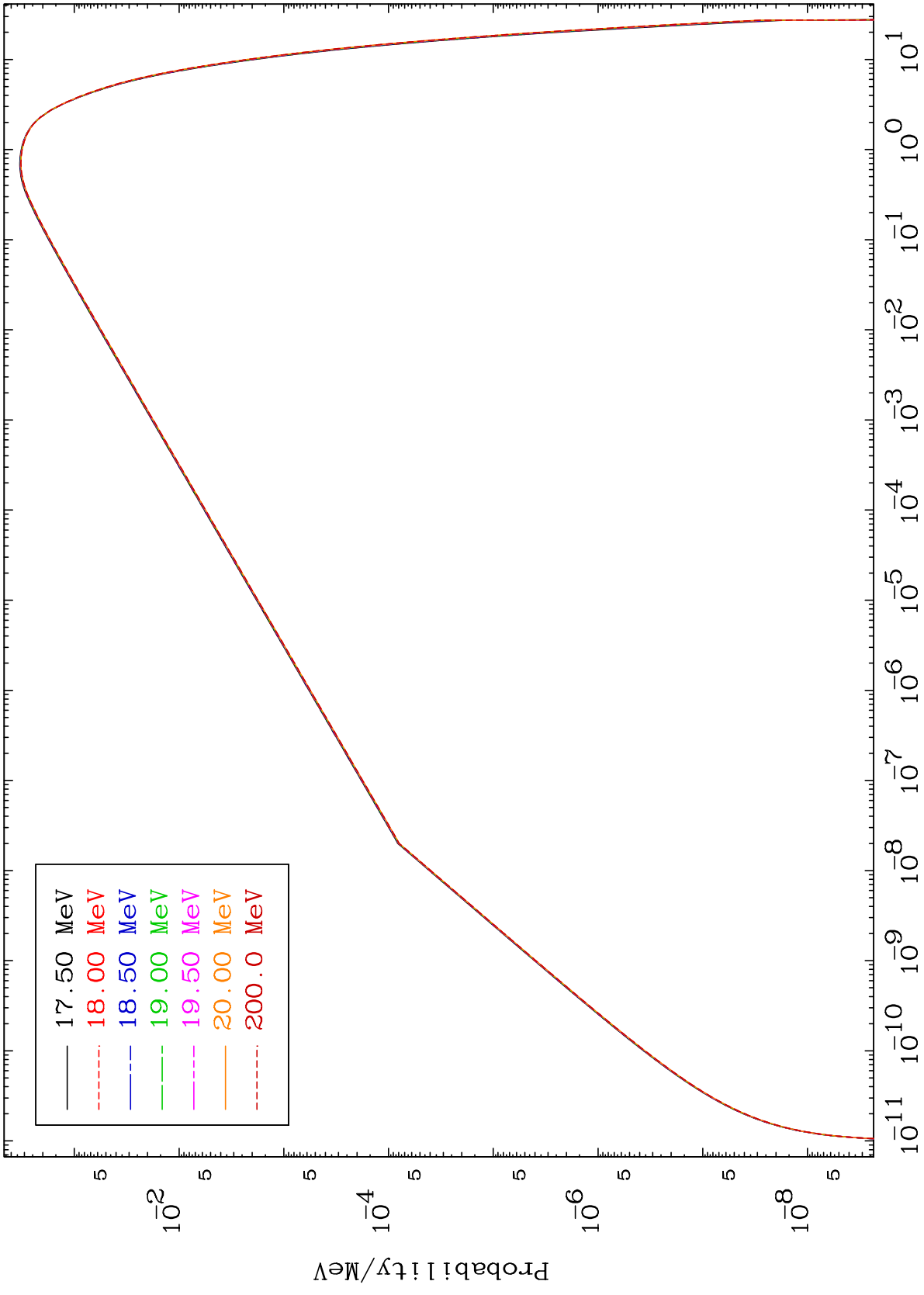


103-Lr-257

MAT 9986

Fission  
Energy Distribution

103-Lr-257

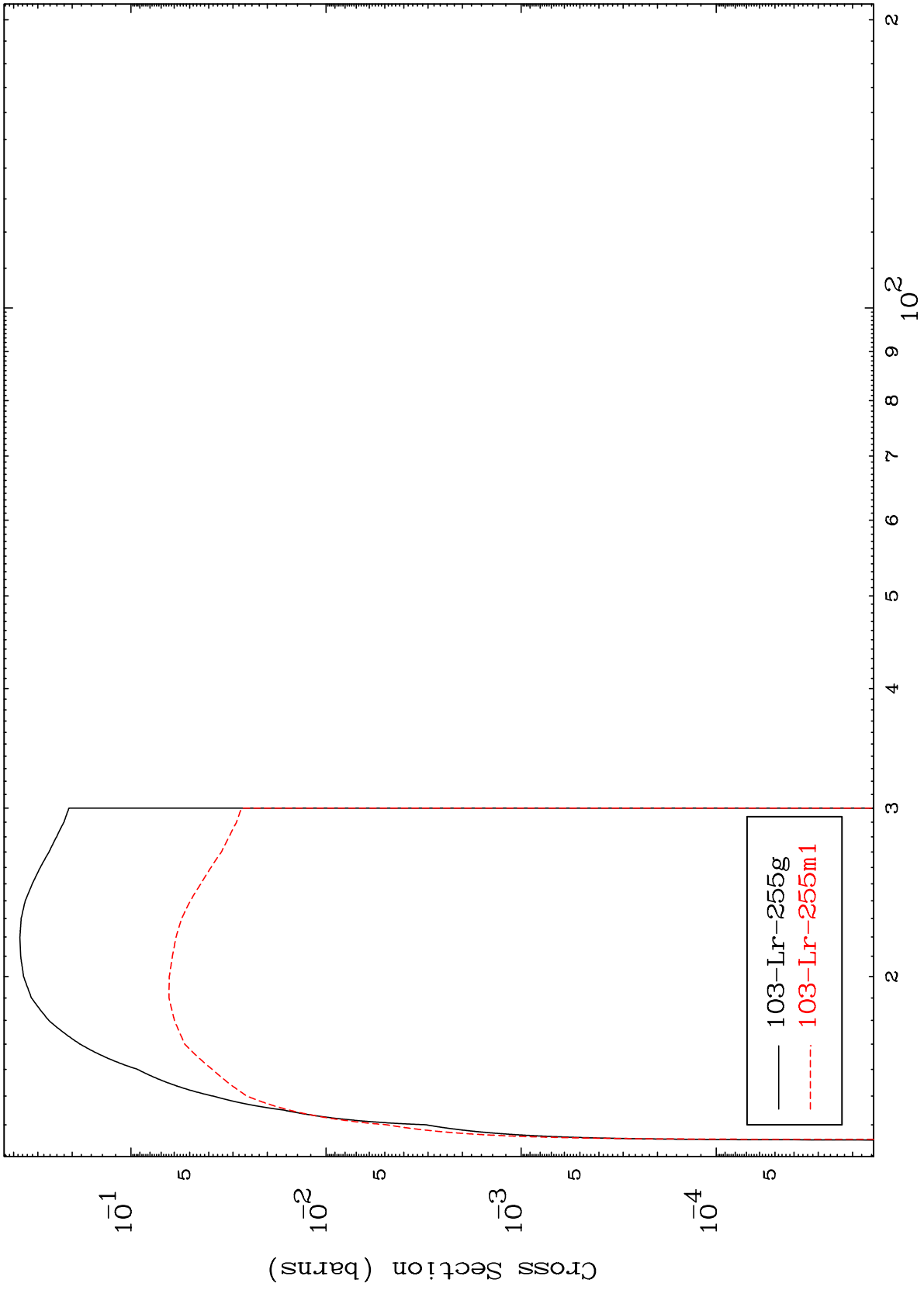


103-Lr-257

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103-Lr-257

(n,3n)  
Radionuclide Production Cross Section



27

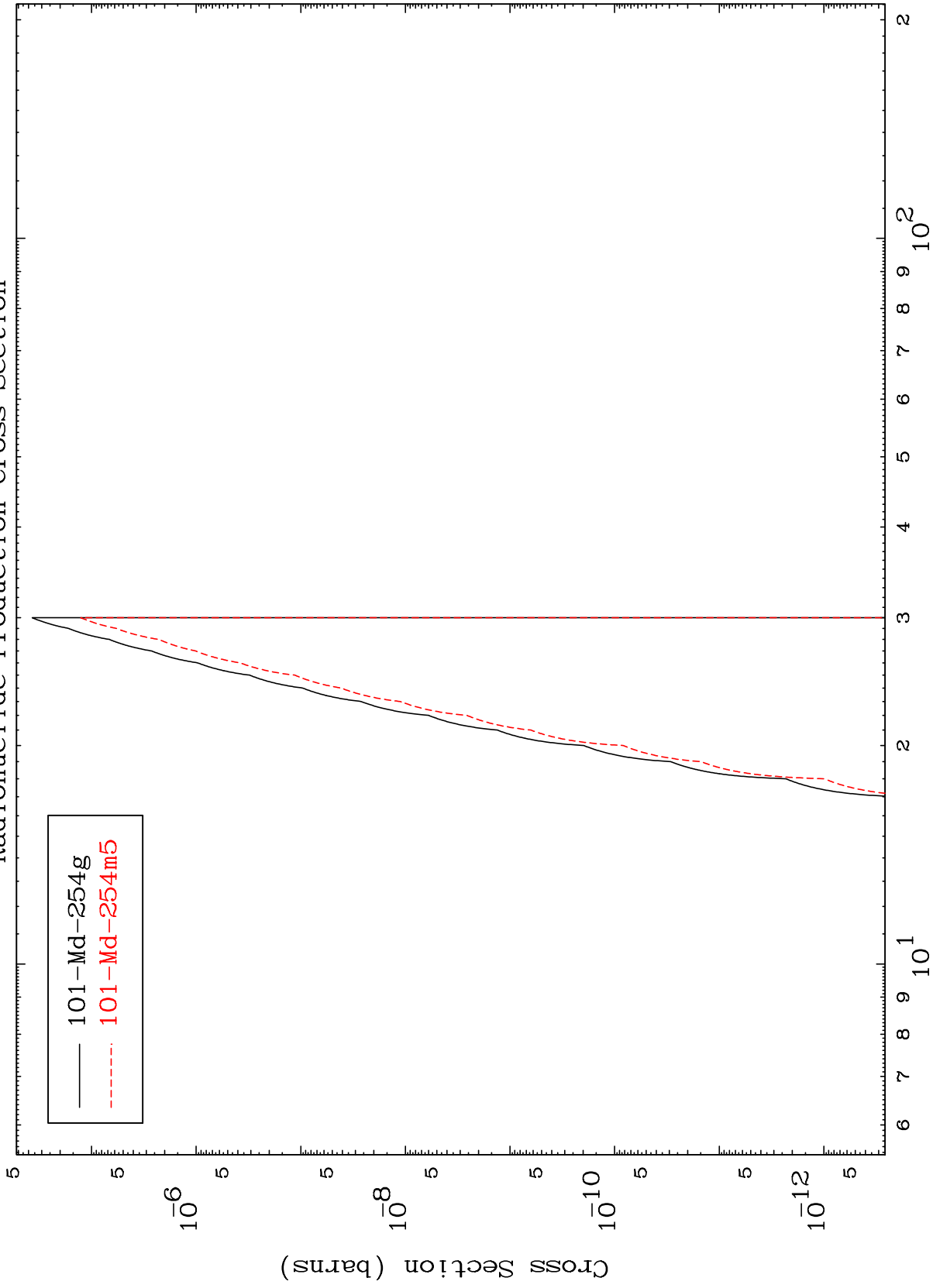
103-Lr-257

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(n, n') He-3

103-Lr-257

Radionuclide Production Cross Section



28

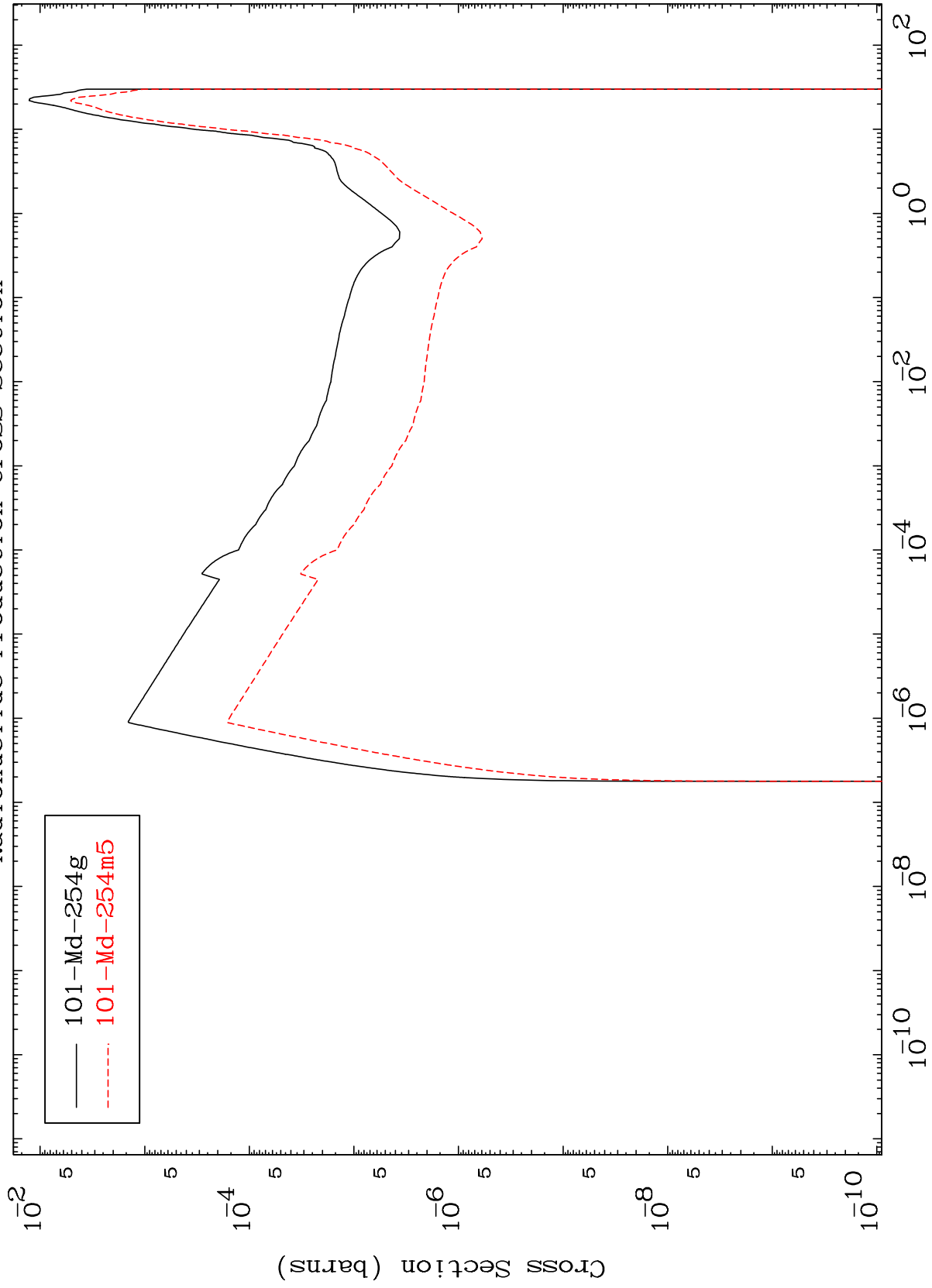
Incident Energy (MeV)

103-Lr-257

MAT 9986

103-Lr-257

(n,  $\alpha$ )  
Radionuclide Production Cross Section



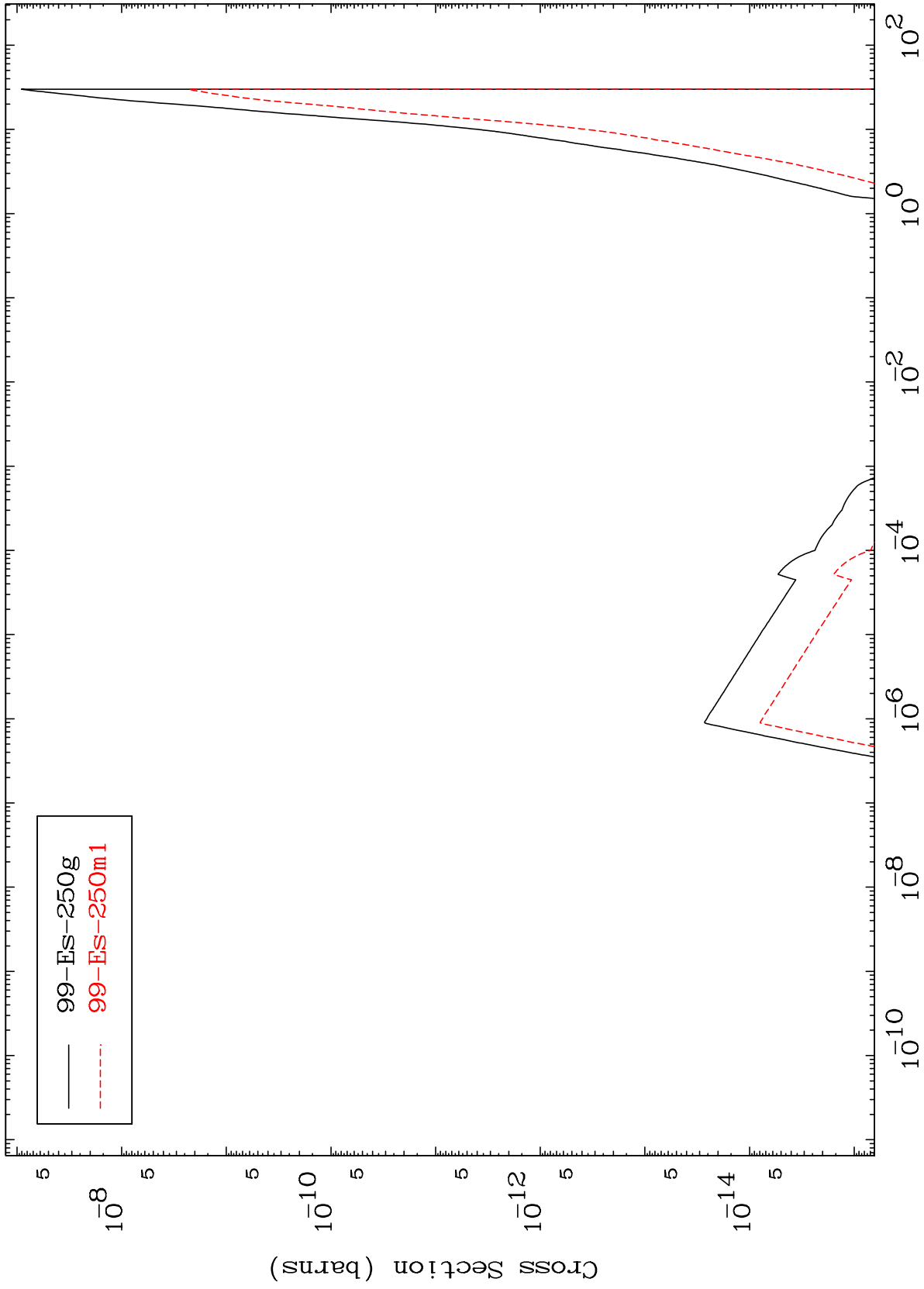
— 101-Md-254g  
- - - 101-Md-254m5

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(n,2α)

103-Lr-257

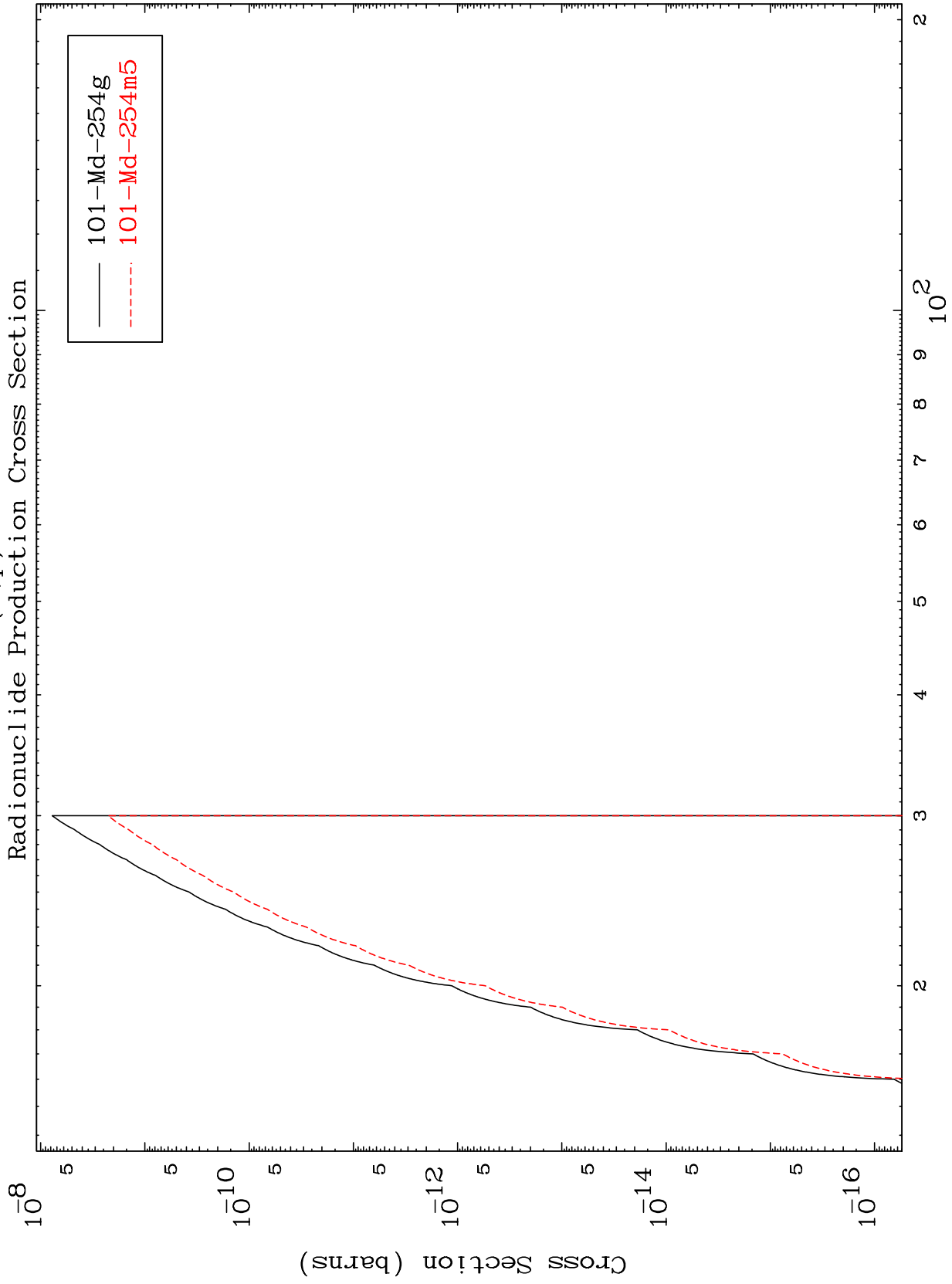
Radionuclide Production Cross Section



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(n,p) t

103-Lr-257



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103-Lr-257