

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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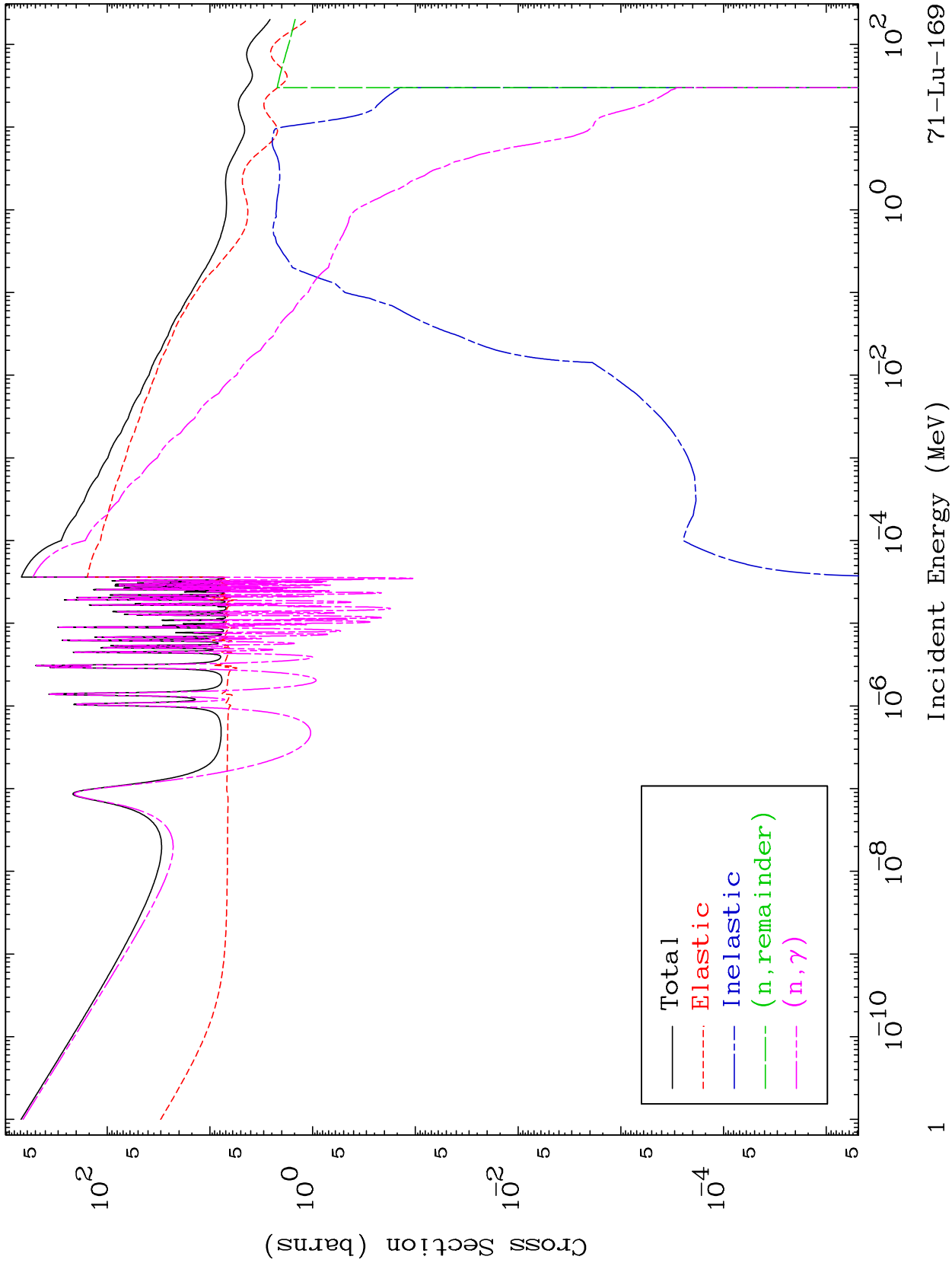
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7108

Major  
293 Kelvin Cross Sections

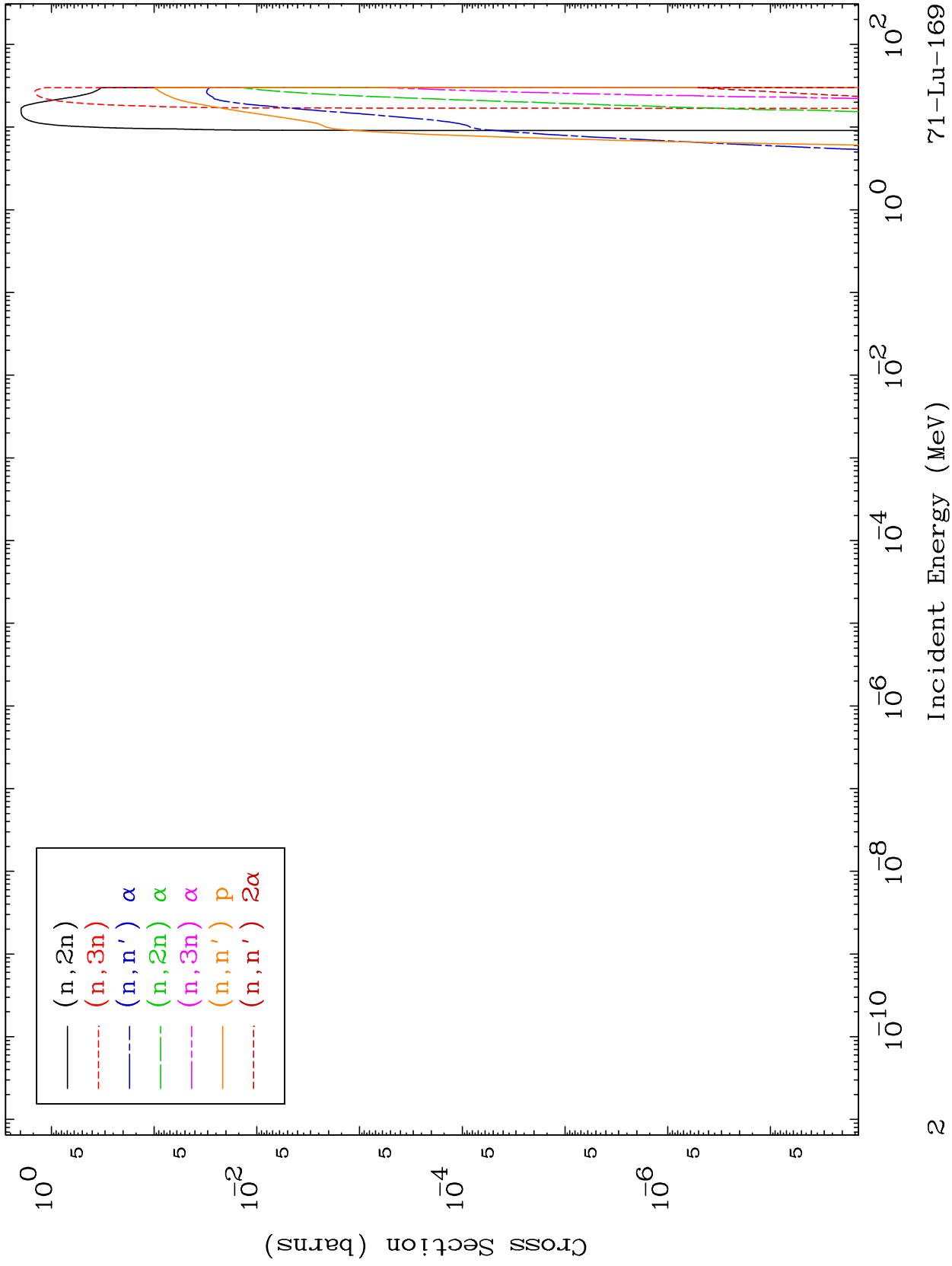
71-Lu-169

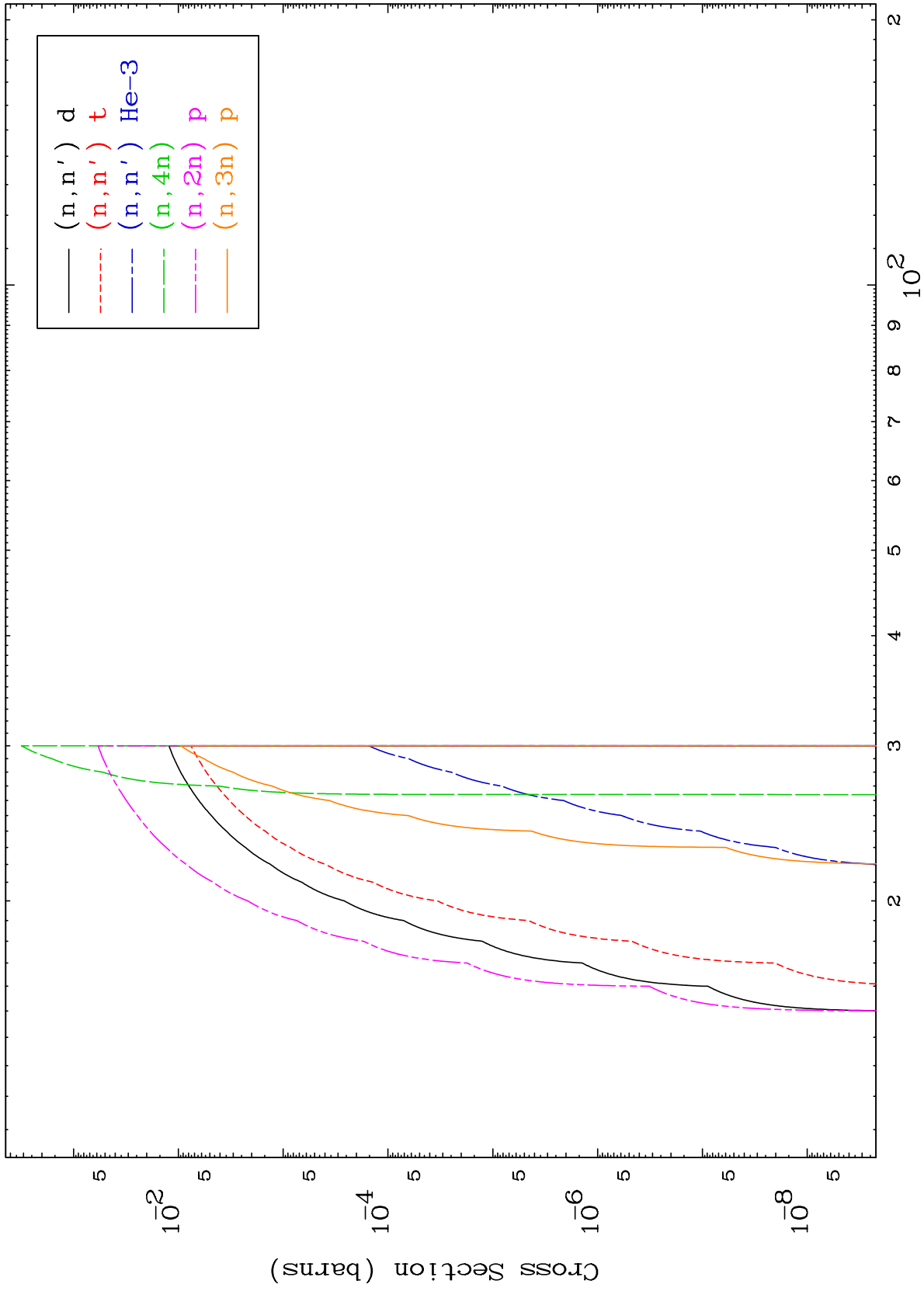


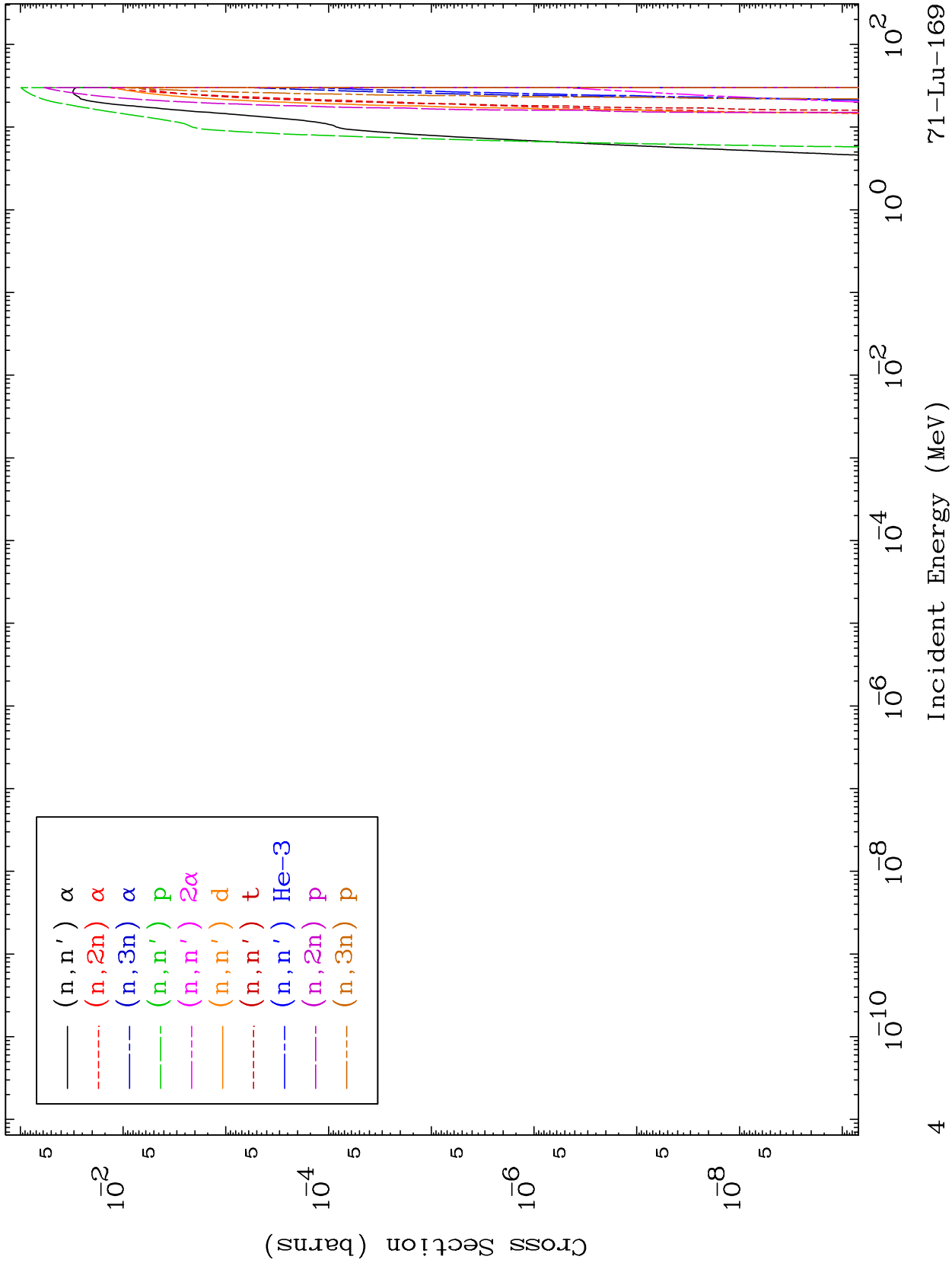
MAT 7108

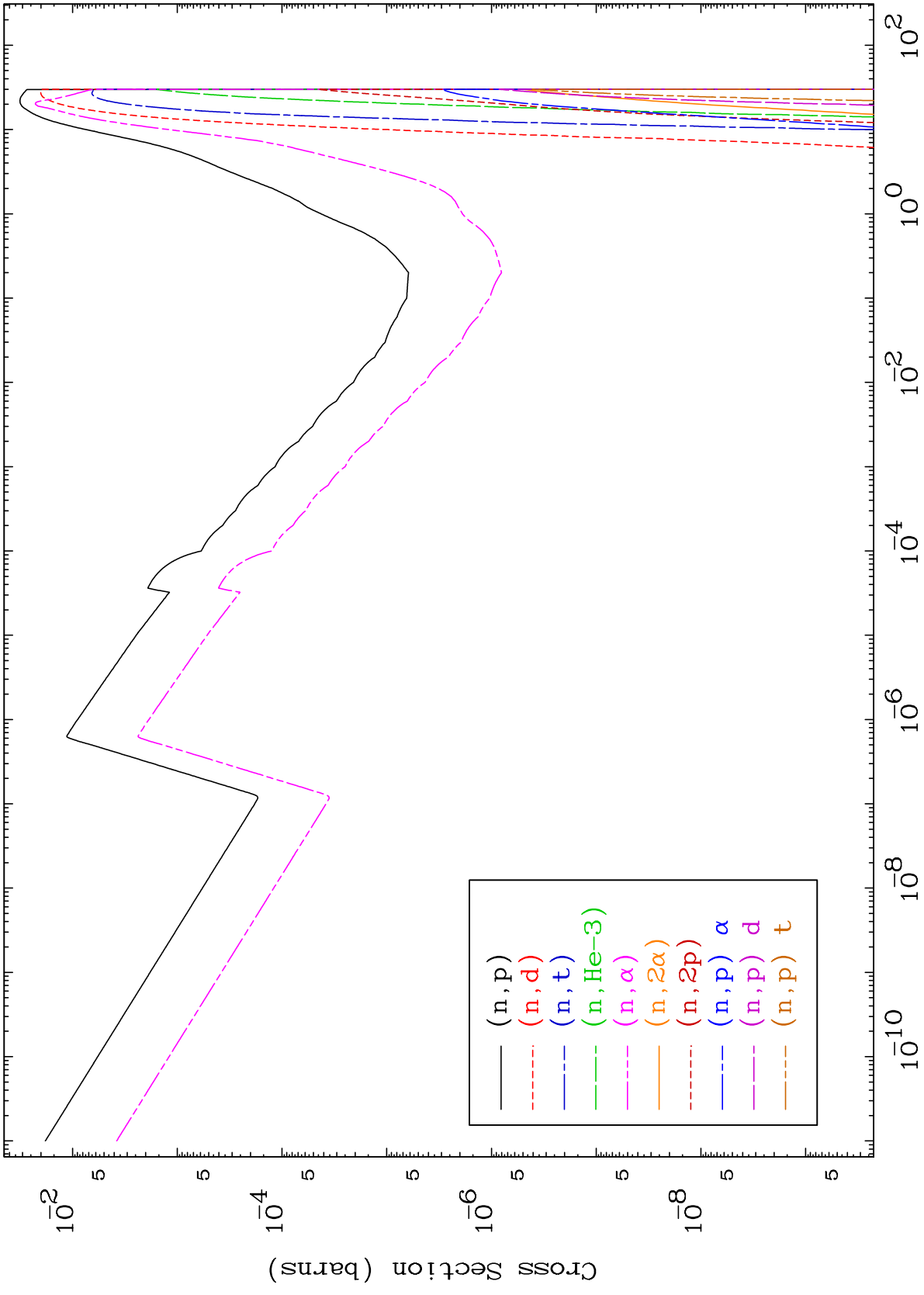
Neutron Production  
293 Kelvin Cross Sections

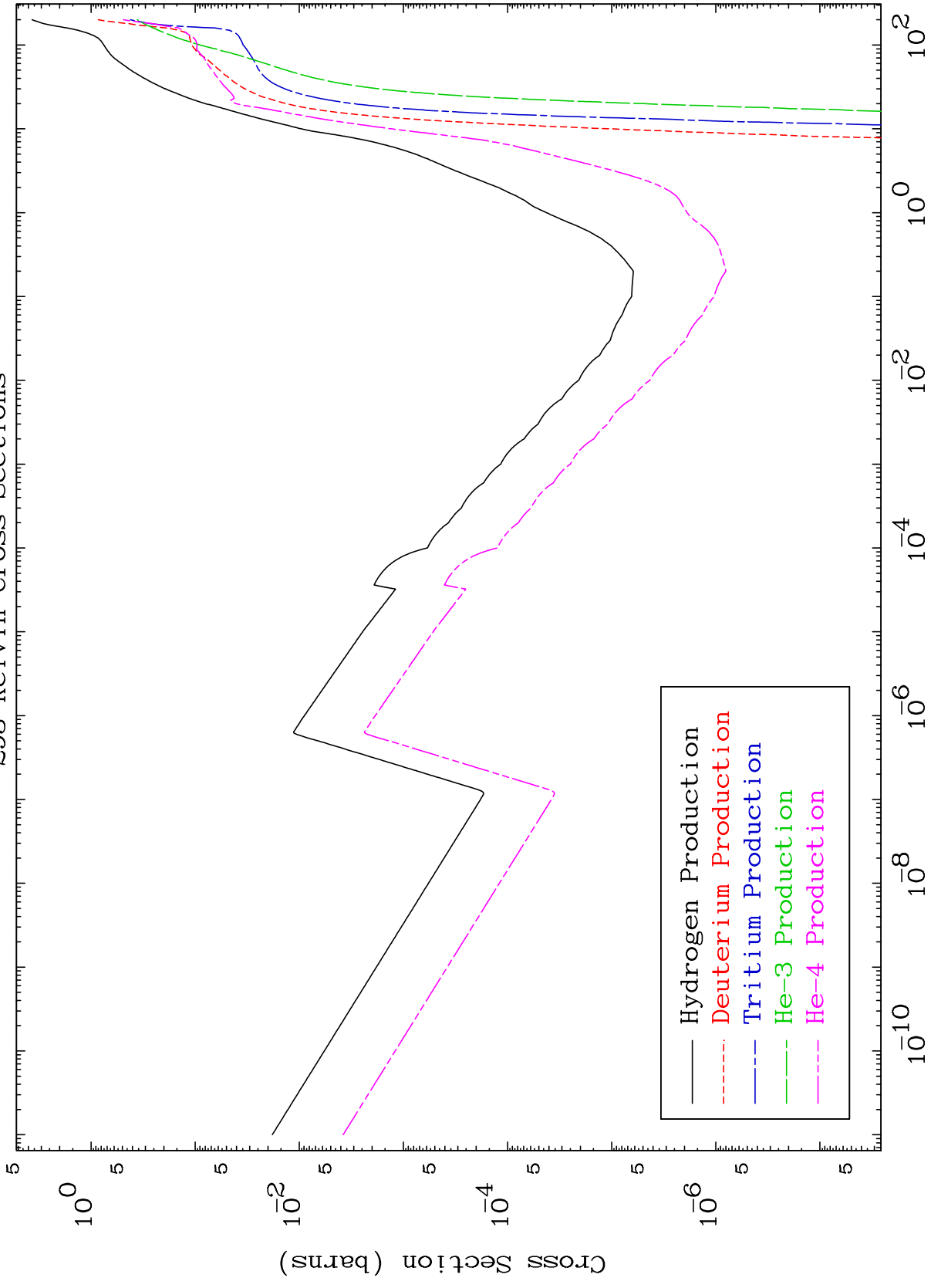
71-Lu-169







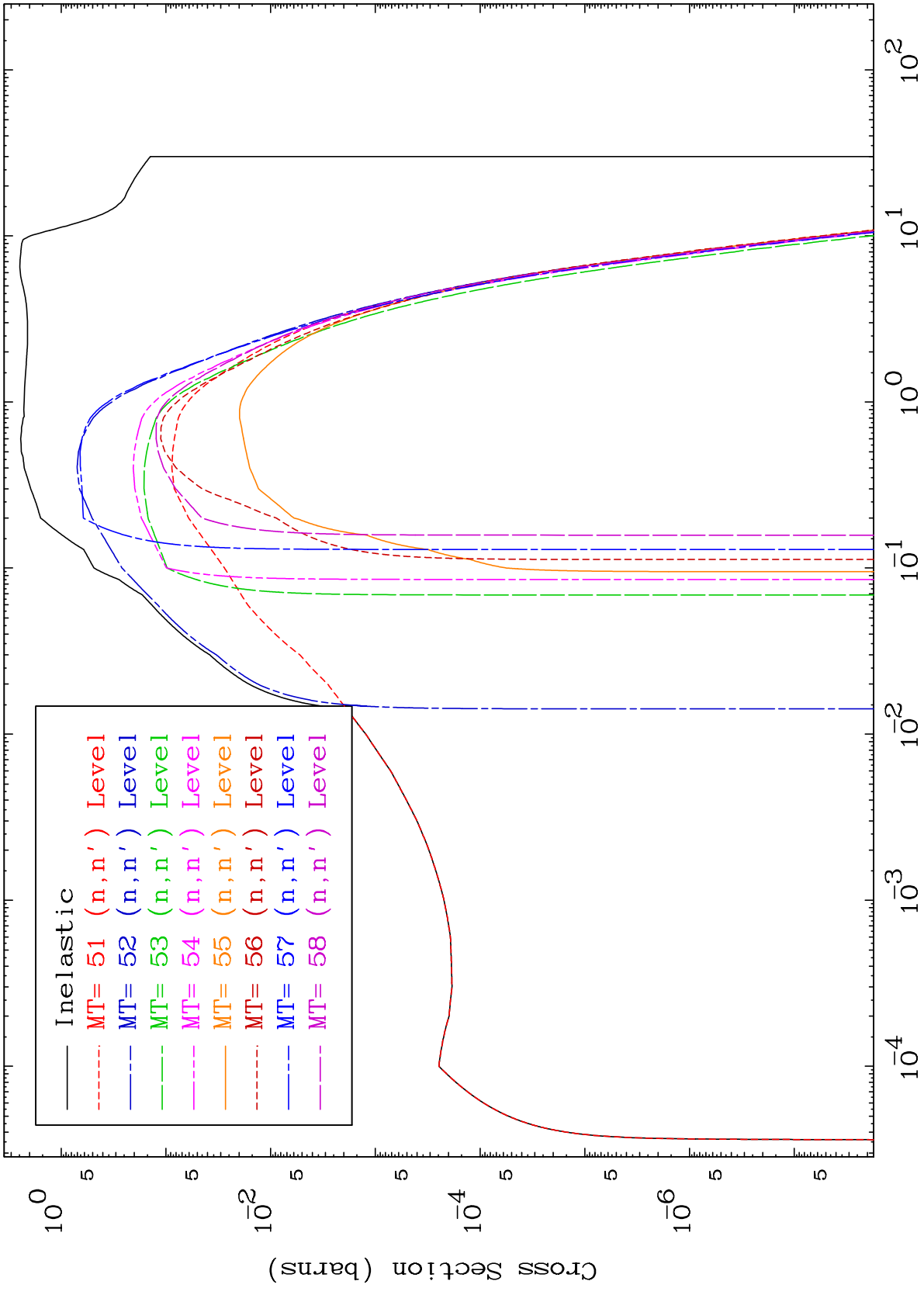




MAT 7108

293 Kelvin Cross Sections  
(n,n') Level

71-Lu-169



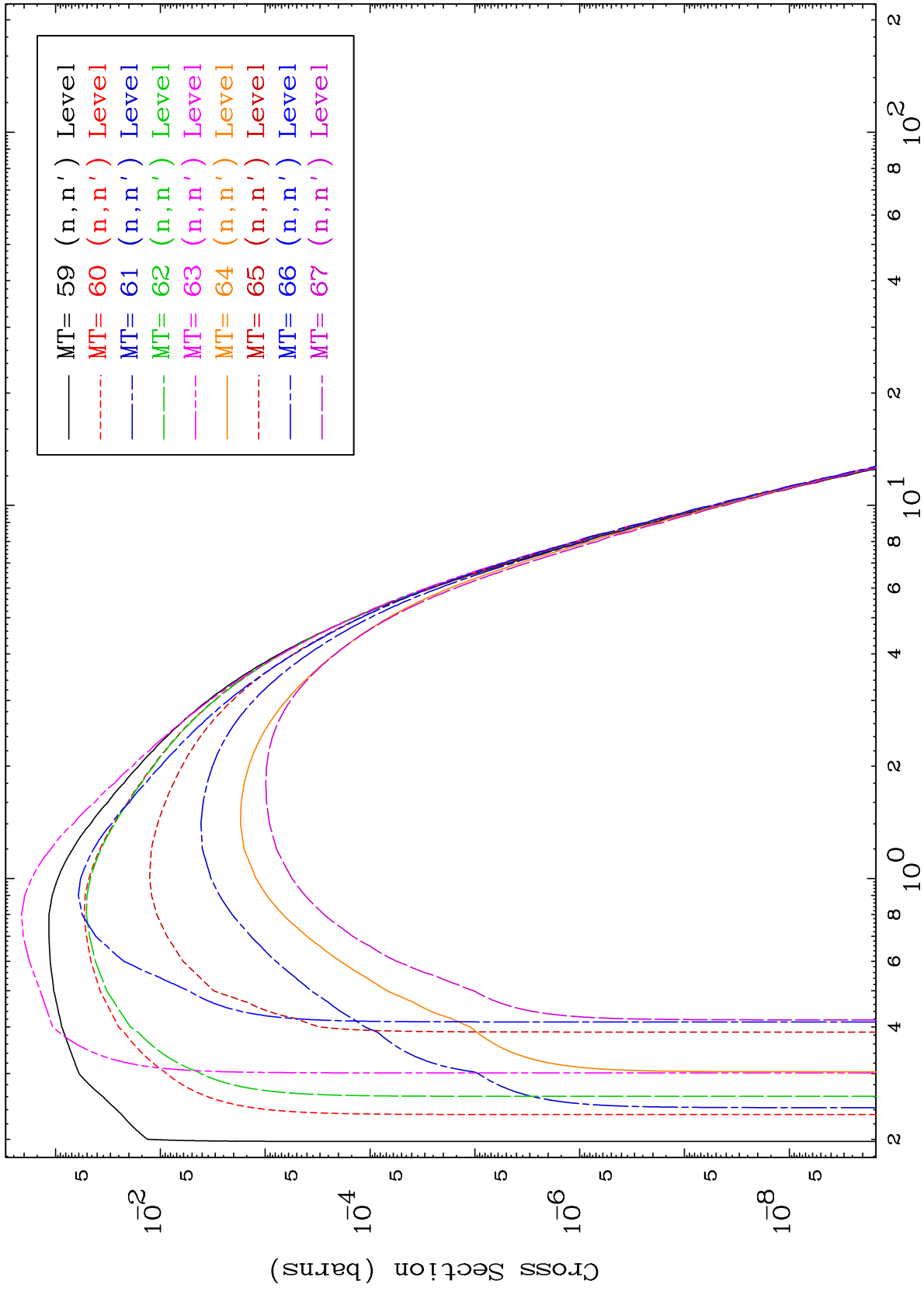
71-Lu-169



MAT 7108

(n,n') Level  
293 Kelvin Cross Sections

71-Lu-169



8

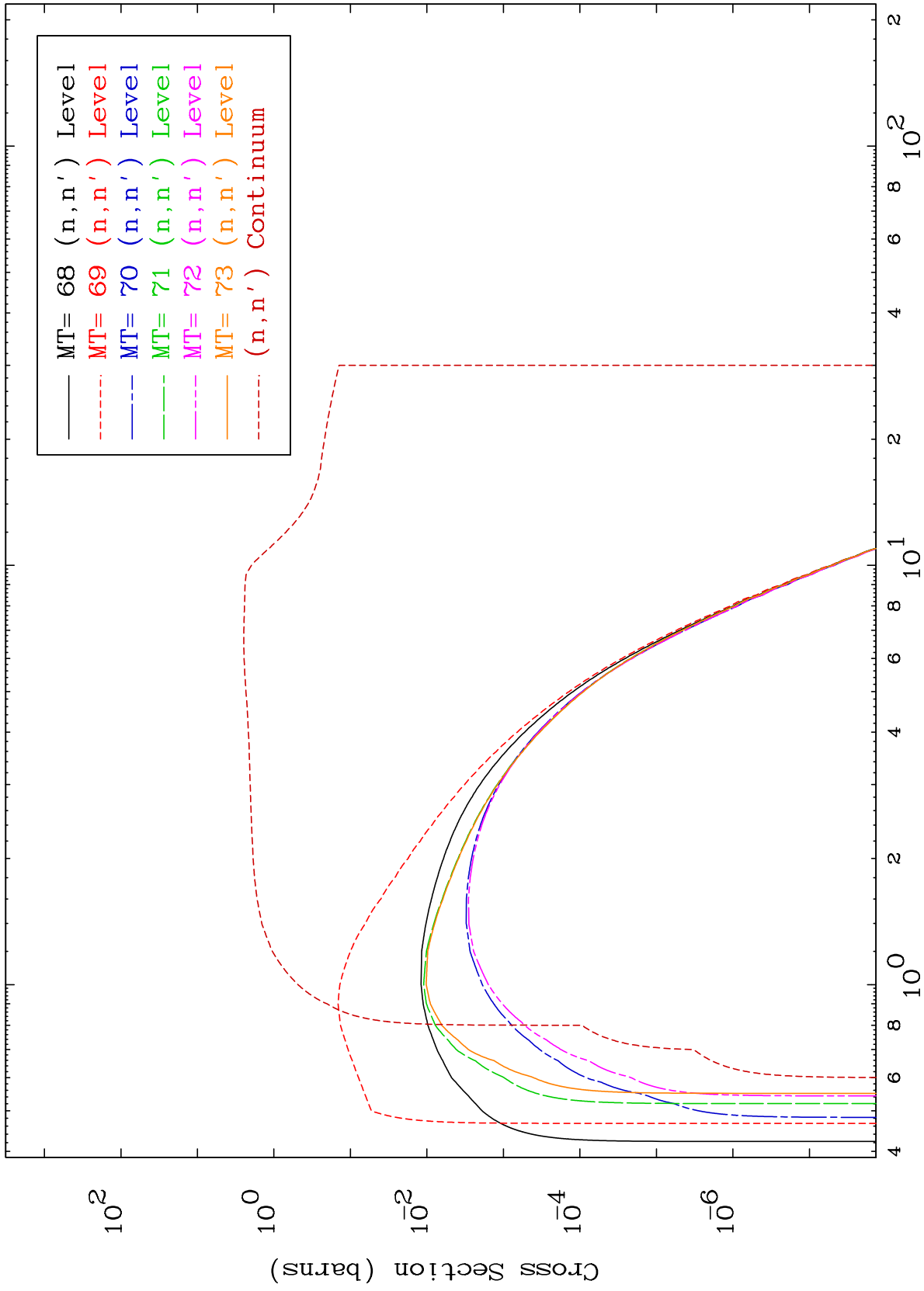
Incident Energy (MeV)

71-Lu-169

MAT 7108

(n,n') Level  
293 Kelvin Cross Sections

71-Lu-169



9

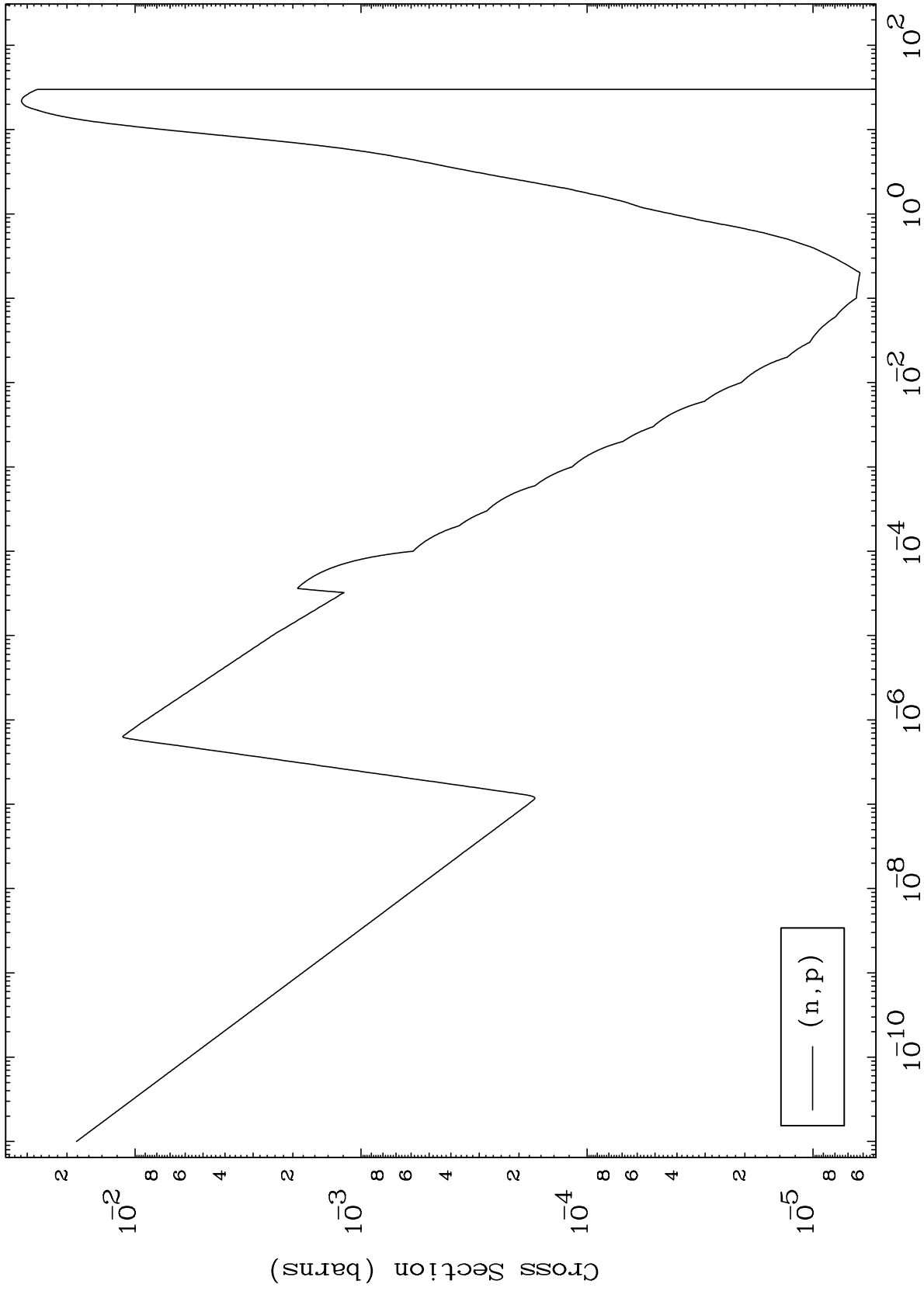
Incident Energy (MeV)

71-Lu-169

MAT 7108

(n,p) Levels  
293 Kelvin Cross Sections

71-Lu-169



10

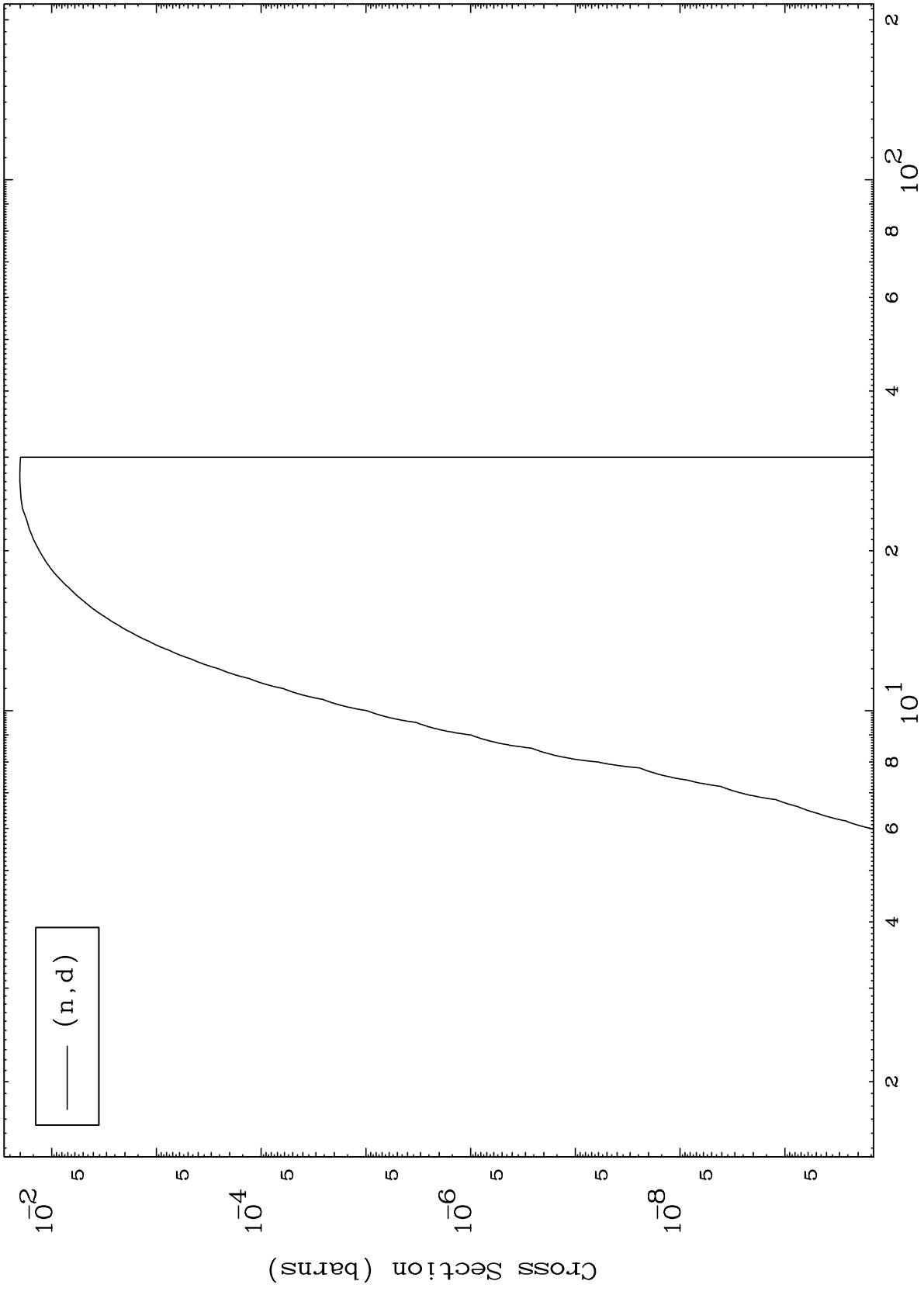
Incident Energy (MeV)

71-Lu-169

MAT 7108

(n,d) Levels  
293 Kelvin Cross Sections

71-Lu-169



11

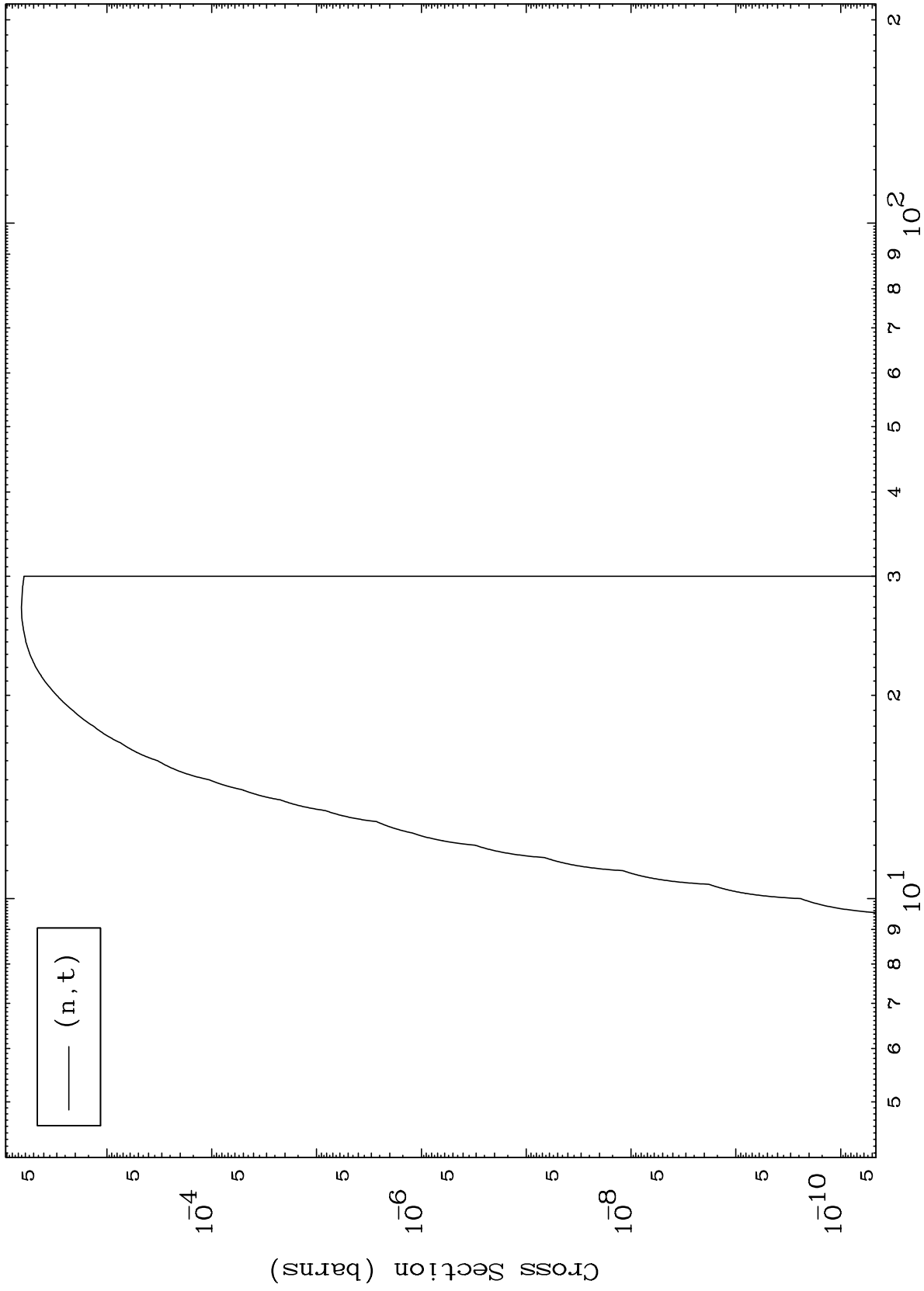
Incident Energy (MeV)

71-Lu-169

MAT 7108

(n,t) Levels  
293 Kelvin Cross Sections

71-Lu-169



12

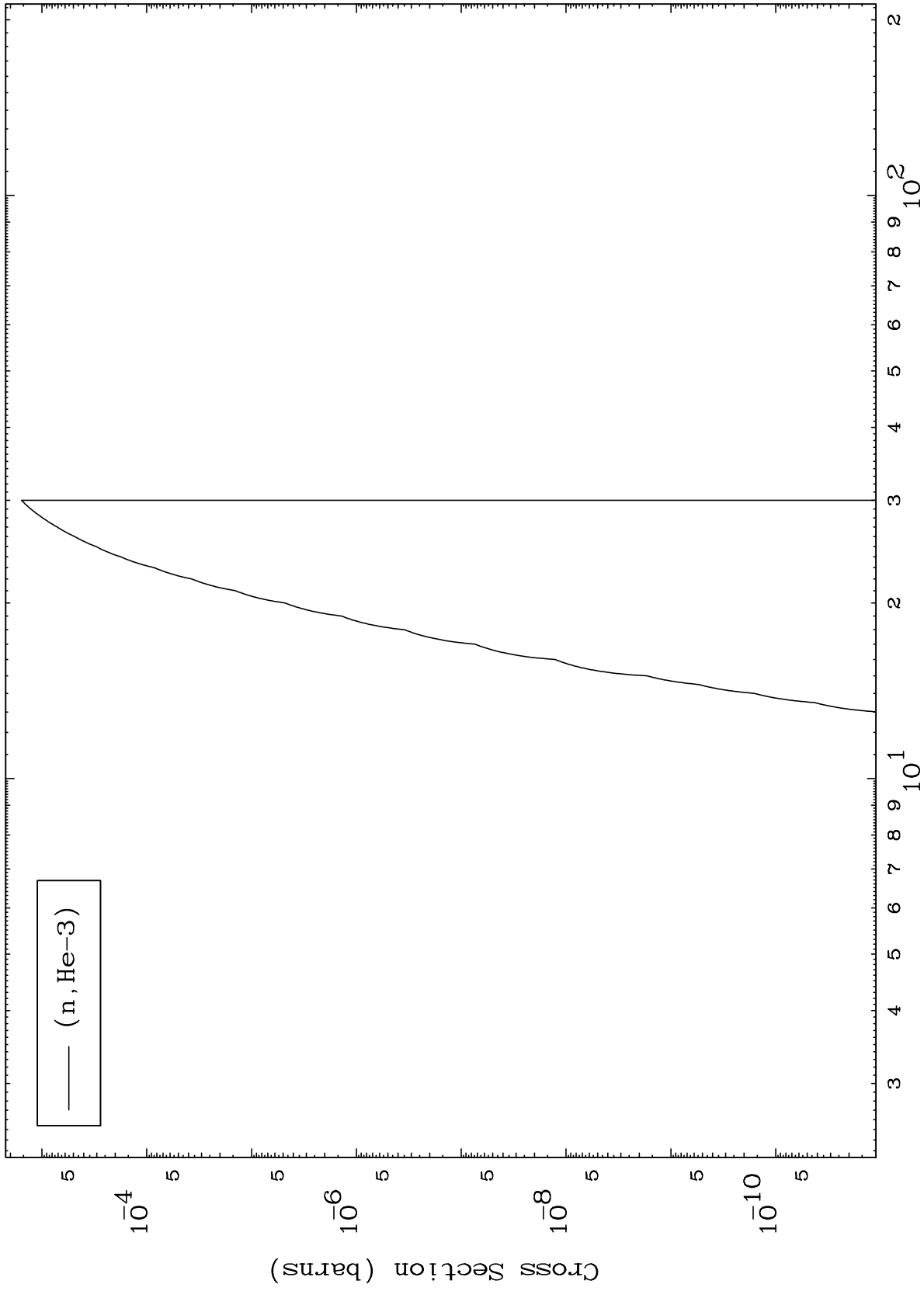
Incident Energy (MeV)

71-Lu-169

MAT 7108

(n,He3) Levels  
293 Kelvin Cross Sections

71-Lu-169



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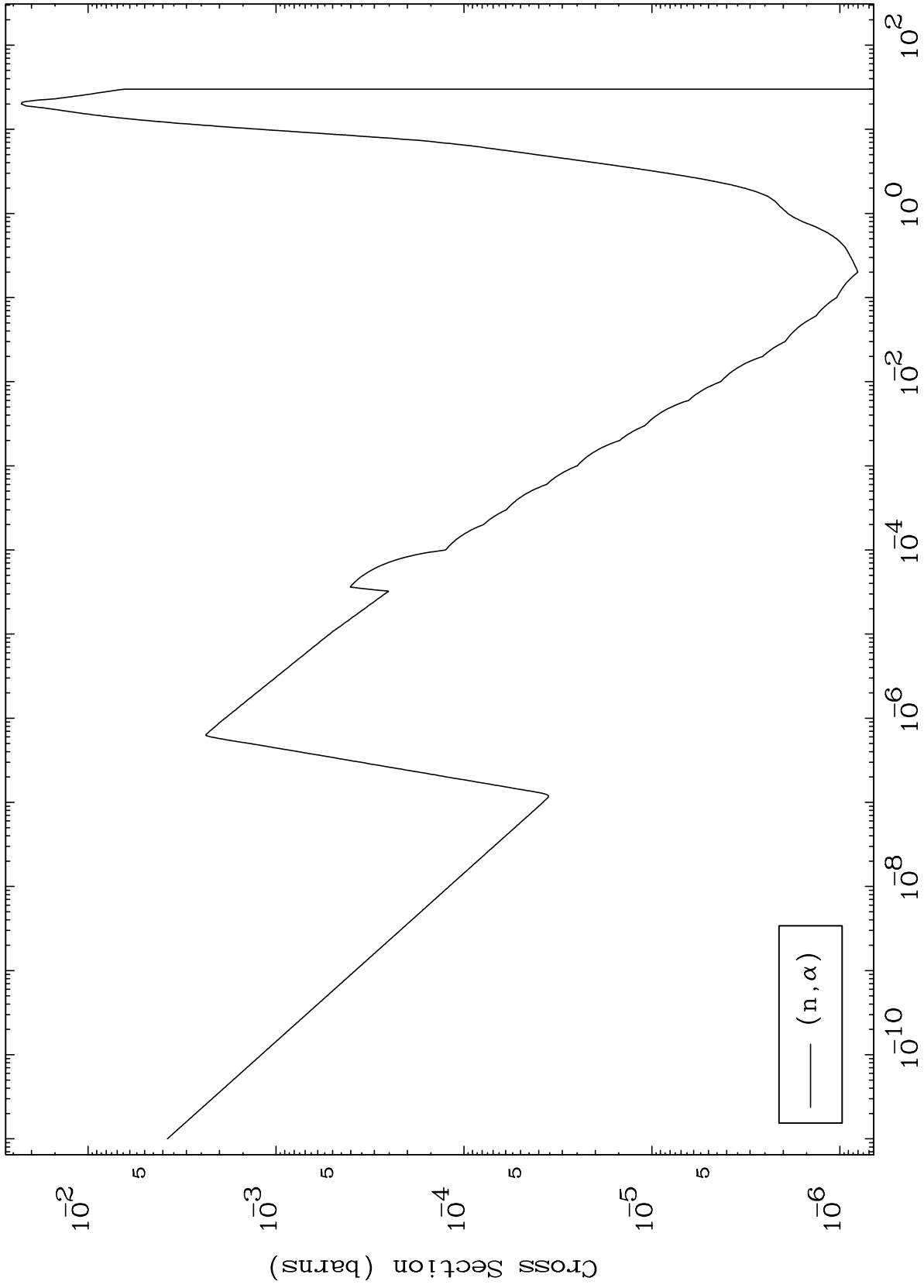
Incident Energy (MeV)

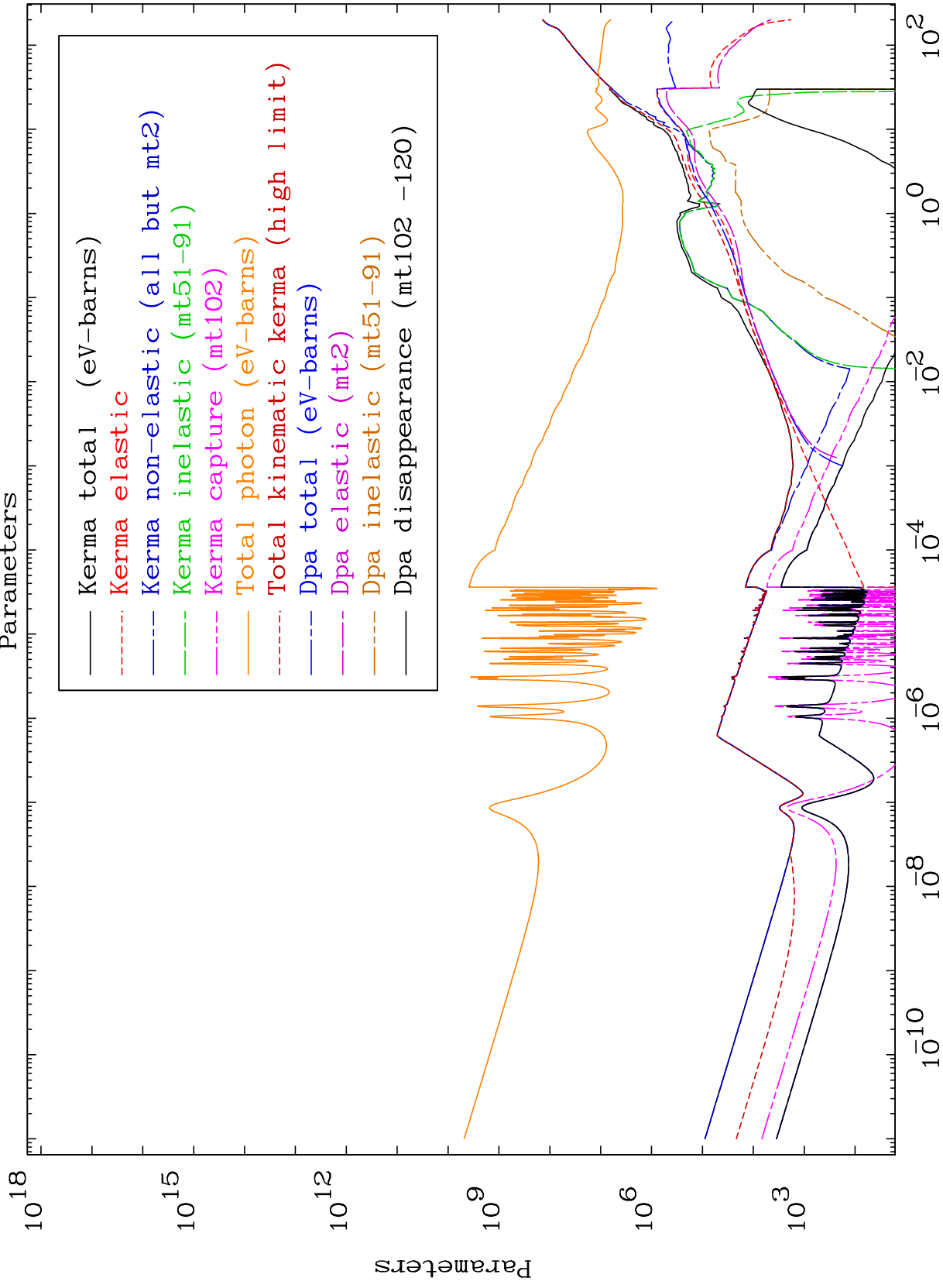
71-Lu-169

MAT 7108

(n,α) Levels  
293 Kelvin Cross Sections

71-Lu-169



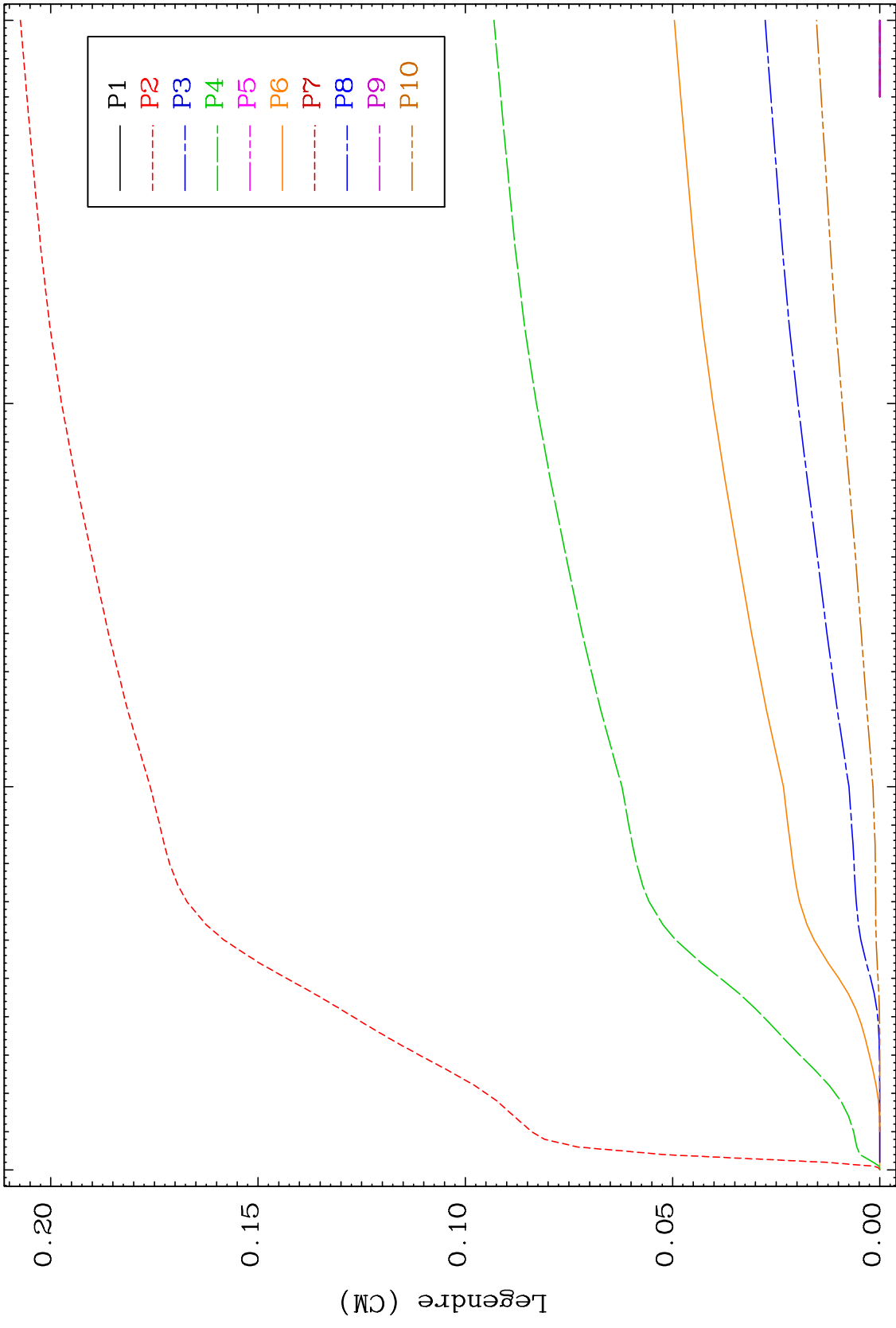




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Elastic Legendre Coefficients

71-Lu-169



71-Lu-169

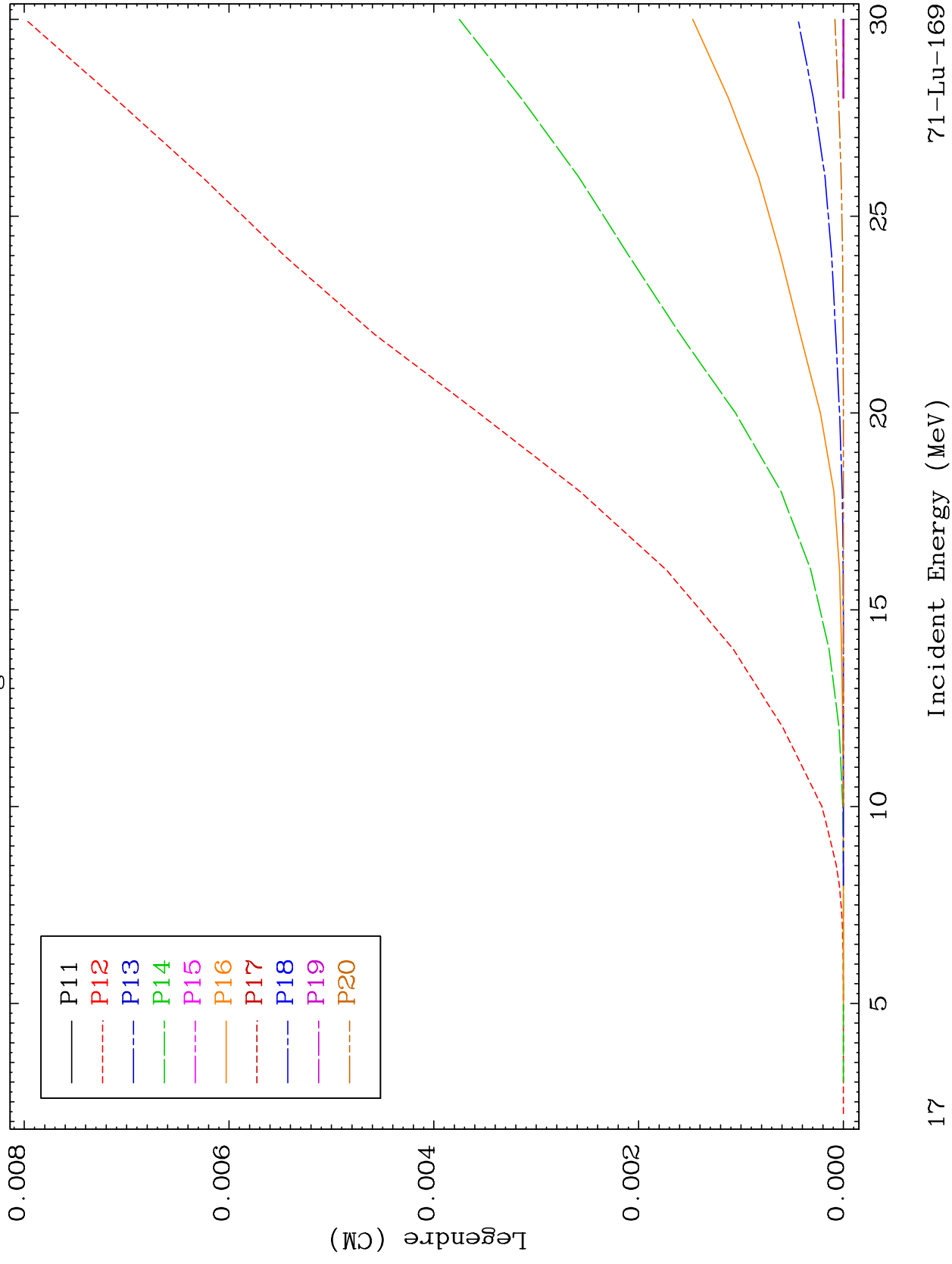
Incident Energy (MeV)

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MAT 7108

Elastic  
Legendre Coefficients

71-Lu-169



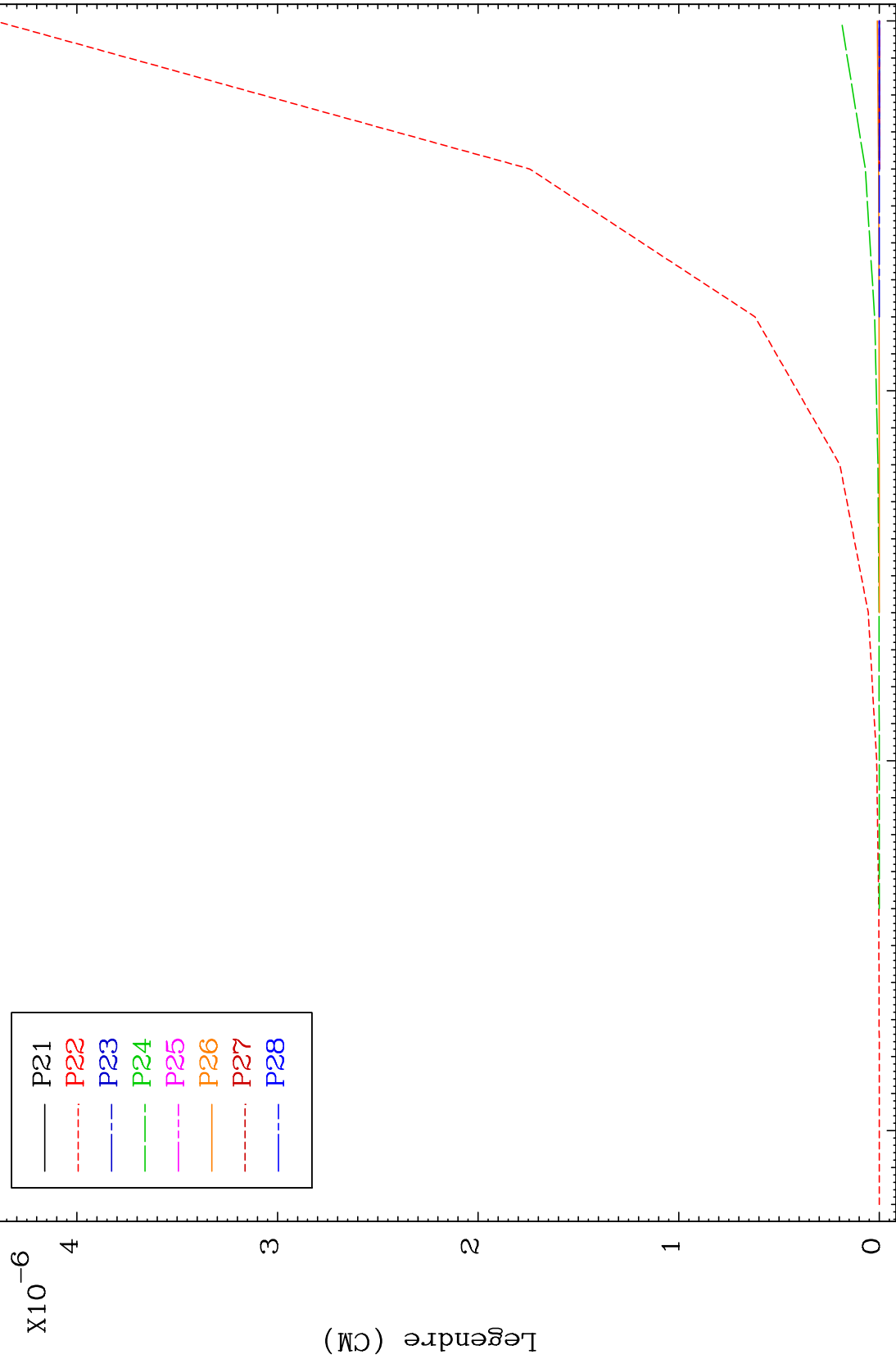
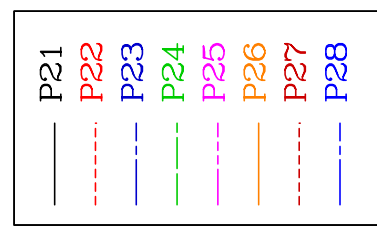
17

71-Lu-169

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Elastic Legendre Coefficients

71-Lu-169



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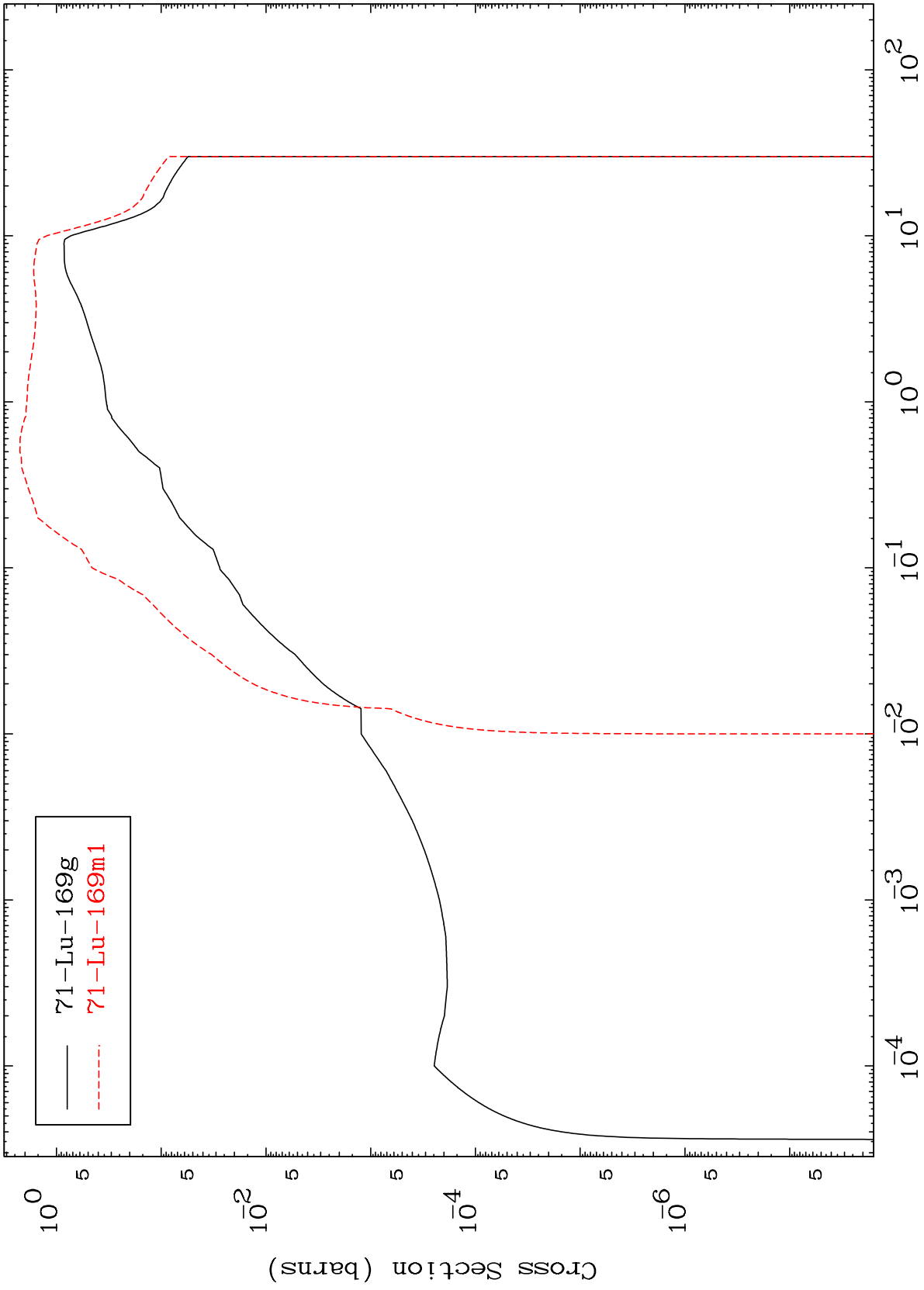
Incident Energy (MeV)

71-Lu-169

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Inelastic  
Radionuclide Production Cross Section

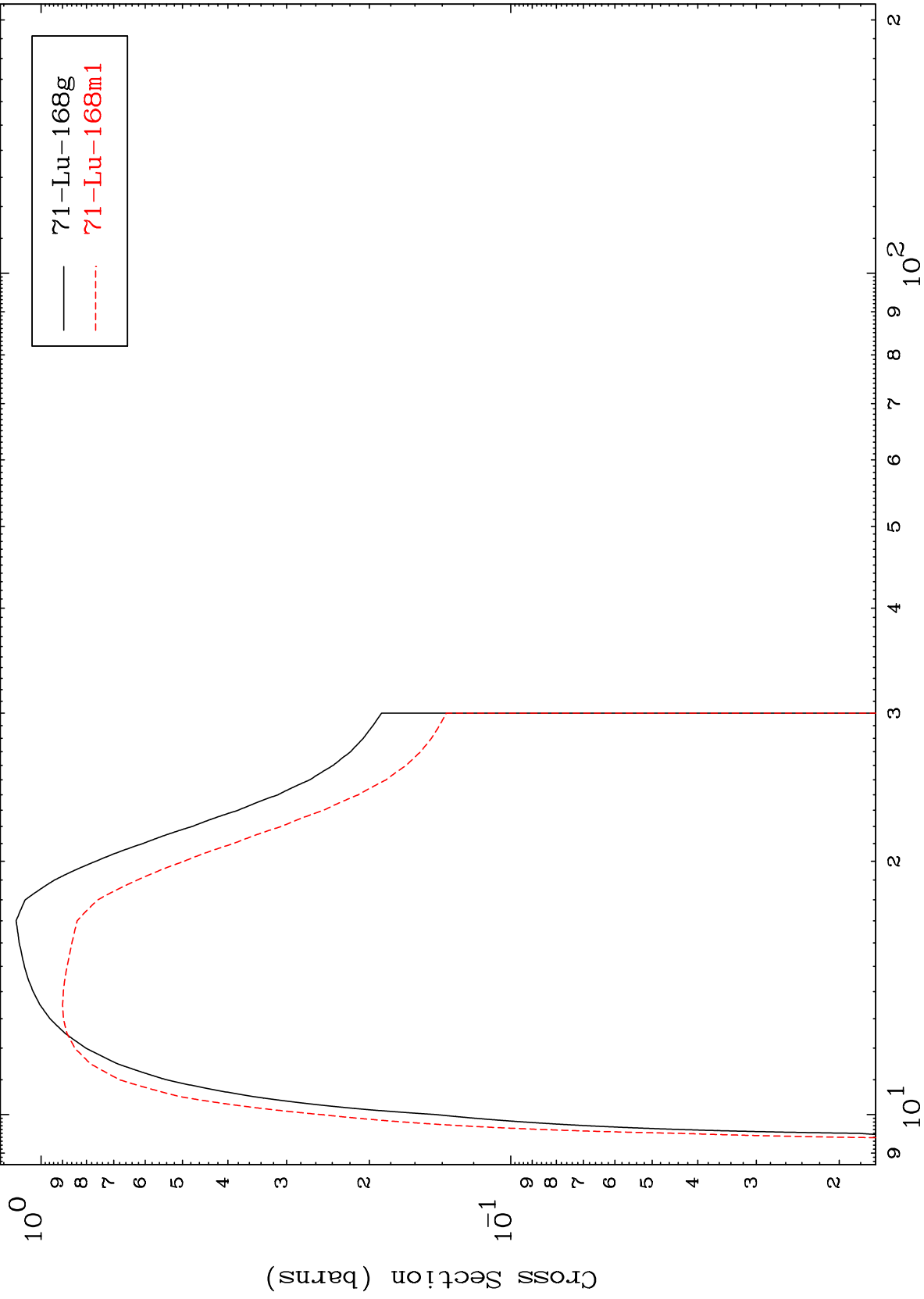
71-Lu-169



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71-Lu-169

(n,2n)  
Radionuclide Production Cross Section



20

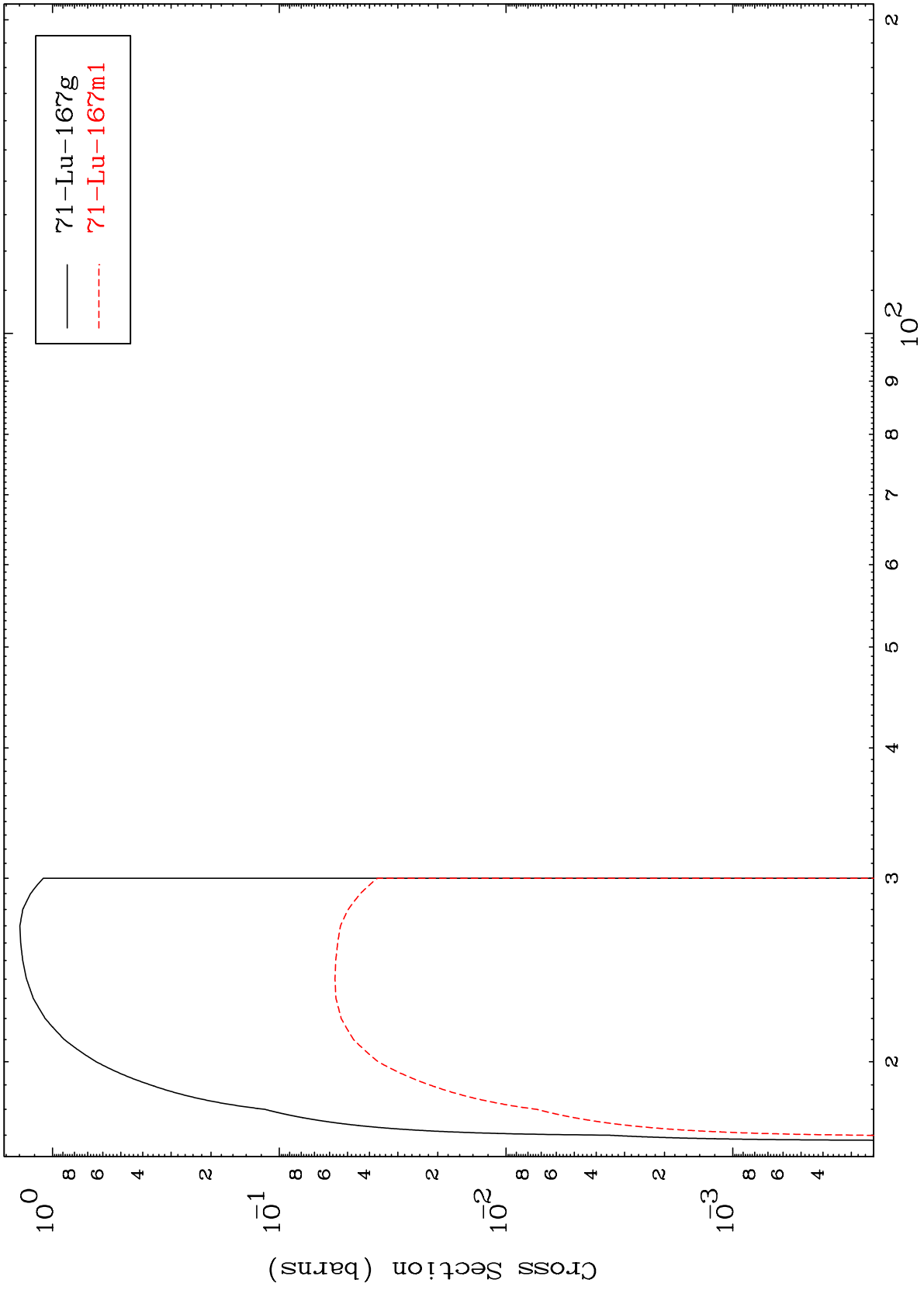
Incident Energy (MeV)

71-Lu-169

MAT 7108

71-Lu-169

(n,3n)  
Radionuclide Production Cross Section



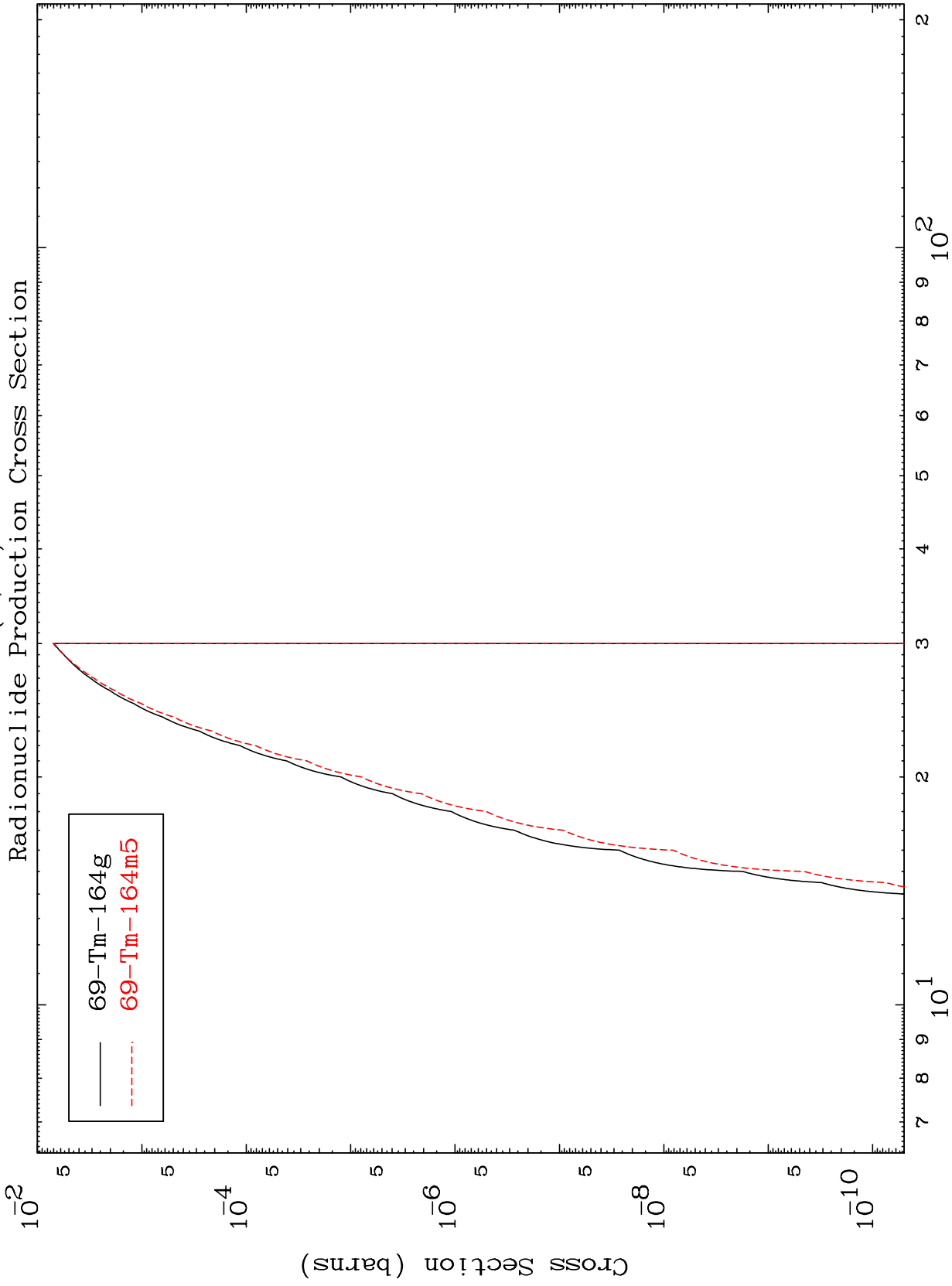
21

71-Lu-169

MAT 7108

$(n, 2n) \alpha$

71-Lu-169



22

Incident Energy (MeV)

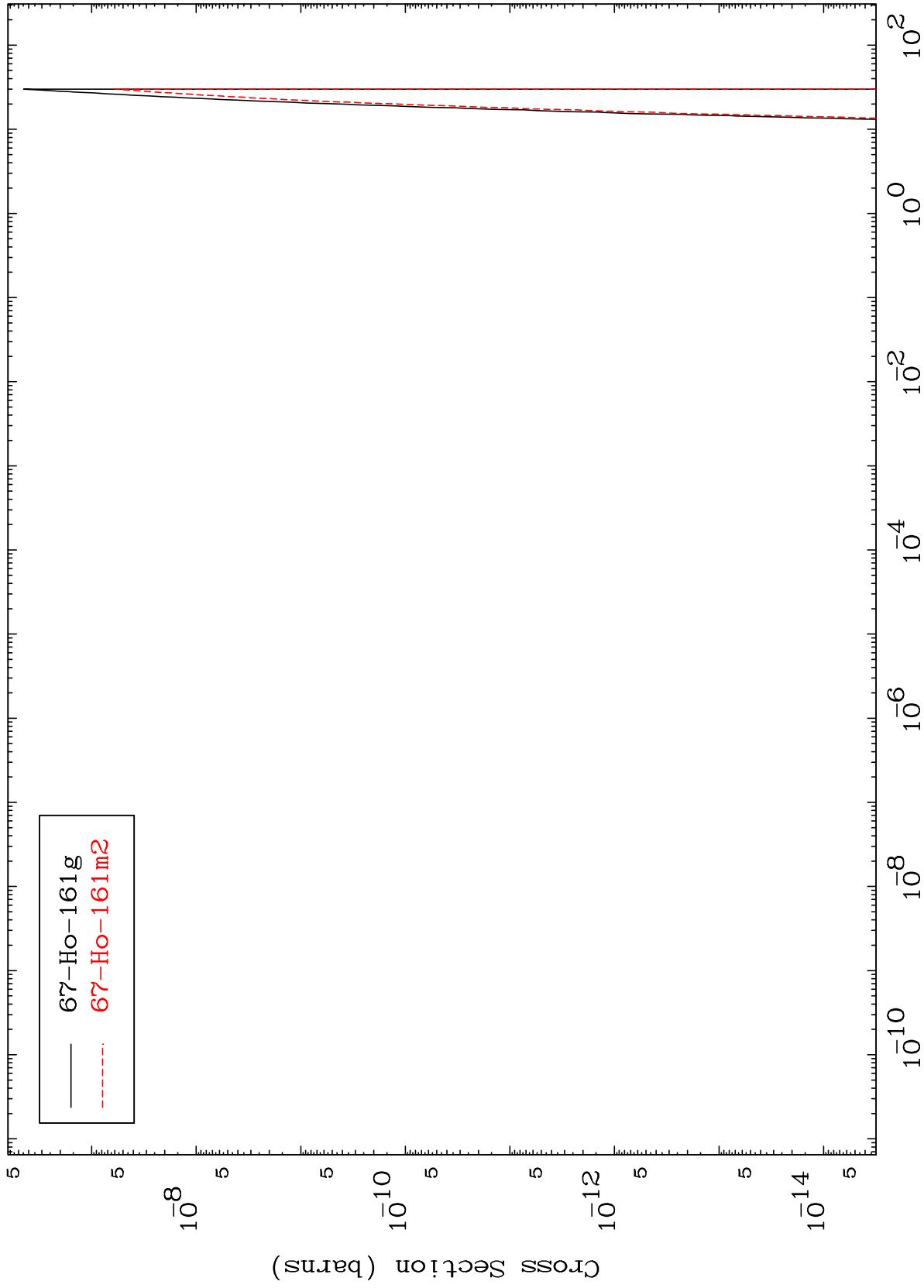
71-Lu-169

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(n,n') 2α

71-Lu-169

Radionuclide Production Cross Section



23

Incident Energy (MeV)

71-Lu-169

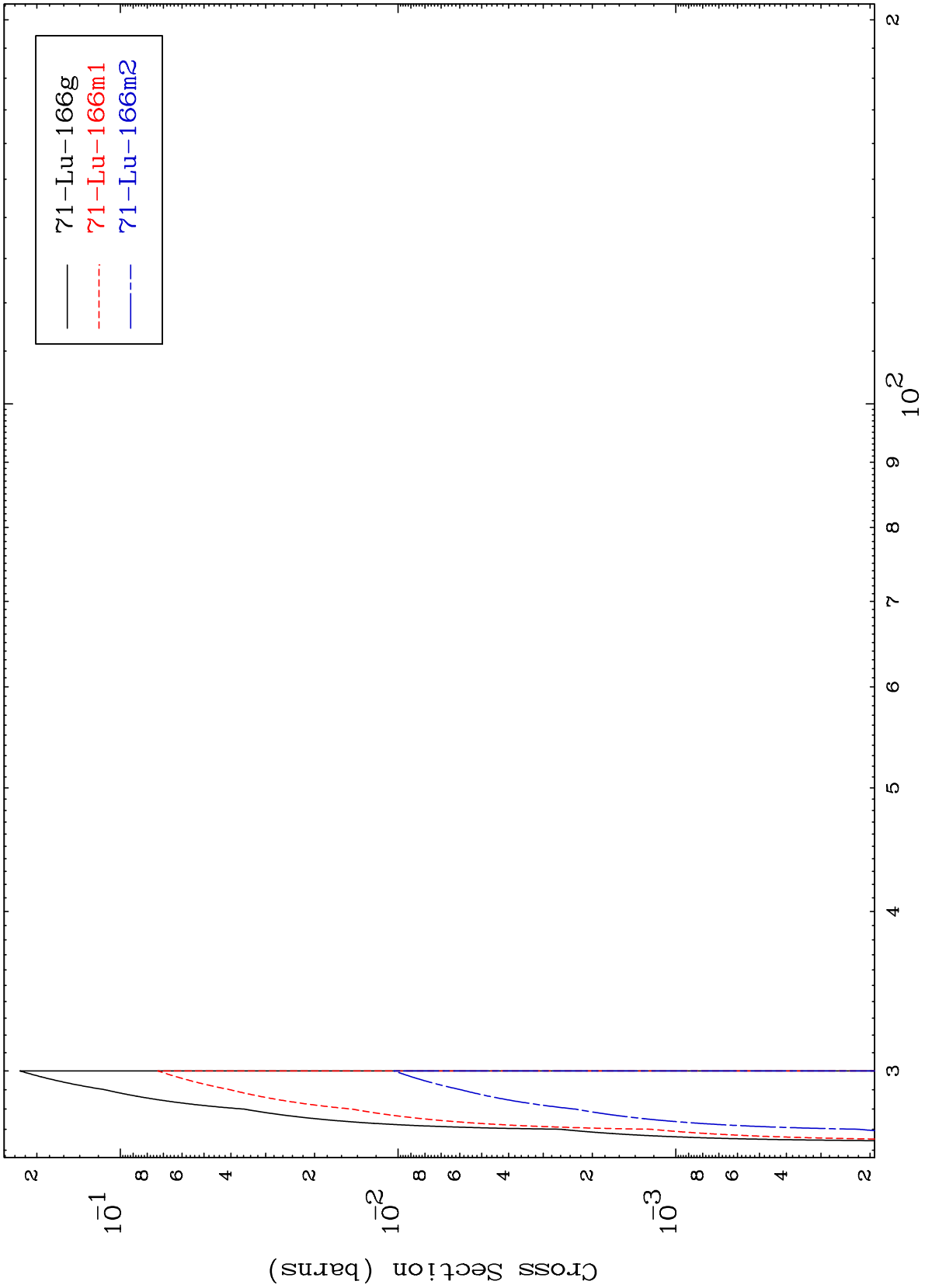


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(n,4n)

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Radionuclide Production Cross Section



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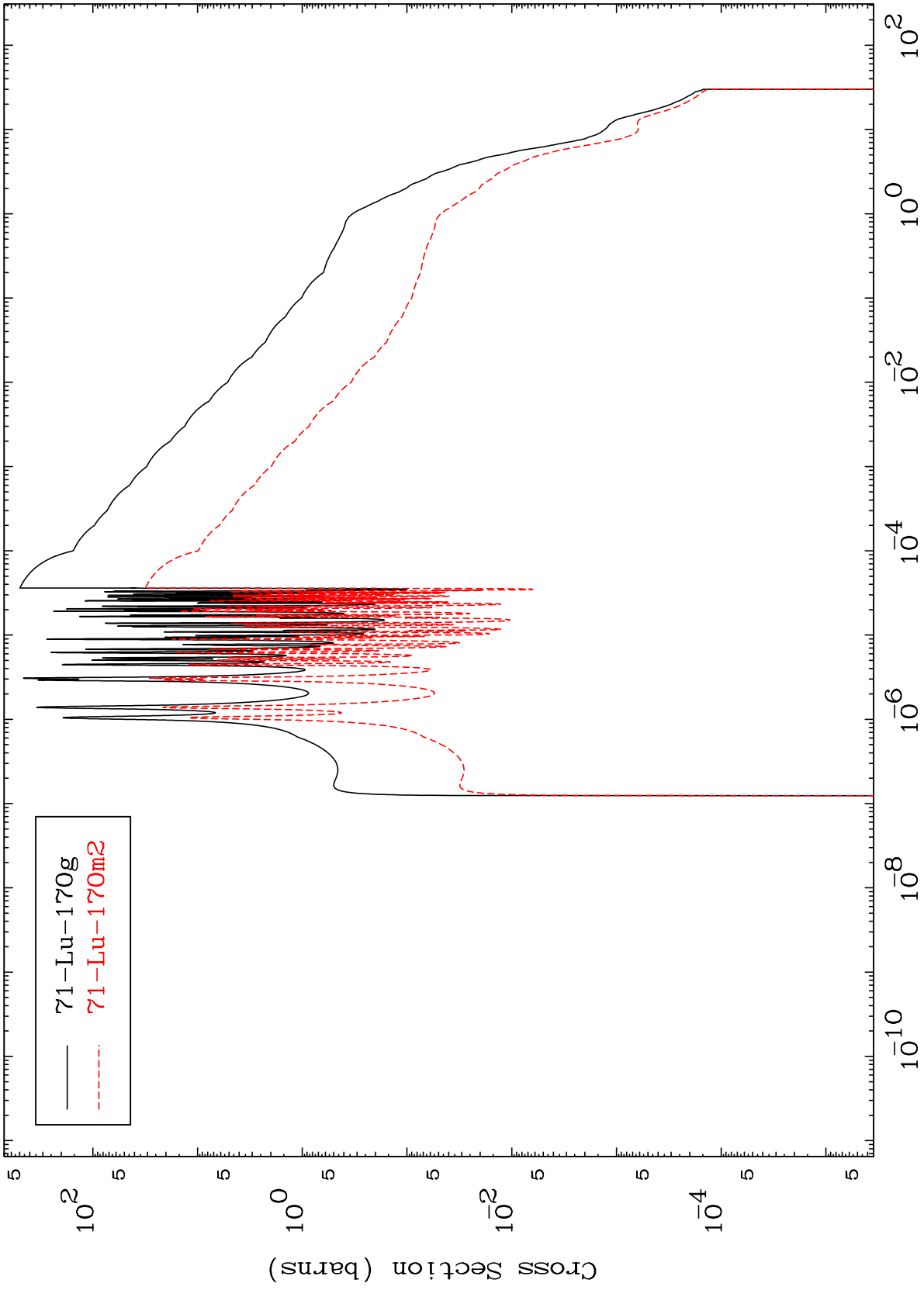
Incident Energy (MeV)

71-Lu-169

MAT 7108

71-Lu-169

Radionuclide Production Cross Section (n,γ)



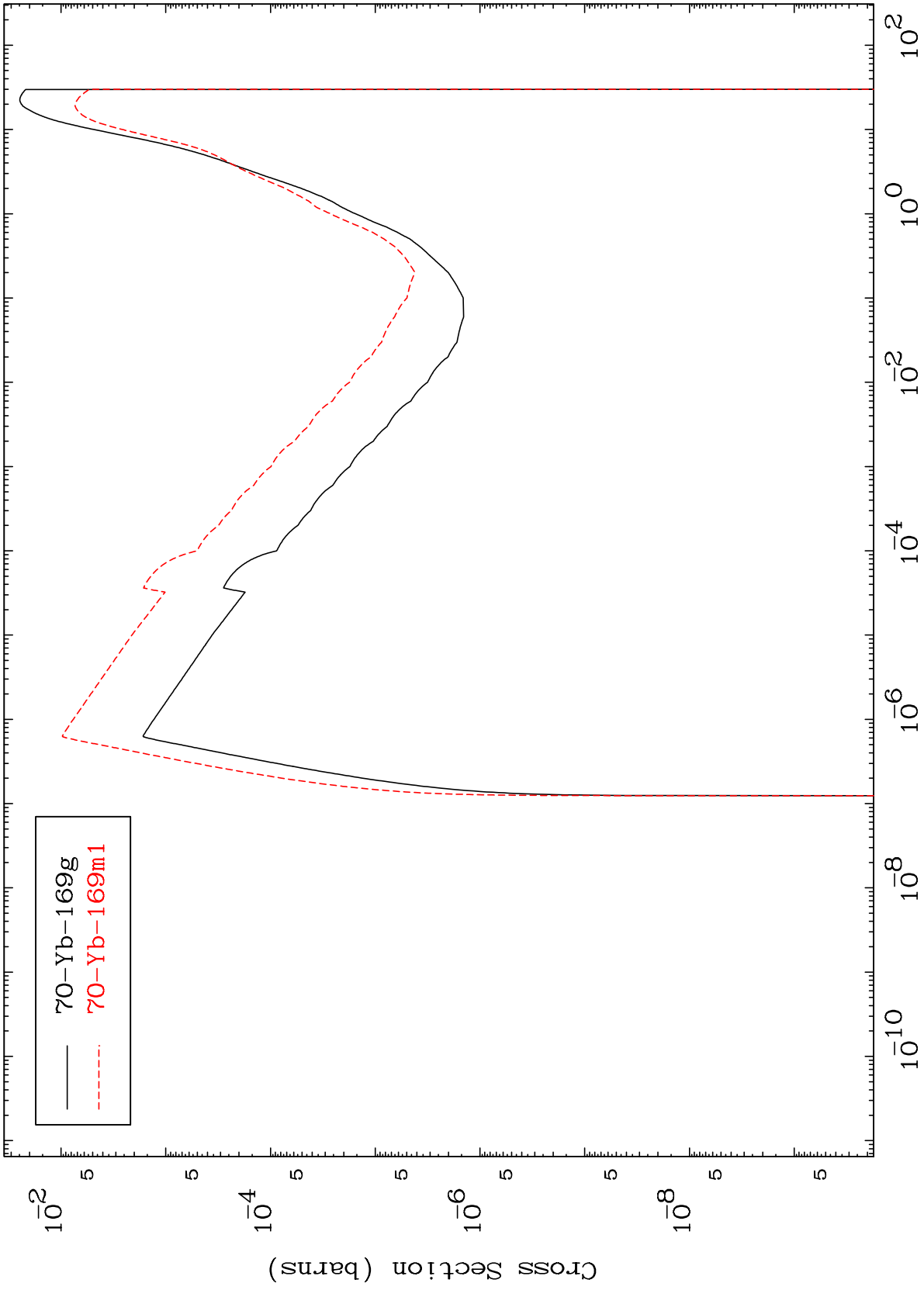
25

71-Lu-169

MAT 7108

71-Lu-169

(n,p)  
Radionuclide Production Cross Section



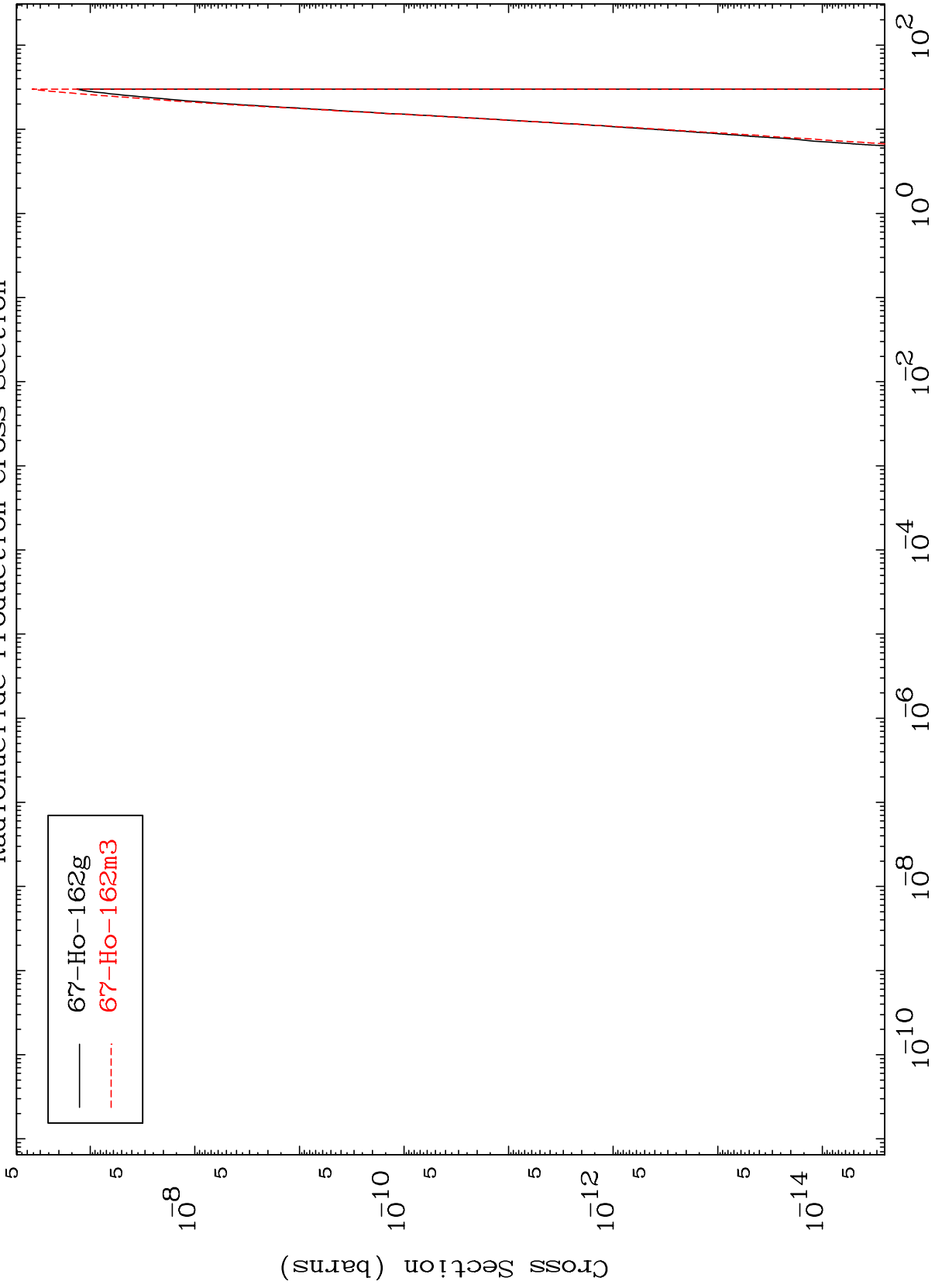
— 70-Yb-169g  
- - - 70-Yb-169m1

MAT 7108

(n,2α)

71-Lu-169

Radionuclide Production Cross Section



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Incident Energy (MeV)

71-Lu-169