

Program EVALPLOT  
(Version 2018-1)

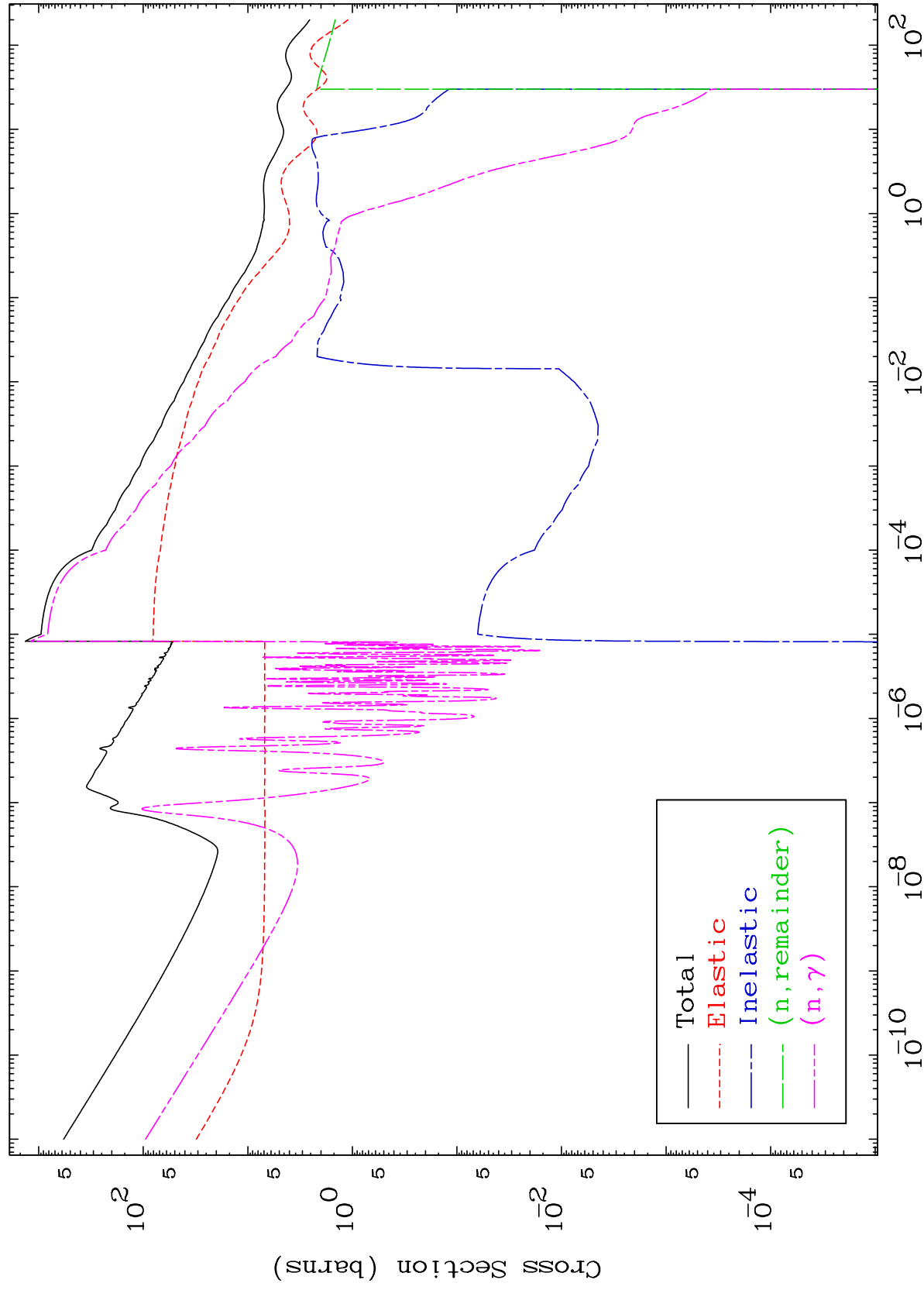
by

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Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

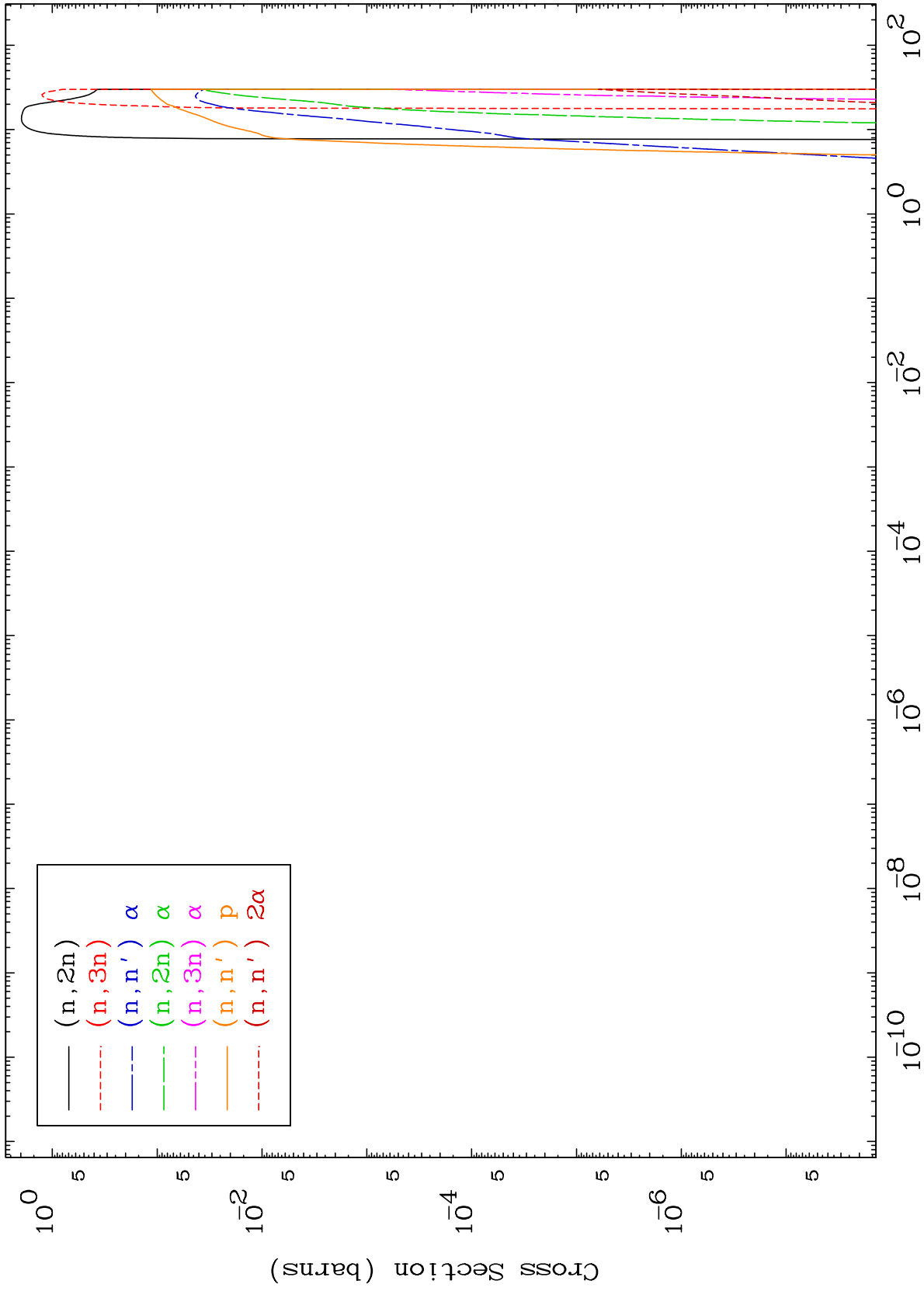
Press Mouse Button to Start



MAT 7100

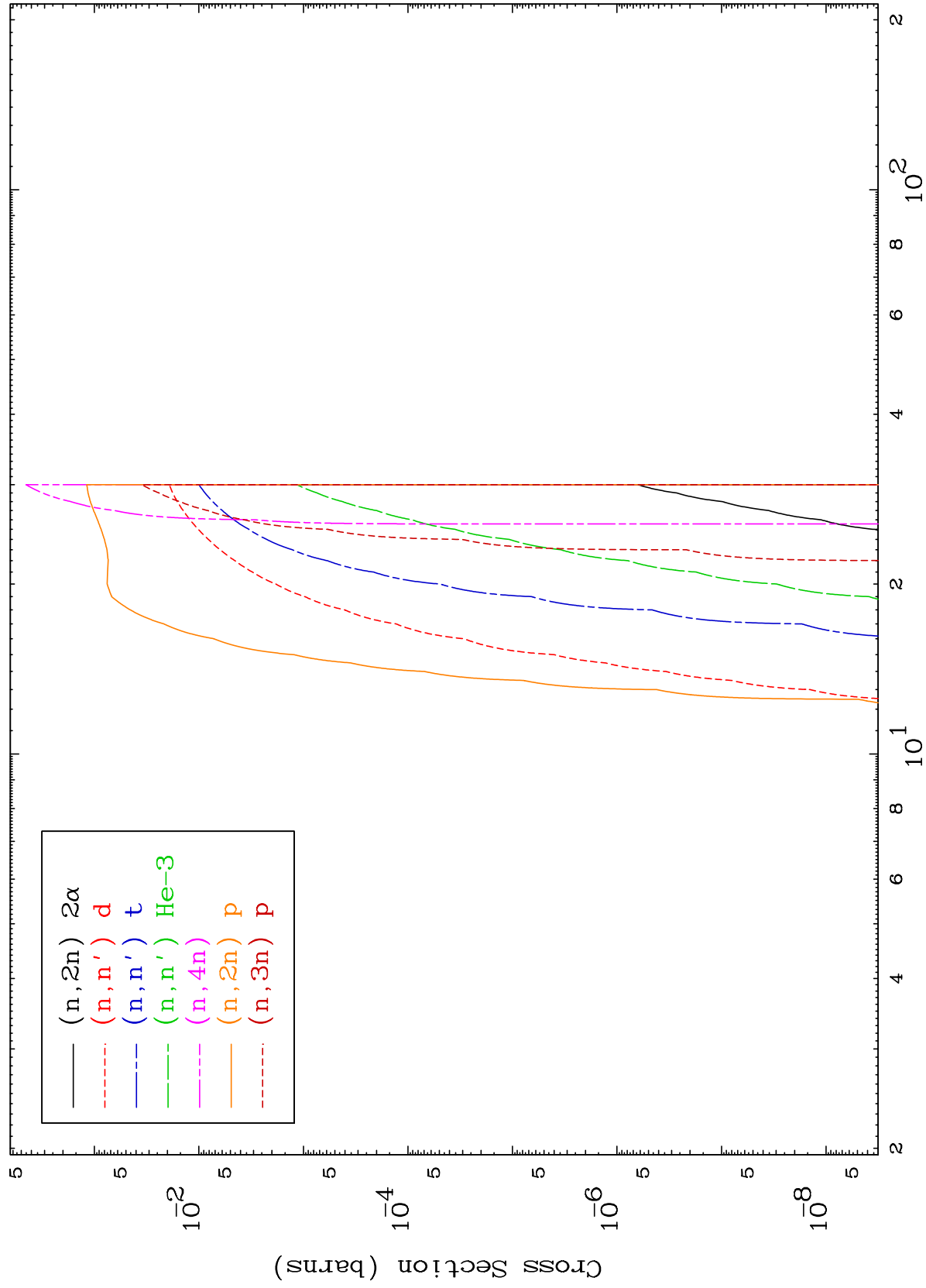
Neutron Production  
293 Kelvin Cross Sections

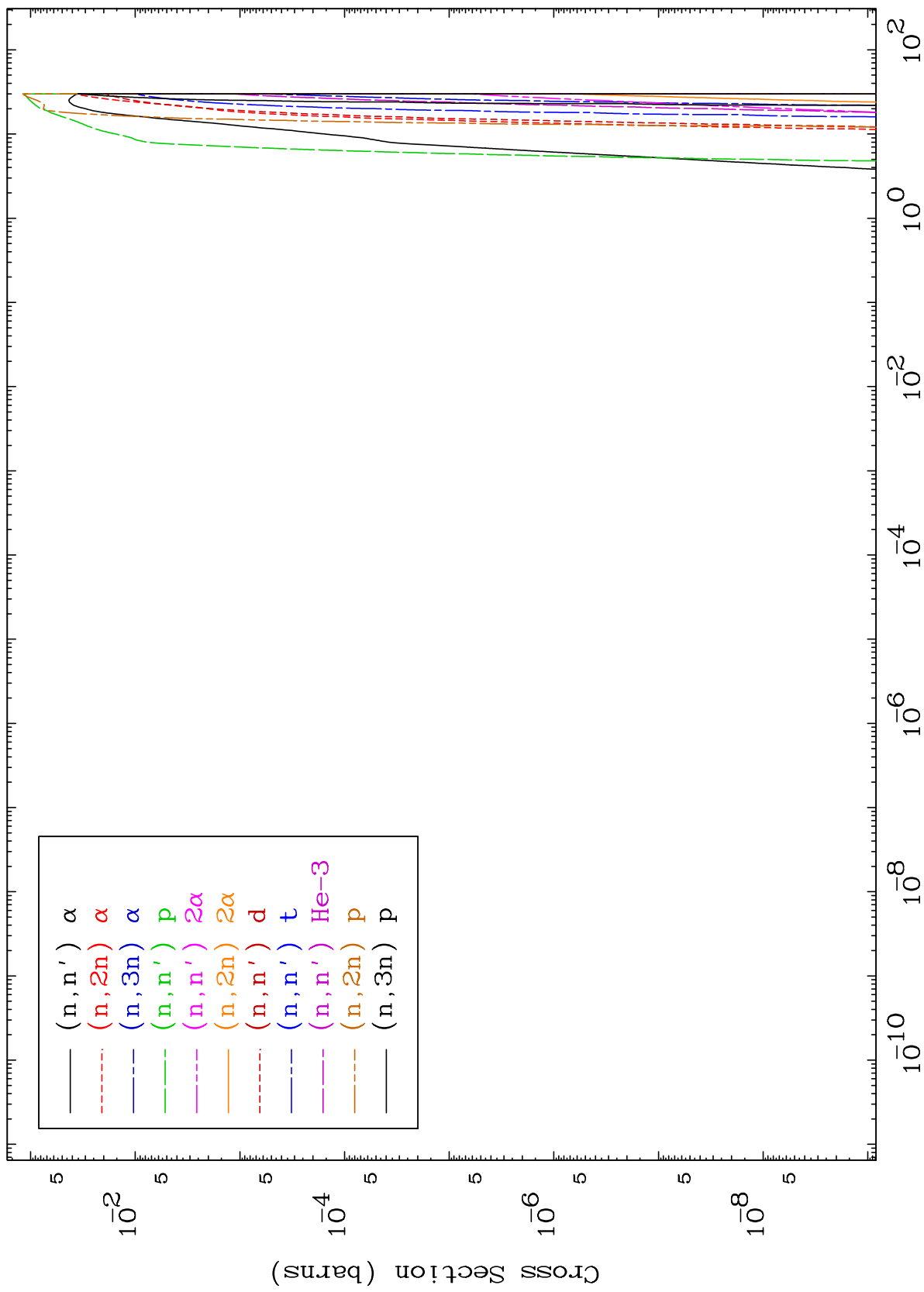
71-Lu-166

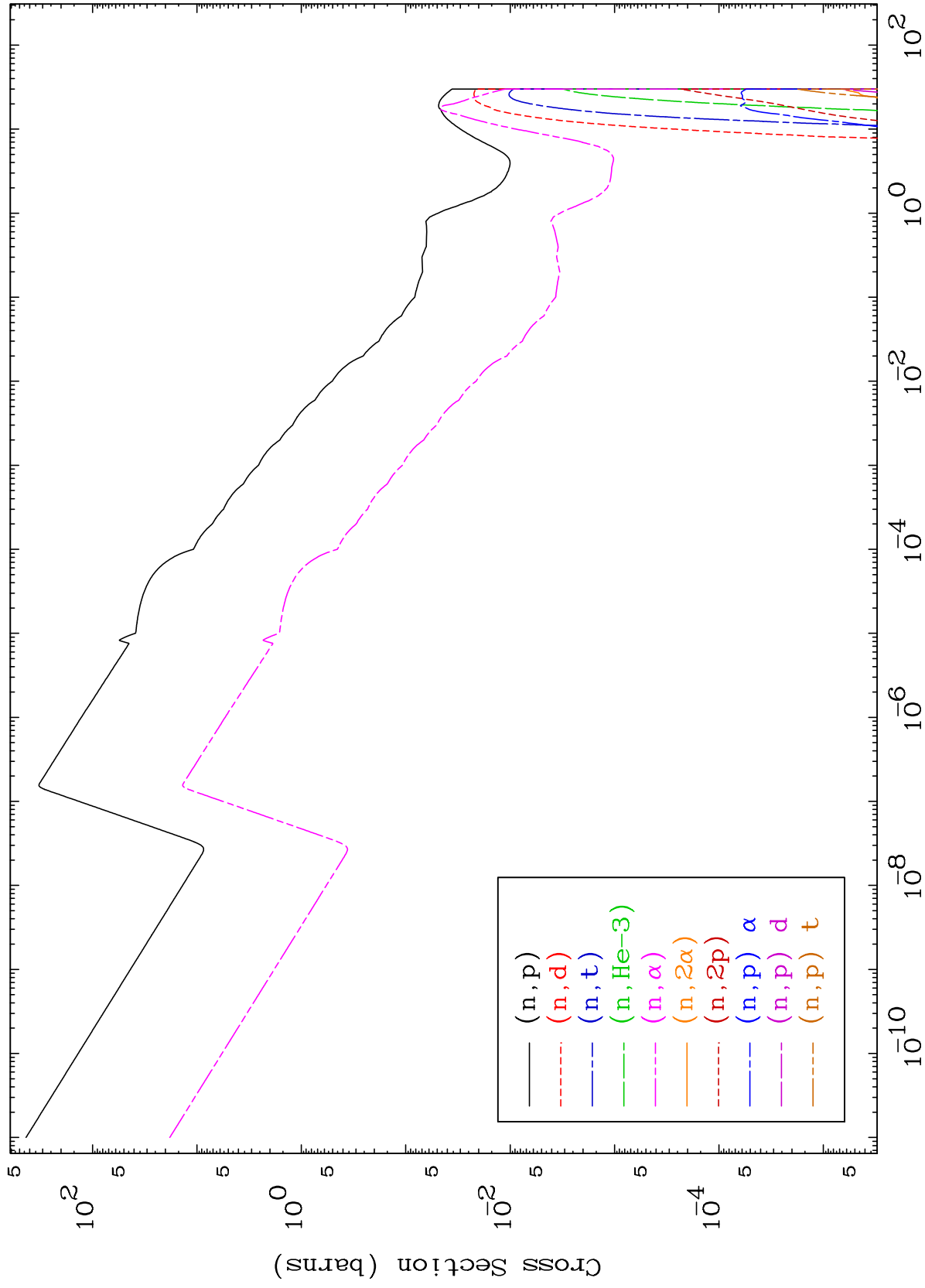


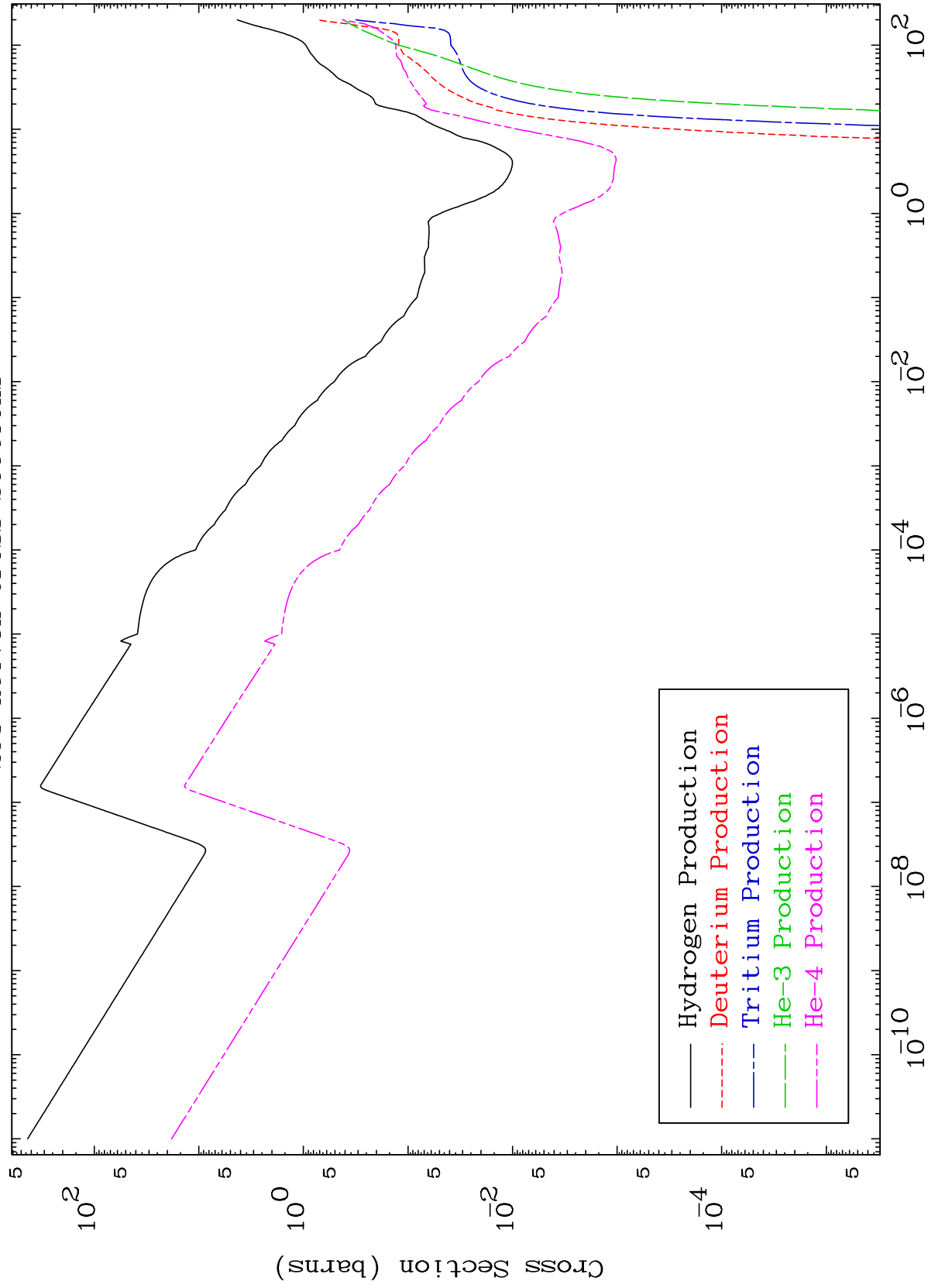
2

71-Lu-166







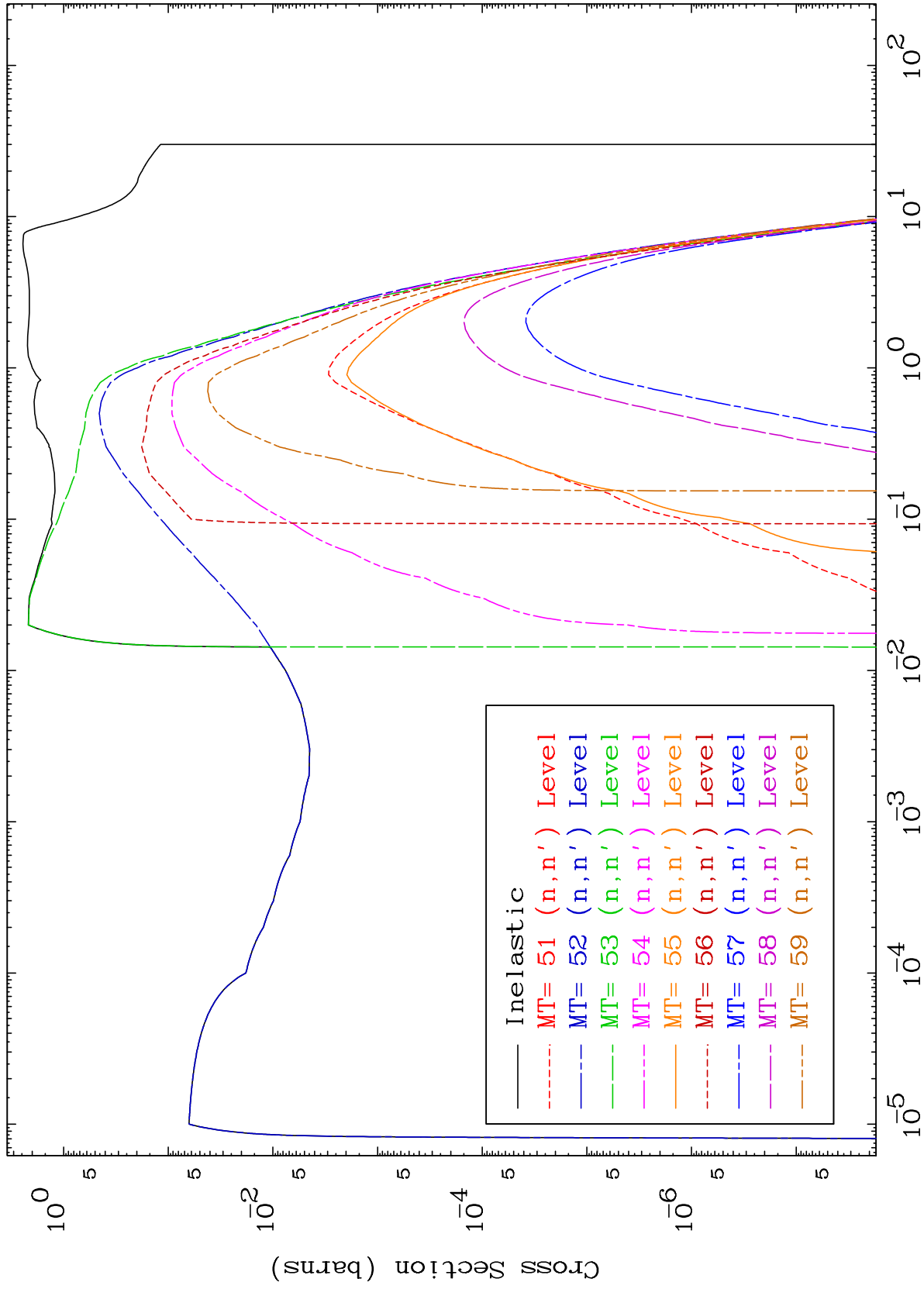


MAT 7100

(n,n') Level

71-Lu-166

293 Kelvin Cross Sections



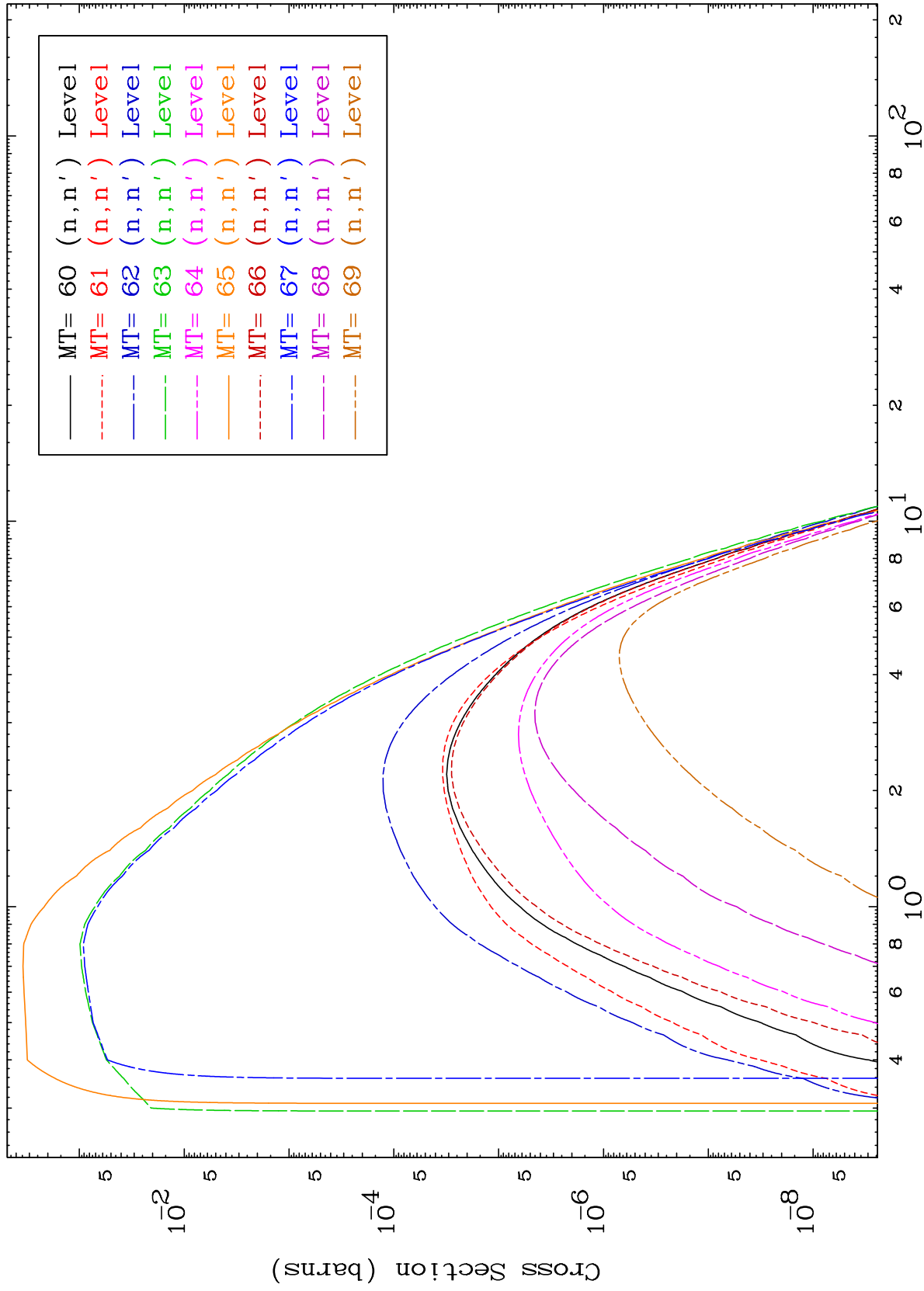


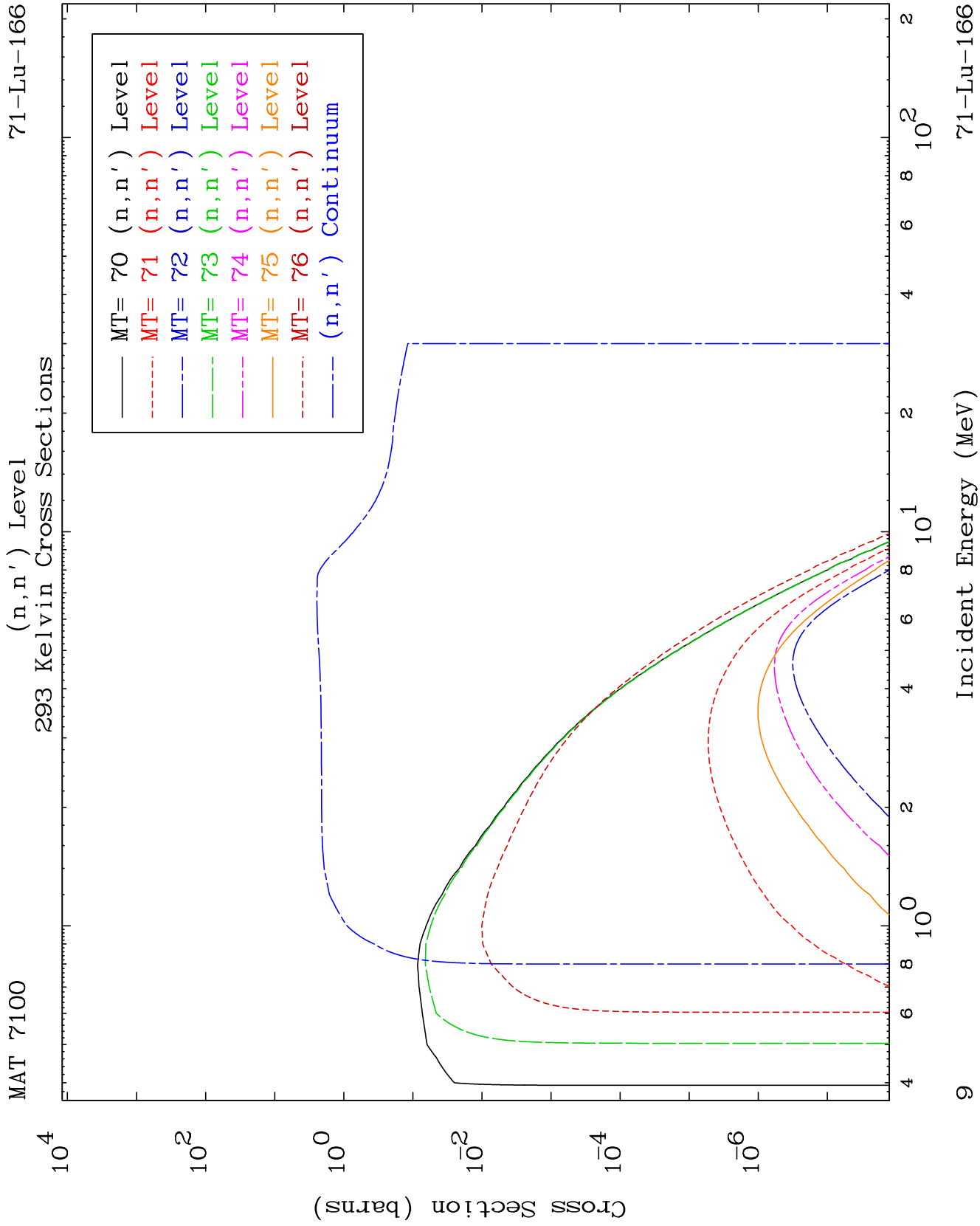
MAT 7100

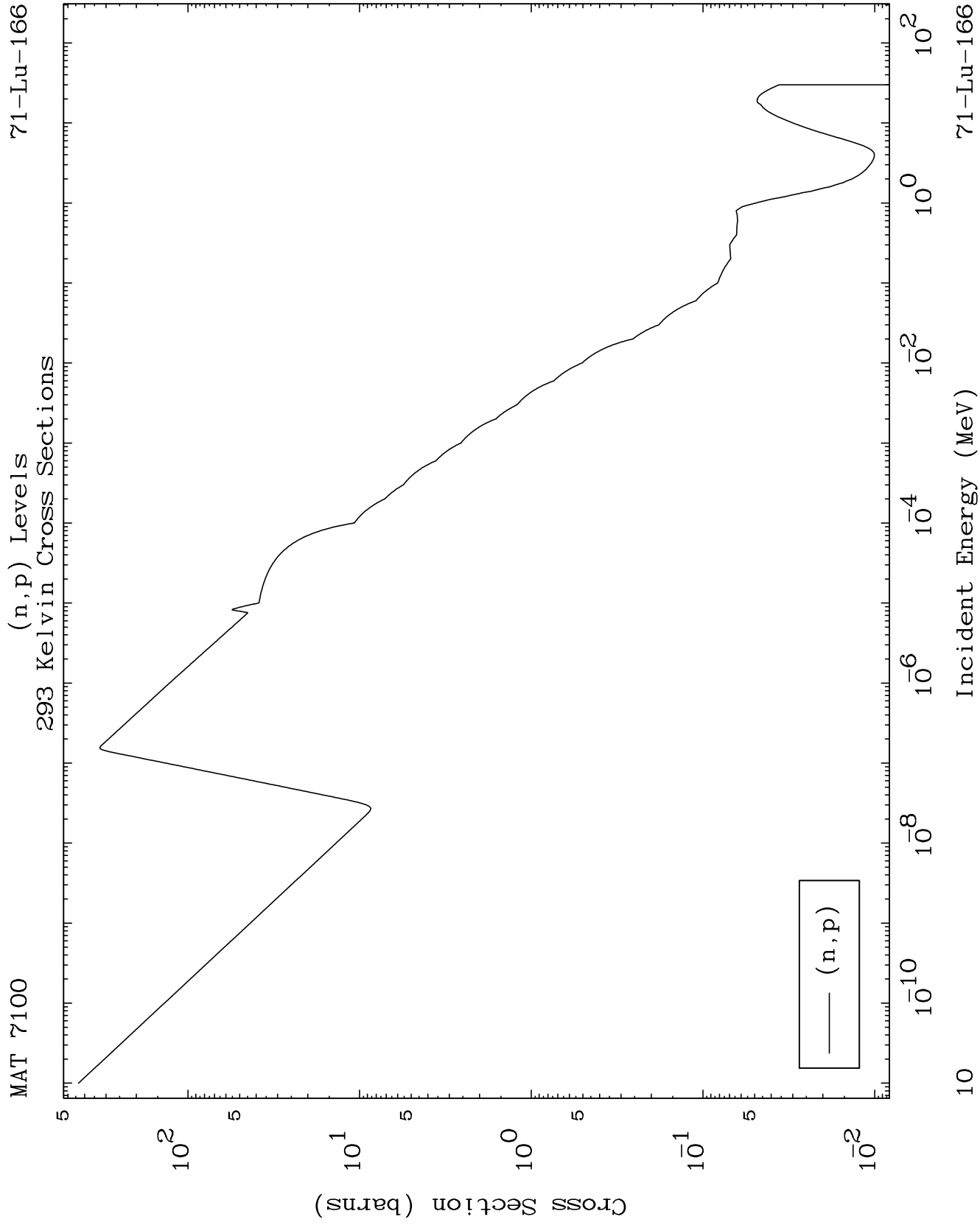
(n,n') Level

71-Lu-166

293 Kelvin Cross Sections



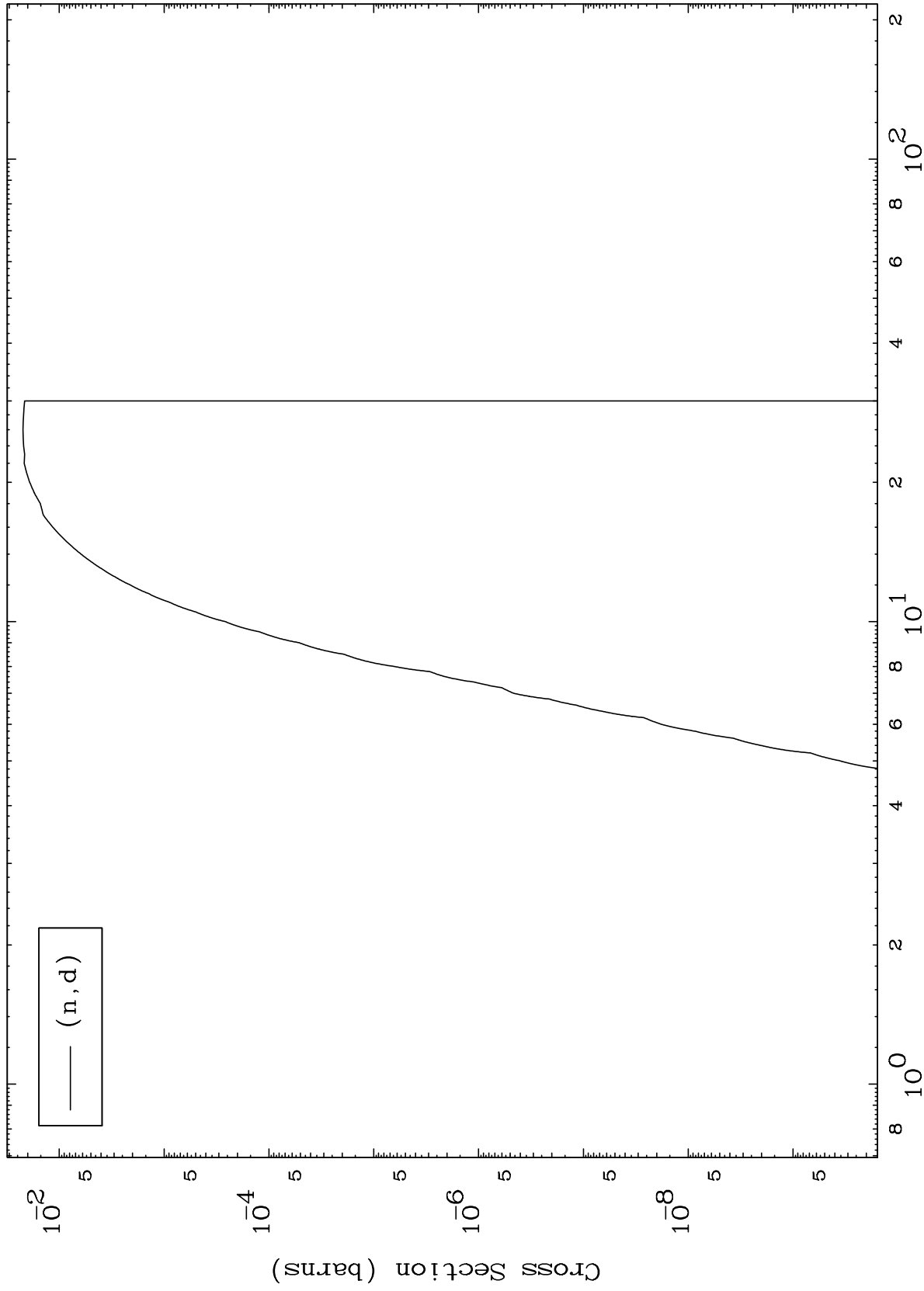




MAT 7100

(n,d) Levels  
293 Kelvin Cross Sections

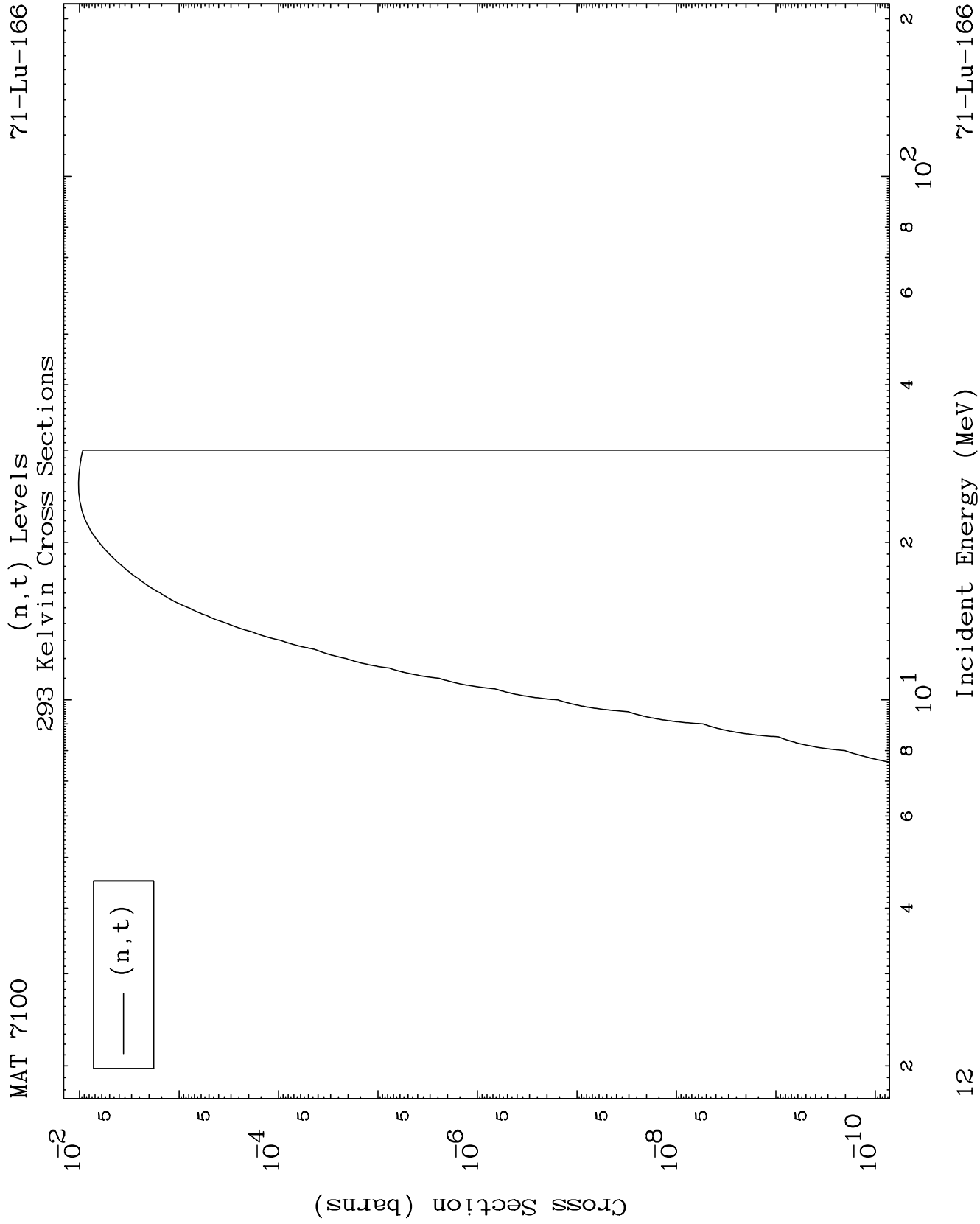
71-Lu-166



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Incident Energy (MeV)

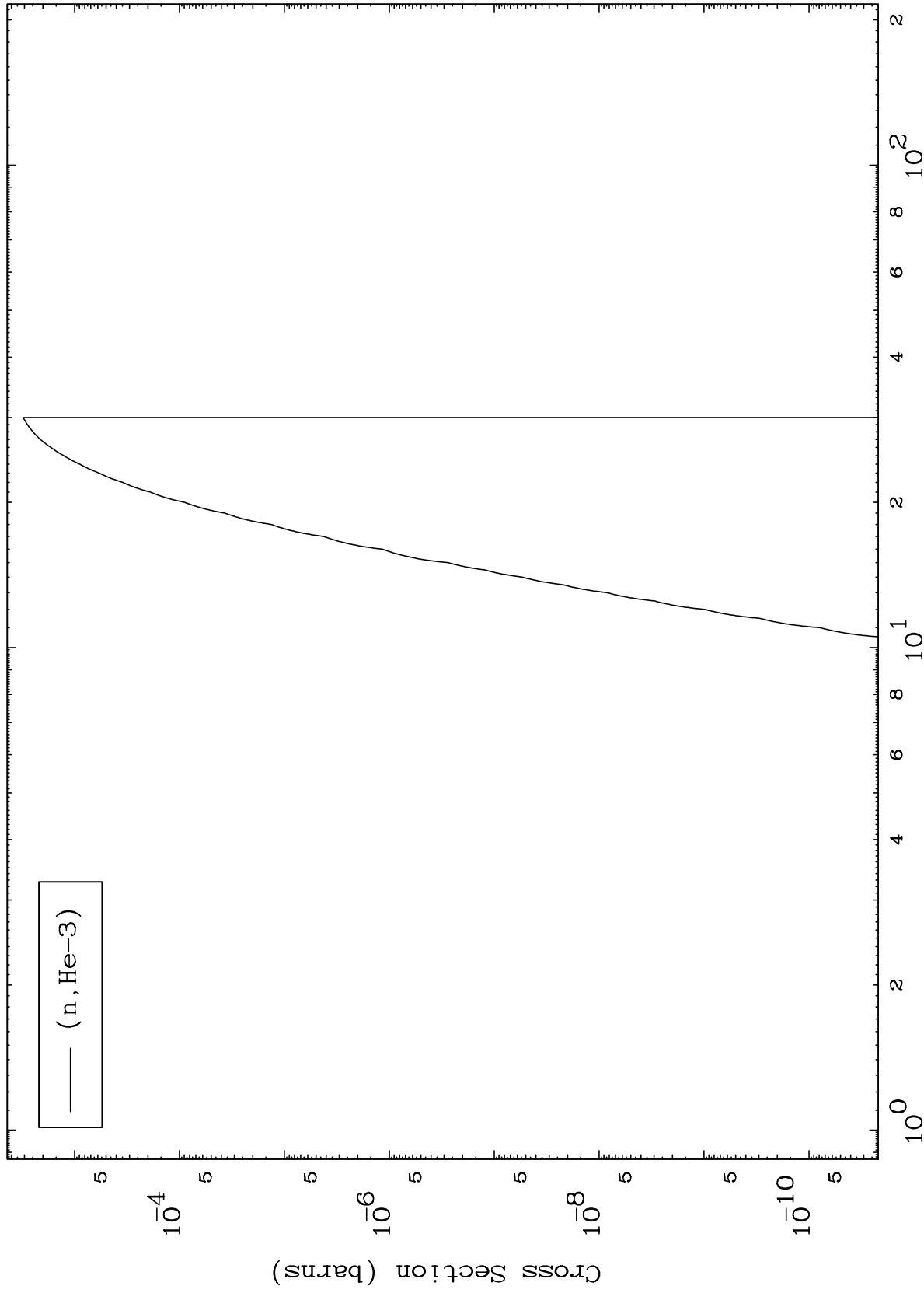
71-Lu-166



MAT 7100

(n,He3) Levels  
293 Kelvin Cross Sections

71-Lu-166



13

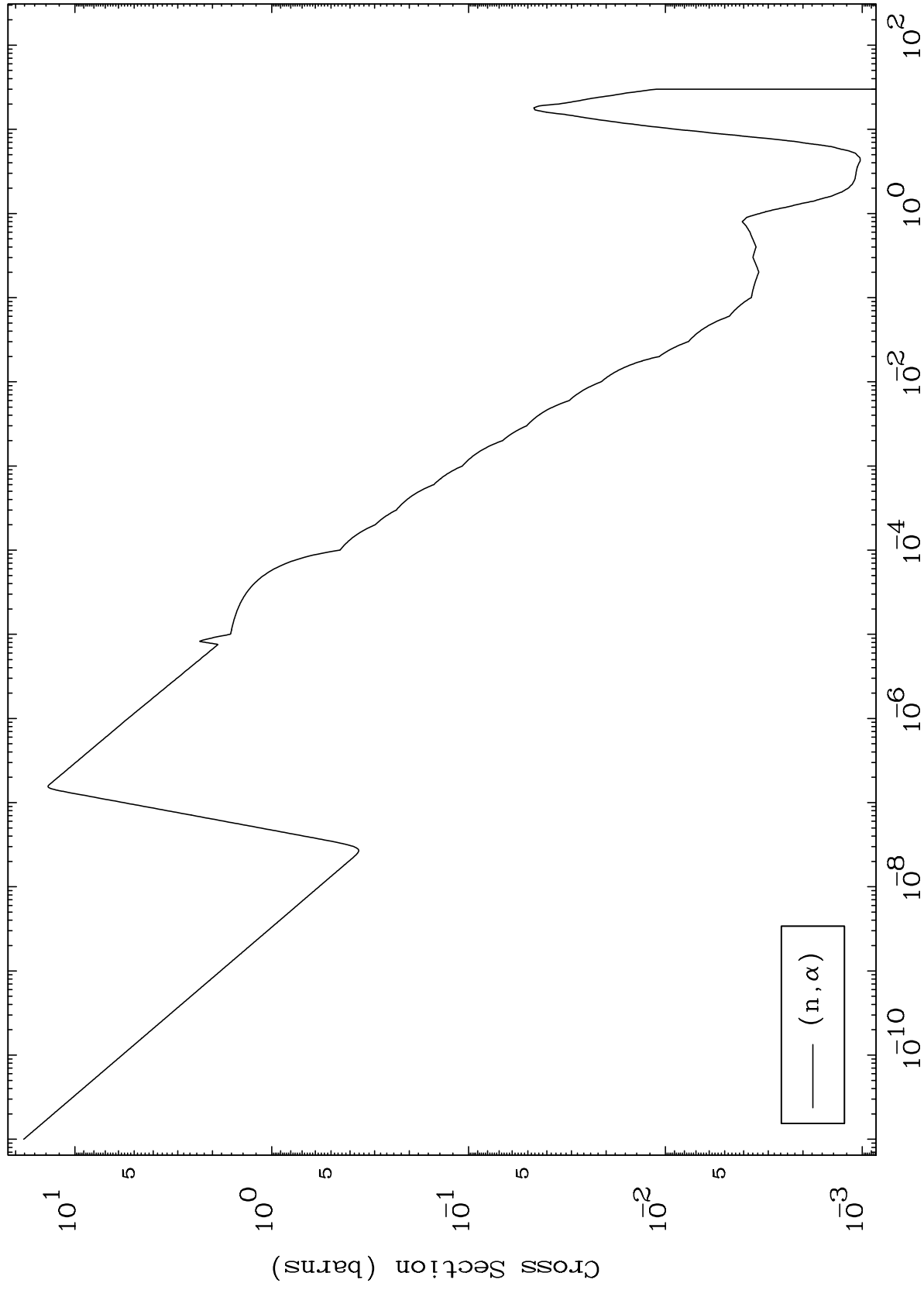
Incident Energy (MeV)

71-Lu-166

MAT 7100

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

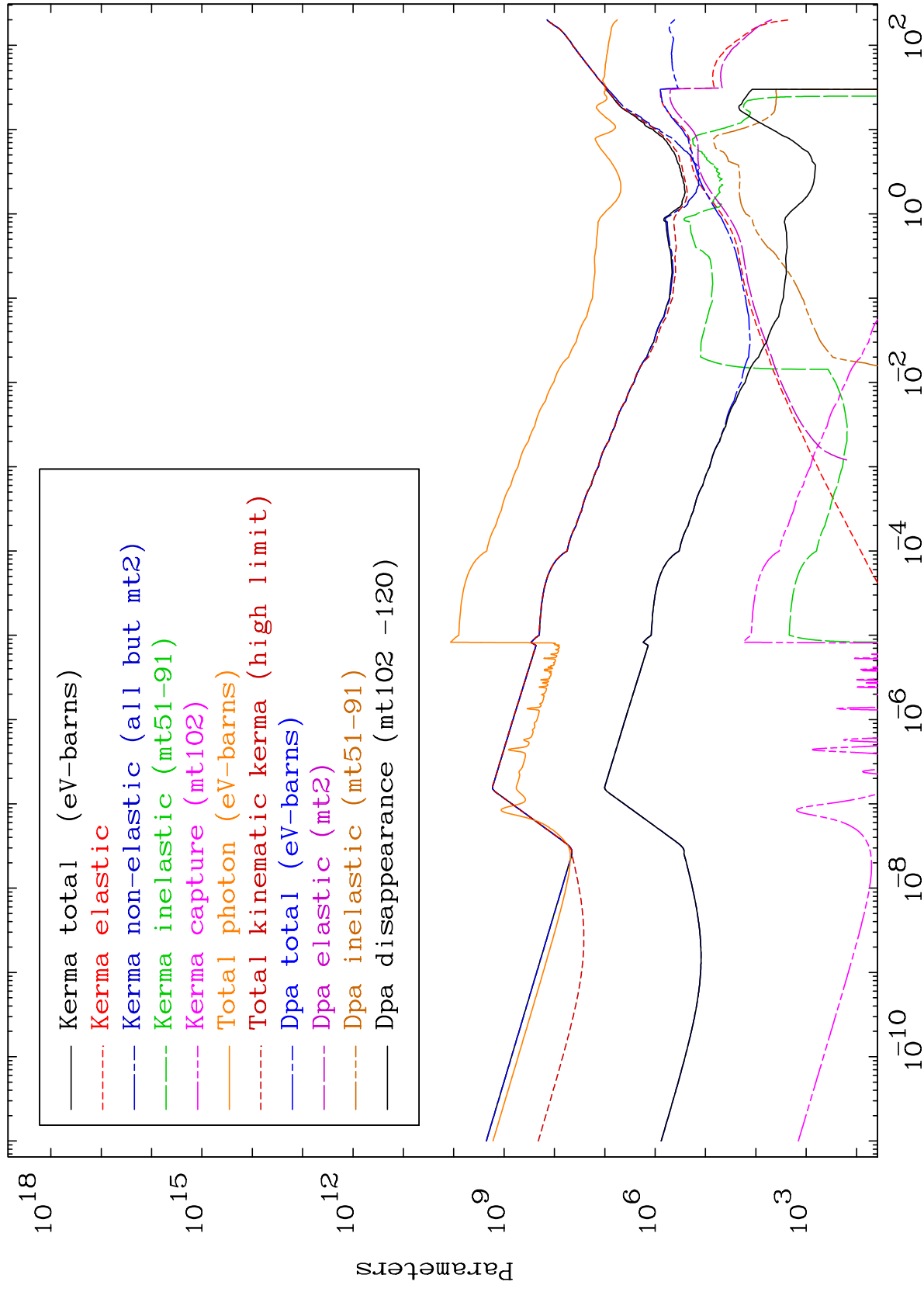
71-Lu-166



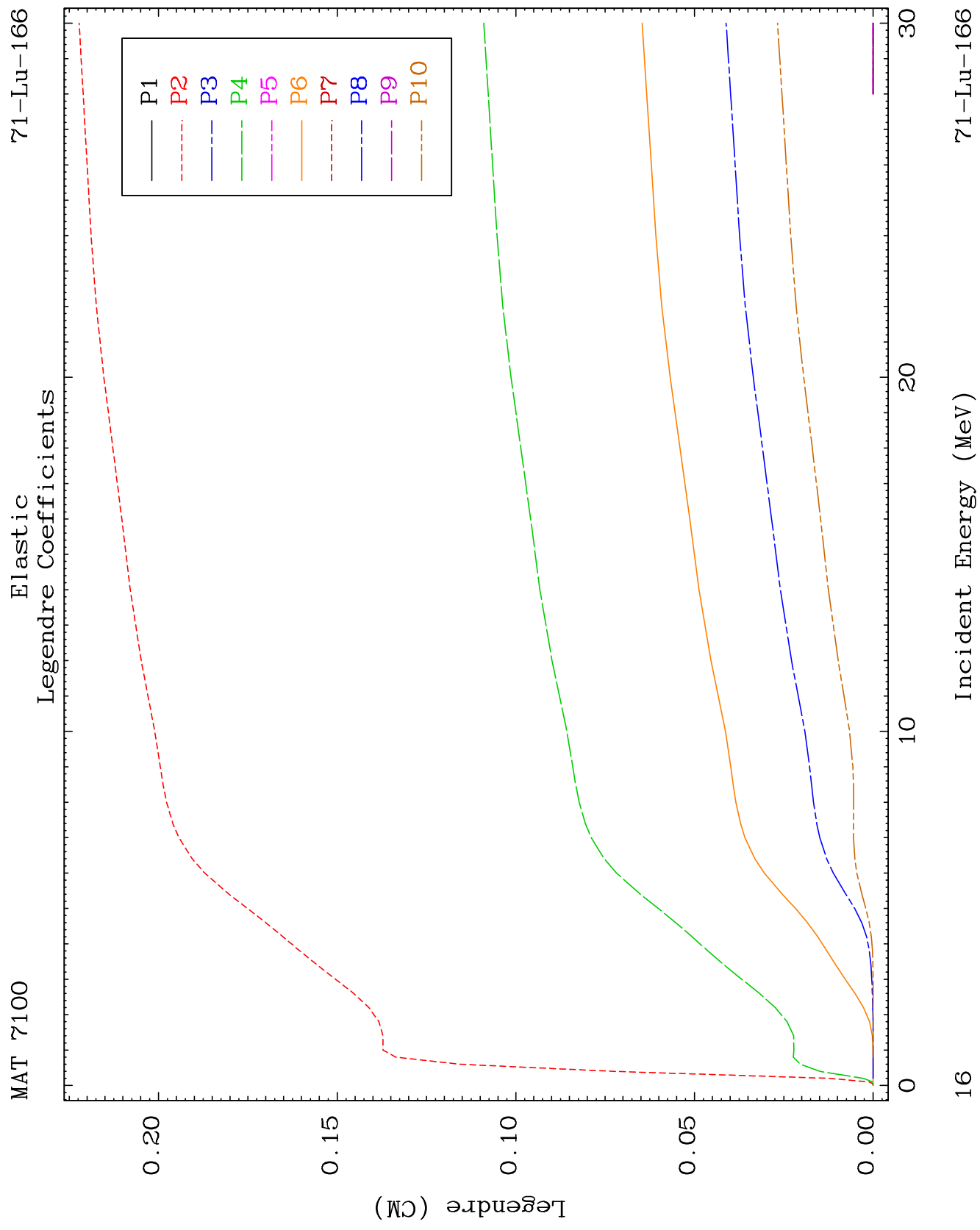
14

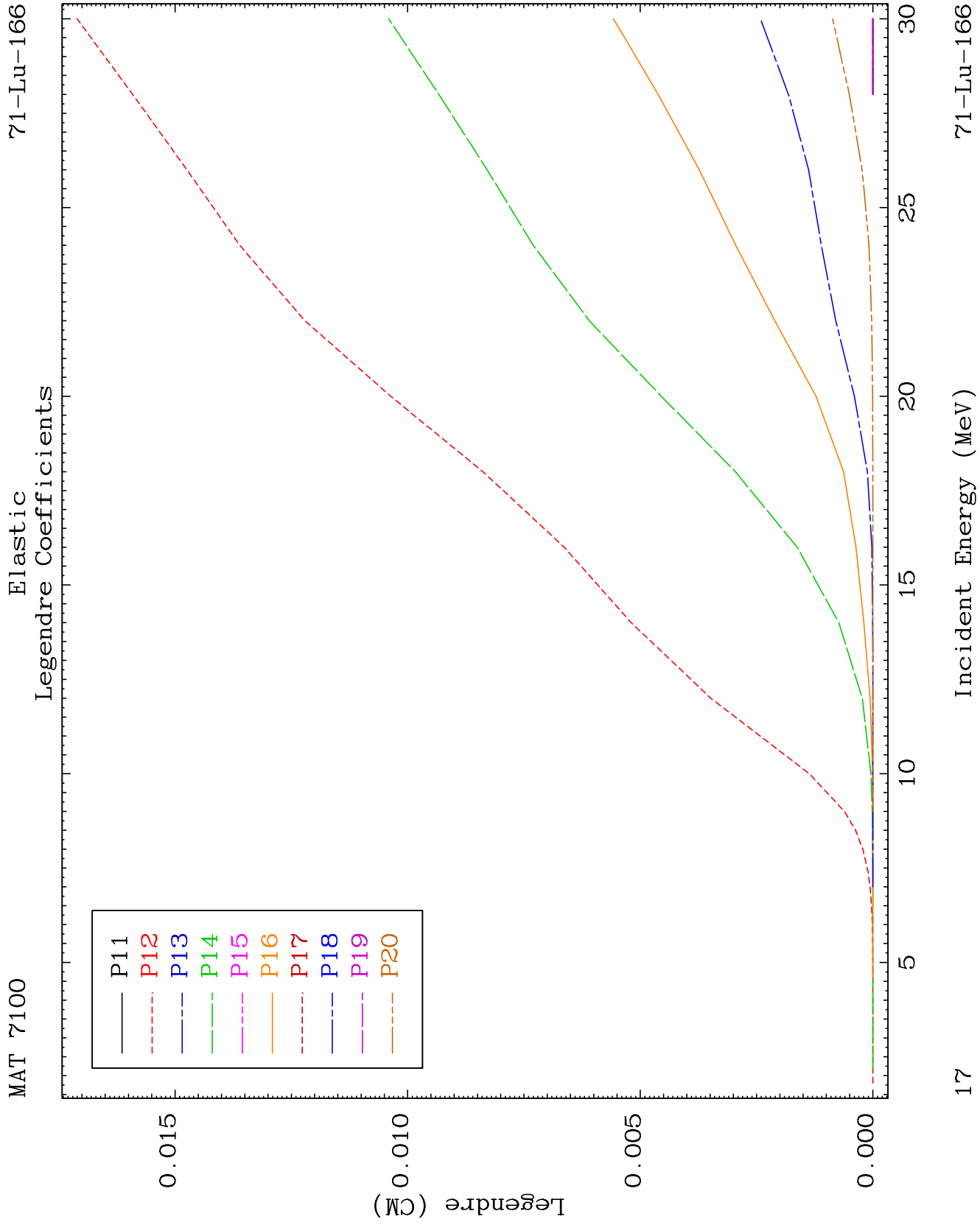
Incident Energy (MeV)

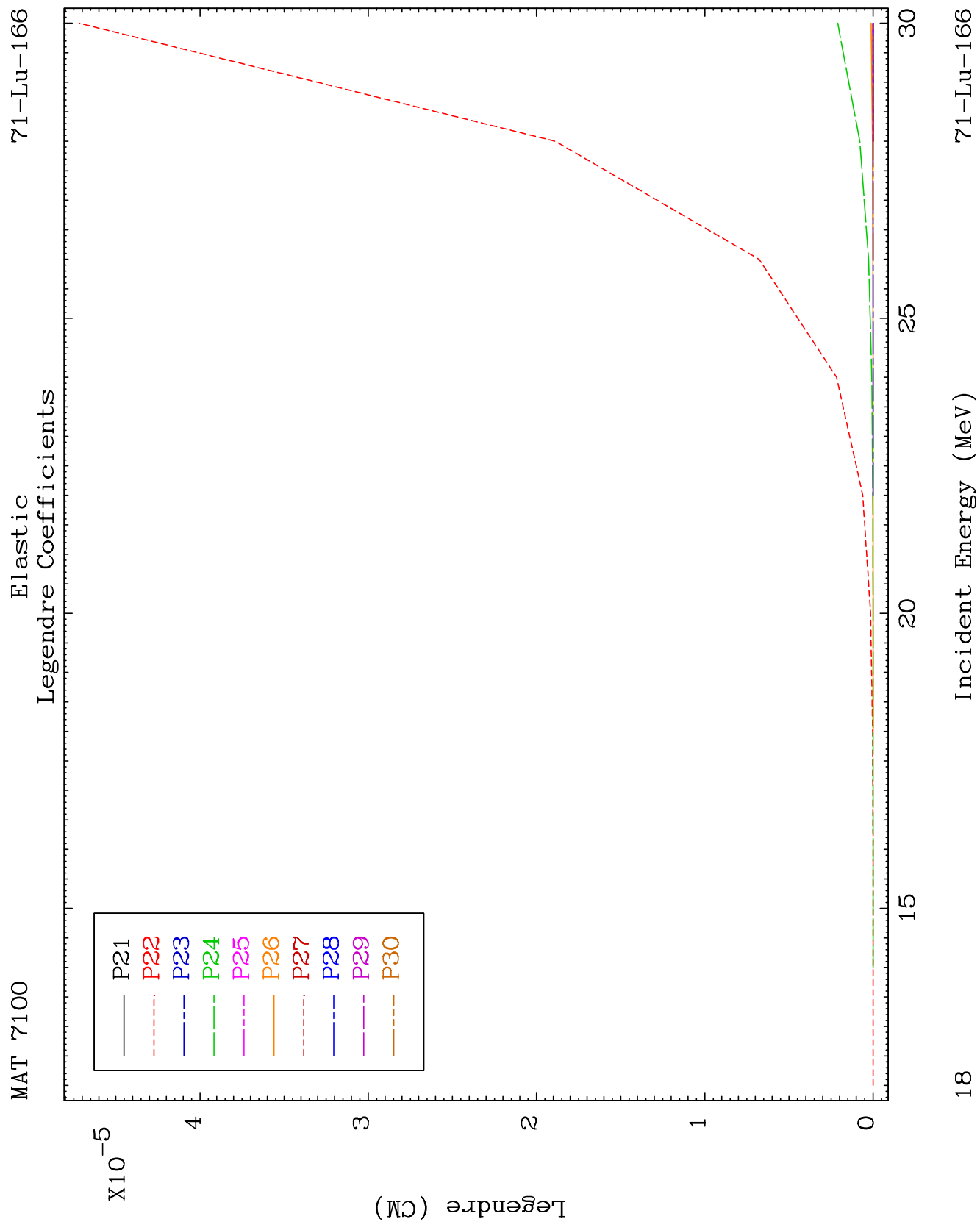
71-Lu-166

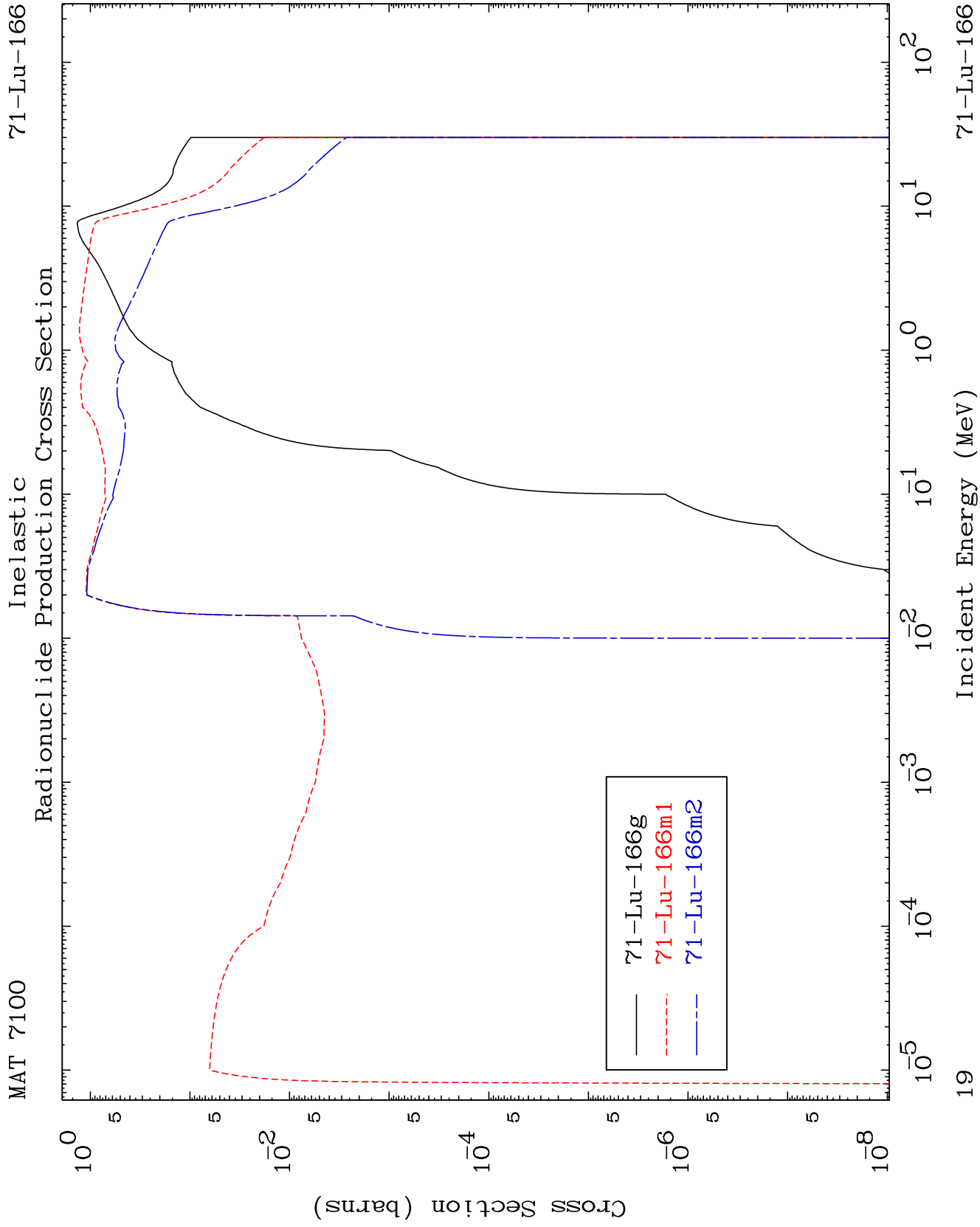










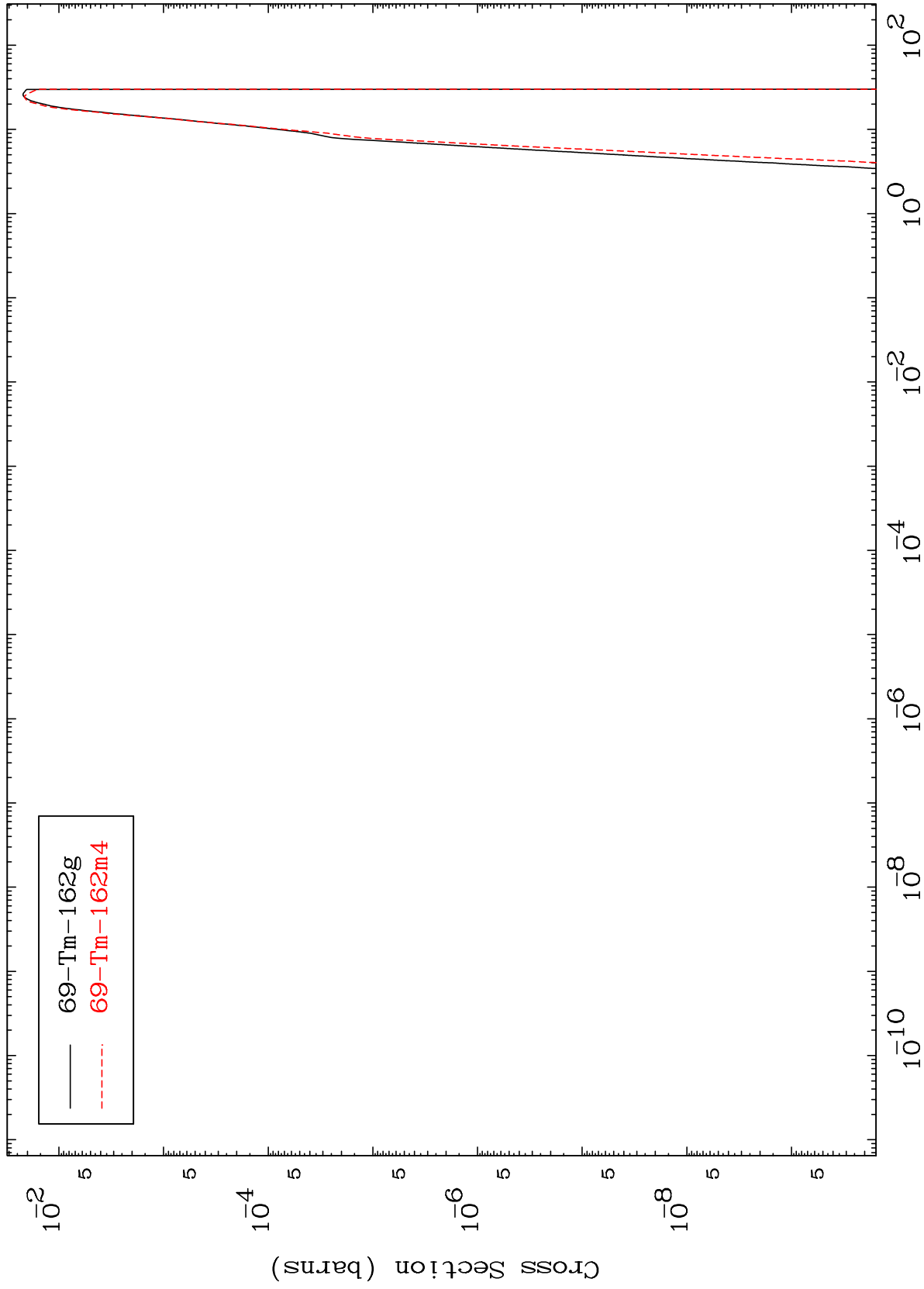


MAT 7100

$(n,n') \alpha$

$^{71}\text{Lu-166}$

Radionuclide Production Cross Section



20

Incident Energy (MeV)

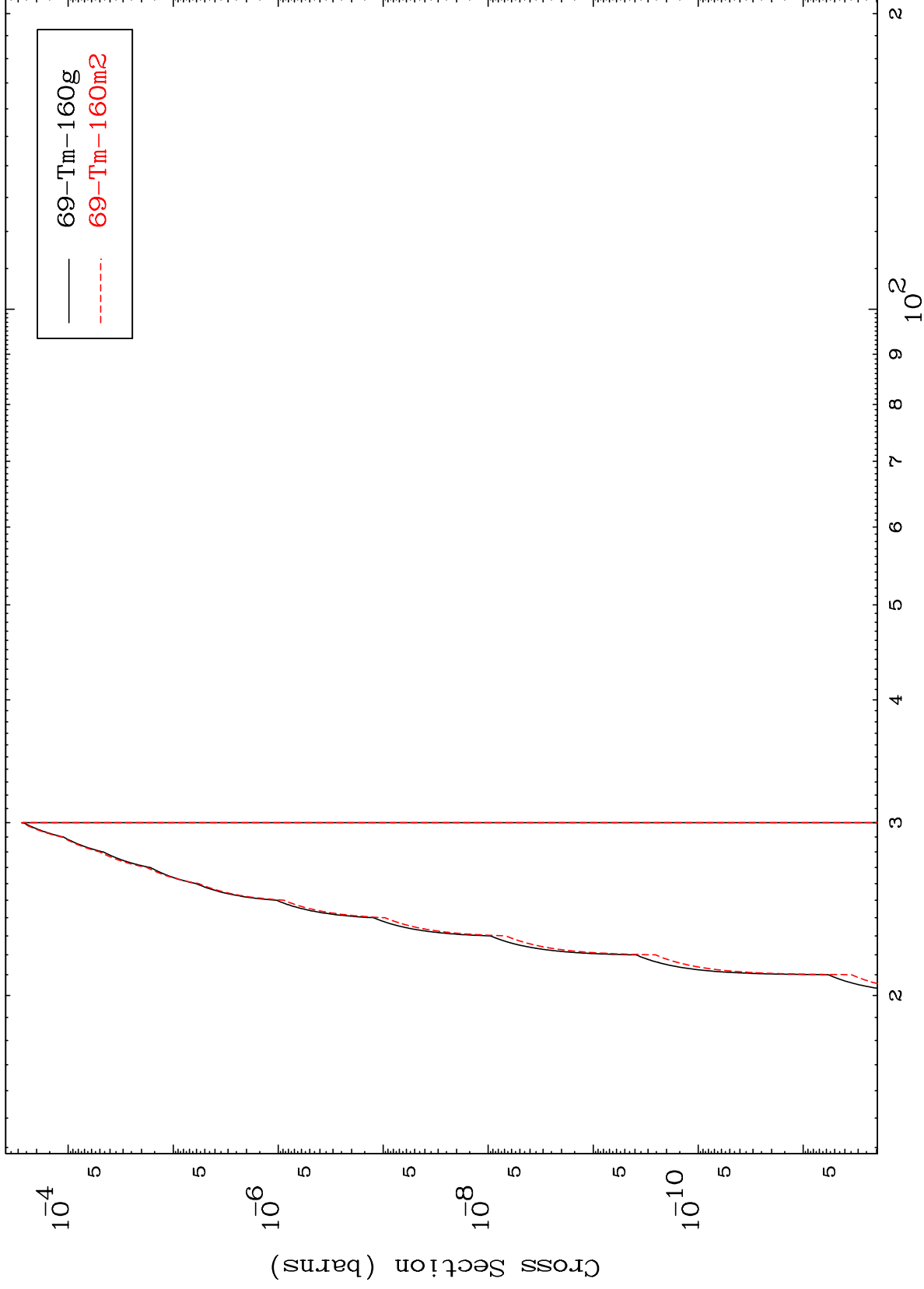
$^{71}\text{Lu-166}$

MAT 7100

$(n,3n) \alpha$

$^{71}\text{Lu-166}$

Radionuclide Production Cross Section



21

Incident Energy (MeV)

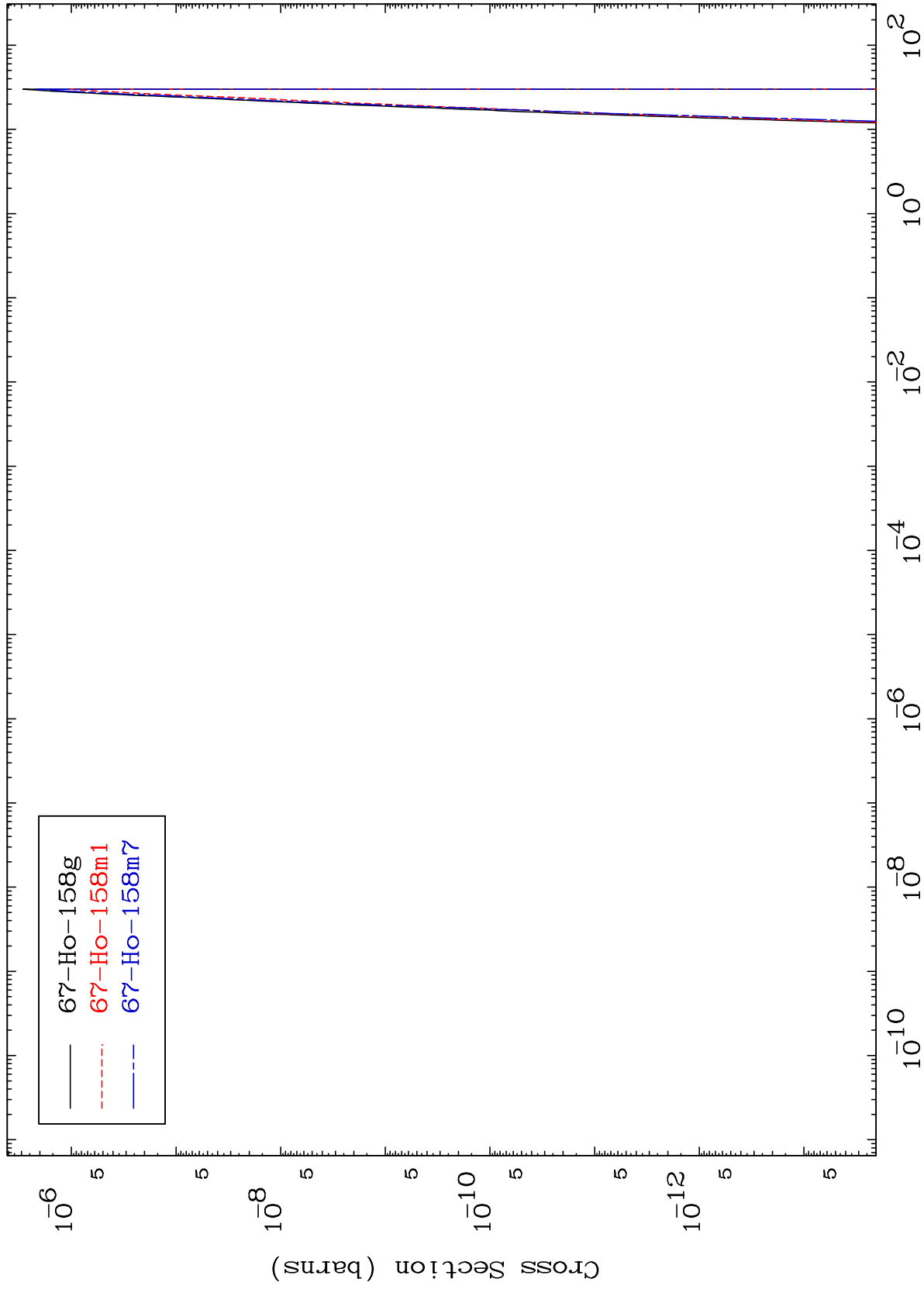
$^{71}\text{Lu-166}$

MAT 7100

(n,n') 2 $\alpha$

<sup>71</sup>Lu-166

Radionuclide Production Cross Section



22

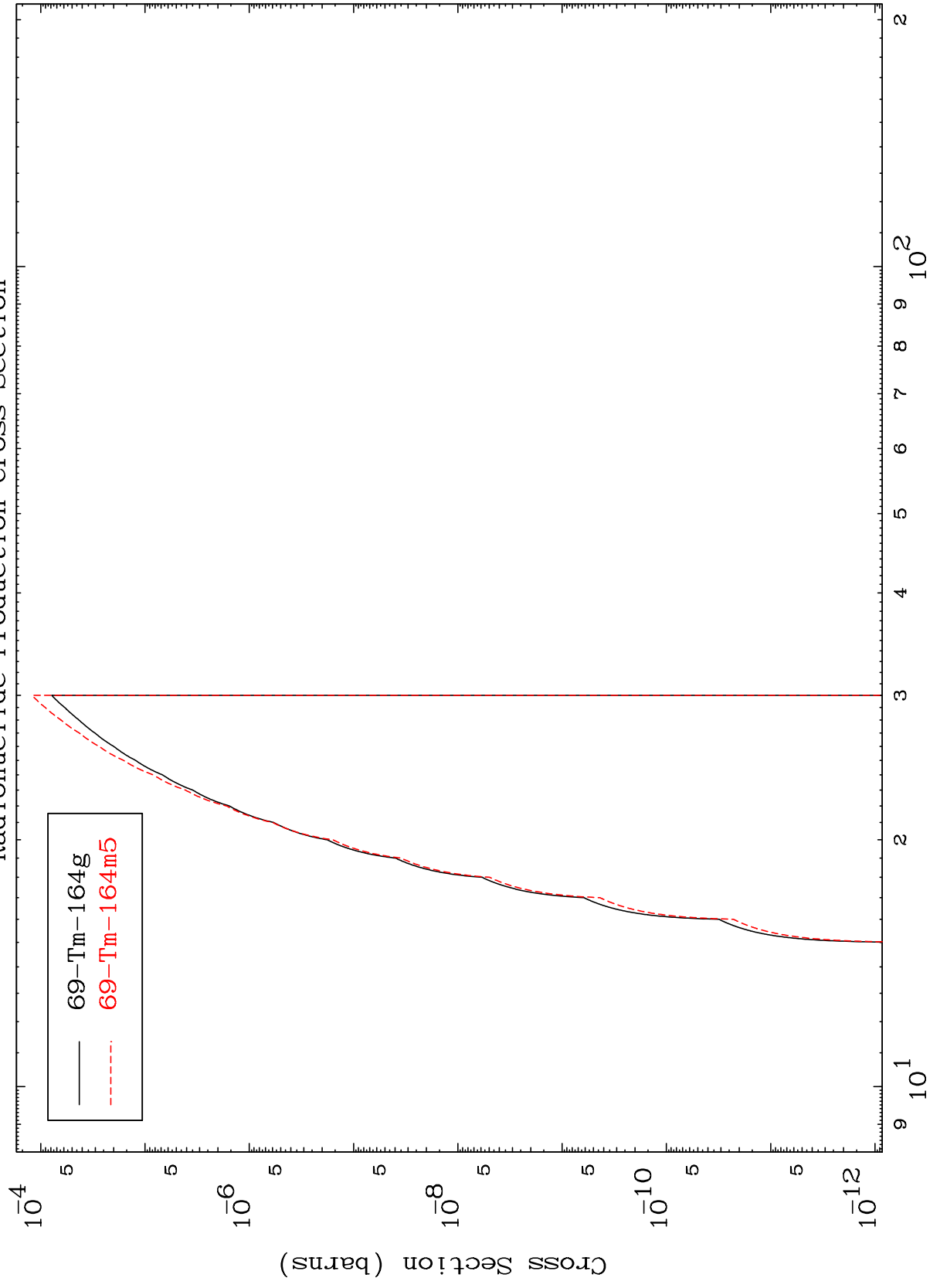
Incident Energy (MeV)

<sup>71</sup>Lu-166

71-Lu-166

MAT 7100

(n,2n) p  
Radionuclide Production Cross Section



71-Lu-166

23

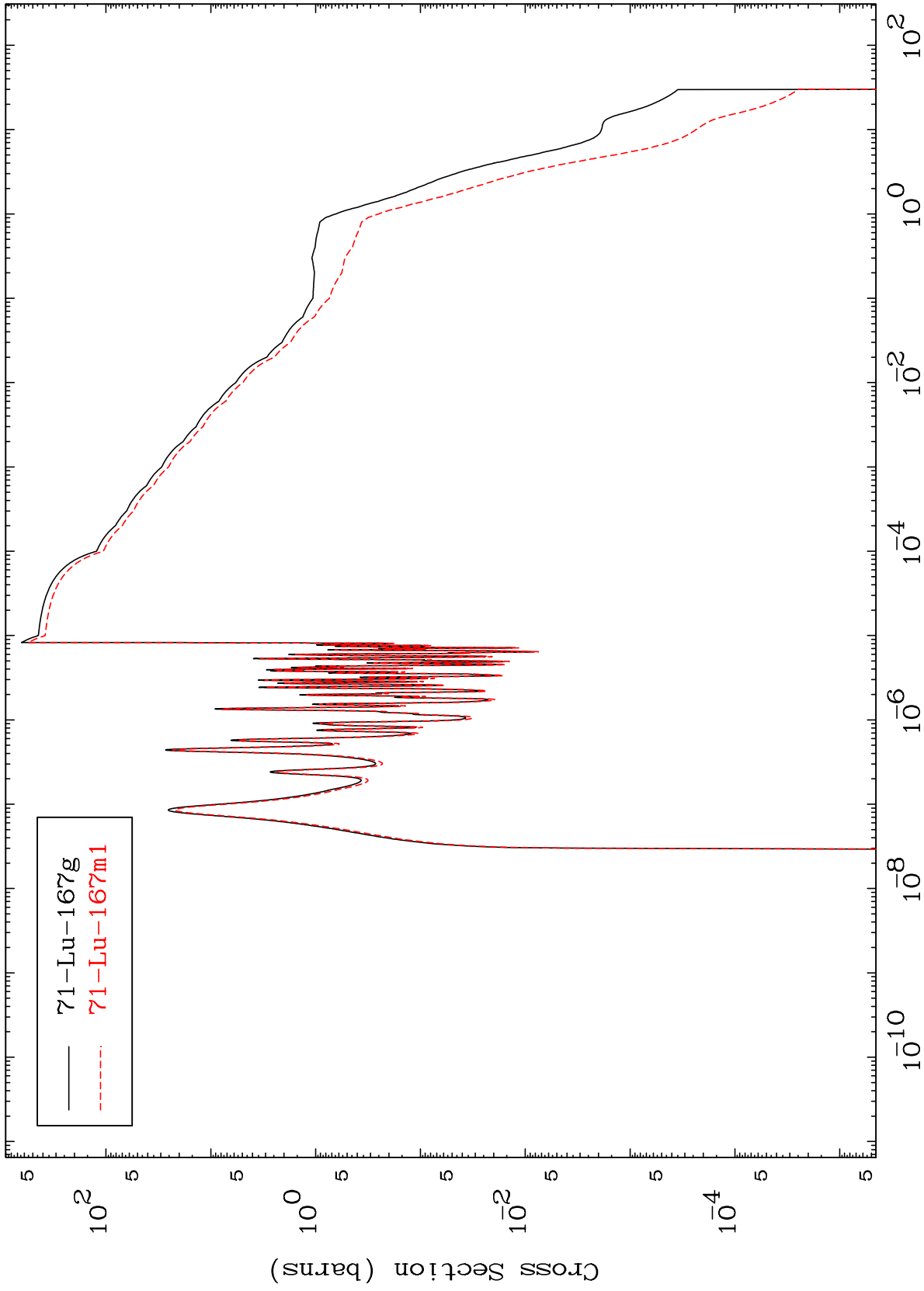
Incident Energy (MeV)



MAT 7100

71-Lu-166

Radionuclide Production Cross Section



24

Incident Energy (MeV)

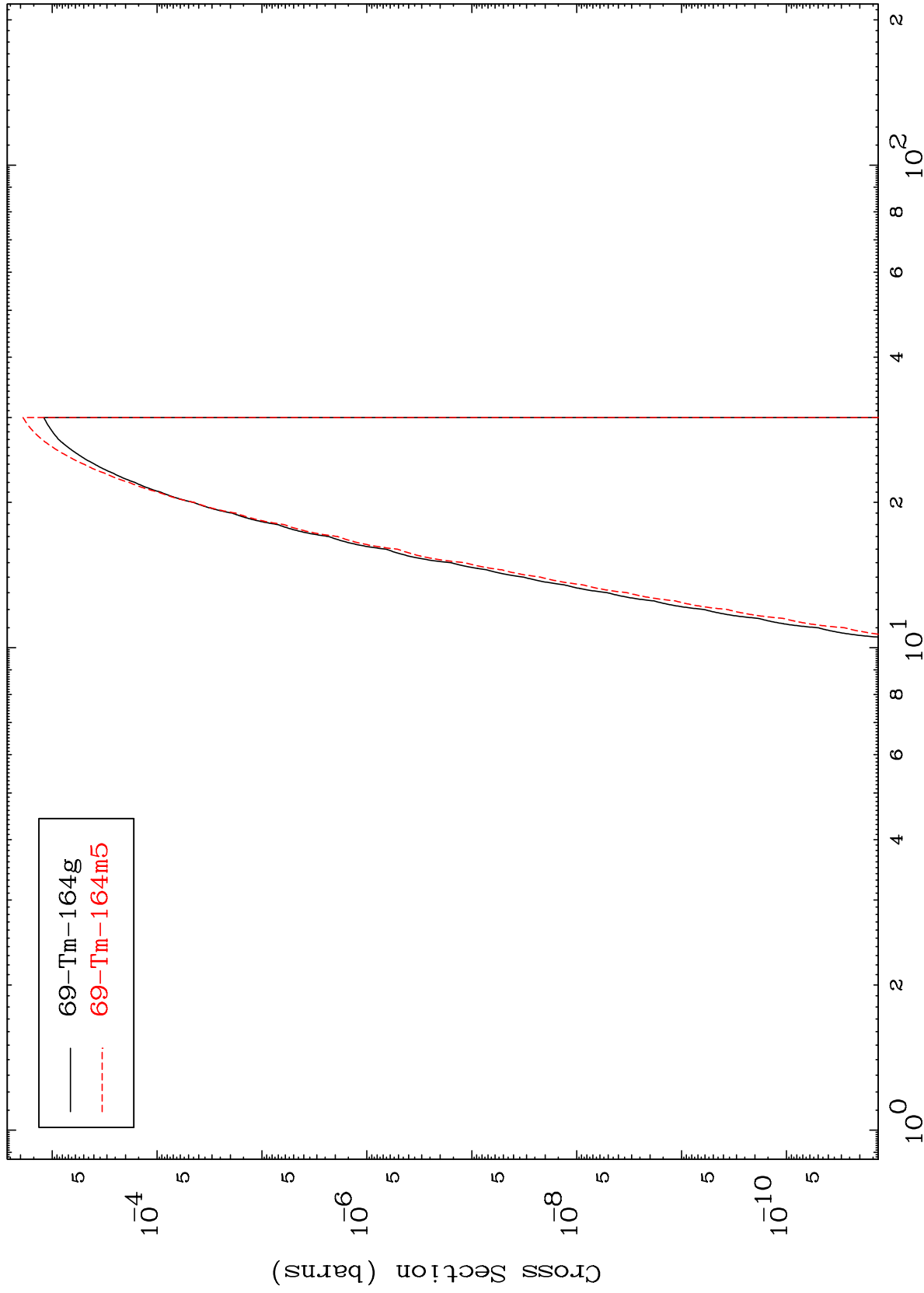
71-Lu-166

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(n,He-3)

71-Lu-166

Radionuclide Production Cross Section



25

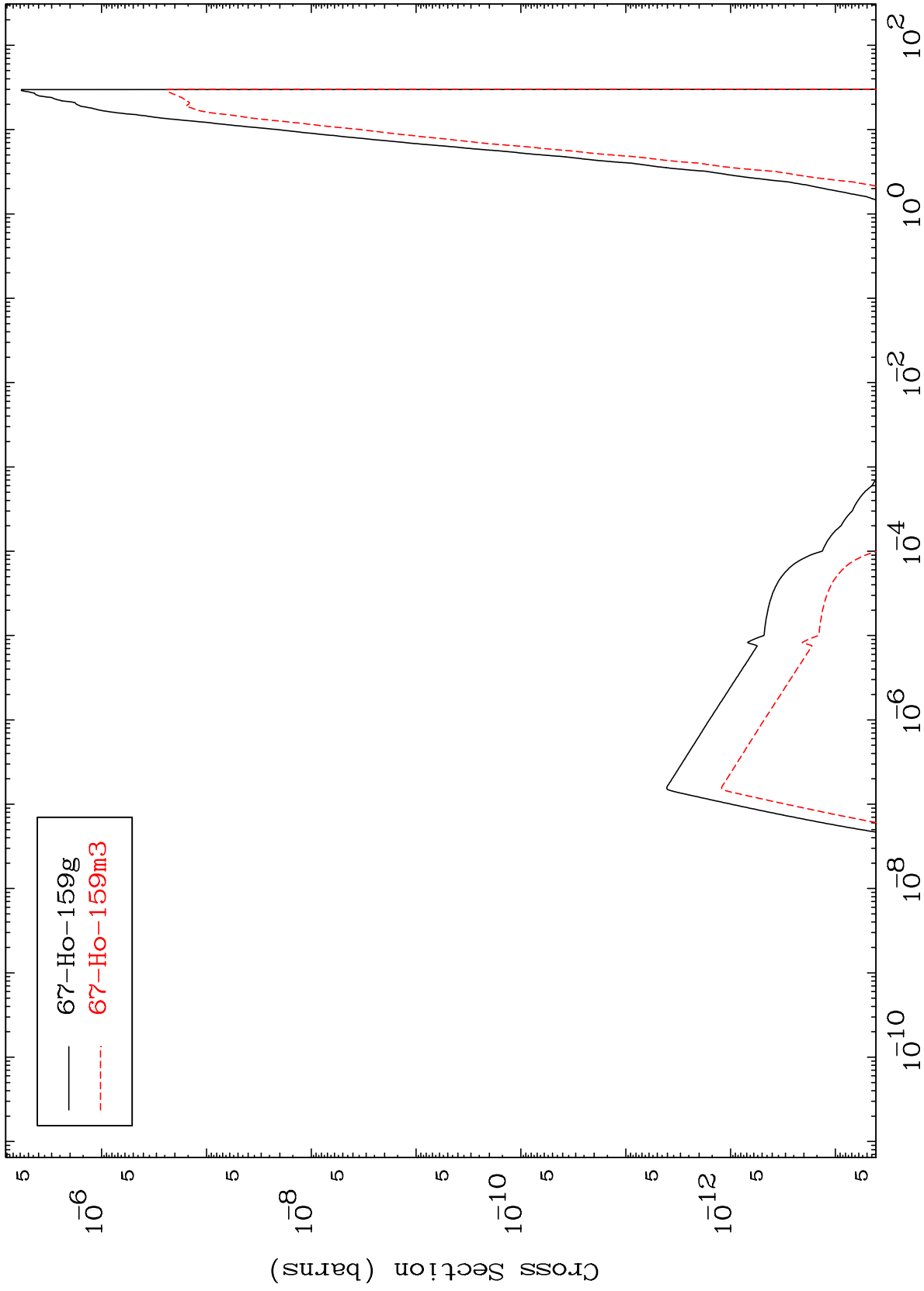
Incident Energy (MeV)

71-Lu-166

MAT 7100

Radionuclide Production Cross Section  
(n,2α)

<sup>71</sup>Lu-166



26

Incident Energy (MeV)

<sup>71</sup>Lu-166

