

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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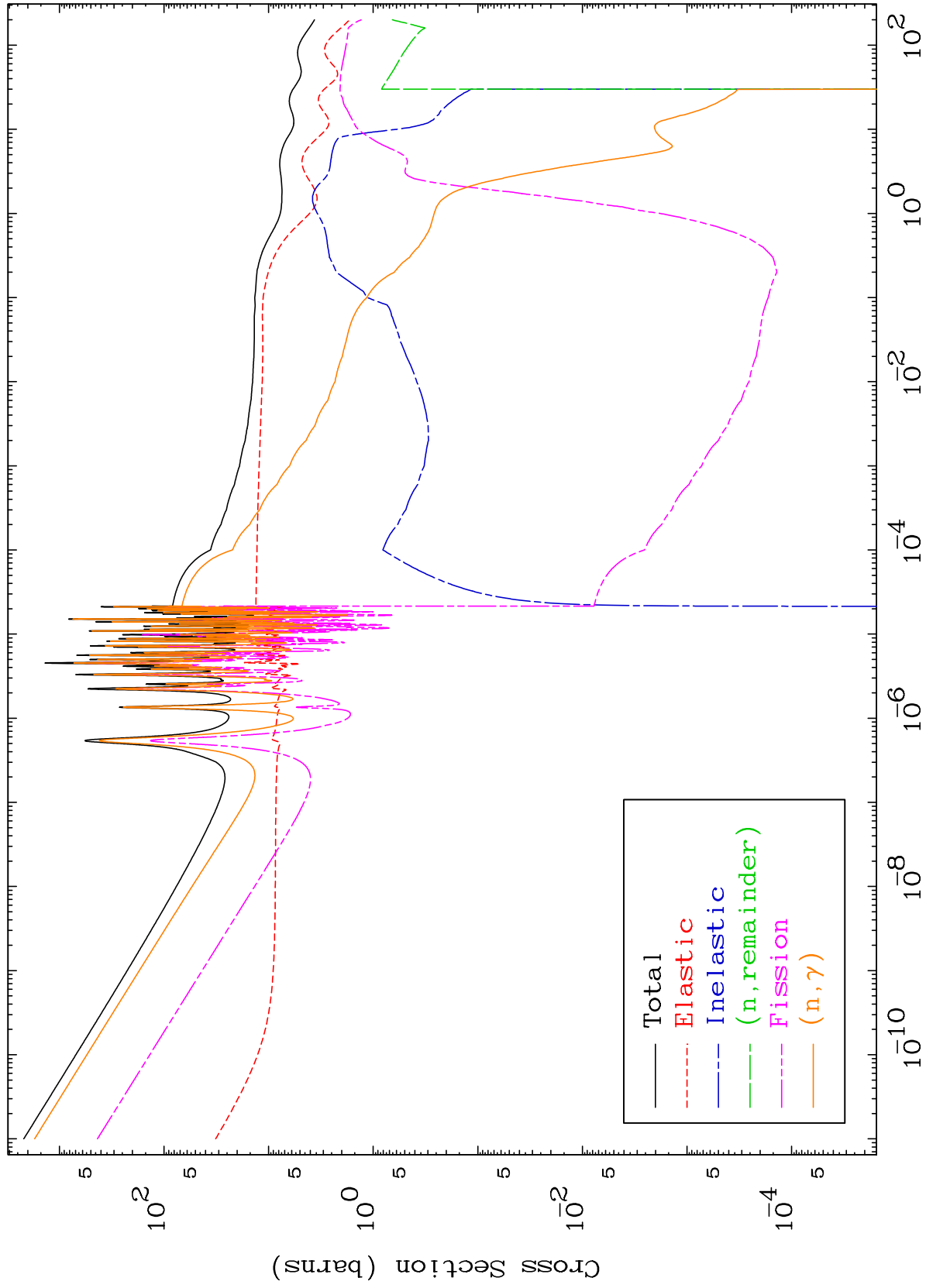
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9955

Major  
293 Kelvin Cross Sections

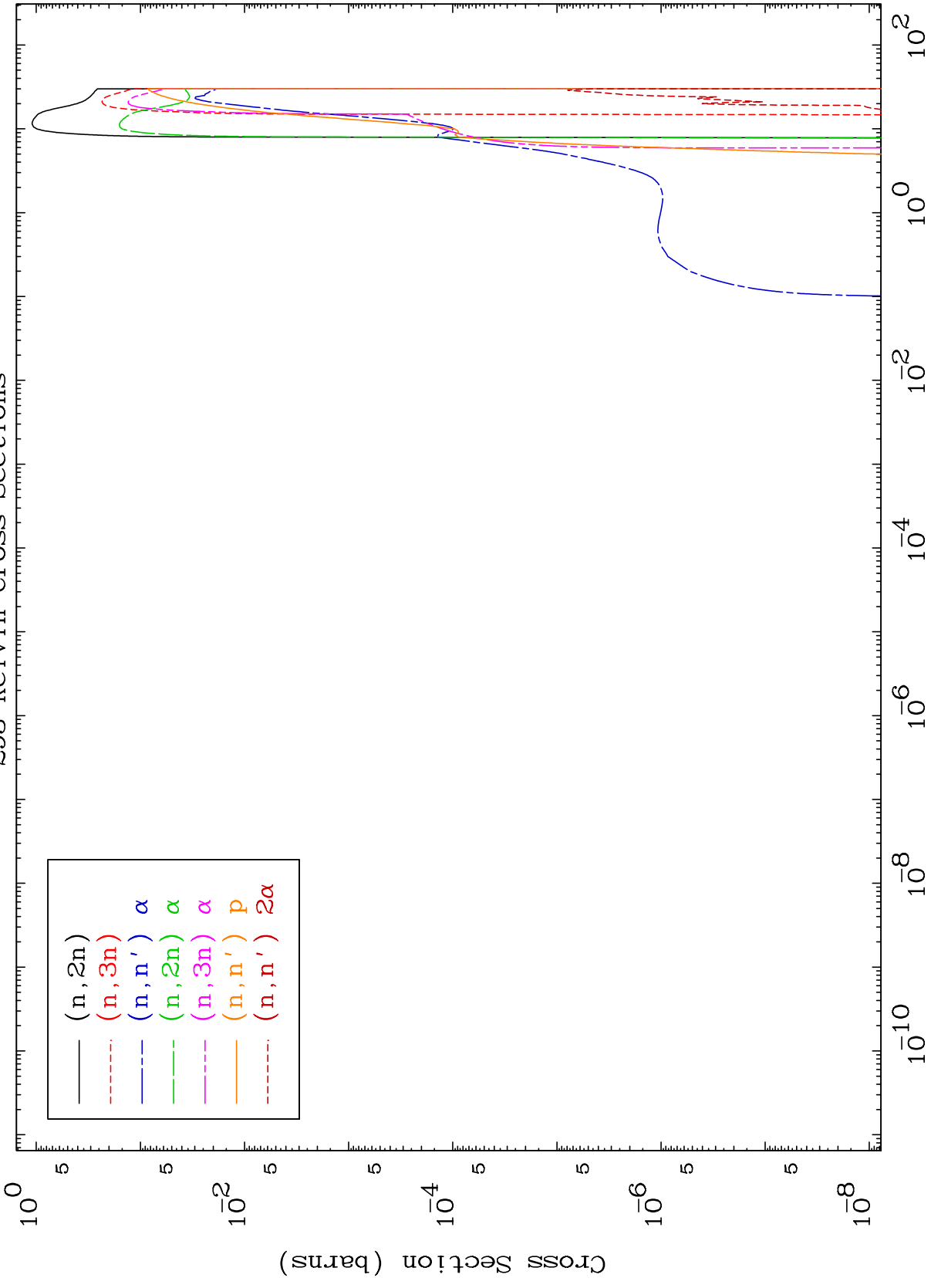
101-Md-249



MAT 9955

Neutron Production  
293 Kelvin Cross Sections

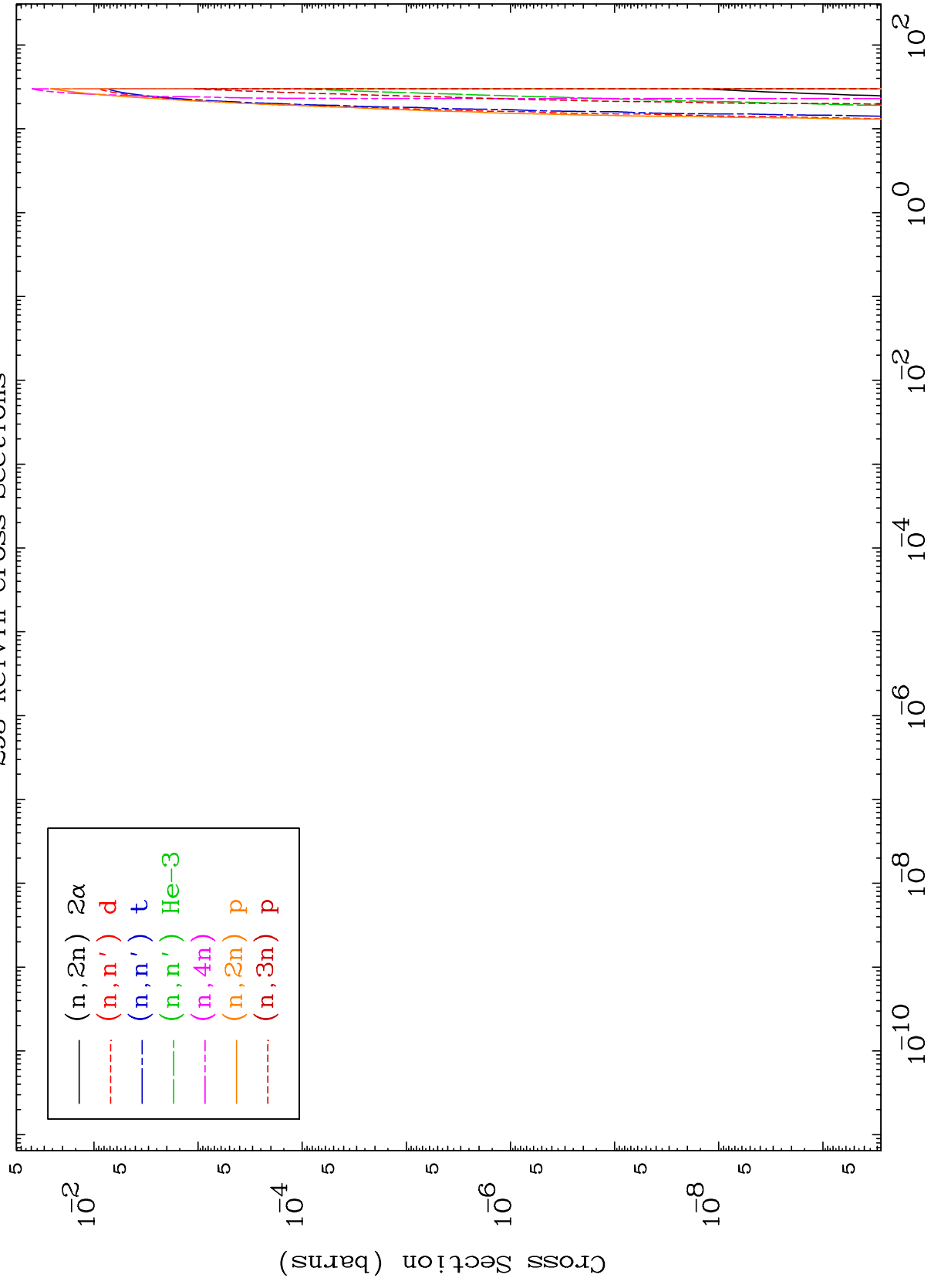
101-Md-249



MAT 9955

Neutron Production  
293 Kelvin Cross Sections

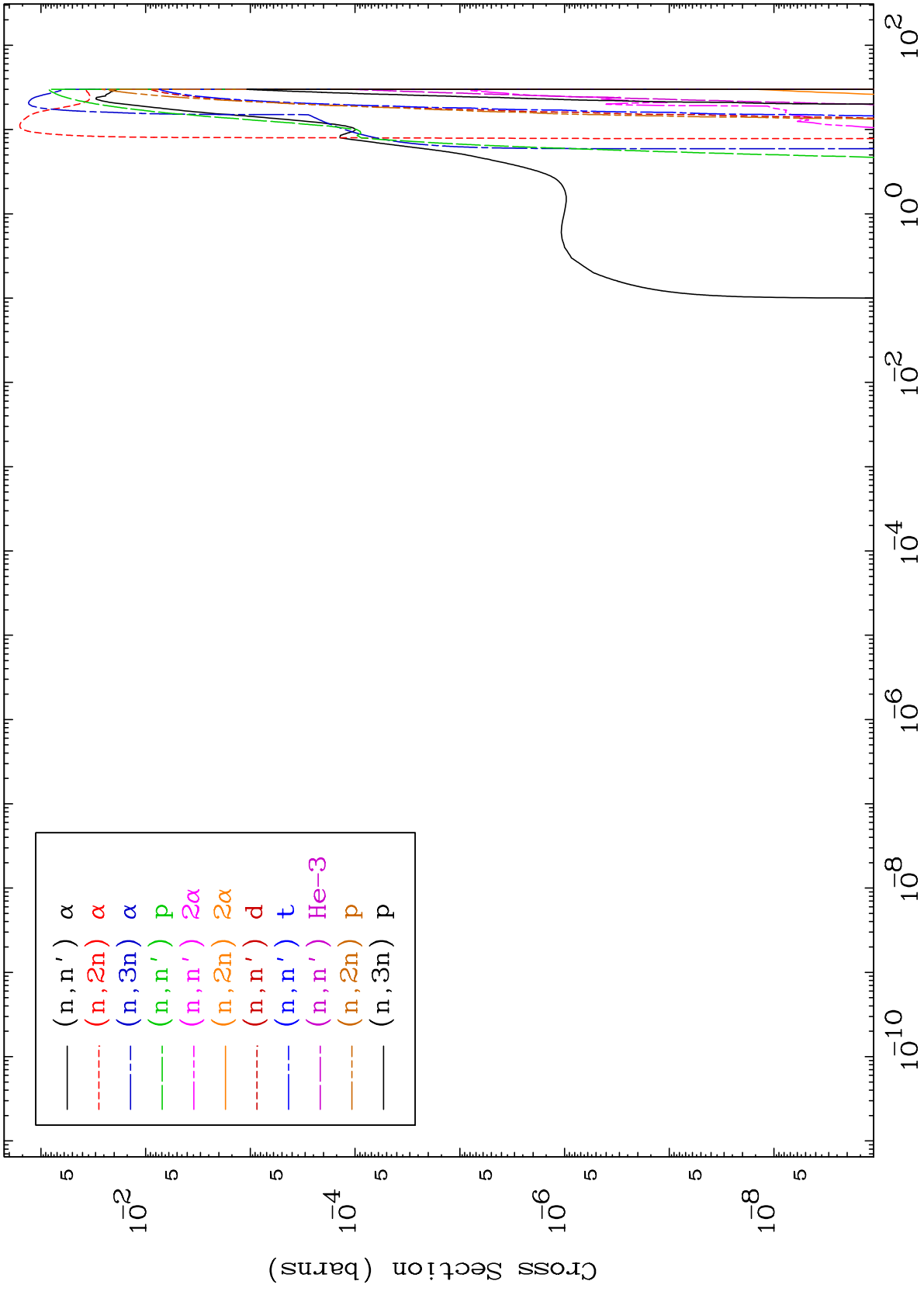
101-Md-249



MAT 9955

Charged Particle  
293 Kelvin Cross Sections

101-Md-249



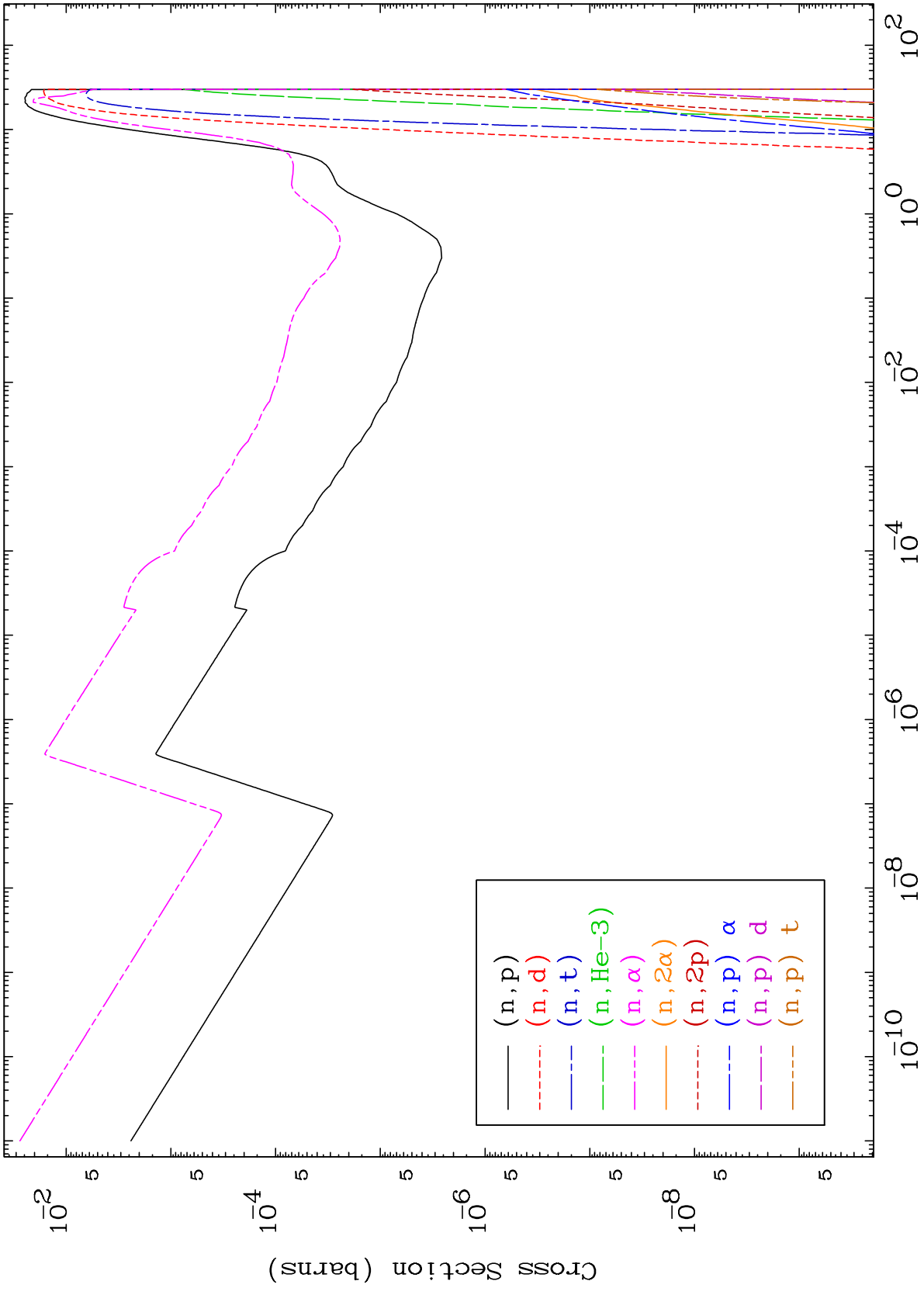
101-Md-249

Incident Energy (MeV)

MAT 9955

Charged Particle  
293 Kelvin Cross Sections

101-Md-249

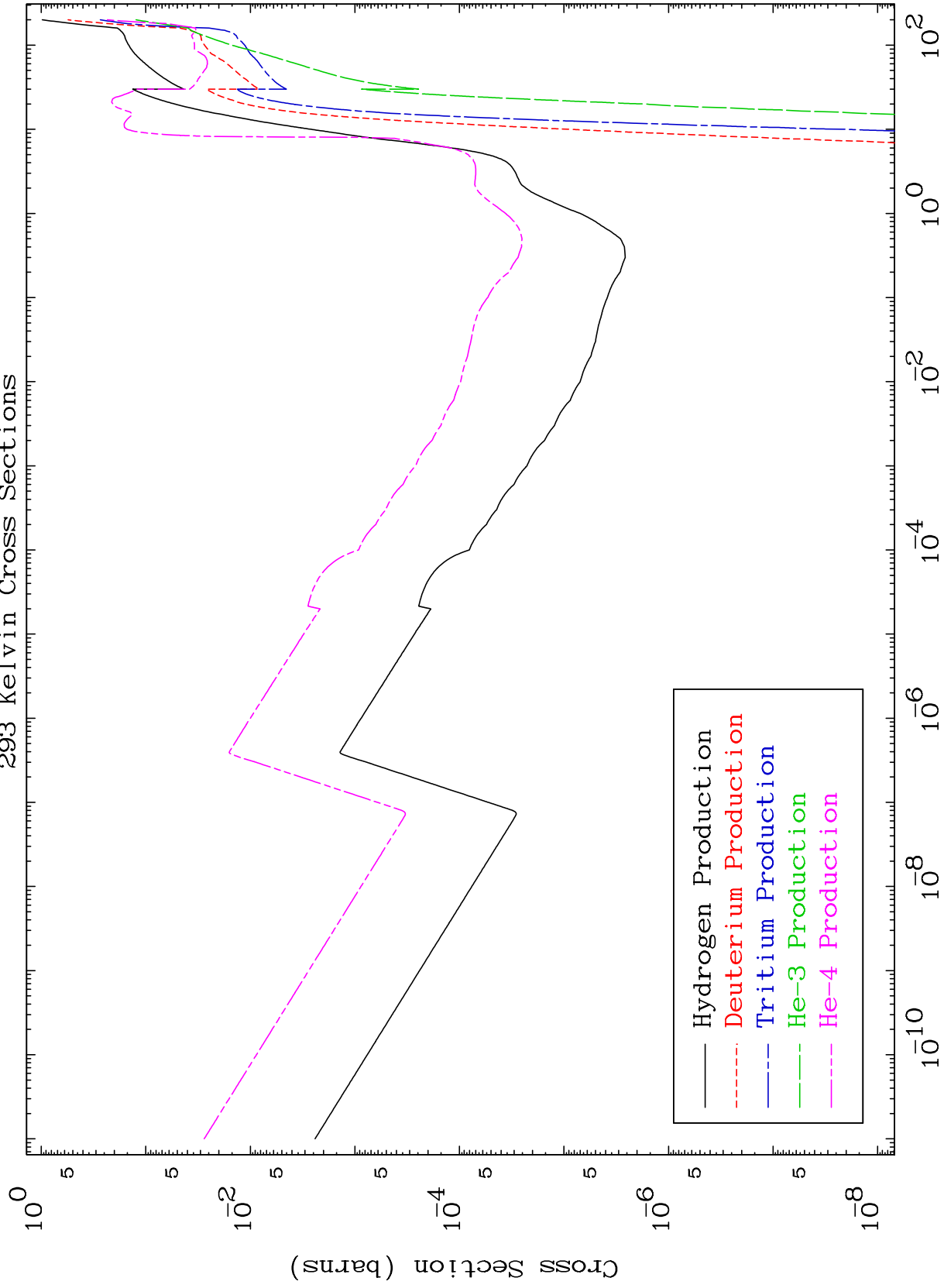


101-Md-249

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Particle Production  
293 Kelvin Cross Sections

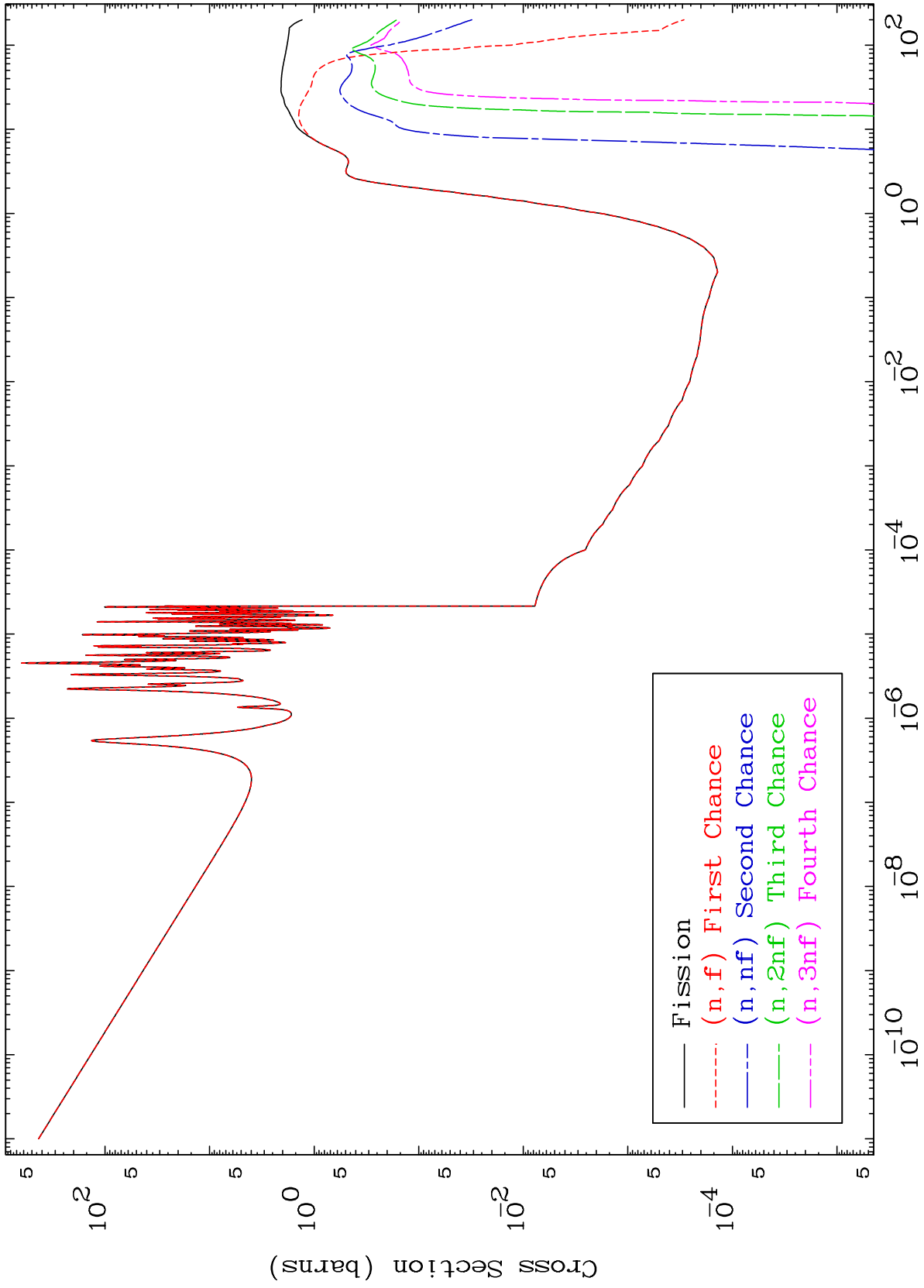
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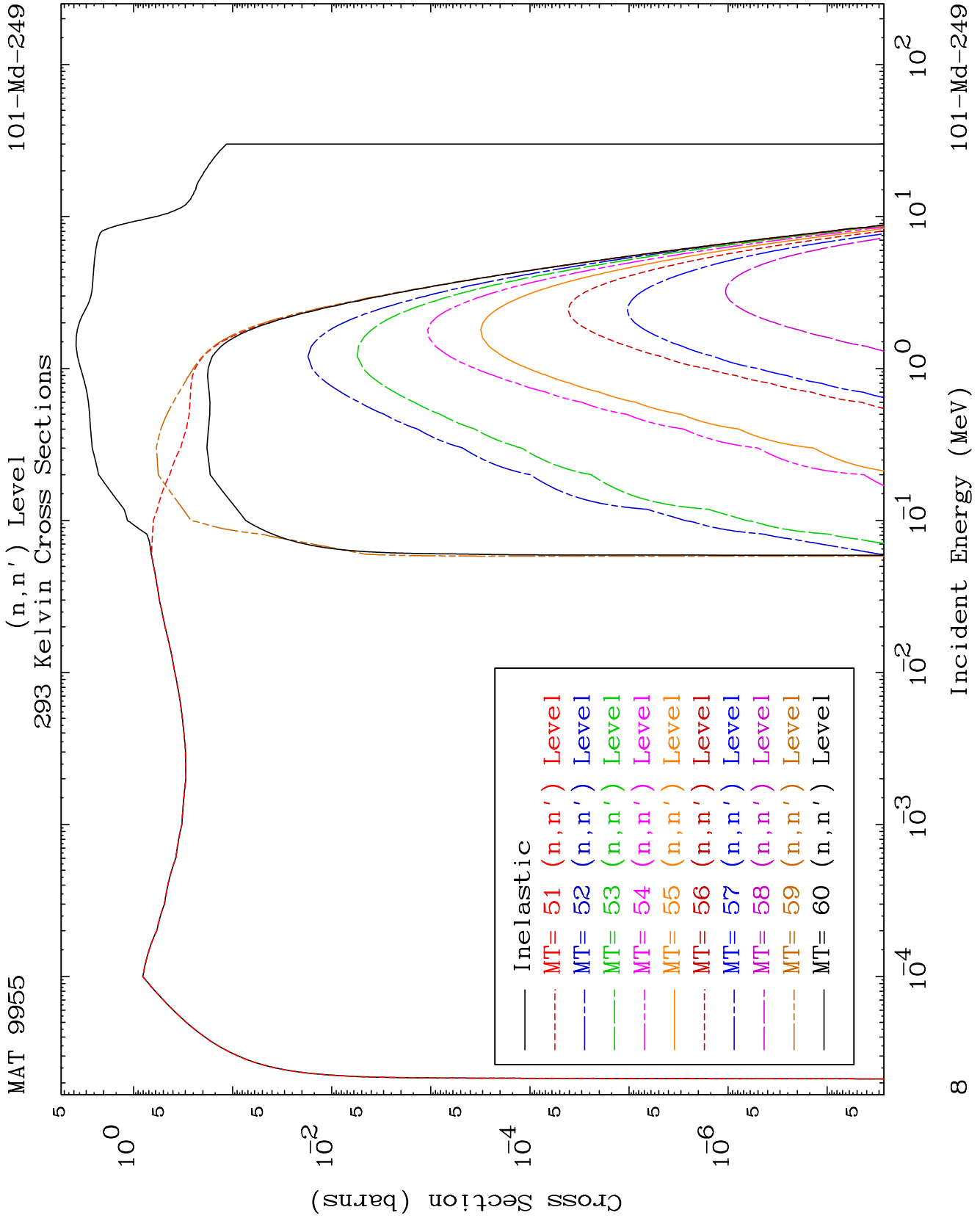
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Fission  
293 Kelvin Cross Sections

101-Md-249



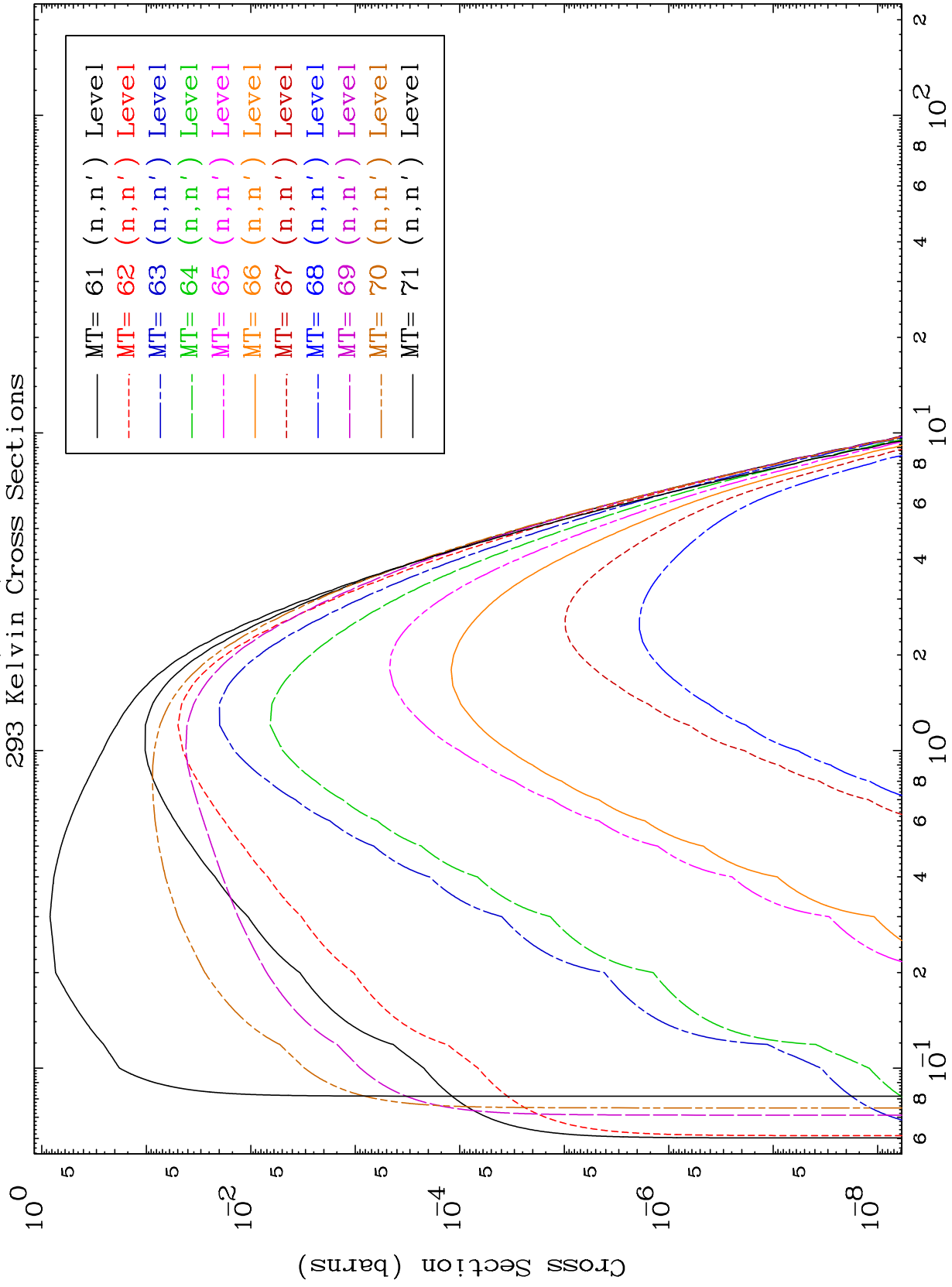




MAT 9955

(n,n') Level

101-Md-249



9

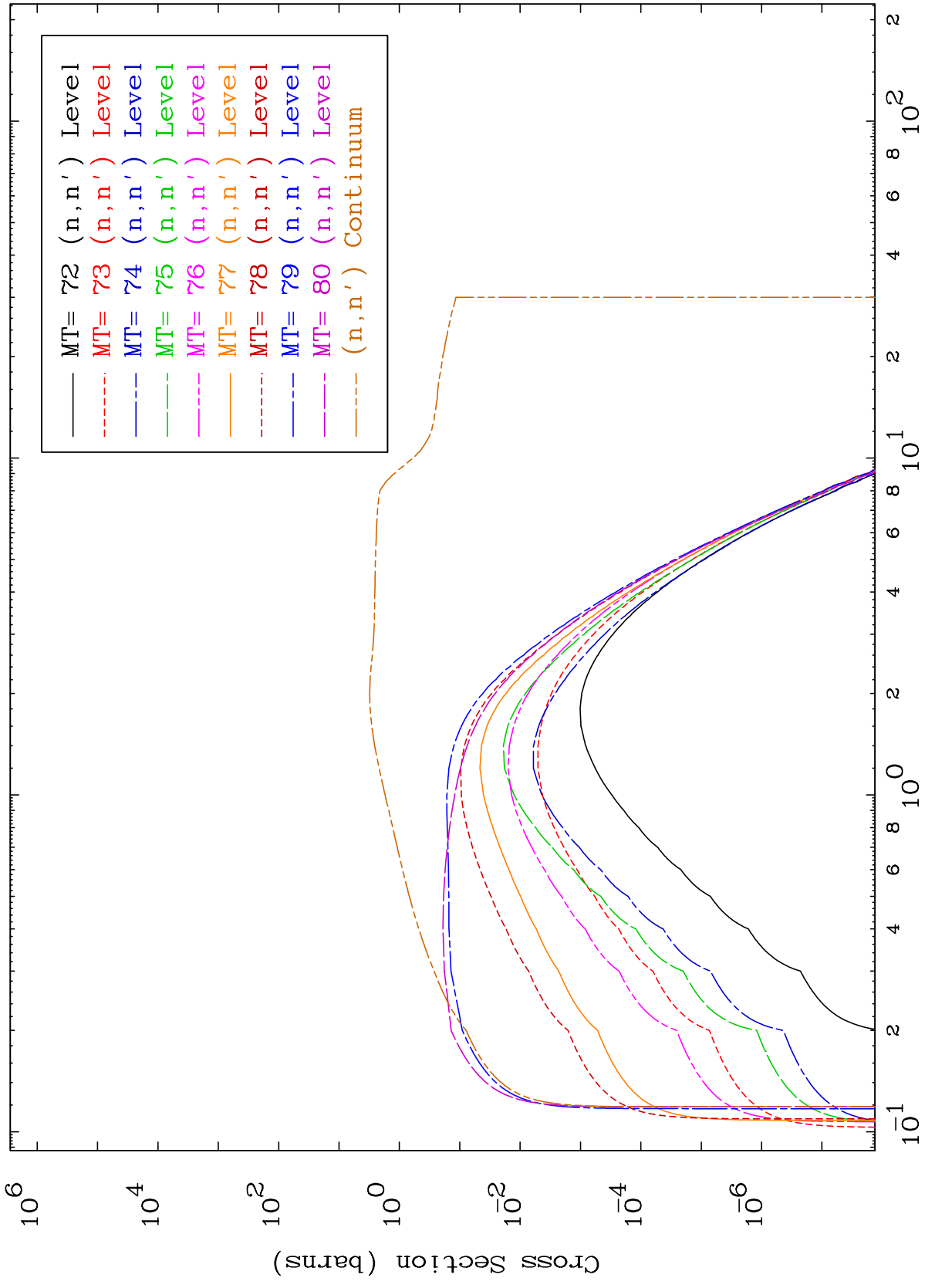
101-Md-249

MAT 9955

(n,n') Level

101-Md-249

293 Kelvin Cross Sections



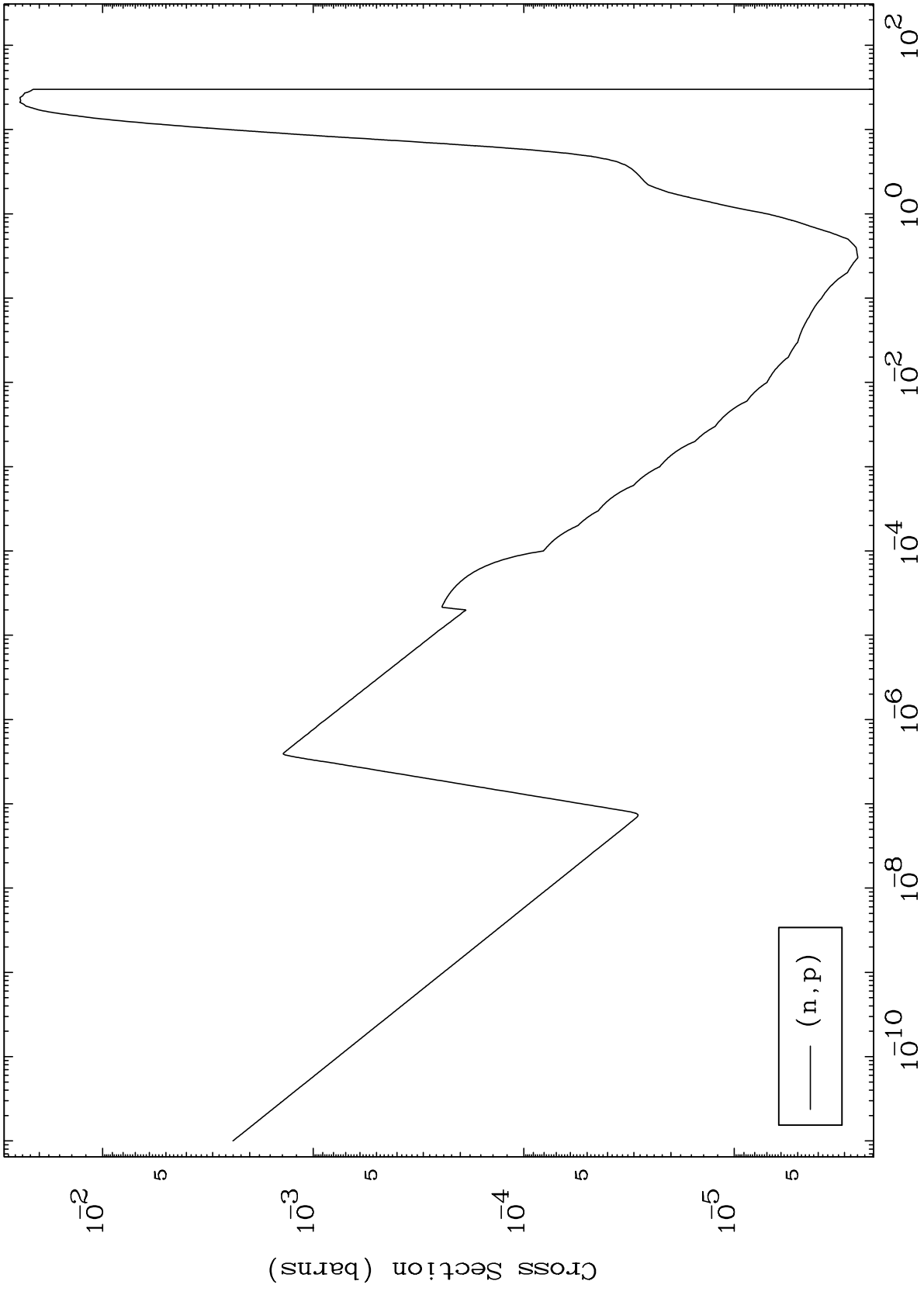
Incident Energy (MeV)

101-Md-249

MAT 9955

(n,p) Levels  
293 Kelvin Cross Sections

101-Md-249

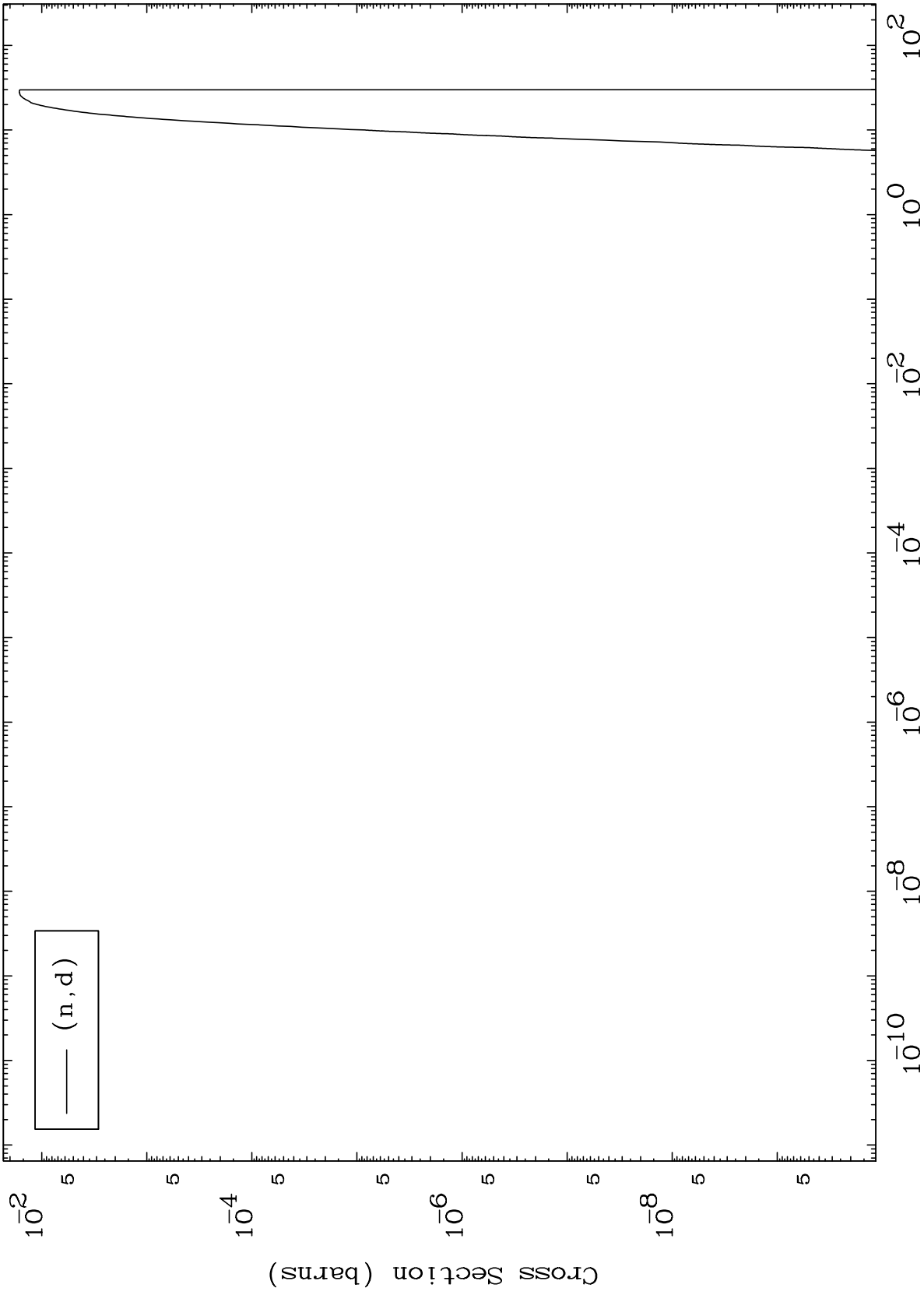


— (n, p)

MAT 9955

(n,d) Levels  
293 Kelvin Cross Sections

101-Md-249



12

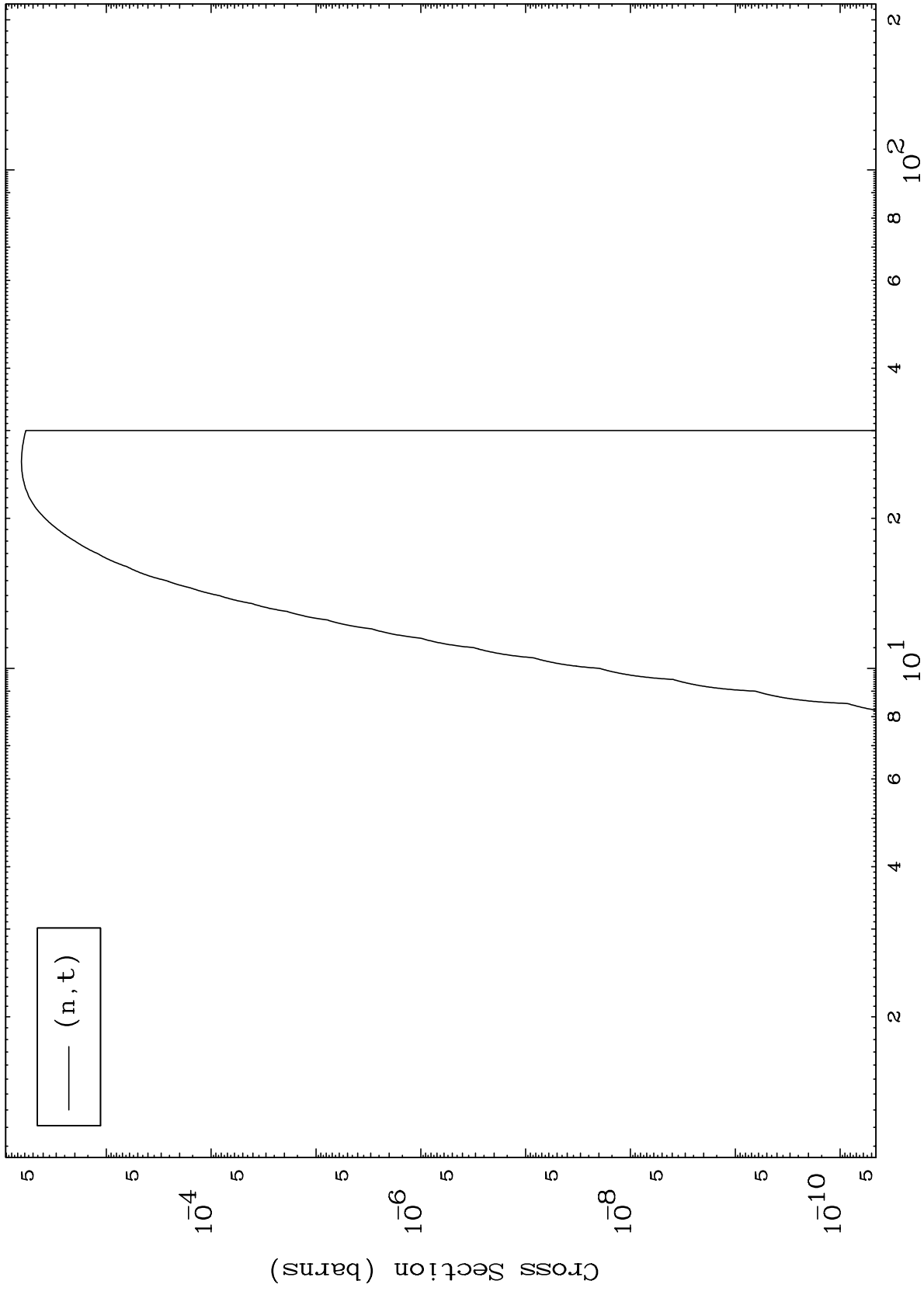
Incident Energy (MeV)

101-Md-249

MAT 9955

(n,t) Levels  
293 Kelvin Cross Sections

101-Md-249



13

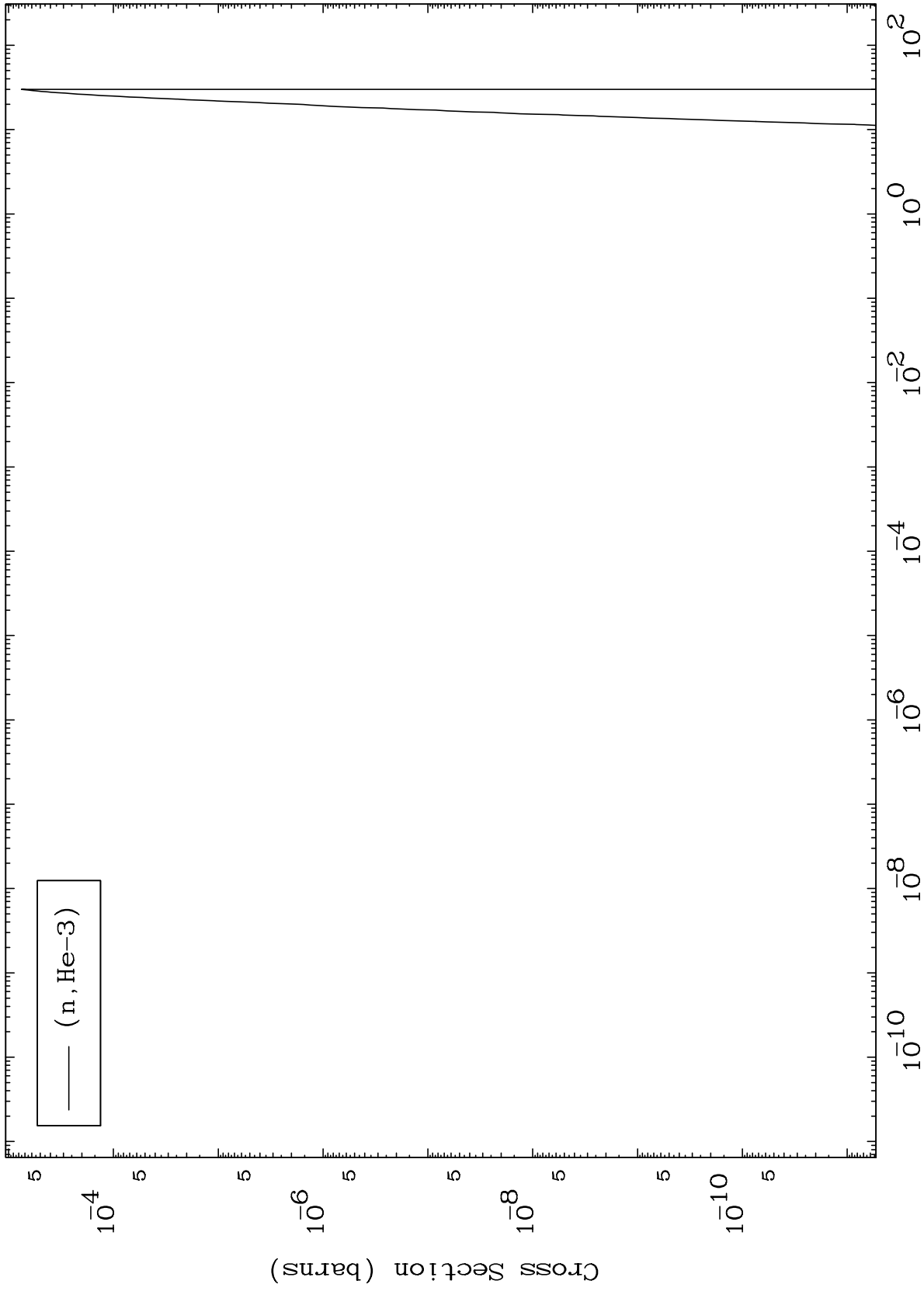
Incident Energy (MeV)

101-Md-249

MAT 9955

(n,He3) Levels  
293 Kelvin Cross Sections

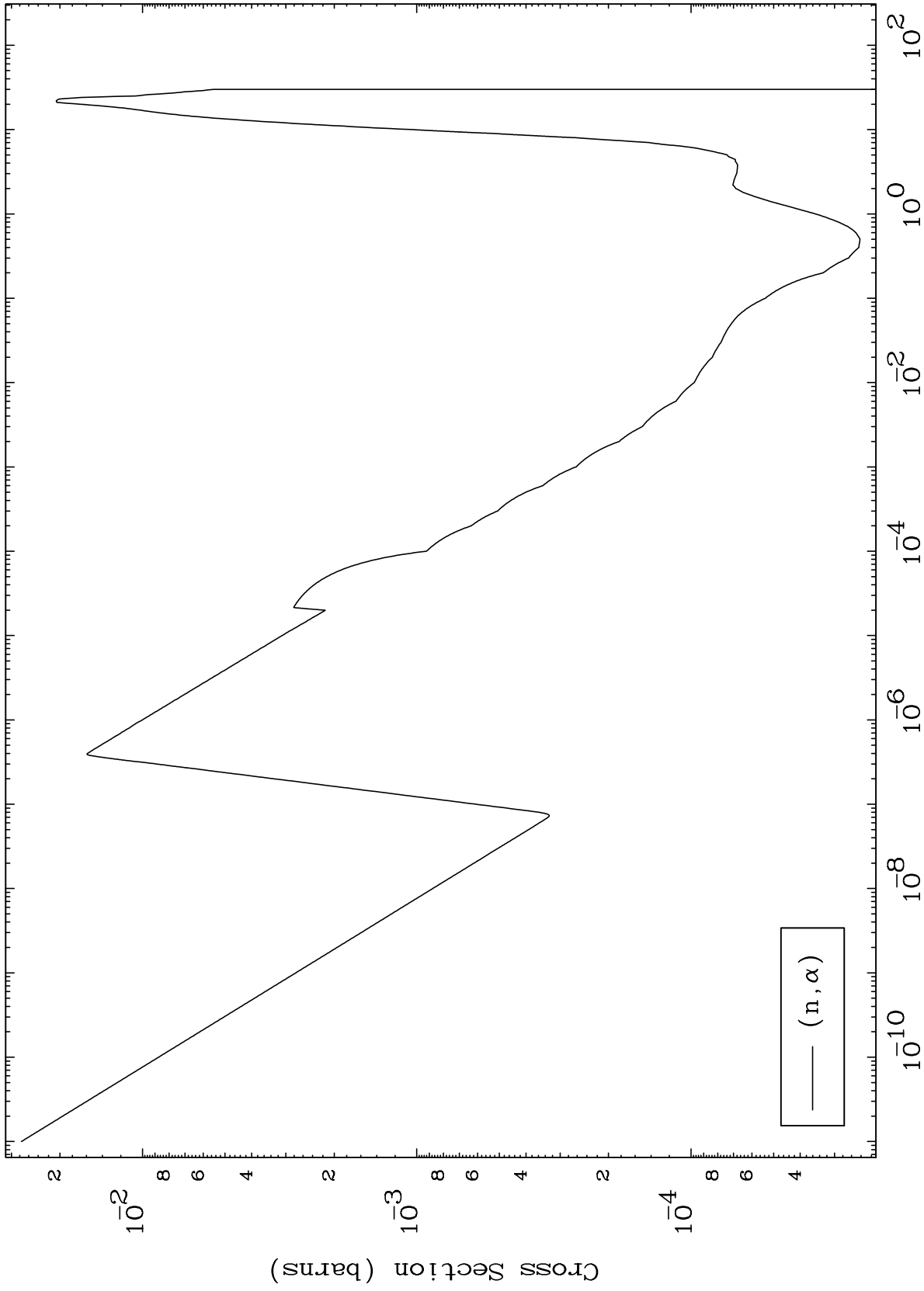
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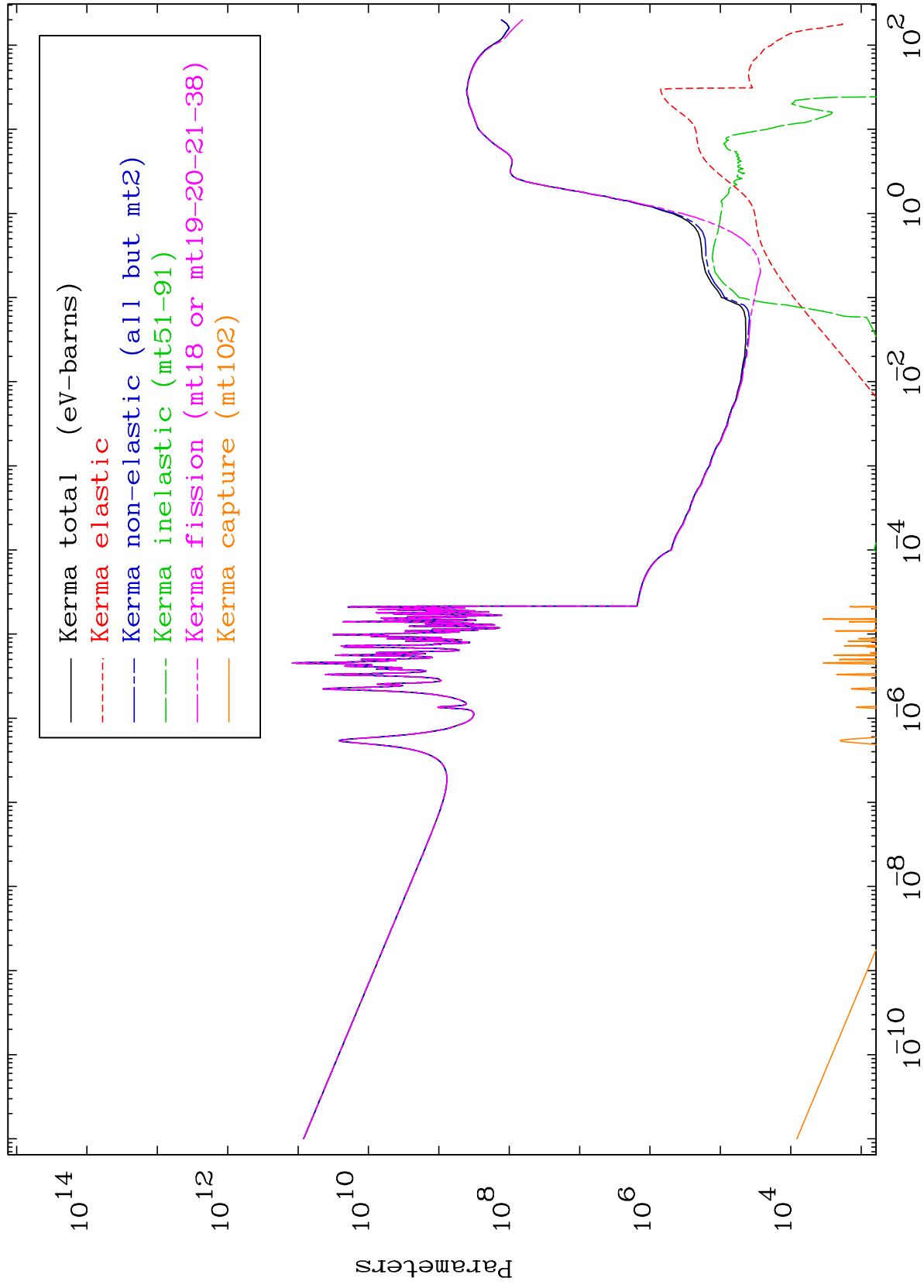
MAT 9955

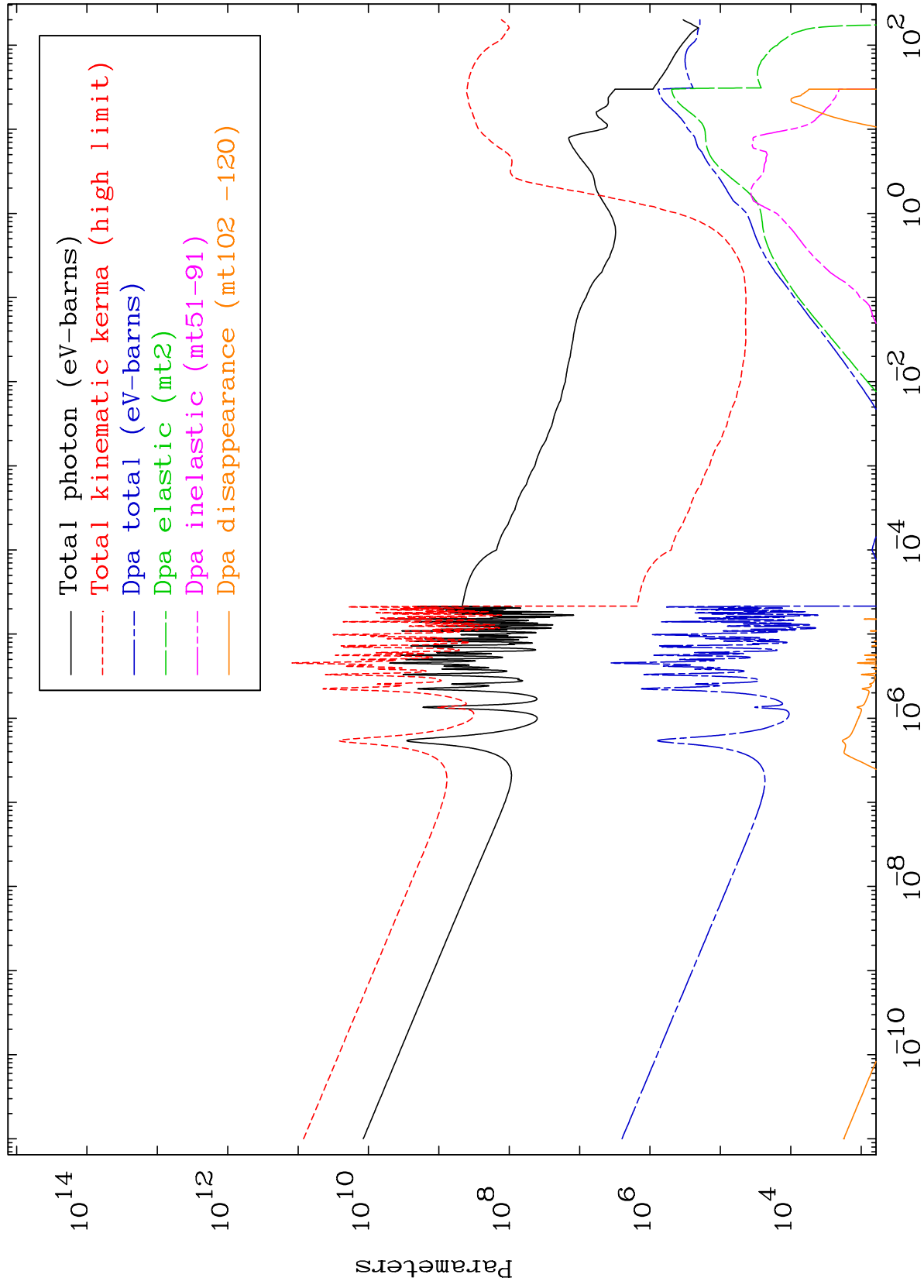
(n,α) Levels  
293 Kelvin Cross Sections

101-Md-249





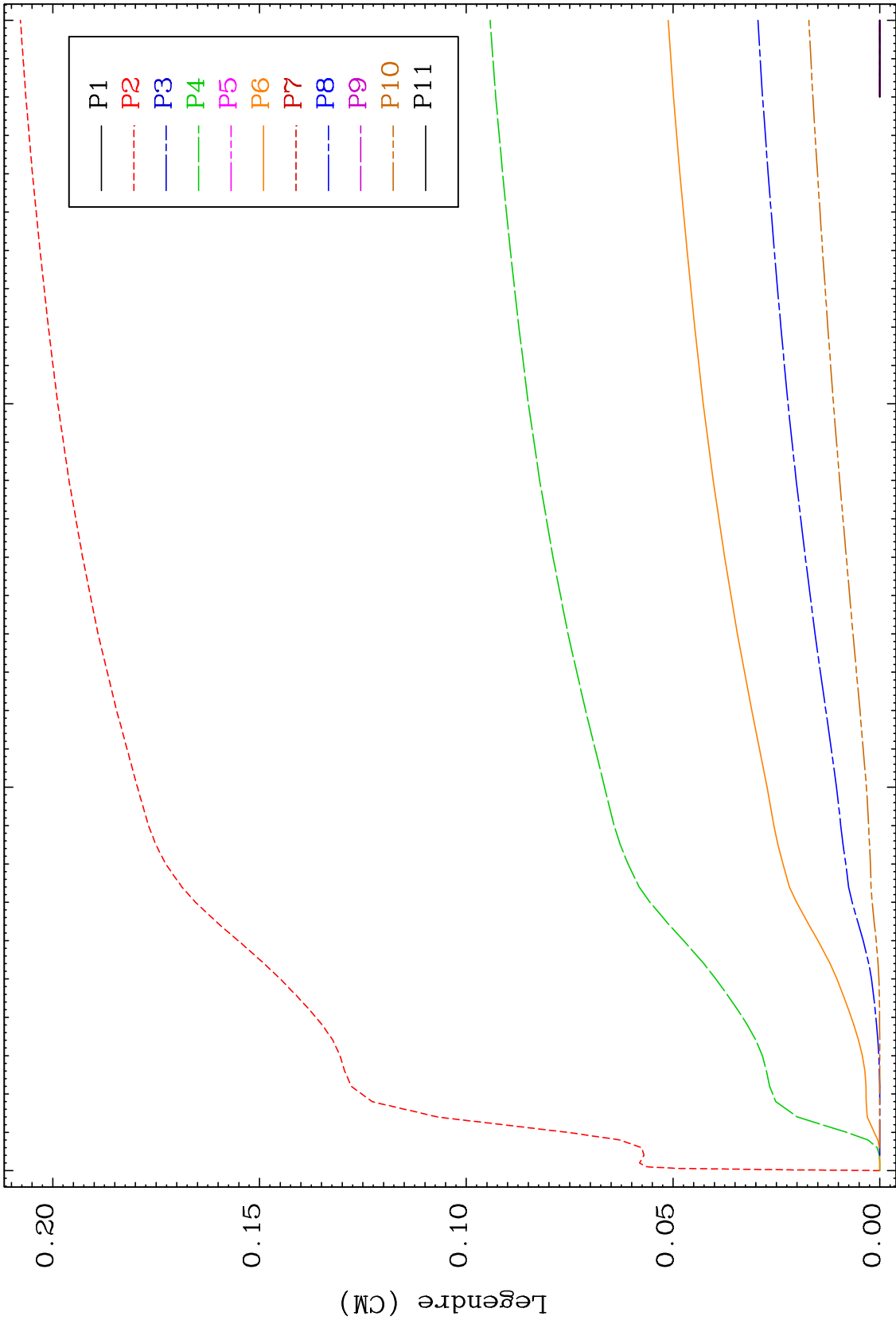




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Elastic Legendre Coefficients

101-Md-249



101-Md-249

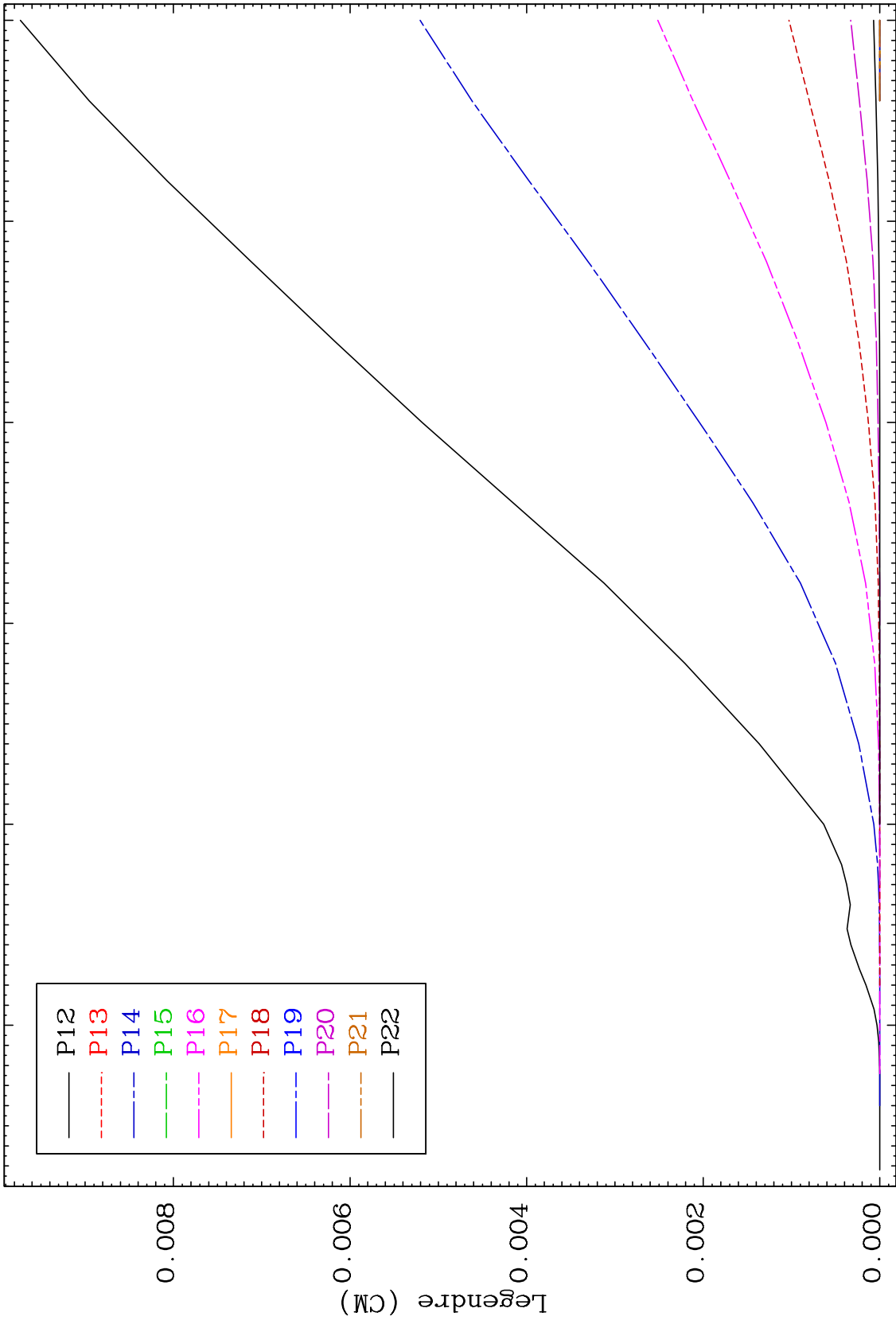
Incident Energy (MeV)

18

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Elastic Legendre Coefficients

101-Md-249



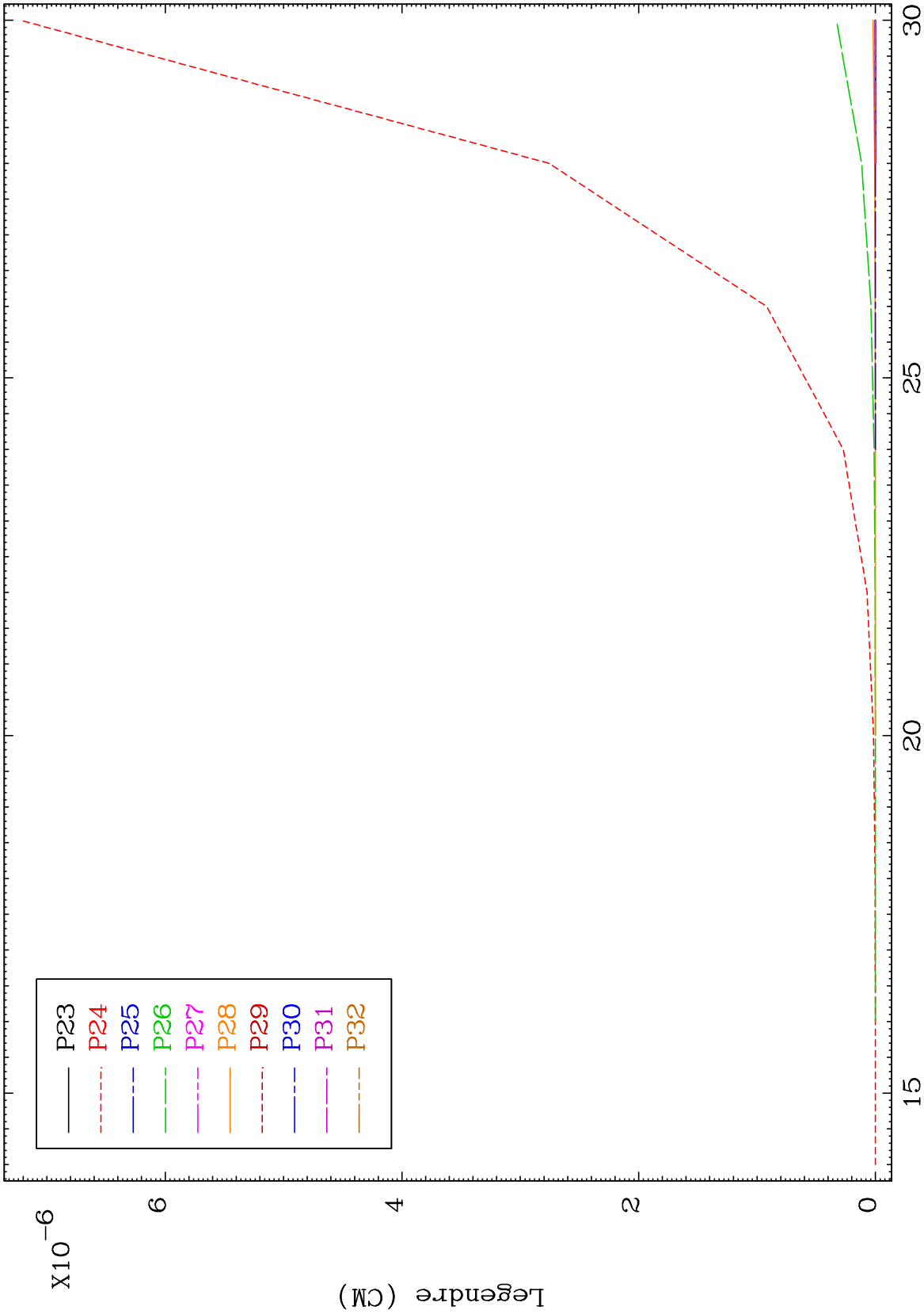
19

101-Md-249

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Elastic Legendre Coefficients

101-Md-249



20

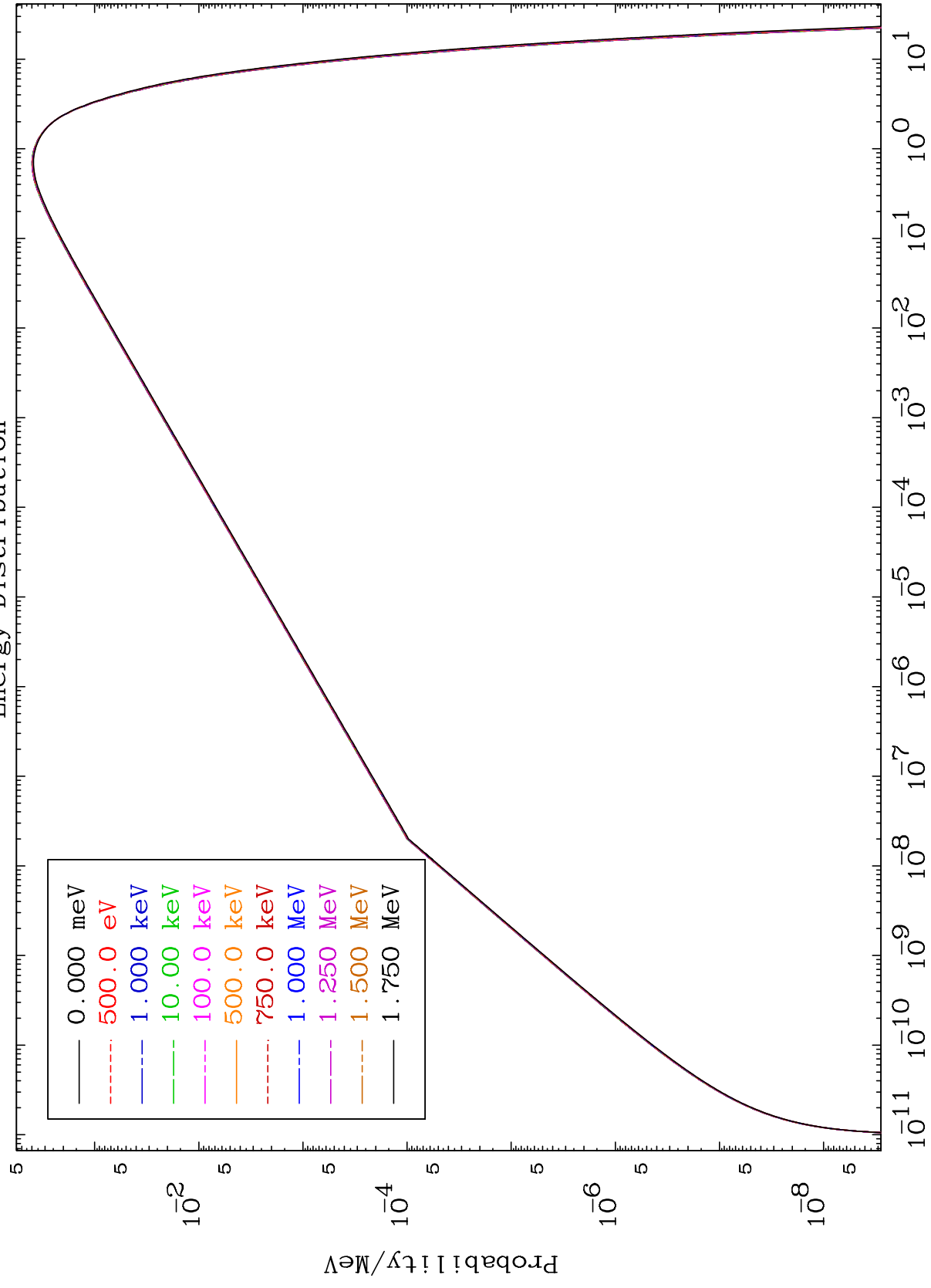
Incident Energy (MeV)

101-Md-249

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### Fission Energy Distribution

101-Md-249

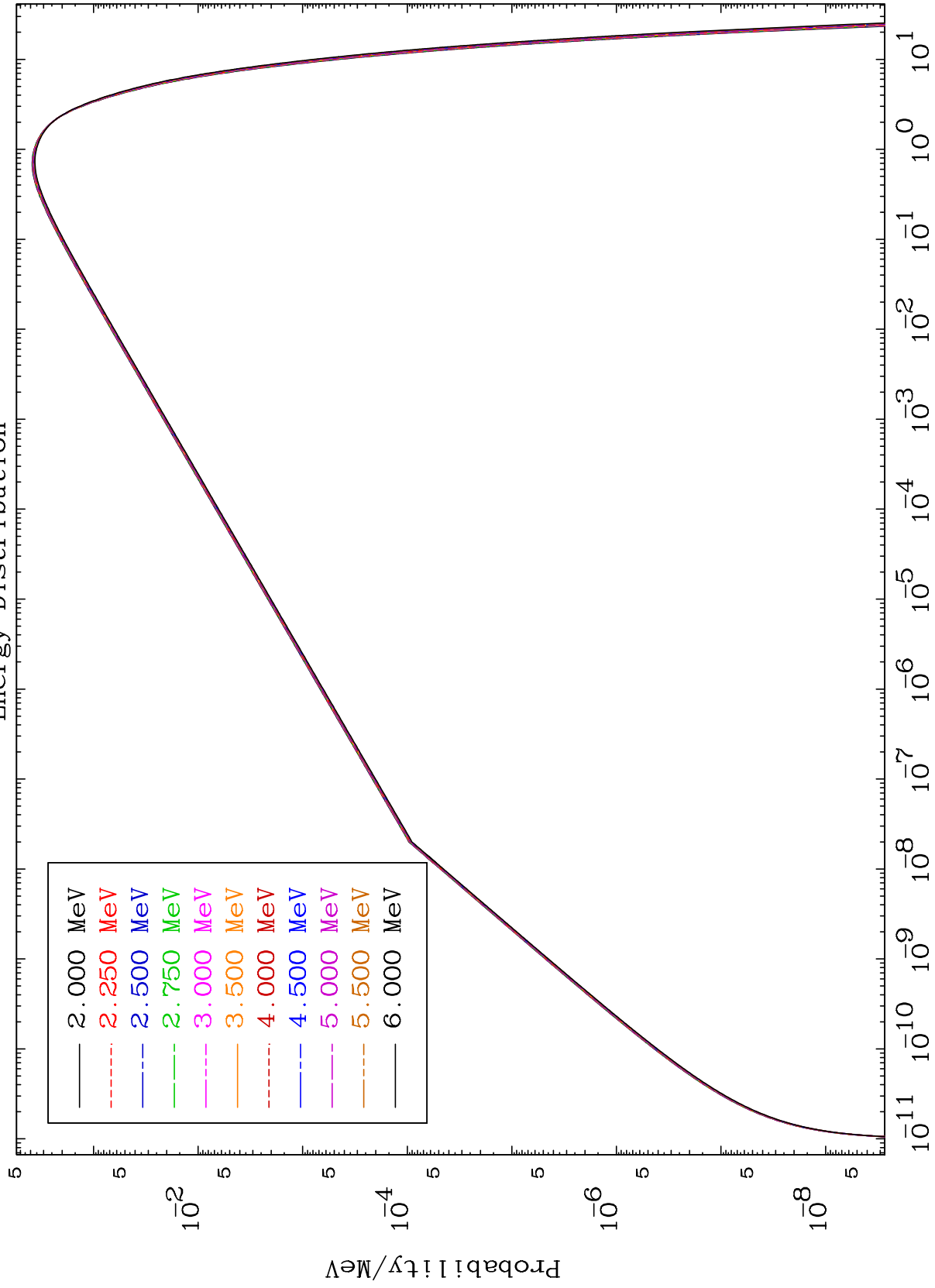


101-Md-249

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# Fission Energy Distribution

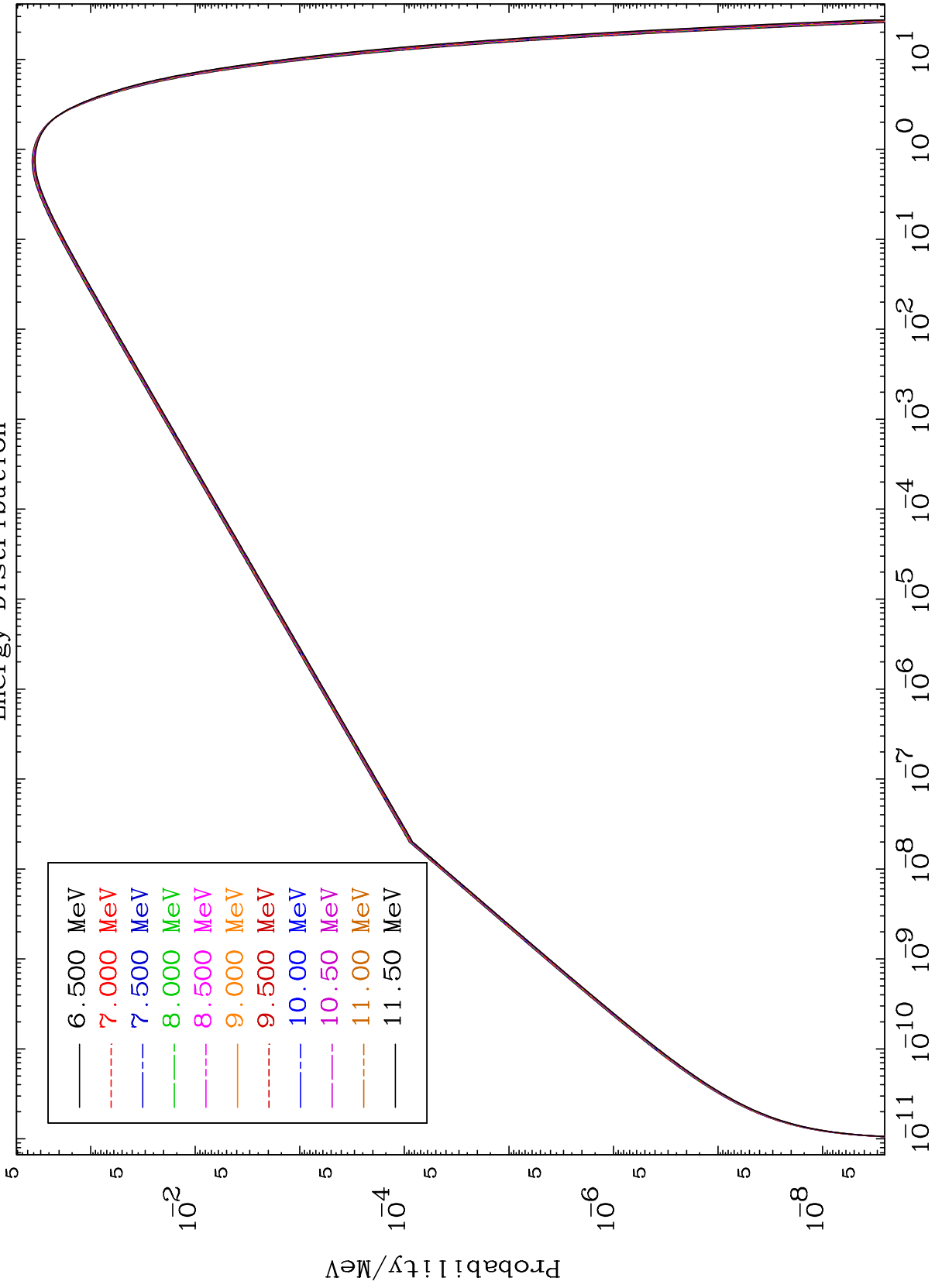
101-Md-249



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# Fission Energy Distribution

101-Md-249



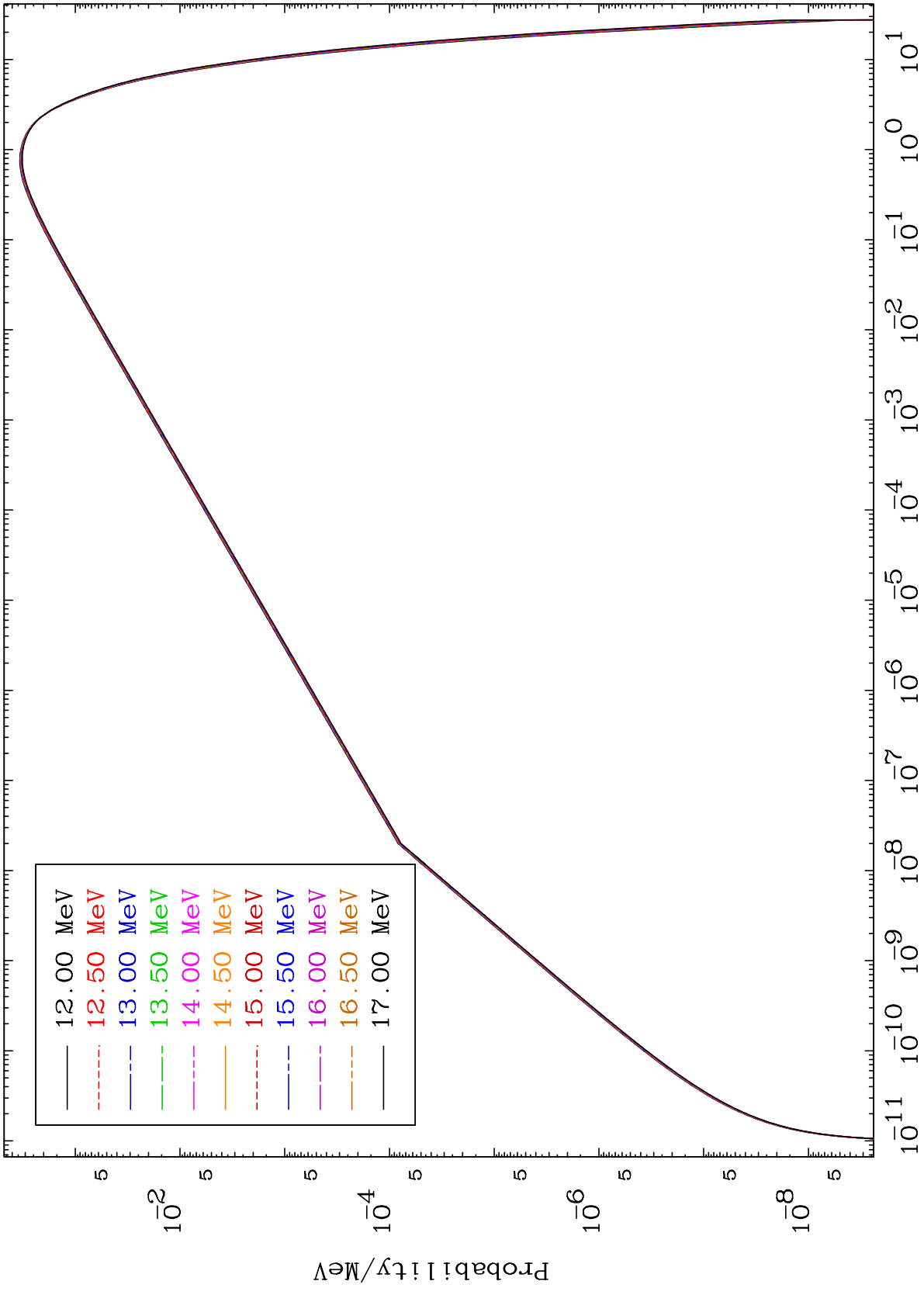
101-Md-249



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Fission  
Energy Distribution

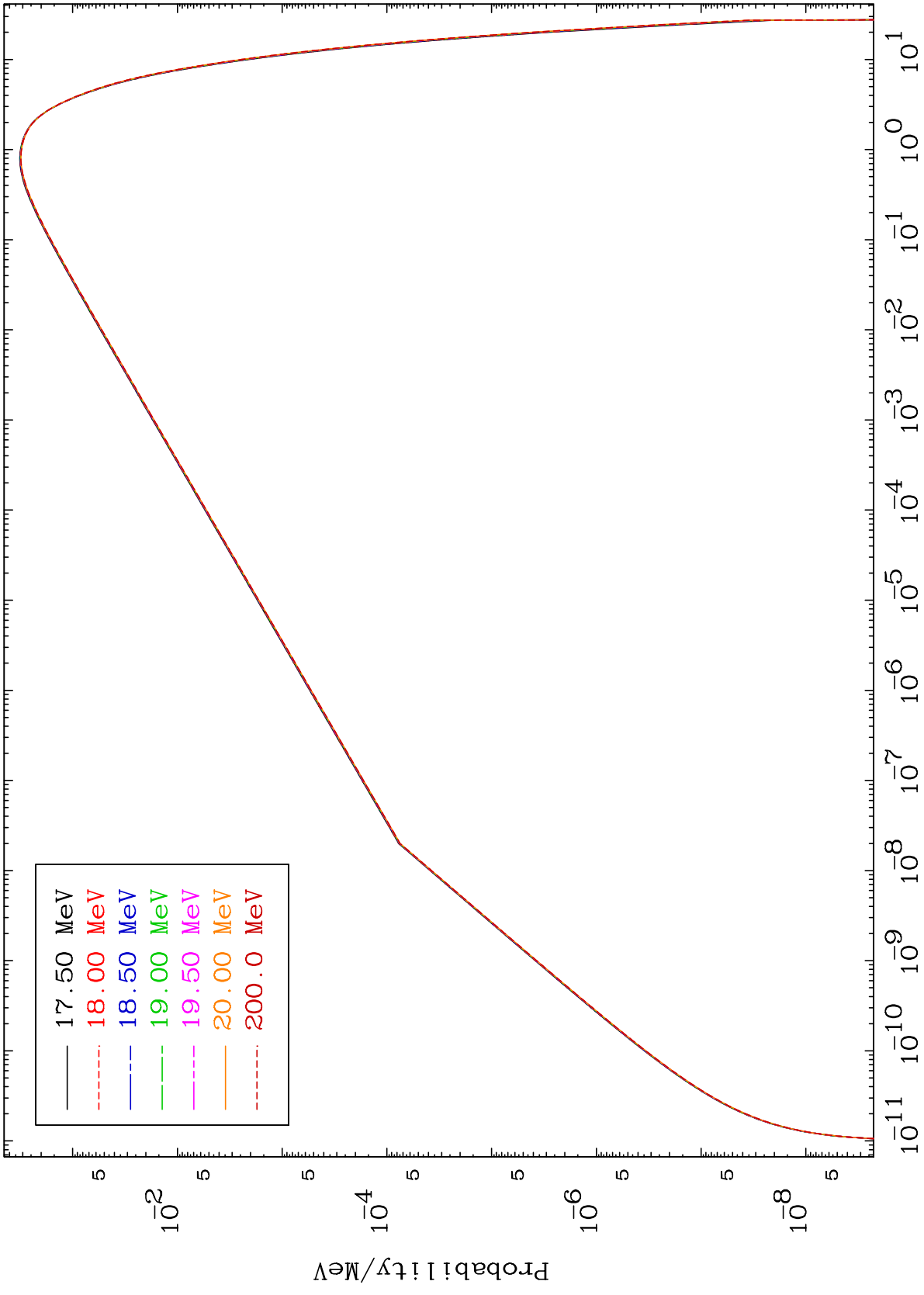
101-Md-249



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### Fission Energy Distribution

101-Md-249

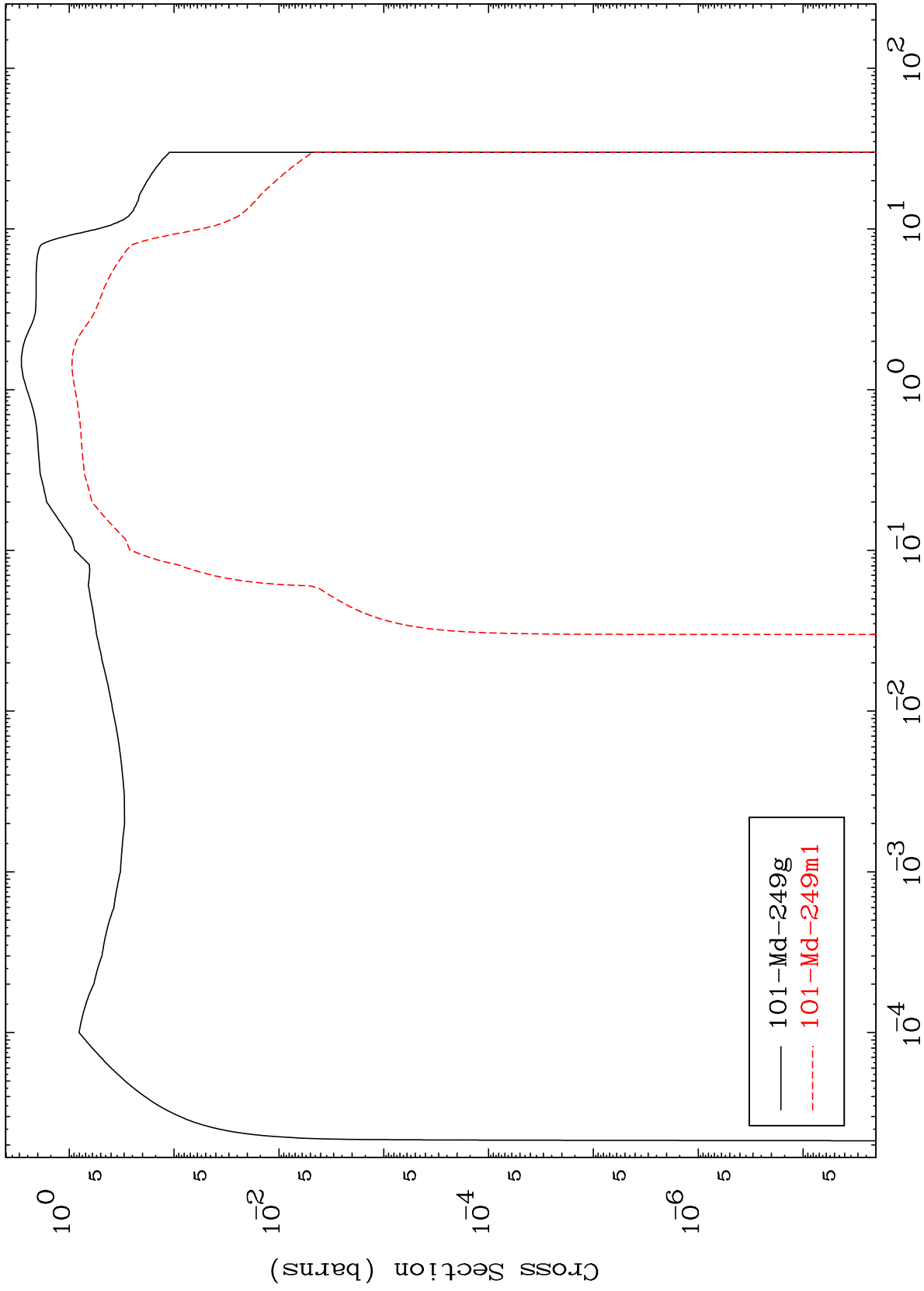


101-Md-249

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101-Md-249

Inelastic  
Radionuclide Production Cross Section



26

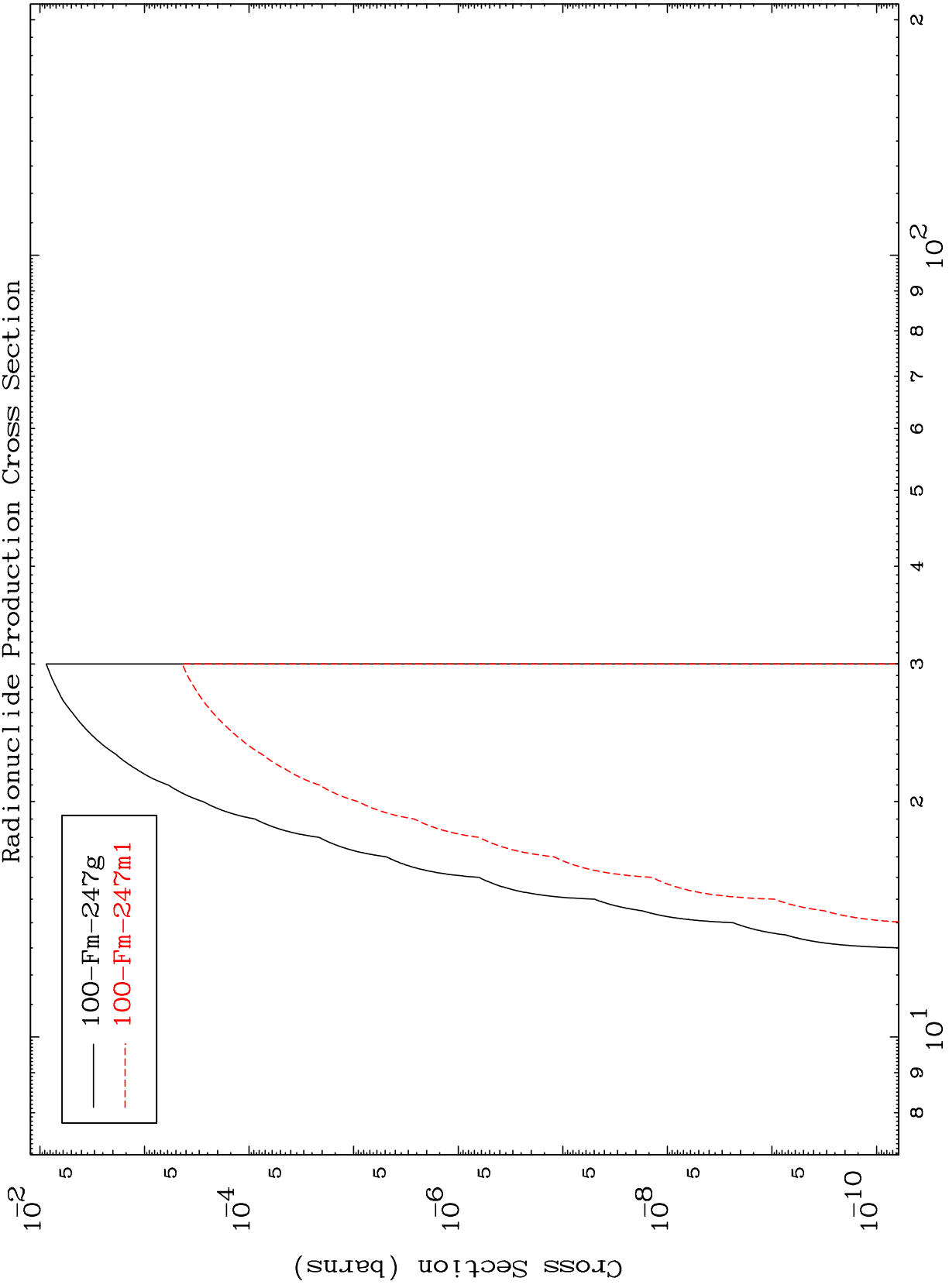
101-Md-249

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(n,n') d

101-Md-249

Radionuclide Production Cross Section



27

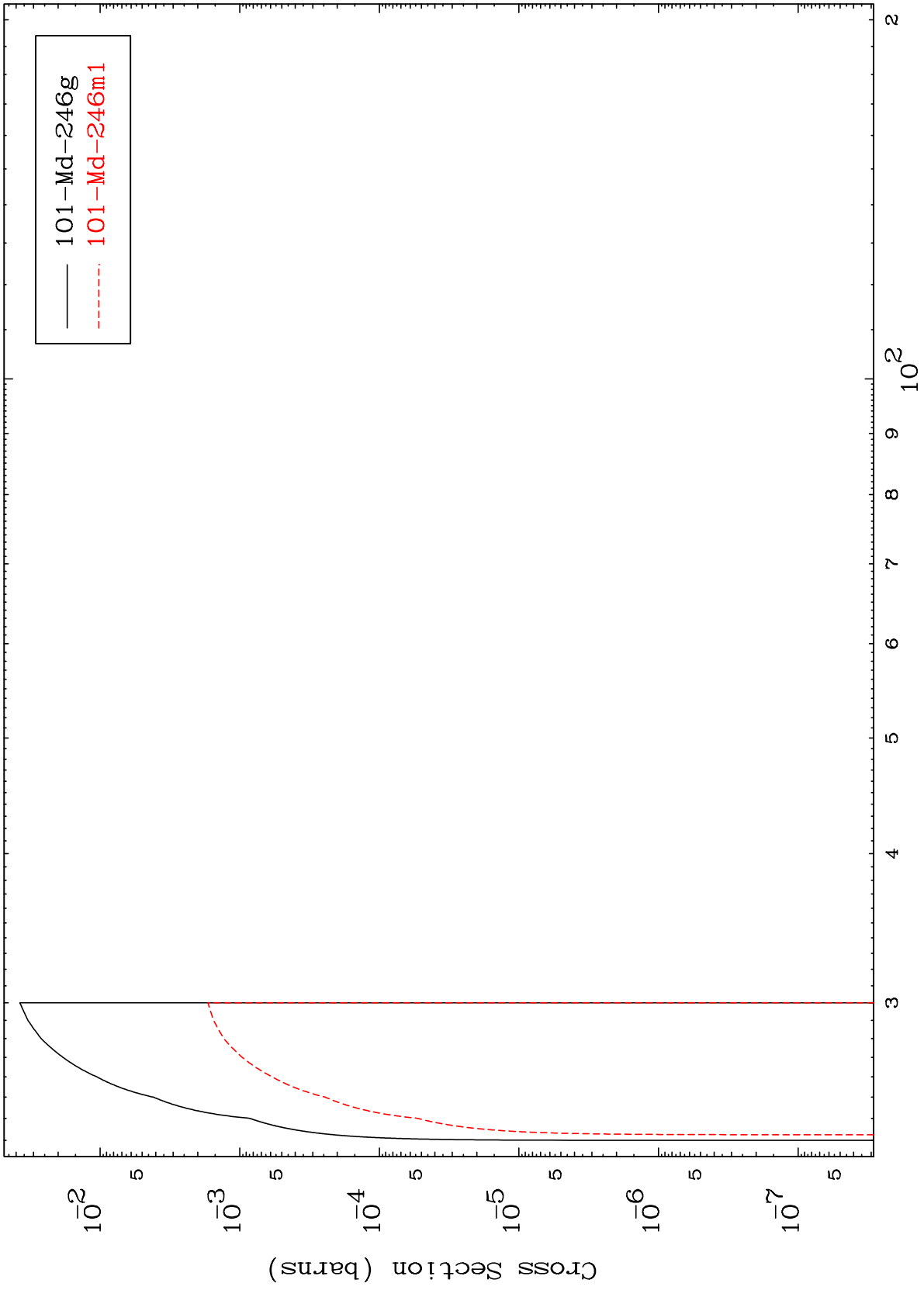
Incident Energy (MeV)

101-Md-249

MAT 9955

101-Md-249

(n,4n)  
Radionuclide Production Cross Section



28

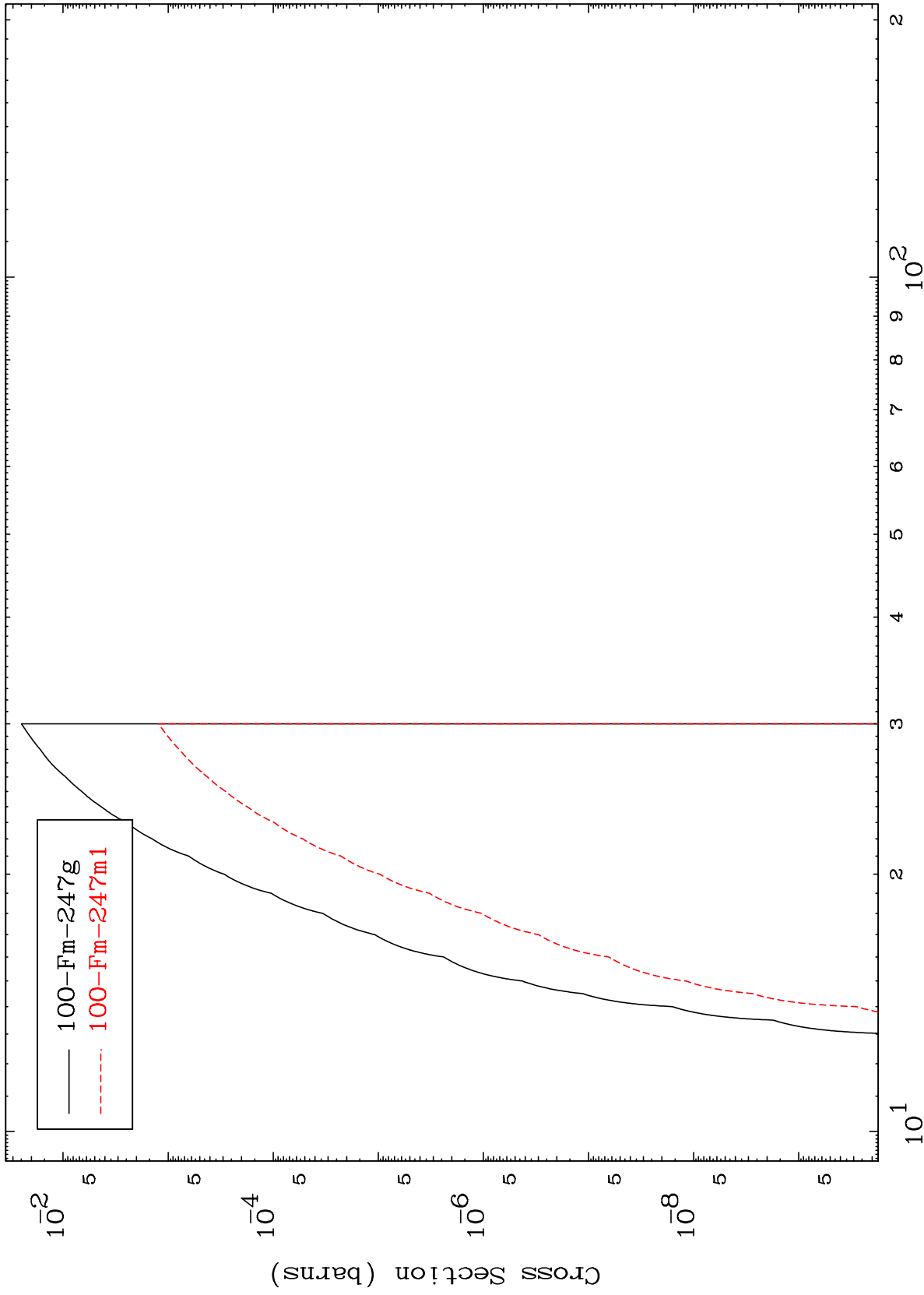
101-Md-249

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(n,2n) p

101-Md-249

Radionuclide Production Cross Section

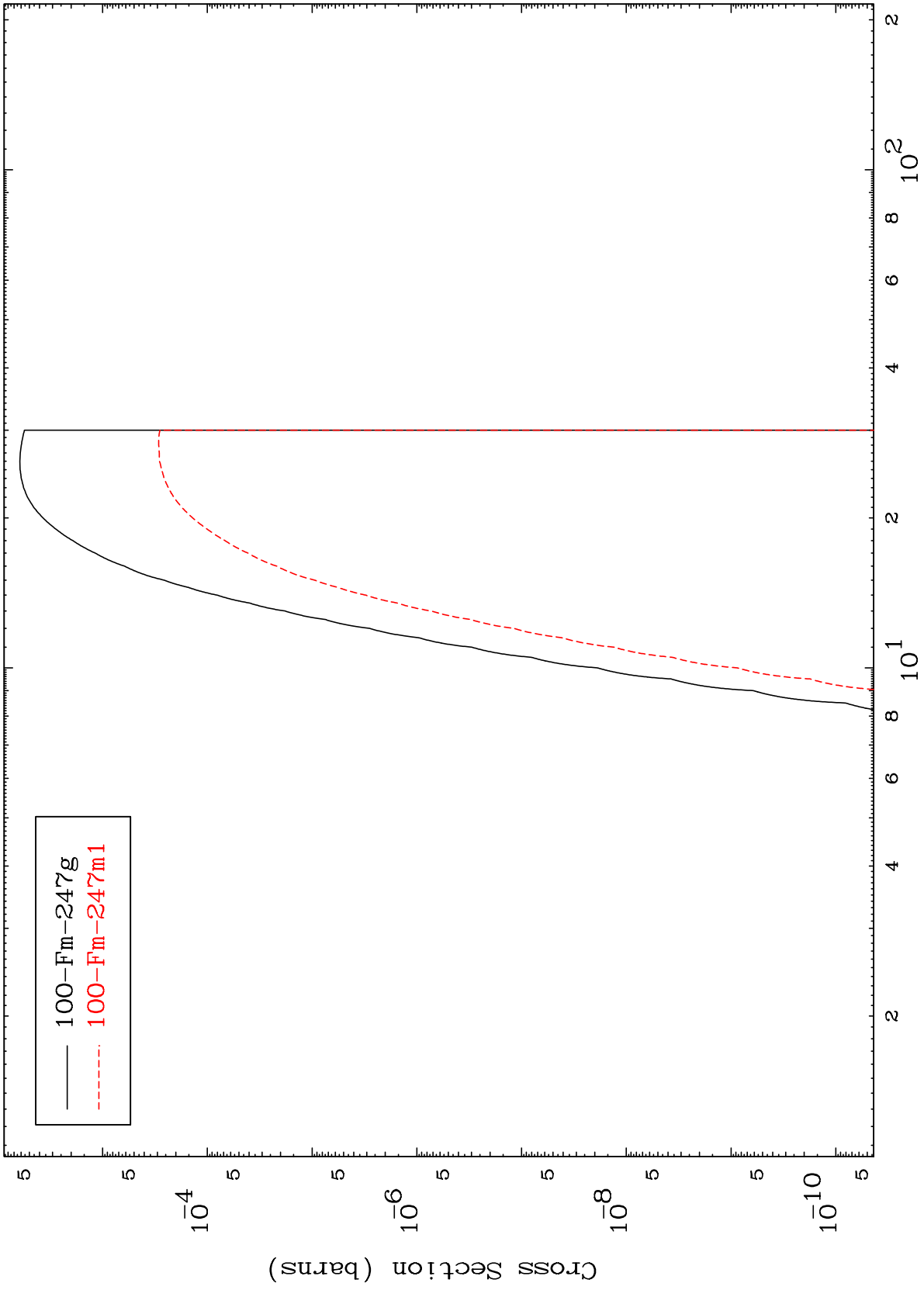


100-Fm-247g  
100-Fm-247m1

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(n, t)  
Radionuclide Production Cross Section

101-Md-249



— 100-Fm-247g  
- - - 100-Fm-247m1

30

Incident Energy (MeV)

101-Md-249