

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550

U.S.A.

Tele: 925-443-1911

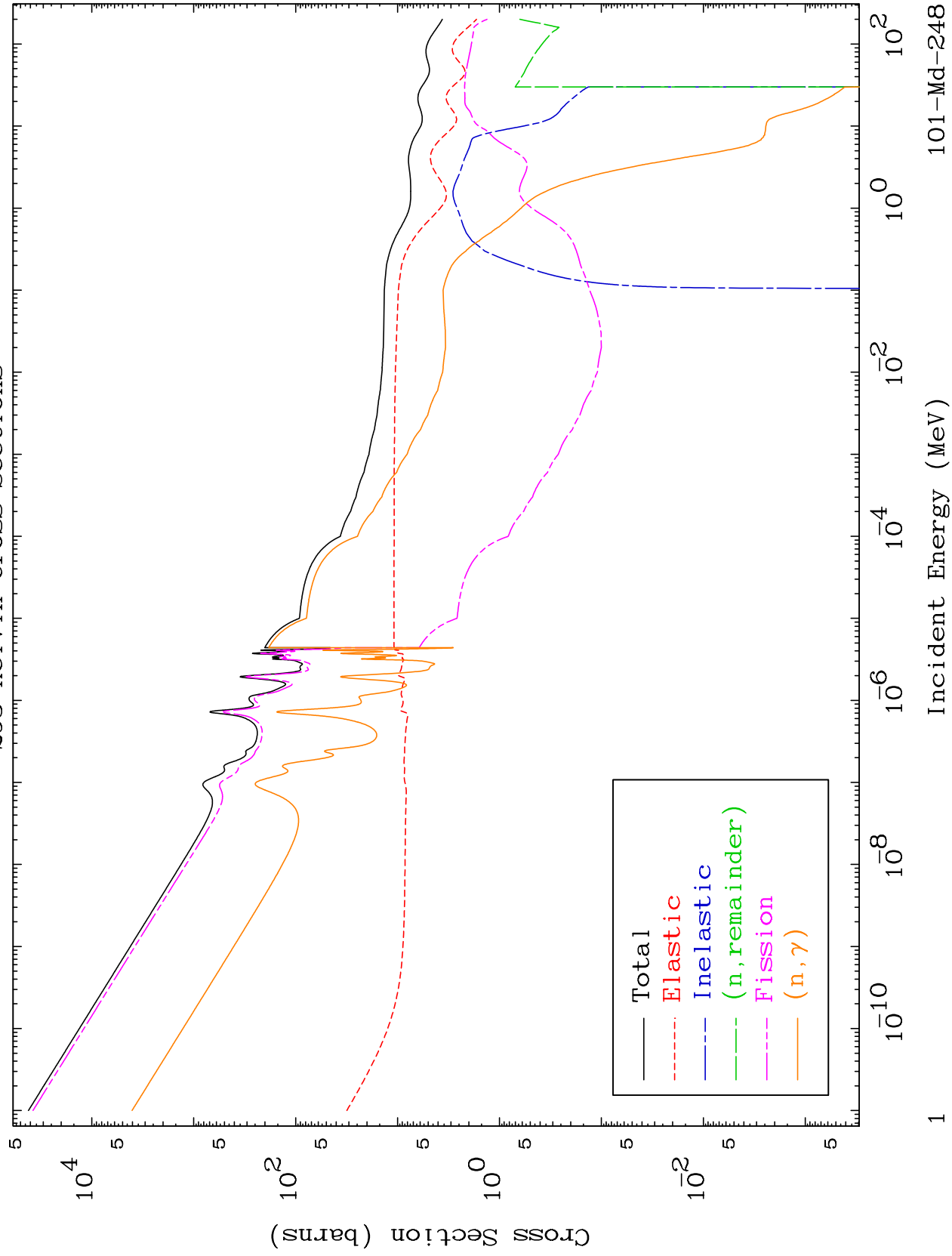
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9949

Major
293 Kelvin Cross Sections

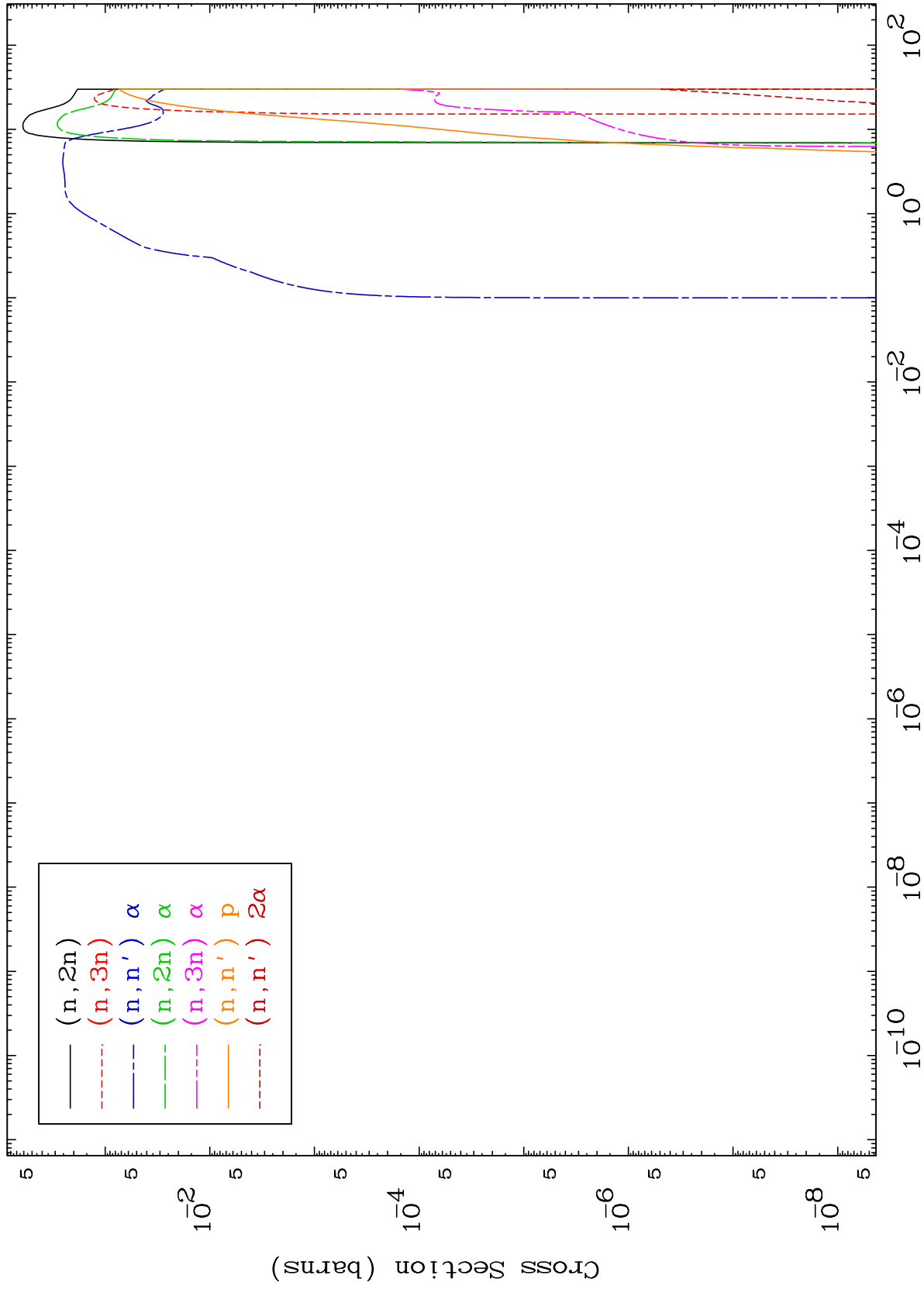
101-Md-248



MAT 9949

Neutron Production
293 Kelvin Cross Sections

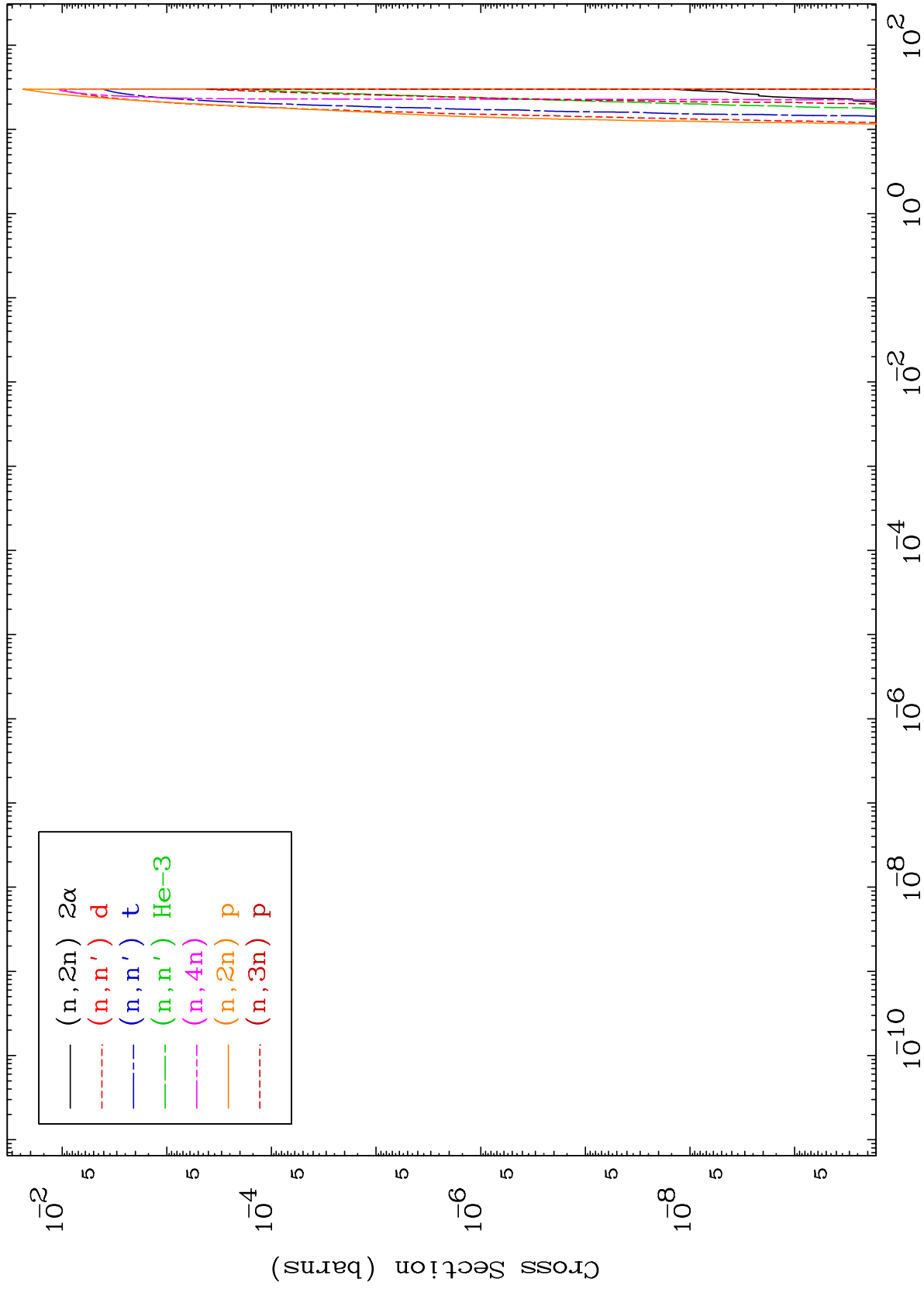
101-Md-248



MAT 9949

Neutron Production
293 Kelvin Cross Sections

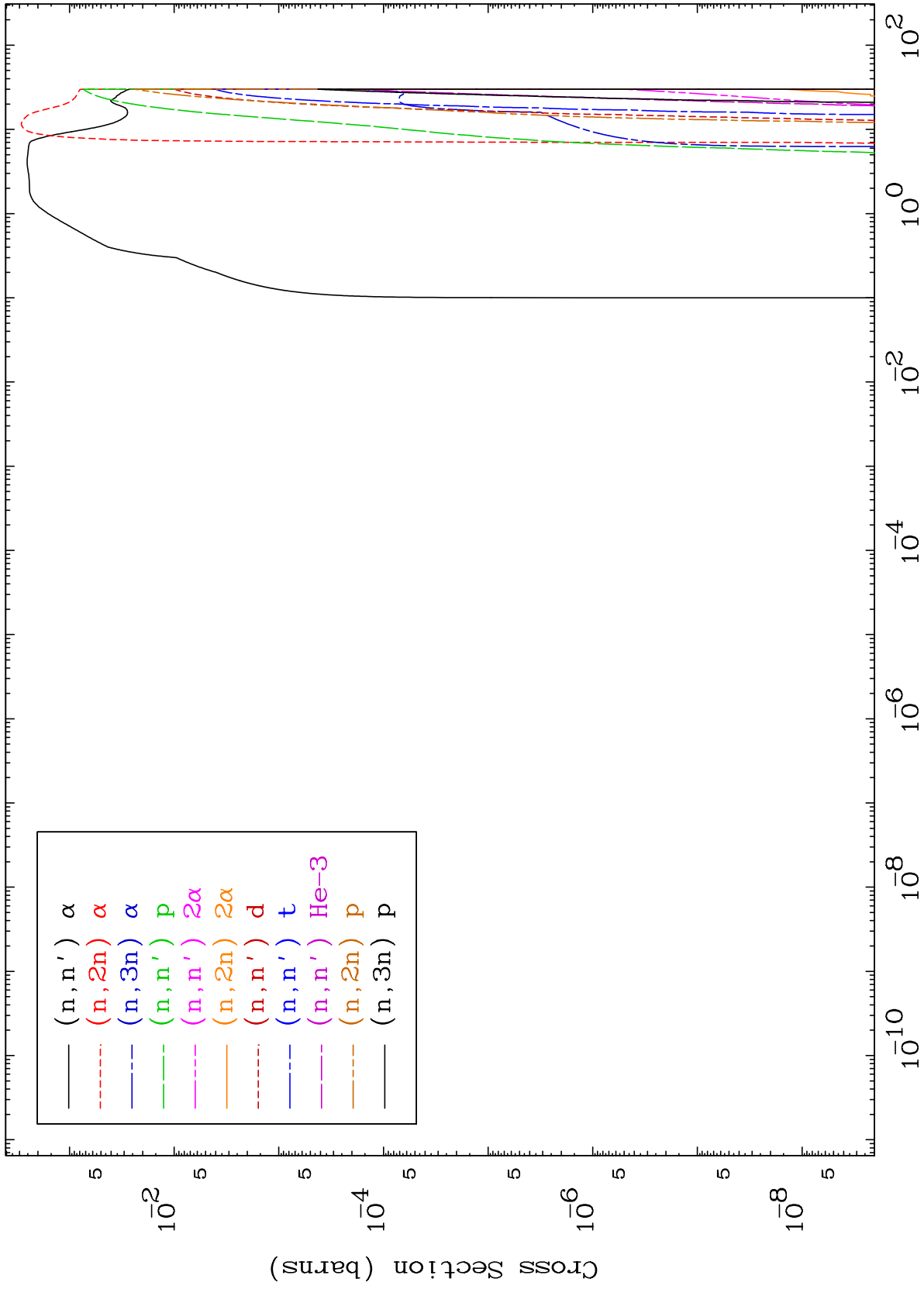
101-Md-248



MAT 9949

Charged Particle
293 Kelvin Cross Sections

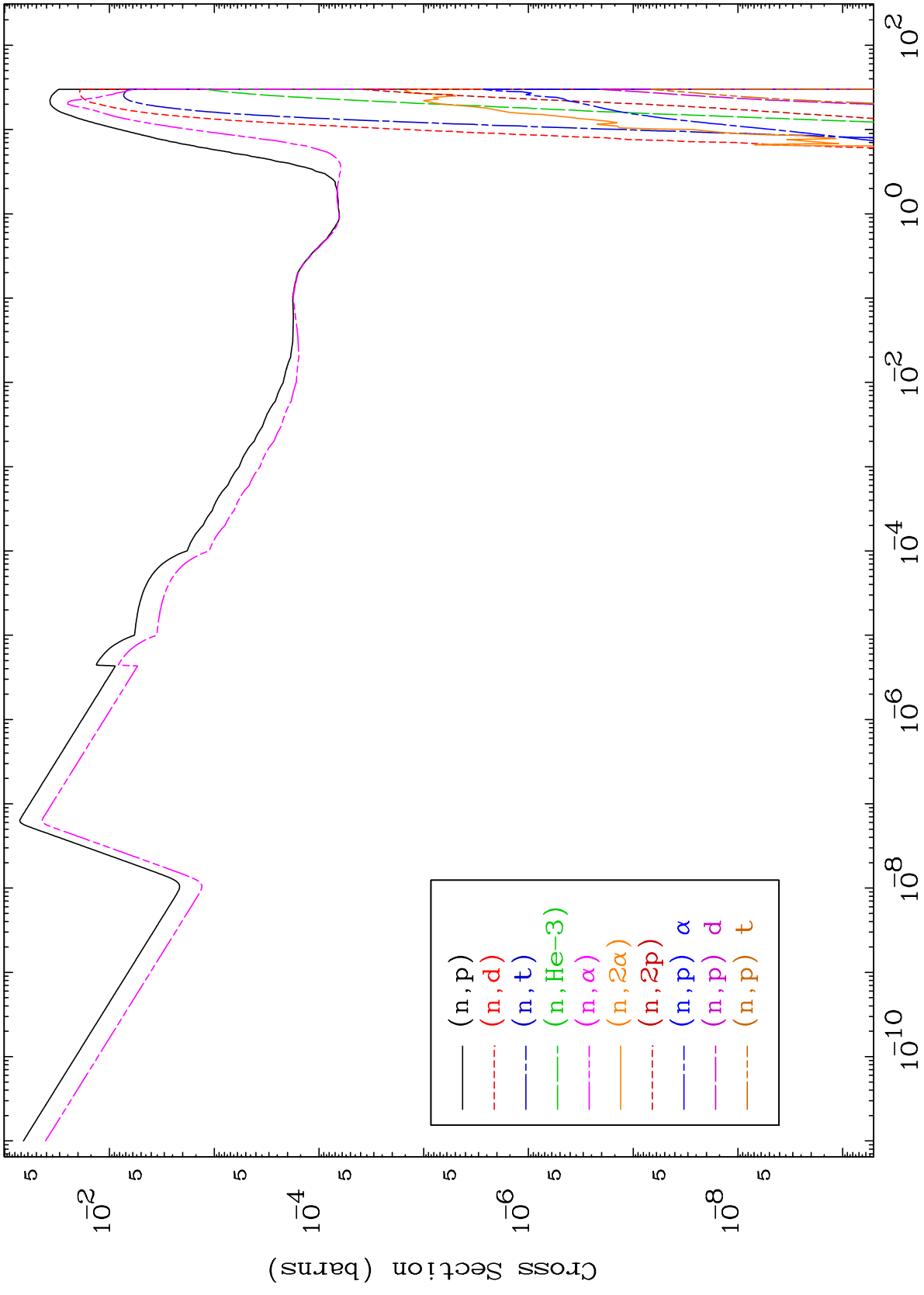
101-Md-248



MAT 9949

293 Kelvin Cross Sections

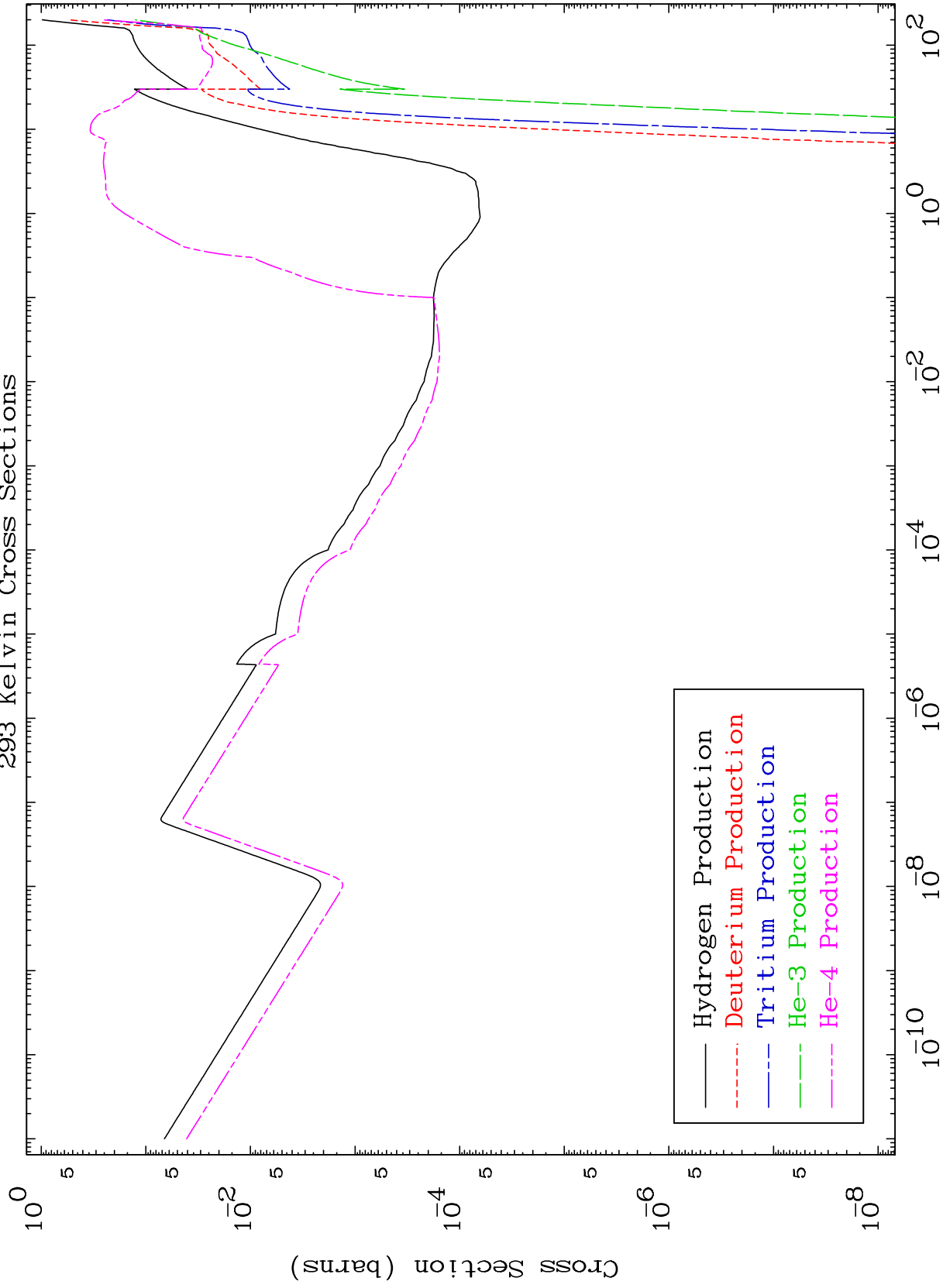
101-Md-248



MAT 9949

Particle Production
293 Kelvin Cross Sections

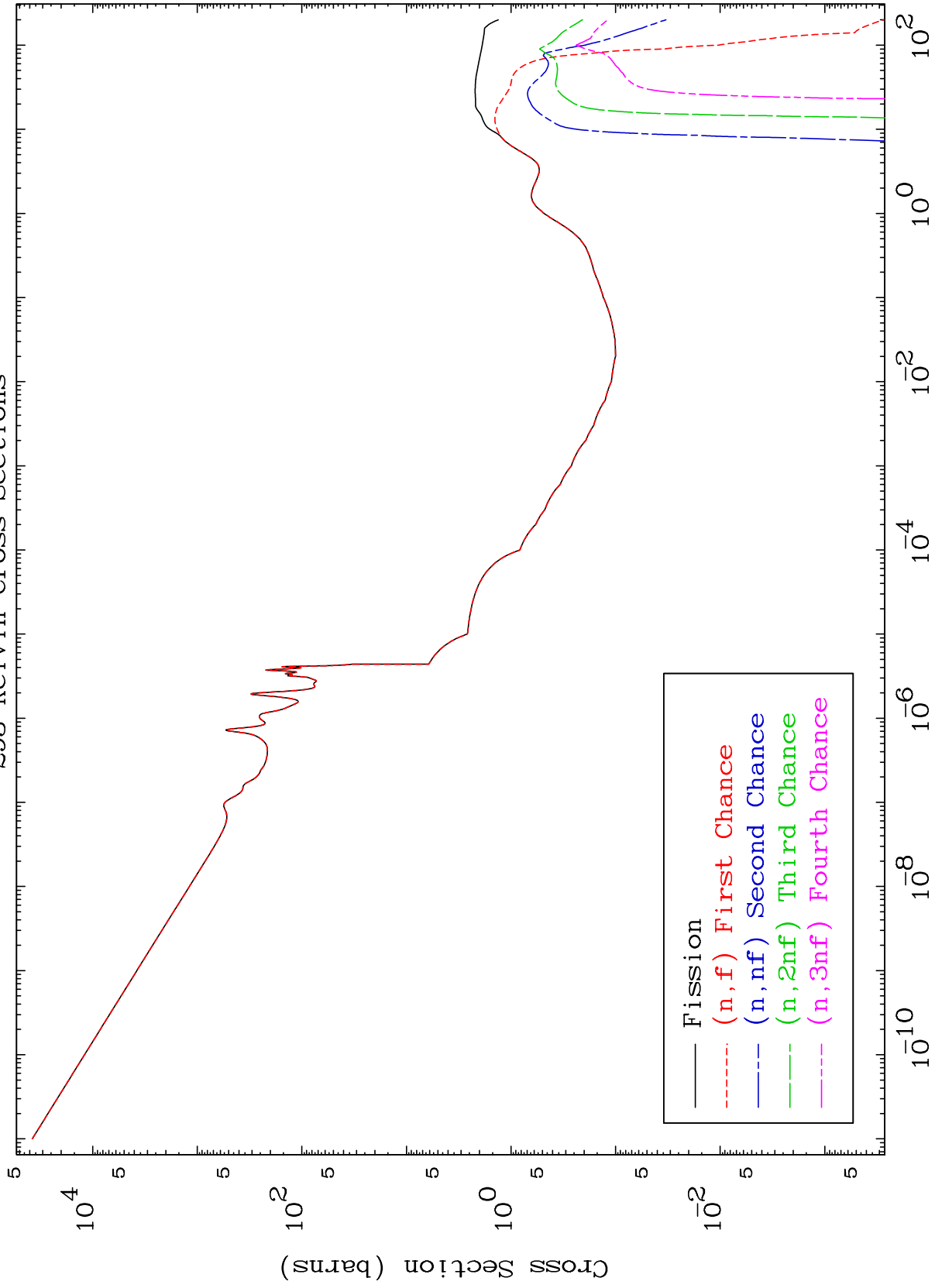
101-Md-248



MAT 9949

Fission
293 Kelvin Cross Sections

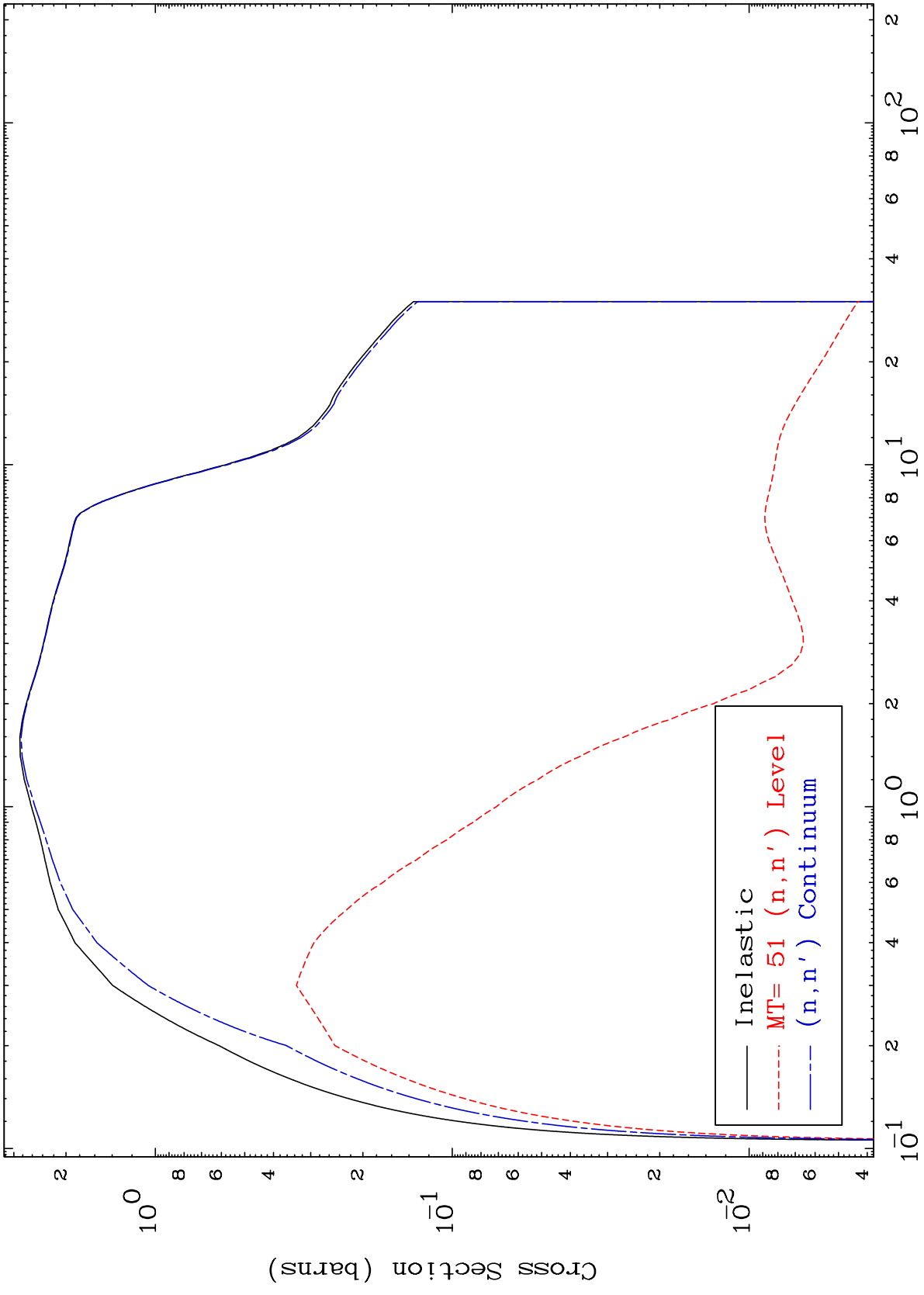
101-Md-248



MAT 9949

(n,n') Level
293 Kelvin Cross Sections

101-Md-248



8

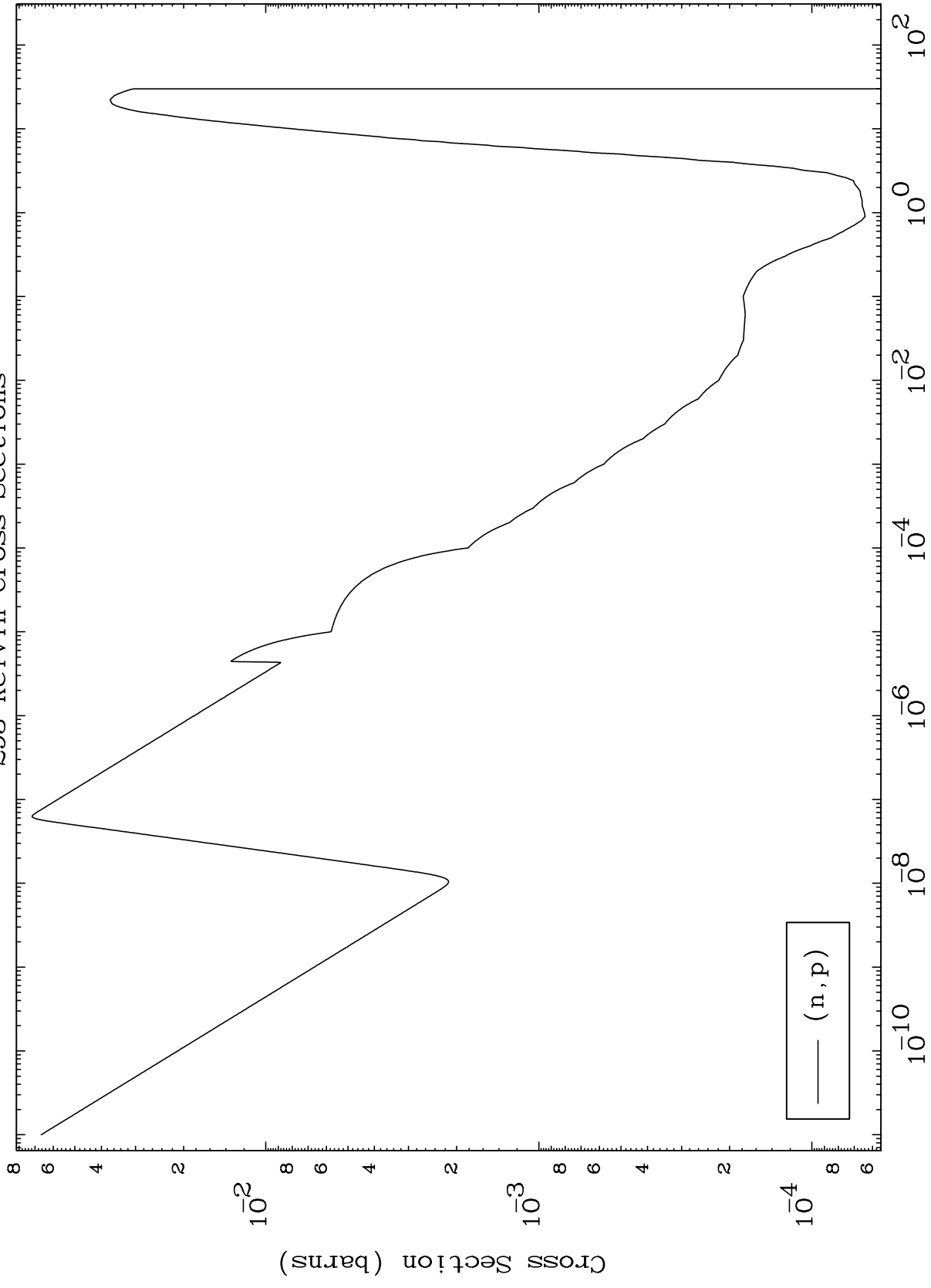
Incident Energy (MeV)

101-Md-248

MAT 9949

(n,p) Levels
293 Kelvin Cross Sections

101-Md-248

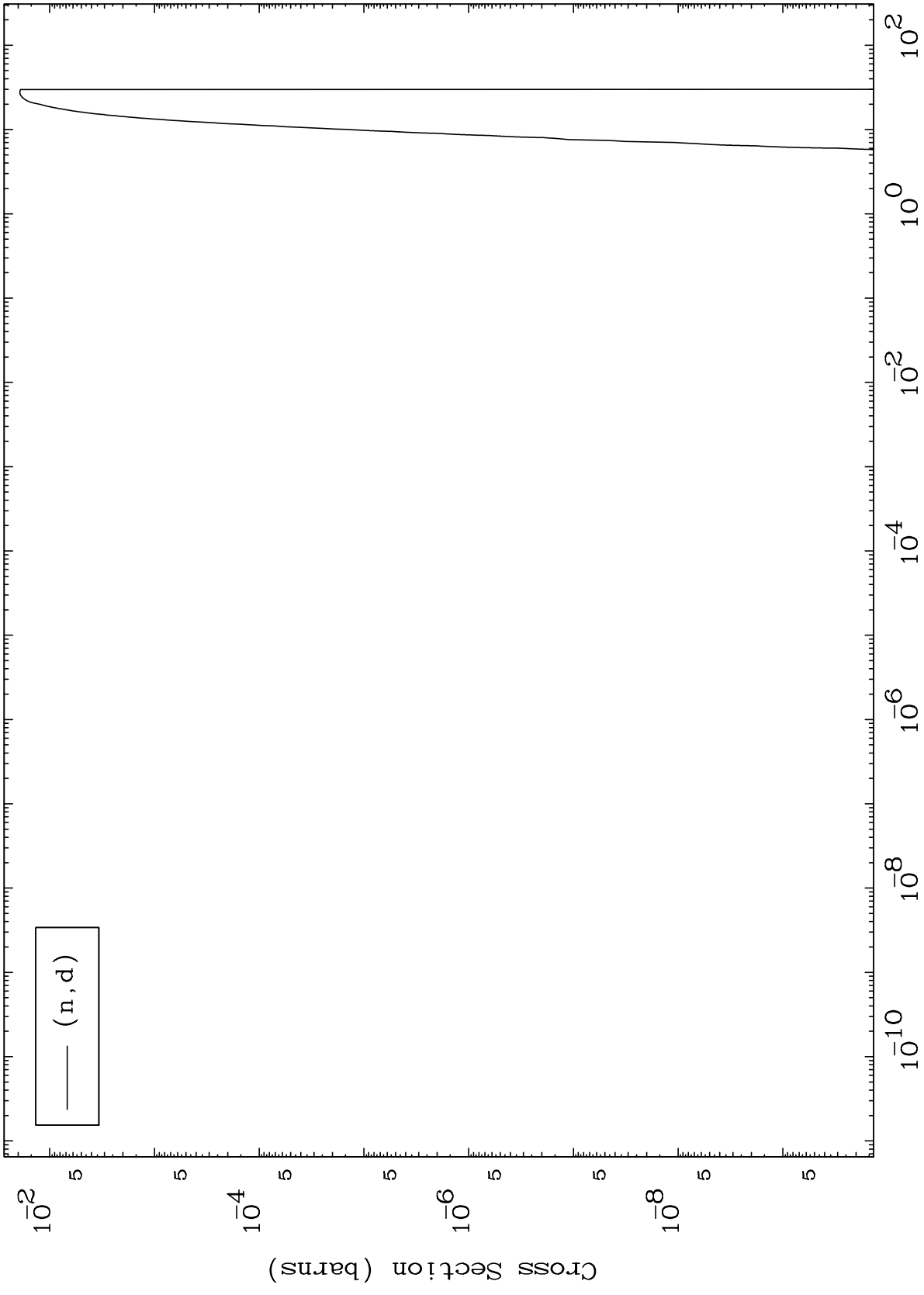


101-Md-248

MAT 9949

(n,d) Levels
293 Kelvin Cross Sections

101-Md-248



10

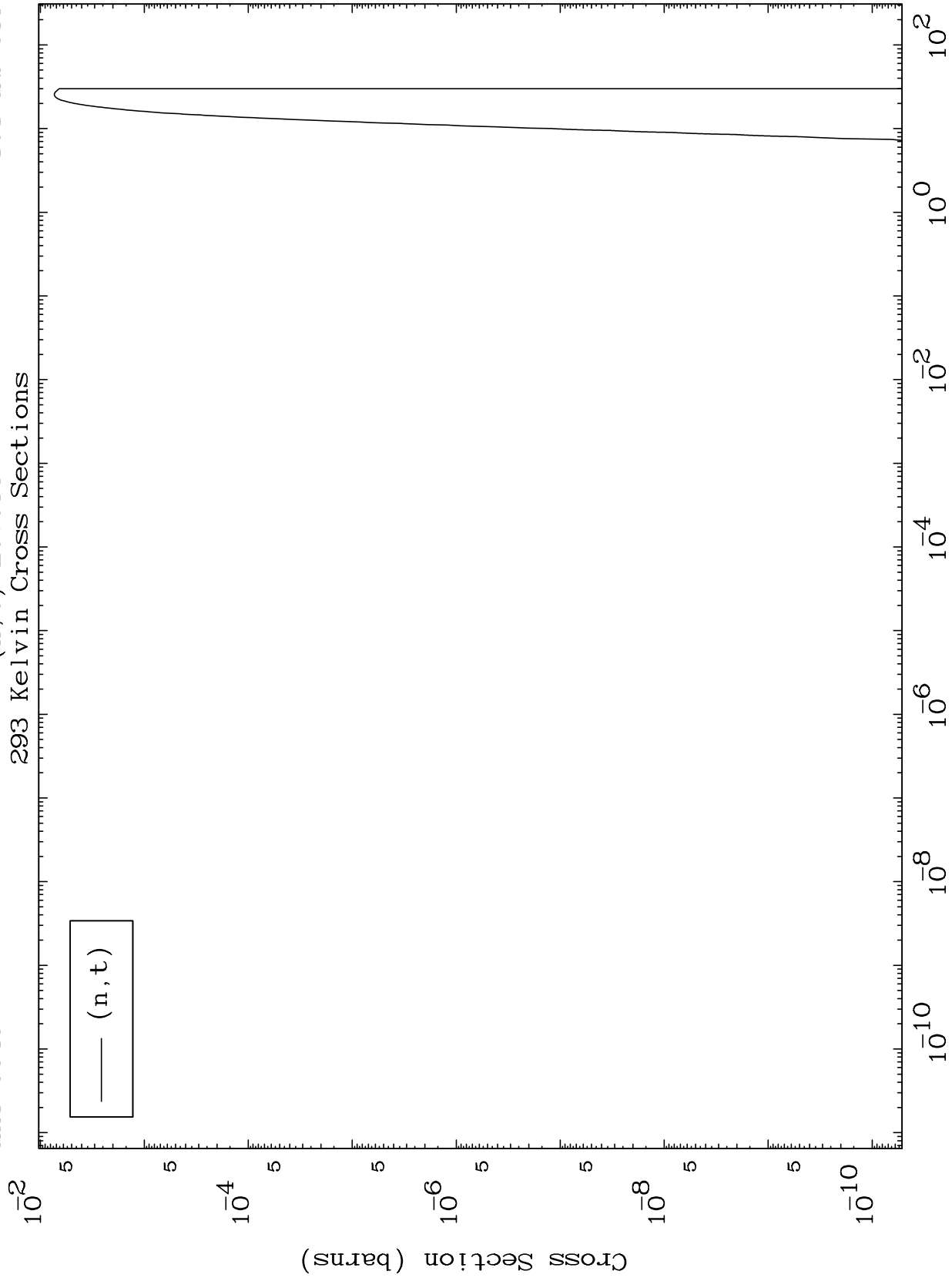
Incident Energy (MeV)

101-Md-248

MAT 9949

(n,t) Levels
293 Kelvin Cross Sections

101-Md-248

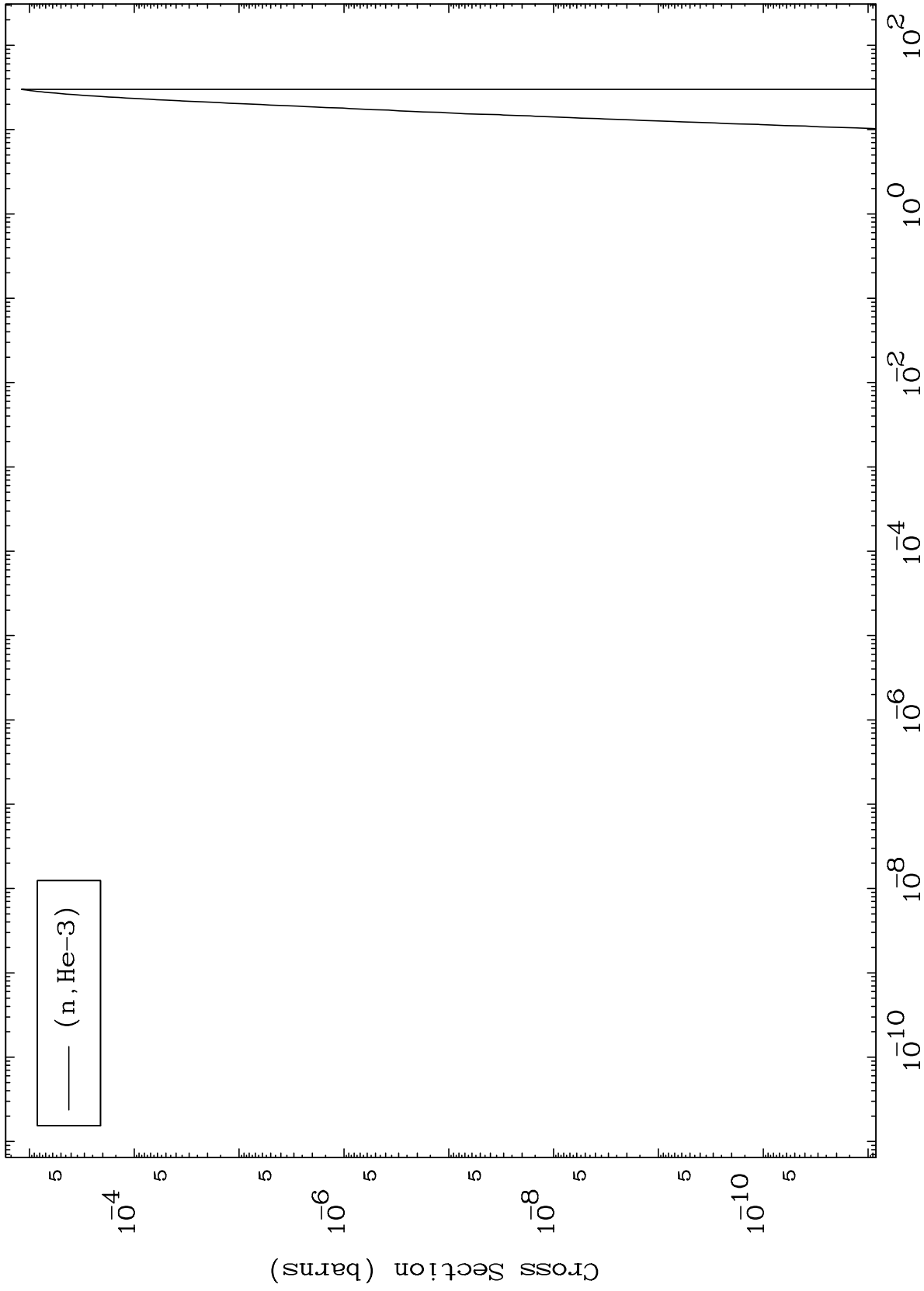


101-Md-248

MAT 9949

(n,He3) Levels
293 Kelvin Cross Sections

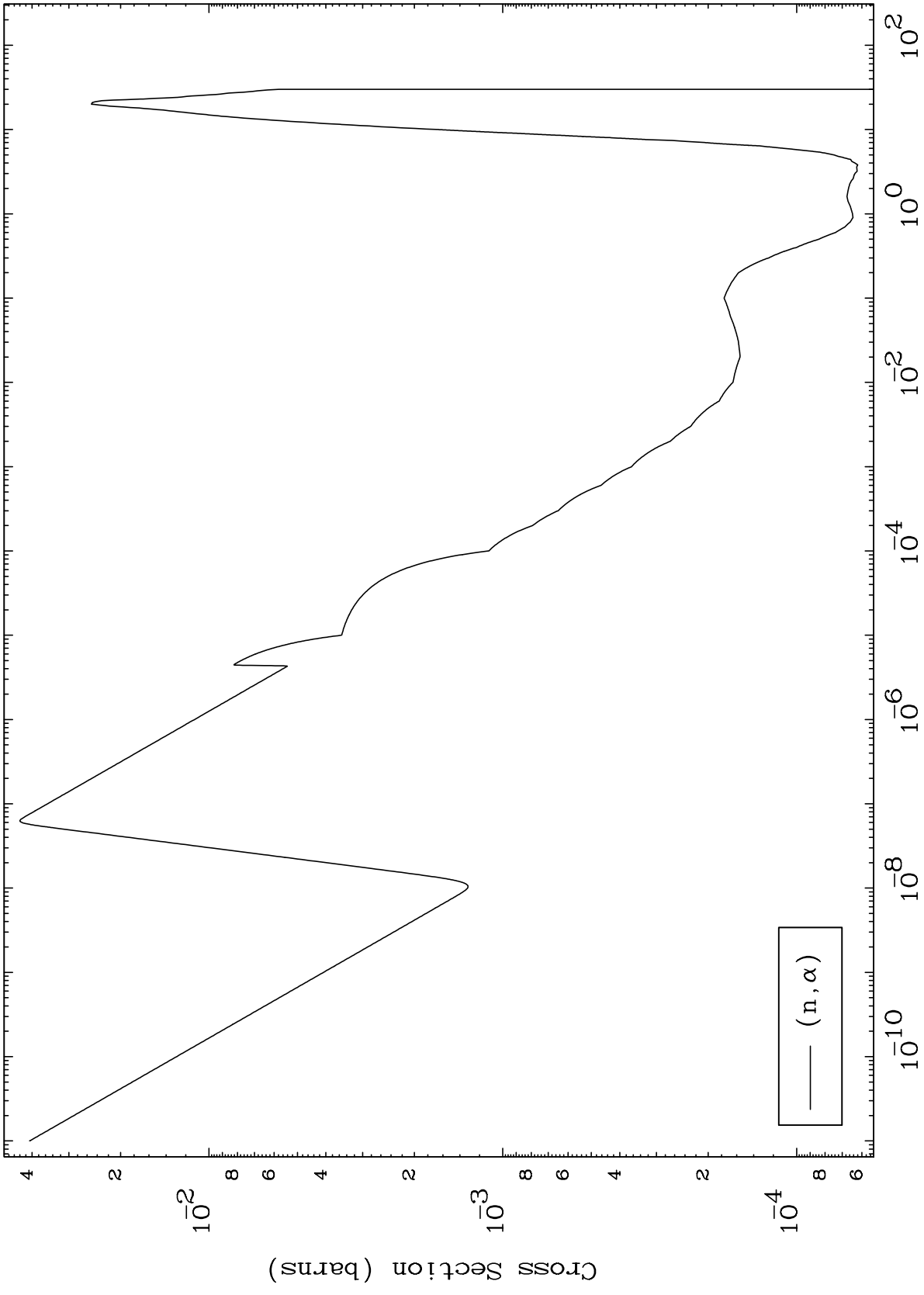
101-Md-248



MAT 9949

(n,α) Levels
293 Kelvin Cross Sections

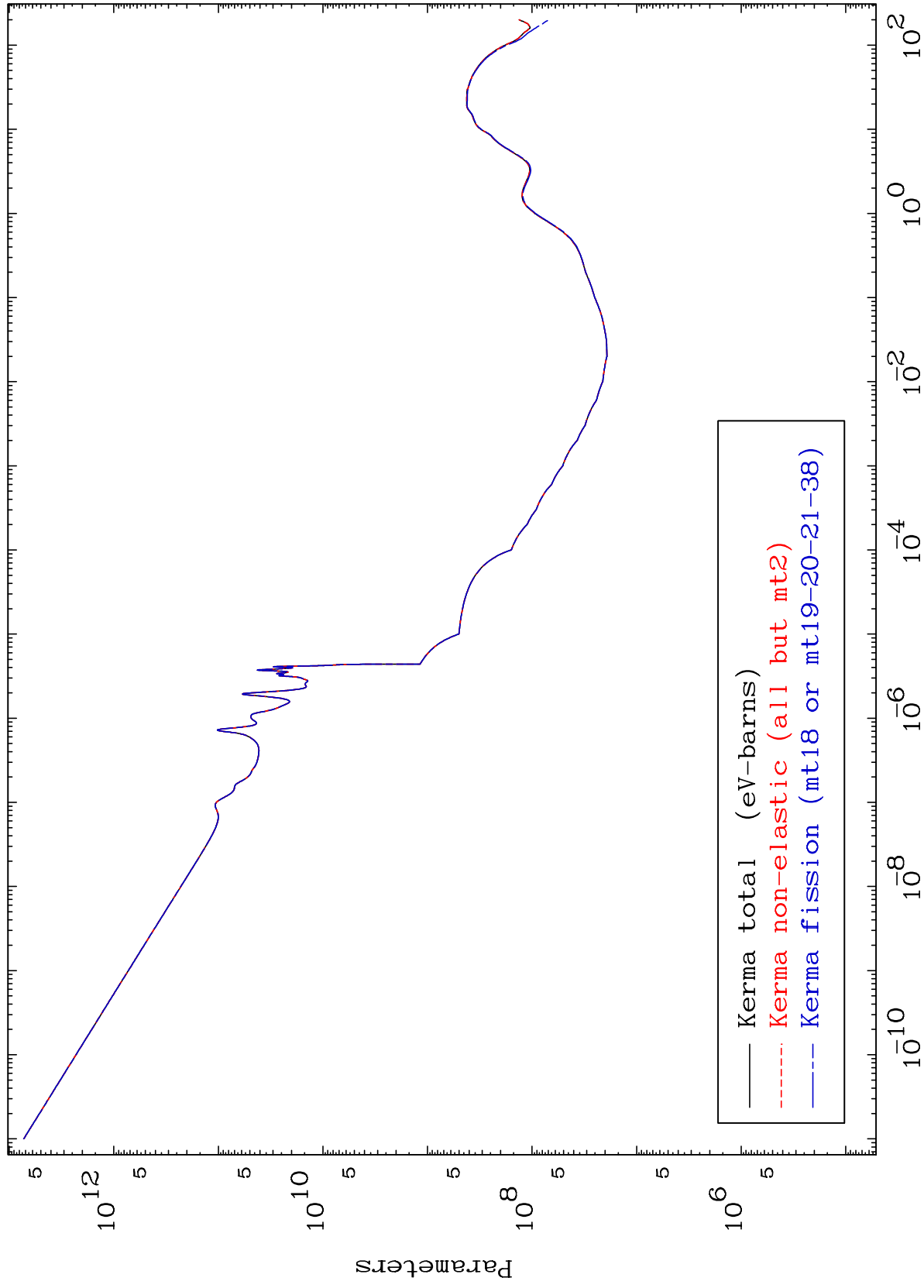
101-Md-248



MAT 9949

Energy Release
Parameters

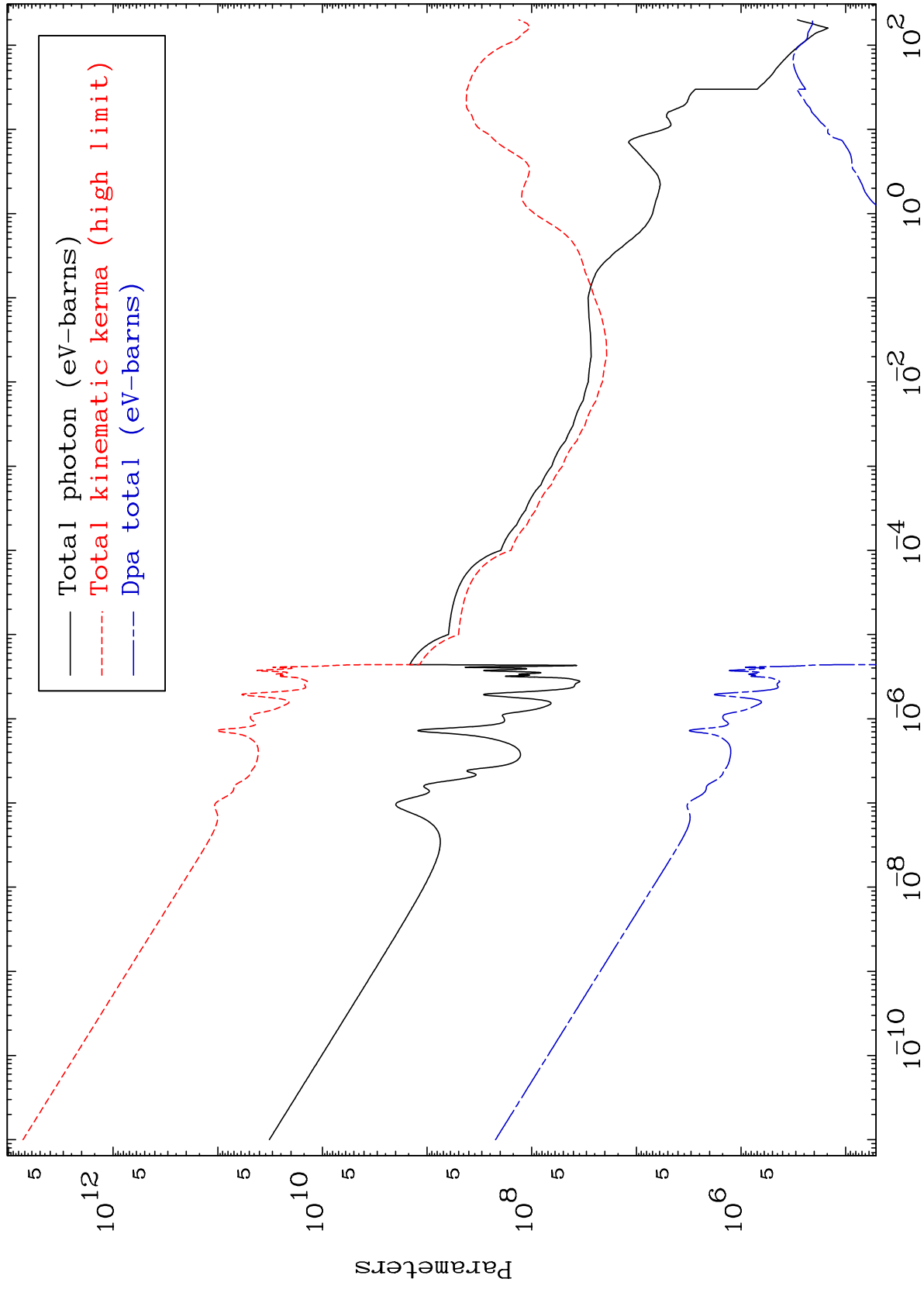
101-Md-248



MAT 9949

Energy Release
Parameters

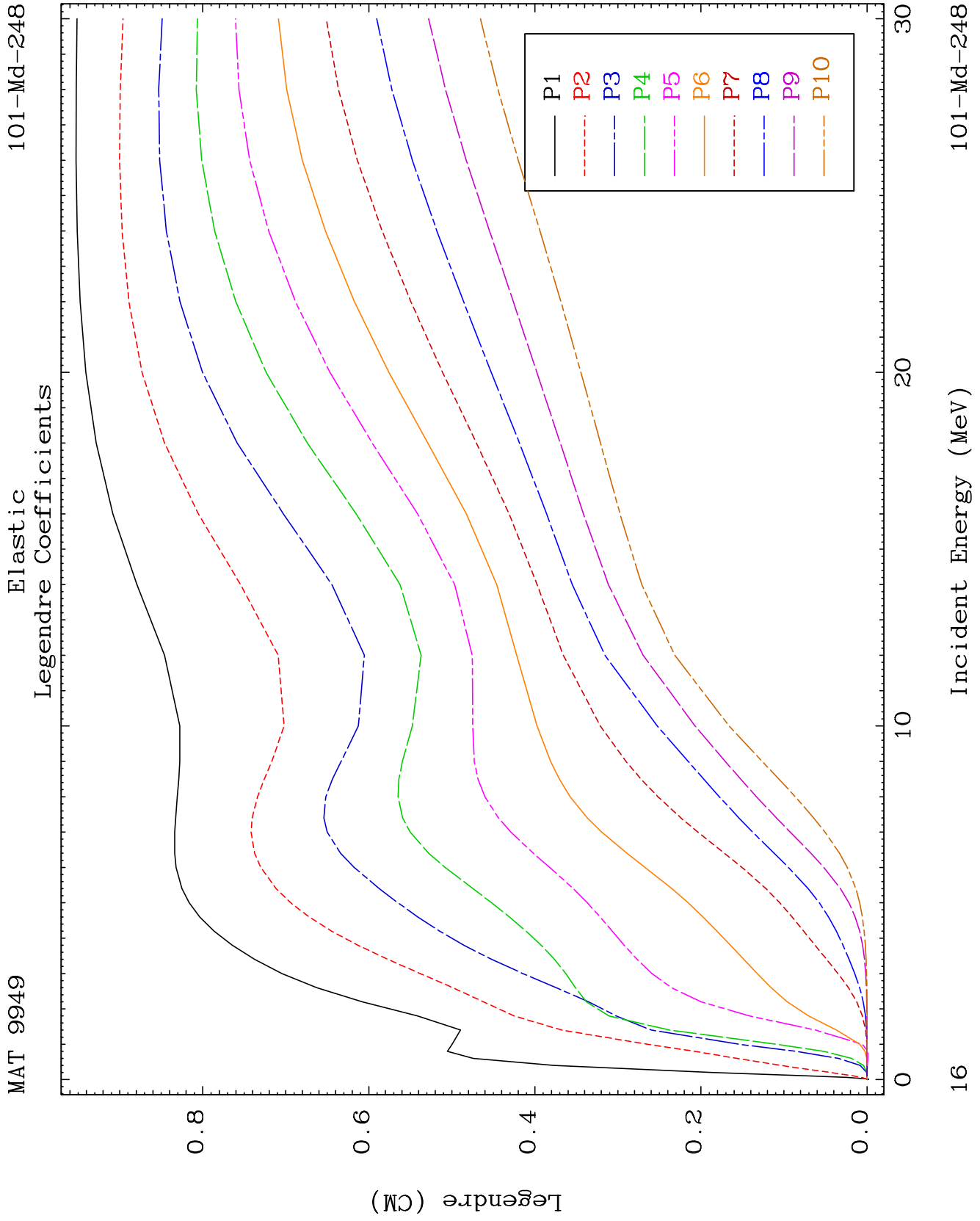
101-Md-248

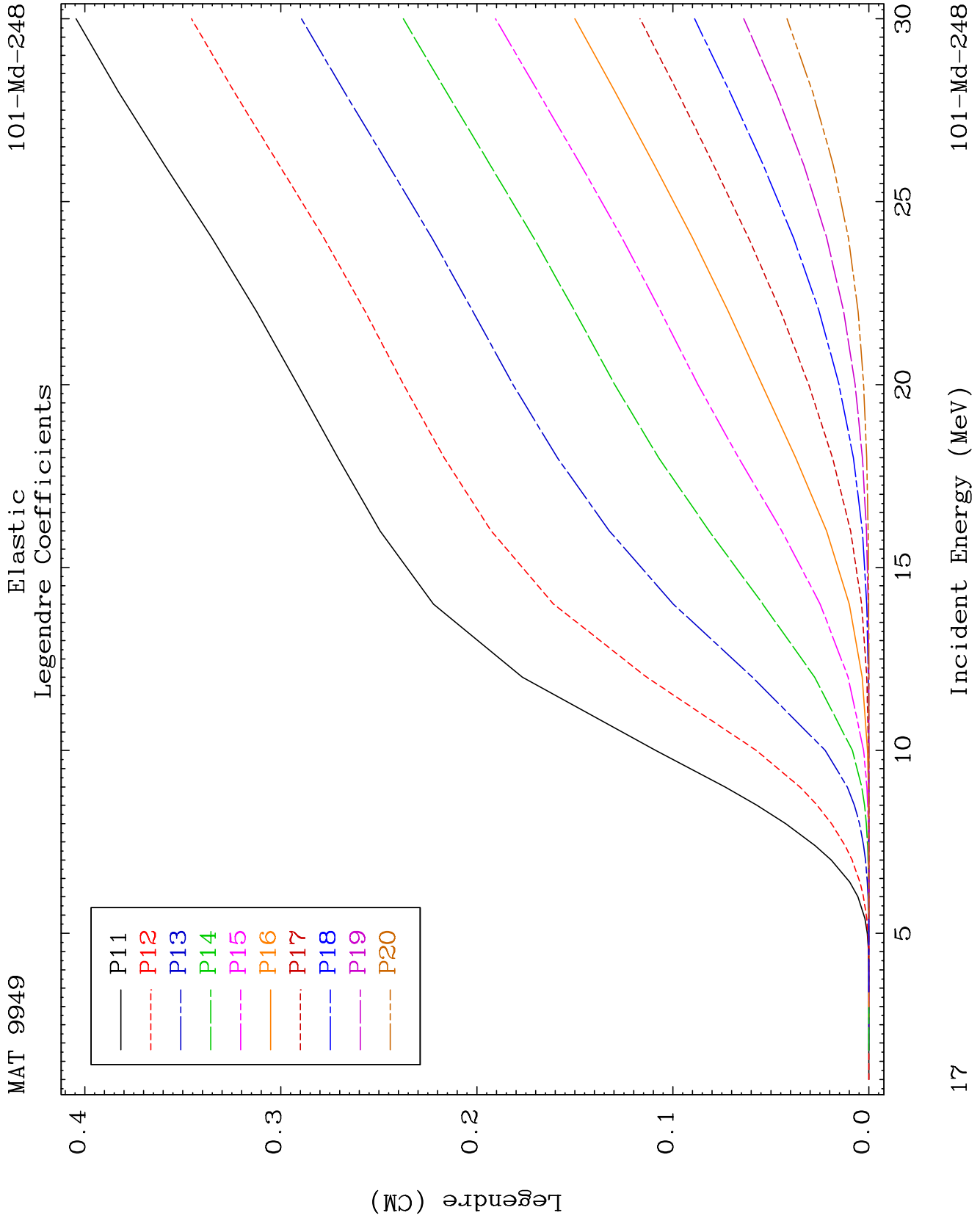


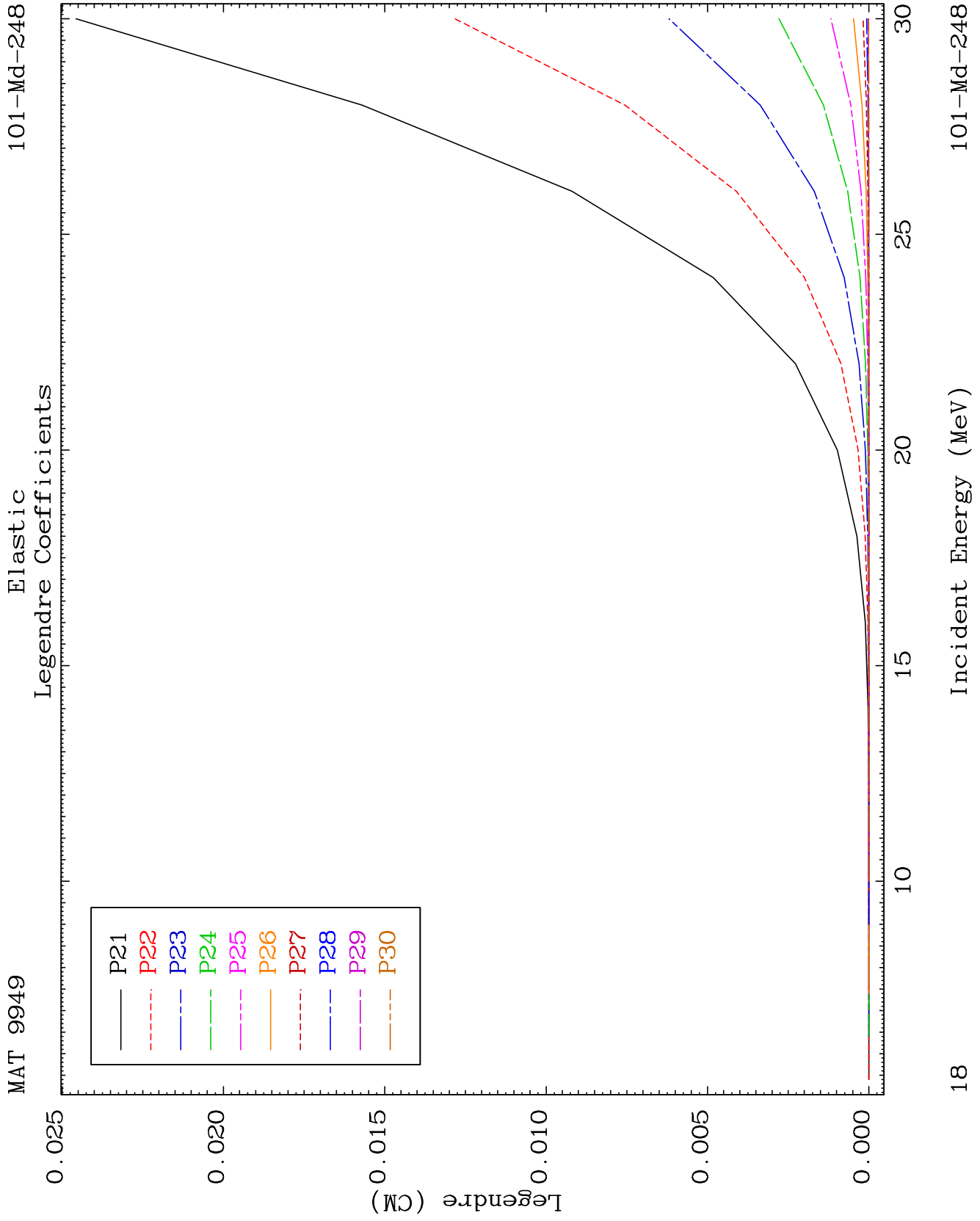
15

Incident Energy (MeV)

101-Md-248



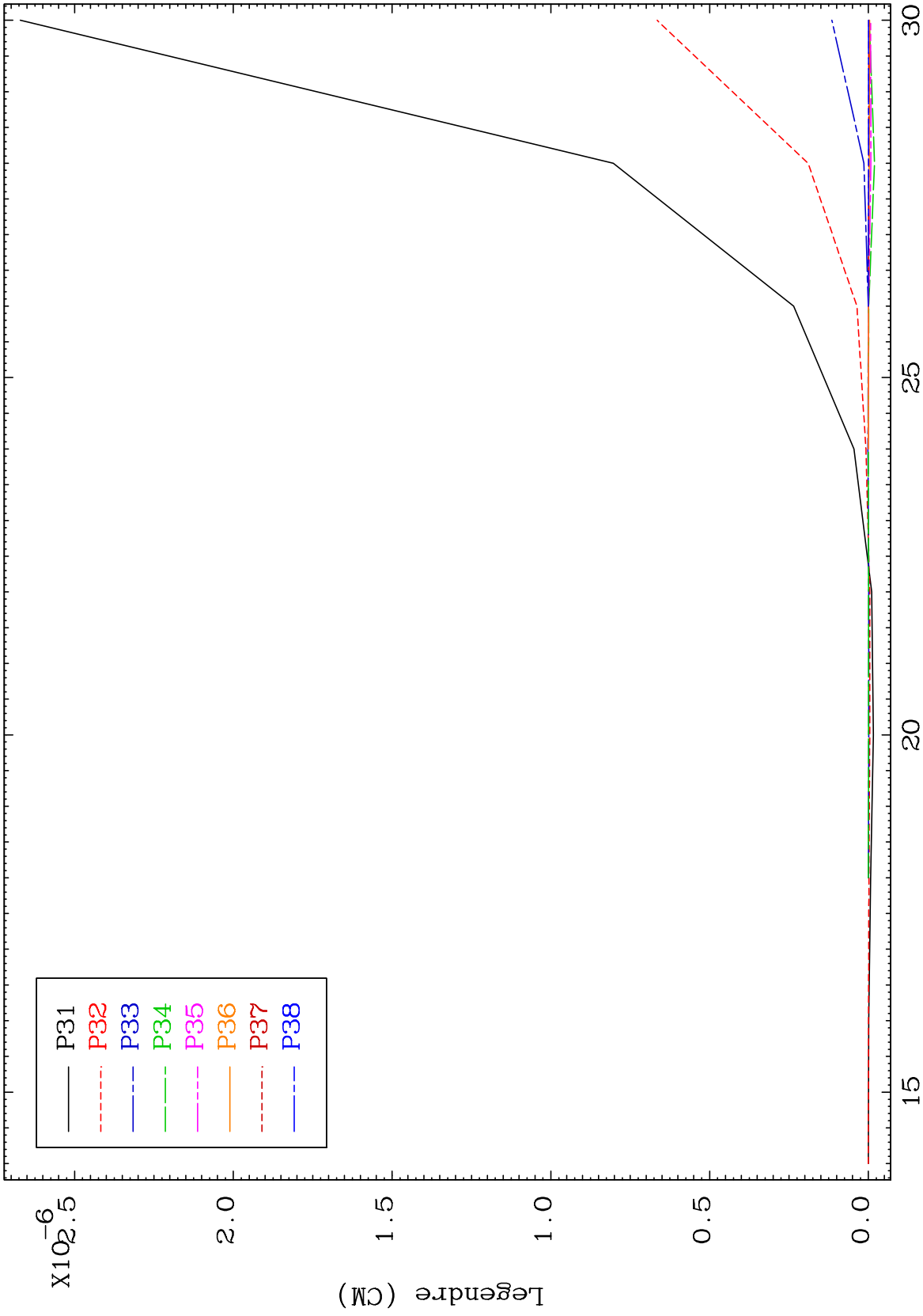




MAT 9949

Elastic Legendre Coefficients

101-Md-248



19

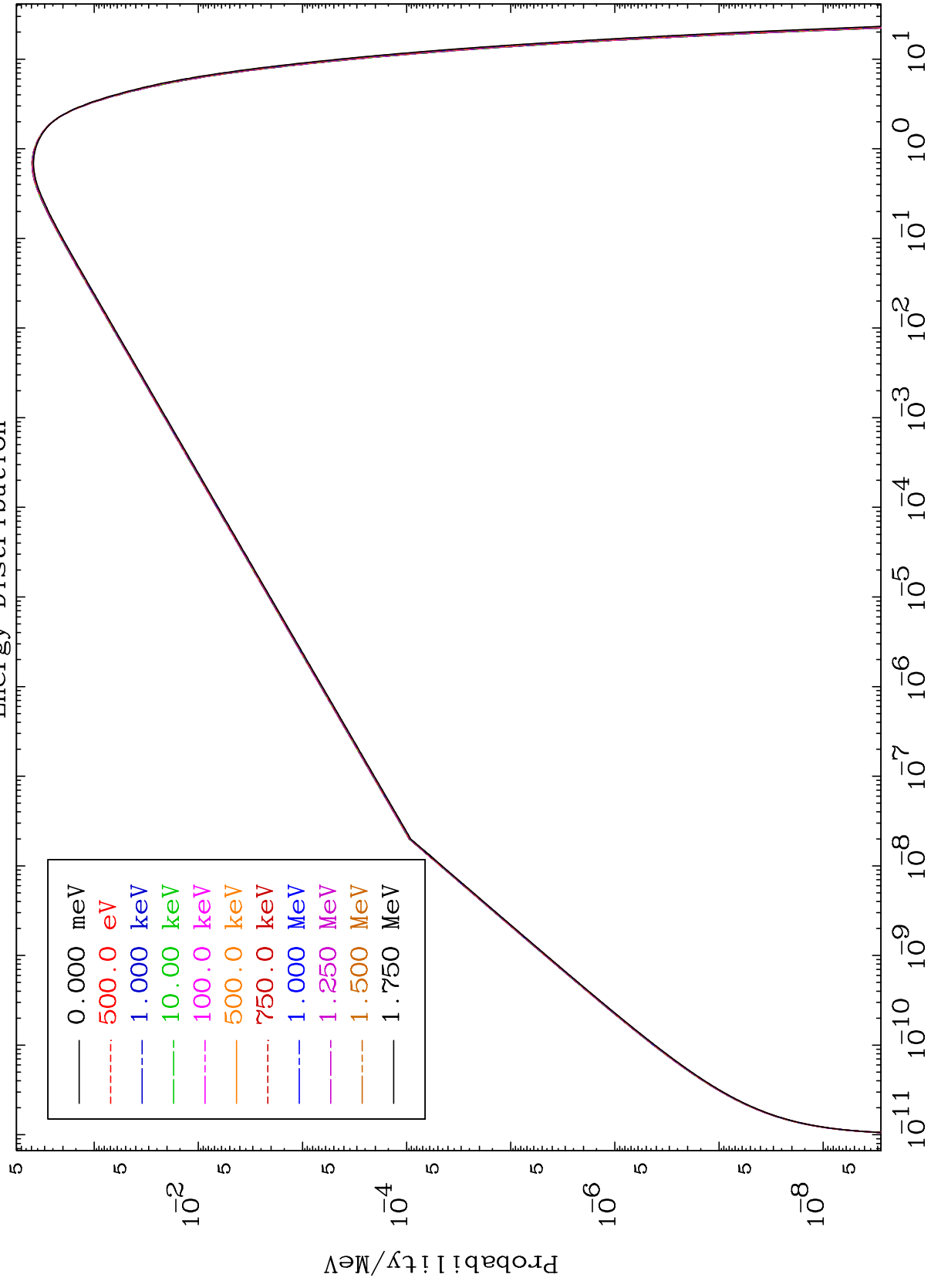
Incident Energy (MeV)

101-Md-248

MAT 9949

Fission Energy Distribution

101-Md-248

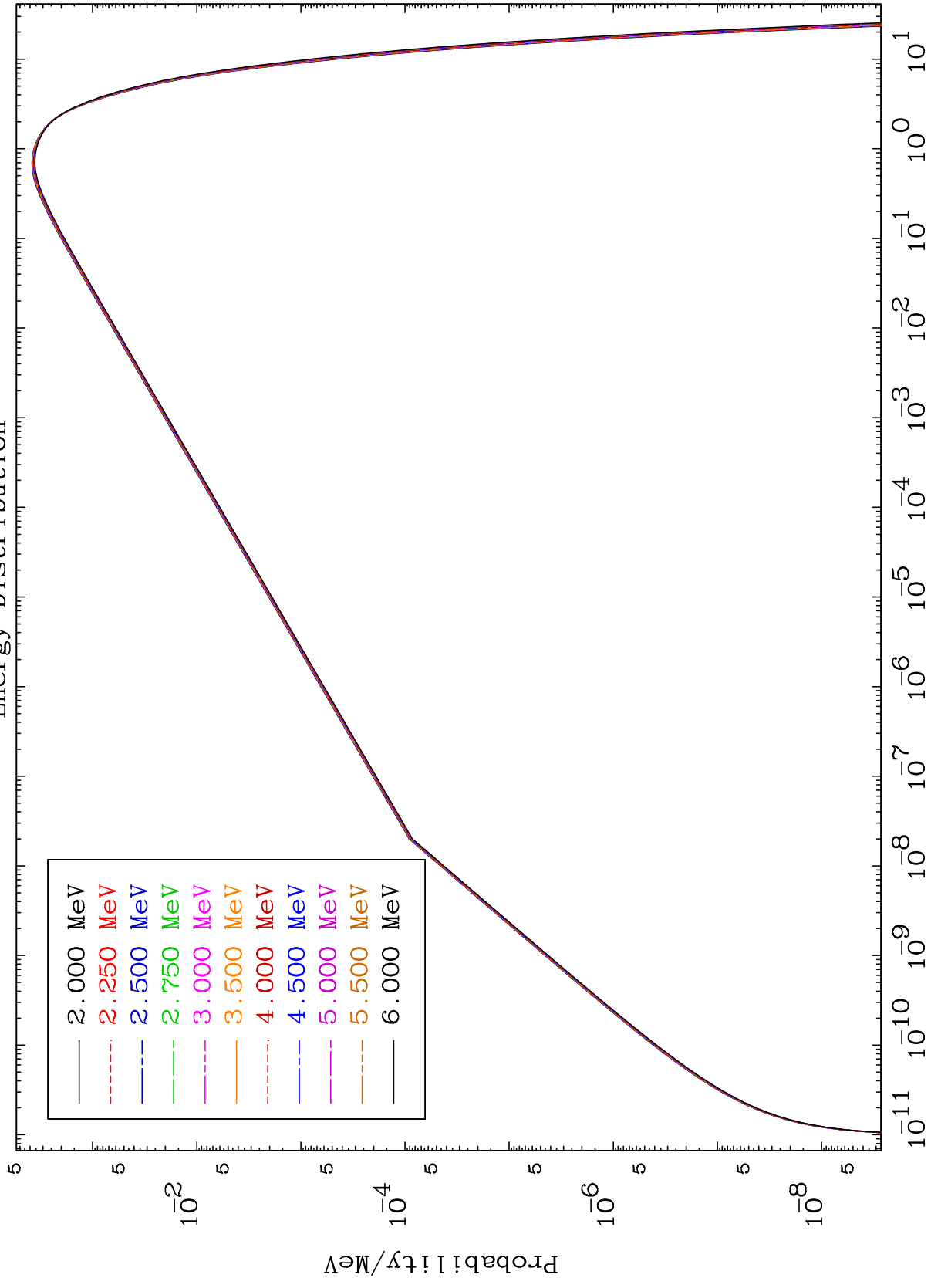


101-Md-248

MAT 9949

Fission Energy Distribution

101-Md-248

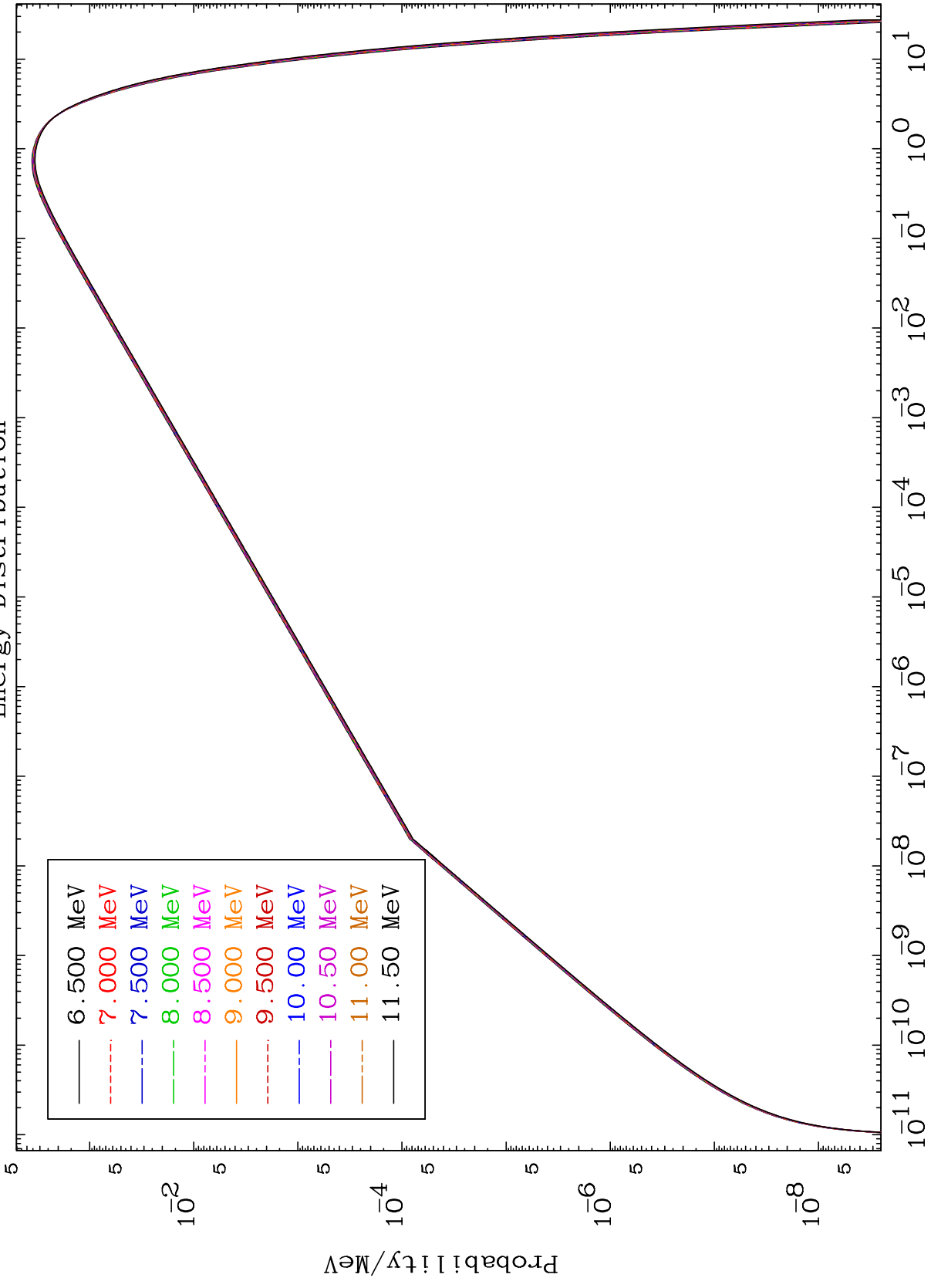


101-Md-248

MAT 9949

Fission Energy Distribution

101-Md-248

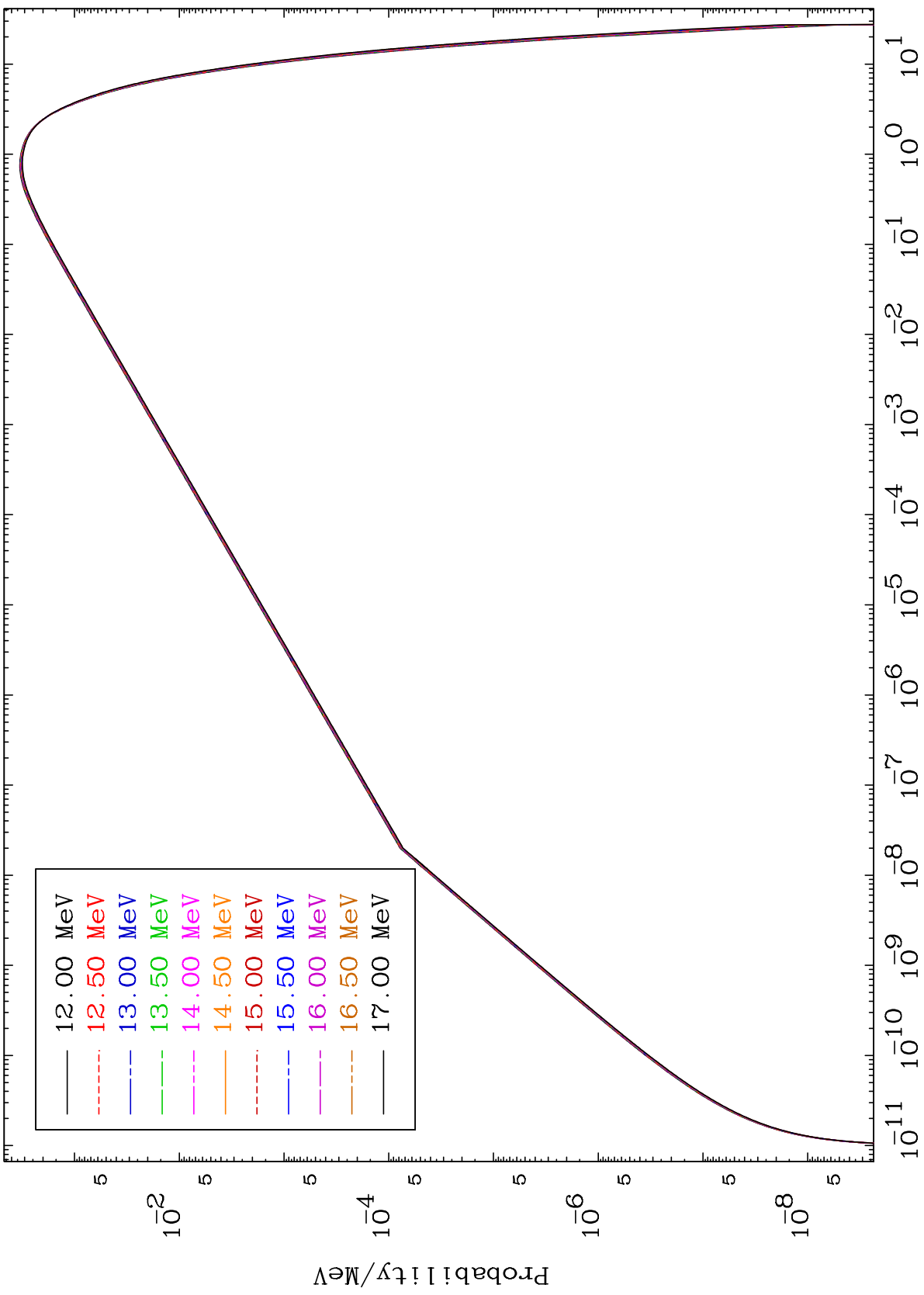


101-Md-248

MAT 9949

Fission Energy Distribution

101-Md-248



23

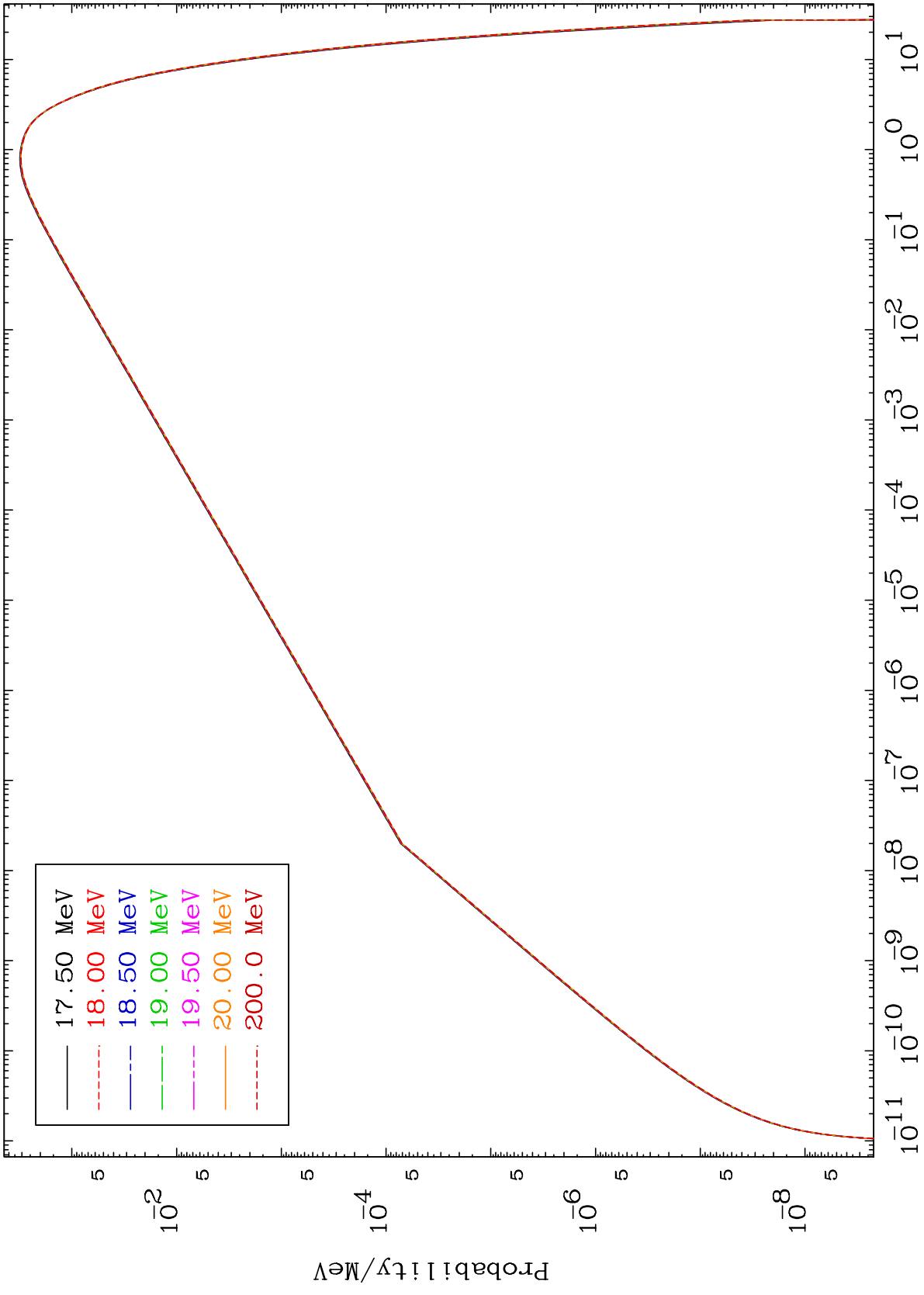
Secondary Energy (MeV)

101-Md-248

MAT 9949

Fission Energy Distribution

101-Md-248

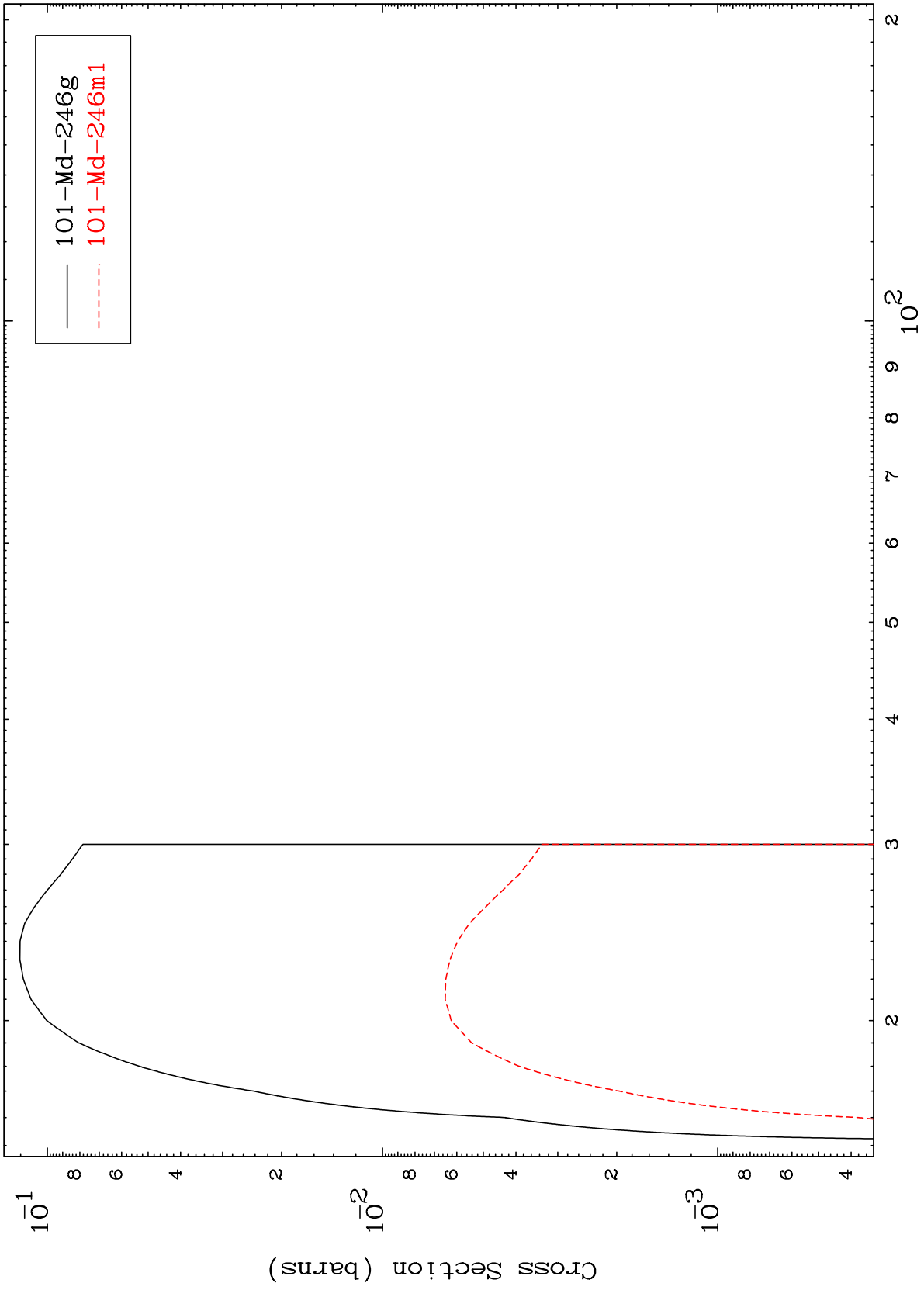


101-Md-248

MAT 9949

101-Md-248

(n,3n)
Radionuclide Production Cross Section



25

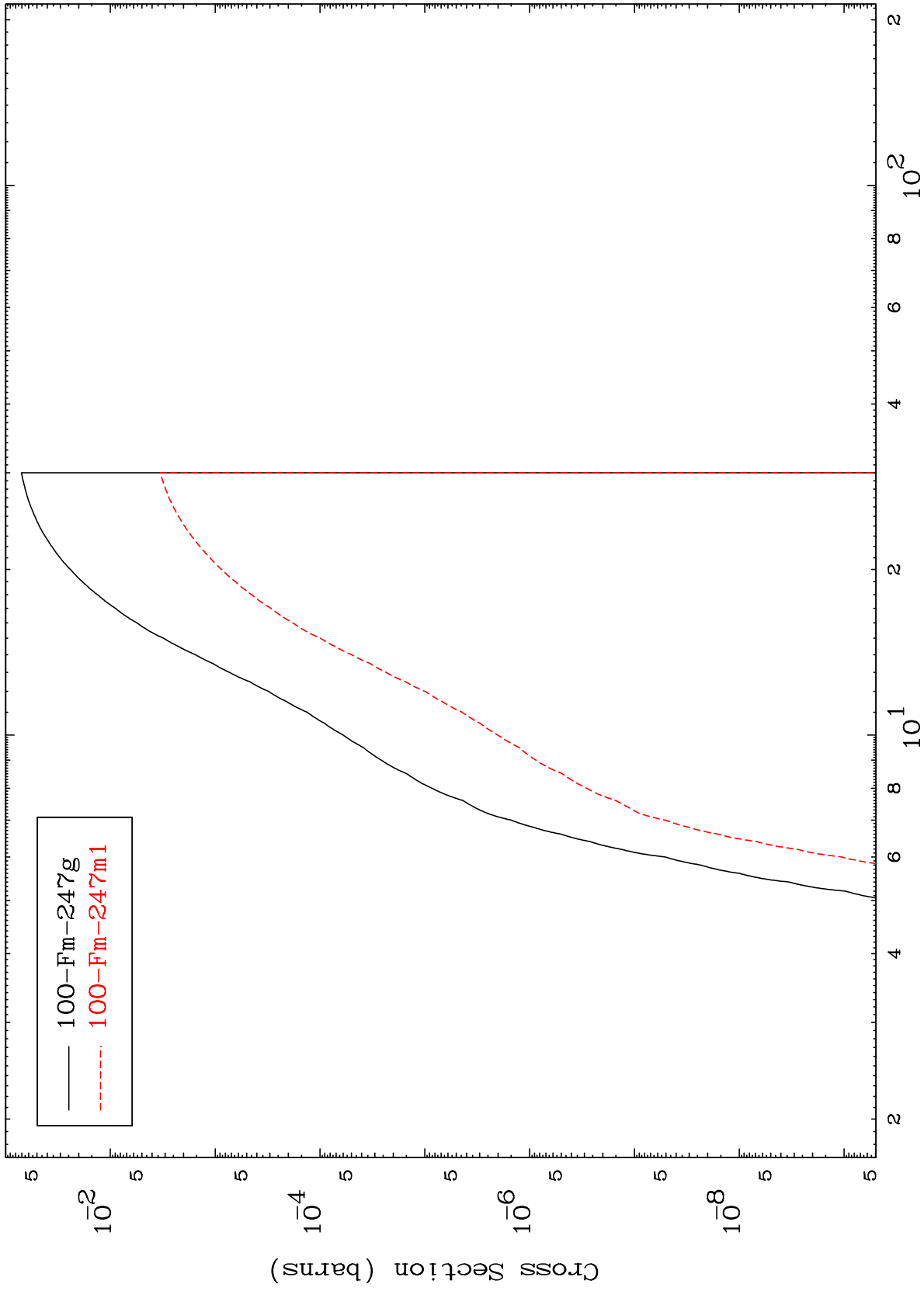
101-Md-248

MAT 9949

(n,n') p

101-Md-248

Radionuclide Production Cross Section



26

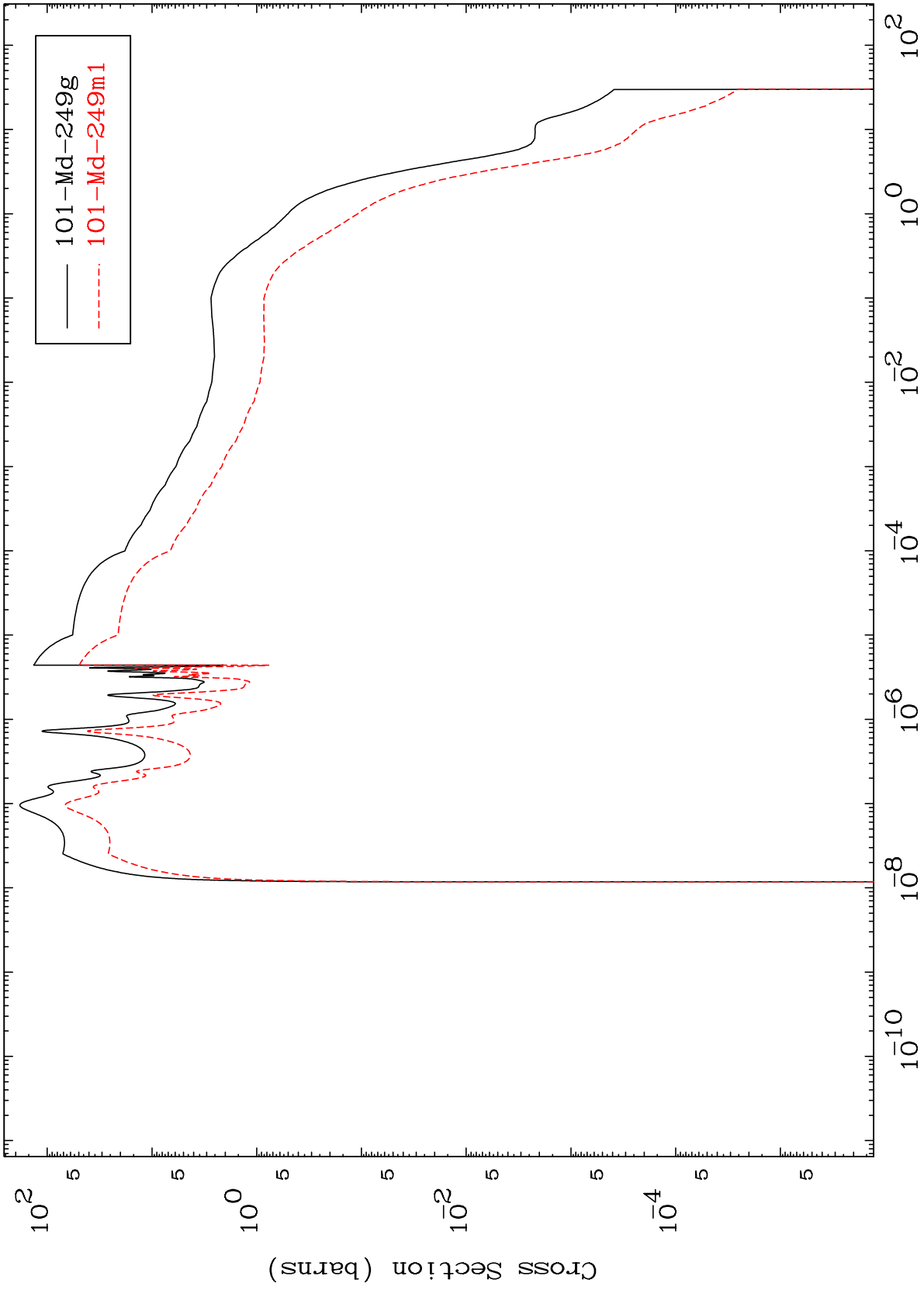
Incident Energy (MeV)

101-Md-248

MAT 9949

101-Md-248

Radionuclide Production Cross Section
(n, γ)



27

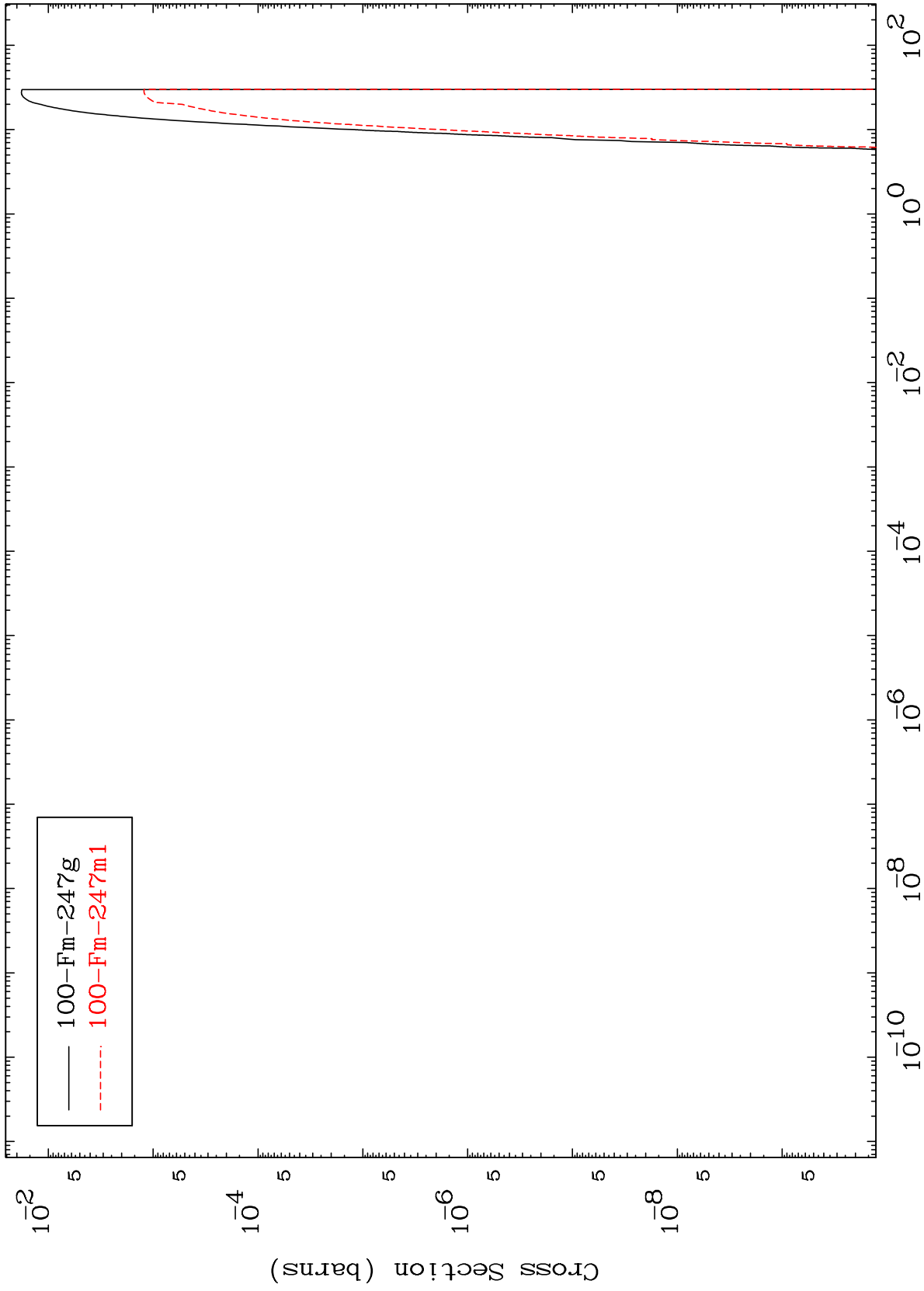
101-Md-248

MAT 9949

(n,d)

101-Md-248

Radionuclide Production Cross Section



28

101-Md-248

101-Md-248