

Program EVALPLOT
(Version 2018-1)

by

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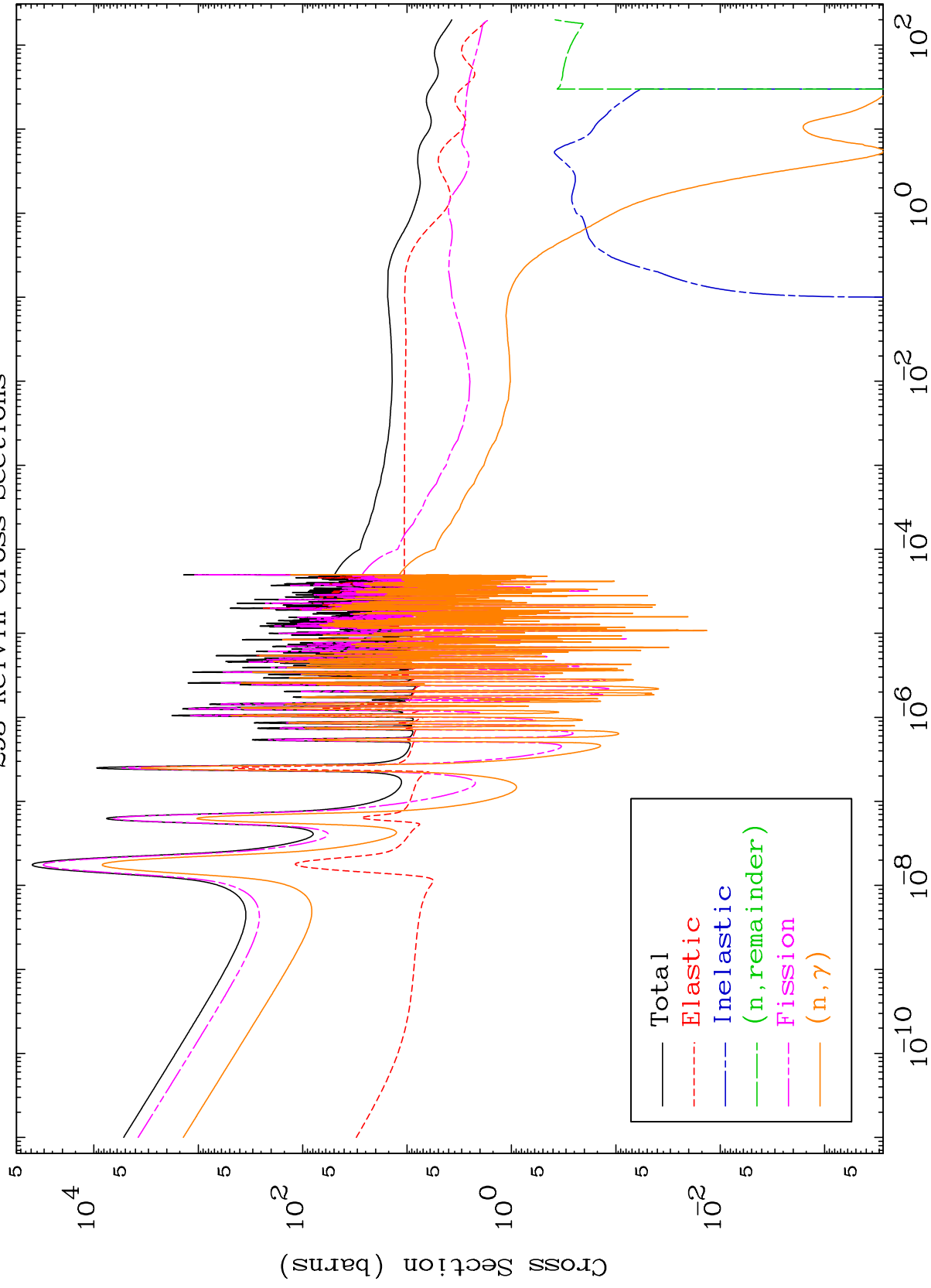
Press Mouse Button to Start

MAT 9992

Major

293 Kelvin Cross Sections

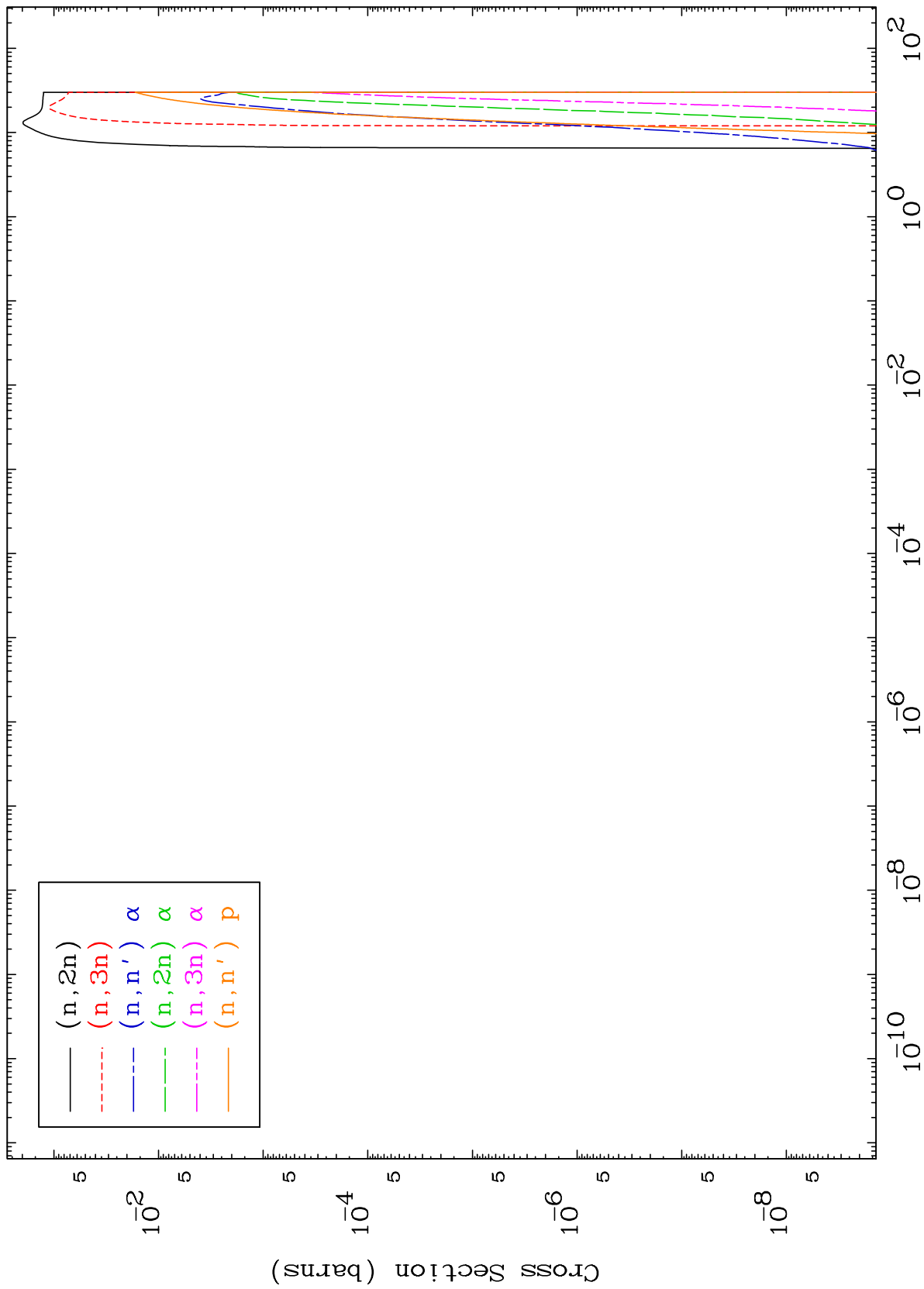
103-Lr-263



MAT 9992

Neutron Production
293 Kelvin Cross Sections

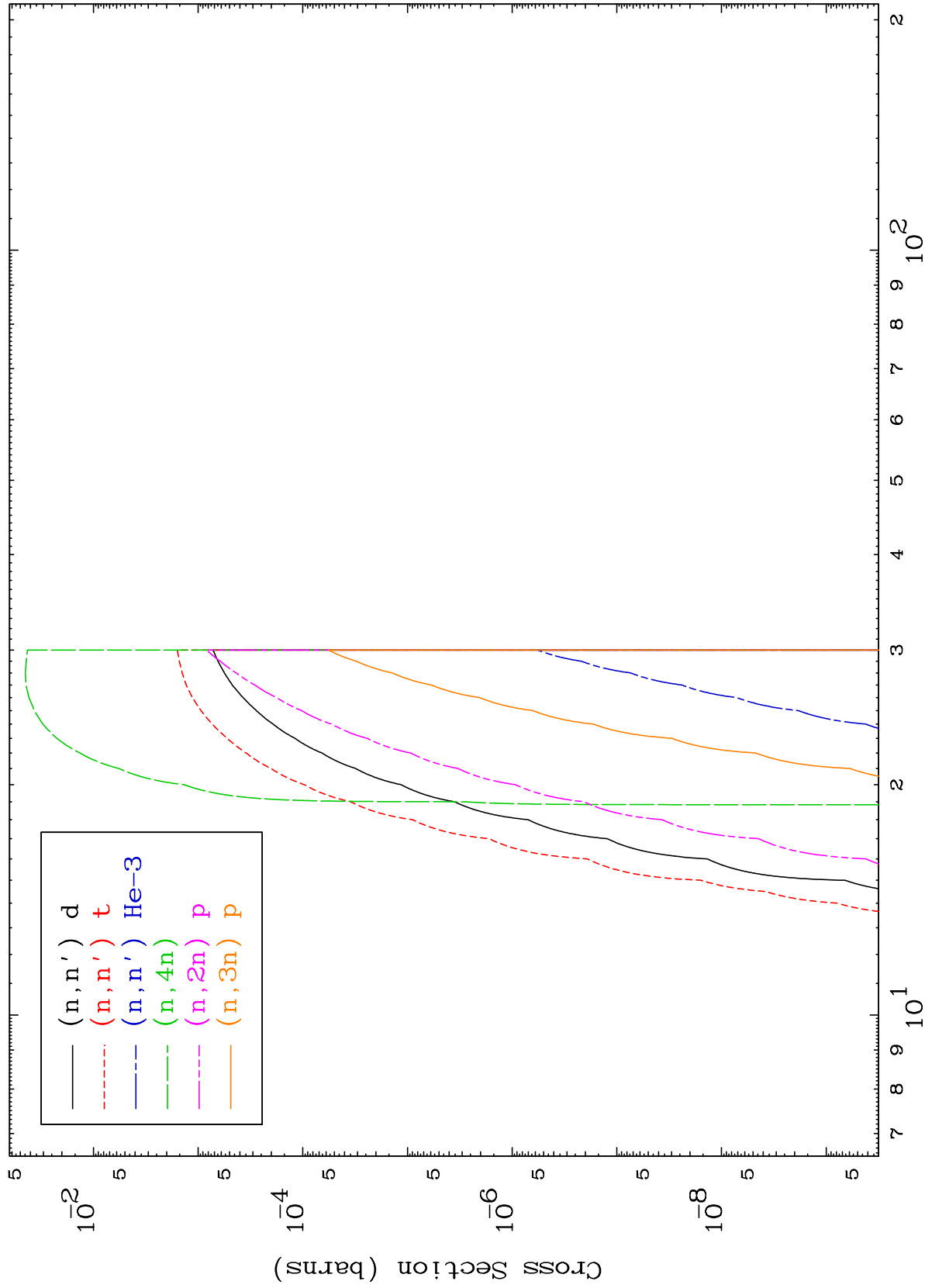
103-Lr-263



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Neutron Production
293 Kelvin Cross Sections

103-Lr-263



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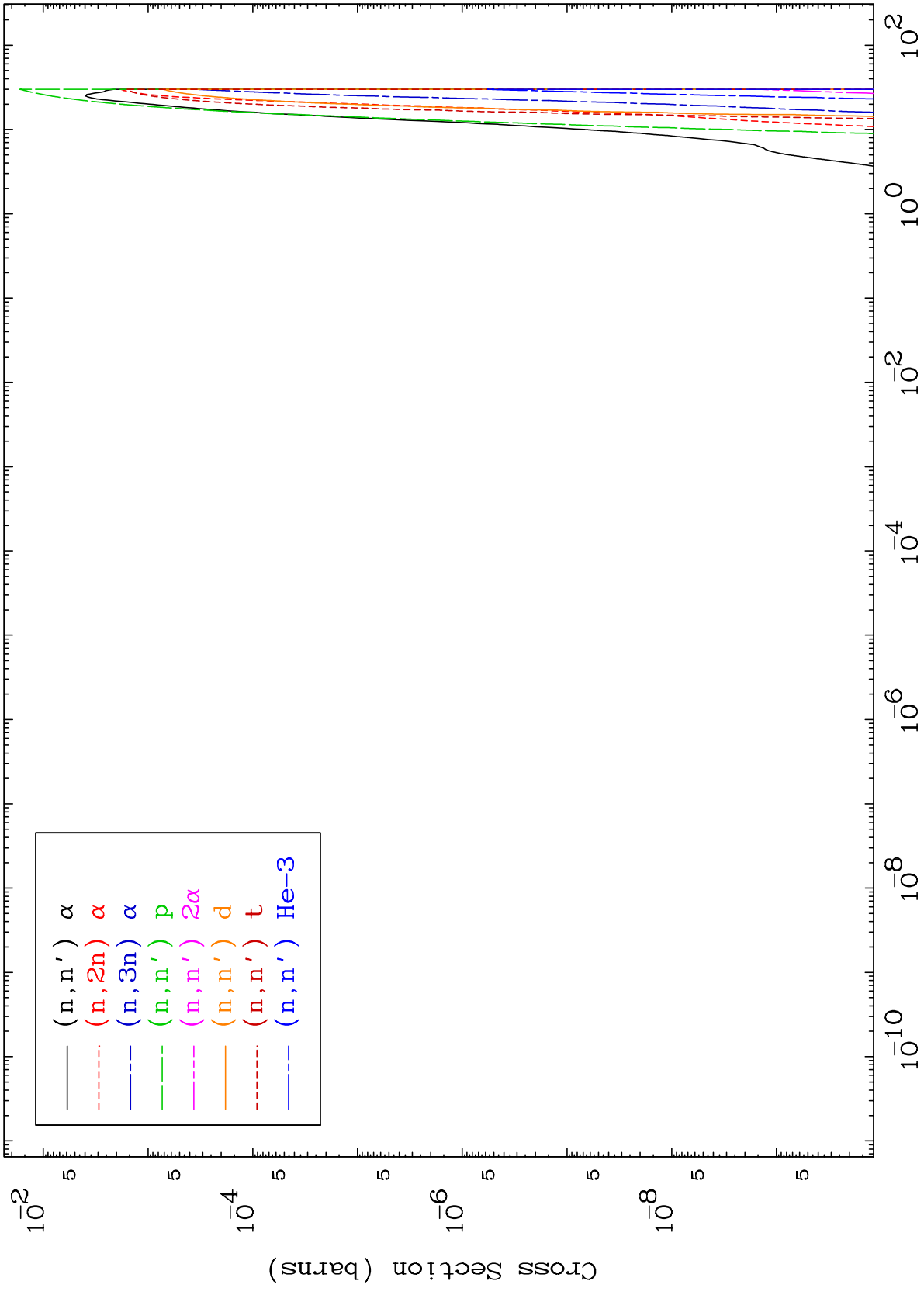
Incident Energy (MeV)

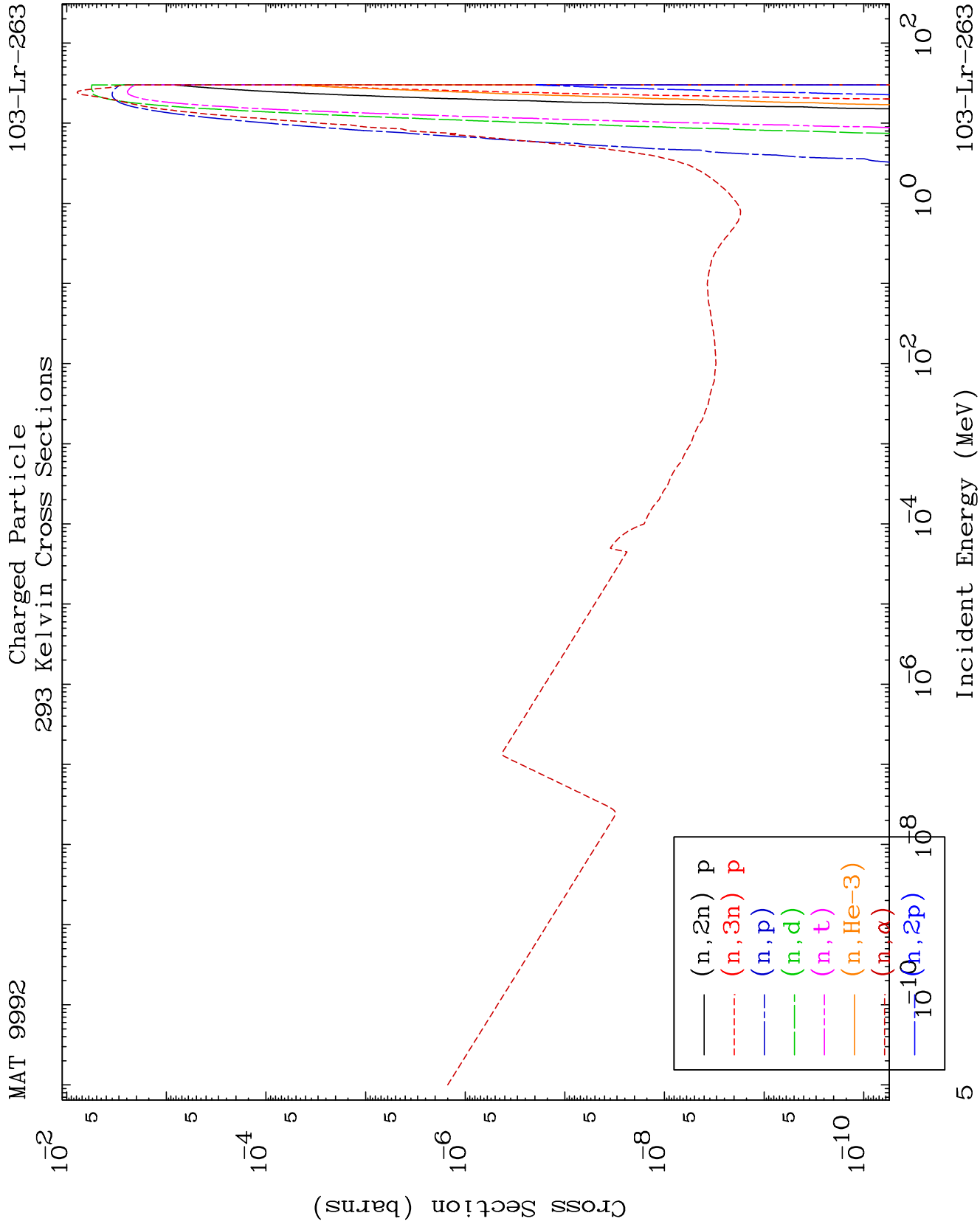
103-Lr-263

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Charged Particle
293 Kelvin Cross Sections

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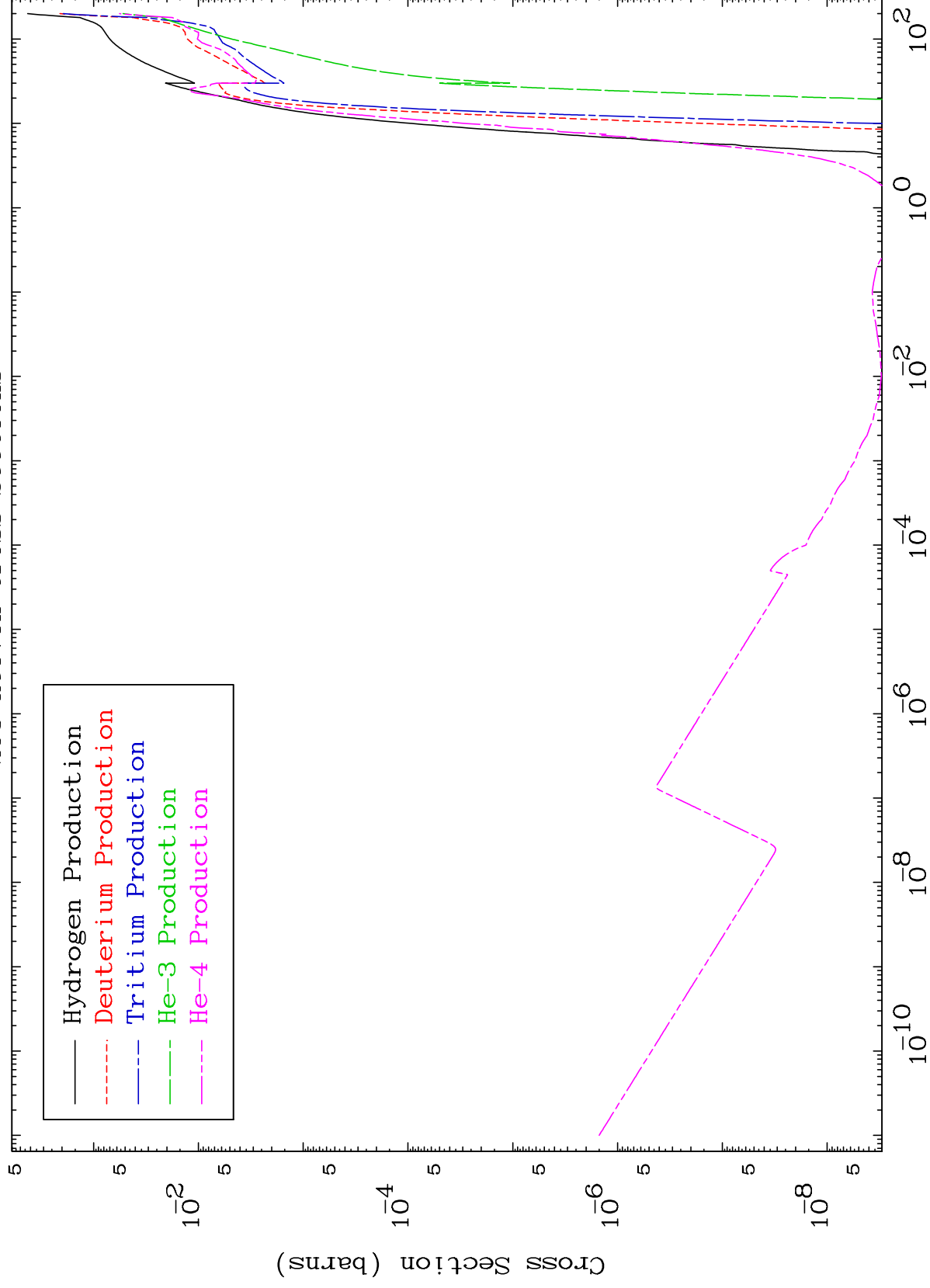




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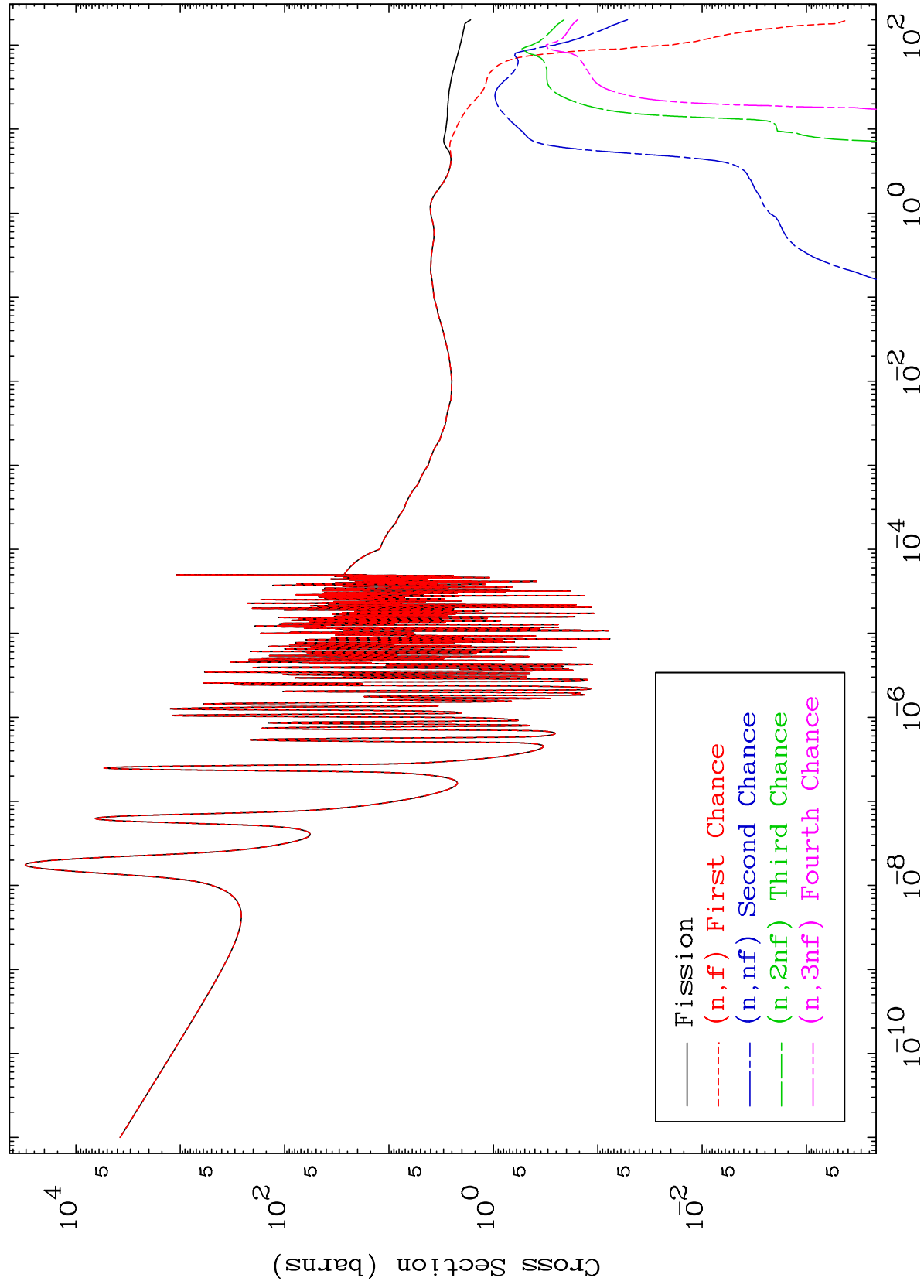
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293 Kelvin Cross Sections

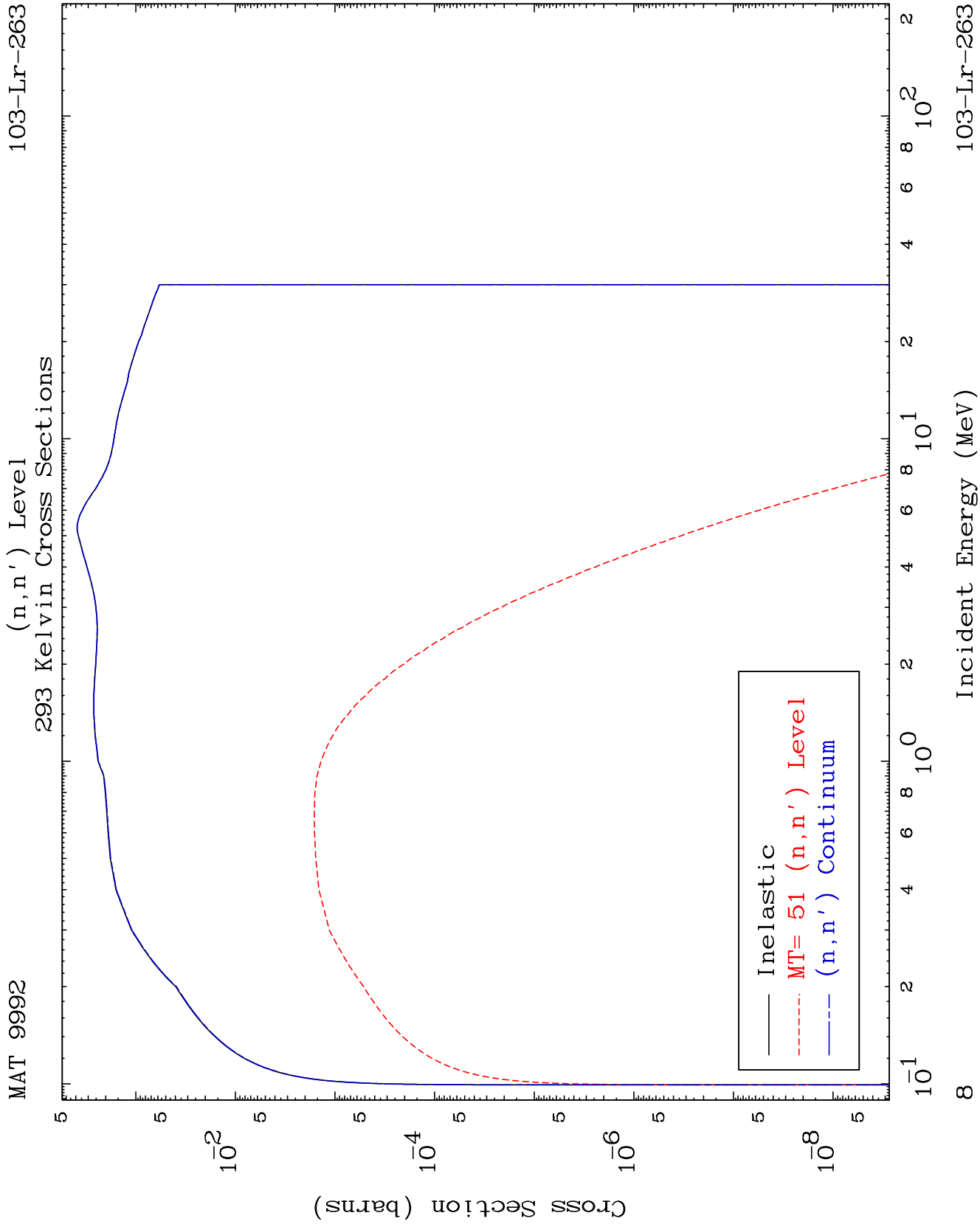
103-Lr-263



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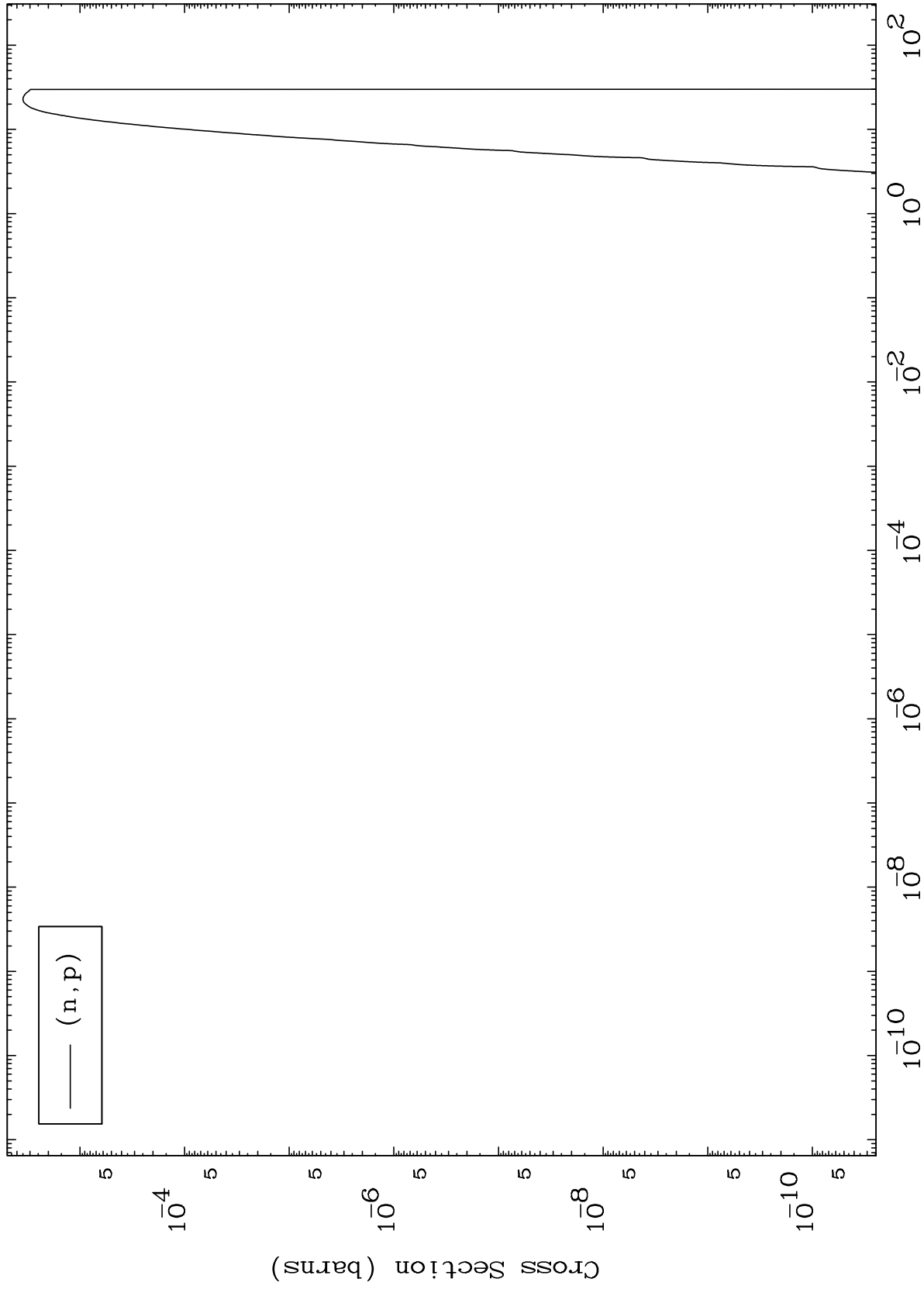




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(n,p) Levels
293 Kelvin Cross Sections

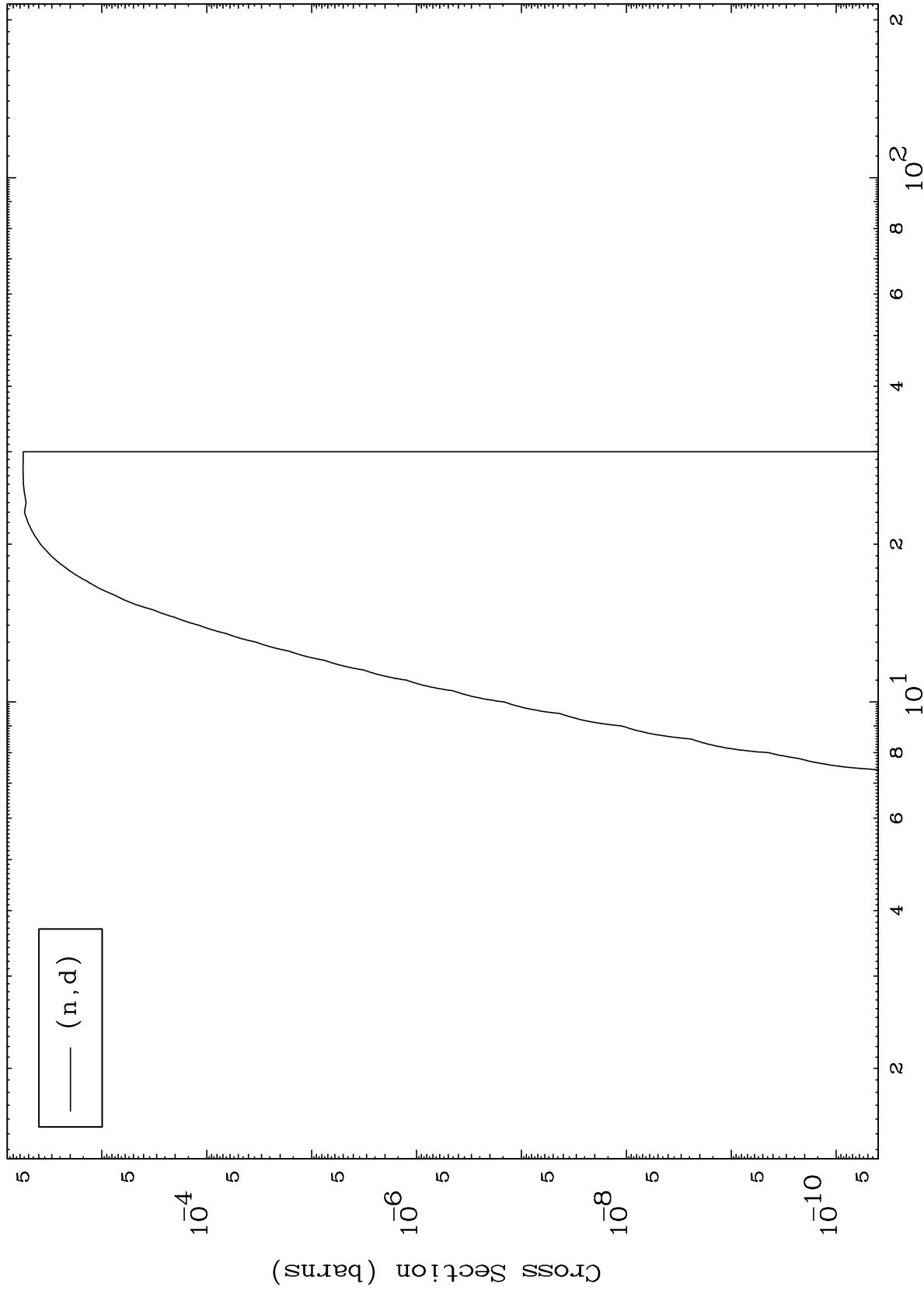
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(n,d) Levels
293 Kelvin Cross Sections

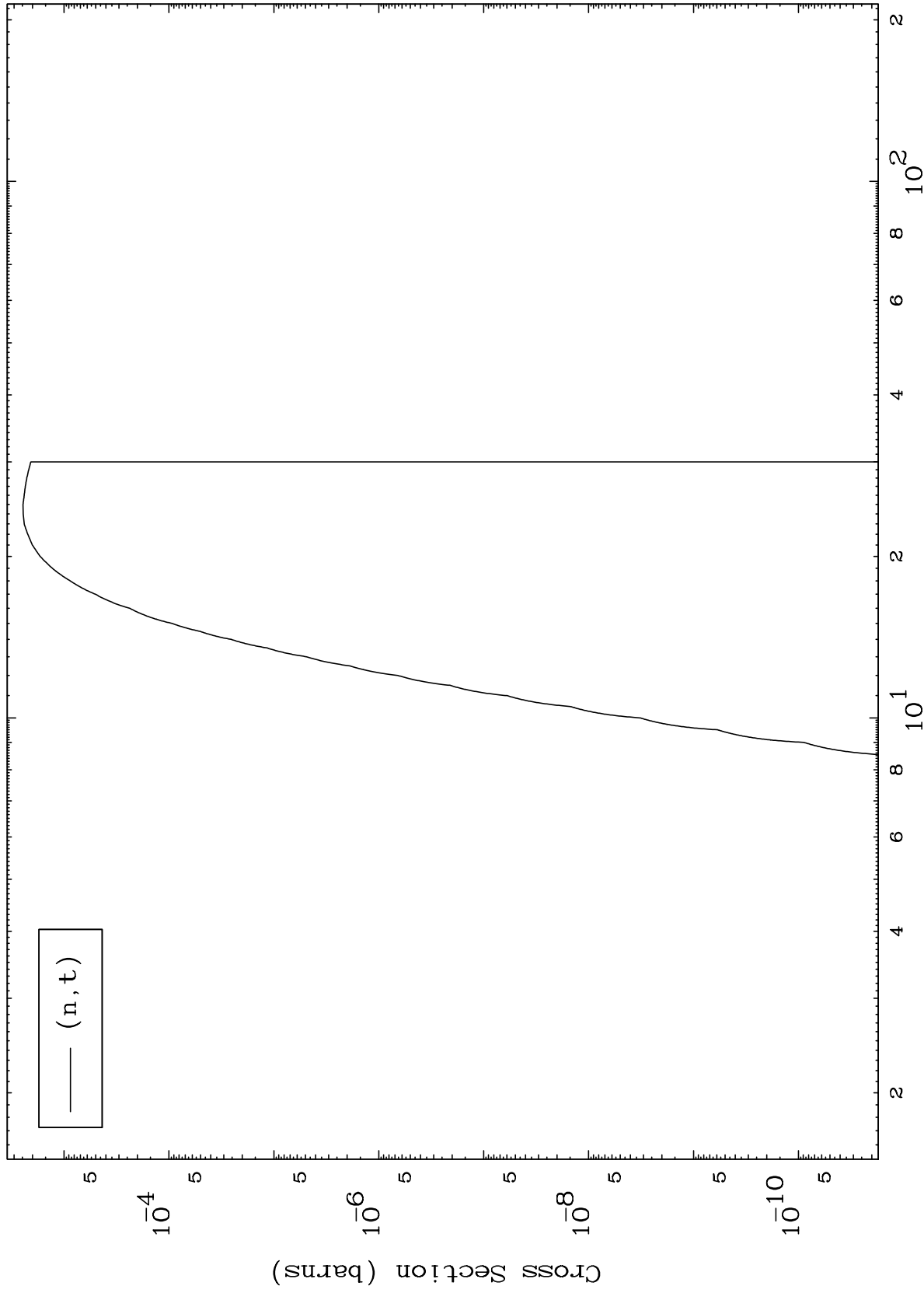
103-Lr-263



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(n,t) Levels
293 Kelvin Cross Sections

103-Lr-263



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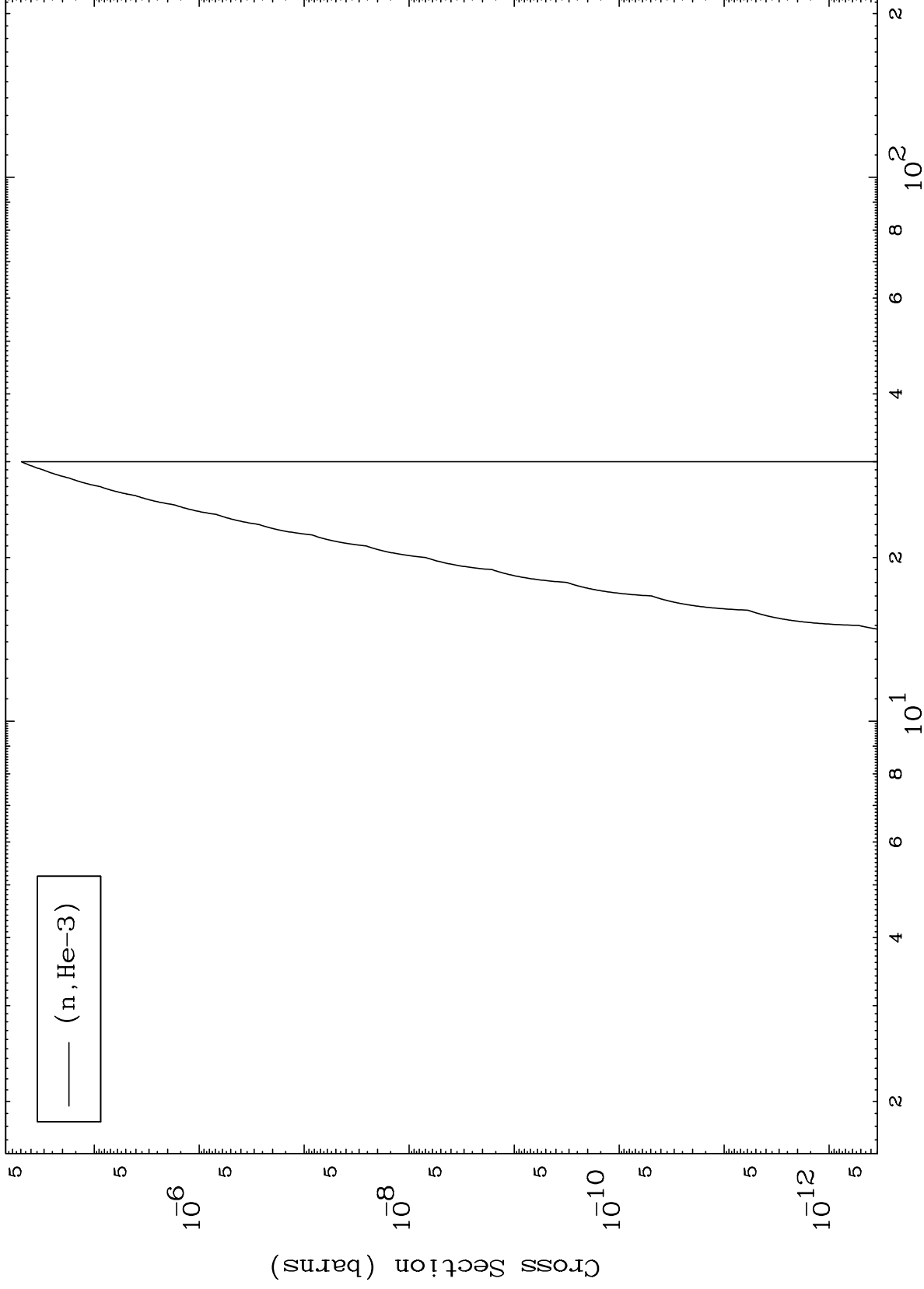
Incident Energy (MeV)

103-Lr-263

MAT 9992

(n,He3) Levels
293 Kelvin Cross Sections

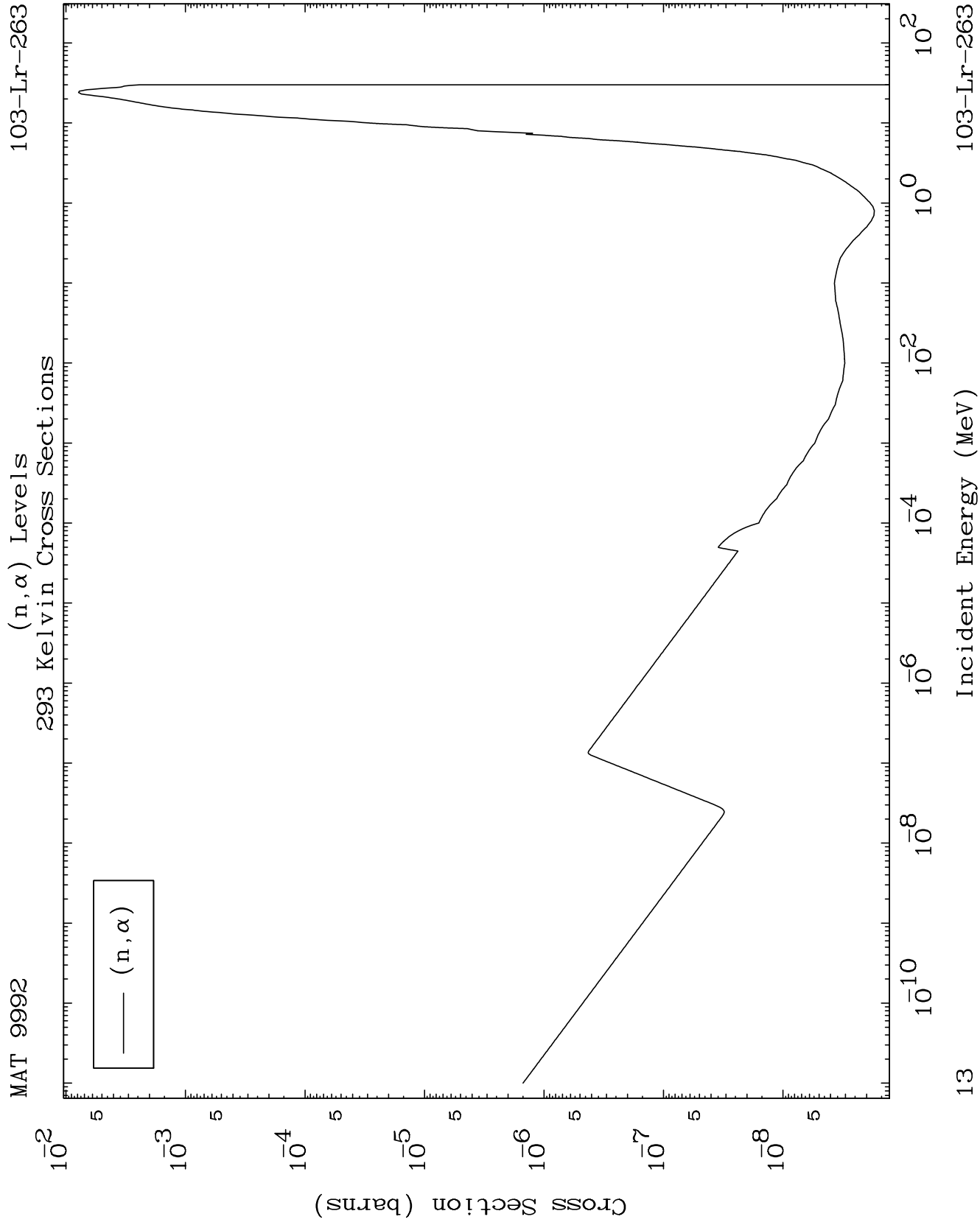
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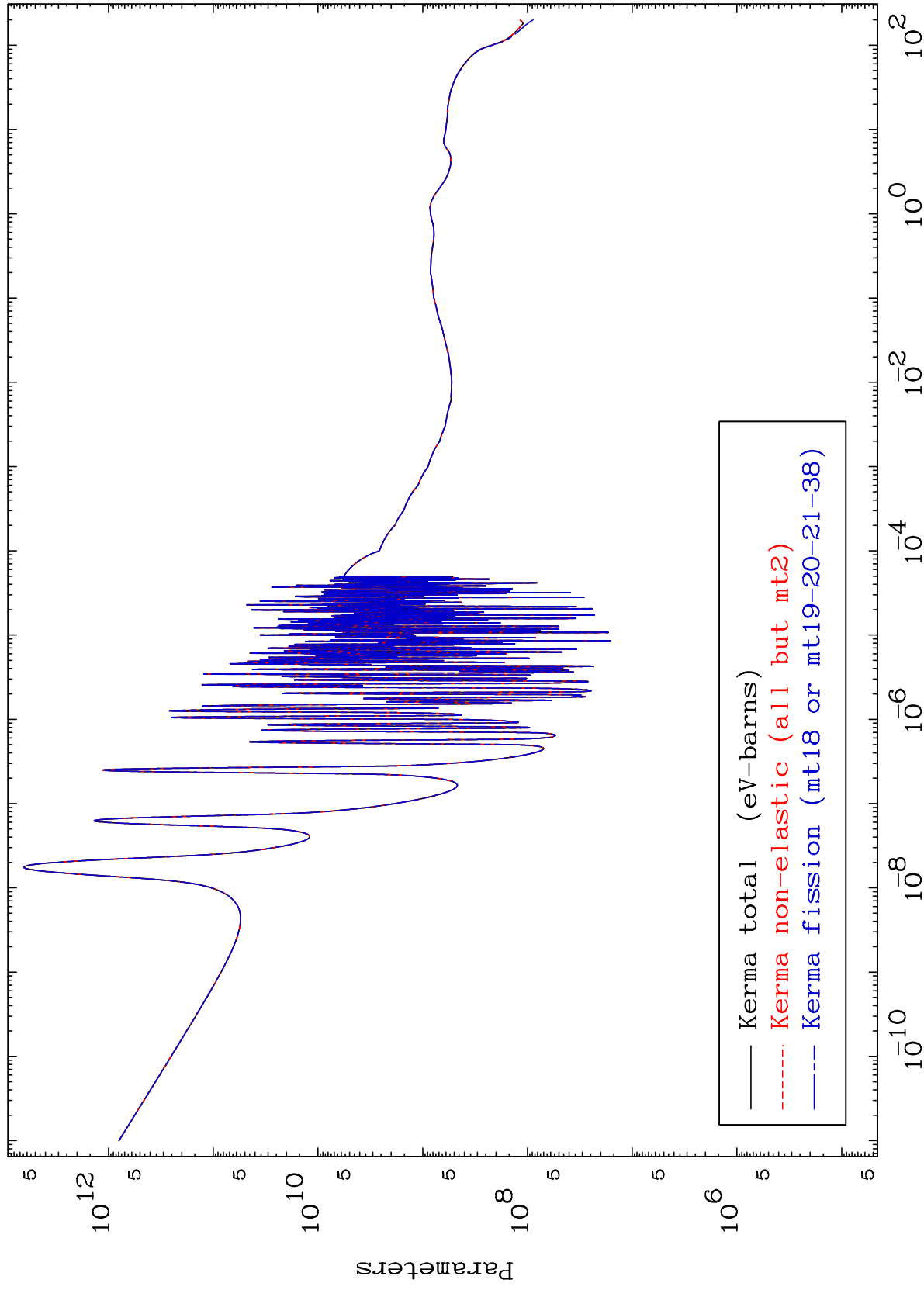


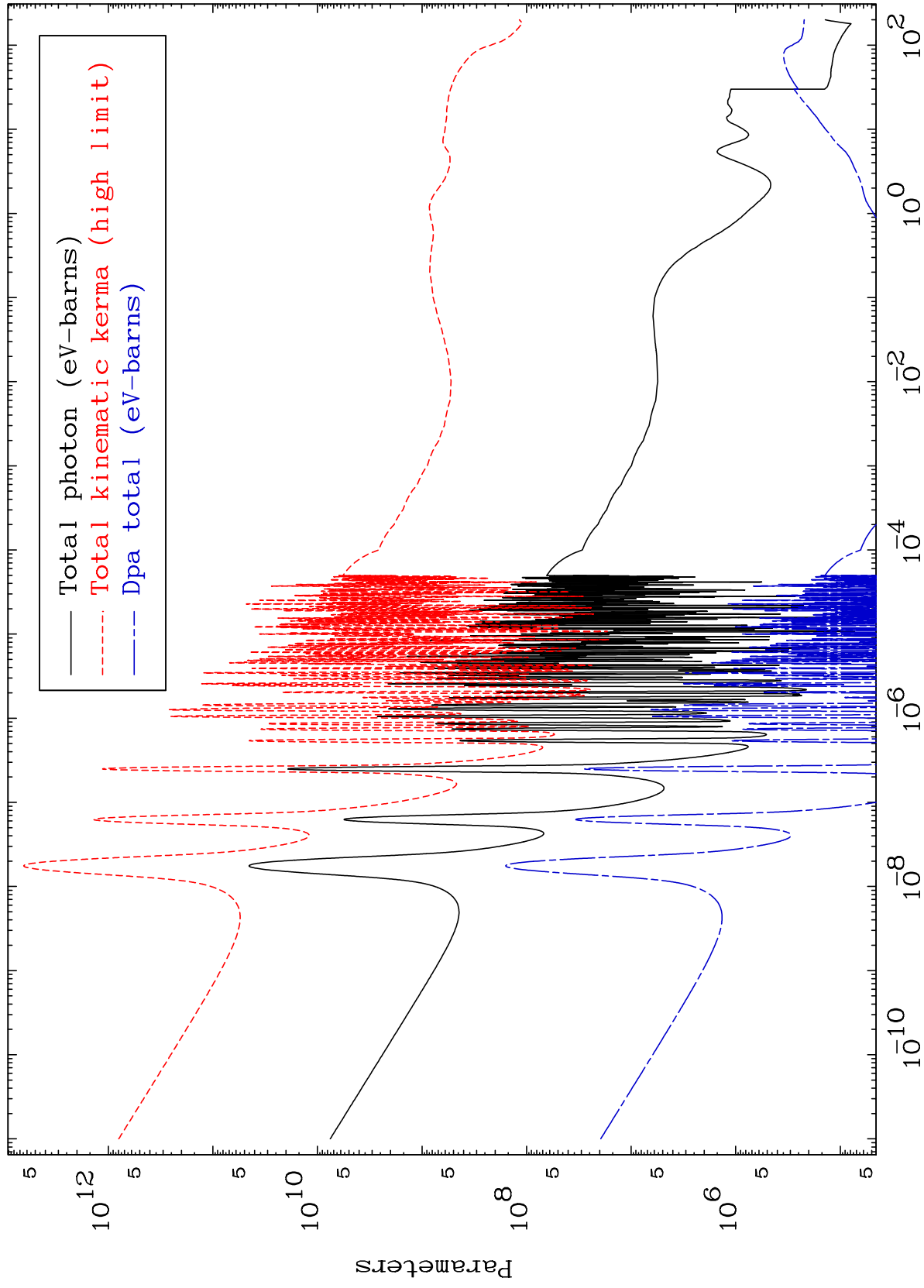
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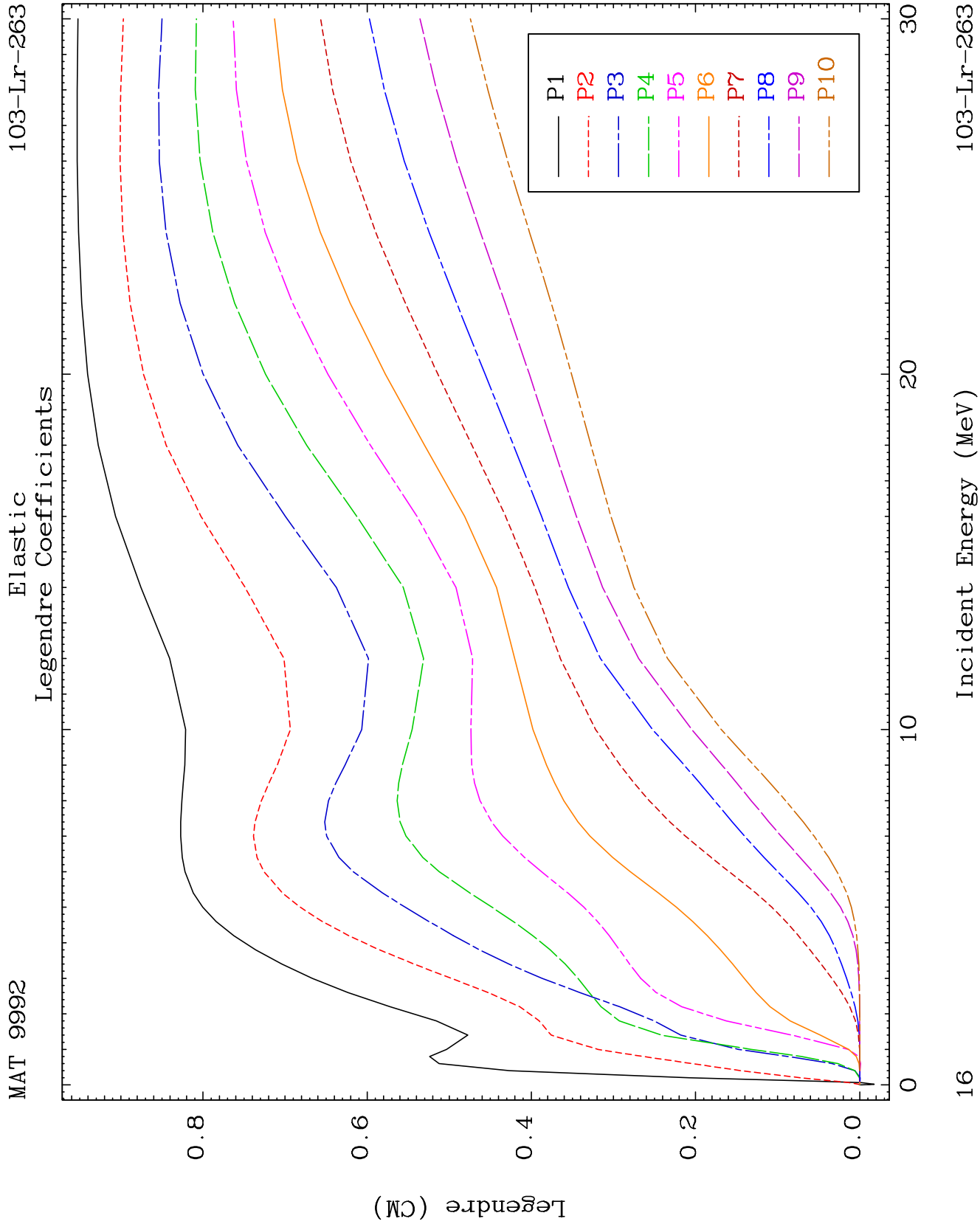
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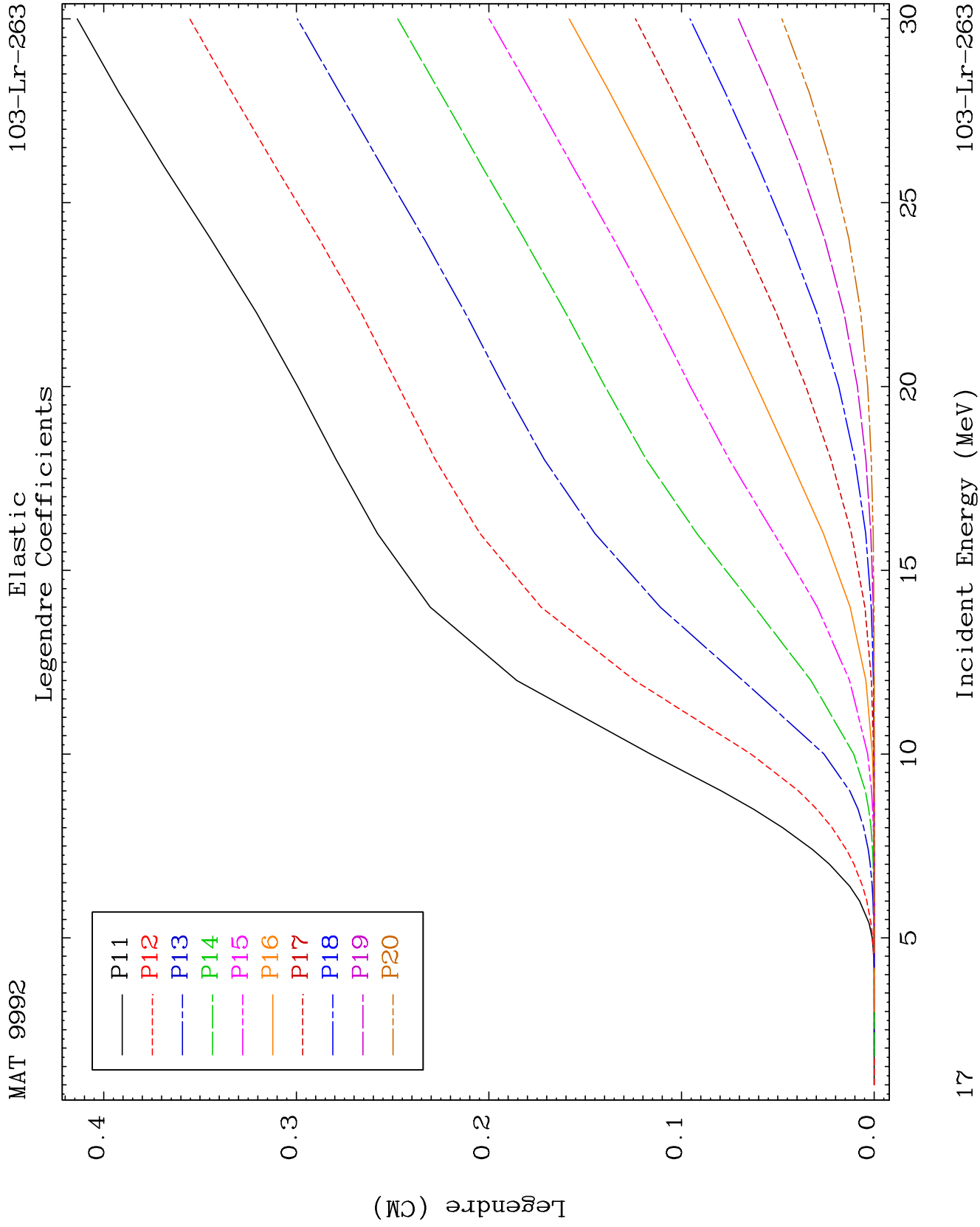
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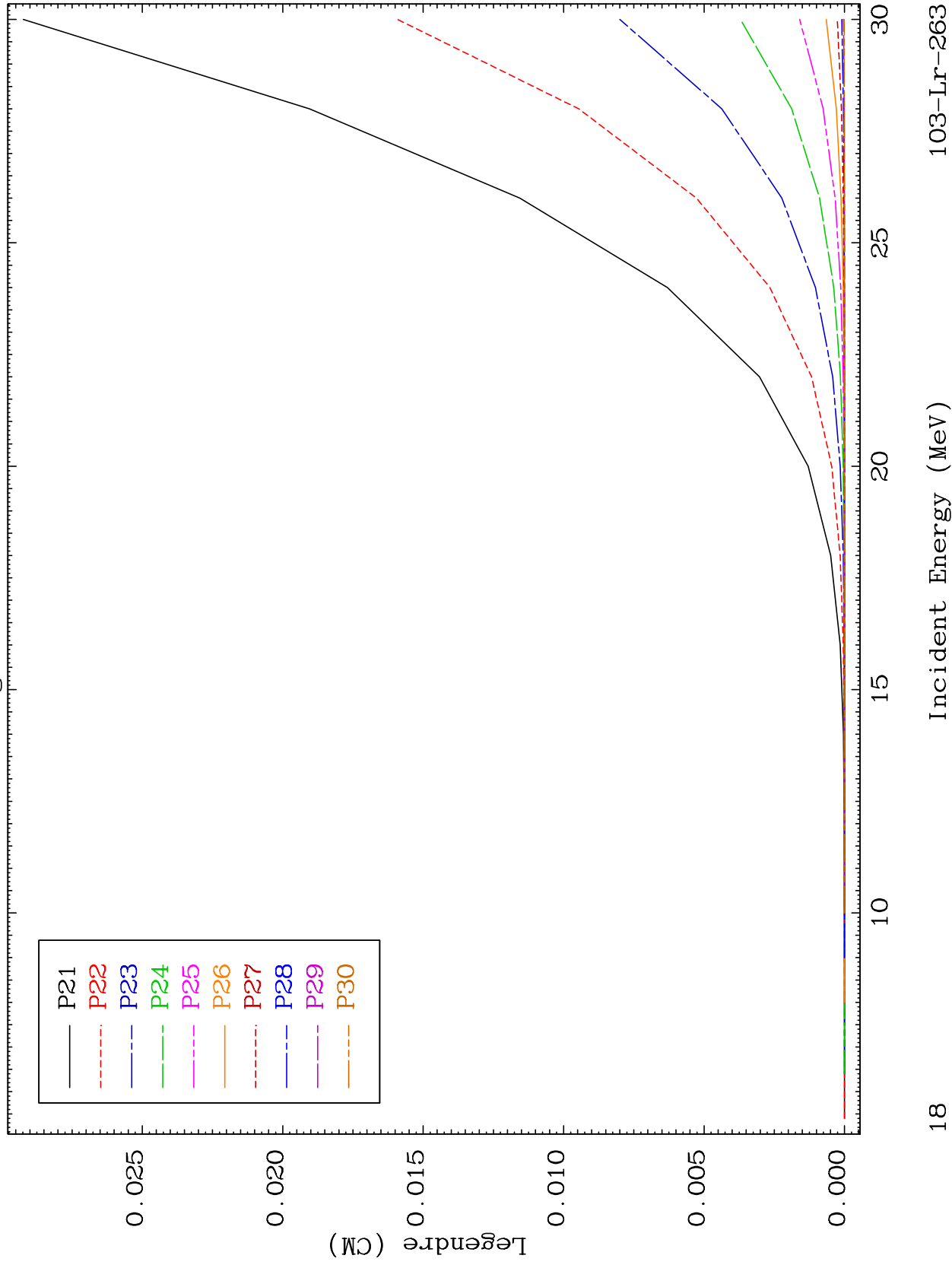


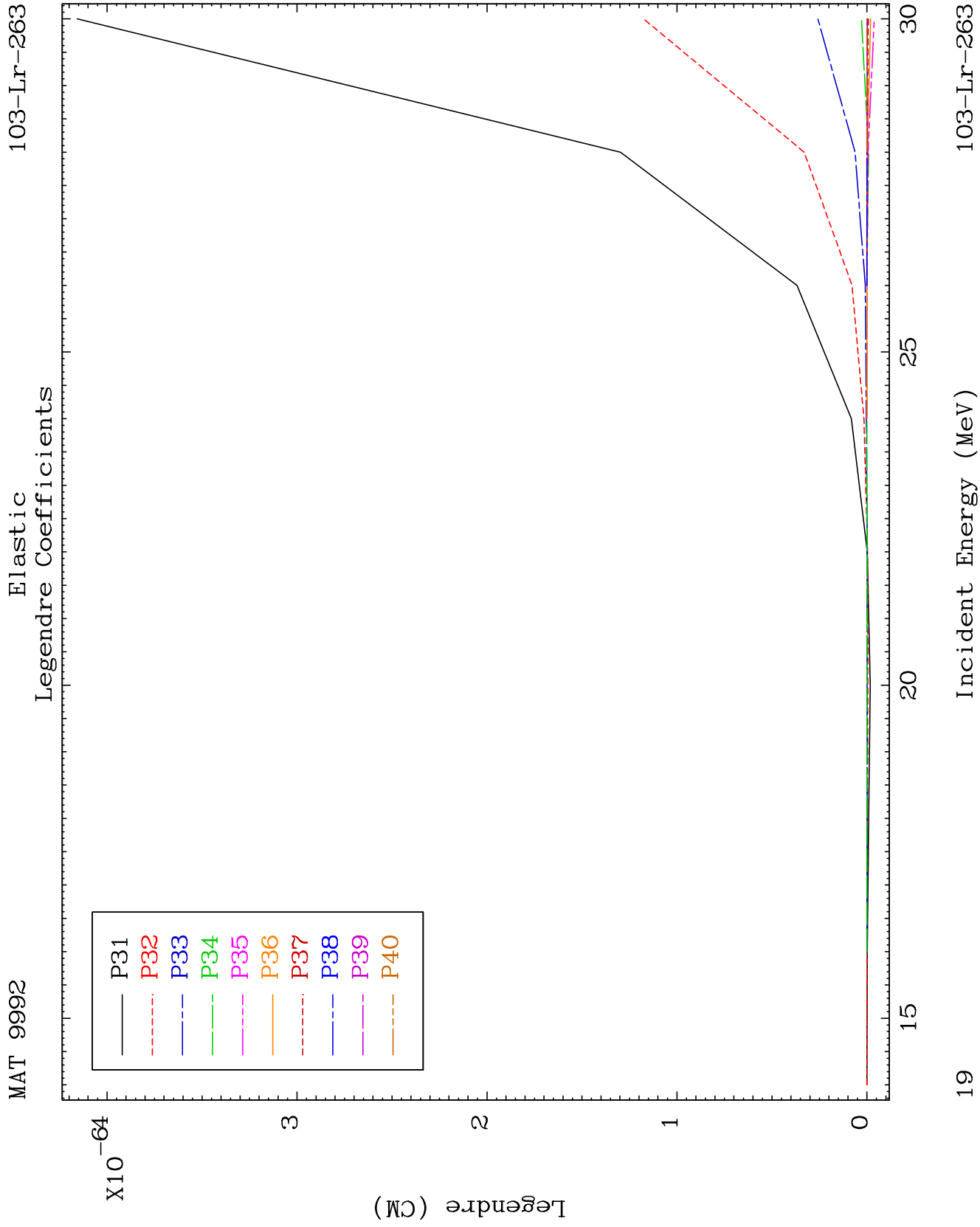




103-Lr-263

103-Lr-263

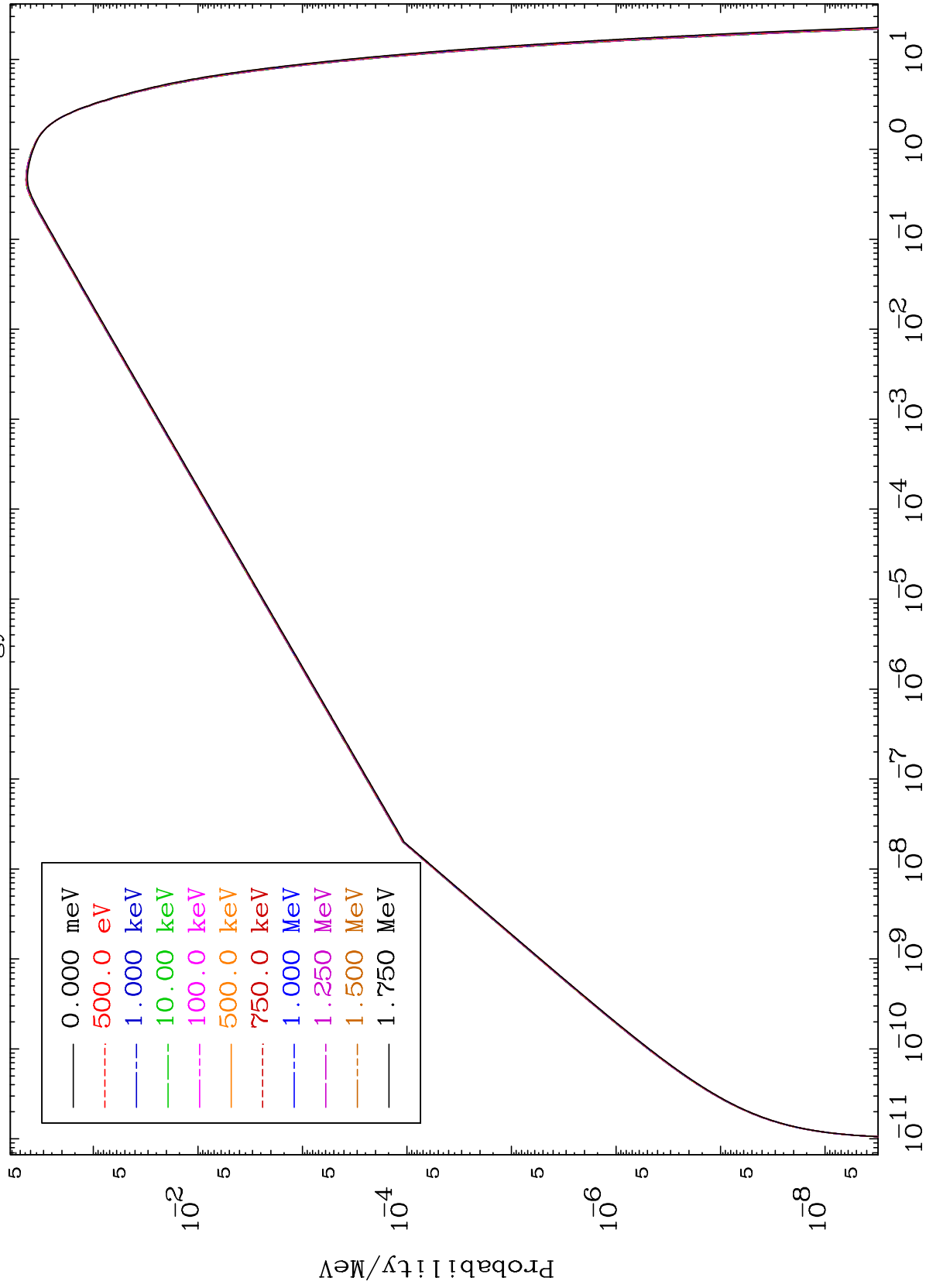




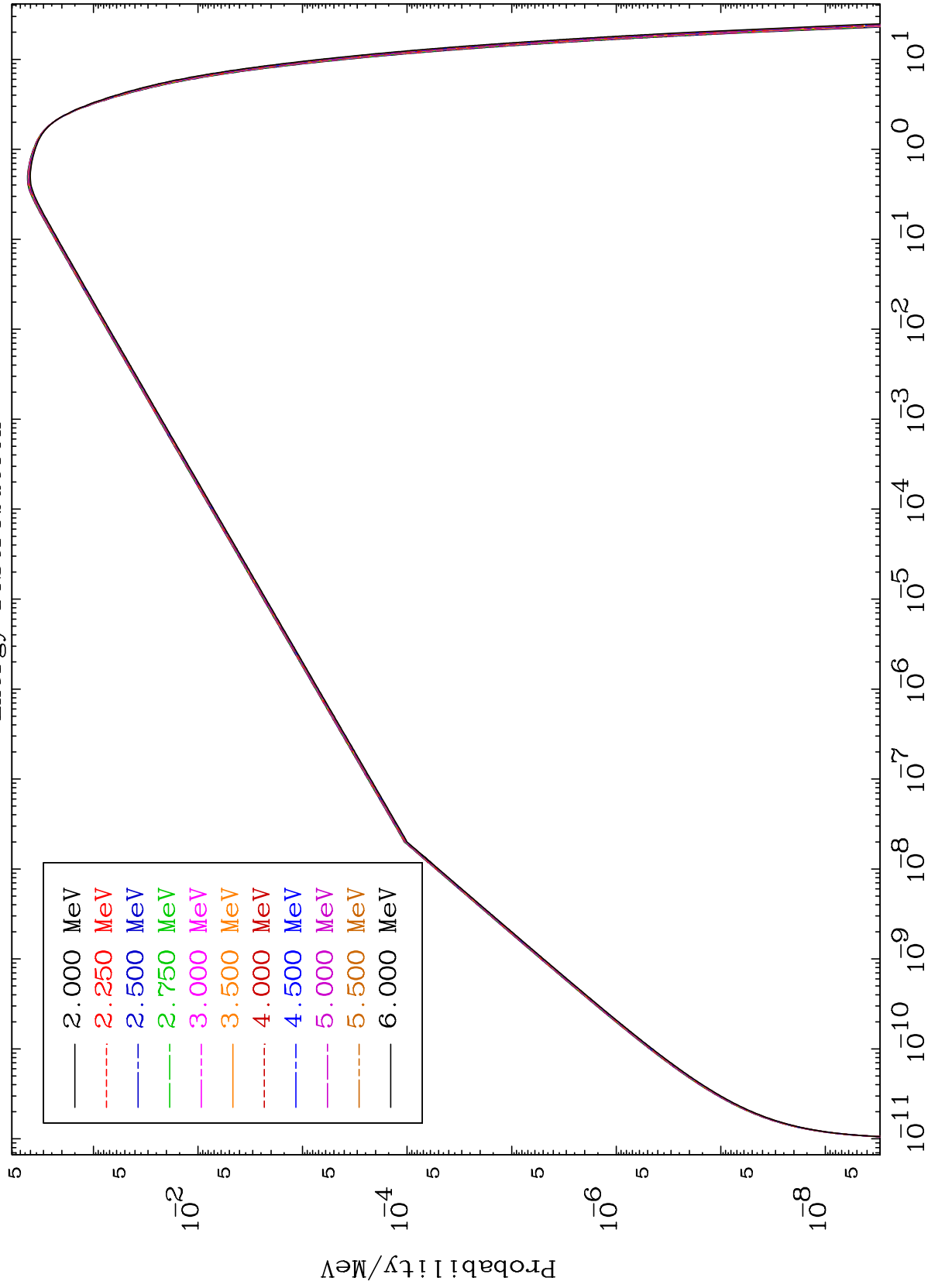
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Fission
Energy Distribution

103-Lr-263



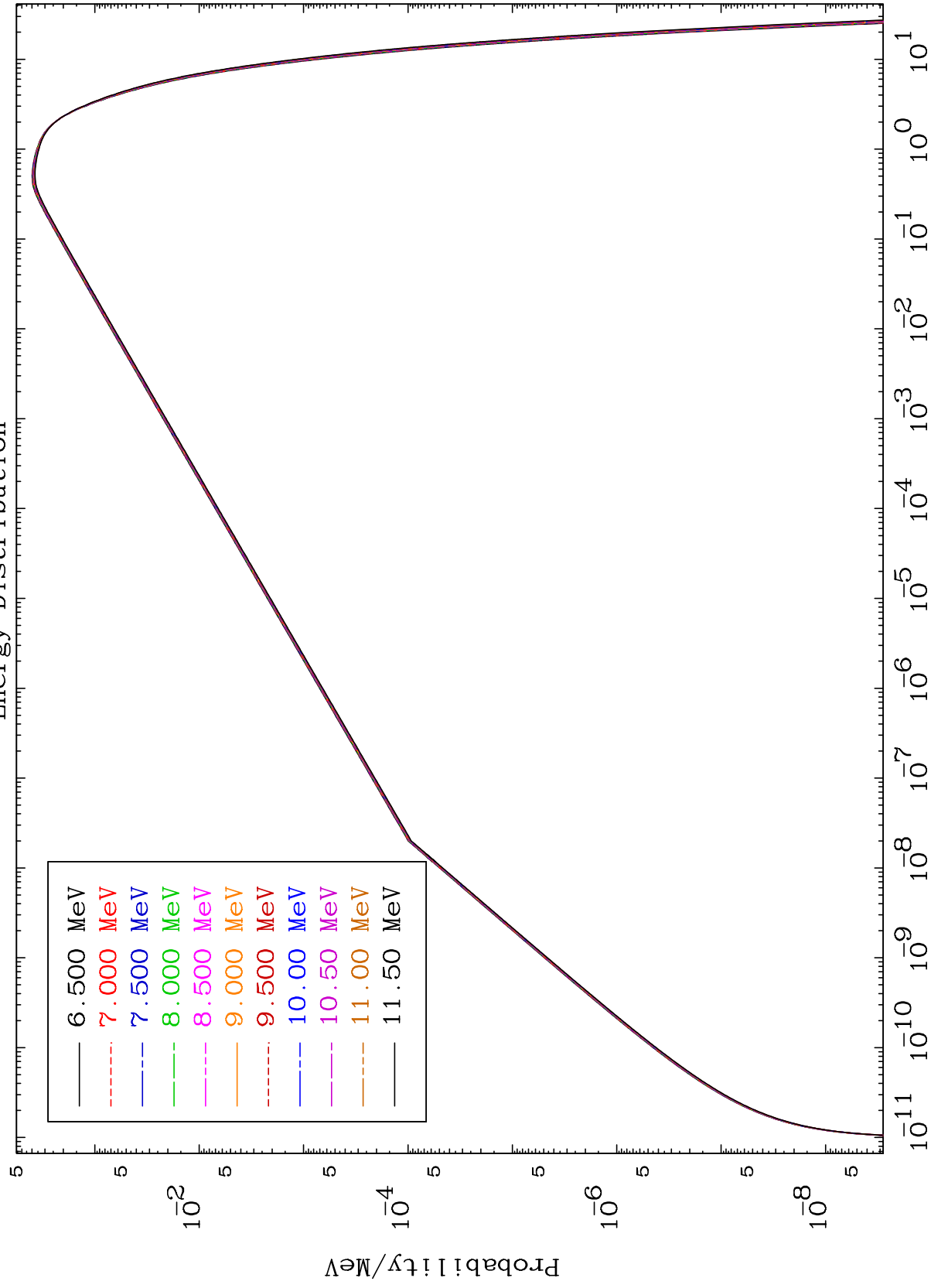
Fission Energy Distribution



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Fission
Energy Distribution

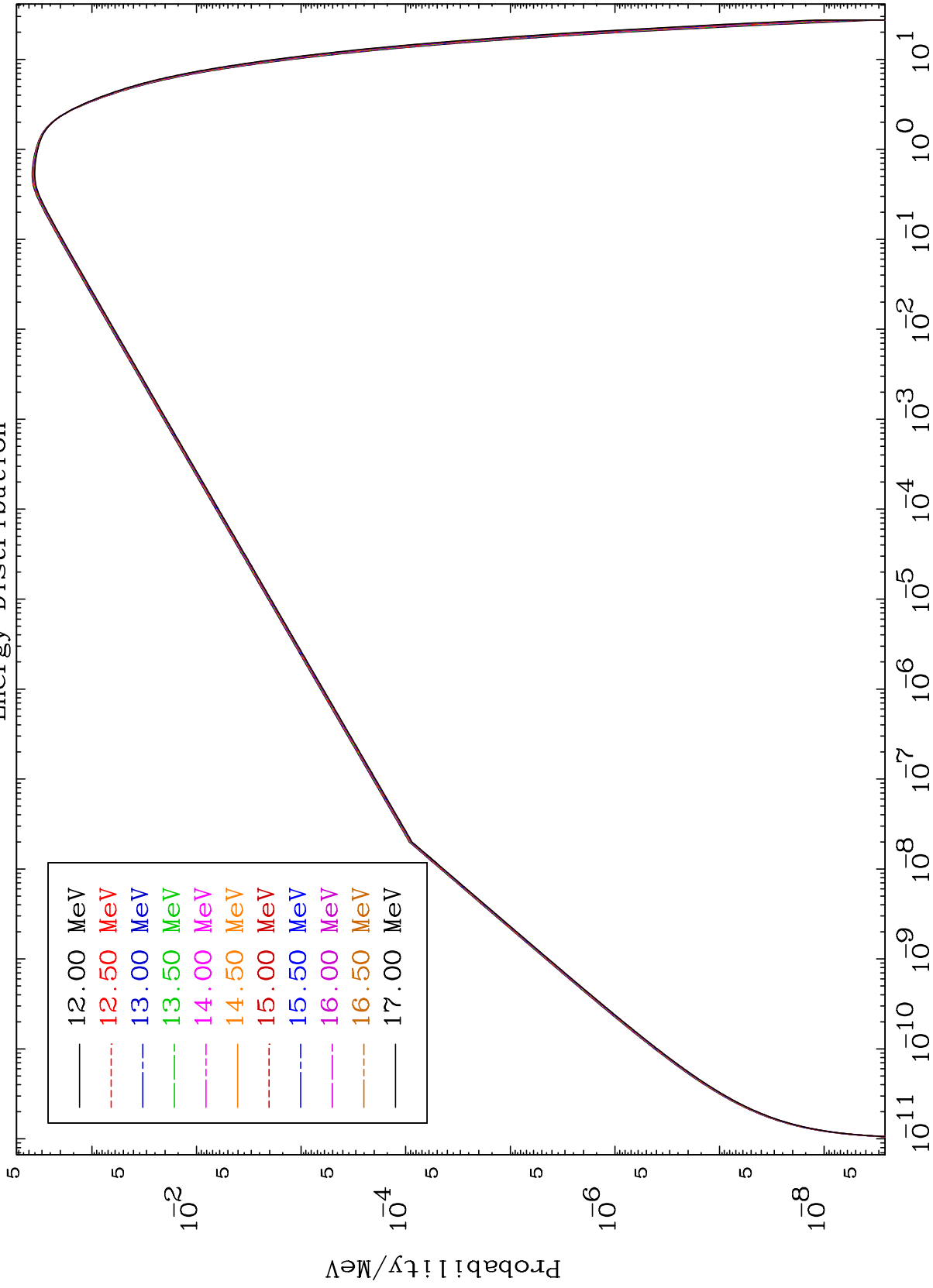
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Fission
Energy Distribution

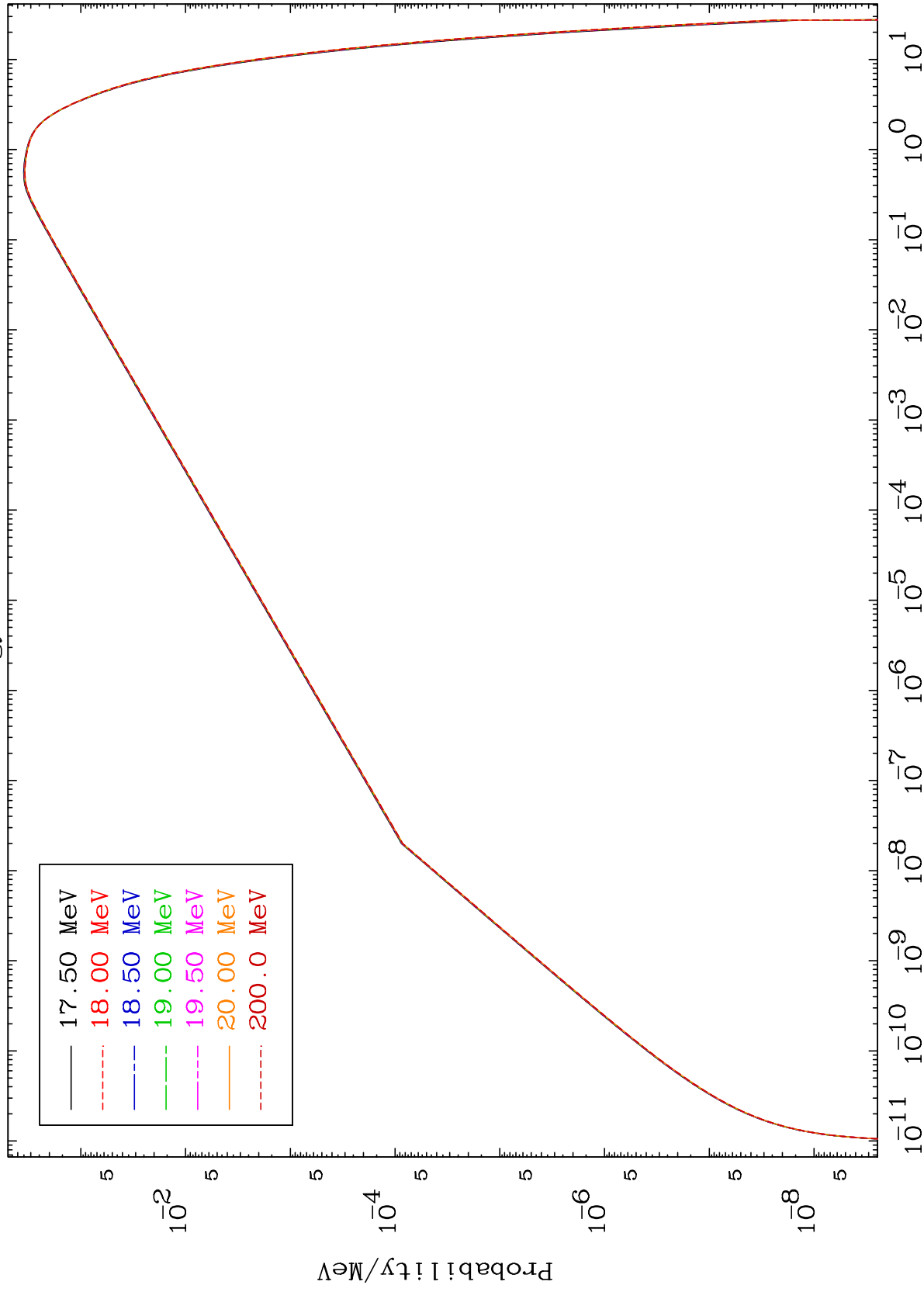
103-Lr-263



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Fission
Energy Distribution

103-Lr-263



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Secondary Energy (MeV)

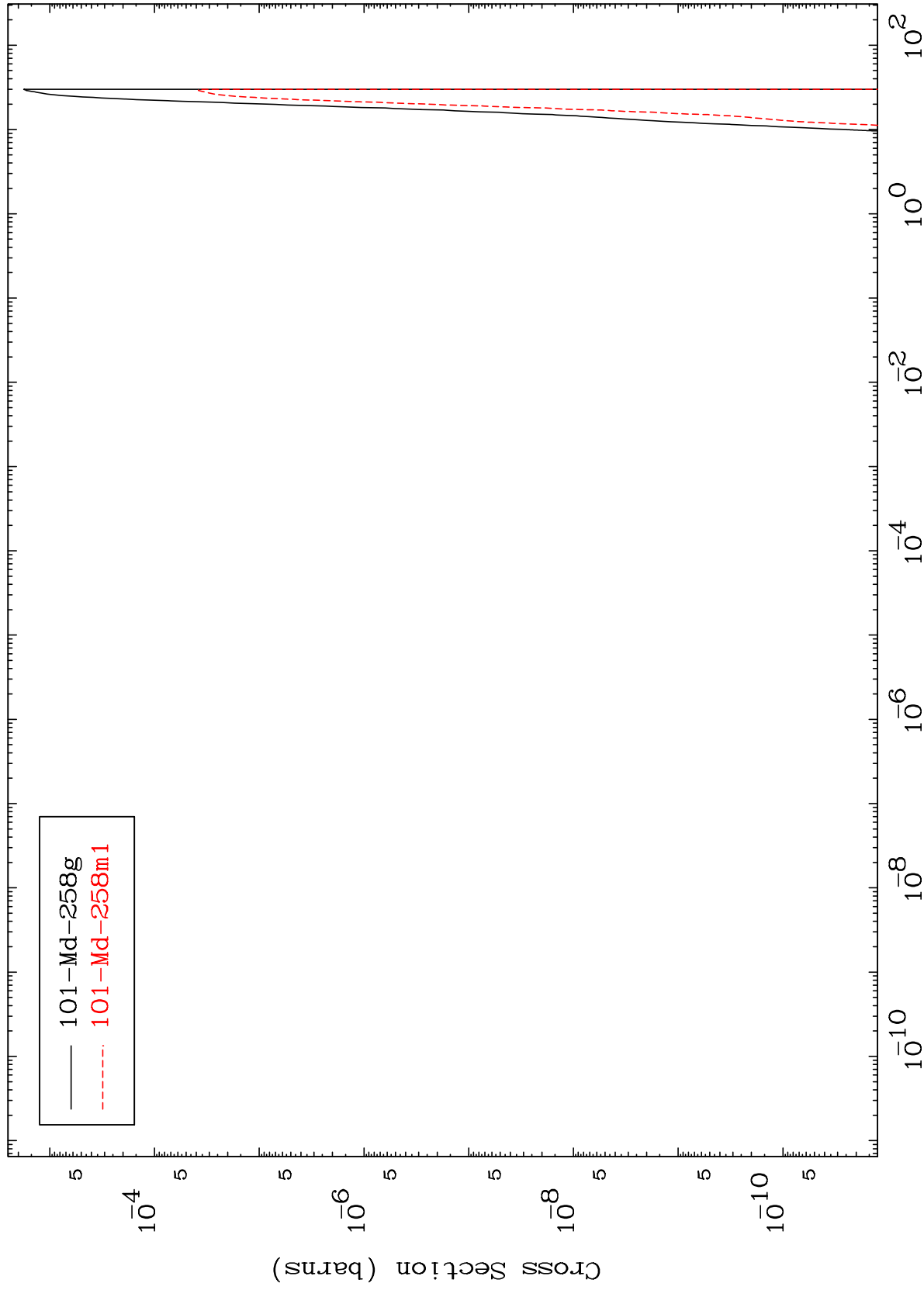
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$(n,2n) \alpha$

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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