

Program EVALPLOT
(Version 2018-1)

by

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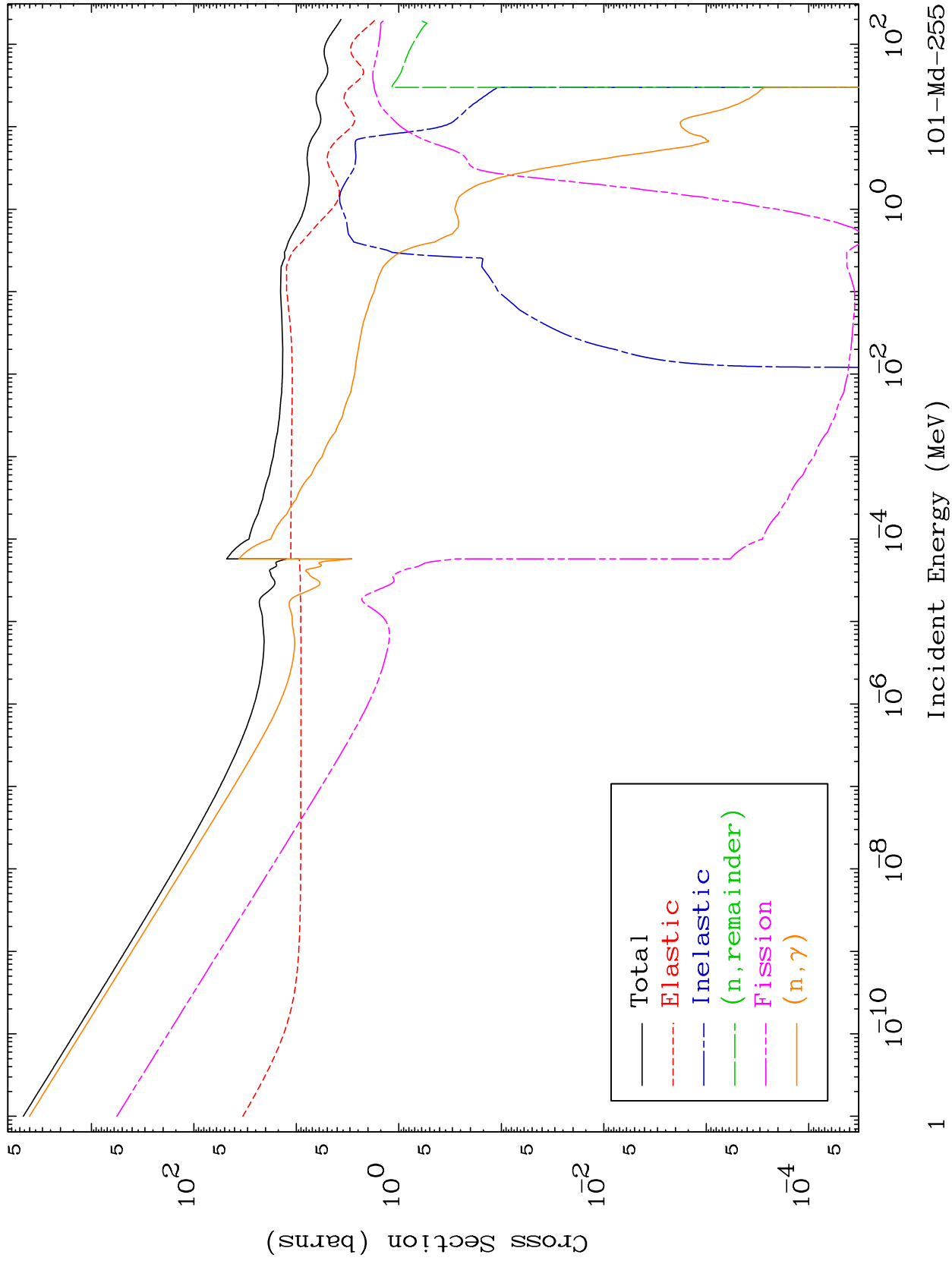
Press Mouse Button to Start

MAT 9956

Major

293 Kelvin Cross Sections

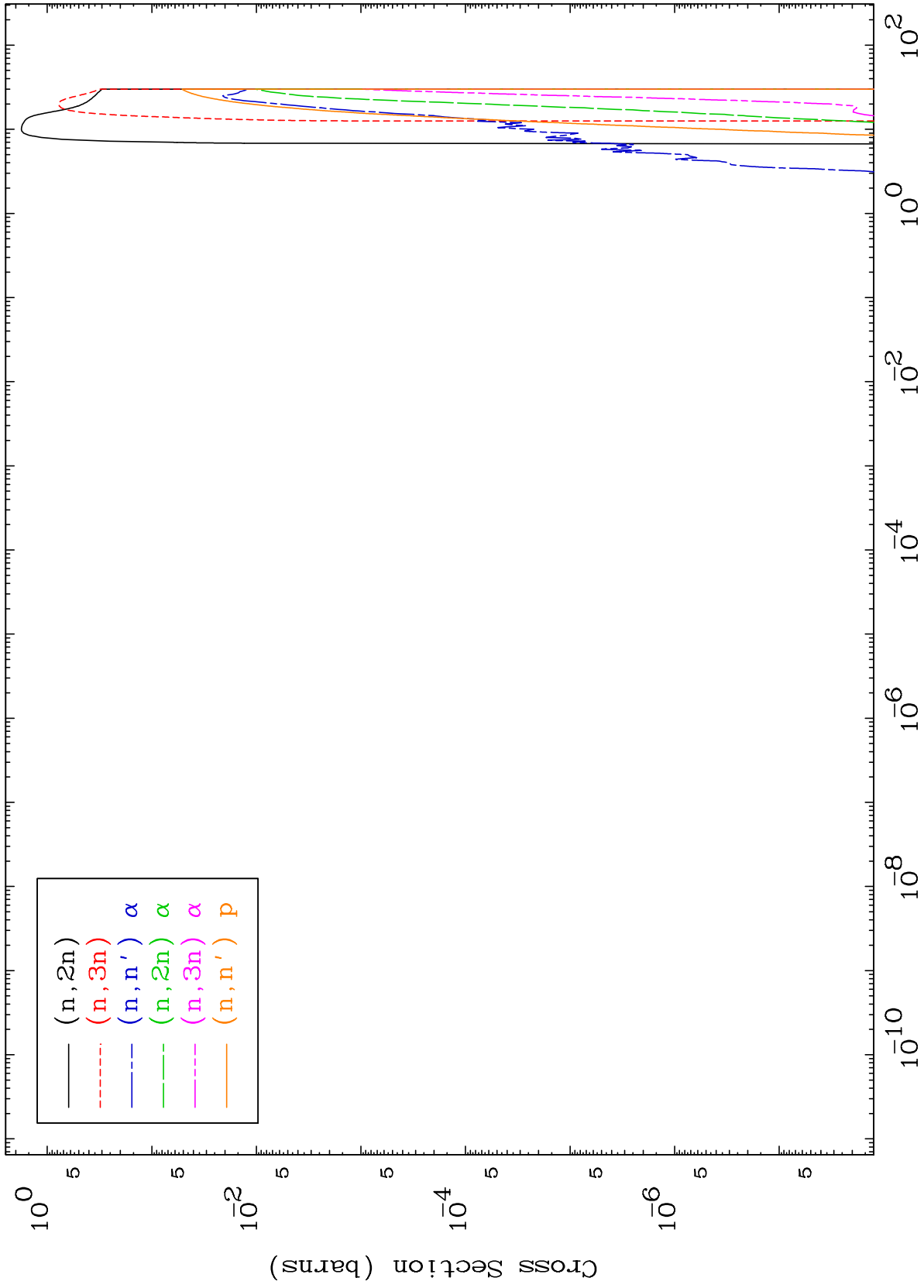
101-Md-255



MAT 9956

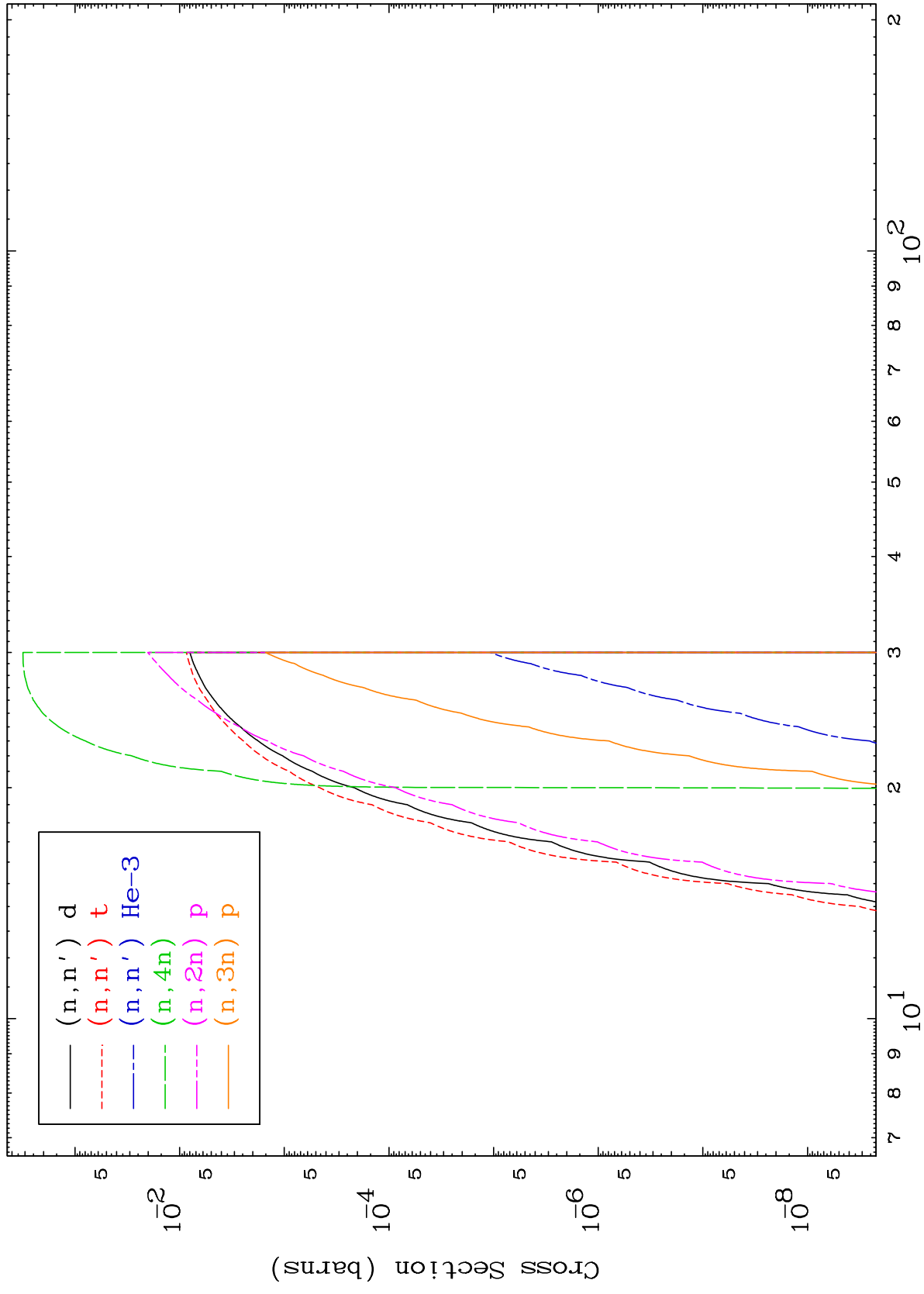
Neutron Production
293 Kelvin Cross Sections

101-Md-255



2

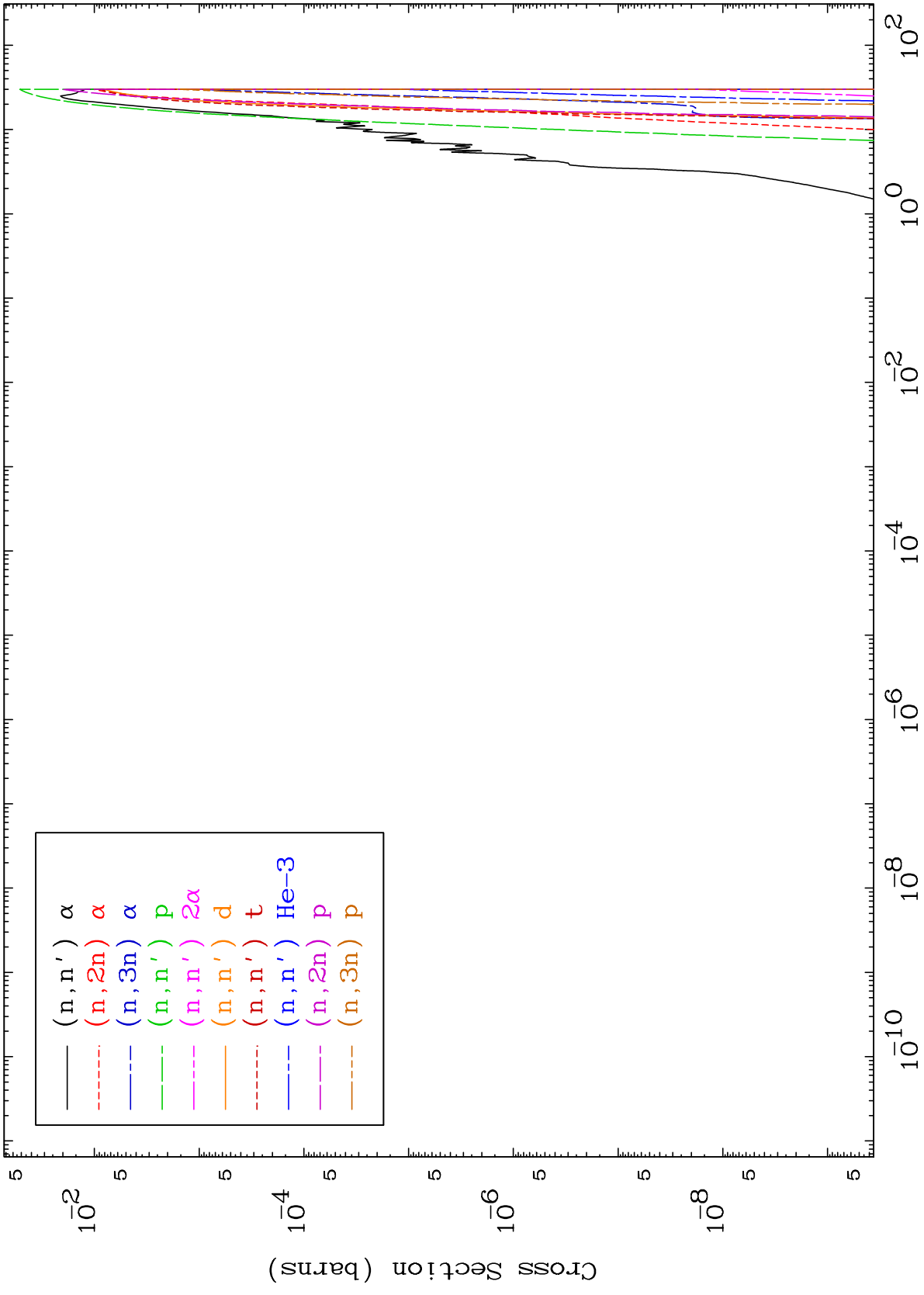
101-Md-255



MAT 9956

Charged Particle
293 Kelvin Cross Sections

101-Md-255

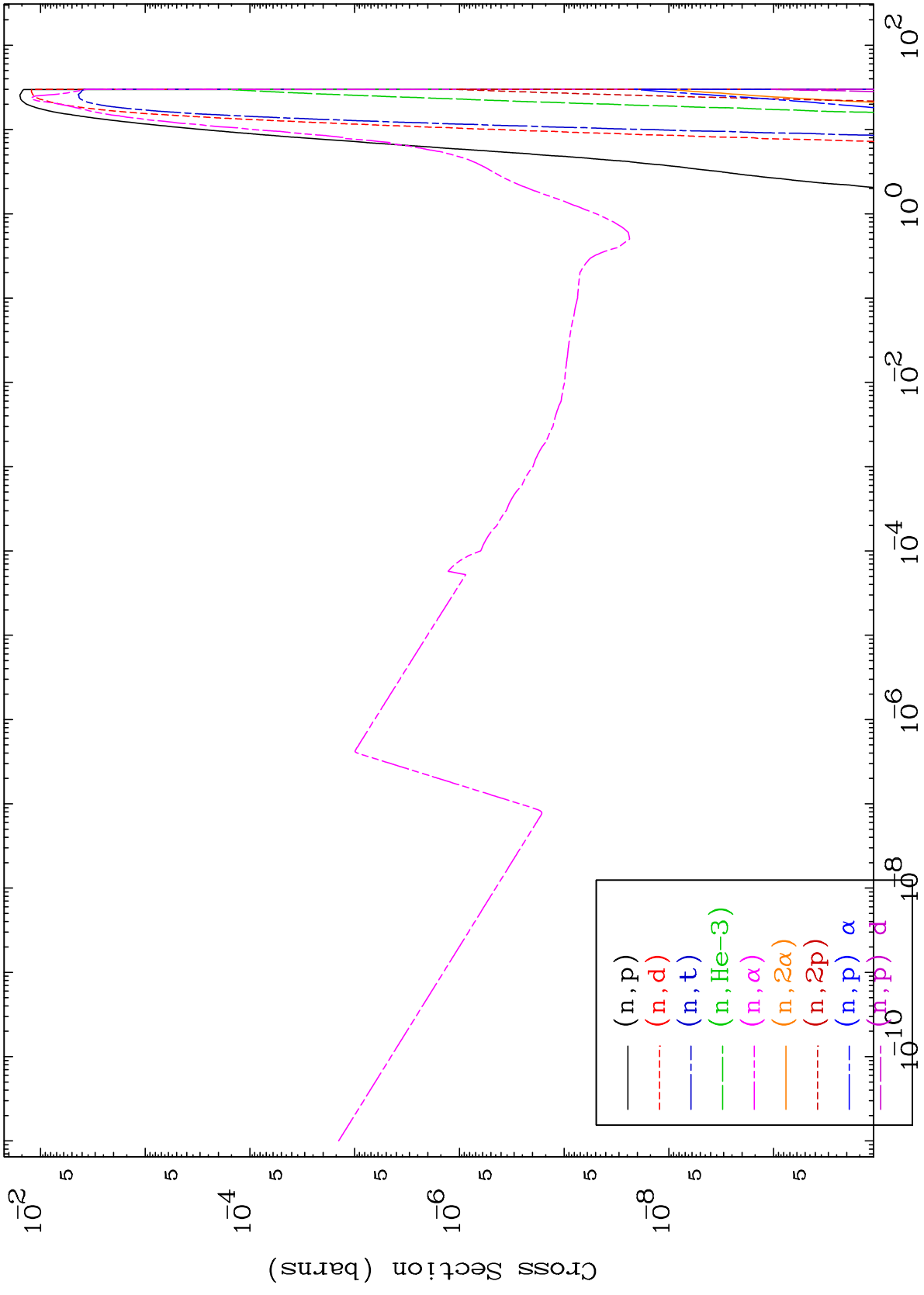


101-Md-255

MAT 9956

Charged Particle
293 Kelvin Cross Sections

101-Md-255



101-Md-255

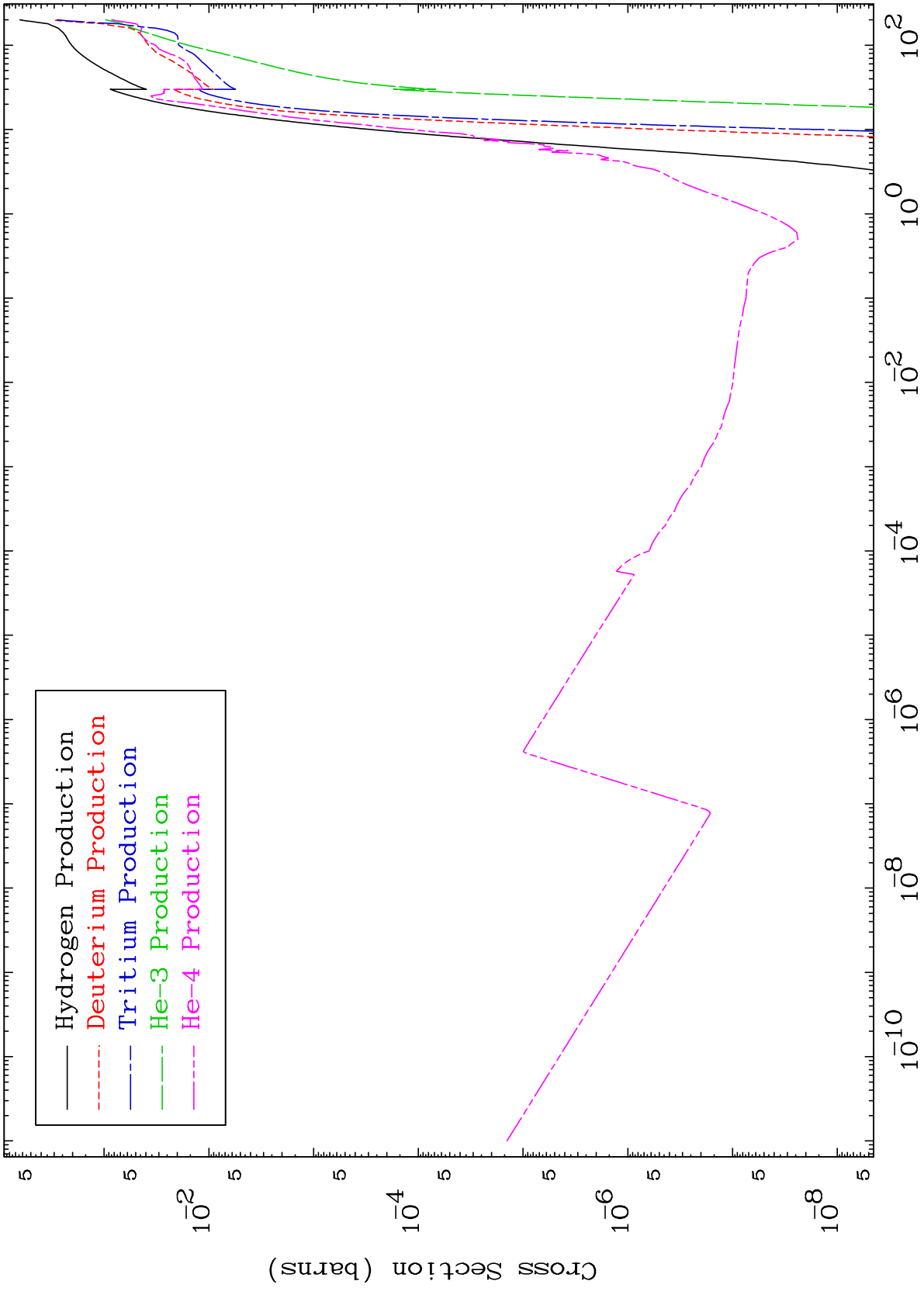
Incident Energy (MeV)

5

MAT 9956

Particle Production
293 Kelvin Cross Sections

101-Md-255



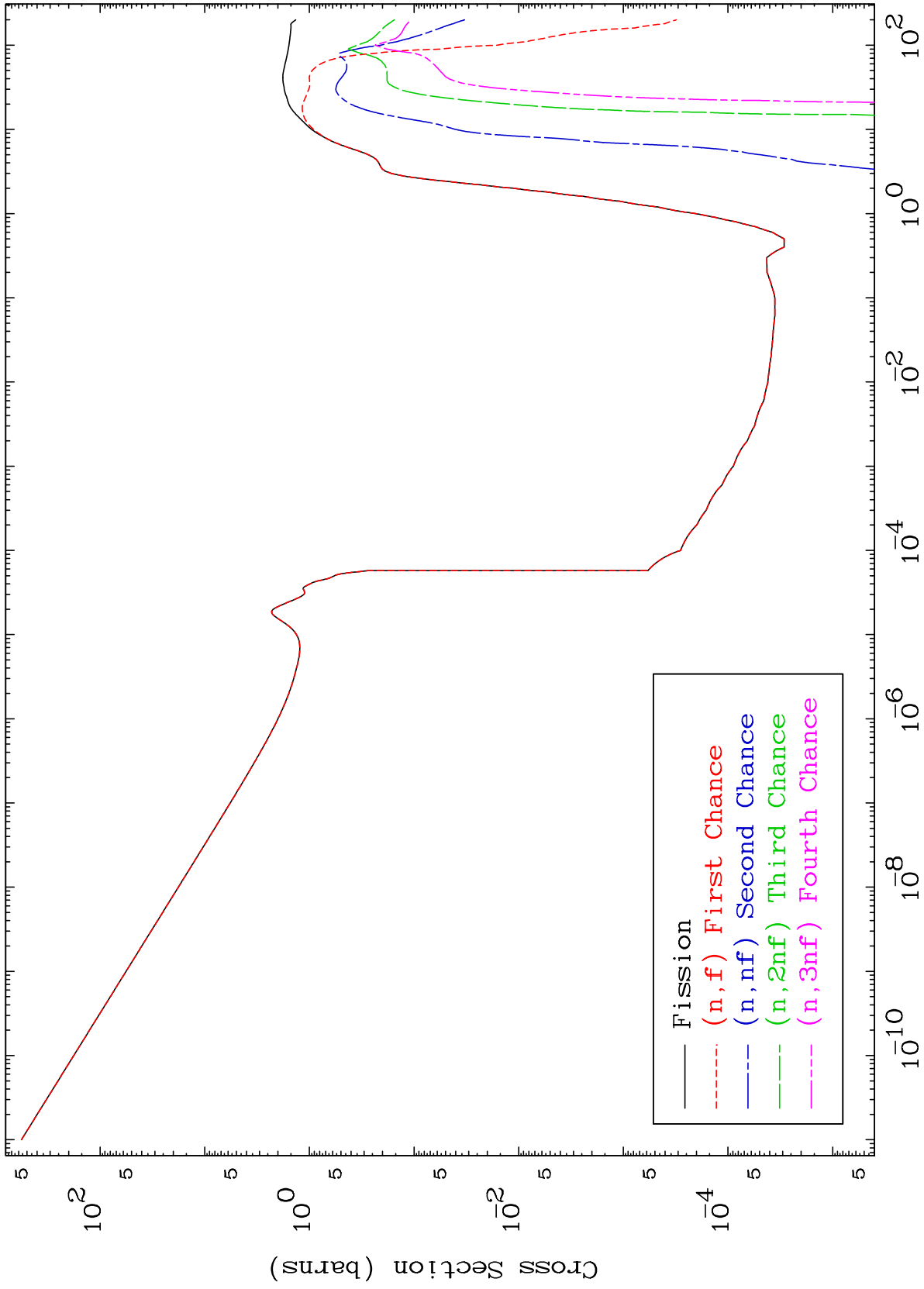
6

101-Md-255

MAT 9956

Fission
293 Kelvin Cross Sections

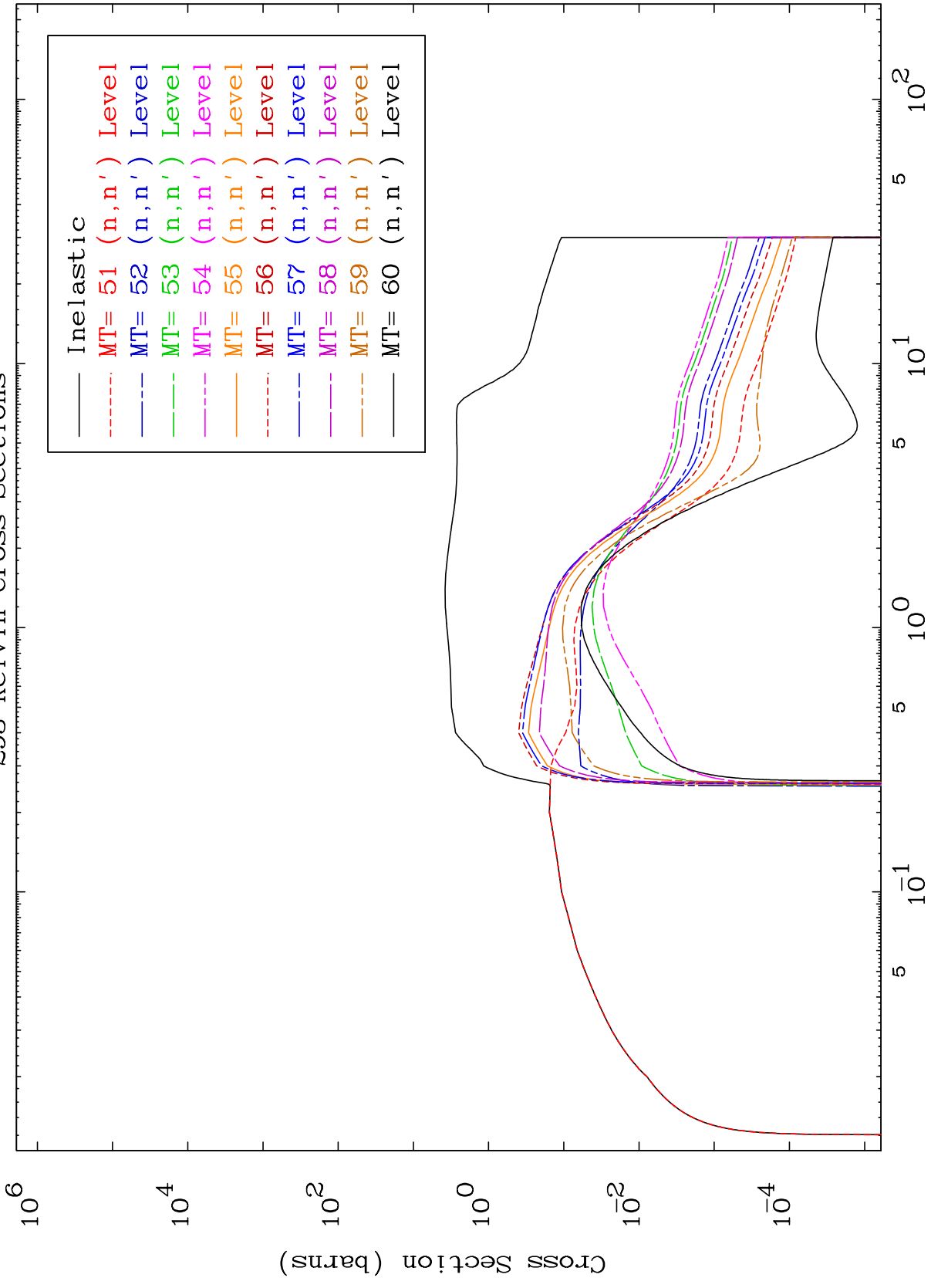
101-Md-255



MAT 9956

(n,n') Level
293 Kelvin Cross Sections

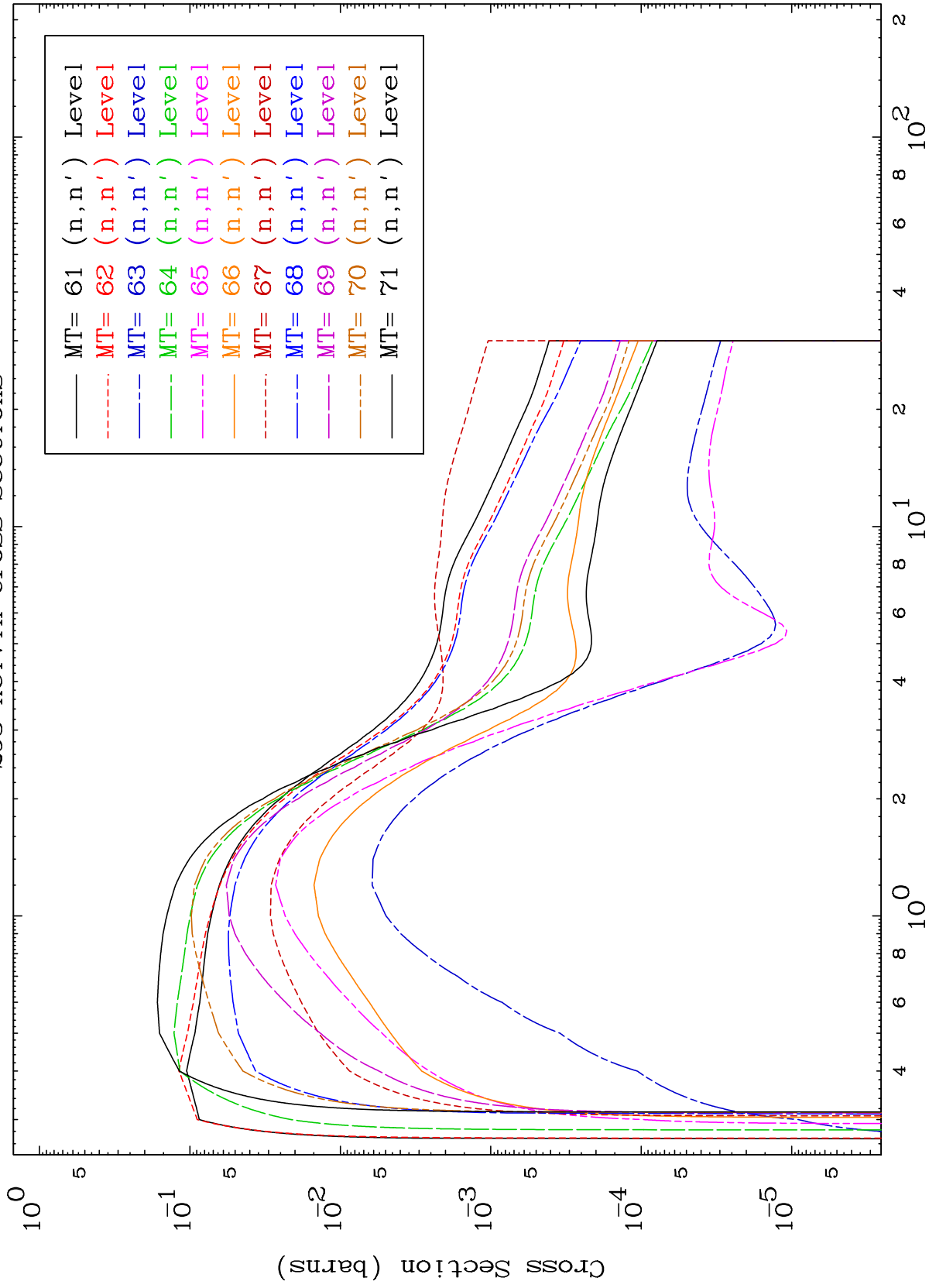
101-Md-255



MAT 9956

(n,n') Level
293 Kelvin Cross Sections

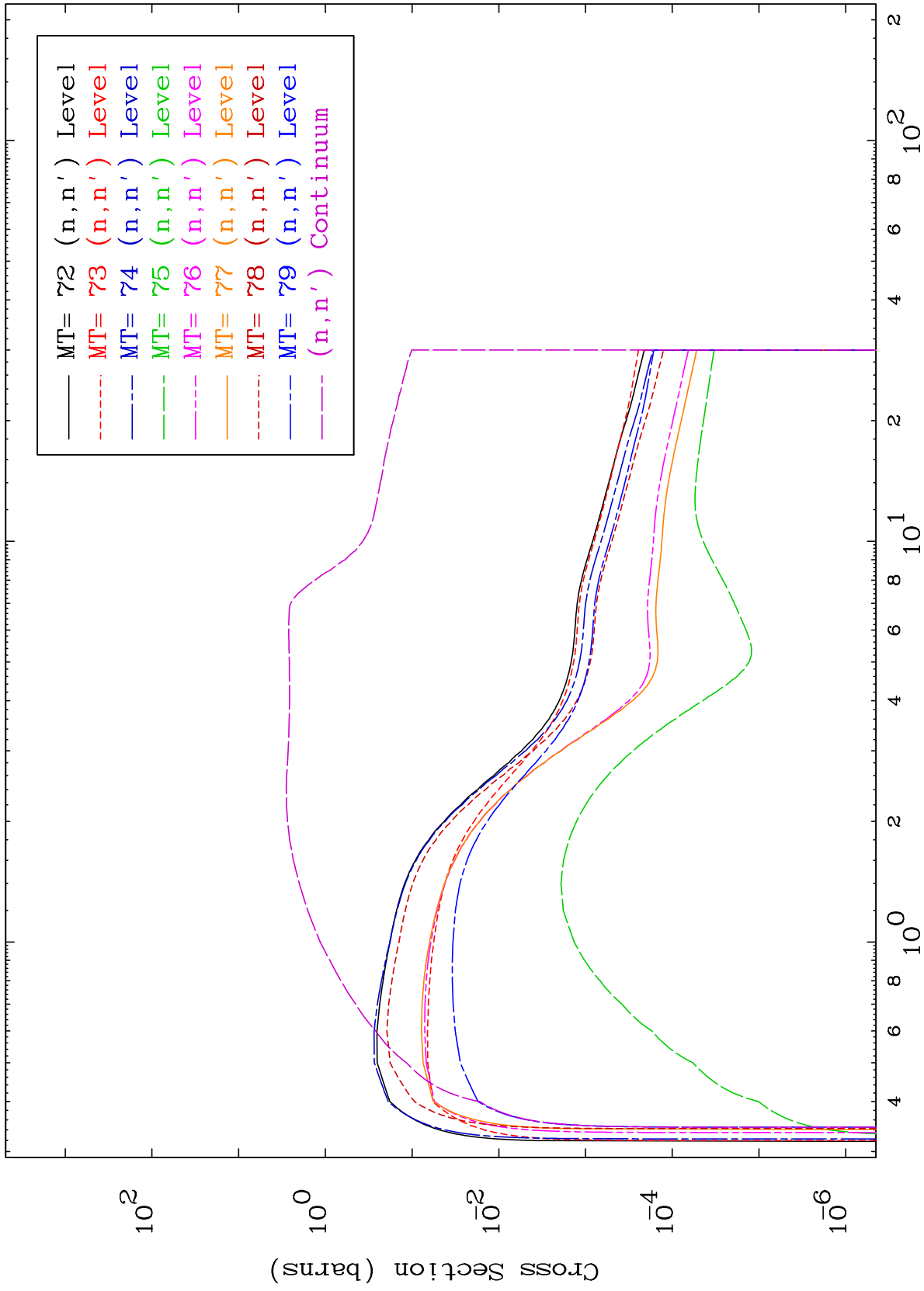
101-Md-255



MAT 9956

(n,n') Level
293 Kelvin Cross Sections

101-Md-255



10

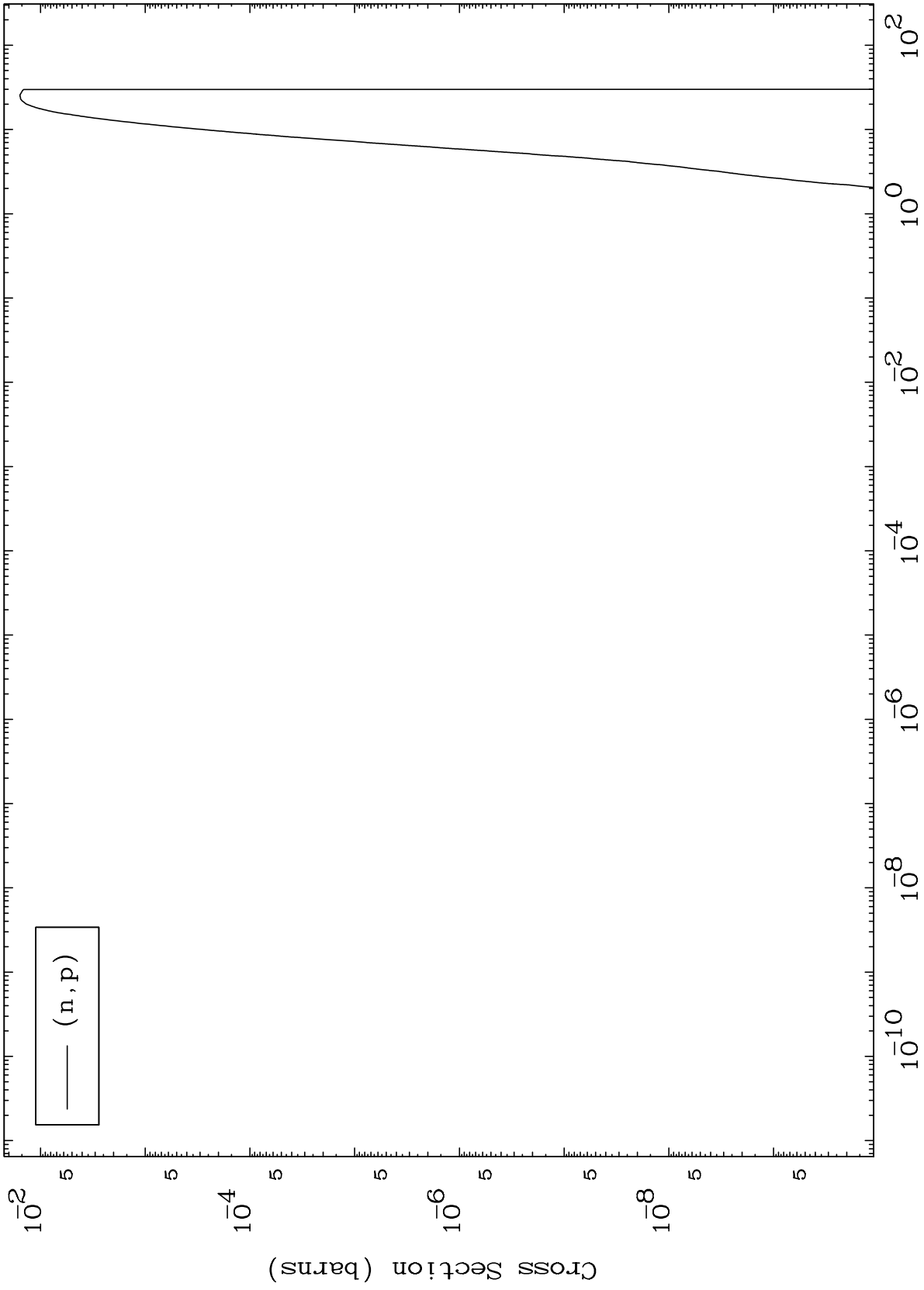
Incident Energy (MeV)

101-Md-255

MAT 9956

(n,p) Levels
293 Kelvin Cross Sections

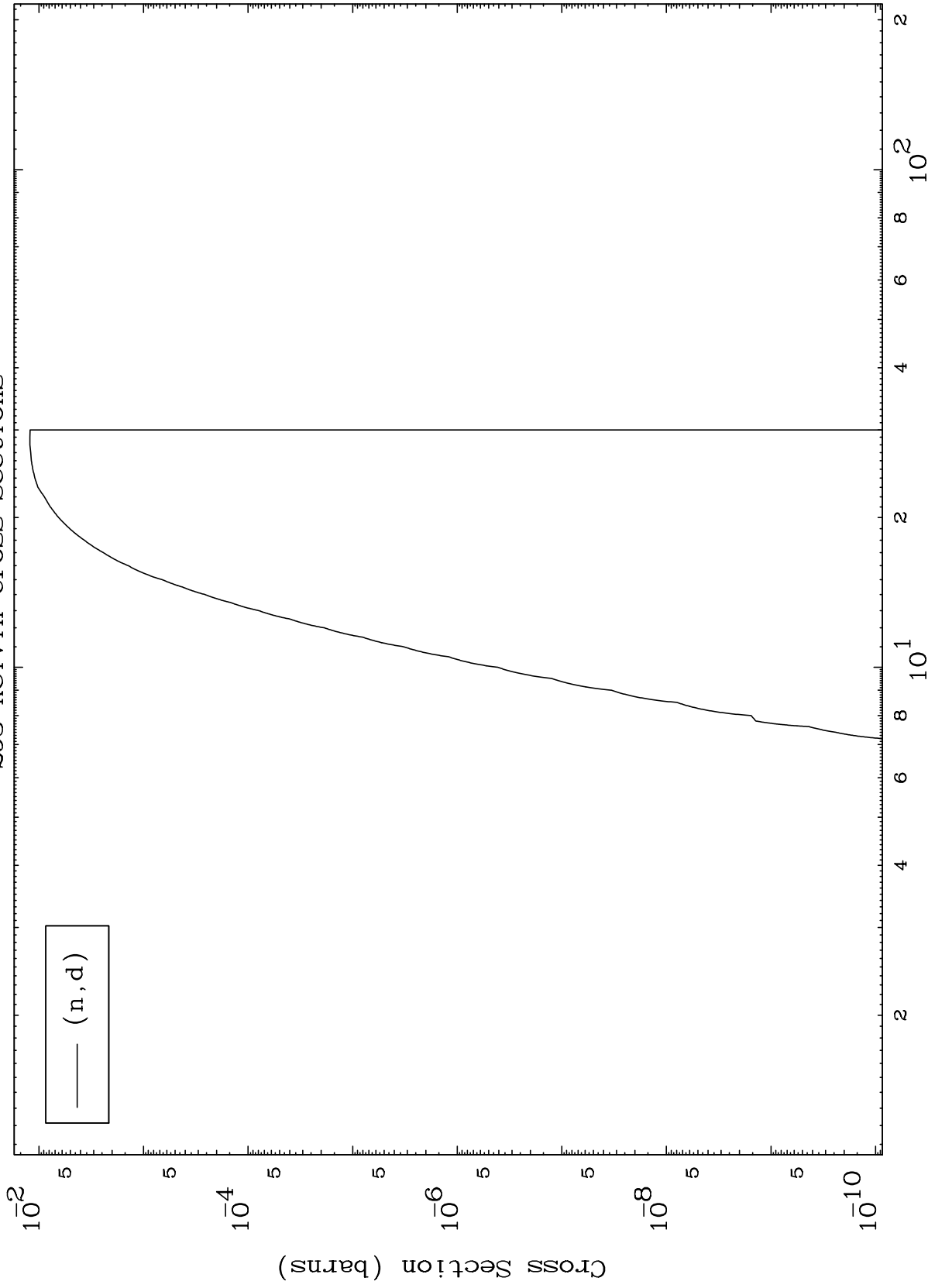
101-Md-255



MAT 9956

(n,d) Levels
293 Kelvin Cross Sections

101-Md-255



12

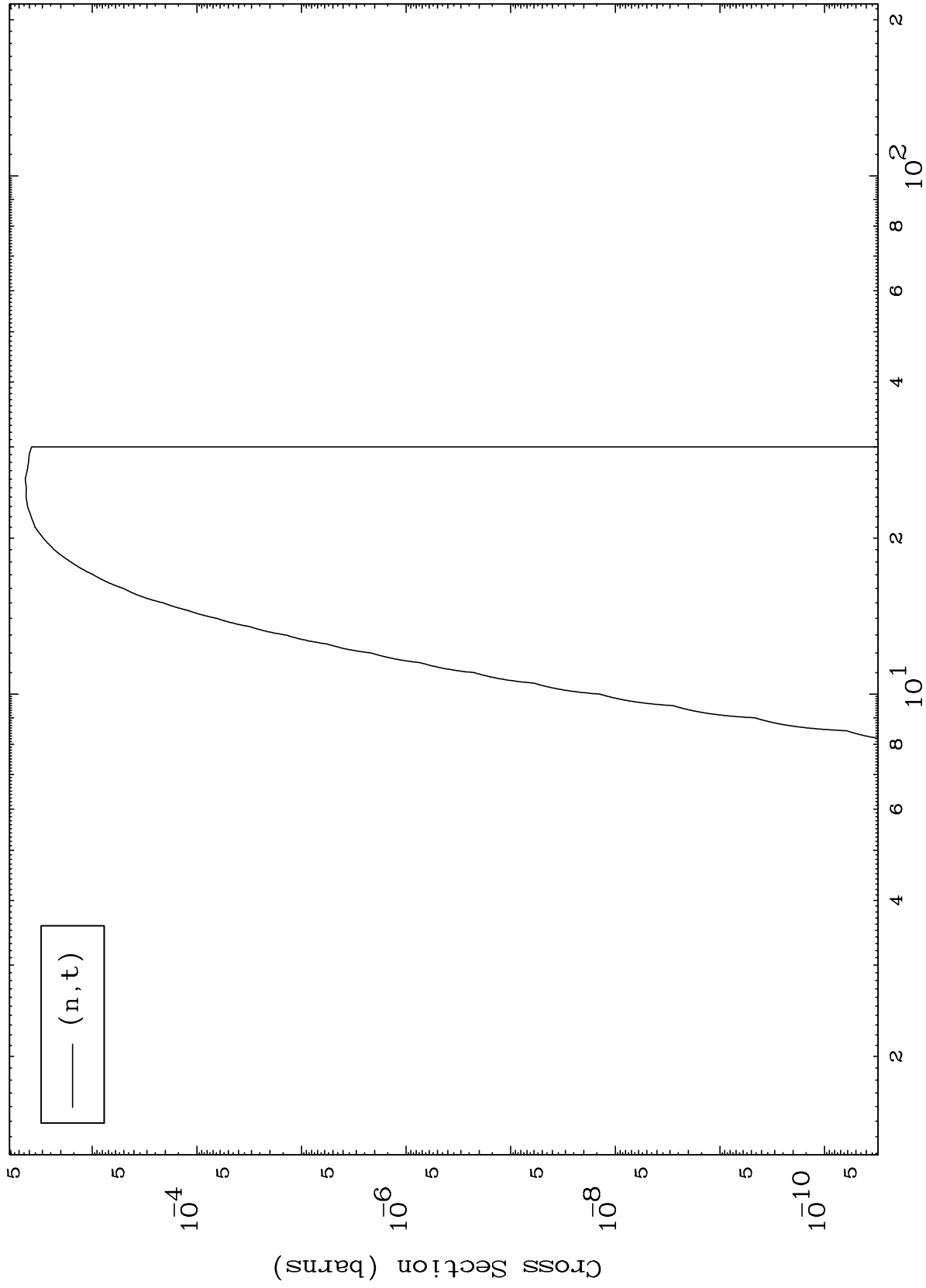
Incident Energy (MeV)

101-Md-255

MAT 9956

(n,t) Levels
293 Kelvin Cross Sections

101-Md-255



13

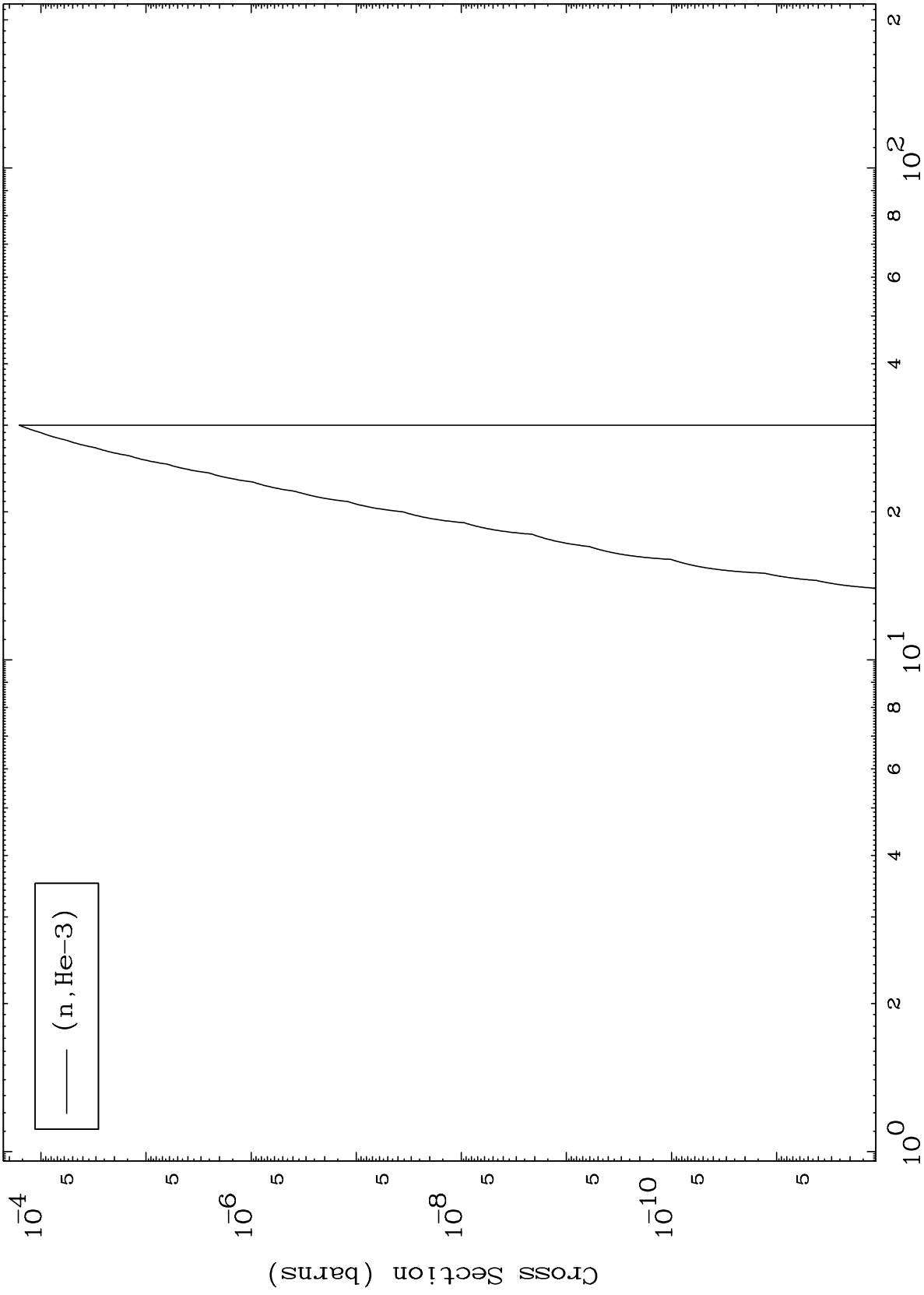
Incident Energy (MeV)

101-Md-255

MAT 9956

(n,He3) Levels
293 Kelvin Cross Sections

101-Md-255



14

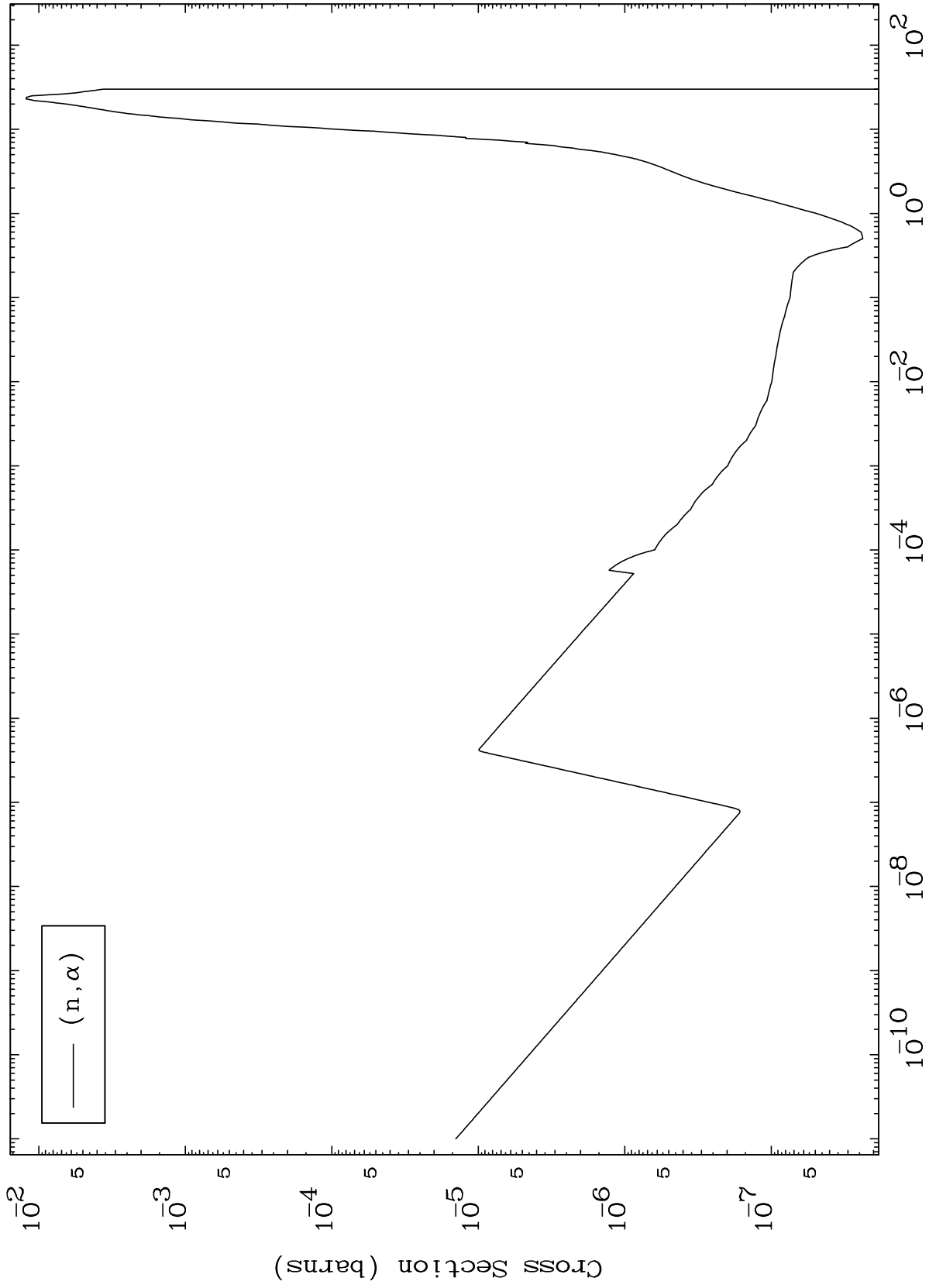
Incident Energy (MeV)

101-Md-255

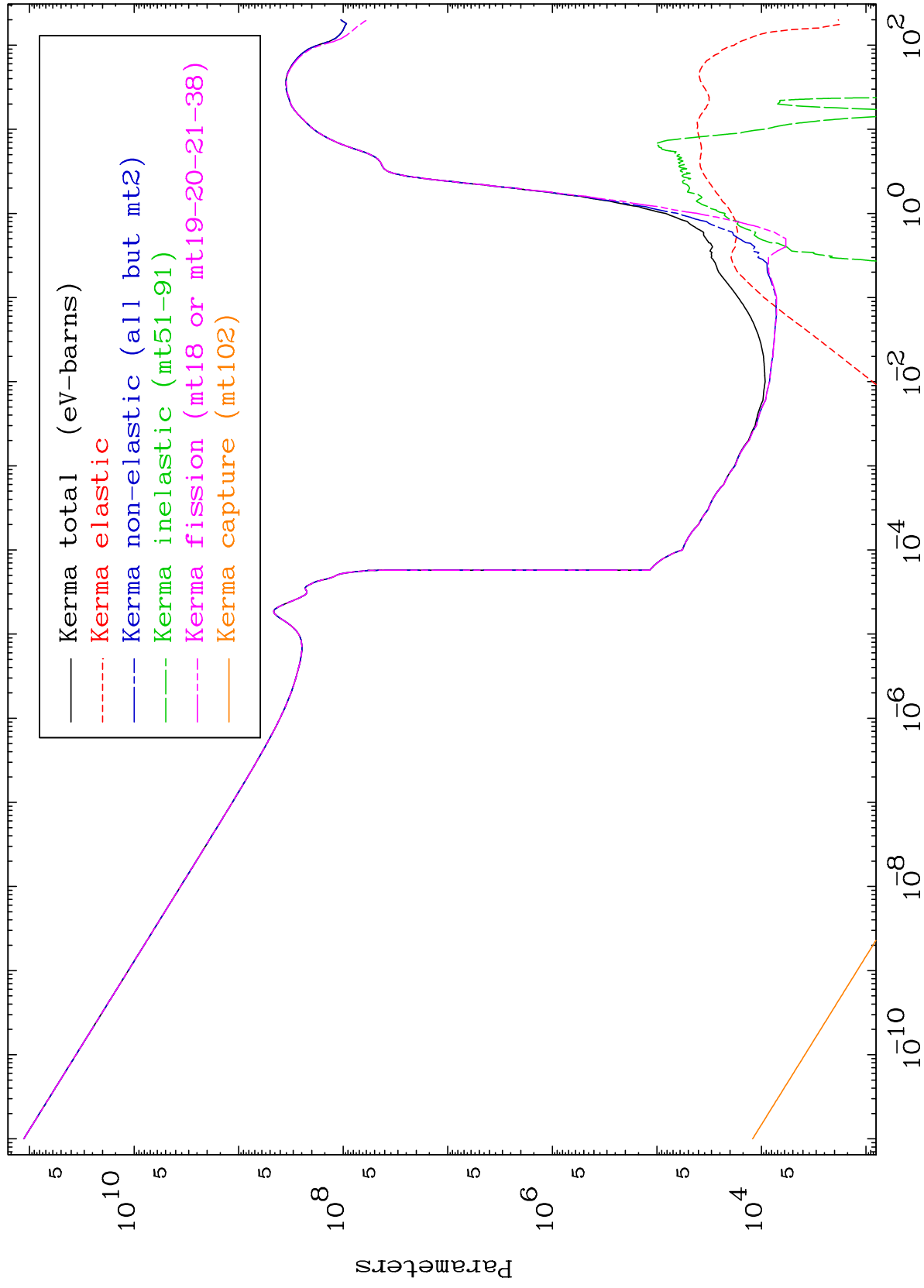
MAT 9956

(n,α) Levels
293 Kelvin Cross Sections

101-Md-255



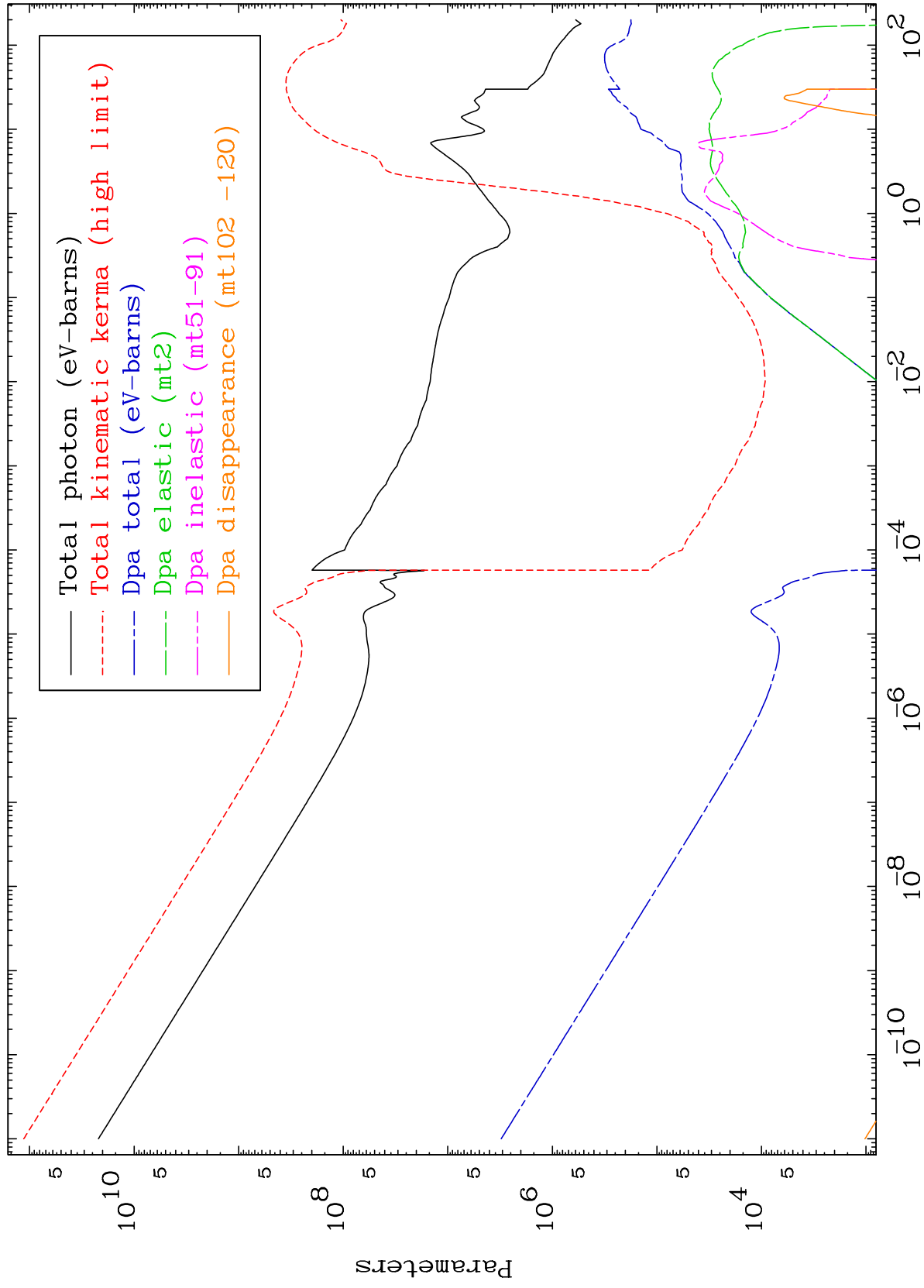
(n,α)



MAT 9956

Energy Release
Parameters

101-Md-255

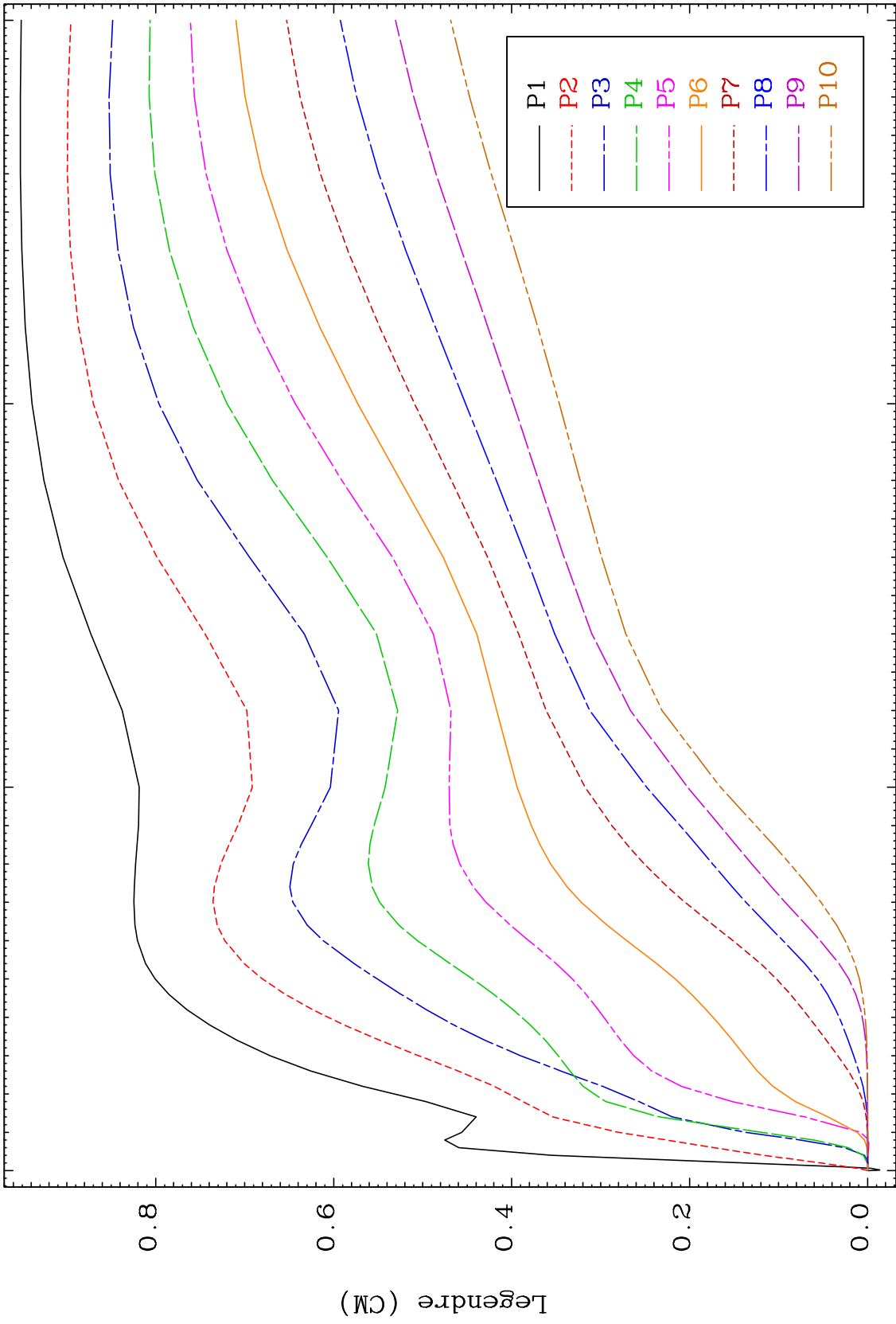


101-Md-255

MAT 9956

Elastic Legendre Coefficients

101-Md-255



18

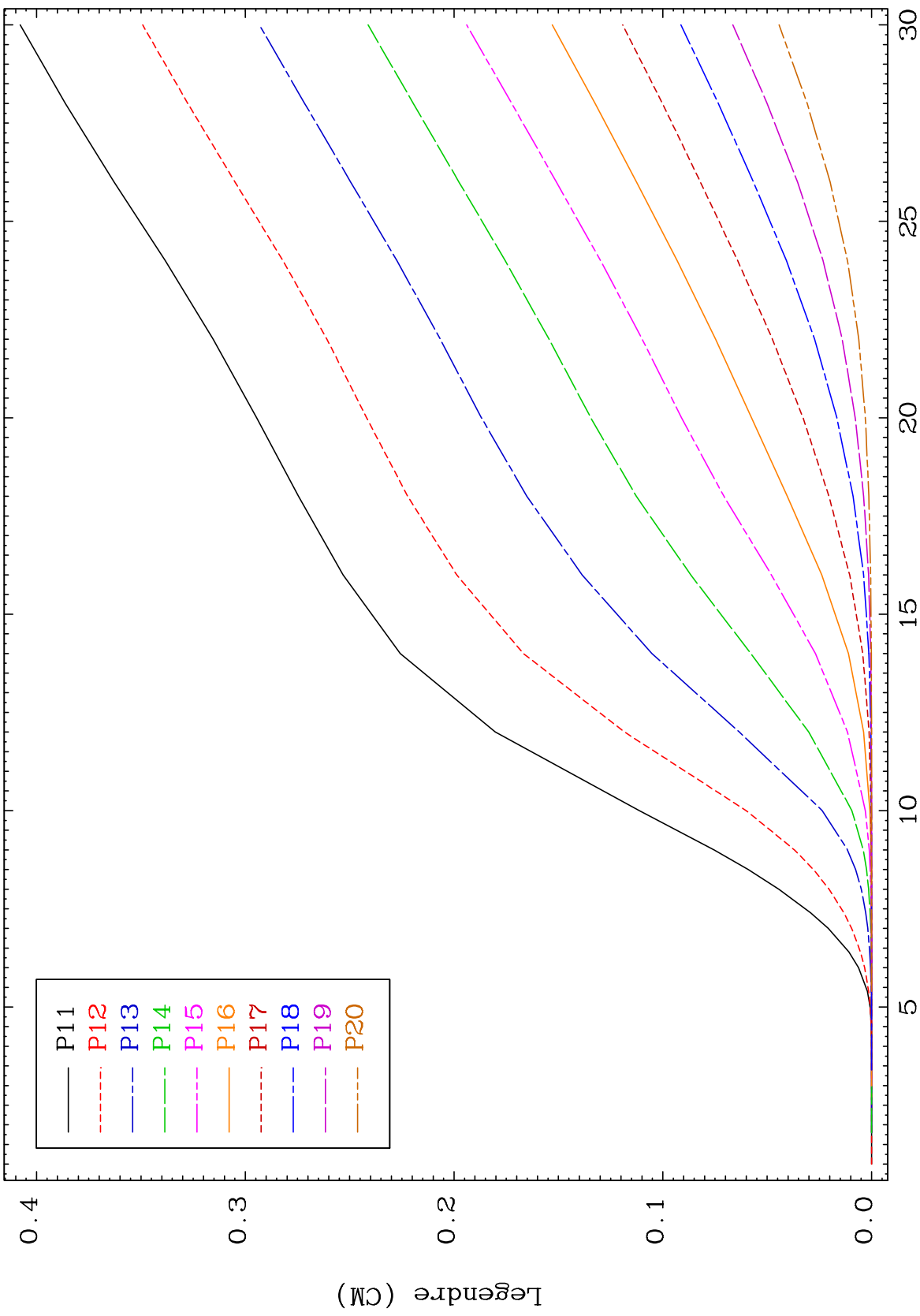
Incident Energy (MeV)

101-Md-255

MAT 9956

Elastic Legendre Coefficients

101-Md-255



19

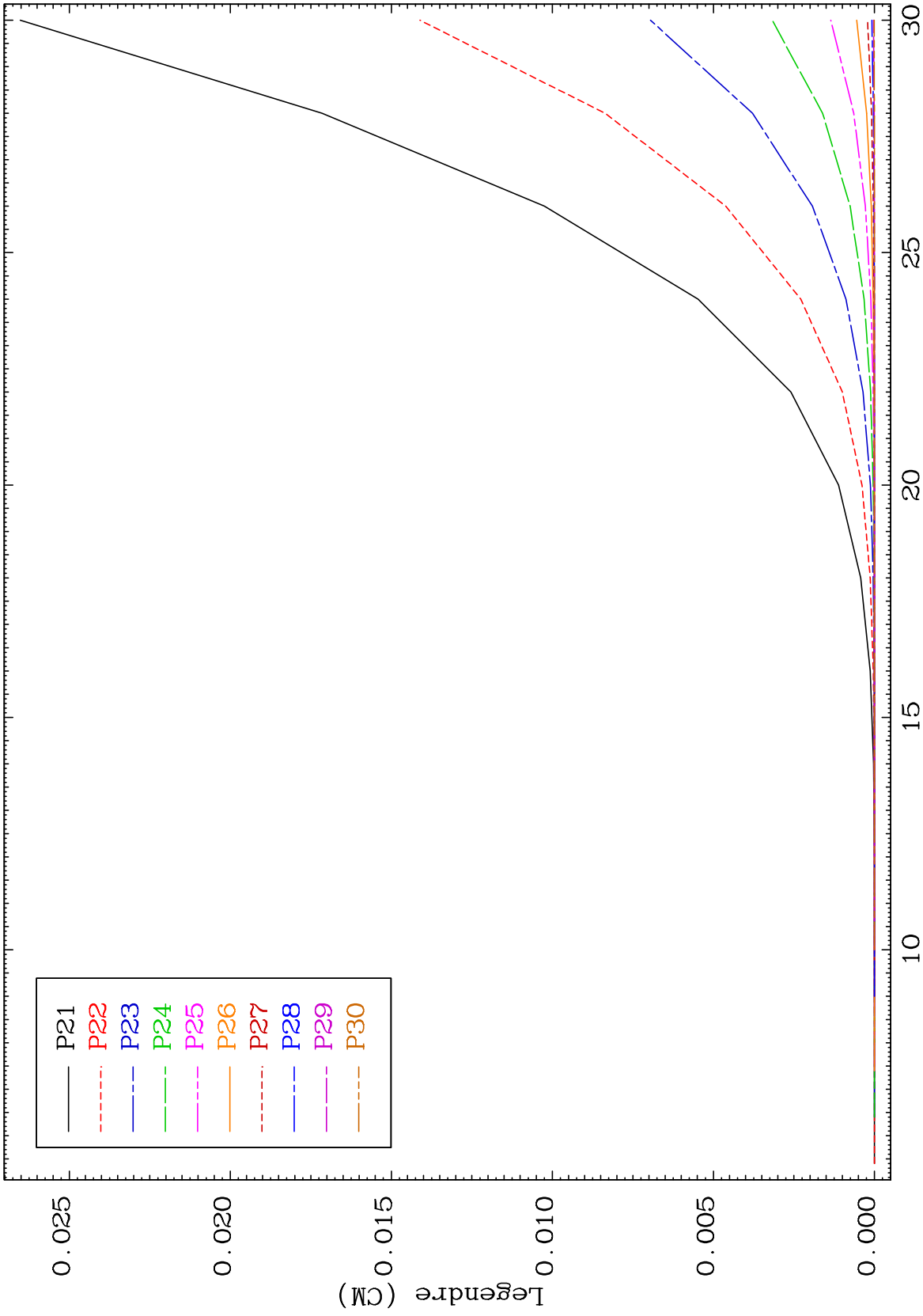
Incident Energy (MeV)

101-Md-255

MAT 9956

Elastic Legendre Coefficients

101-Md-255



20

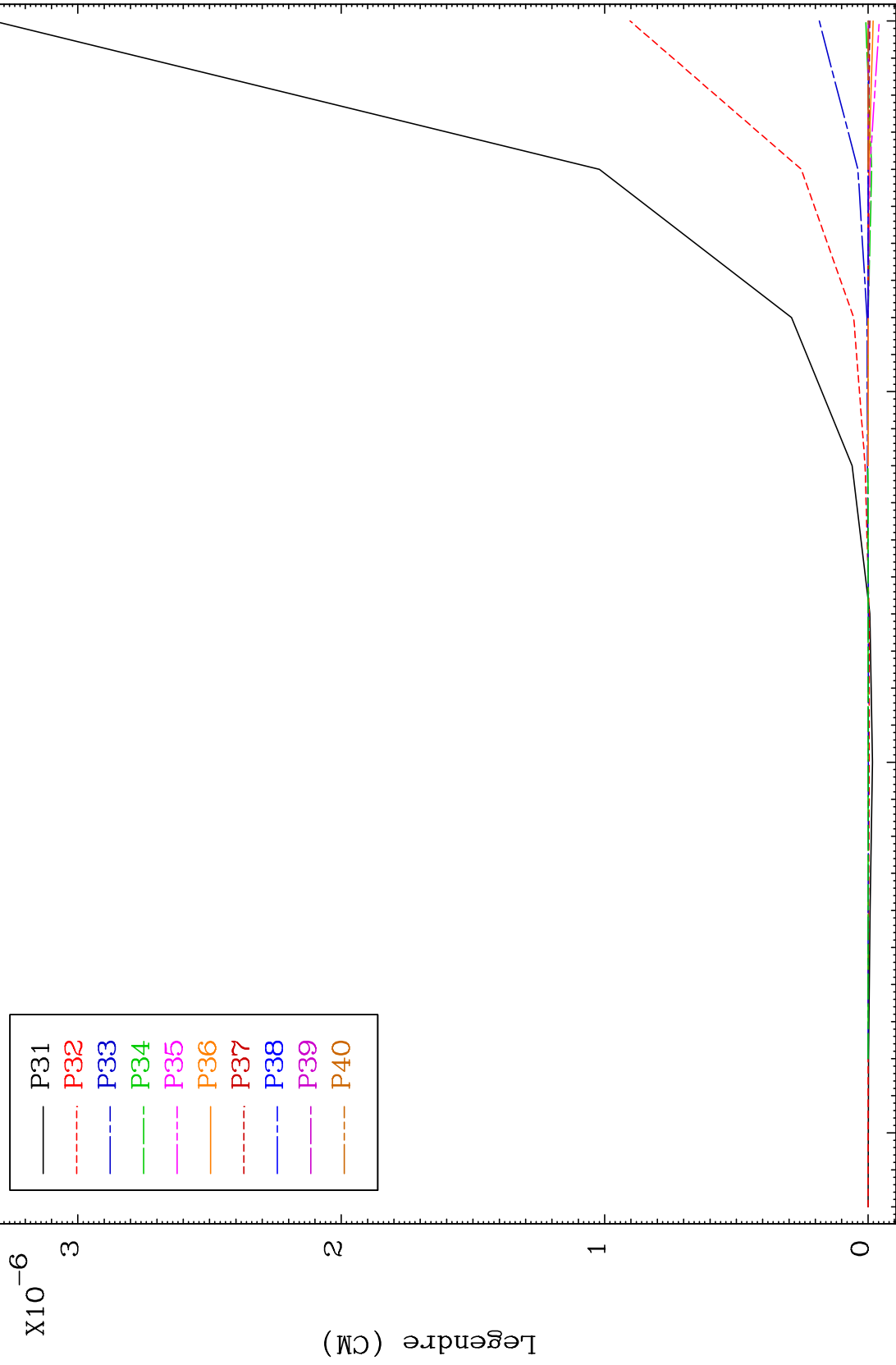
Incident Energy (MeV)

101-Md-255

MAT 9956

Elastic Legendre Coefficients

101-Md-255



21

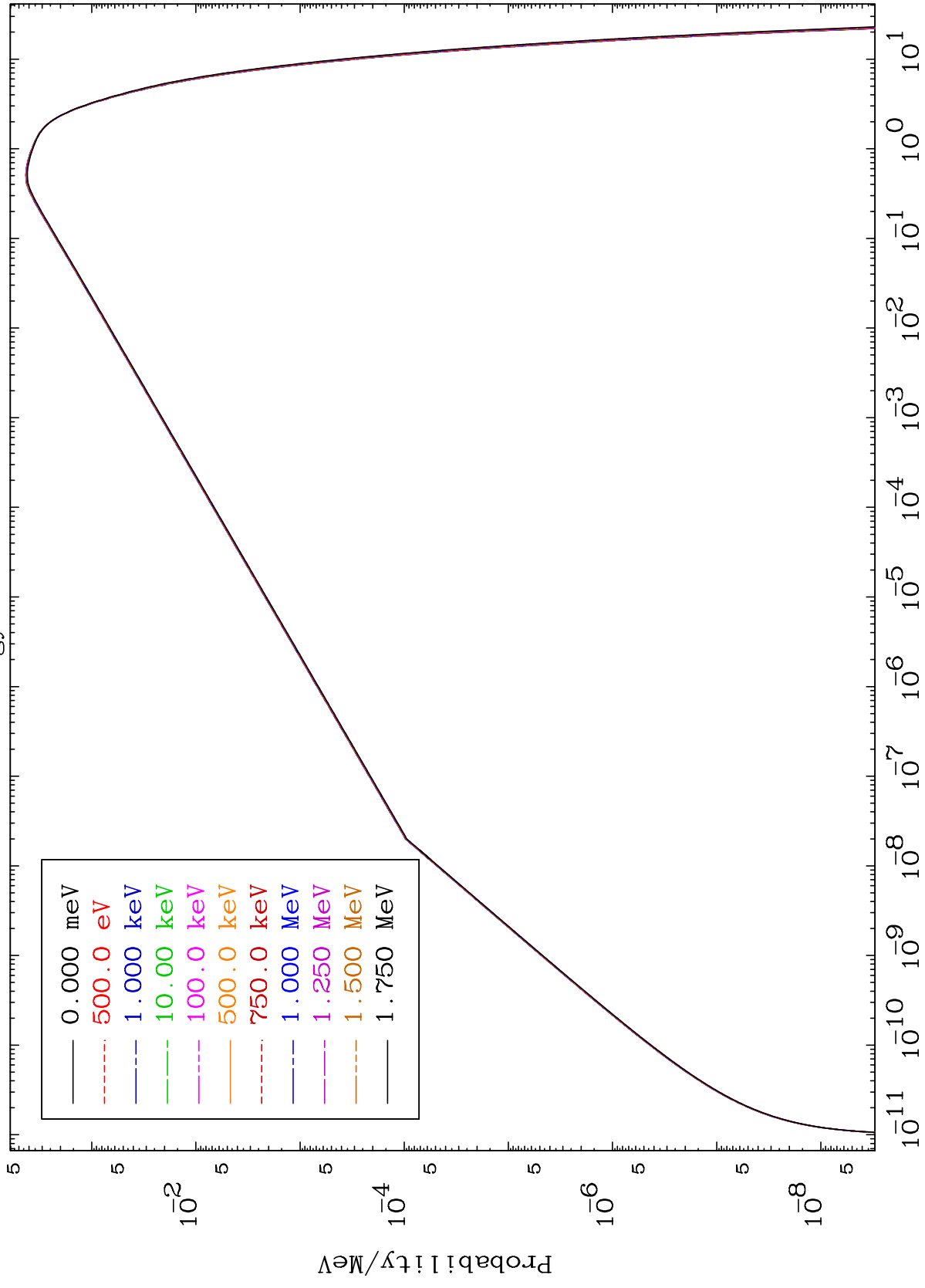
Incident Energy (MeV)

101-Md-255

MAT 9956

Fission Energy Distribution

101-Md-255

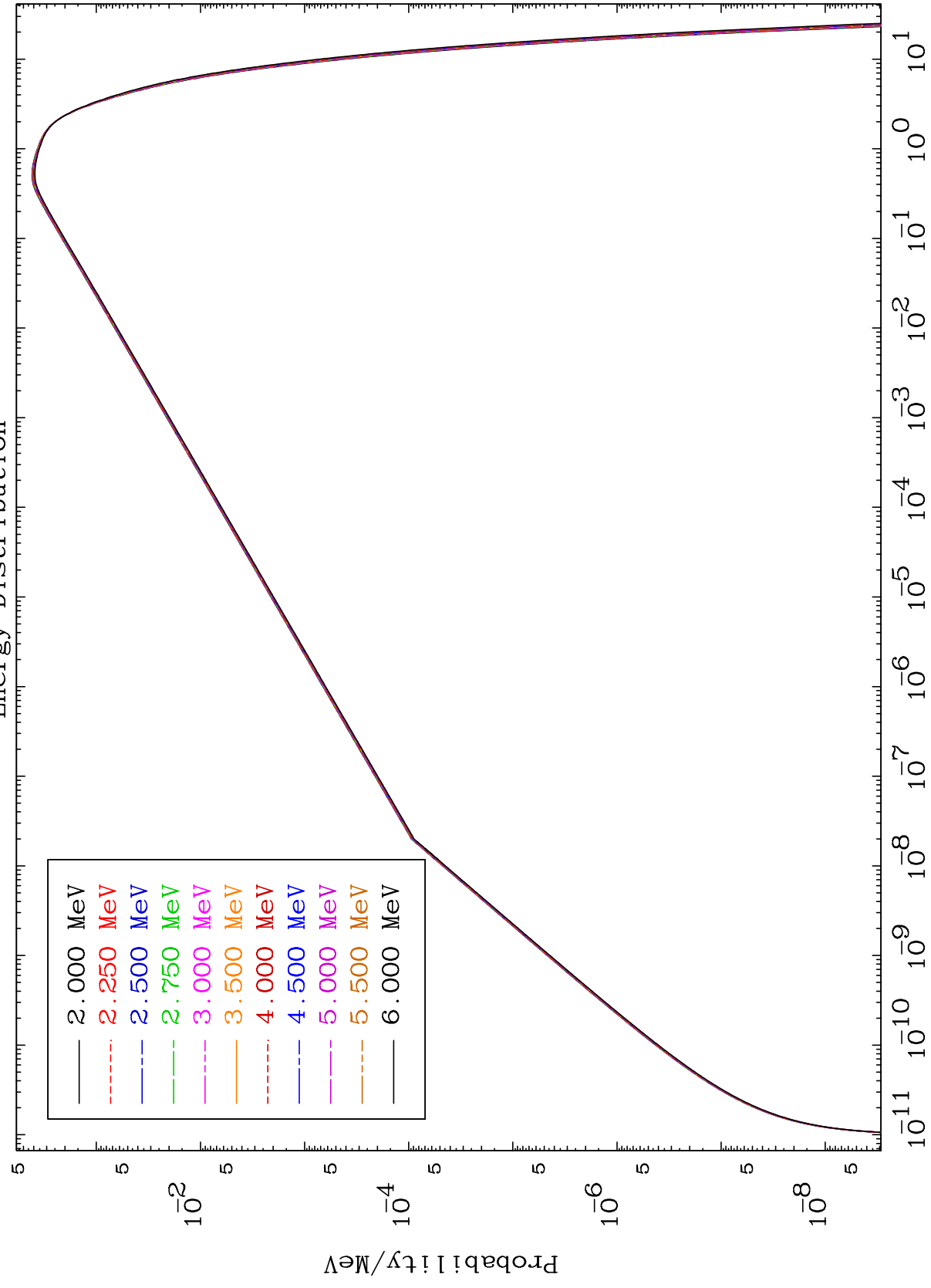


101-Md-255

MAT 9956

Fission Energy Distribution

101-Md-255

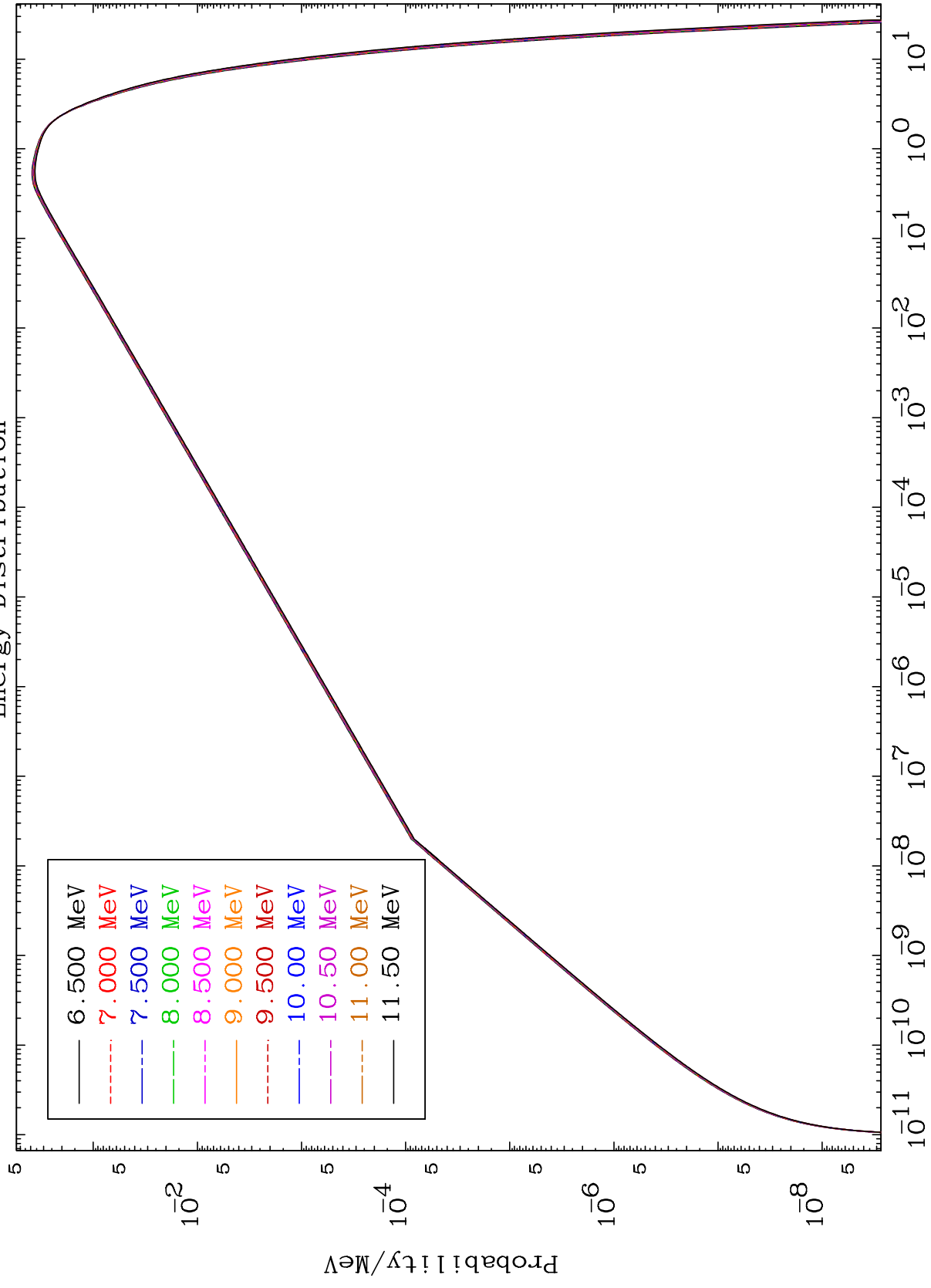


101-Md-255

MAT 9956

Fission
Energy Distribution

101-Md-255

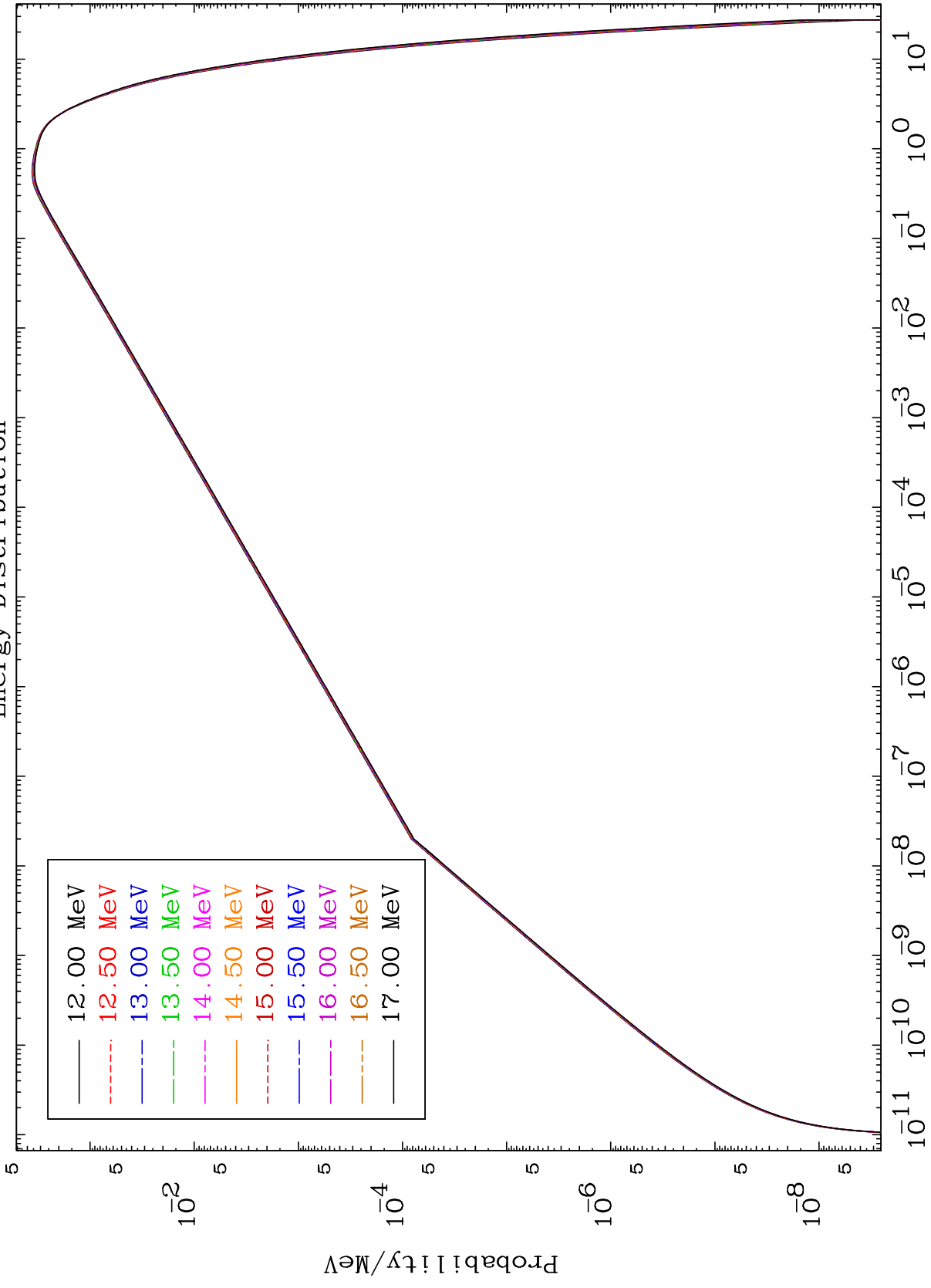


101-Md-255

MAT 9956

Fission Energy Distribution

101-Md-255

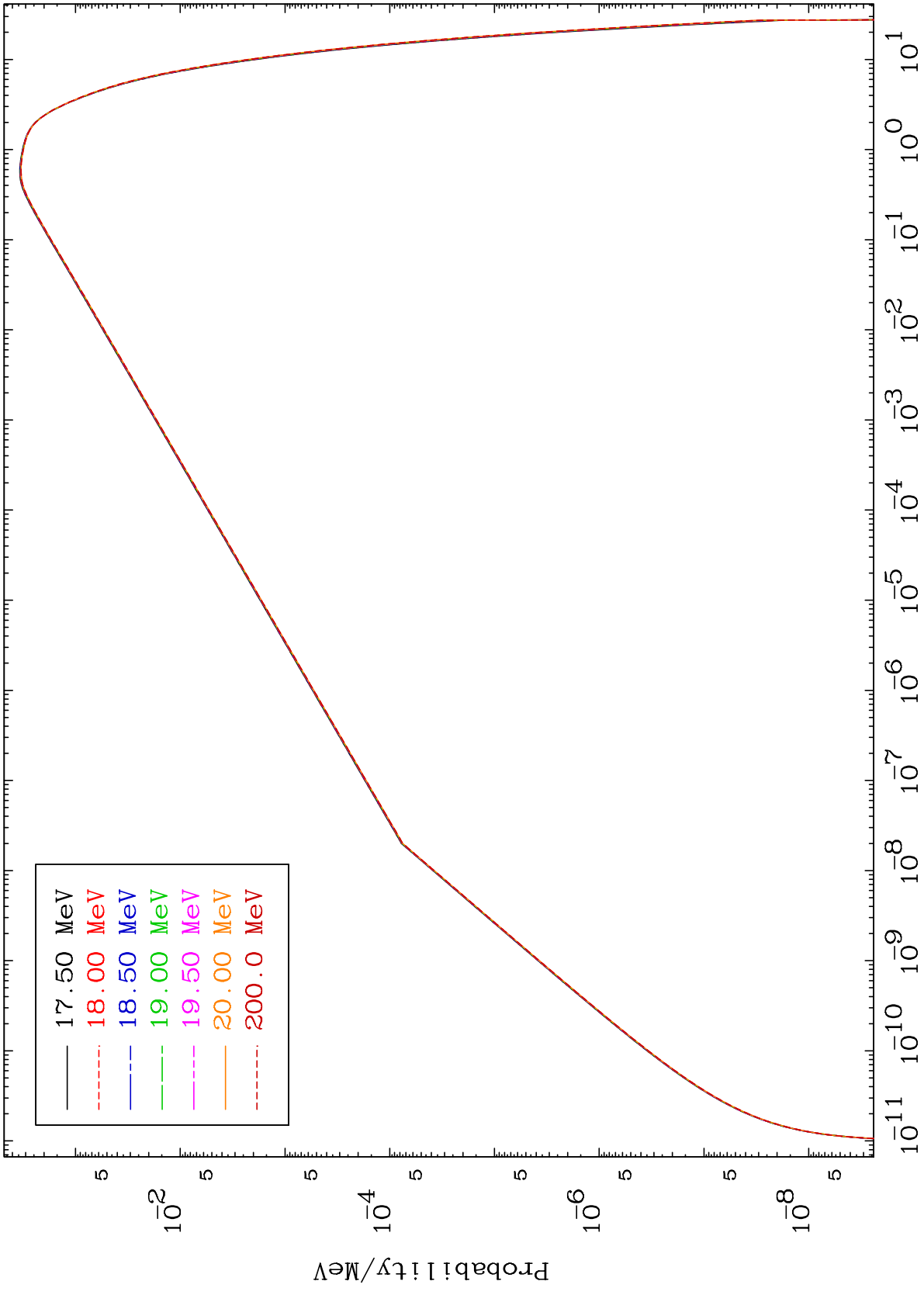


101-Md-255

MAT 9956

Fission Energy Distribution

101-Md-255

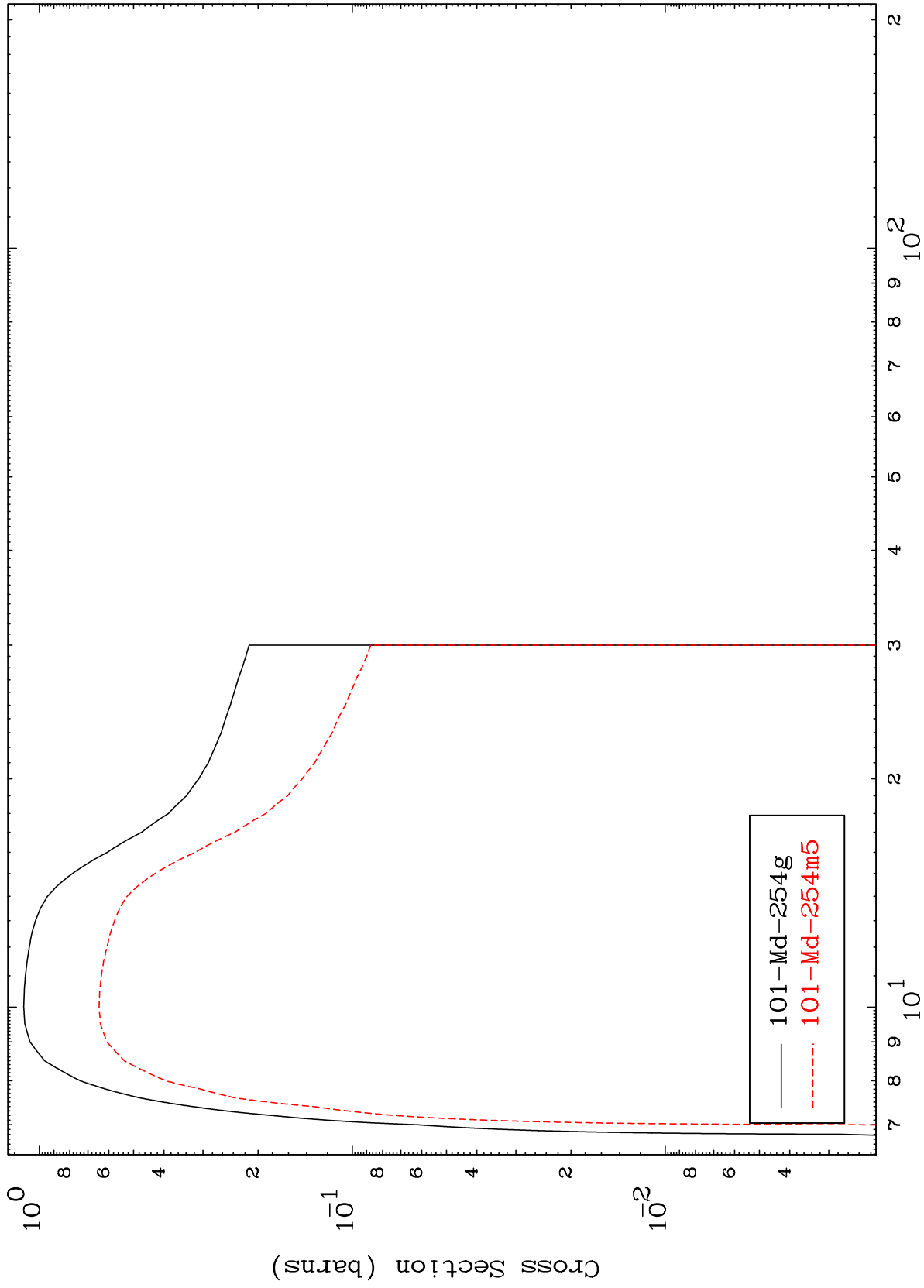


101-Md-255

MAT 9956

101-Md-255

(n,2n)
Radionuclide Production Cross Section



27

Incident Energy (MeV)

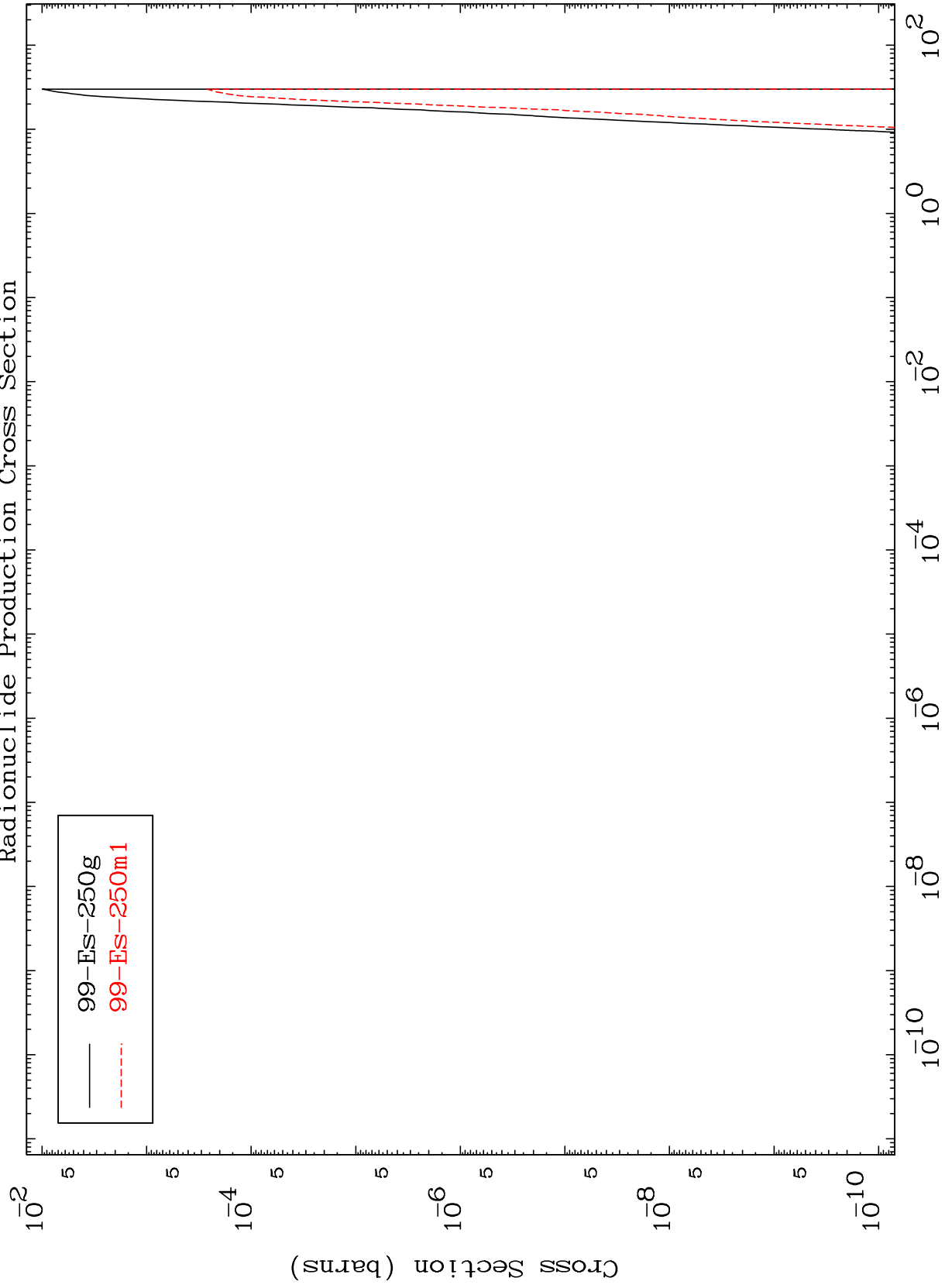
101-Md-255

MAT 9956

(n,2n) α

101-Md-255

Radionuclide Production Cross Section



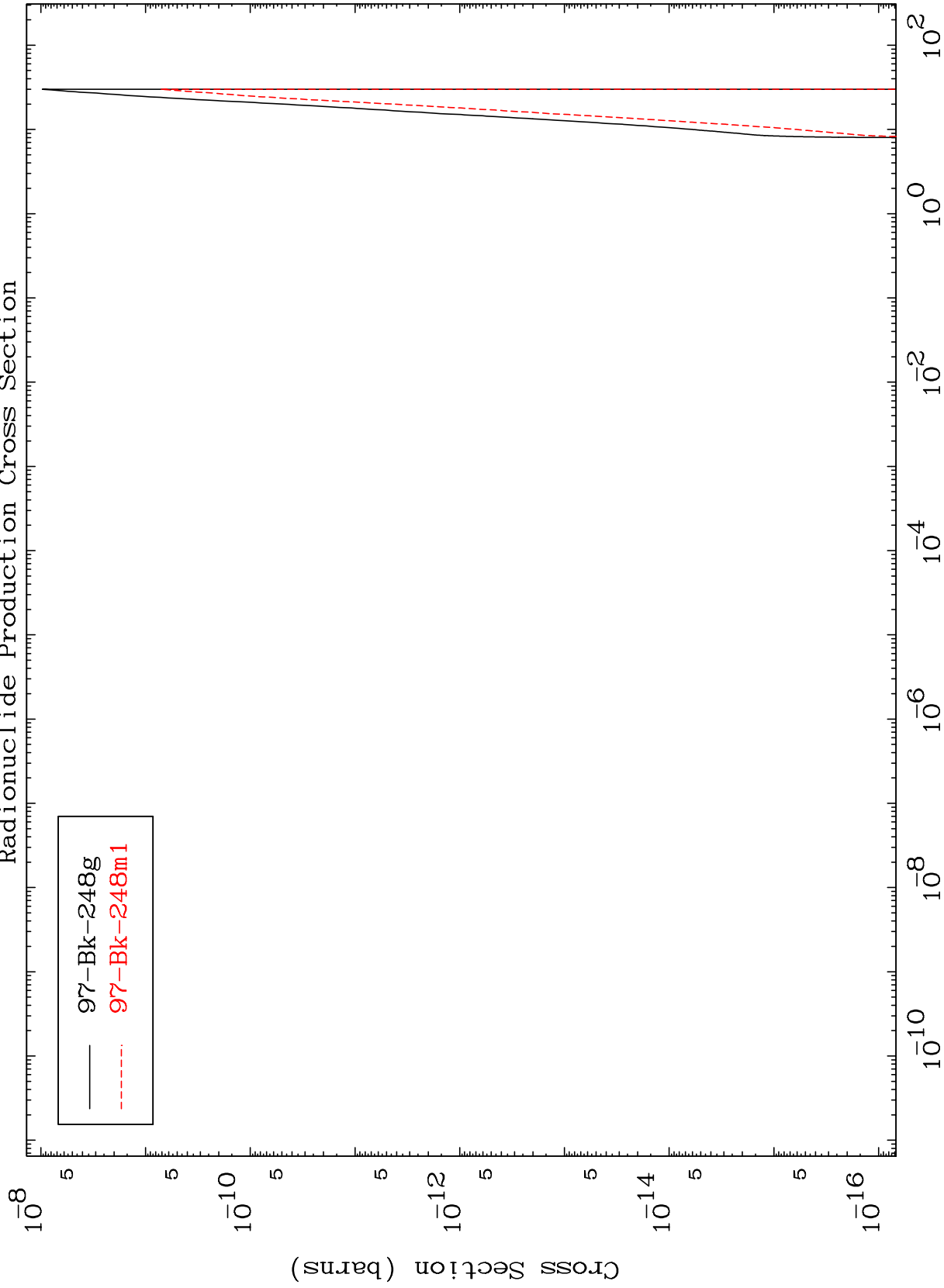
99-Es-250g
99-Es-250ml

MAT 9956

(n,2α)

101-Md-255

Radionuclide Production Cross Section



97-Bk-248g
97-Bk-248m1

29

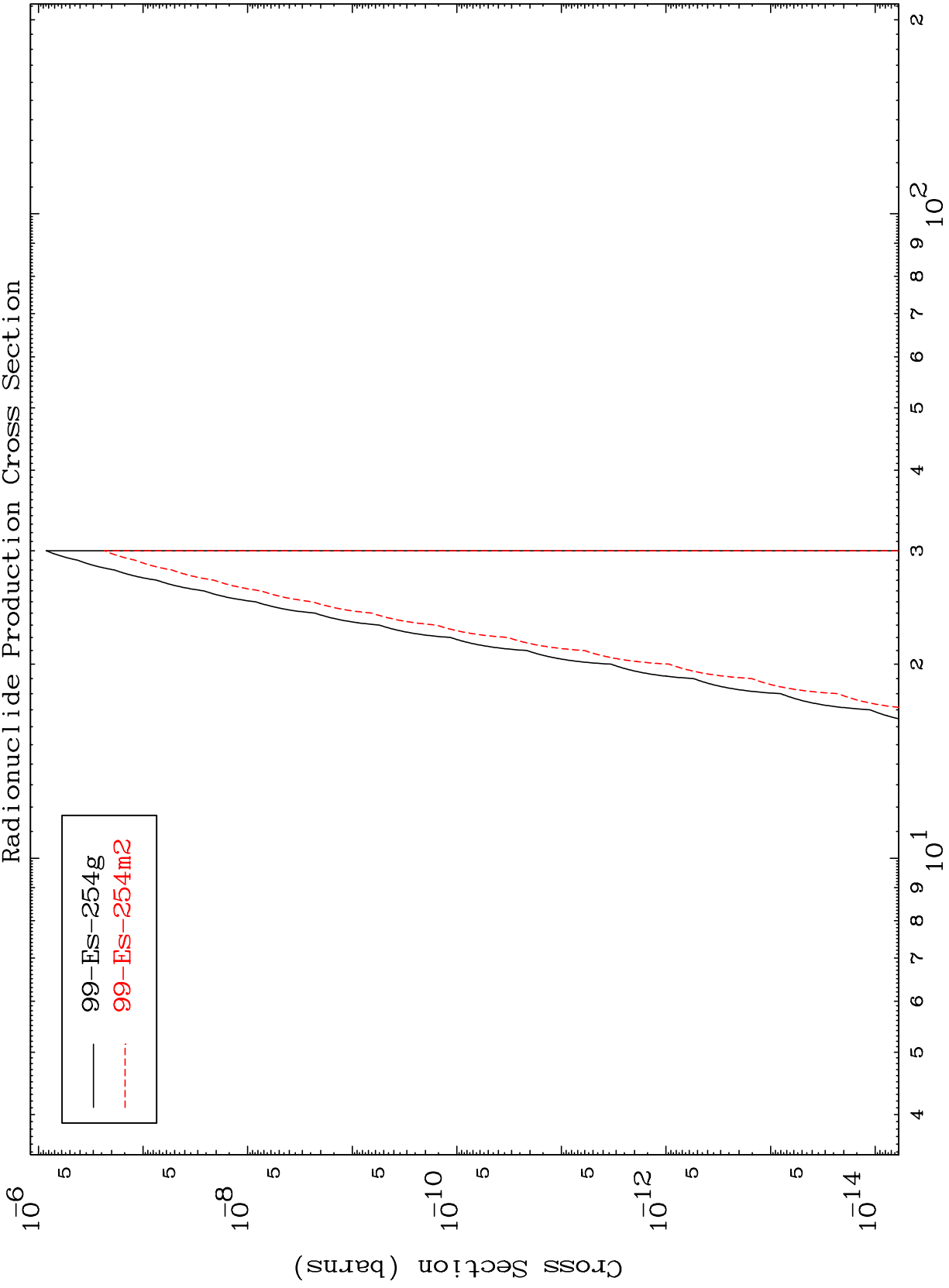
Incident Energy (MeV)

101-Md-255

MAT 9956

101-Md-255

(n,2p)
Radionuclide Production Cross Section



— 99-Es-254g
- - - 99-Es-254m2

30

101-Md-255