

Program EVALPLOT
(Version 2018-1)

by

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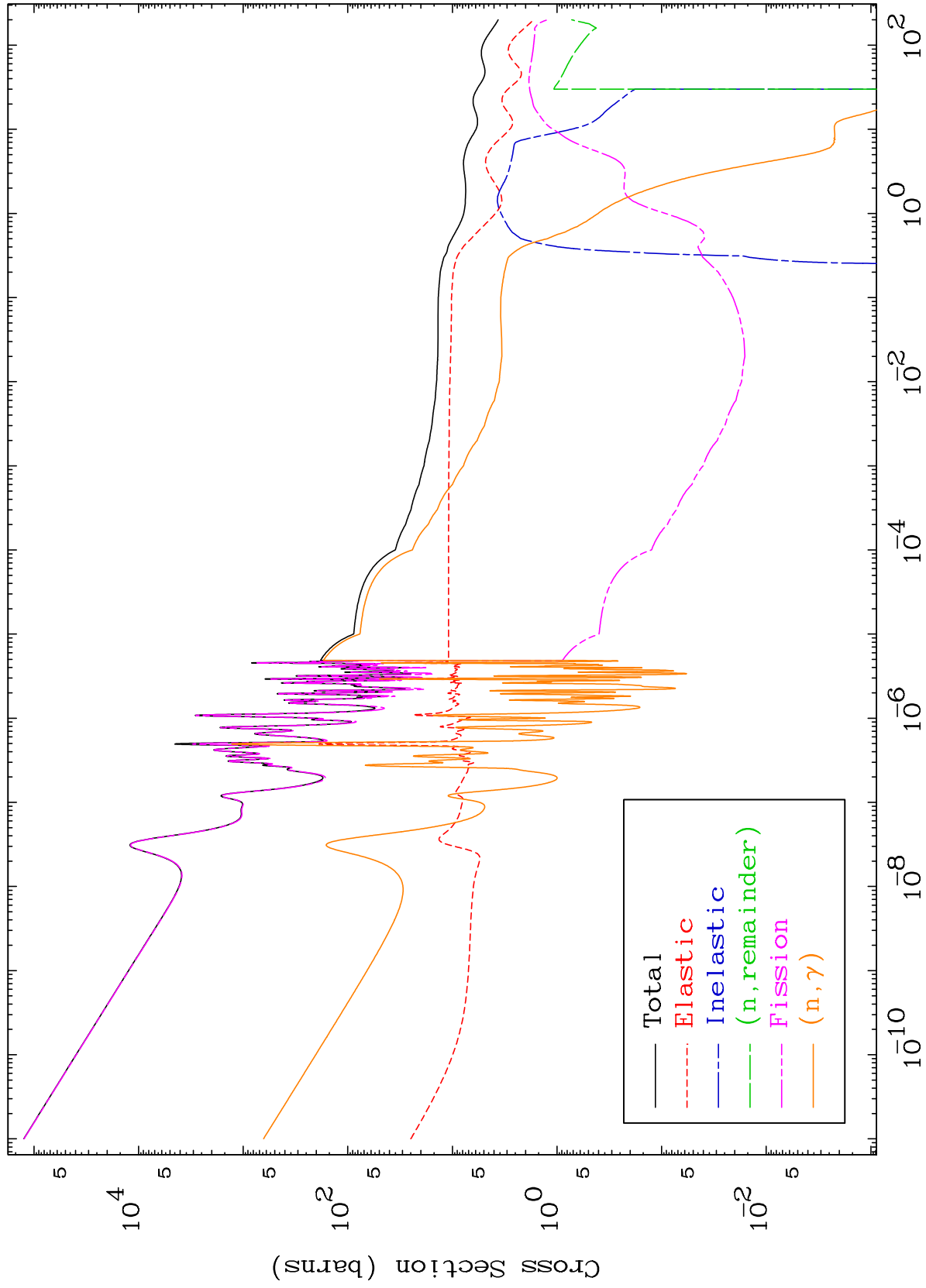
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9951

Major
293 Kelvin Cross Sections

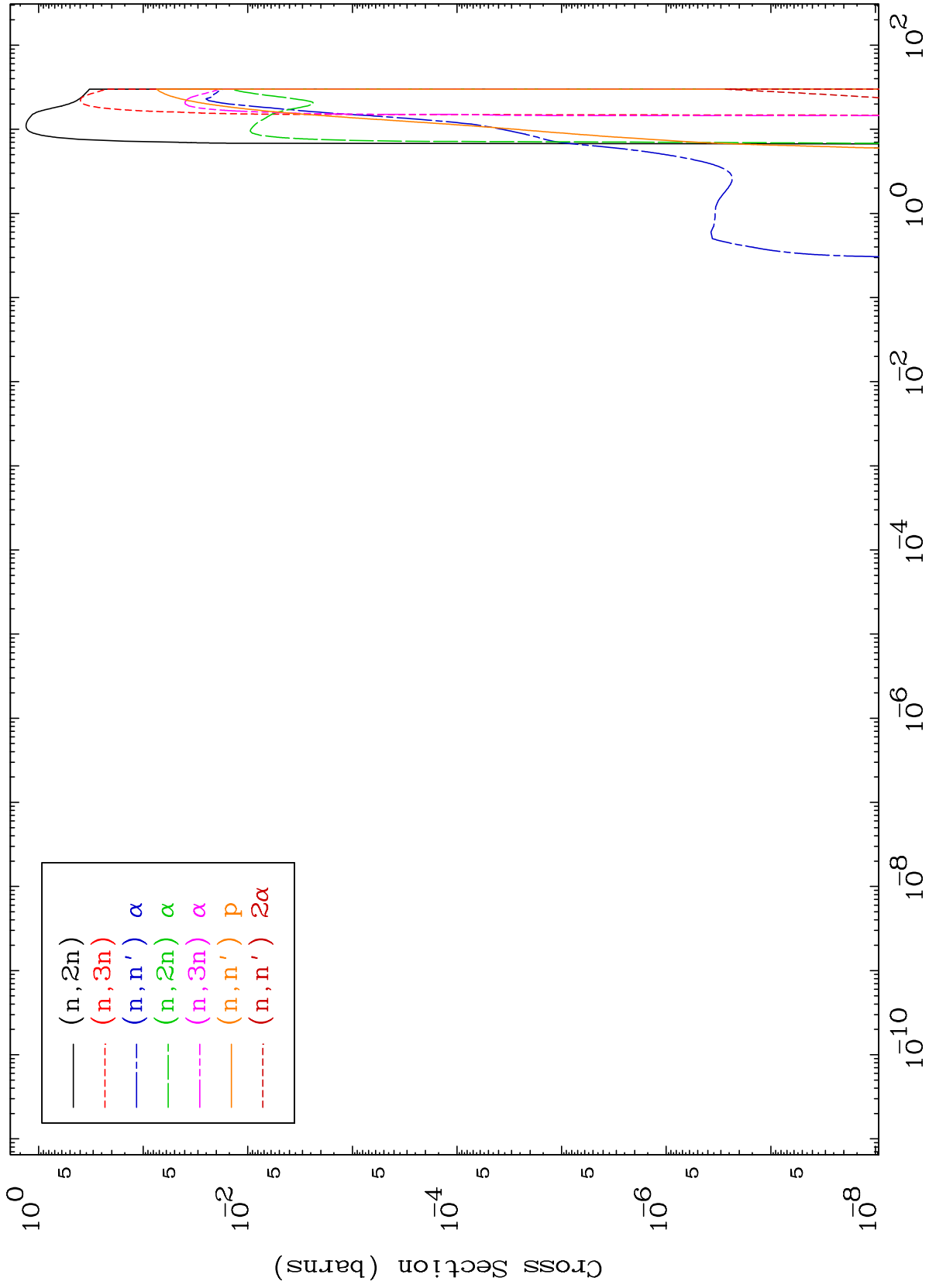
101-Md-250



MAT 9951

Neutron Production
293 Kelvin Cross Sections

101-Md-250



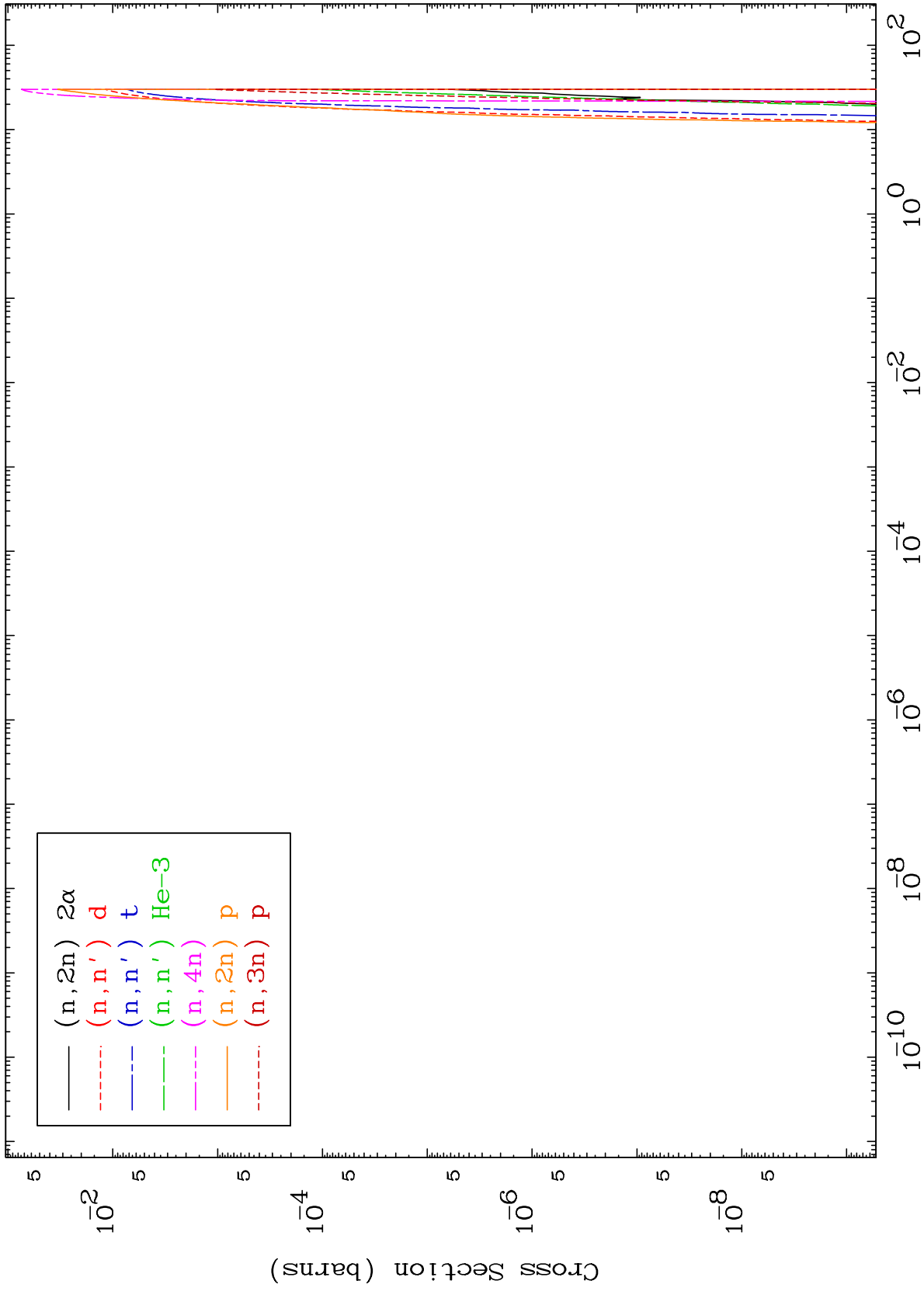
2

101-Md-250

MAT 9951

Neutron Production
293 Kelvin Cross Sections

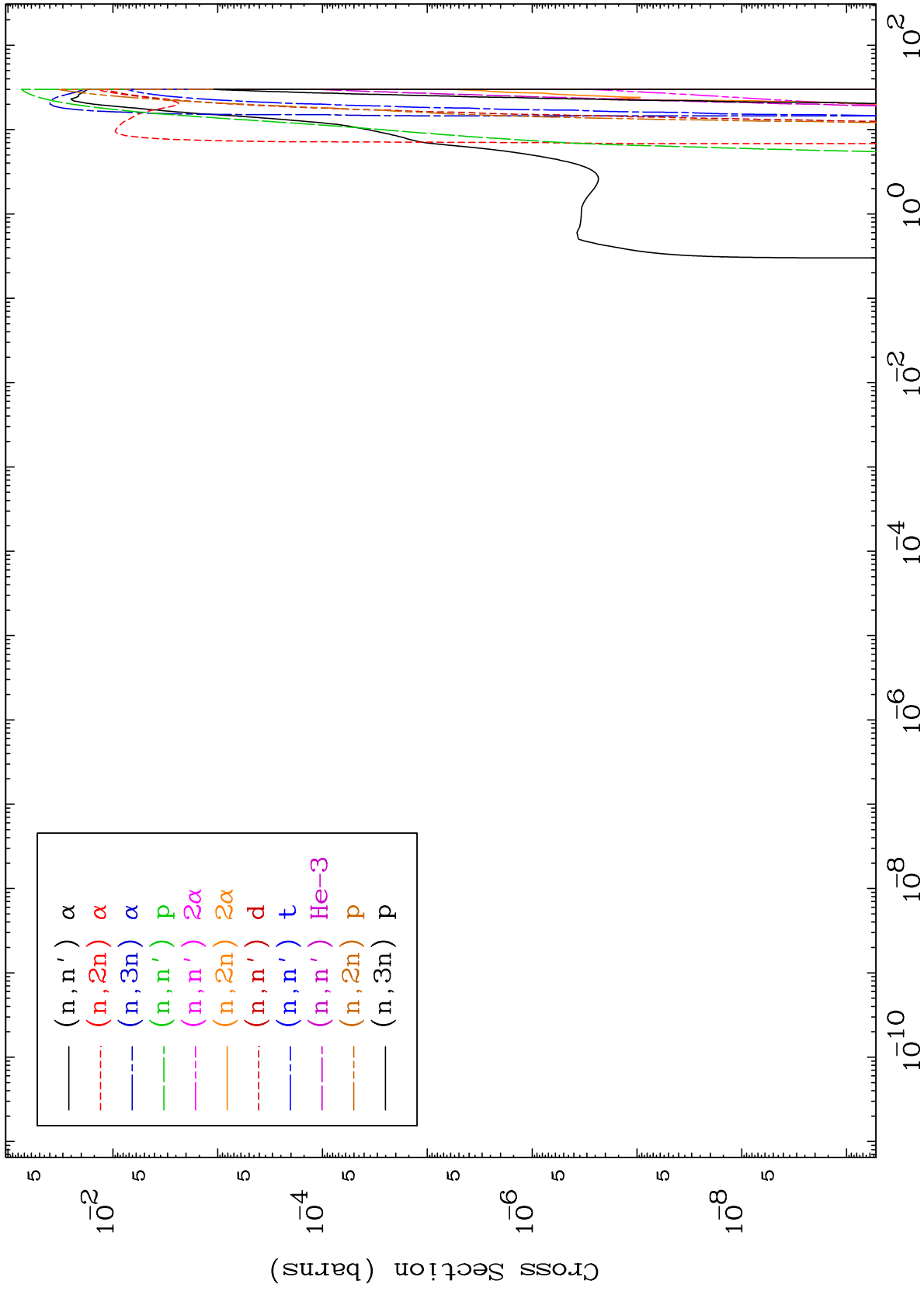
101-Md-250



MAT 9951

293 Kelvin Cross Sections

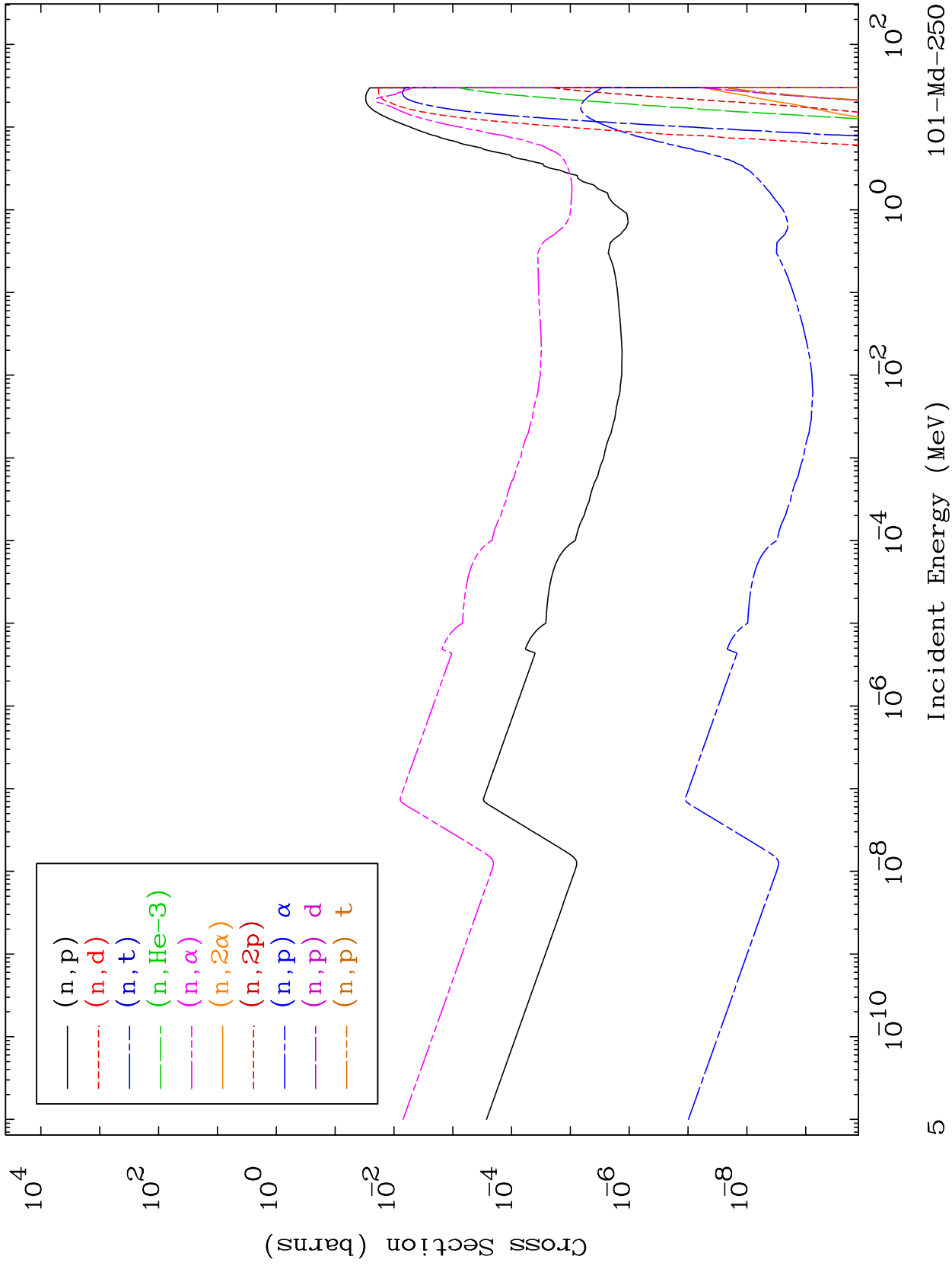
101-Md-250



MAT 9951

293 Kelvin Cross Sections

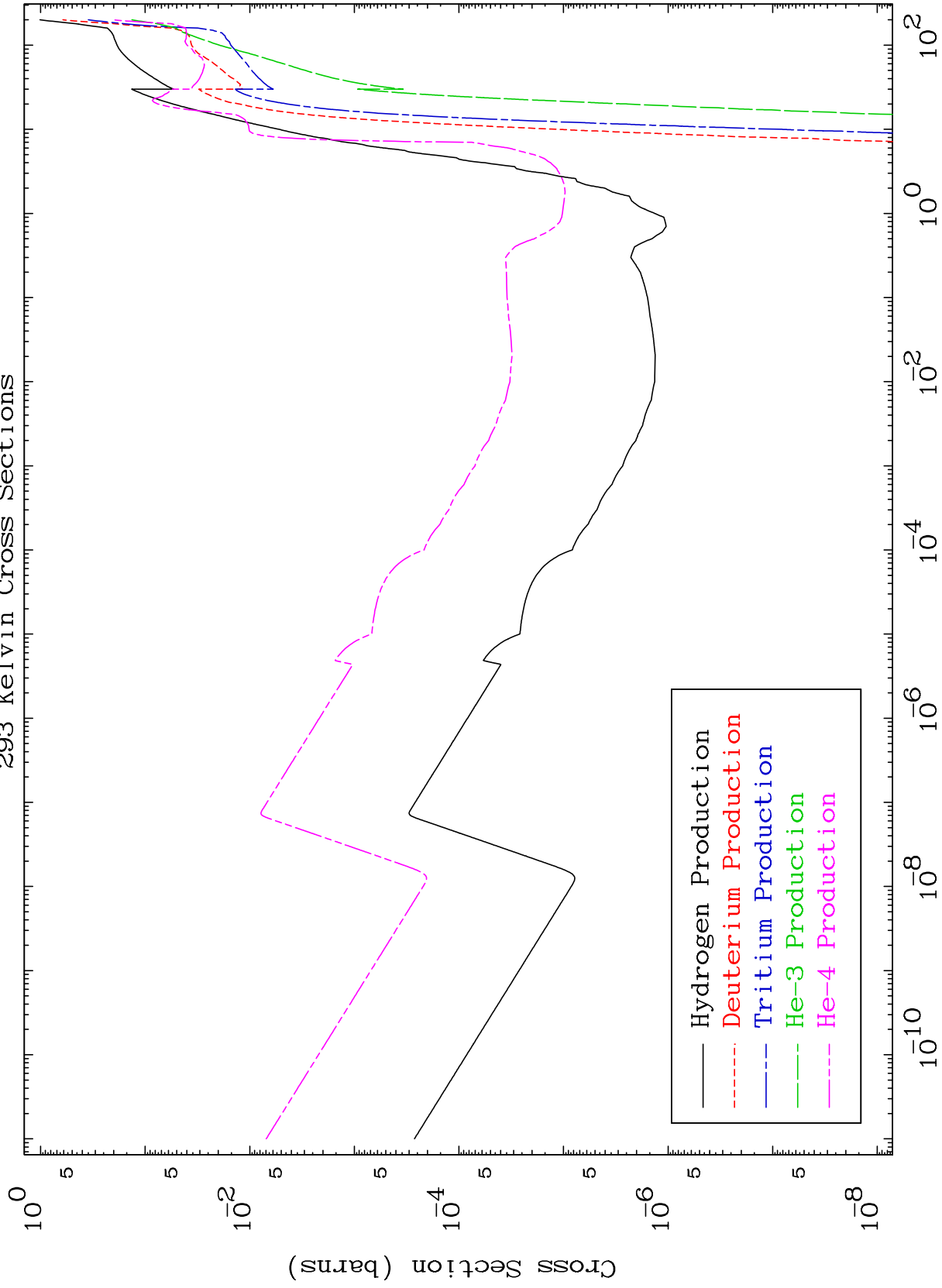
101-Md-250



MAT 9951

Particle Production
293 Kelvin Cross Sections

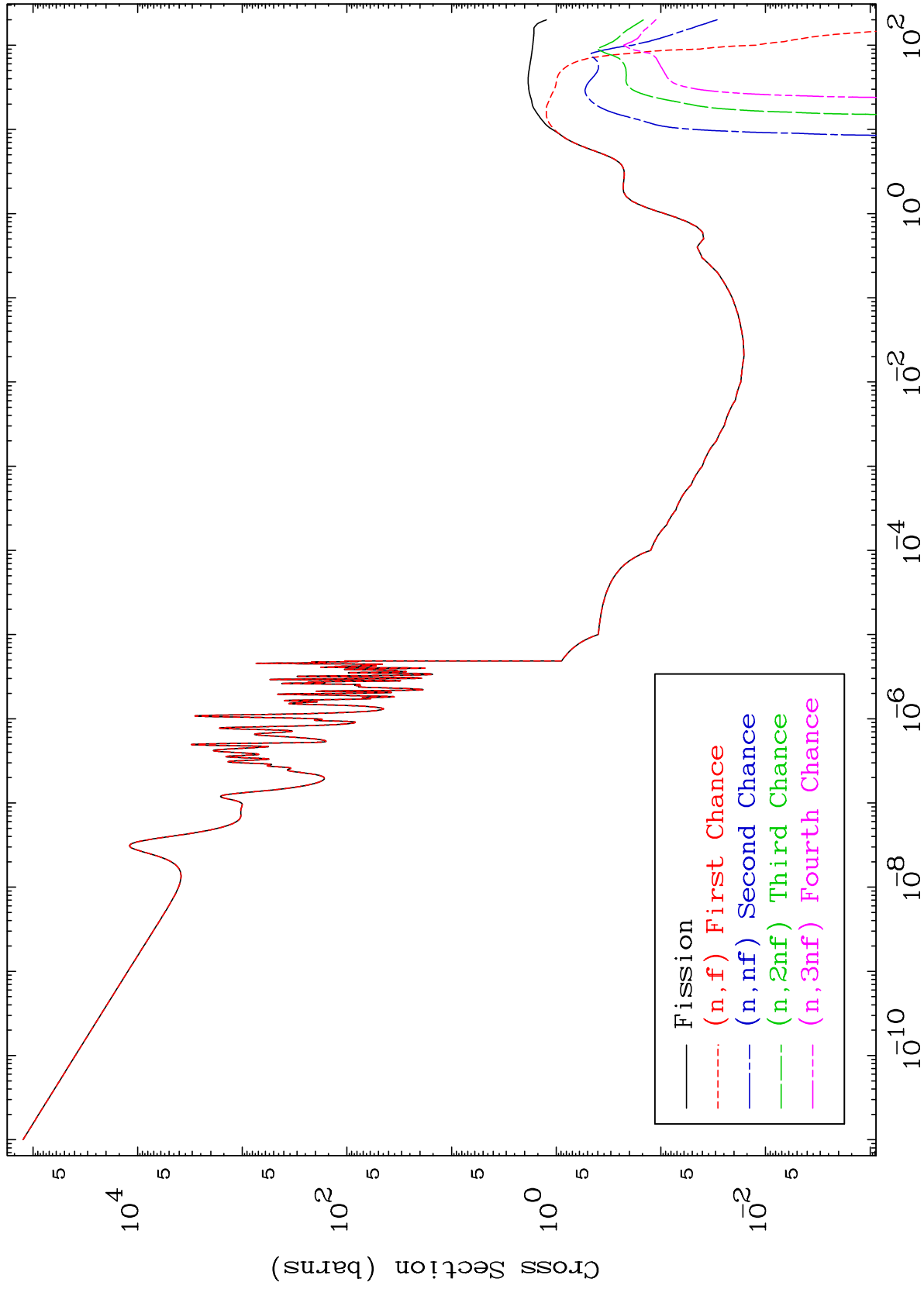
101-Md-250



MAT 9951

Fission
293 Kelvin Cross Sections

101-Md-250

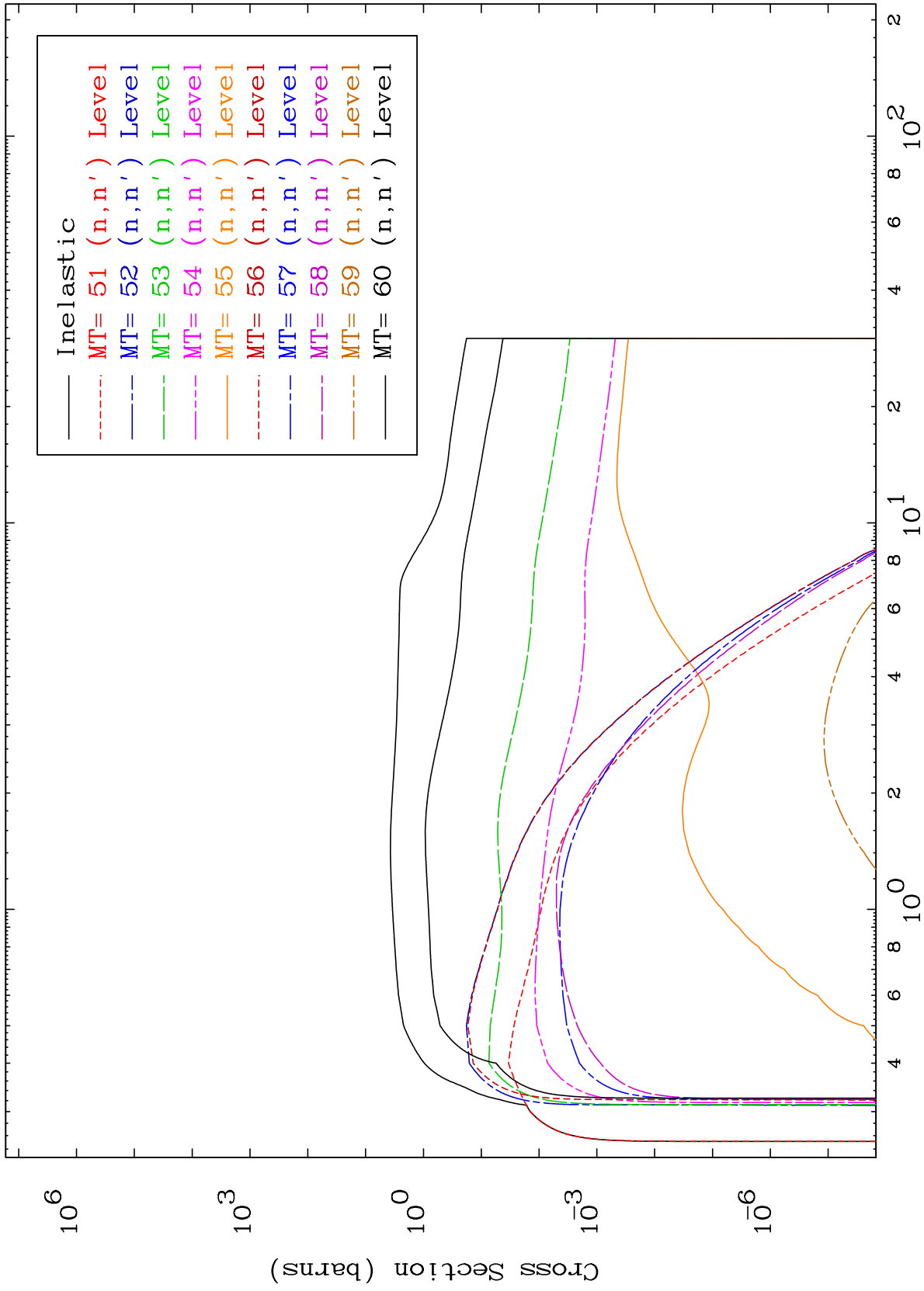


MAT 9951

(n,n') Level

101-Md-250

293 Kelvin Cross Sections



8

Incident Energy (MeV)

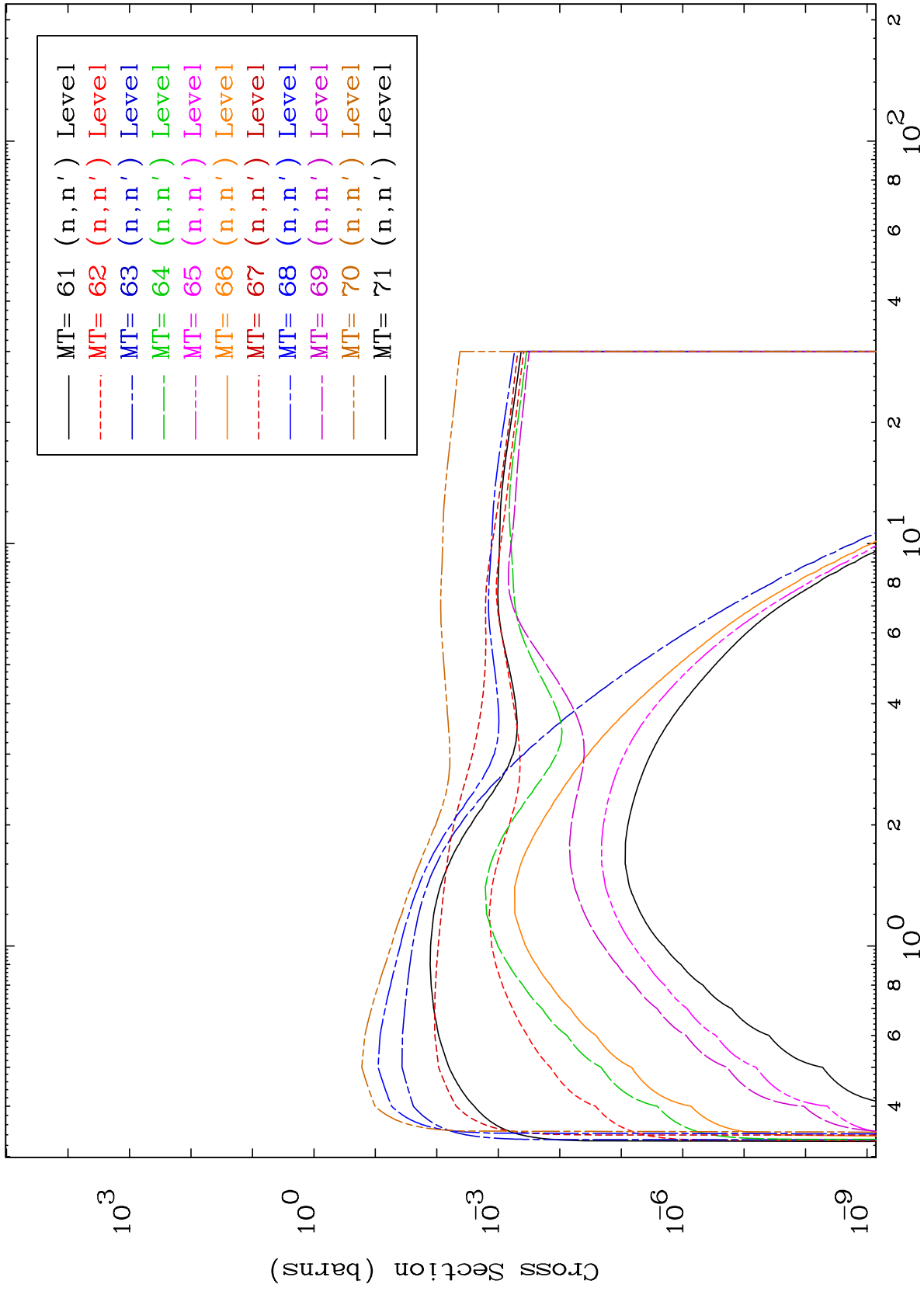
101-Md-250

MAT 9951

(n,n') Level

101-Md-250

293 Kelvin Cross Sections



9

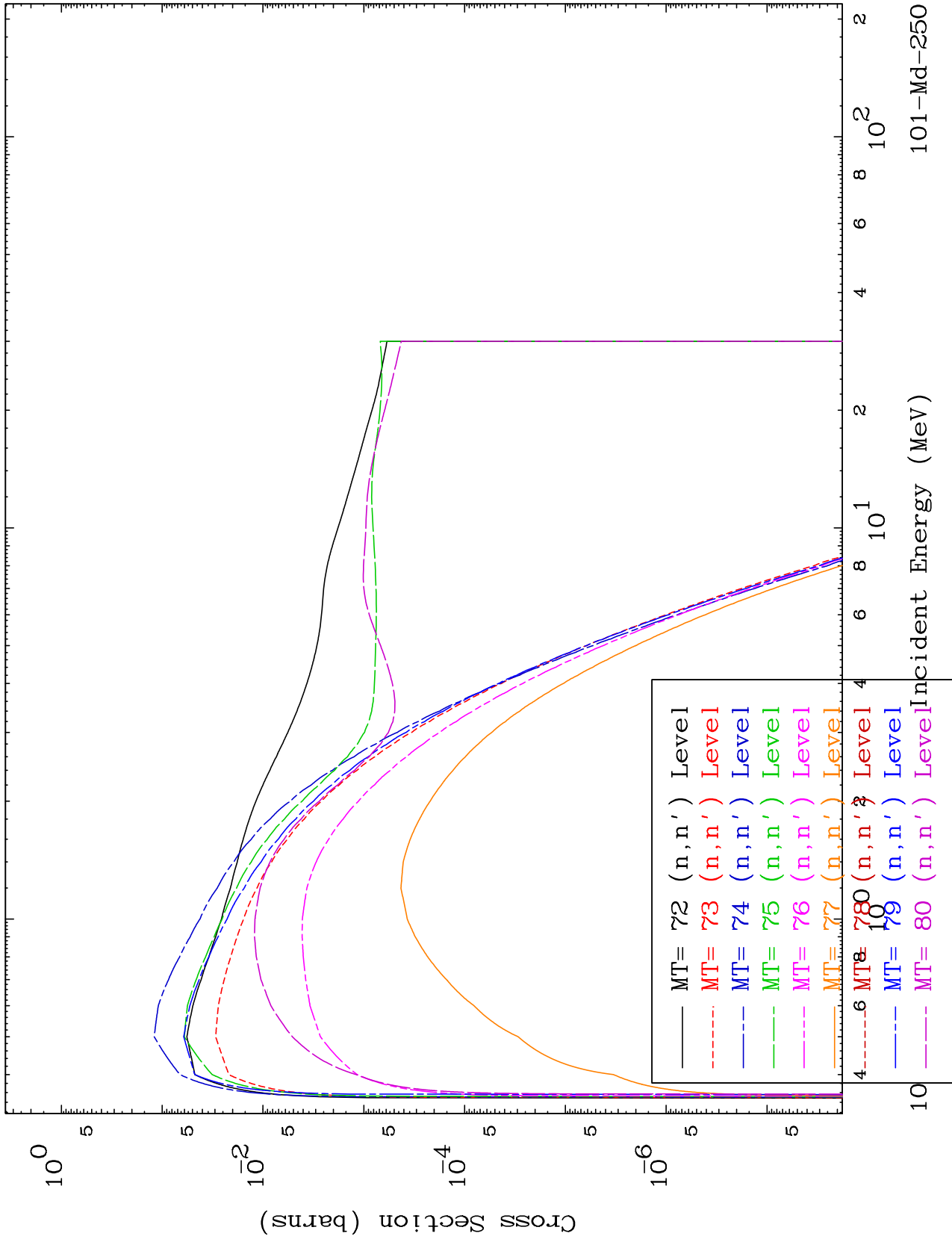
Incident Energy (MeV)

101-Md-250

MAT 9951

(n,n') Level
293 Kelvin Cross Sections

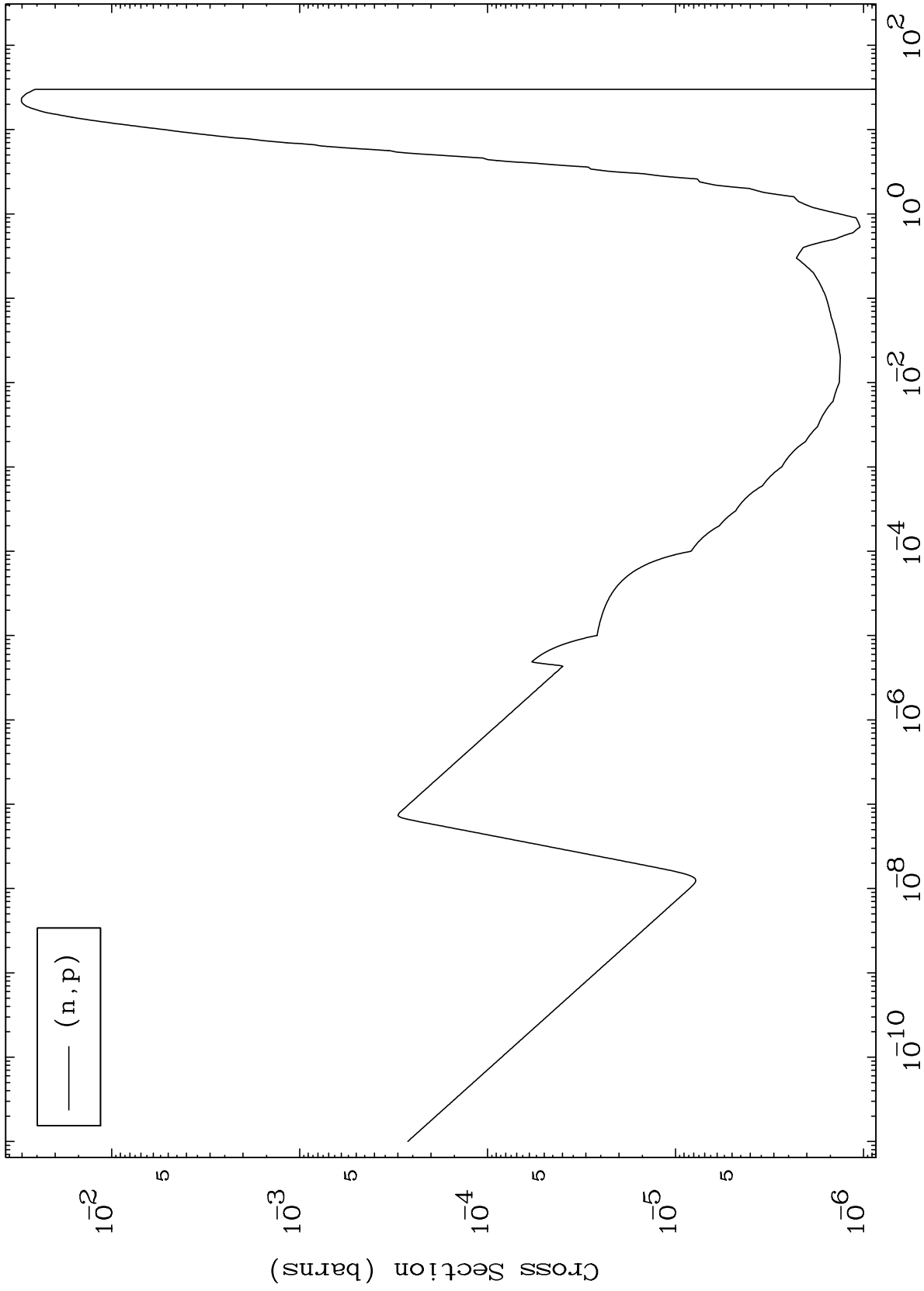
101-Md-250



MAT 9951

(n,p) Levels
293 Kelvin Cross Sections

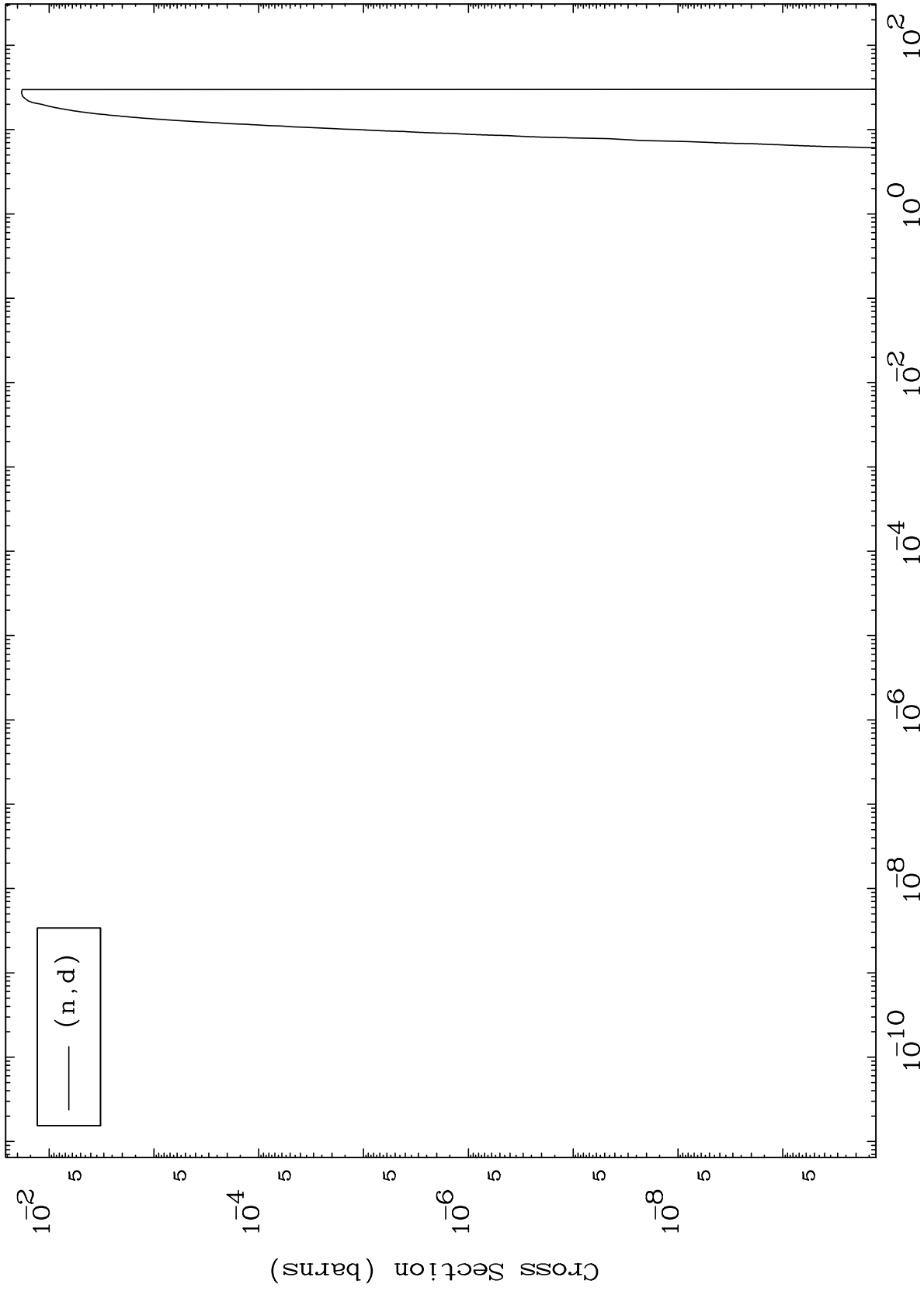
101-Md-250



MAT 9951

(n,d) Levels
293 Kelvin Cross Sections

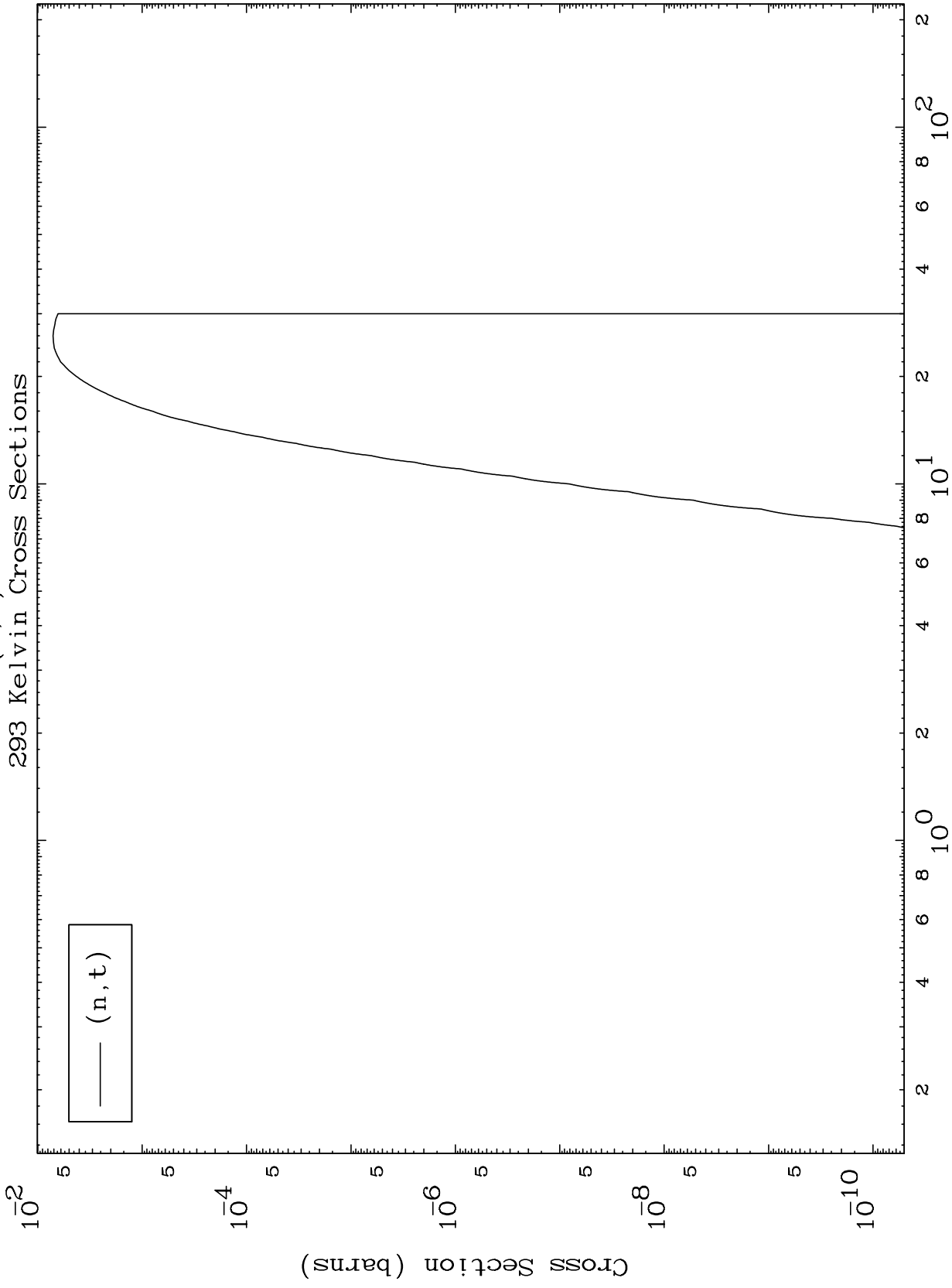
101-Md-250



MAT 9951

(n,t) Levels

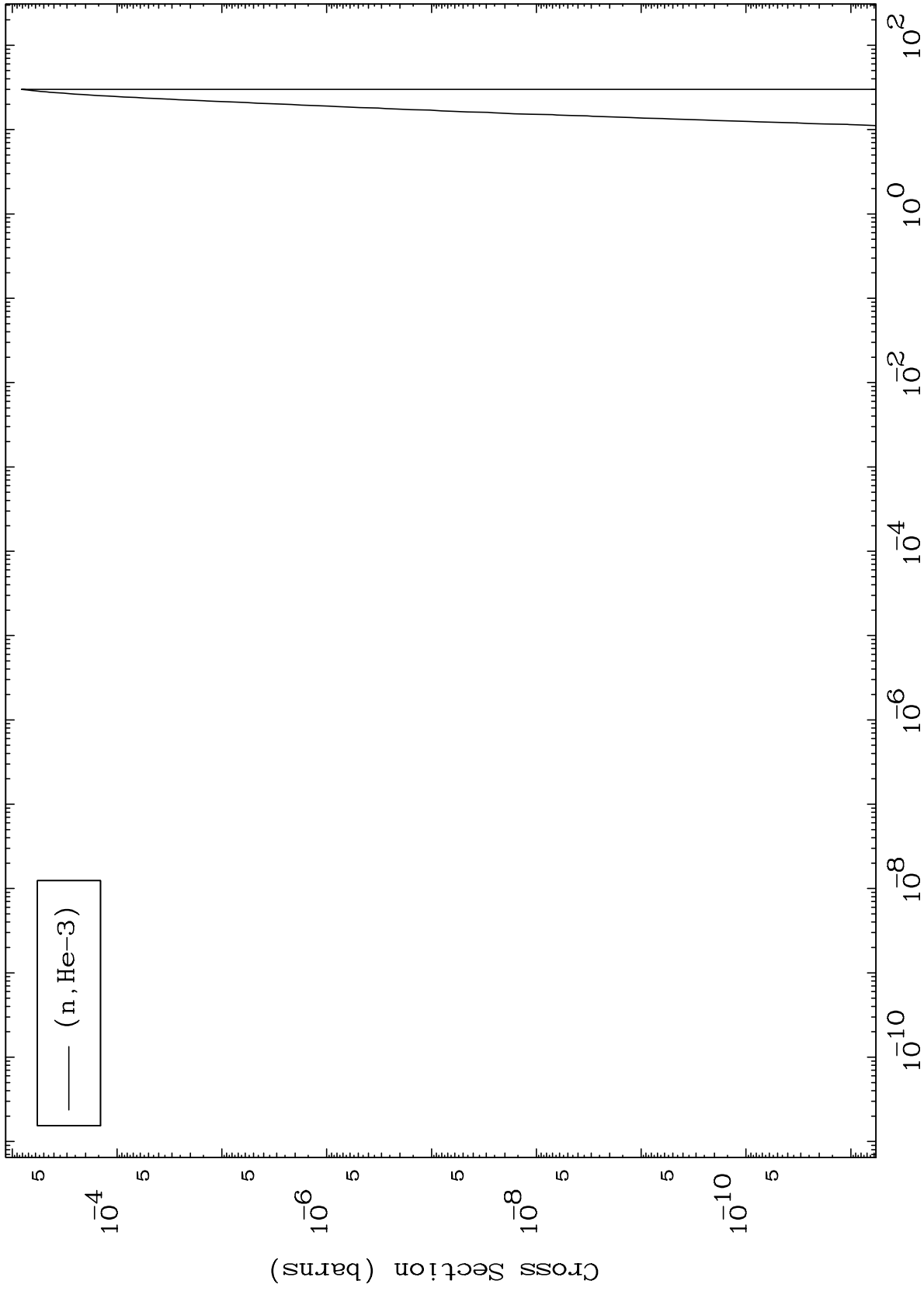
101-Md-250



MAT 9951

(n,He3) Levels
293 Kelvin Cross Sections

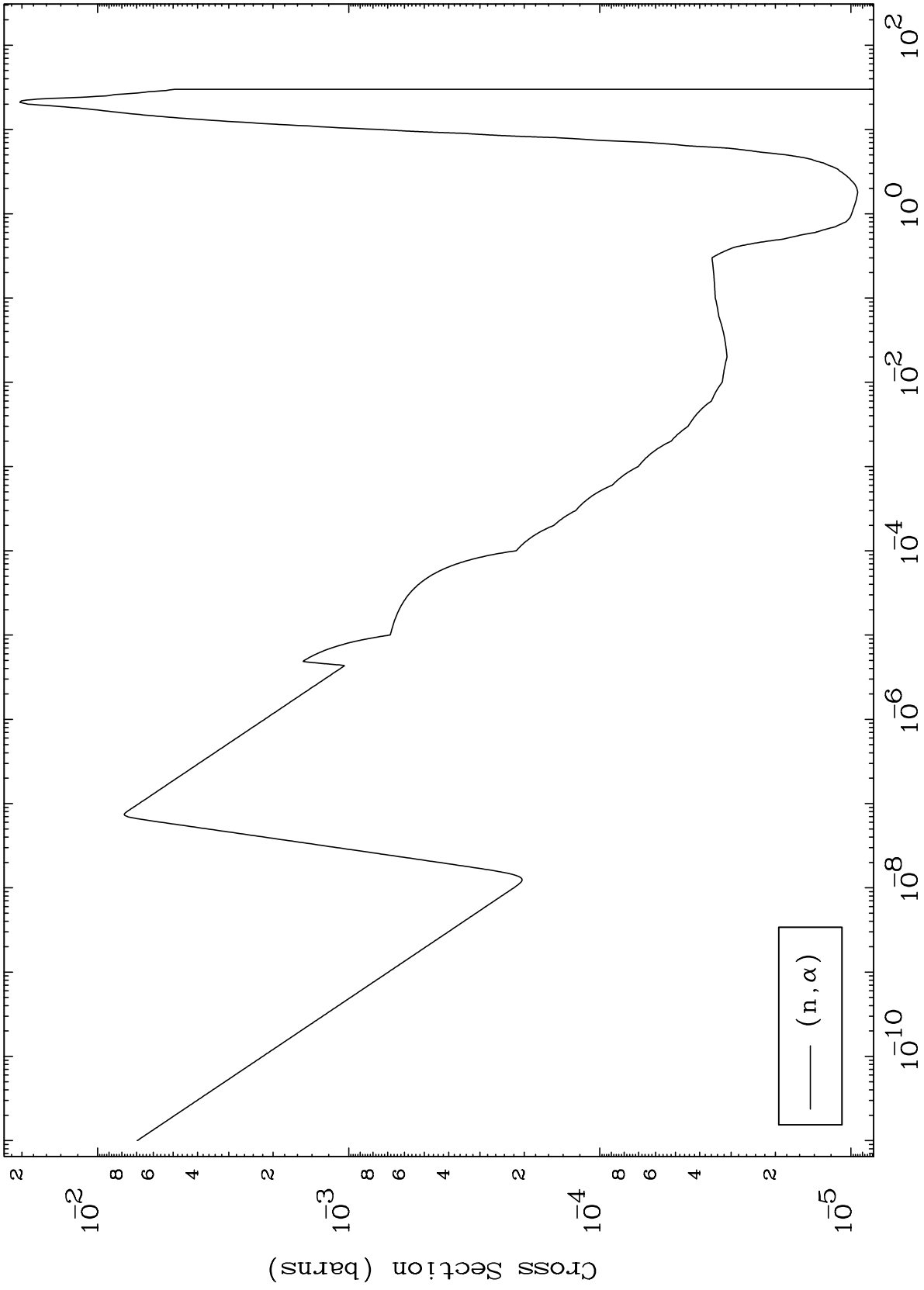
101-Md-250



MAT 9951

(n,α) Levels
293 Kelvin Cross Sections

101-Md-250

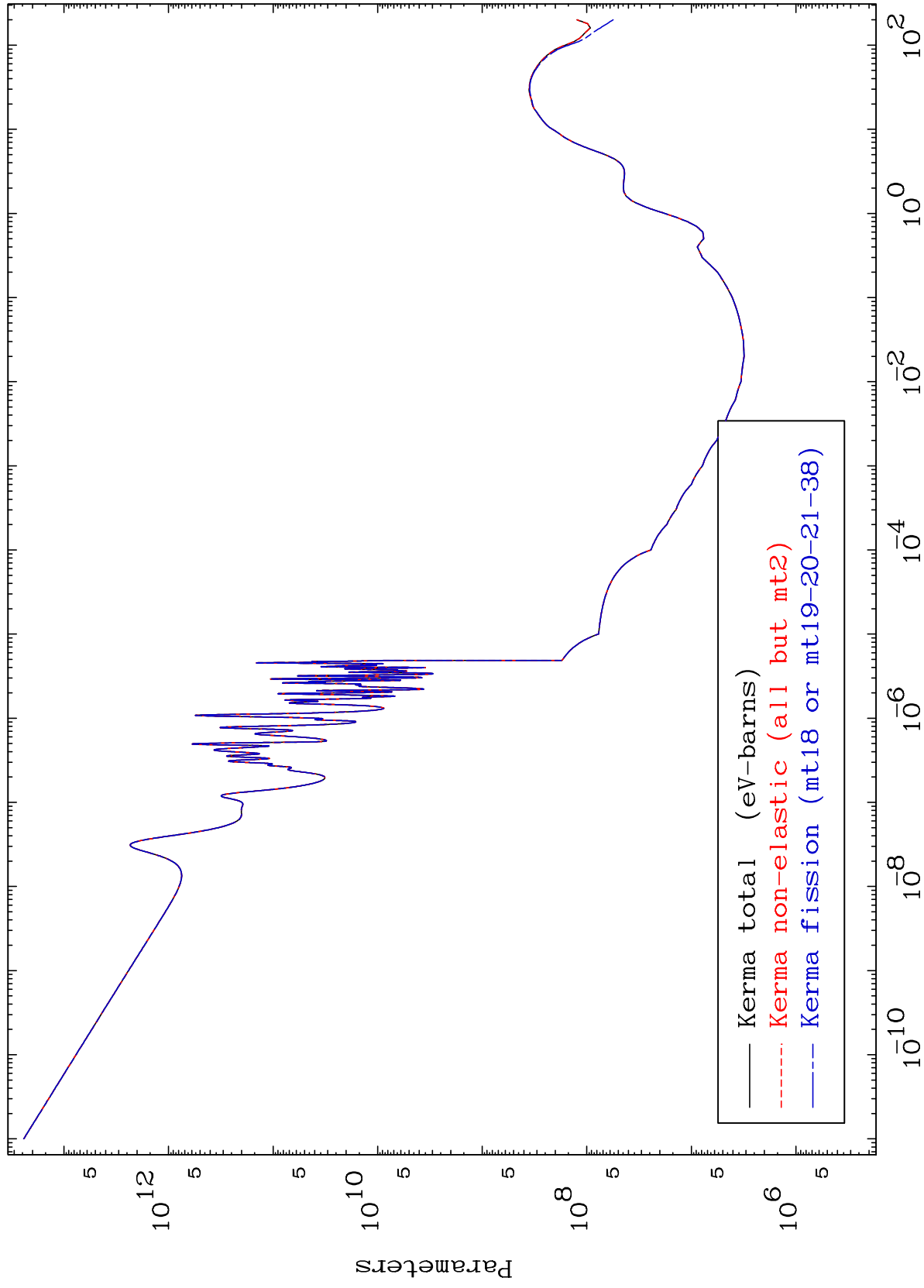


— (n,α)

MAT 9951

Energy Release
Parameters

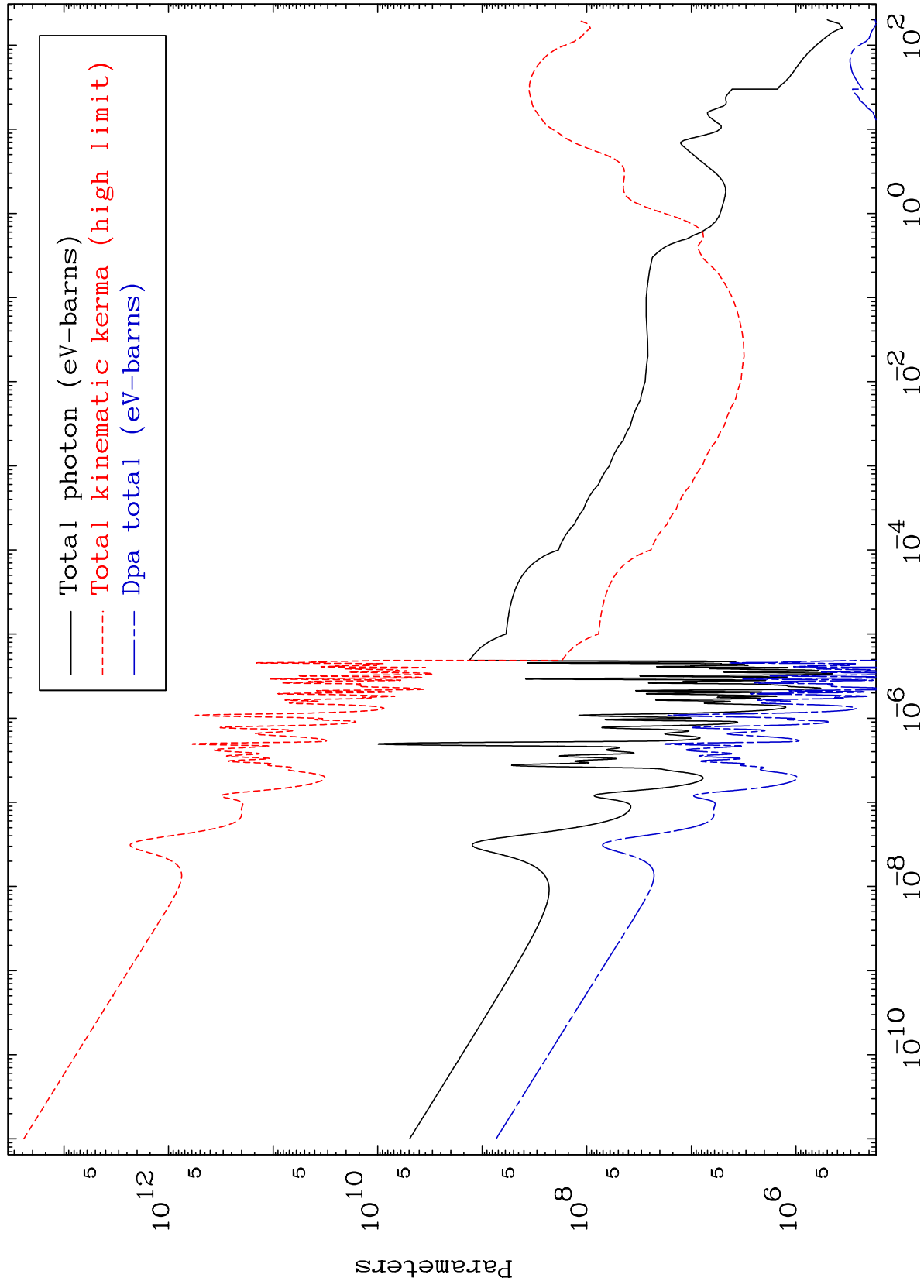
101-Md-250



16

Incident Energy (MeV)

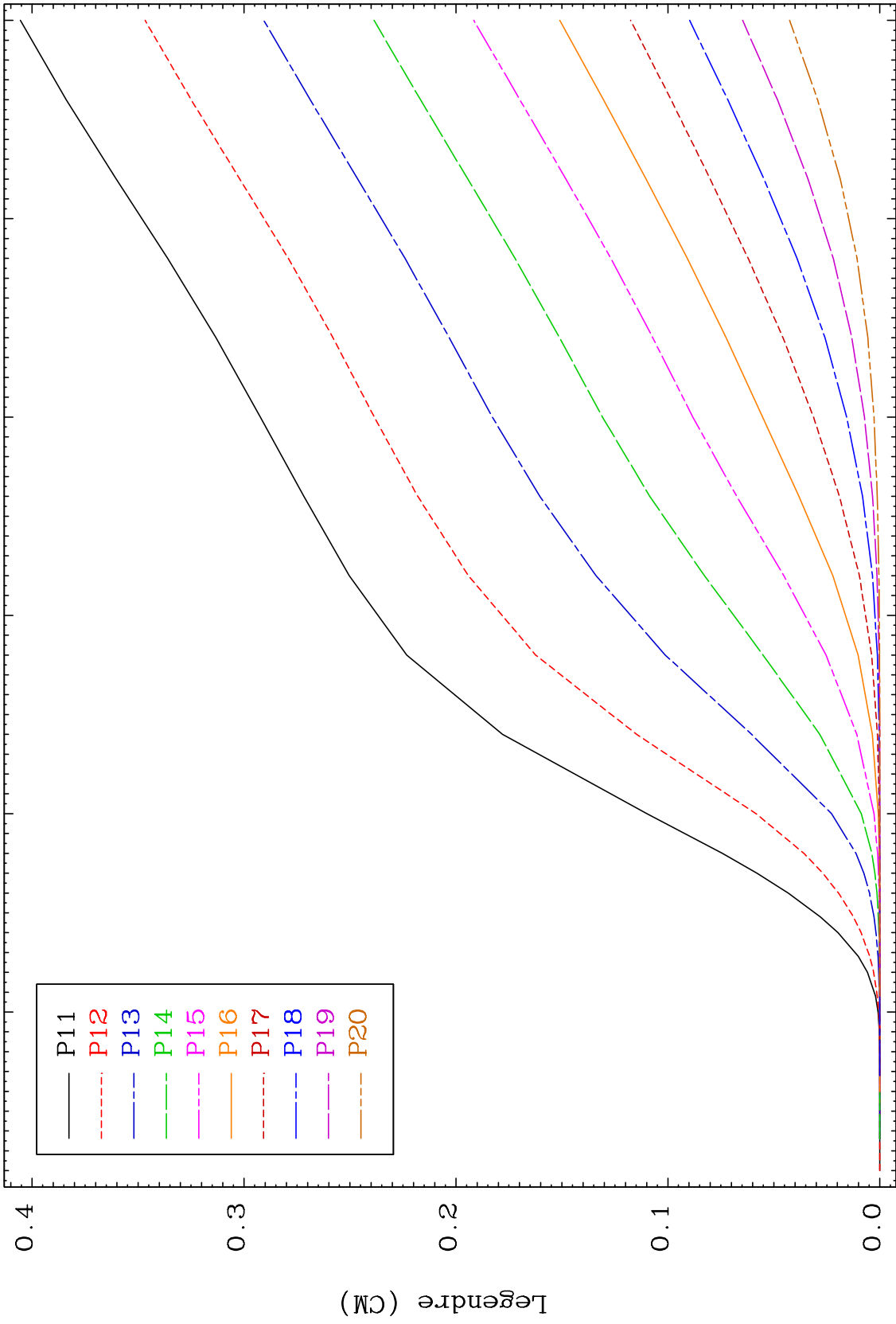
101-Md-250



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Elastic Legendre Coefficients

101-Md-250



19

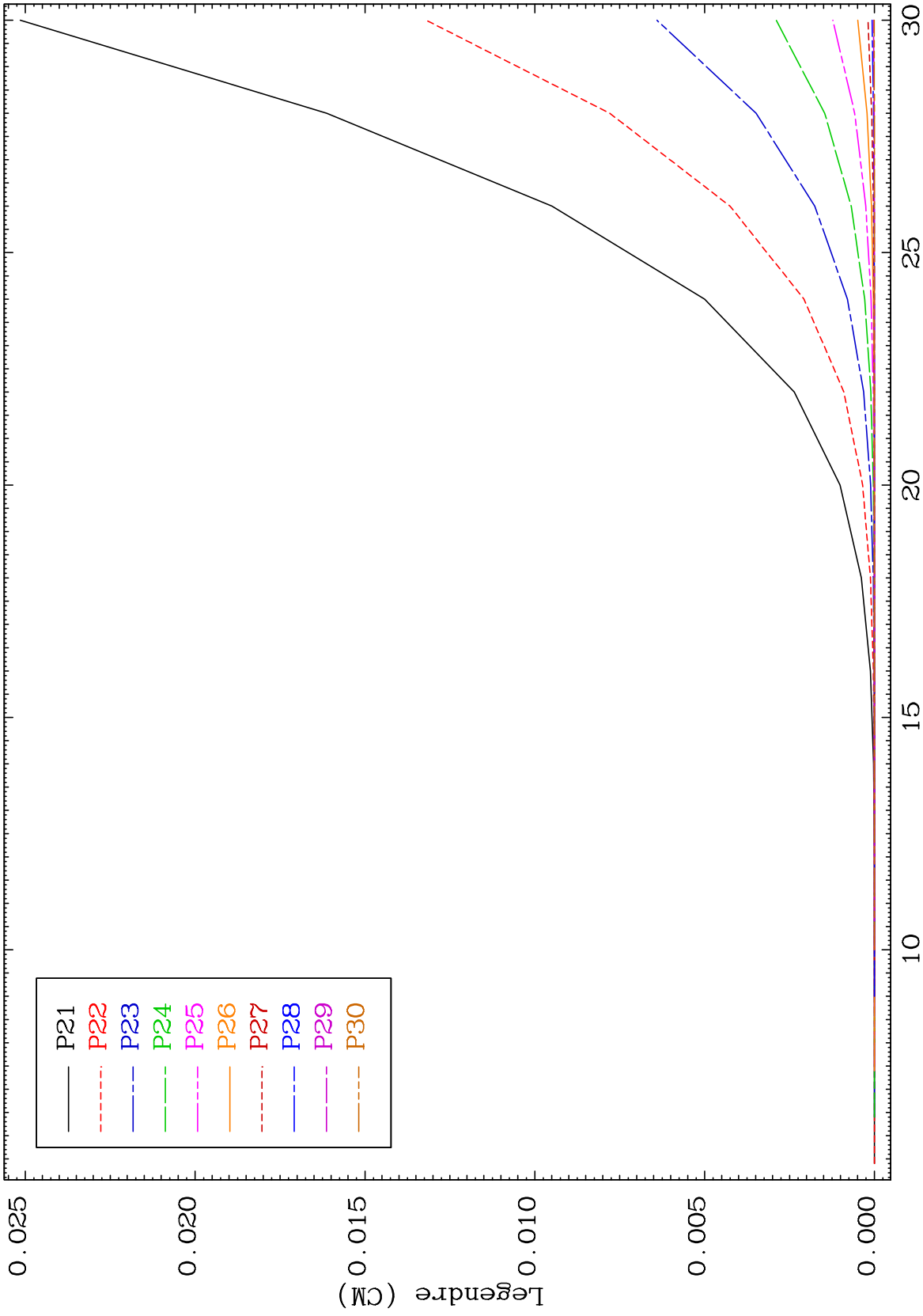
Incident Energy (MeV)

101-Md-250

MAT 9951

Elastic
Legendre Coefficients

101-Md-250



20

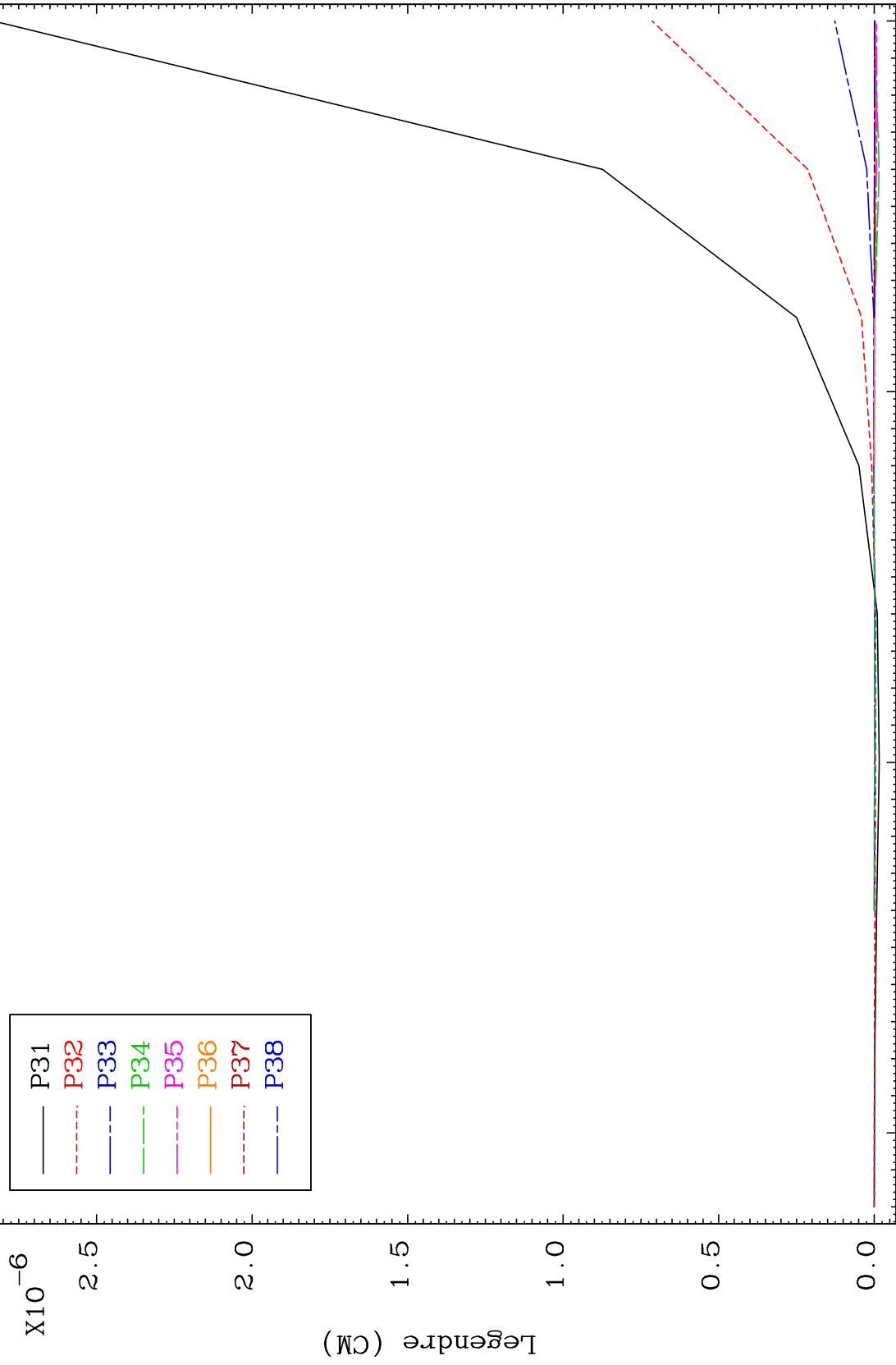
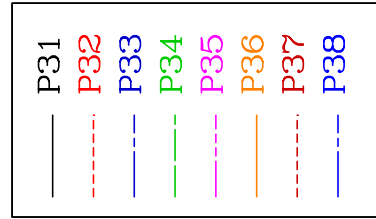
Incident Energy (MeV)

101-Md-250

MAT 9951

Elastic Legendre Coefficients

101-Md-250



21

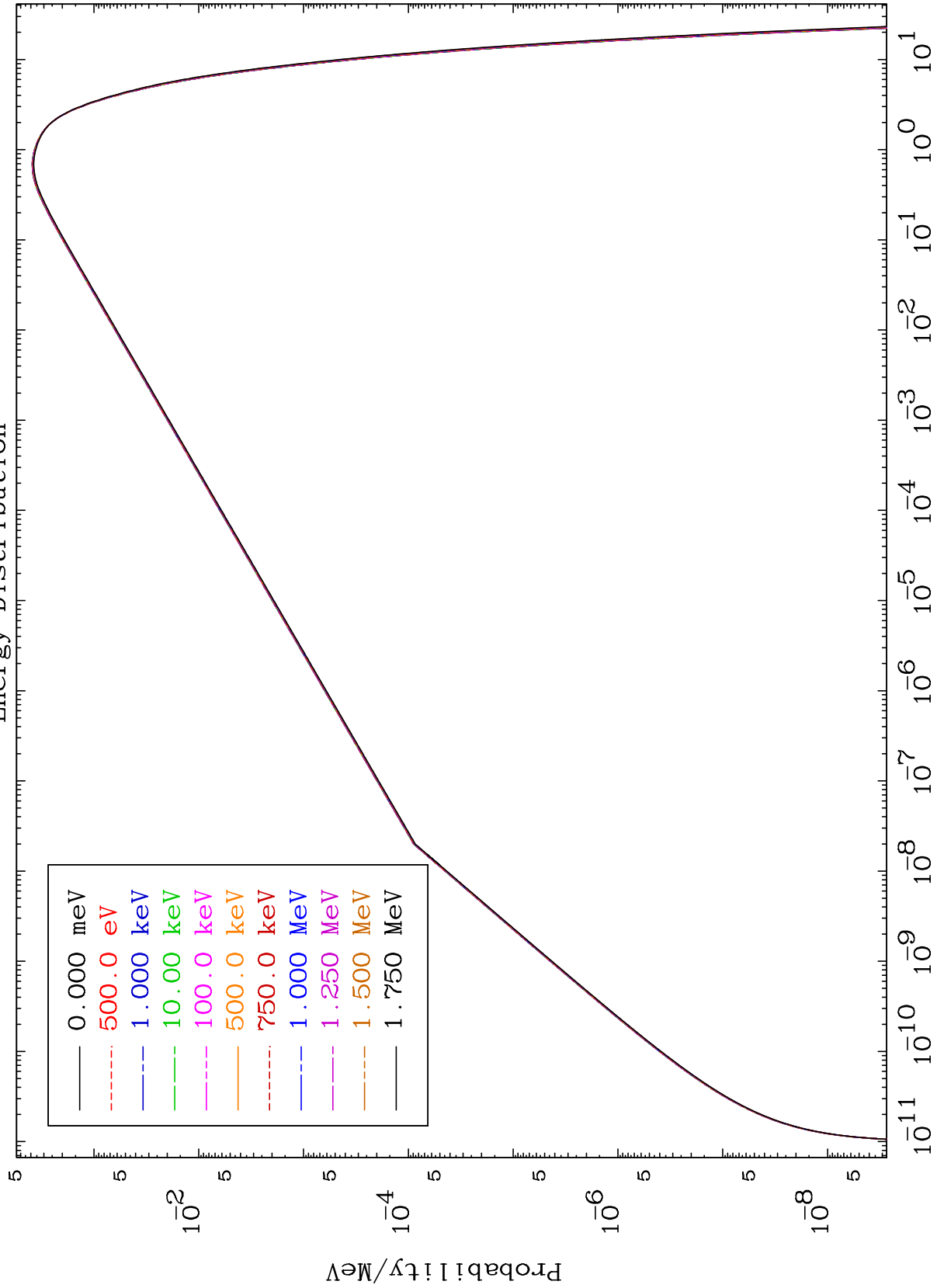
Incident Energy (MeV)

101-Md-250

MAT 9951

Fission Energy Distribution

101-Md-250

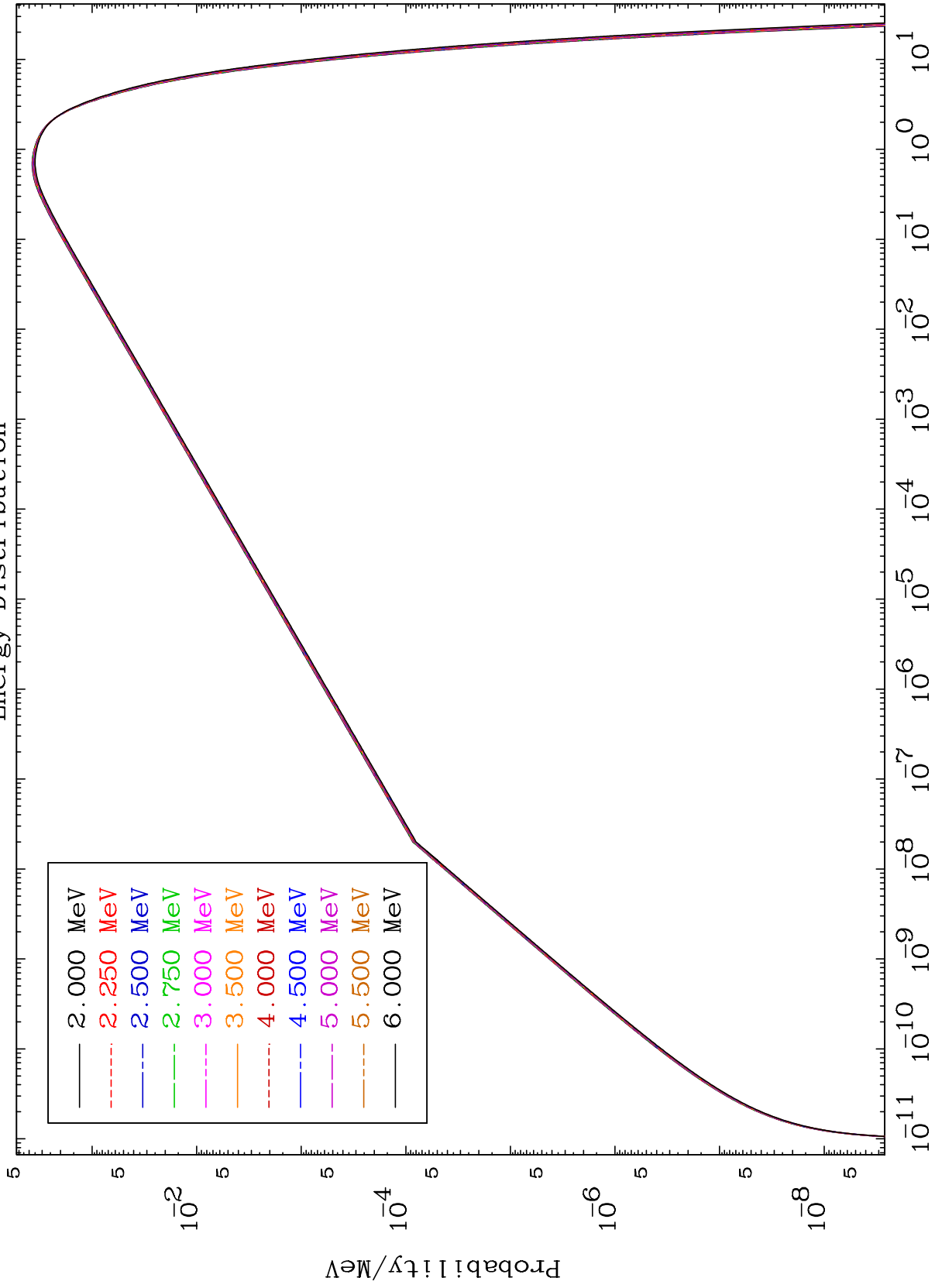


101-Md-250

MAT 9951

Fission Energy Distribution

101-Md-250

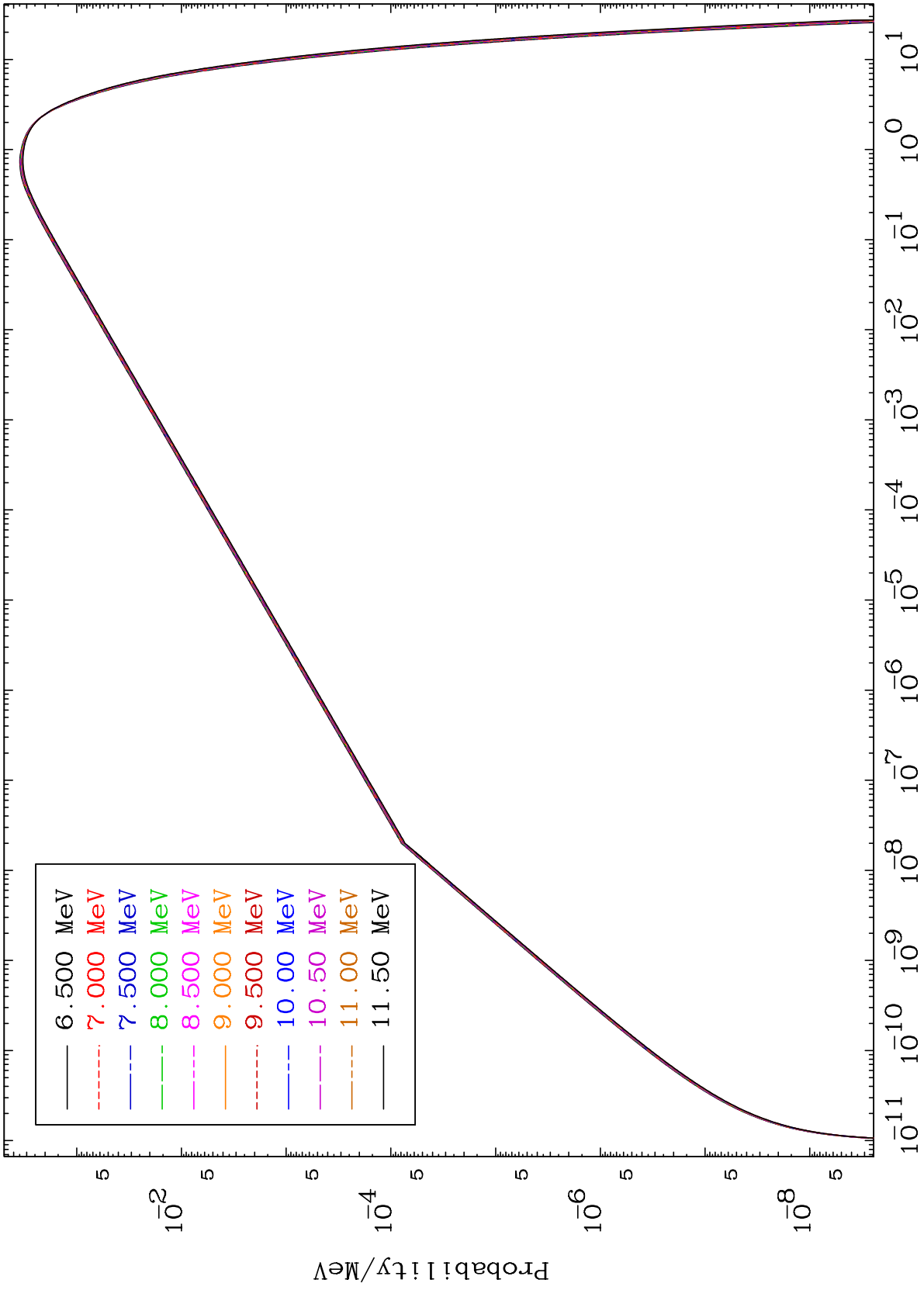


101-Md-250

MAT 9951

Fission Energy Distribution

101-Md-250

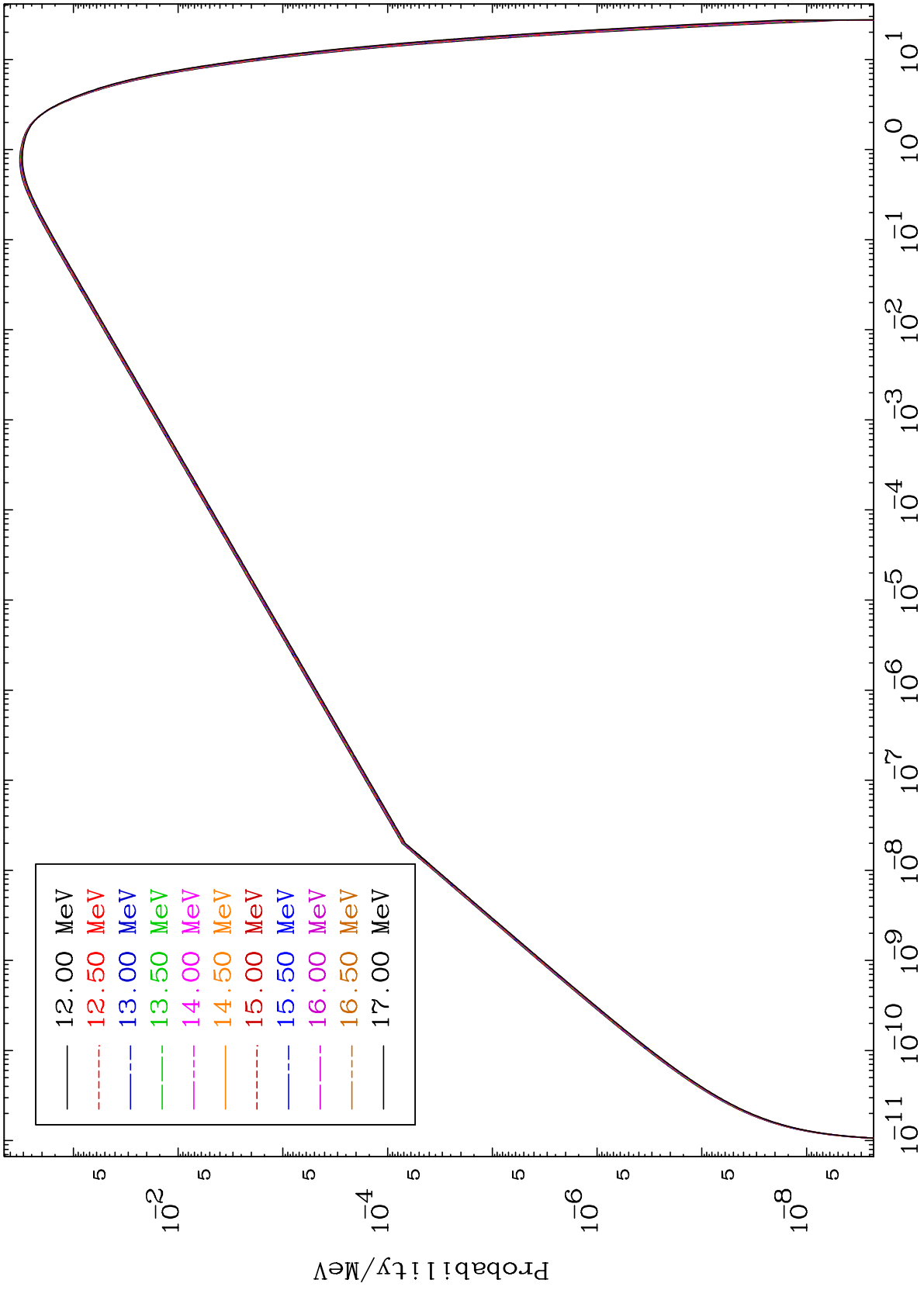


101-Md-250

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Fission Energy Distribution

101-Md-250

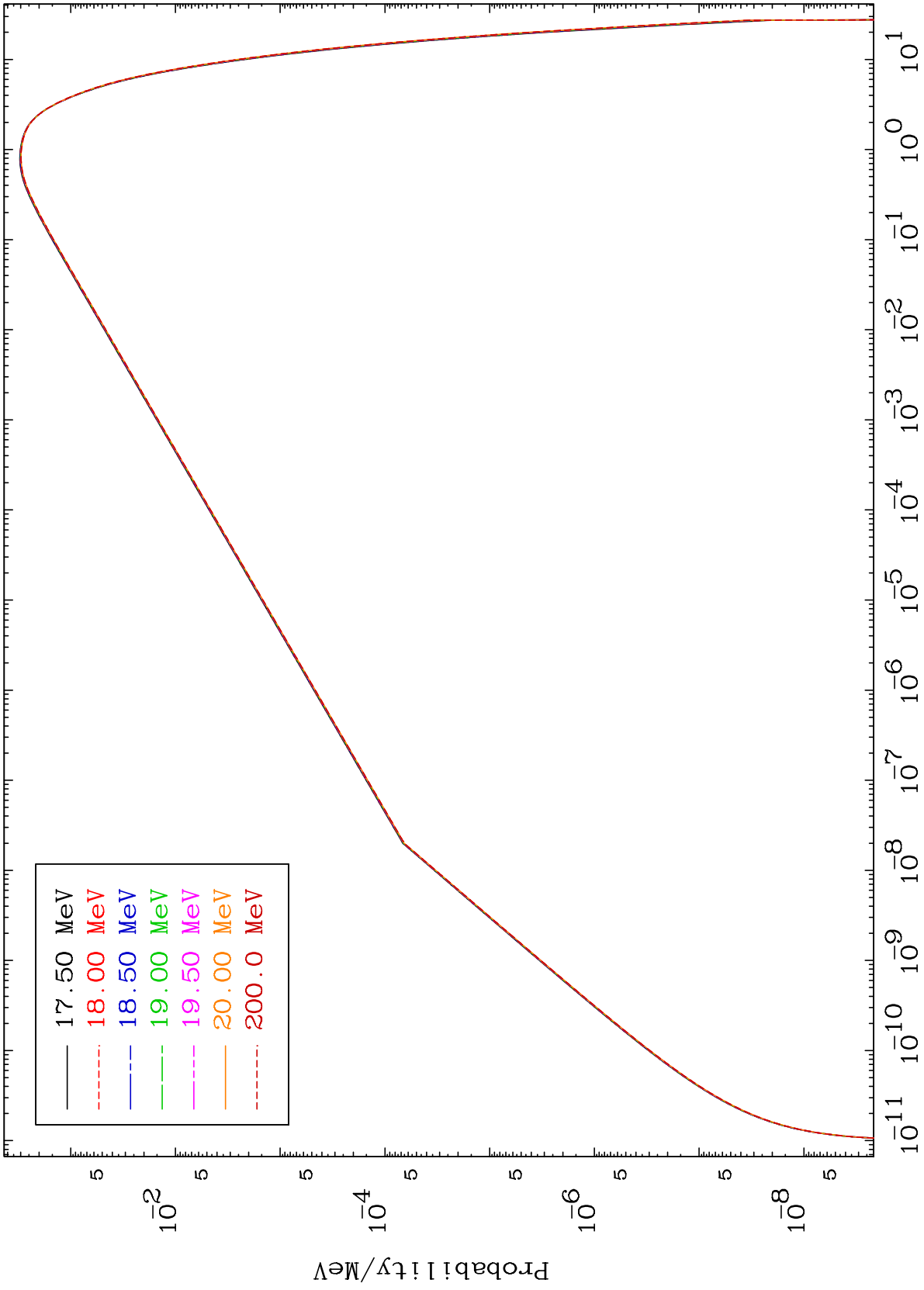


101-Md-250

MAT 9951

Fission Energy Distribution

101-Md-250



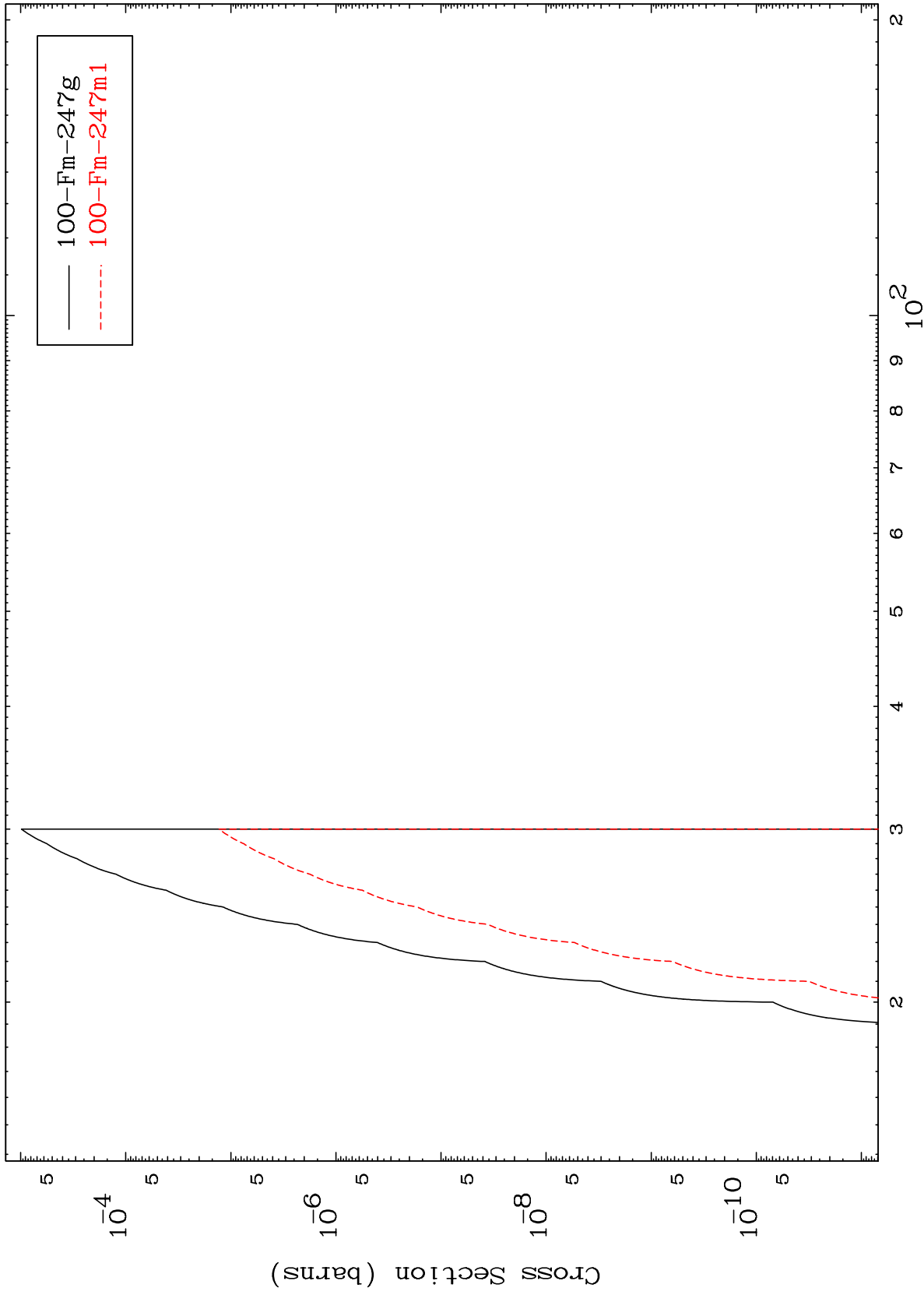
101-Md-250

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(n,2n) d

101-Md-250

Radionuclide Production Cross Section



27

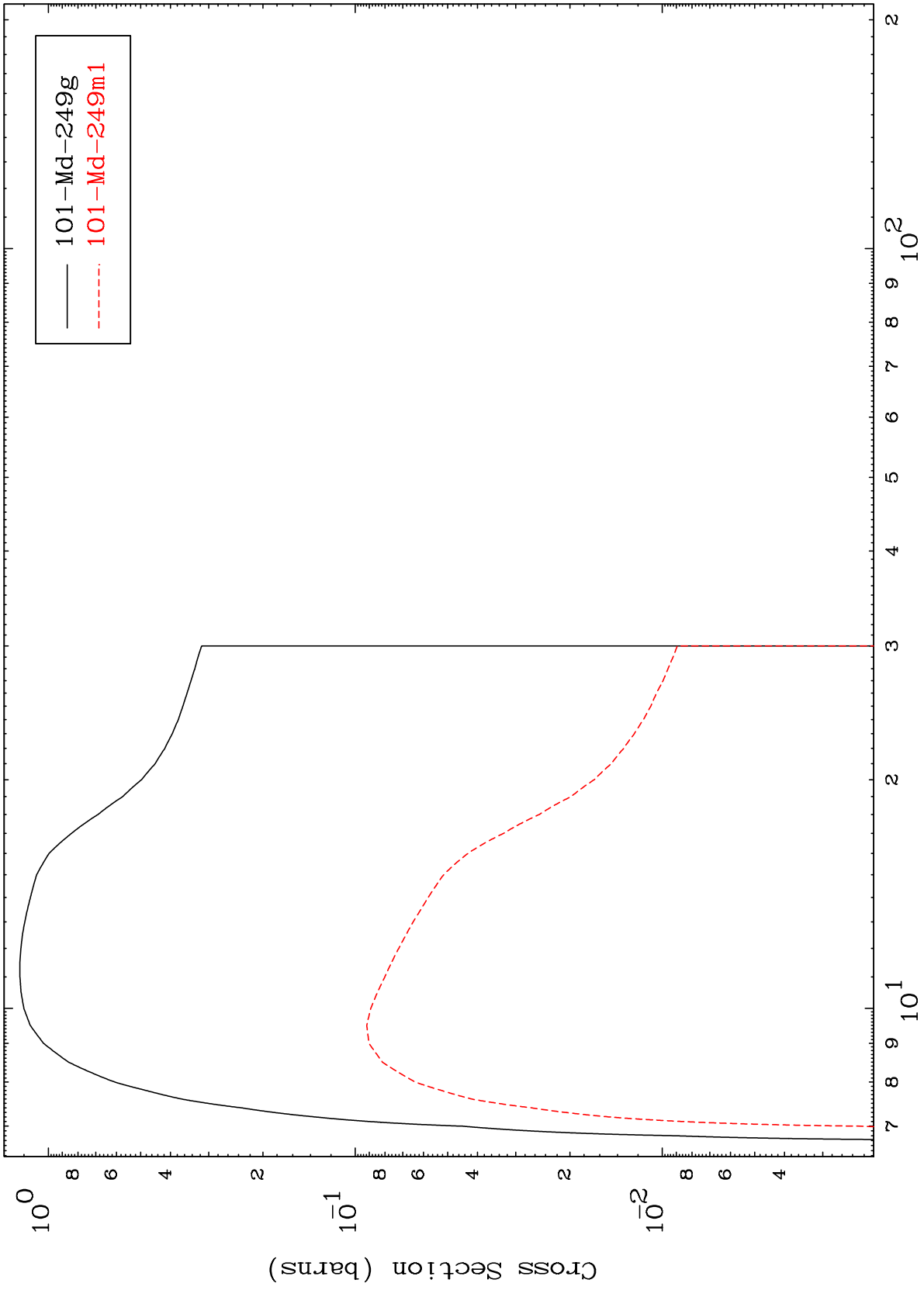
Incident Energy (MeV)

101-Md-250

MAT 9951

101-Md-250

(n,2n)
Radionuclide Production Cross Section



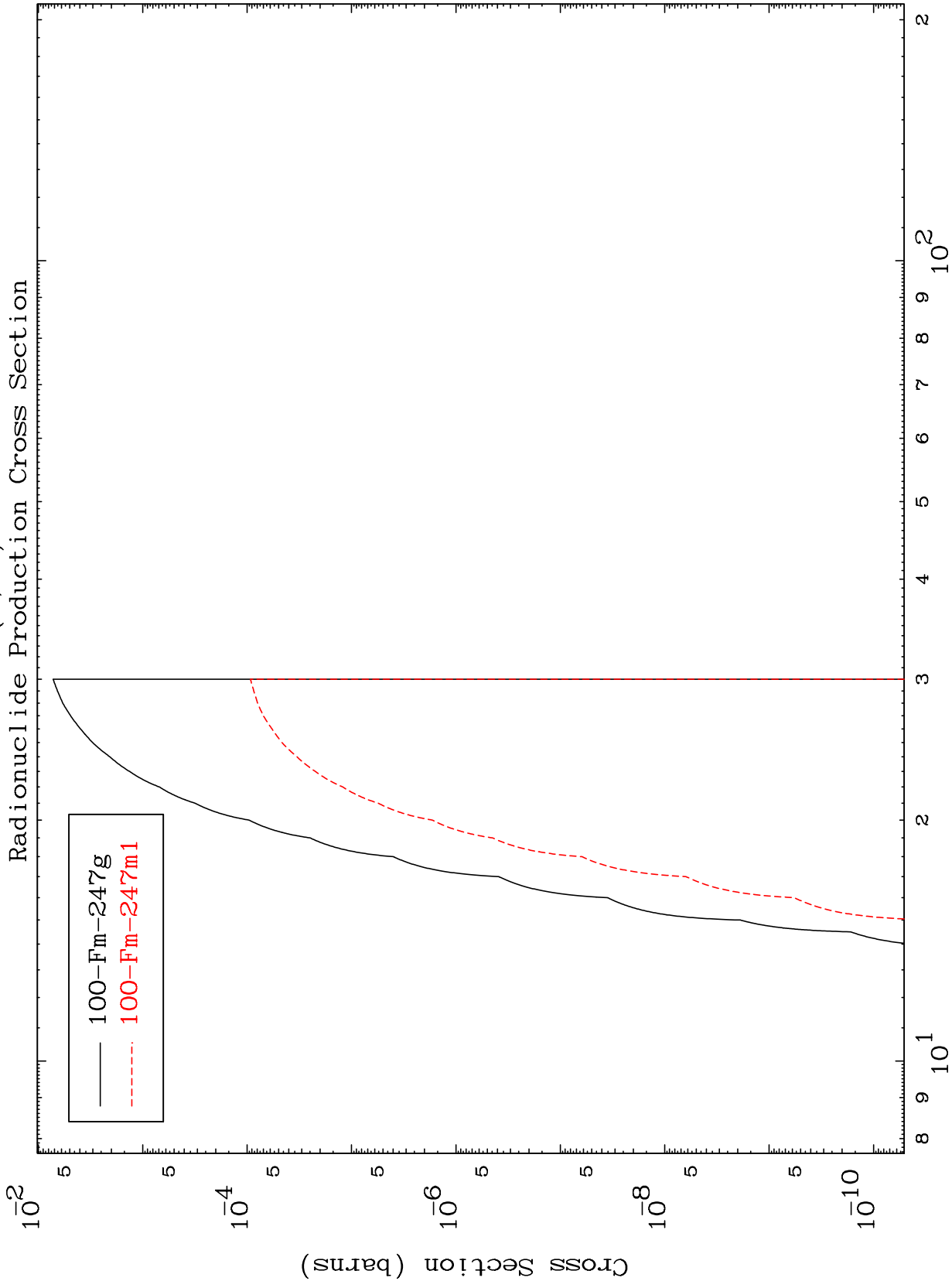
28

101-Md-250

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(n,n') t

101-Md-250



101-Md-250

Incident Energy (MeV)

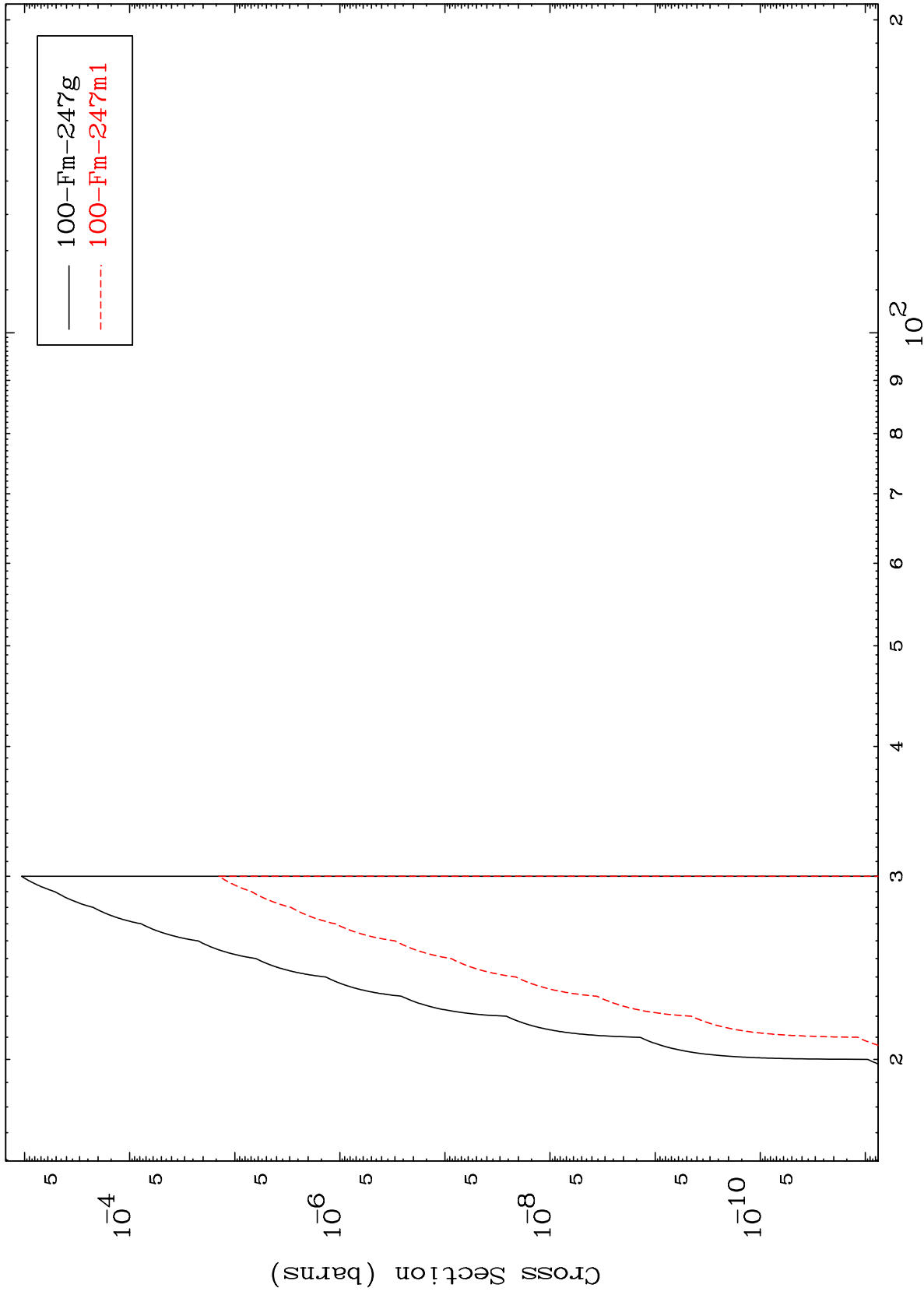
29

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(n,3n) p

101-Md-250

Radionuclide Production Cross Section



30

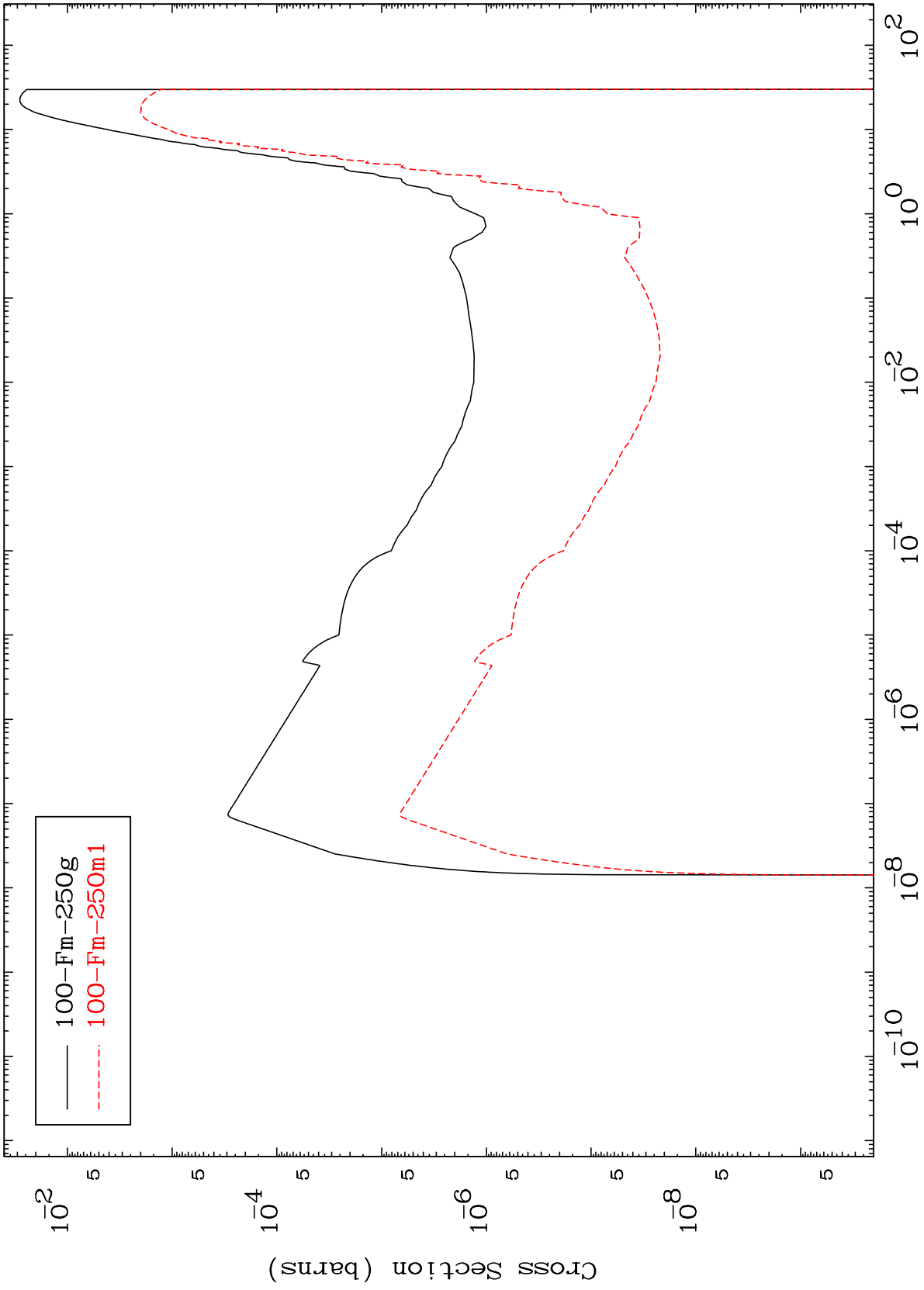
Incident Energy (MeV)

101-Md-250

MAT 9951

(n,p)
Radionuclide Production Cross Section

101-Md-250



— 100-Fm-250g
- - - 100-Fm-250ml