

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

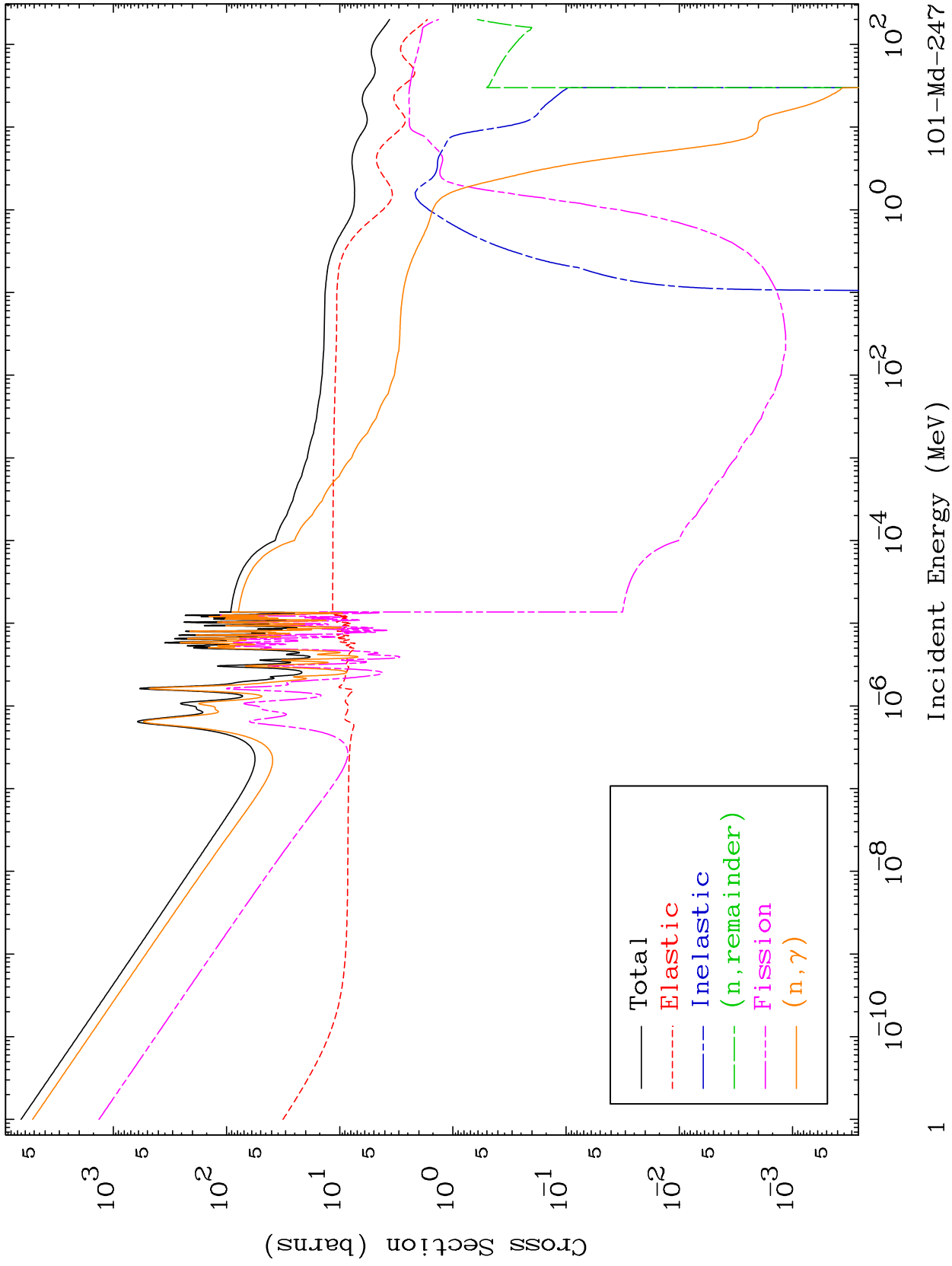
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9948

Major
293 Kelvin Cross Sections

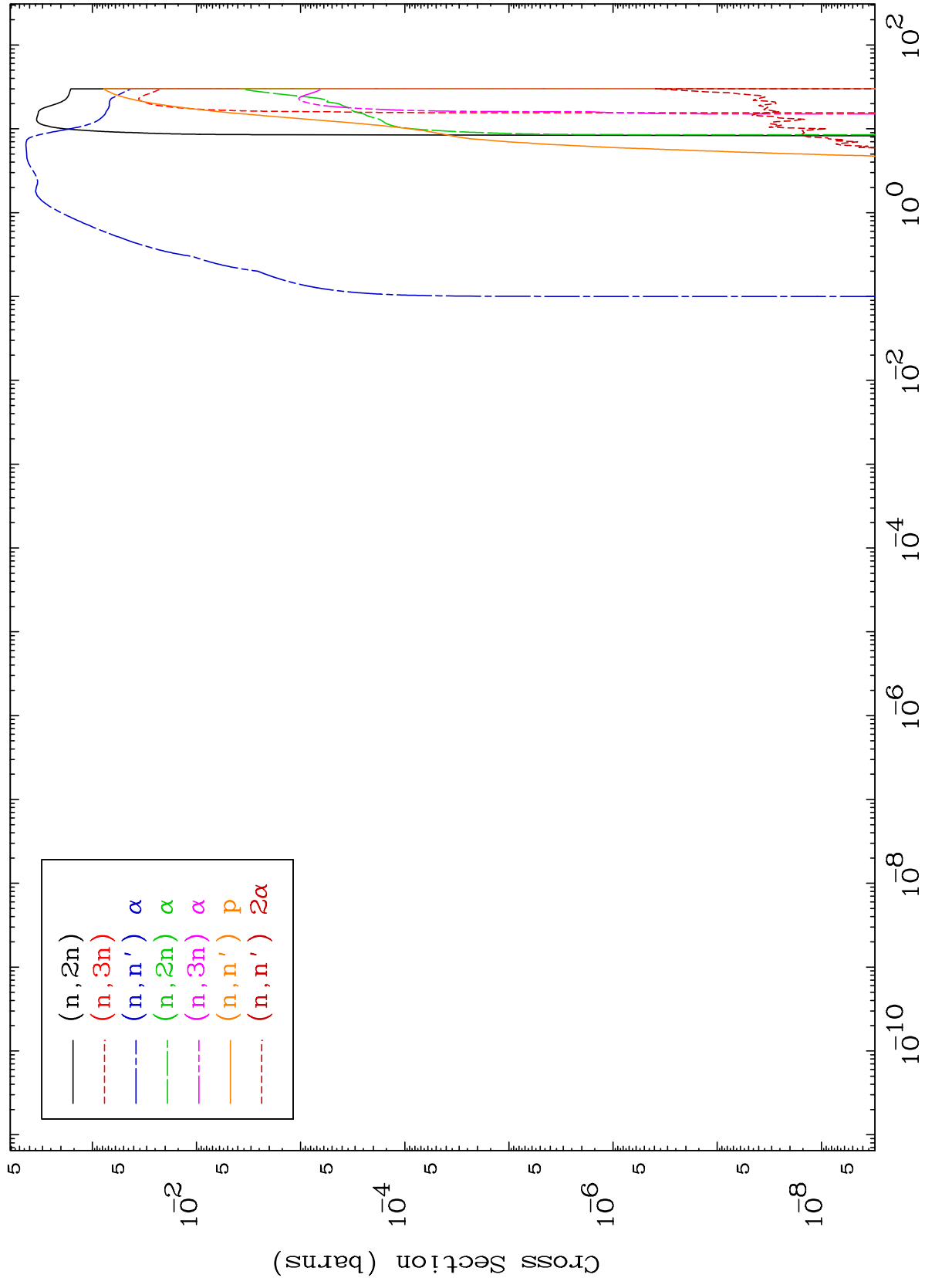
101-Md-247



MAT 9948

Neutron Production
293 Kelvin Cross Sections

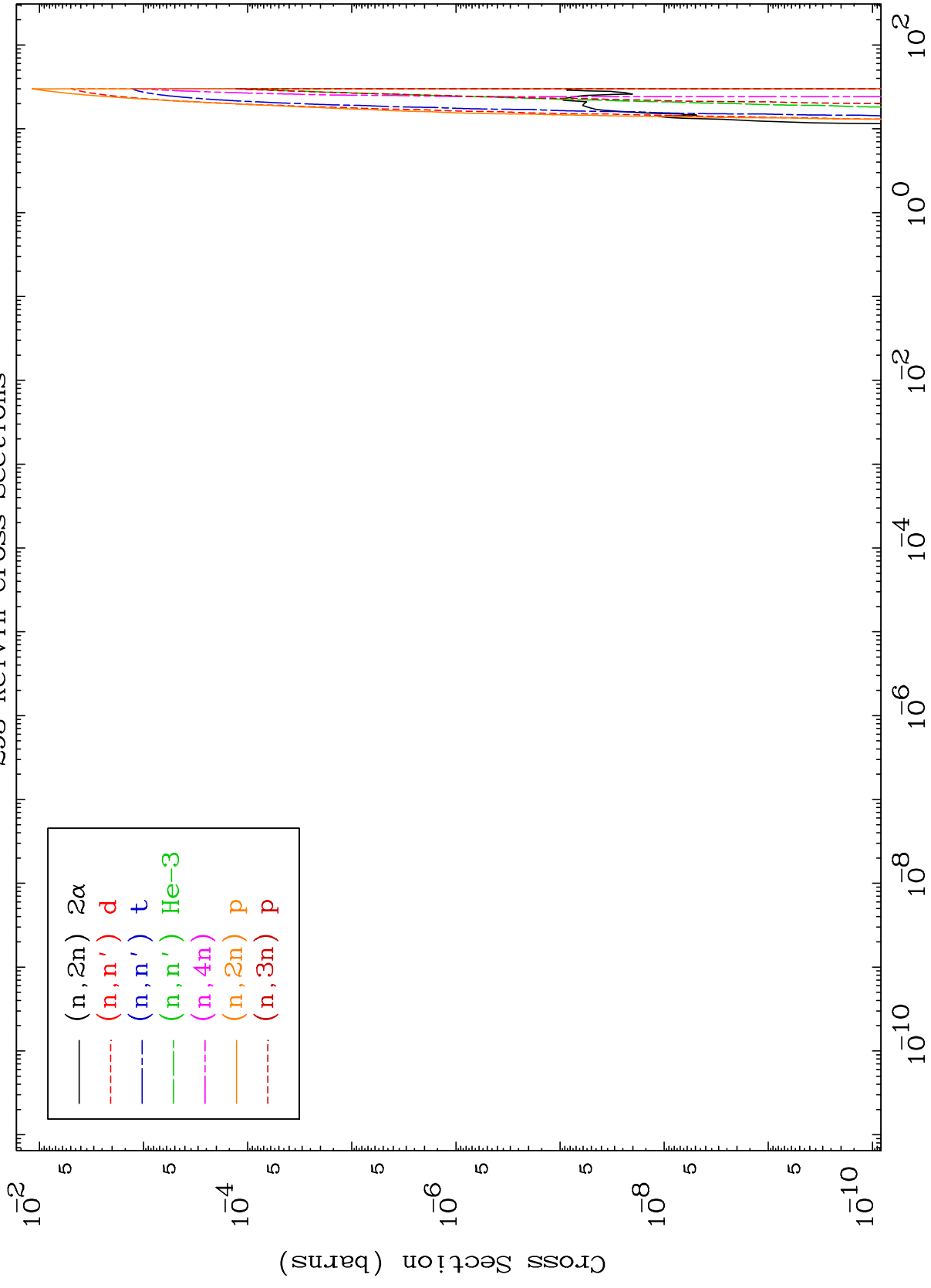
101-Md-247



MAT 9948

Neutron Production
293 Kelvin Cross Sections

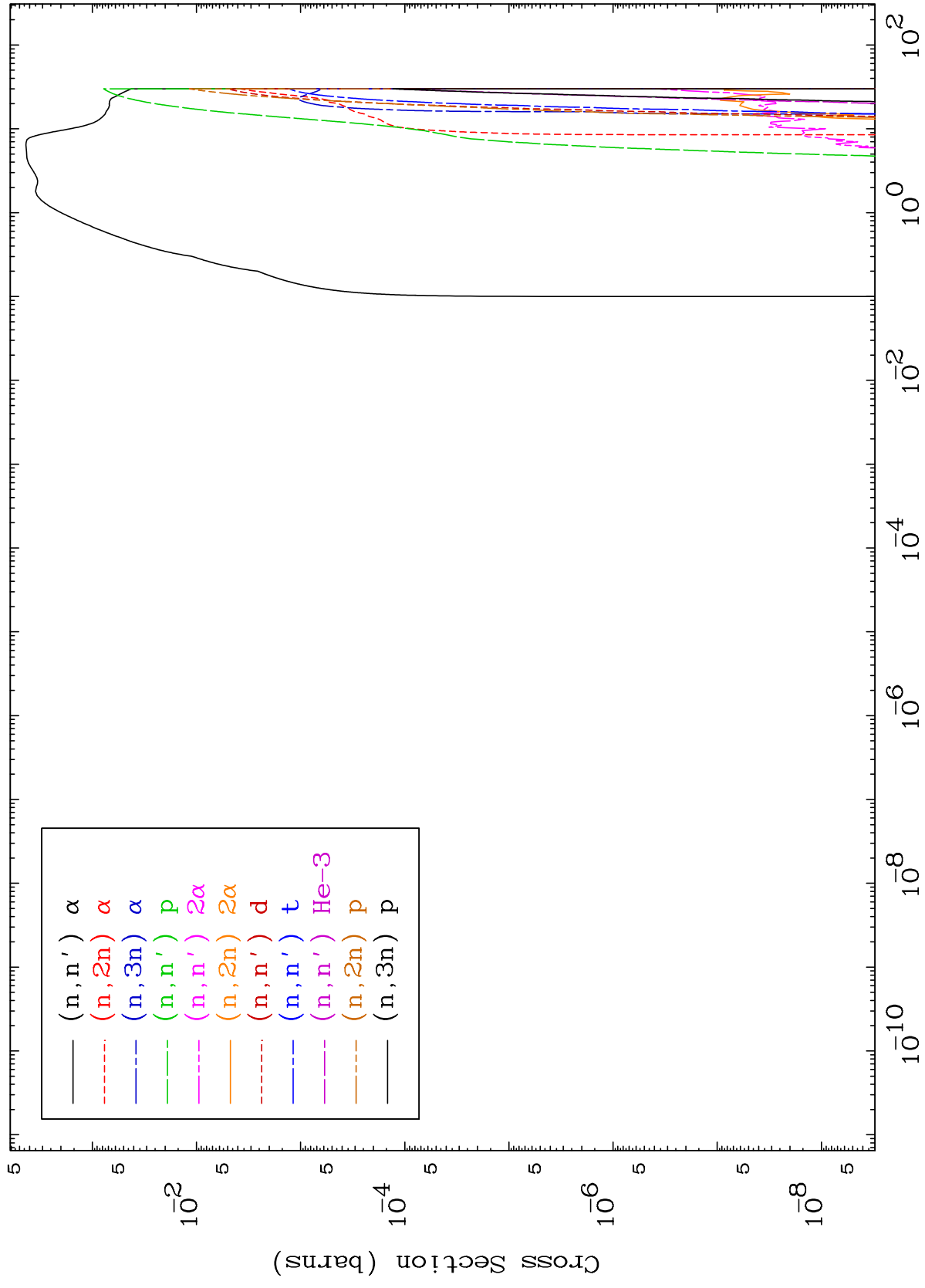
101-Md-247



MAT 9948

Charged Particle
293 Kelvin Cross Sections

101-Md-247



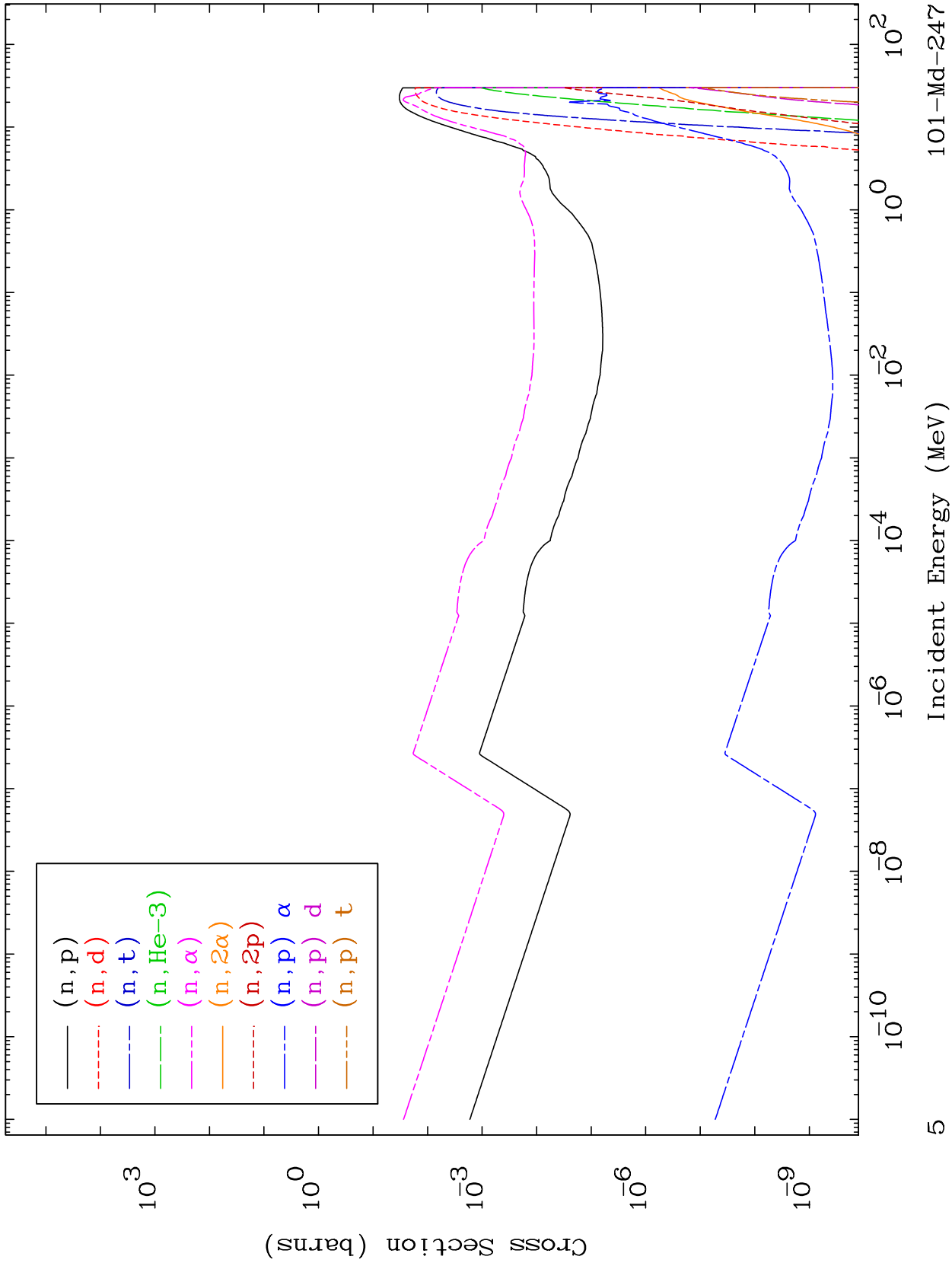
101-Md-247

Incident Energy (MeV)

MAT 9948

293 Kelvin Cross Sections

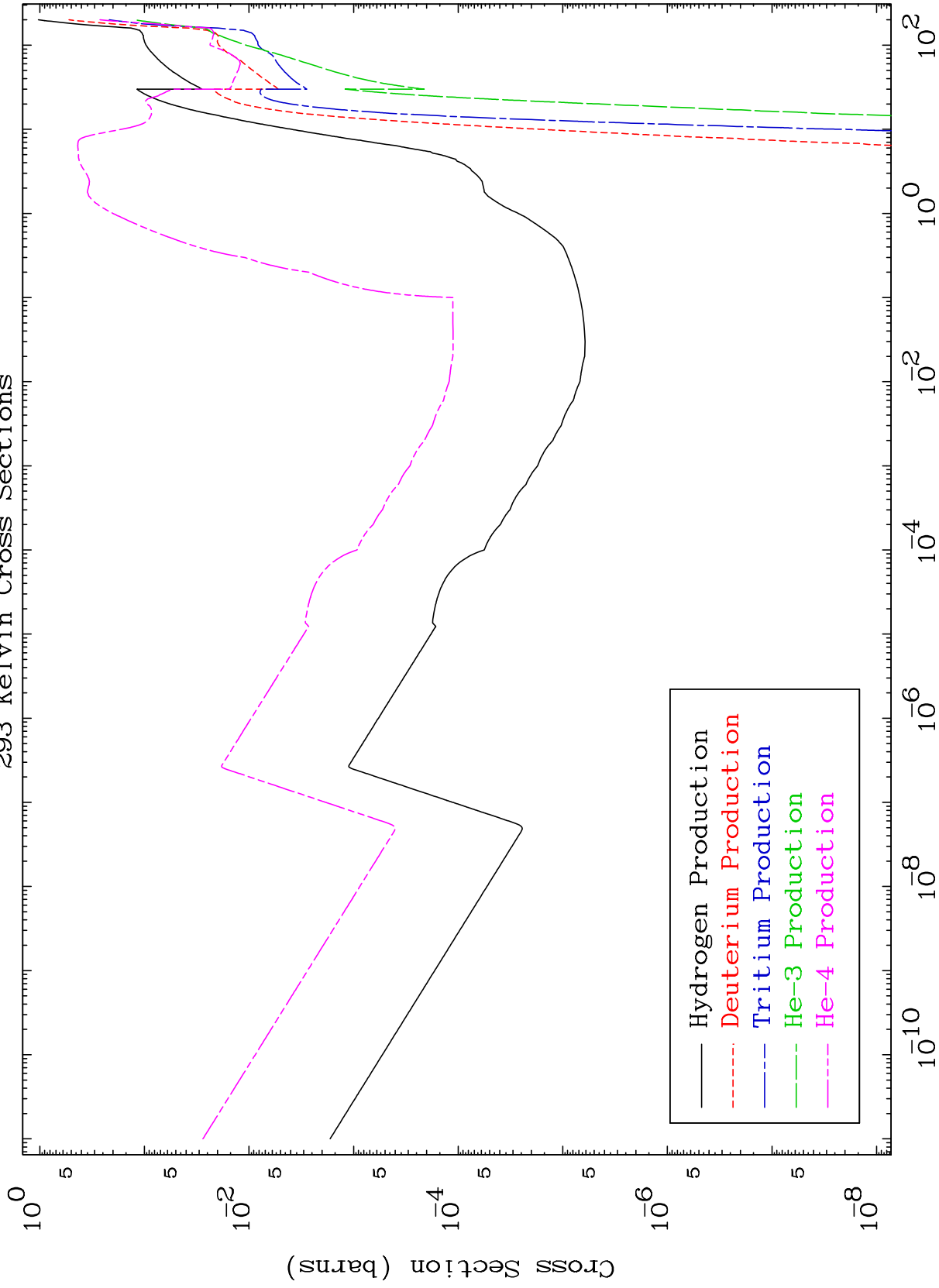
101-Md-247



MAT 9948

Particle Production
293 Kelvin Cross Sections

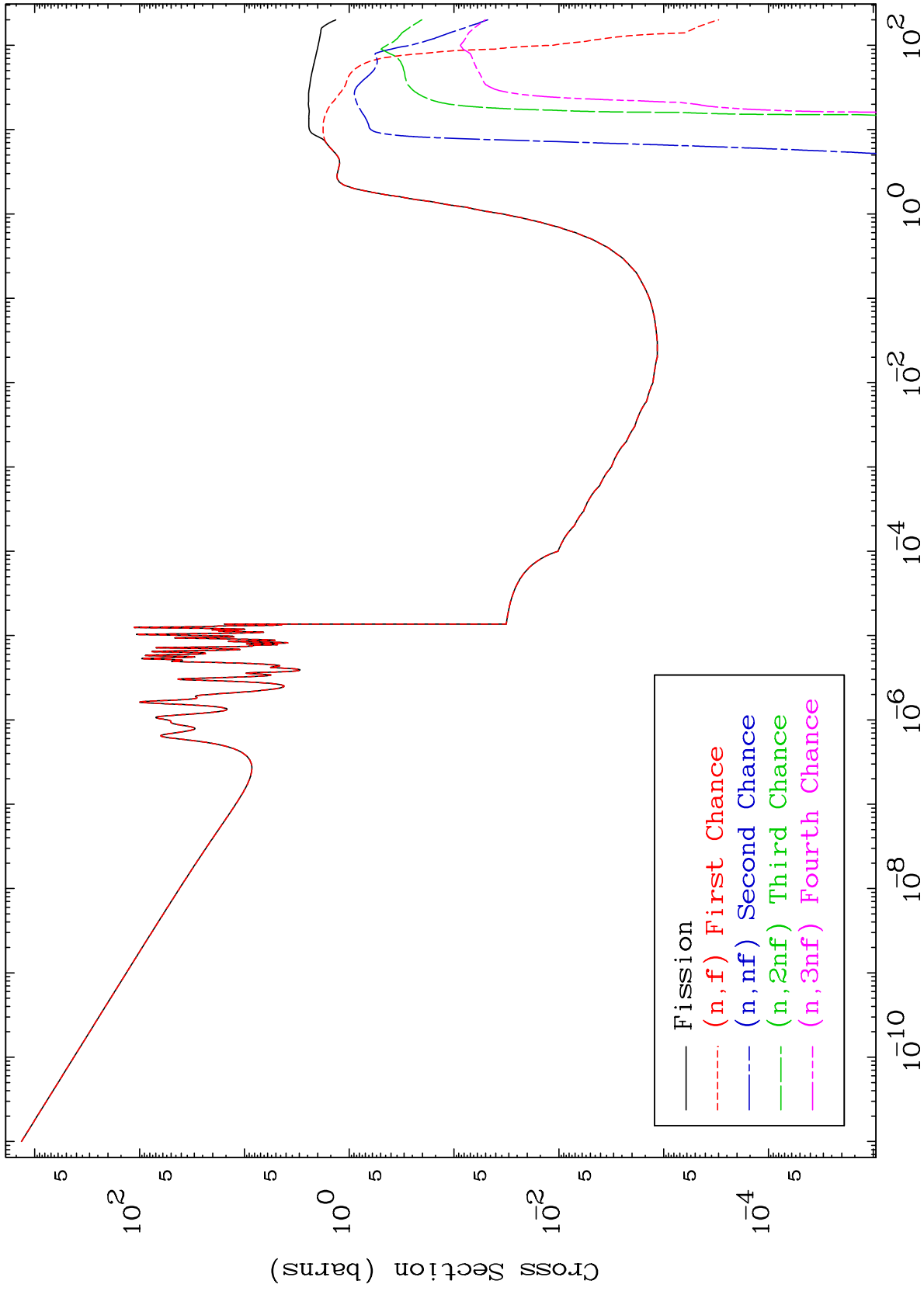
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MAT 9948

Fission
293 Kelvin Cross Sections

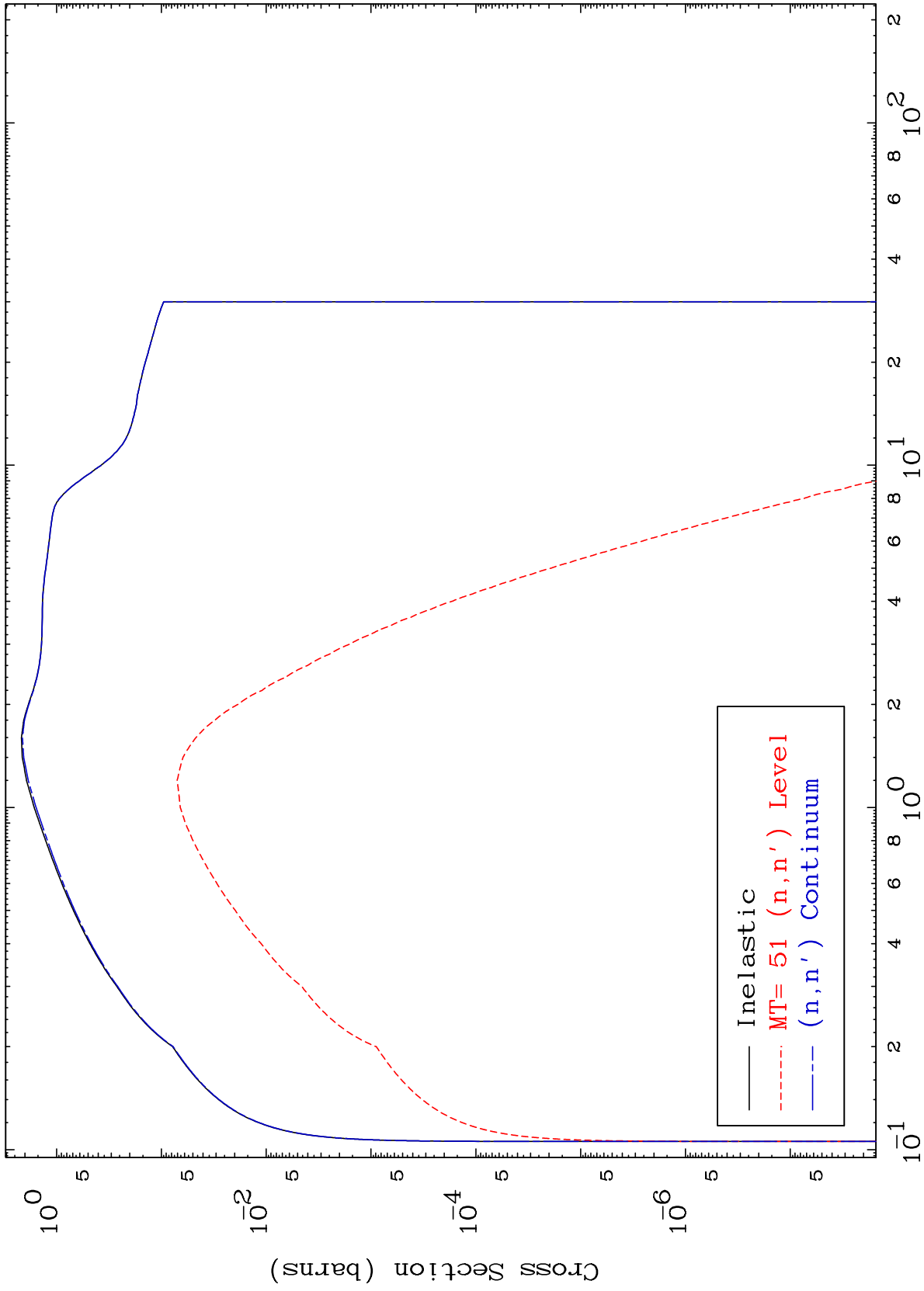
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MAT 9948

(n,n') Level
293 Kelvin Cross Sections

101-Md-247



8

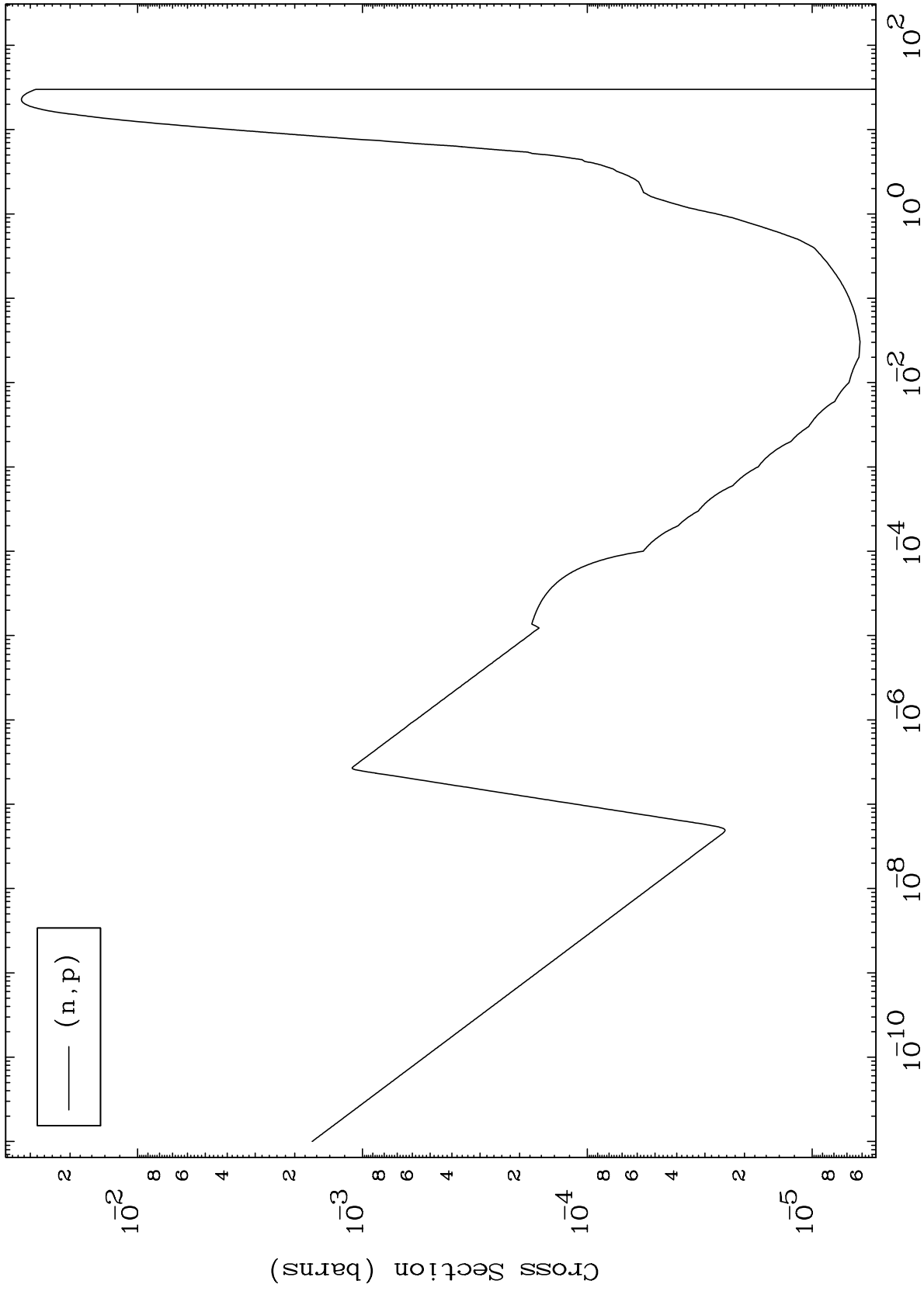
Incident Energy (MeV)

101-Md-247

MAT 9948

(n,p) Levels
293 Kelvin Cross Sections

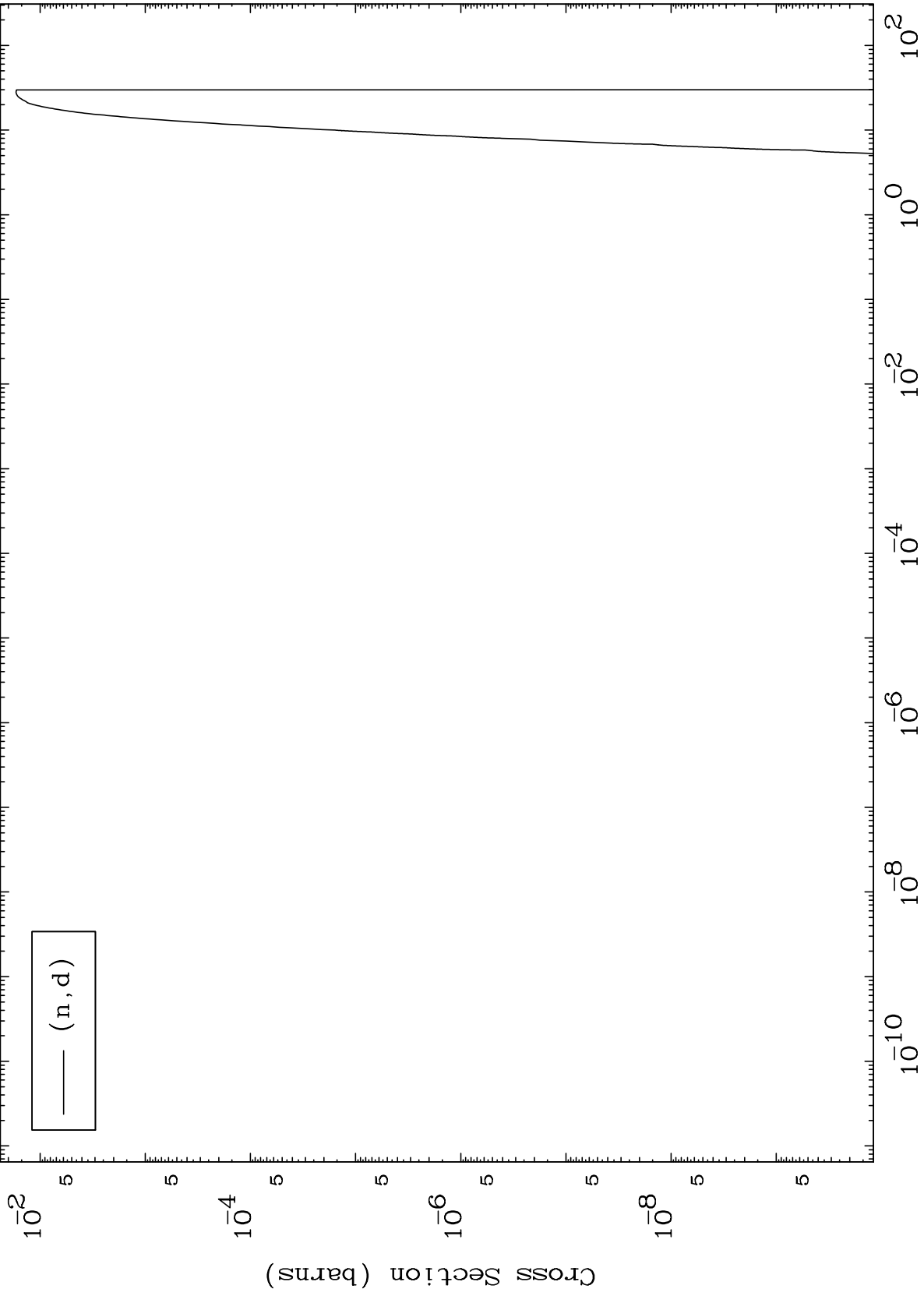
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(n,d) Levels
293 Kelvin Cross Sections

101-Md-247



10

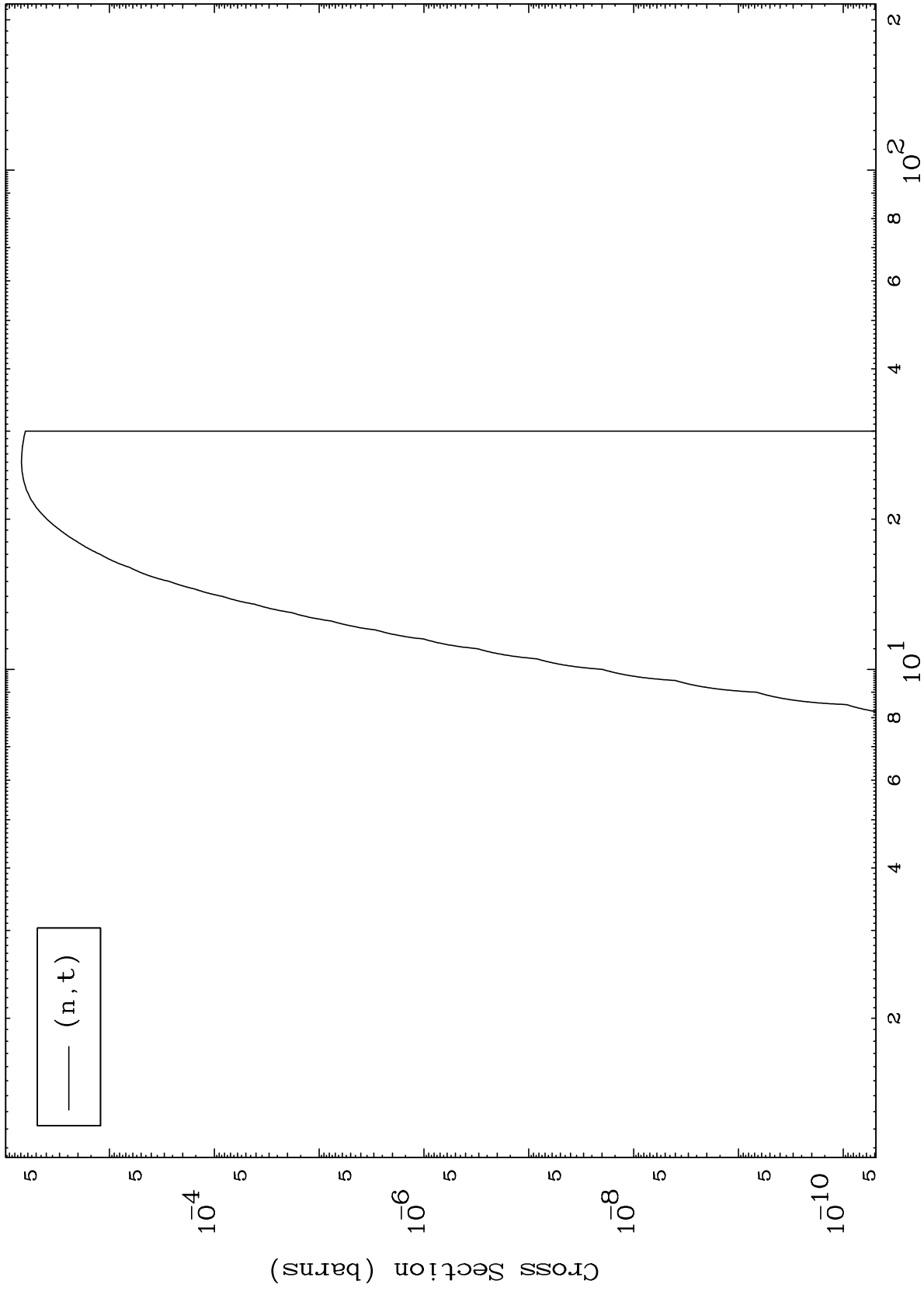
Incident Energy (MeV)

101-Md-247

MAT 9948

(n,t) Levels
293 Kelvin Cross Sections

101-Md-247



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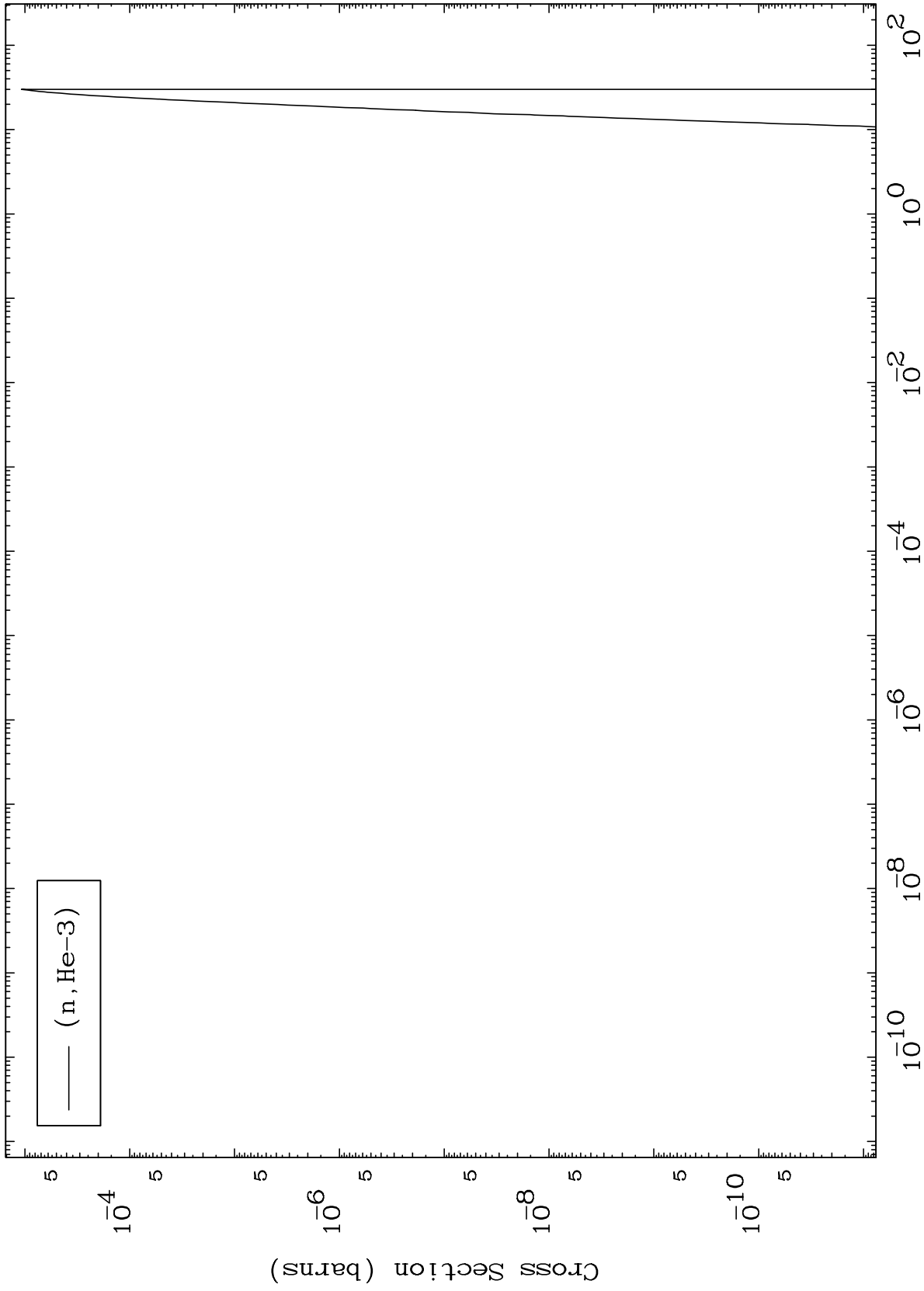
Incident Energy (MeV)

101-Md-247

MAT 9948

(n,He3) Levels
293 Kelvin Cross Sections

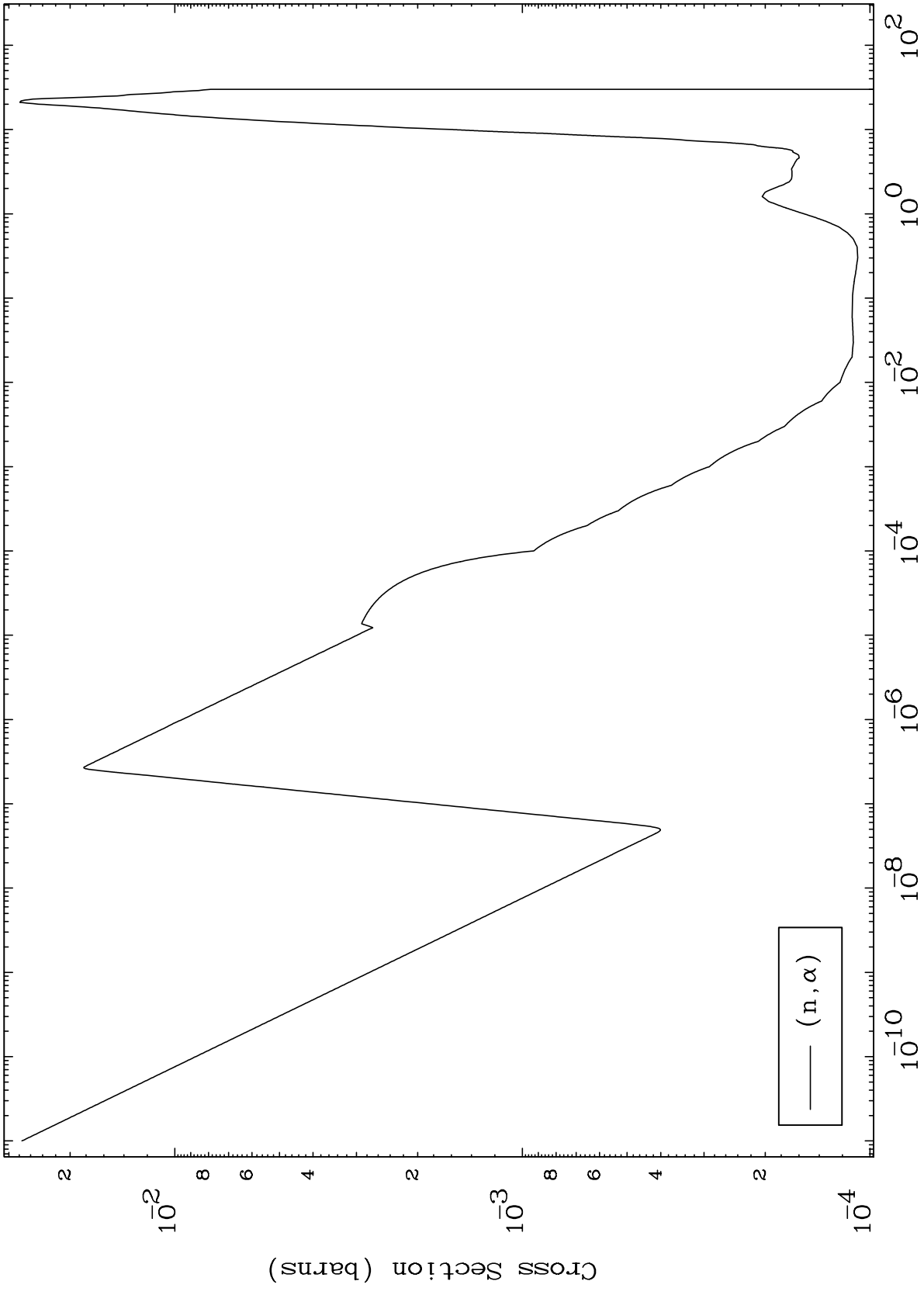
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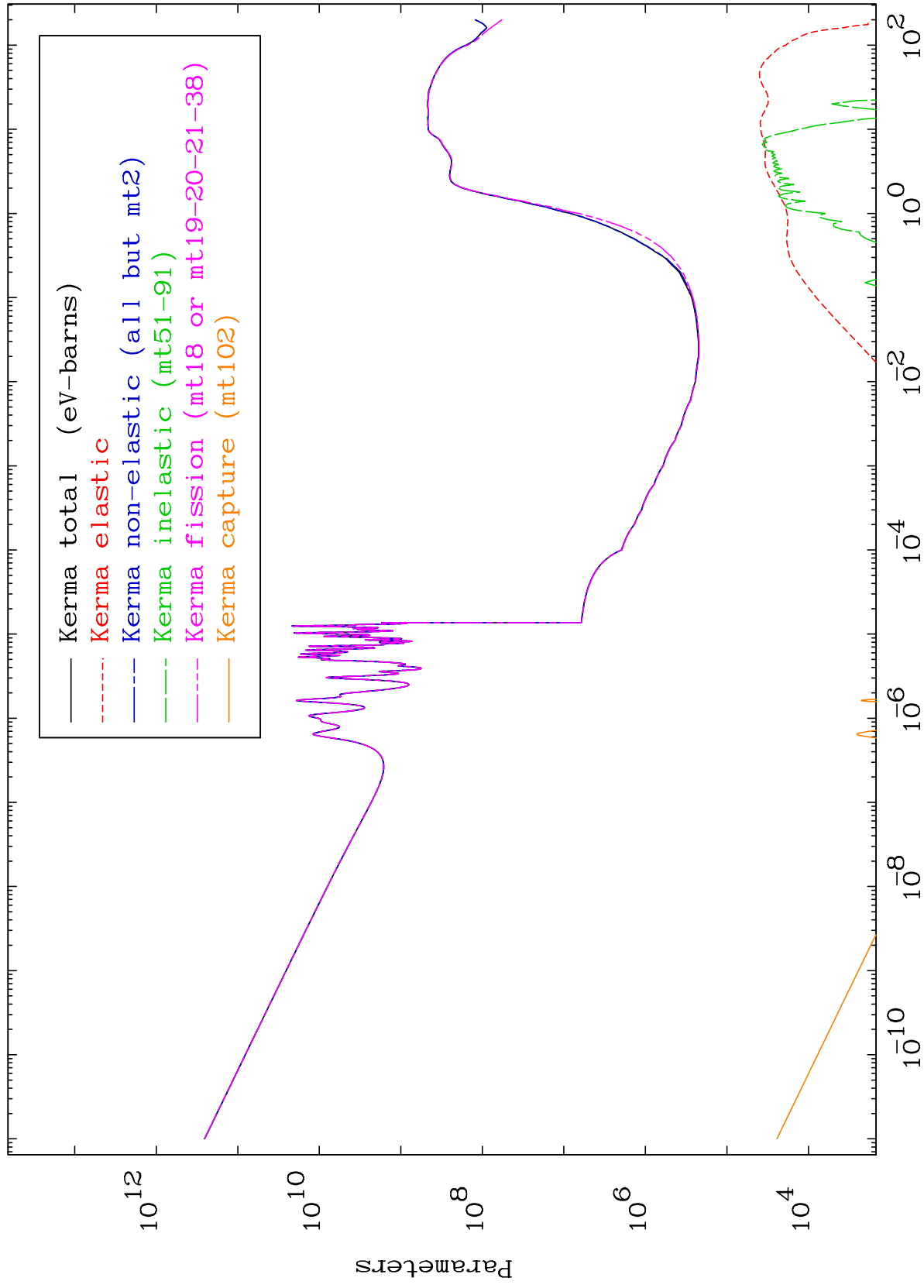


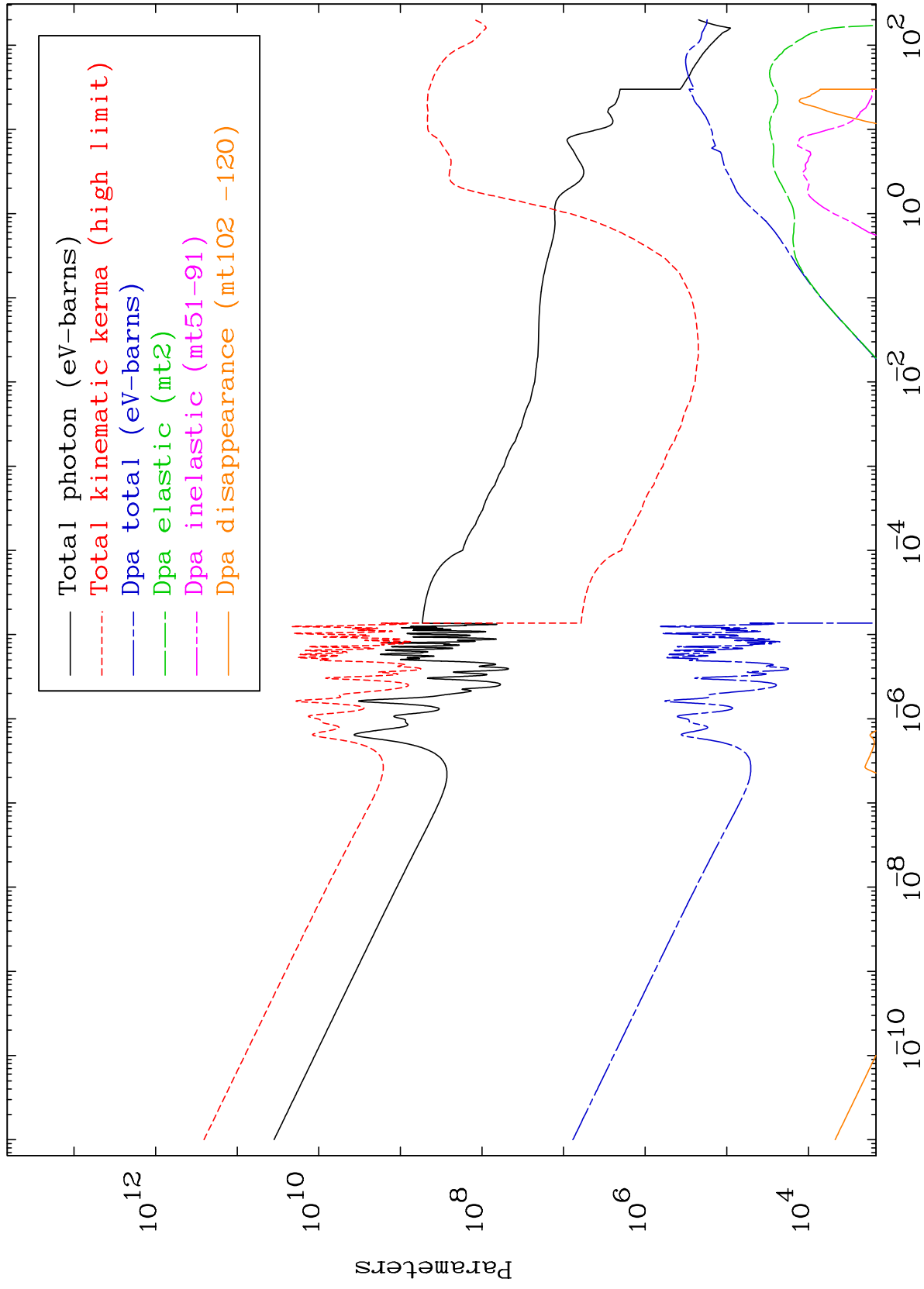
MAT 9948

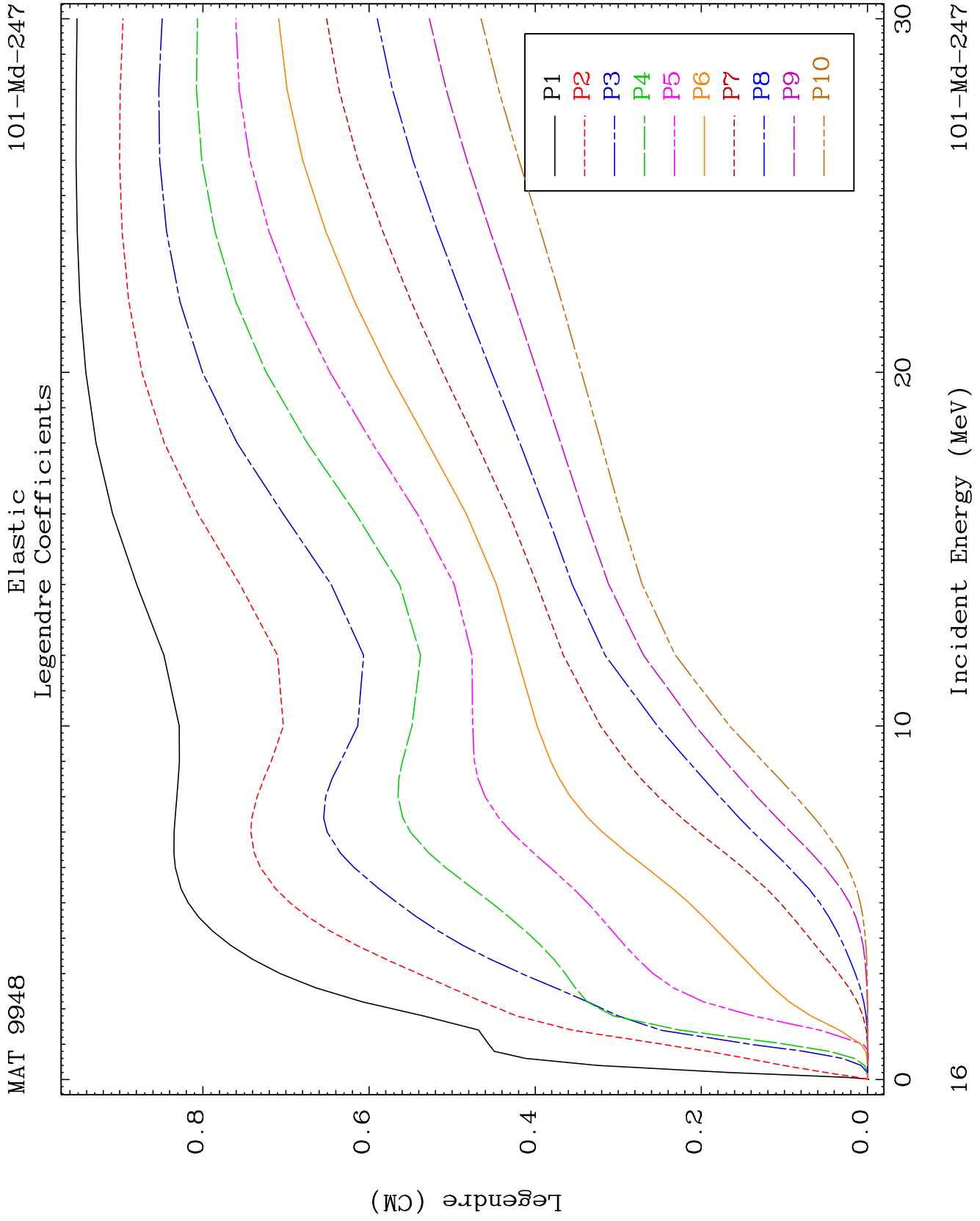
(n,α) Levels
293 Kelvin Cross Sections

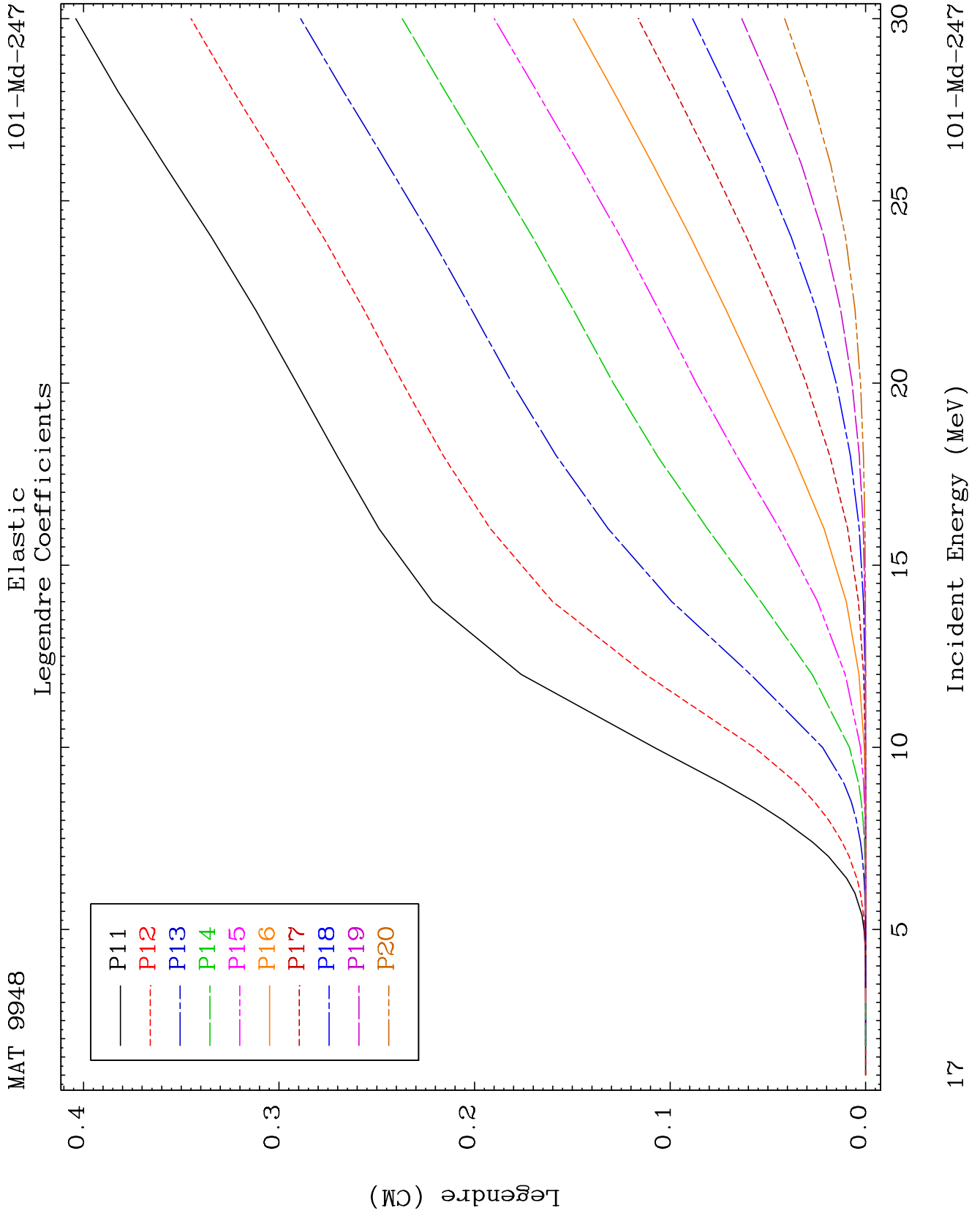
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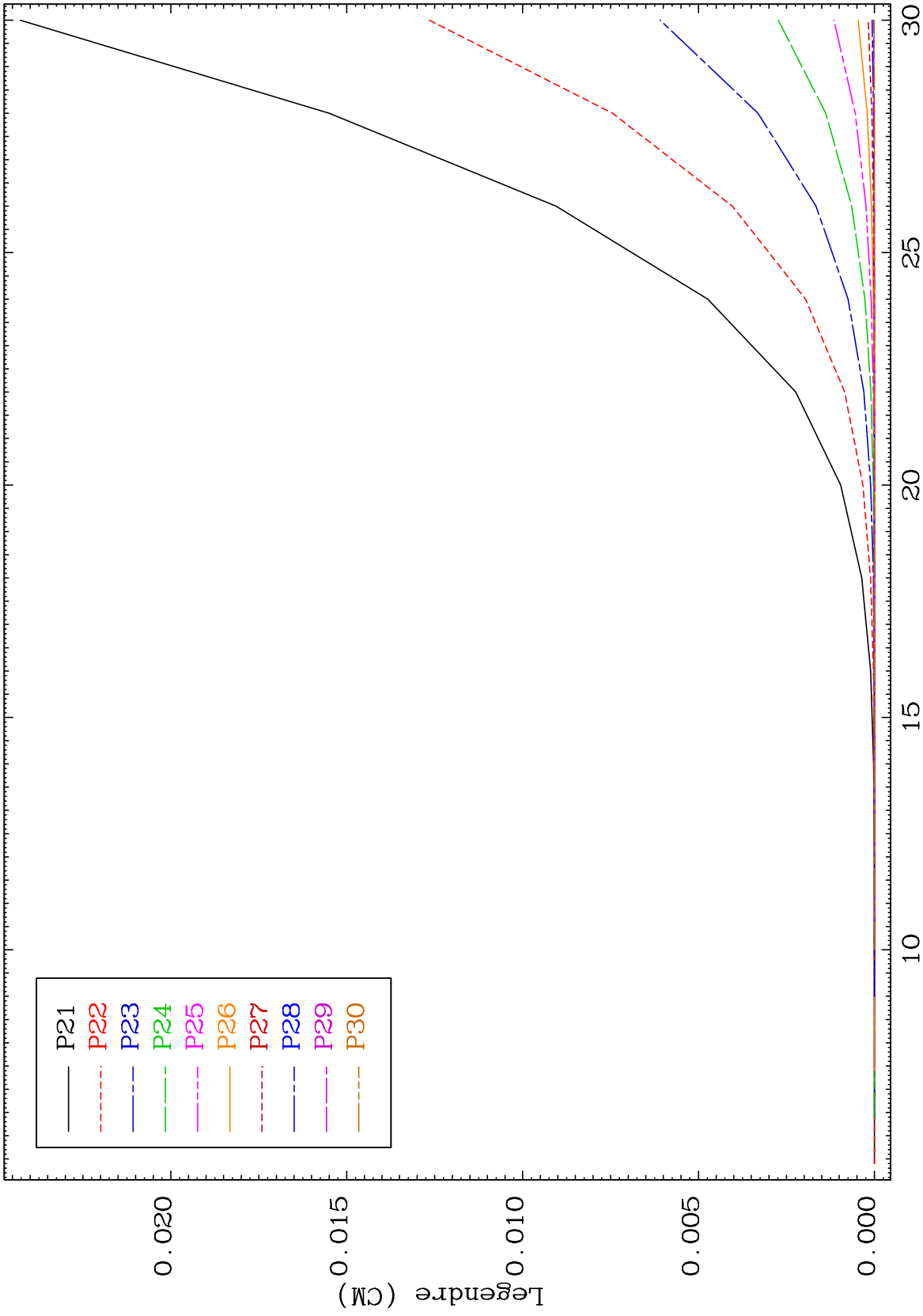




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Elastic Legendre Coefficients

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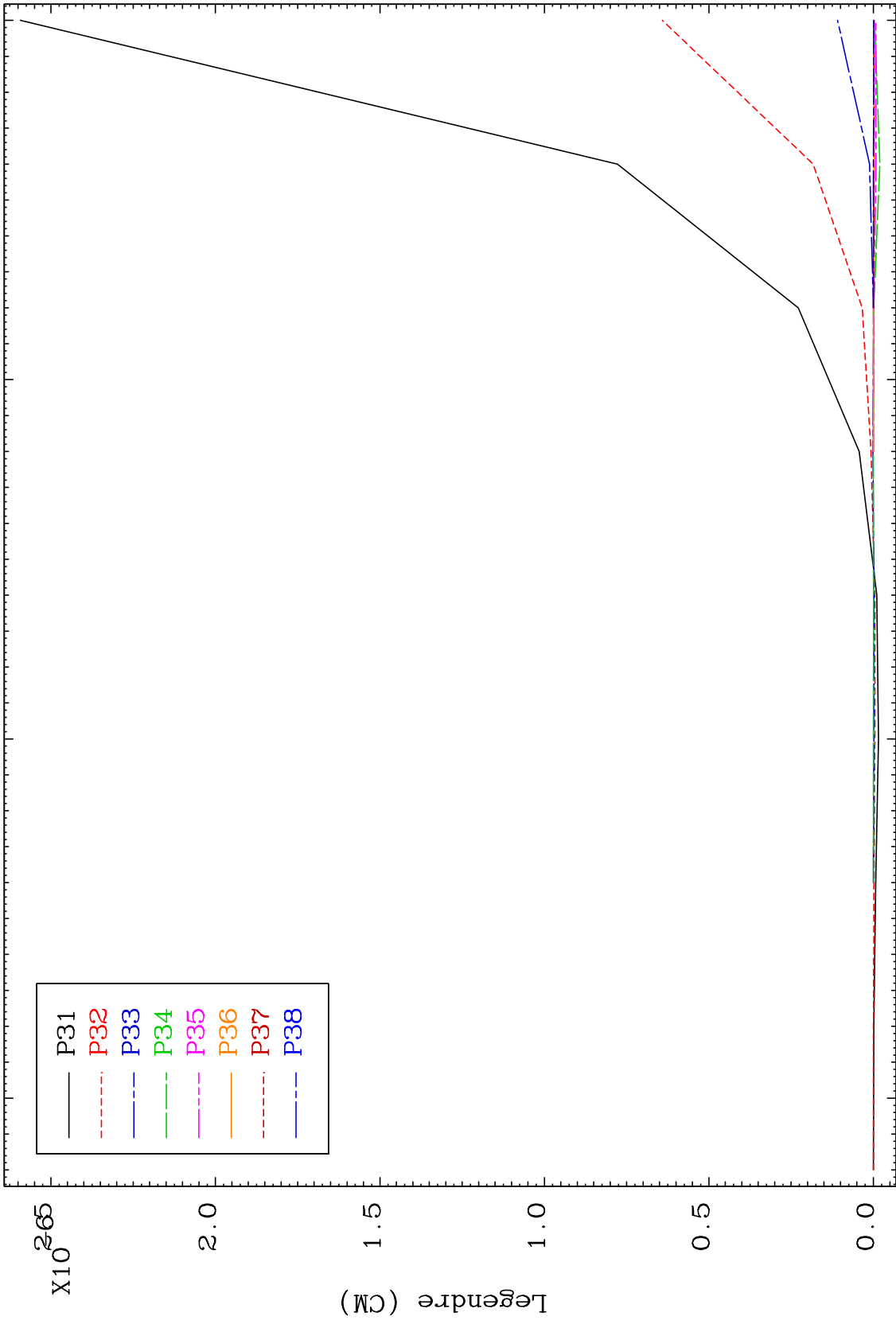
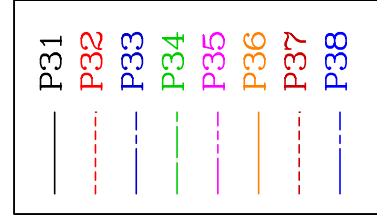
Incident Energy (MeV)

101-Md-247

MAT 9948

Elastic Legendre Coefficients

101-Md-247



19

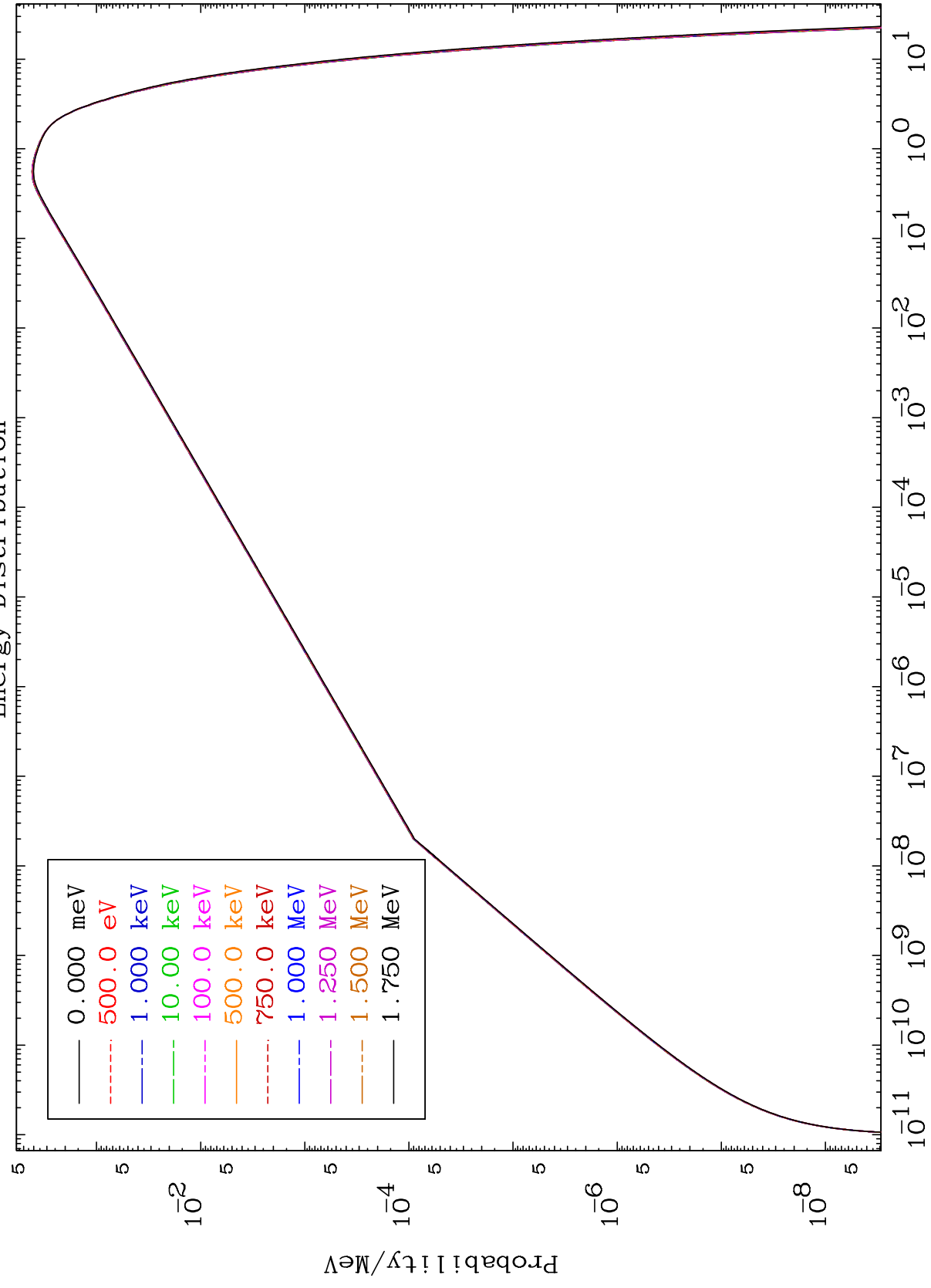
Incident Energy (MeV)

101-Md-247

MAT 9948

Fission Energy Distribution

101-Md-247

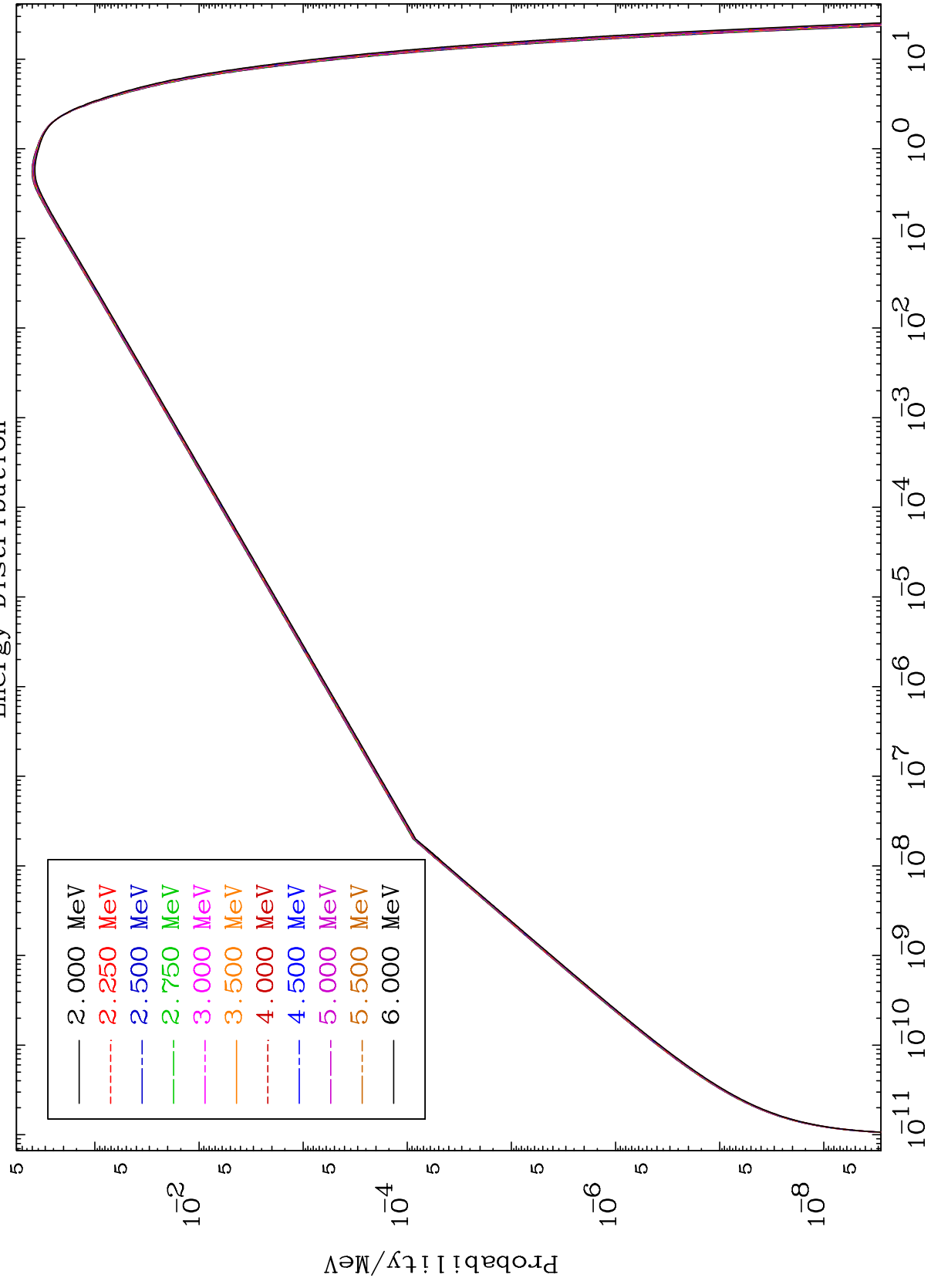


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MAT 9948

Fission Energy Distribution

101-Md-247

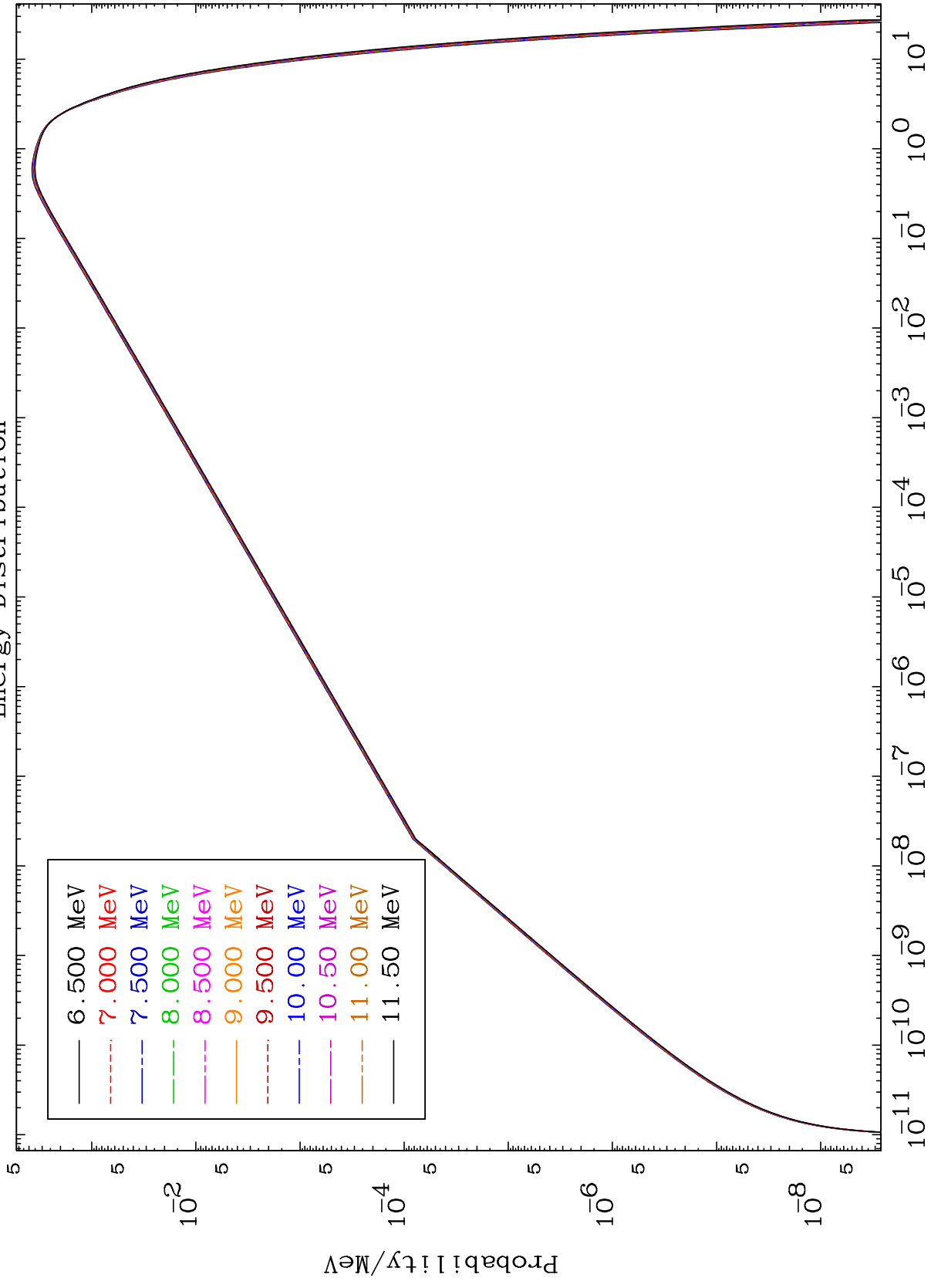


101-Md-247

MAT 9948

Fission Energy Distribution

101-Md-247

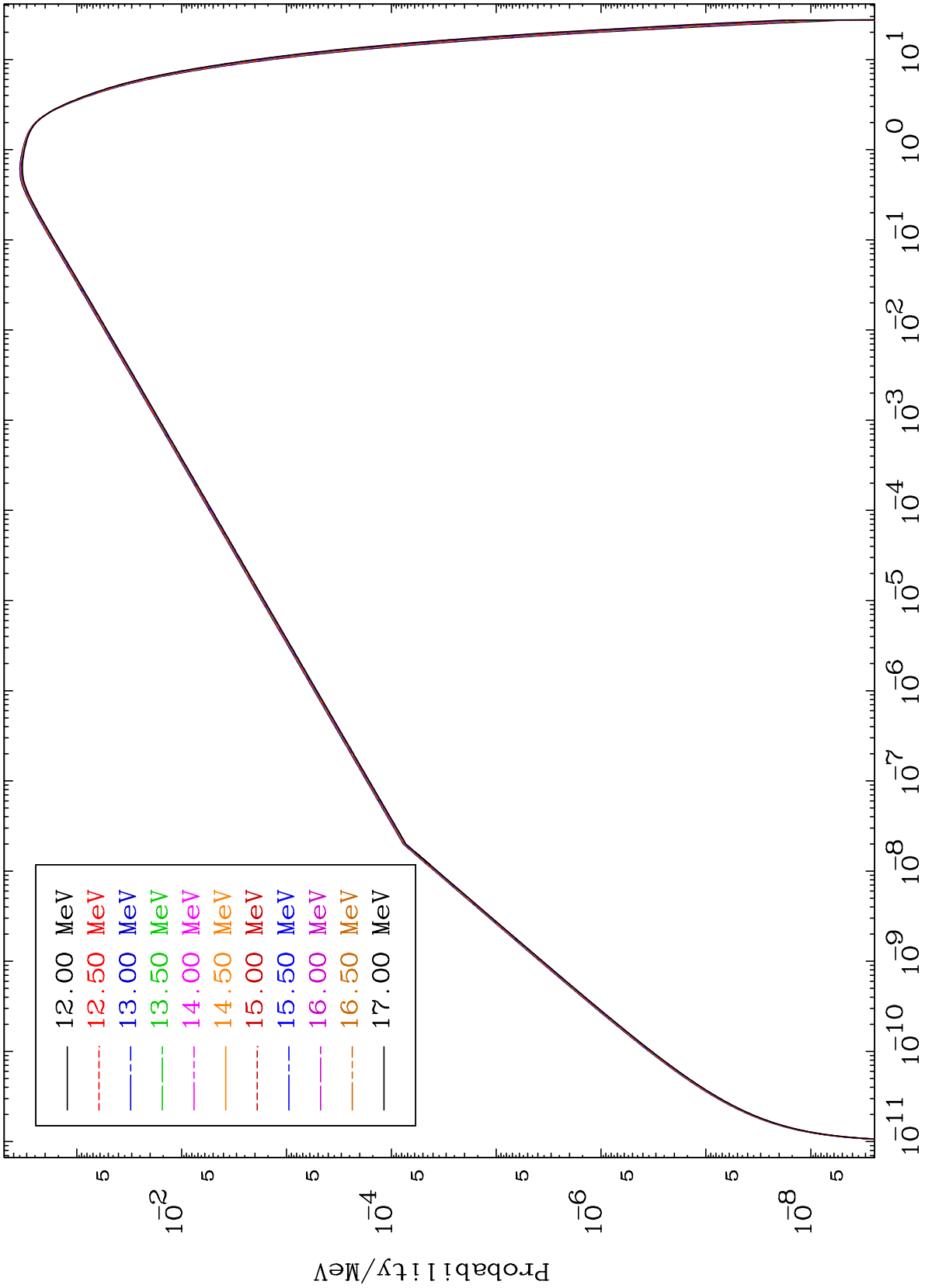


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MAT 9948

Fission Energy Distribution

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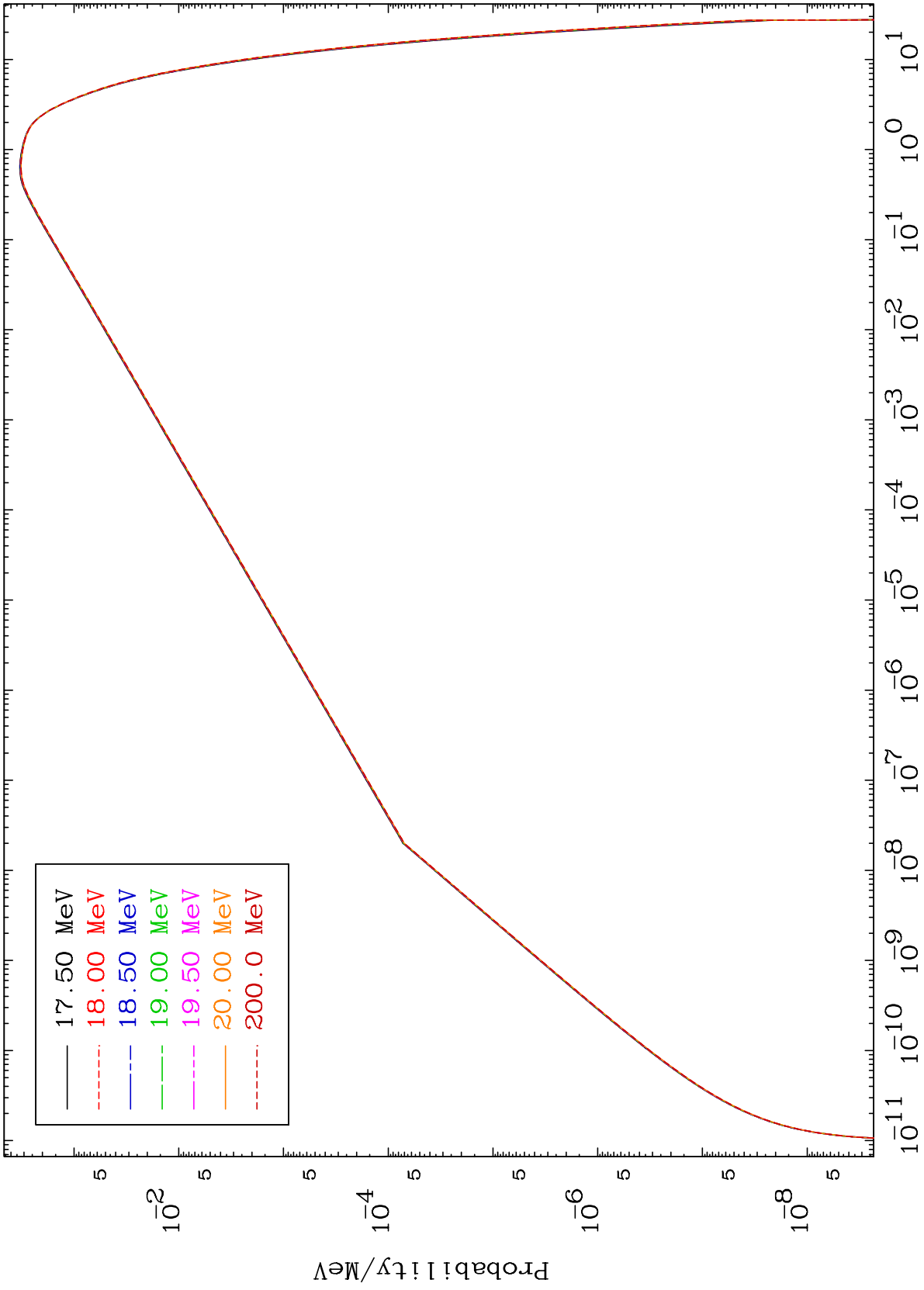


101-Md-247

MAT 9948

Fission Energy Distribution

101-Md-247

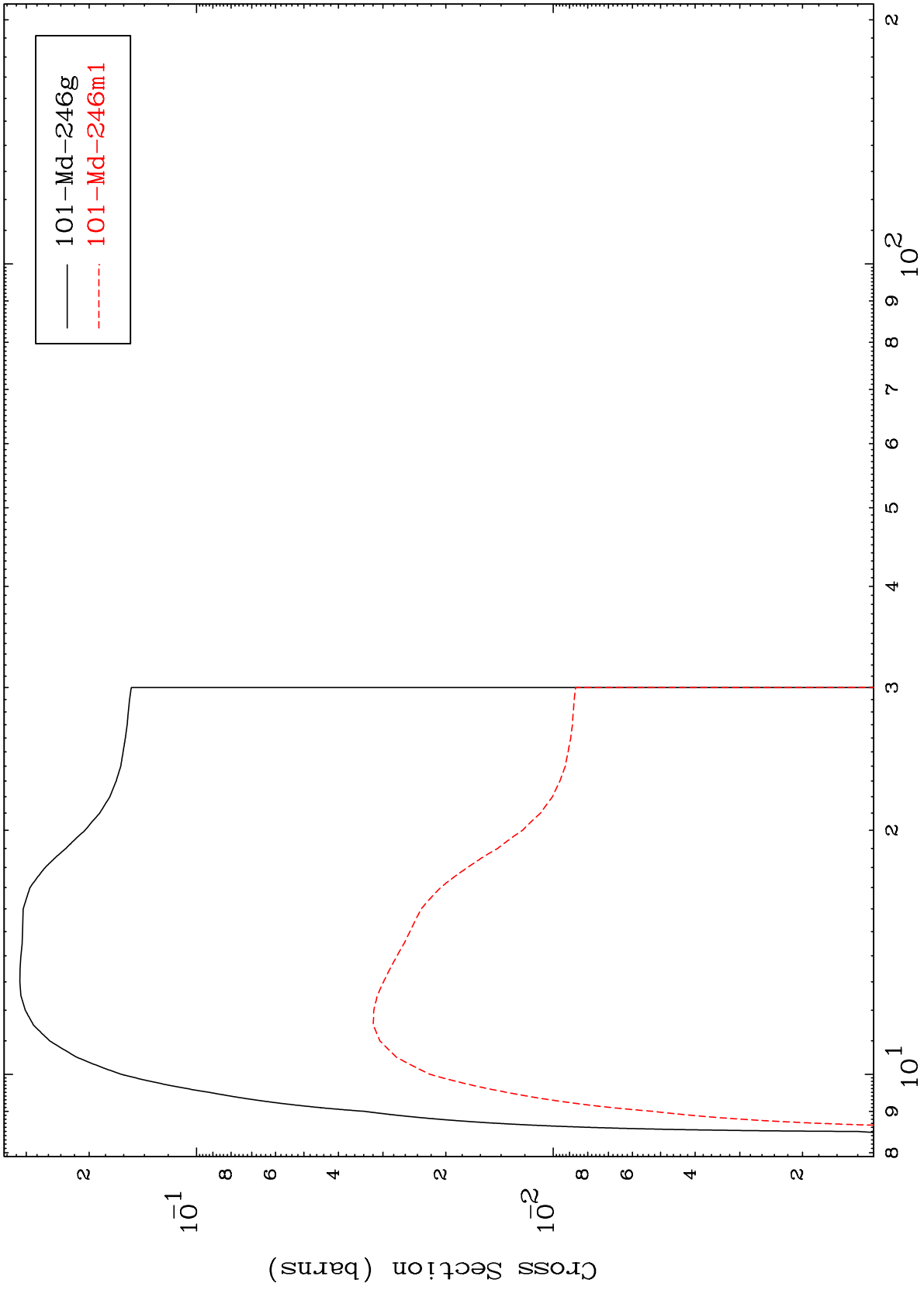


| | | |
|---------|-------|-----|
| — | 17.50 | MeV |
| - - - | 18.00 | MeV |
| - · - · | 18.50 | MeV |
| - - - | 19.00 | MeV |
| - · - · | 19.50 | MeV |
| - - - | 20.00 | MeV |
| - - - | 200.0 | MeV |

MAT 9948

101-Md-247

(n,2n)
Radionuclide Production Cross Section



25

101-Md-247

MAT 9948

Radionuclide Production Cross Section
(n,p)

101-Md-247

